

# Overview of current activities for the IE / GRS / NRG Collaborative project

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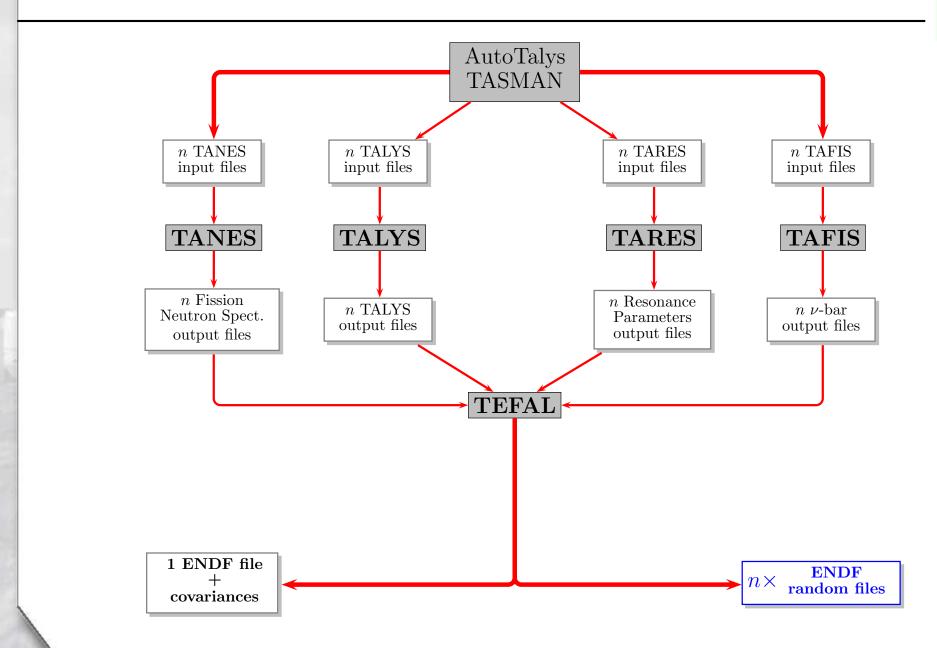
#### NRG field of expertise



In this project we will at first focuss on nuclear data production. For a given isotope, we can provide:

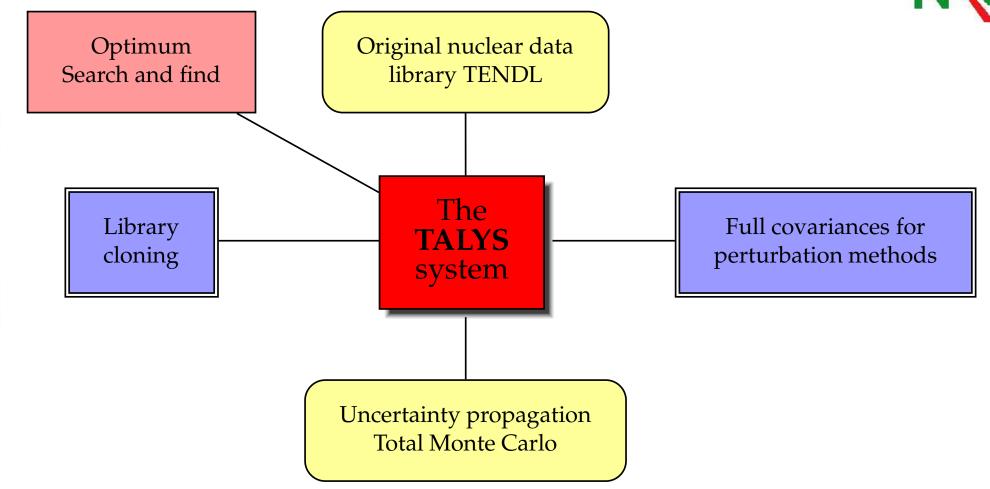
- 1. full evaluation in the ENDF format
- 2. covariance files from thermal to 20 MeV
- 3. "random" files consistent with the covariance file
- 4. Processed files:
  - $\square$  ACE files
  - ☐ Processed covariances in tabular form (different energy groups)
  - □ "covr, ampx" format

