

Nuclear data of the future: daring selection via adjustment and Total Monte Carlo

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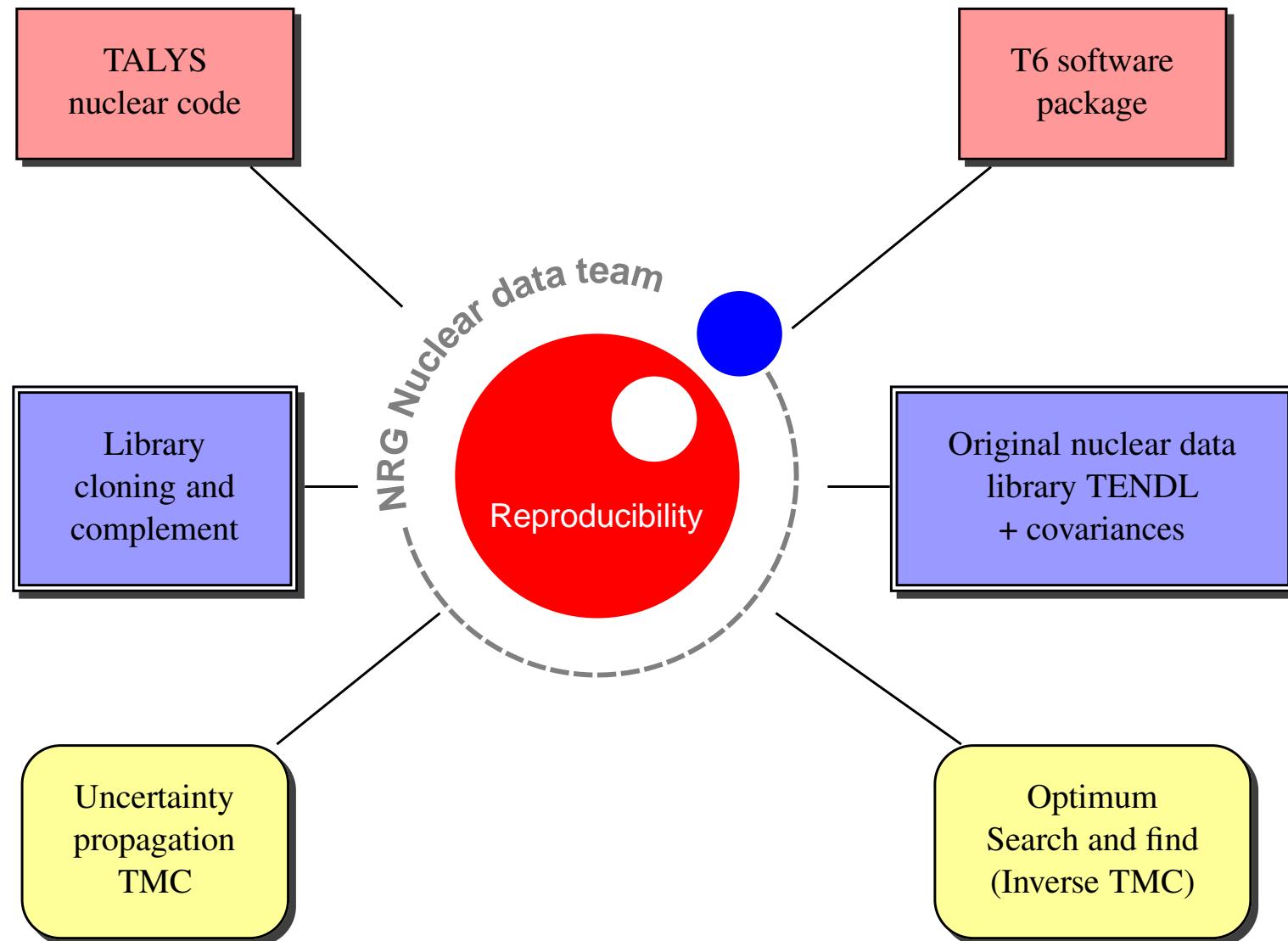
Contents

- ① How can we adjust nuclear data using TMC
(feedback from experimental data)
- ② Explanations
- ③ Examples
- ④ Conclusions



Pavlov's dog eating Schrodinger's cat
(deterministic vs. Monte carlo approach)

Backbone of our methodology: REPRODUCIBILITY



Optimum Search and find (Inverse TMC)

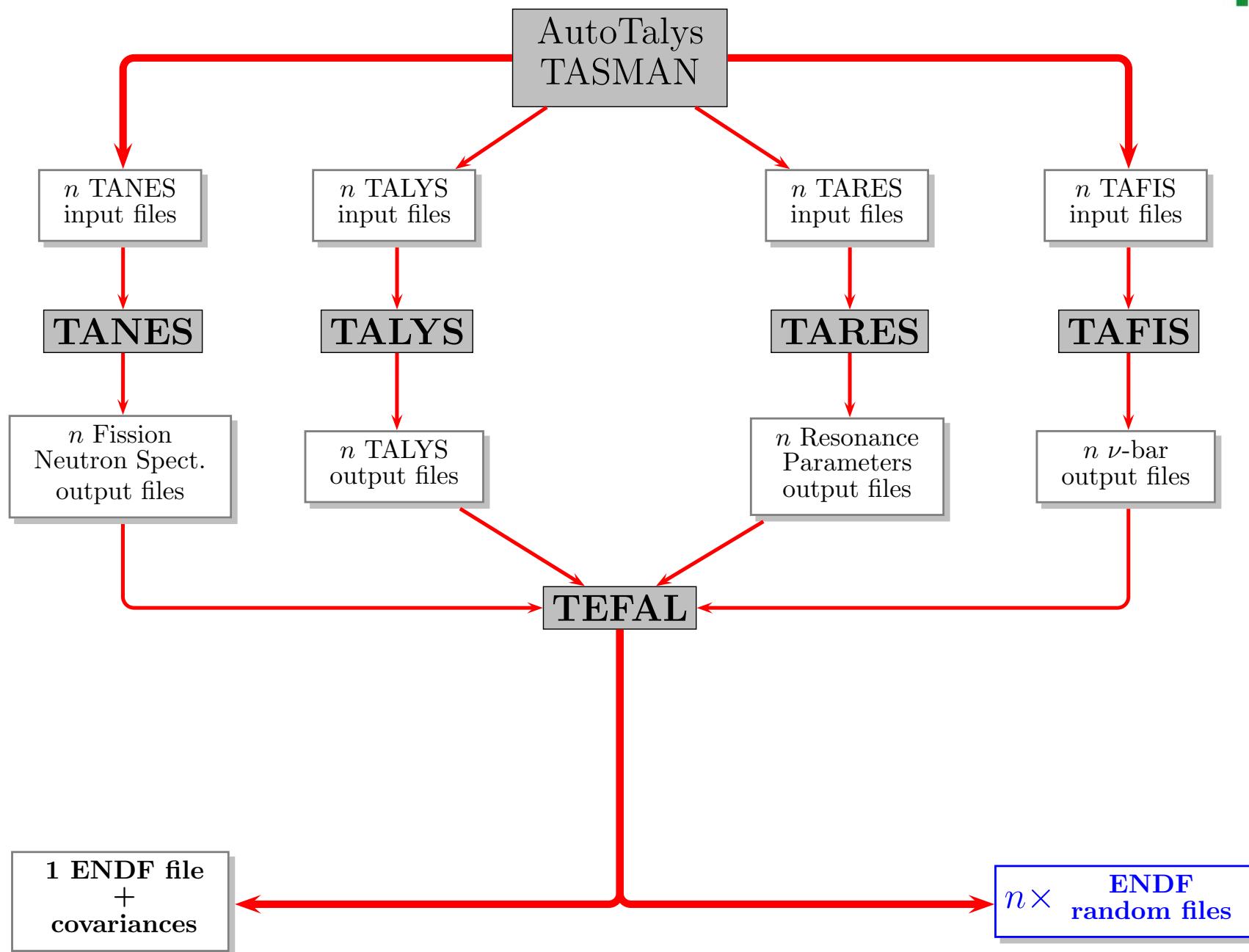
- Started in 2010
- Two publications so far (plus 2 submitted)
- Controversial (if understood at all)
- We believe this is the future of nuclear data evaluation work
- It might be the only way to sensibly improve C/E

Total Monte Carlo + selection
 $\implies \frac{1}{TMC}$

- ① Use TALYS to create a single ^{239}Pu evaluation close or equal to ENDF/B-VII.0 or JEFF-3.1.1
- ② Randomize all model parameters (resonances, nubar, fission neutron spectrum, TALYS parameters) to create 500 random ^{239}Pu evaluations
- ③ Benchmarks the $n \geq 500$ files with the same set of criticality benchmarks
- ④ Select the best random file

Example: 100 benchmarks, 500 random files \implies 500 TALYS + NJOY and $100 \times 500 = 5 \cdot 10^4$ MCNP loops,
1.4 years on a single processor, or 5 days on 100 processors (3 GHz)

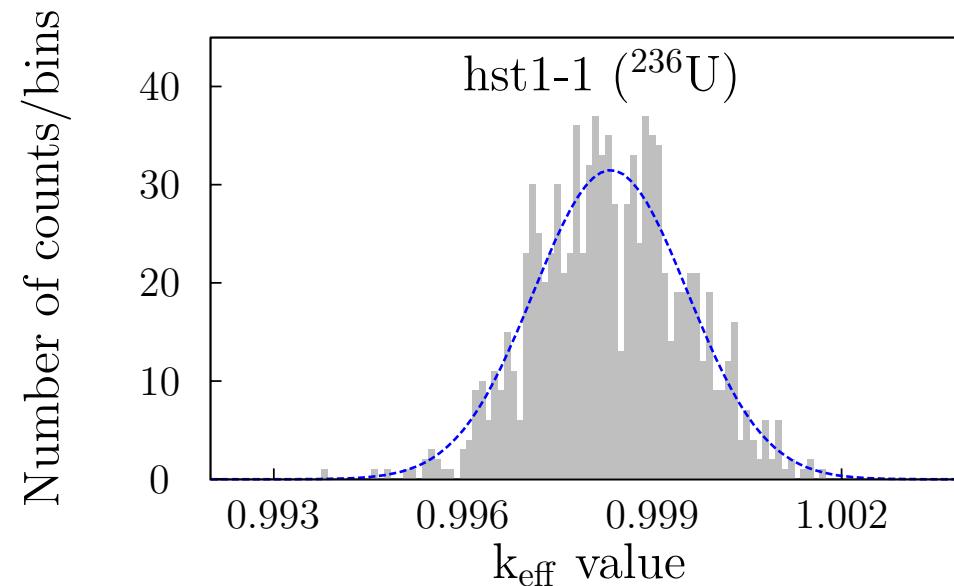
The TALYS system



Total Monte Carlo: examples

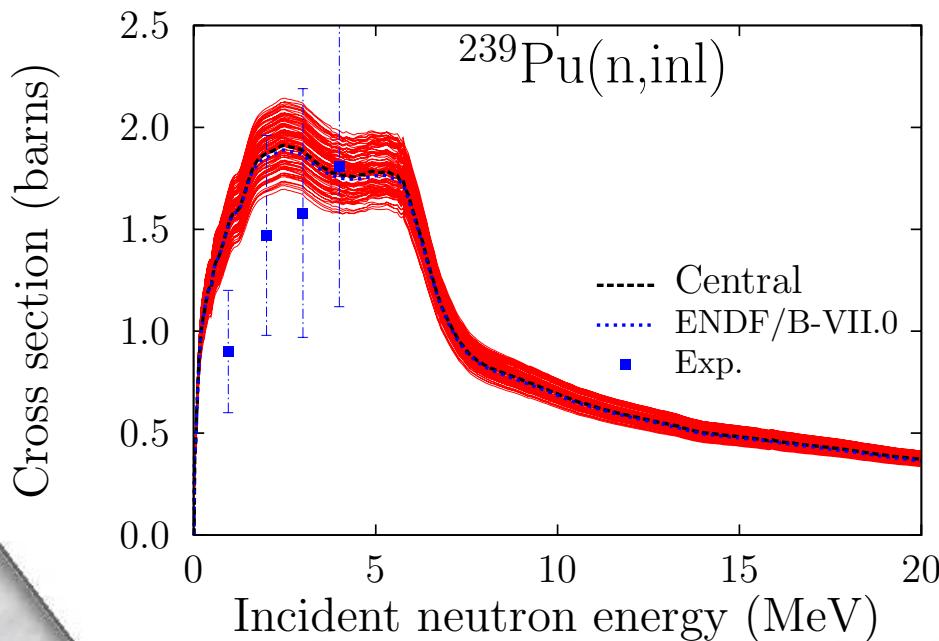
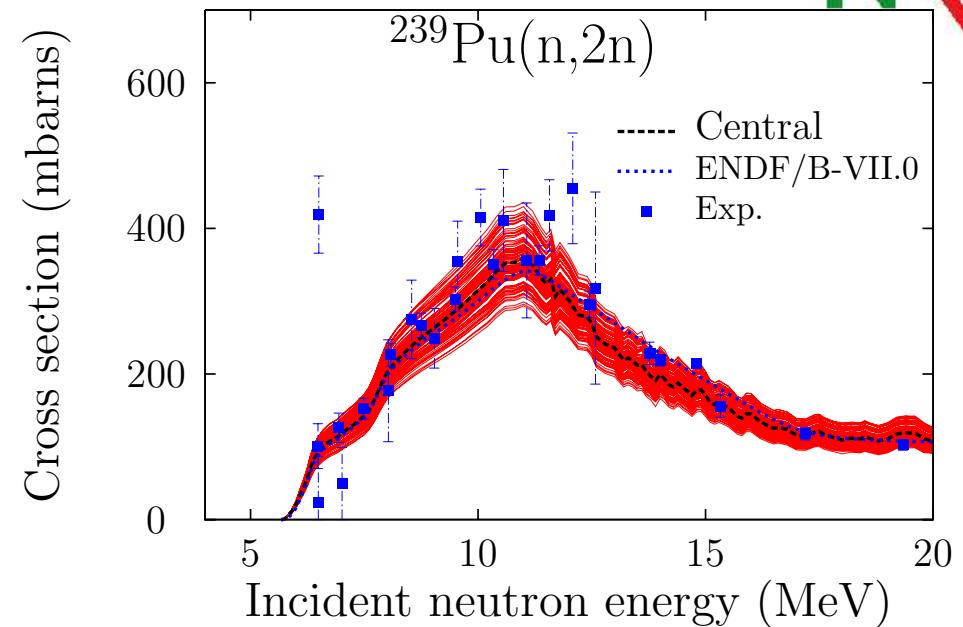
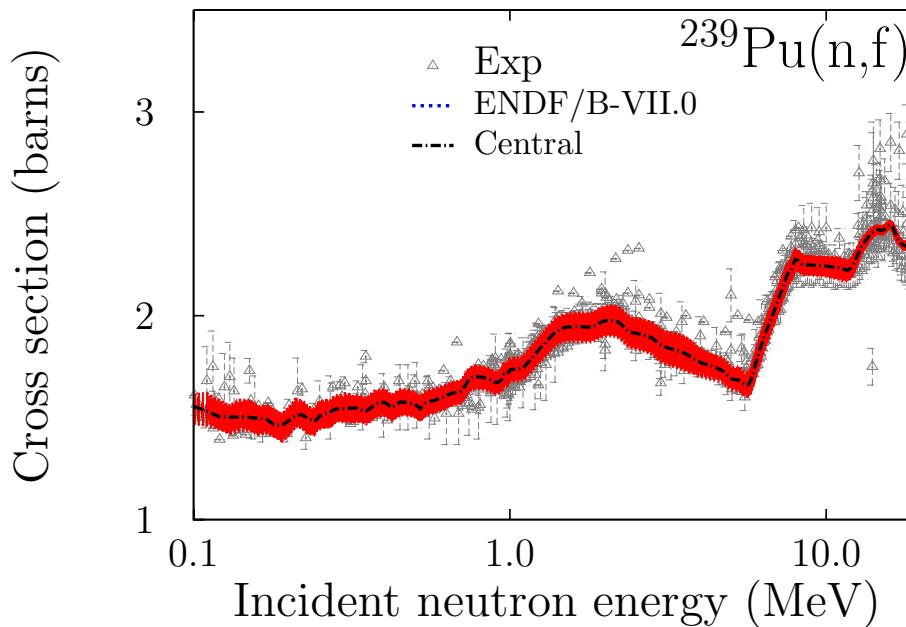