#### Nuclear data file evaluation and production with the TALYS system.



#### Possible outcomes based on the TALYS system





#### Our preferred choice: <sup>238</sup>U



We recently have been working on precise evaluation of <sup>238</sup>U:

- 1. Cross sections, nubar, fission neutron spectrum: similar to ENDF/B-VII.0,
- 2. Unique set of covariances: MF31, 32, 33, 34 and 35,
- 3. Covariance files processed with NJOY (33, 44, 187 groups), PUFF (33 groups),
- 4. Partial benchmarking (criticality-safety, burn-up credit),
- 5. A few hundreds of random files will be ready soon.

## Examples of <sup>238</sup>U criticality-safety benchmarking



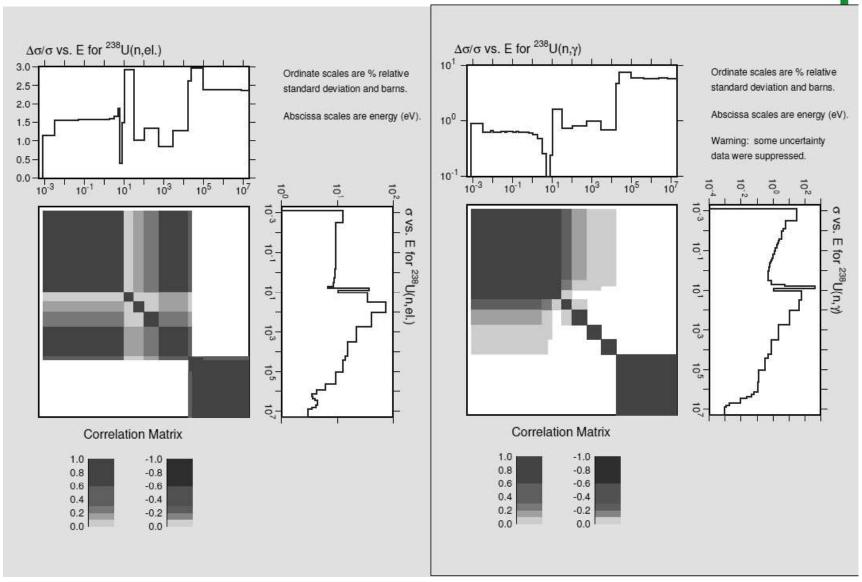
Table 1: Criticality-safety benchmarks calculated with MCNP, changing the <sup>238</sup>U evaluation. The other isotopes are kept equal to ENDF/B-VII.0.

Name	Benchmark		ENDF/B-VII.0		NRG		ENDF/B-VII.0 - NRG	
		(pcm)		(pcm)		(pcm)	(pcm)	(pcm)
hmf1	1.00000	$\pm 100$	1.00009	$\pm 40$	1.00021	$\pm 41$	-12	± 57
hmf28	1.00000	$\pm 300$	1.00243	$\pm$ 42	1.00364	$\pm$ 38	-121	$\pm$ 57
ici1-1	0.96900	$\pm 500$	0.98148	$\pm 31$	0.98180	$\pm 32$	-32	$\pm$ 44
lct1-1	1.00000	$\pm 310$	1.00015	$\pm 73$	0.99858	$\pm 71$	157	$\pm 102$
lct1-2	0.99980	$\pm 300$	1.00039	$\pm 76$	0.99978	$\pm 71$	60	$\pm 104$
lct1-3	0.99980	$\pm 300$	0.99774	$\pm$ 75	0.99764	$\pm$ 66	10	$\pm 100$

Average absolute deviation between NRG and ENDF/B-VII.0 for 120 benchmarks:  $108 \pm 101$ 

### **Covariance examples**





#### **Conclusions**



- 1. We have a large experience in nuclear data and ENDF file production (contributions to different libraries: JEFF, ENDF/B-VII.0, FENDL, TENDL...).
- 2. The <sup>238</sup>U isotope would be our preferred choice for this project, but other isotopes can be added later.