For each random ENDF file, the benchmark calculation is performed with MCNP. At the end of the n calculations, n different k_{eff} values are obtained. In the obtained probability distribution of k_{eff} , the standard deviation σ_{total} reflects two different effects:

$$\sigma_{\text{total}}^2 = \sigma_{\text{statistics}}^2 + \sigma_{\text{nuclear data}}^2. \quad (1)$$



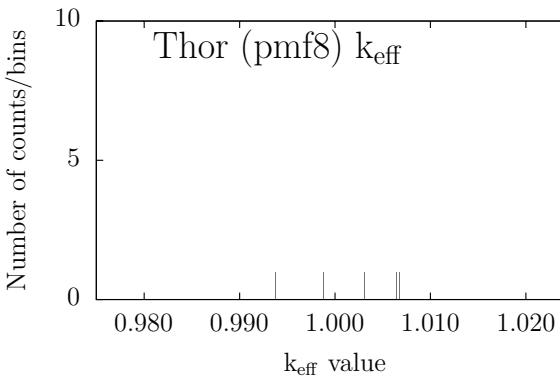
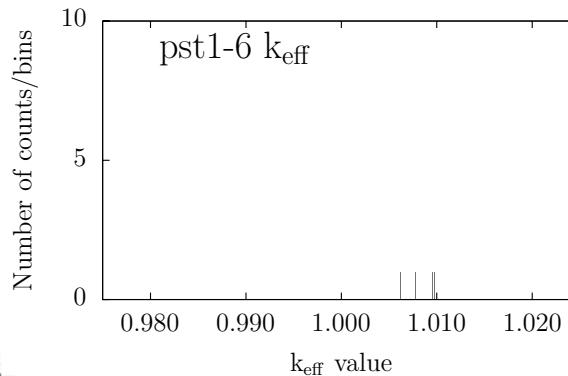
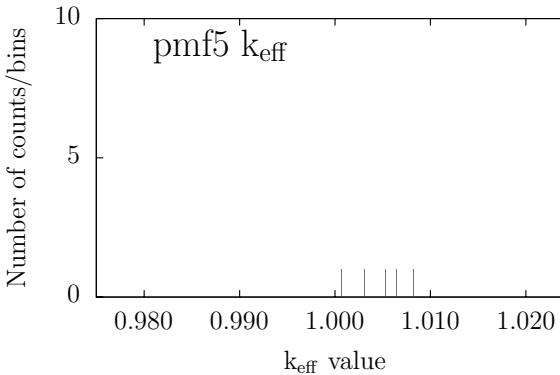
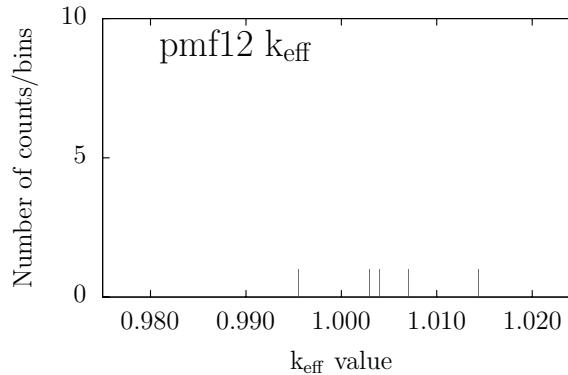
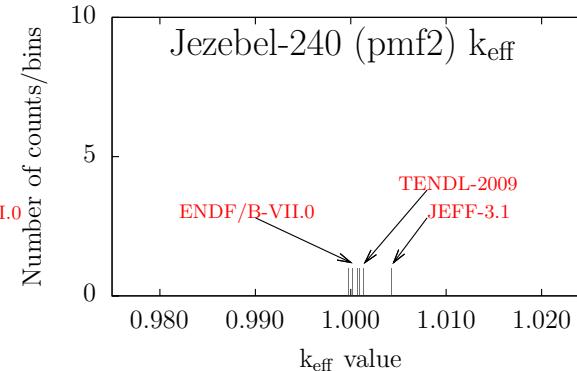
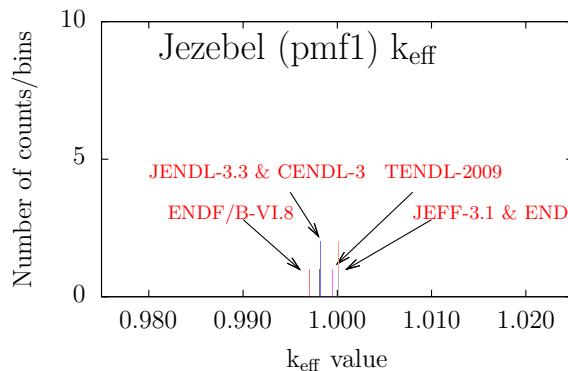
Each random file is completely different from another one: nu-bar ("MF1"), resonance parameters ("MF2"), cross sections ("MF3"), also MF4, MF5, MF6, etc.

Nuclear data: random ^{239}Pu in the thermal and fast range



- (1) Central cross sections *almost* equal to ENDF/B-VII.0 or JEFF-3.1
- (2) Random cross sections obtained from random model parameters
- (3) Similar results in MF1, 2, 4, 5 and 6

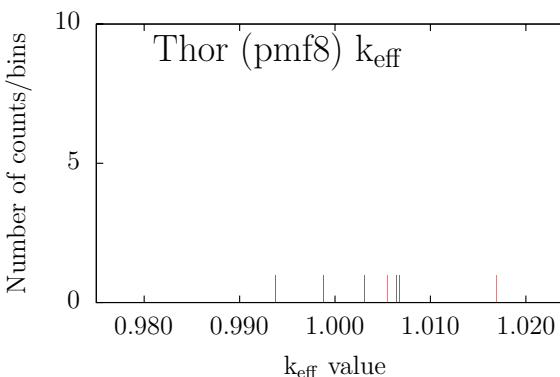
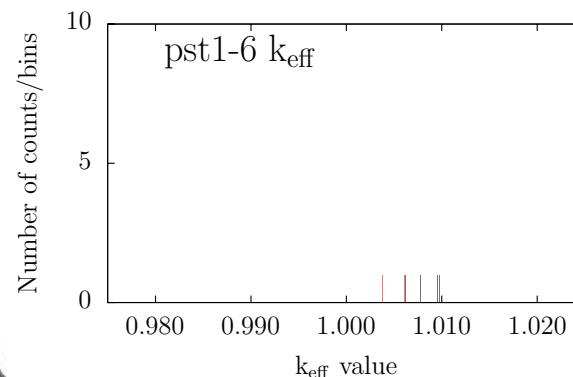
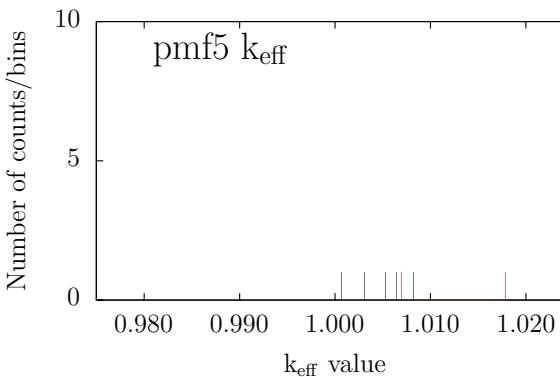
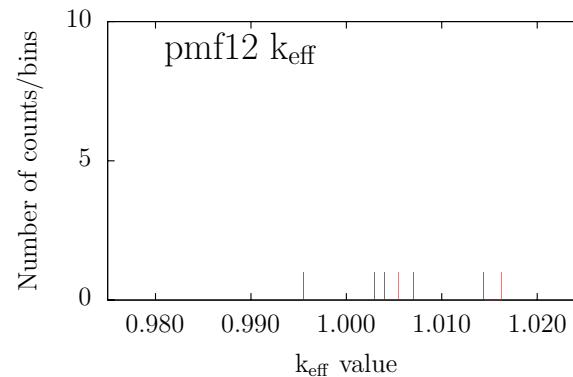
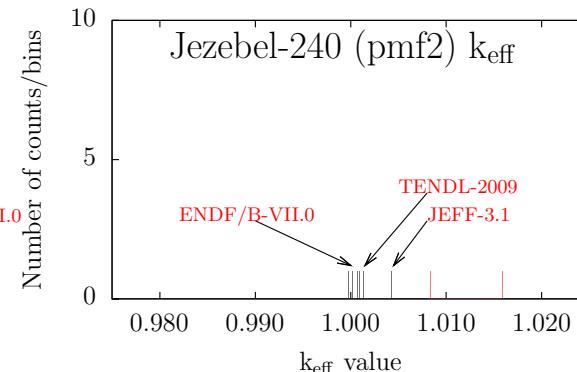
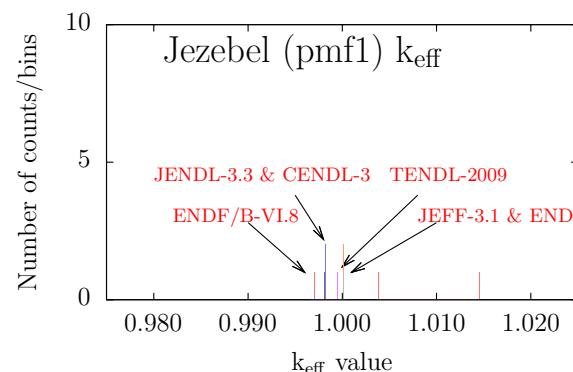
Inverse TMC: simple example with 6 k_{eff} benchmarks



α	
JEFF-3.1.1:	$1.14e^{-4}$
JENDL-3.3:	$1.71e^{-4}$
TENDL-2009:	$3.66e^{-4}$
ENDF/B-VI.8:	$1.72e^{-4}$
ENDF/B-VII.0:	$1.69e^{-4}$

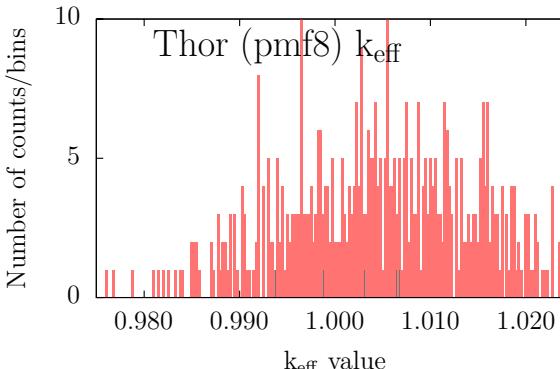
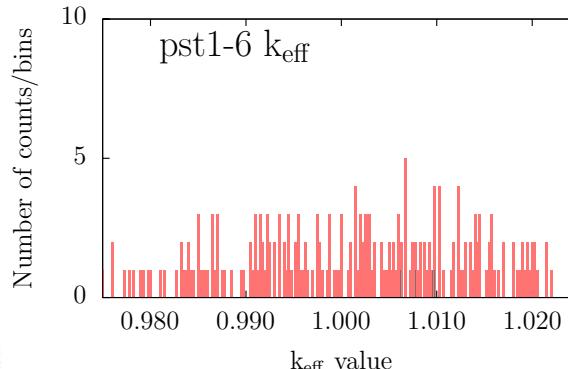
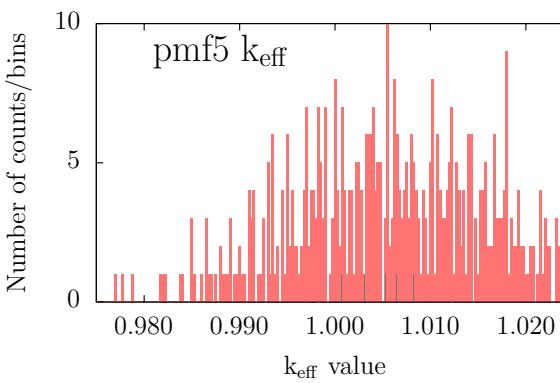
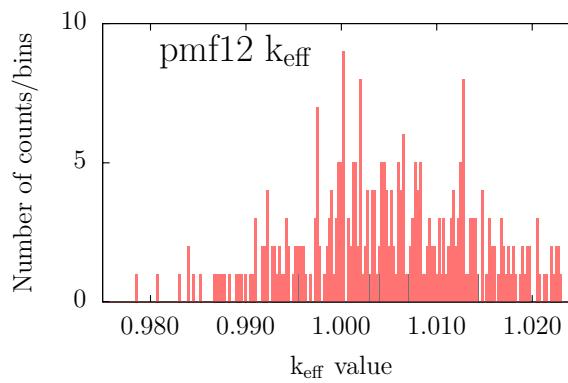
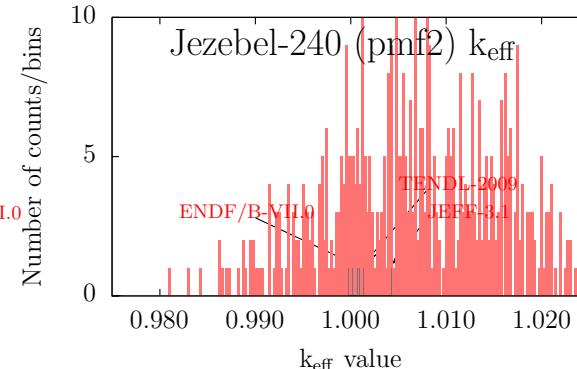
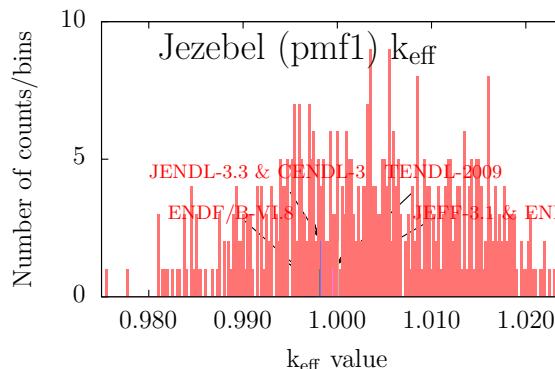
$$\alpha = \sum_{i=0}^n \frac{(C_i - E_i)^2}{C_i},$$

Inverse TMC: simple example with 6 k_{eff} benchmarks



α	$1.14e^{-4}$
JEFF-3.1.1:	$1.71e^{-4}$
JENDL-3.3:	$3.66e^{-4}$
TENDL-2009:	$1.72e^{-4}$
ENDF/B-VI.8:	$1.69e^{-4}$
random 0:	$2.29e^{-4}$
random 1:	$13.4e^{-4}$

Inverse TMC: 6 k_{eff} benchmarks with random ^{239}Pu



Real case: 120 ^{239}Pu benchmarks



Table 1: List of plutonium benchmarks selected for the random search.

Name	Cases	Name	Cases	Name	Cases	Name	Cases
pmf1	1	pmf2	1	pmf5	1	pmf6	1
pmf8	1	pmf12	1	pmf13	1	pc1	1
pmi2	1	pst1	6	pst2	6	pst3	8
pst4	13	pst5	9	pst6	3	pst7	9
pst8	29	pst12	22	pmm1	6		

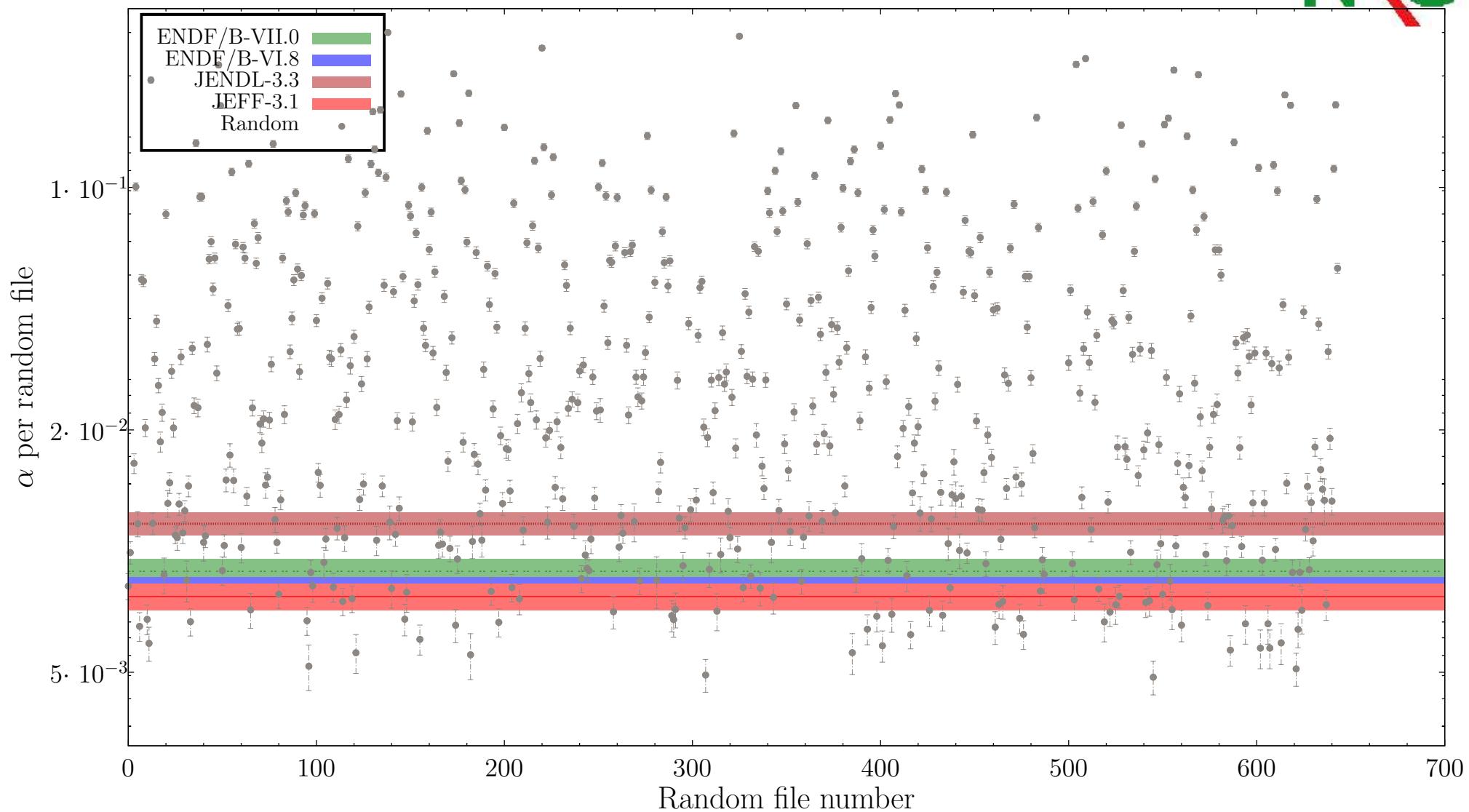
$$\alpha = \sum_{i=0}^n \frac{(C_i - E_i)^2}{C_i}, \quad (2)$$

Results independent of the type of factor α , χ^2 ... or

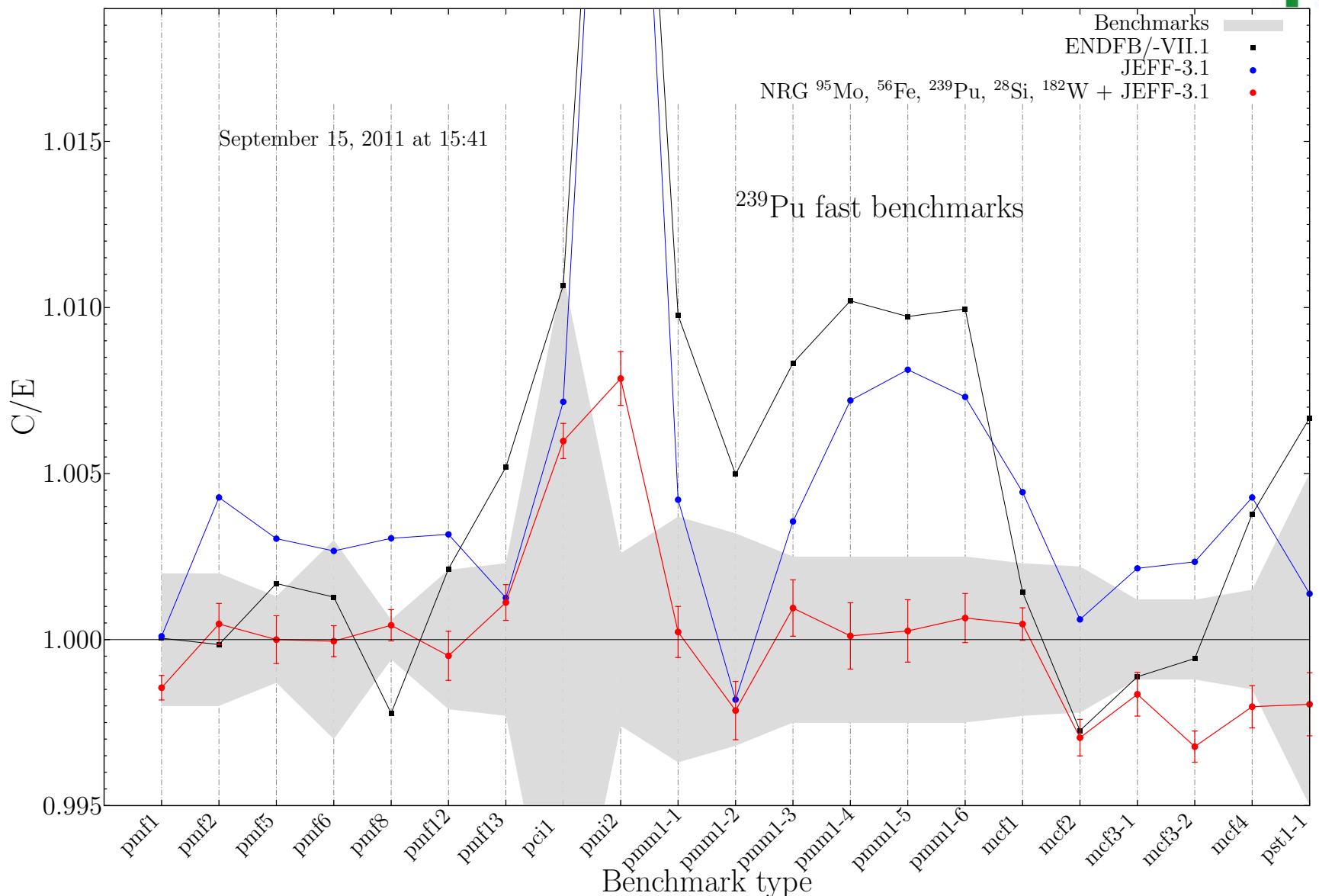
$$F = 1 - 10^{\sqrt{\frac{1}{N} \sum (\log(E_i) - \log(C_i))^2}} \quad (3)$$

α values for random ^{239}Pu evaluations

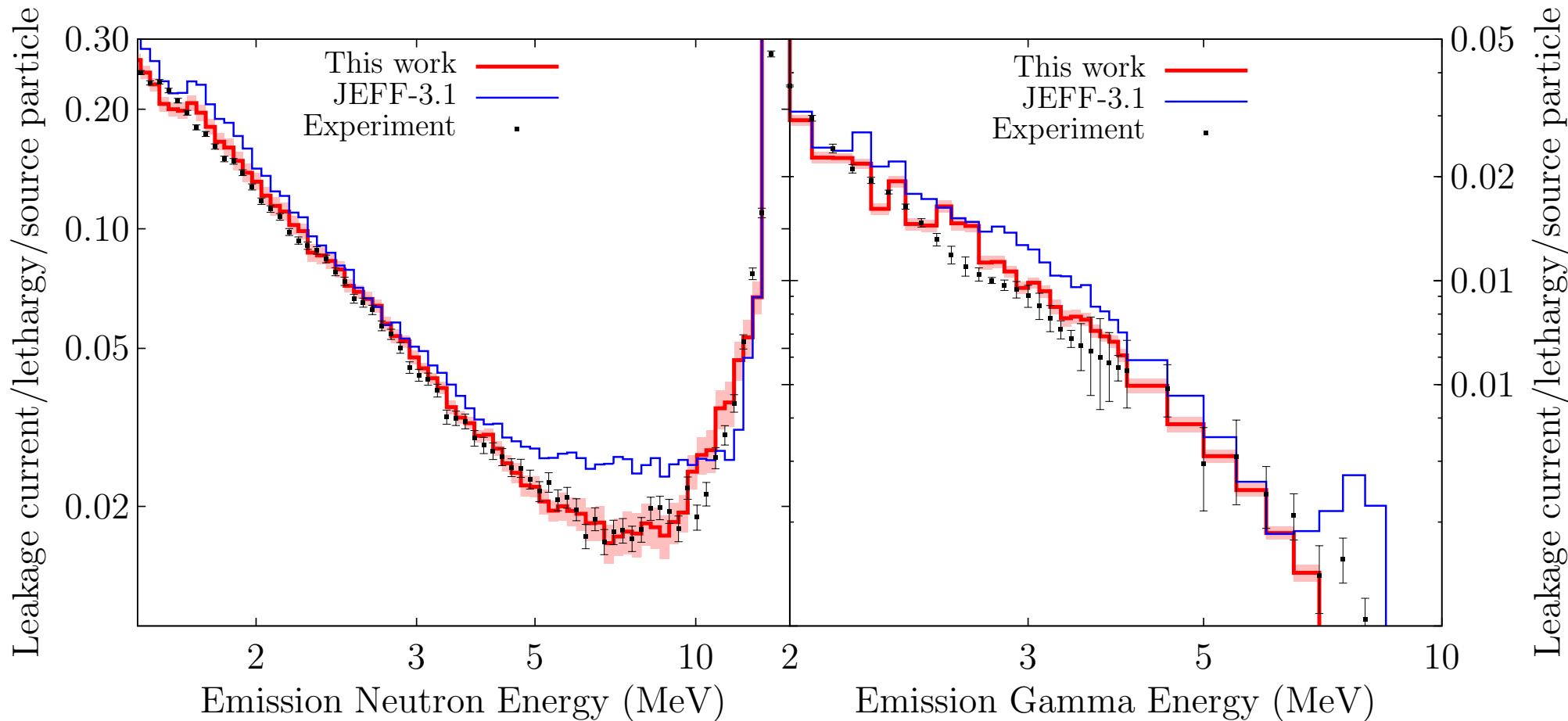
NRG



Inverse TMC: best ^{239}Pu for the ANDES project



Inverse TMC: second example on natural copper and Oktavian benchmark



Inverse TMC: third (and last) example on thermal scattering data H in H₂O

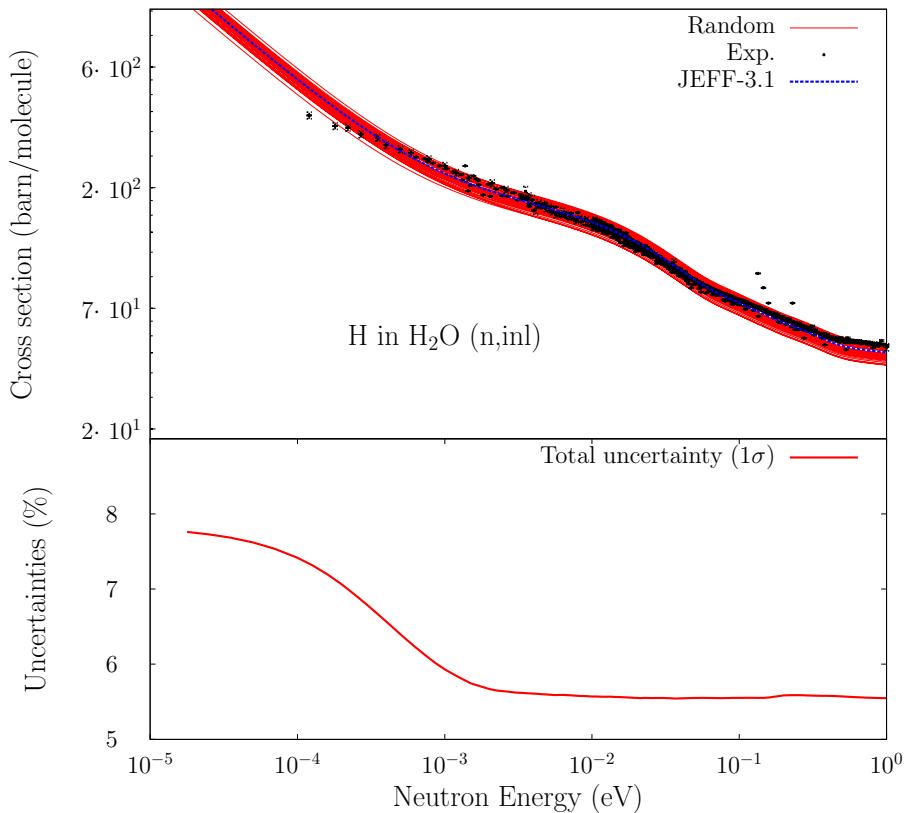


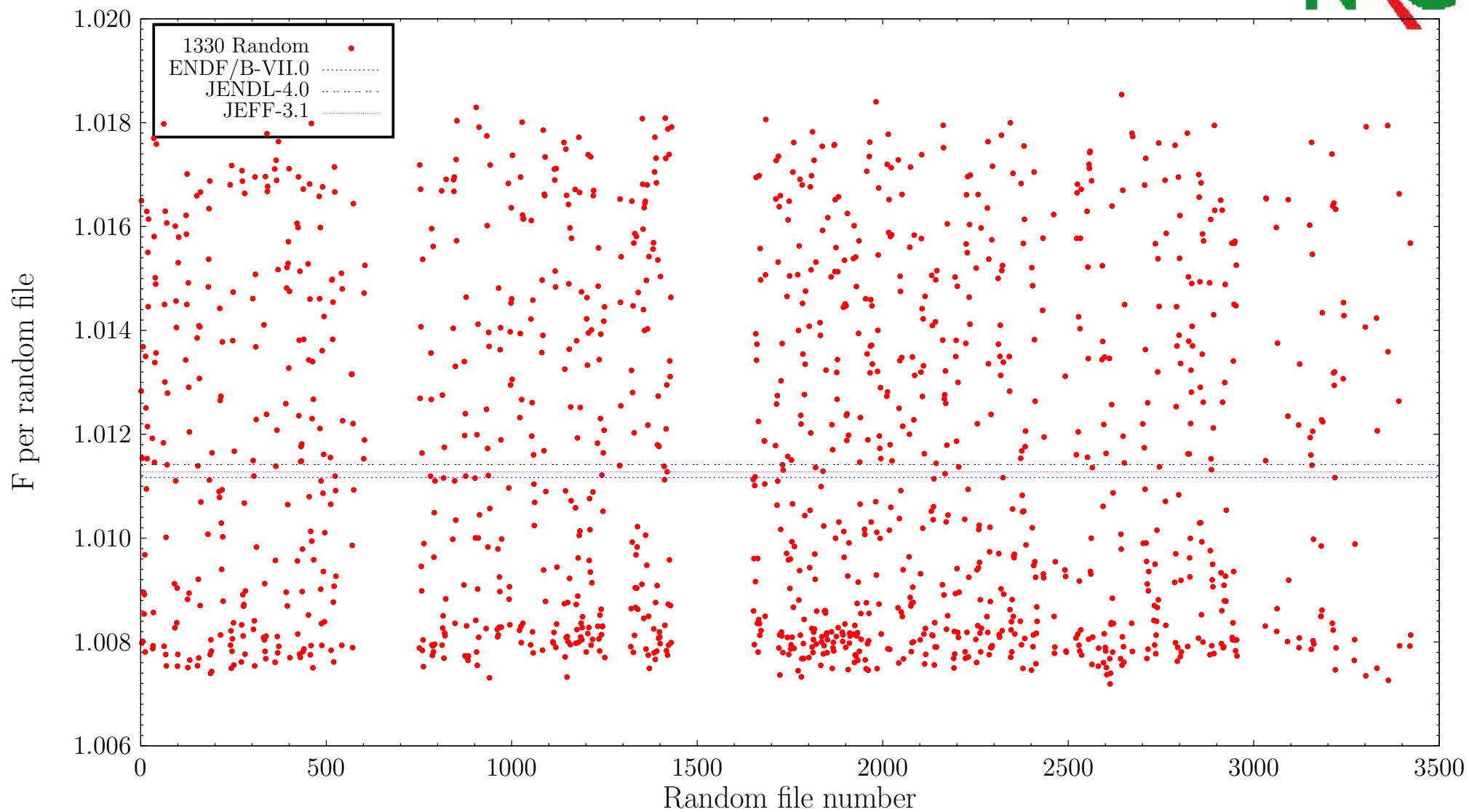
Table 2: List of thermal benchmarks selected for the random search.

Name	Cases	Name	Cases
pst12	22	pst1	6
lct7	10	lct6	18
lst4	7	lmt1	1
ict3	2	hst32	1
hst42	8		

$$F = 10 \sqrt{\frac{1}{N} \sum (\log(E_i) - \log(C_i))^2} \quad (4)$$

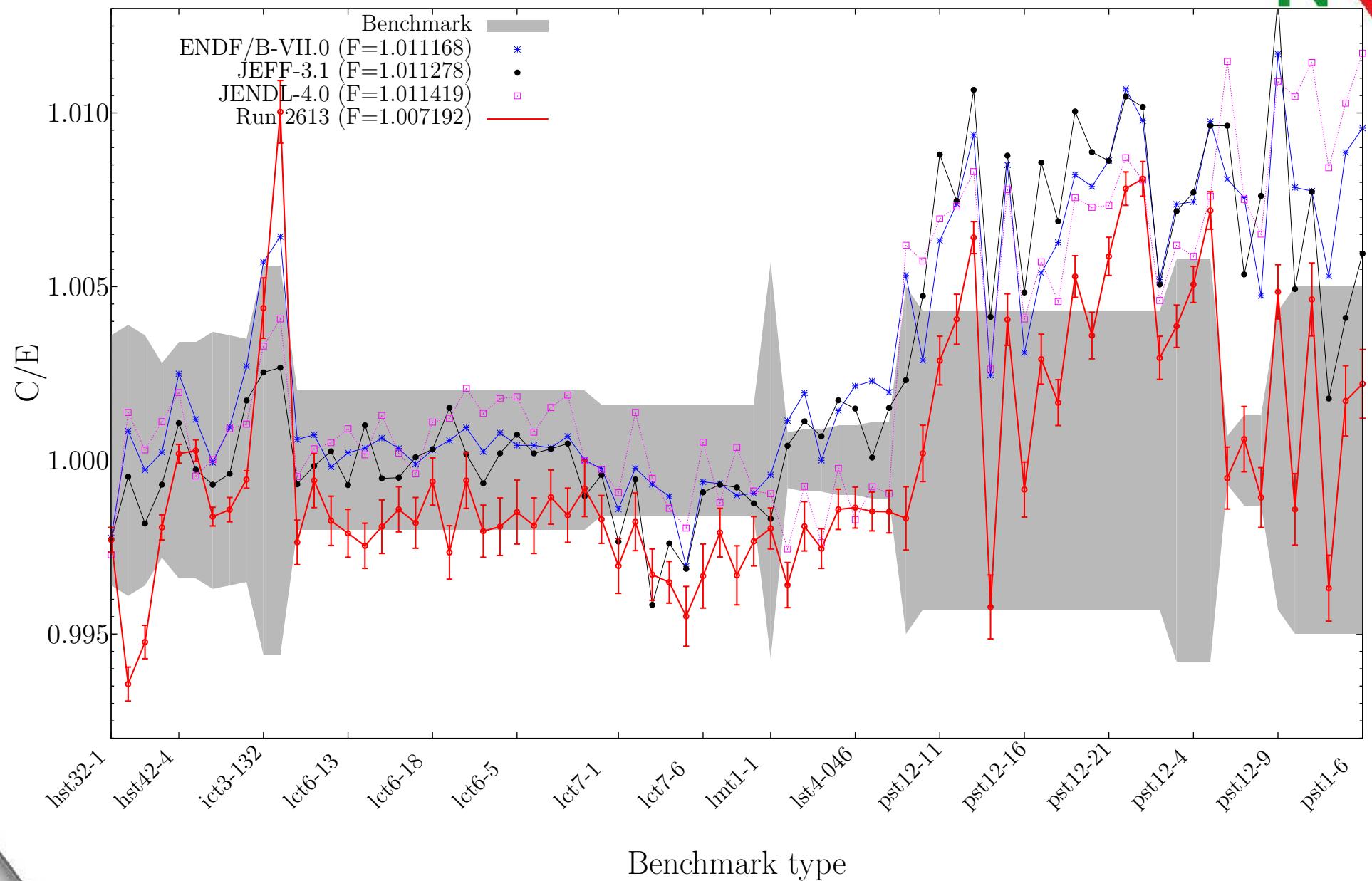
Example on thermal scattering data H in H₂O

NRG



Example on thermal scattering data H in H₂O

NRG



Future work



- ⇒ Continue with TMC (more reactor calculations, applied to current reactors),
- ⇒ Improve the quality of the TENDL library (baseline for TMC, TMC^{-1}),
- ⇒ Apply TMC^{-1} to a large number of isotope (including crit-safety, reactor, dosimetry benchmarks)
- ⇒ Go where covariance methods can not be applied (TMC applied to thermo-hydraulic and transient calculations)
- ⇒ Create the world best nuclear data library (NRG, CCFE, others are welcome)

☺ And finally world domination (and world peace).