

# TENDL-2009: on Evaluations and Monte Carlo Covariances

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# Contents

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① Motivations:

$\implies$  *a roadmap to consistent and state-of-the-art evaluations*

② Concept:

$\implies$  *TALYS system + Monte Carlo = TENDL-2009*

③ Content 1:

$\implies$  *Neutrons, protons, deuterons, tritons, alphas, photons*

④ Content 2:

$\implies$  *Neutrons:  $^{19}\text{F}$  to  $^{281}\text{Ds}$  ( $Z=110$ ), from MF-1 to MF-34*

⑤ Covariances

⑥ Validations

⑦ Conclusions and Future Improvements

## Short-term goal:

We need a **consistent, complete and reproducible** nuclear data library to be integrated in reactor calculations.

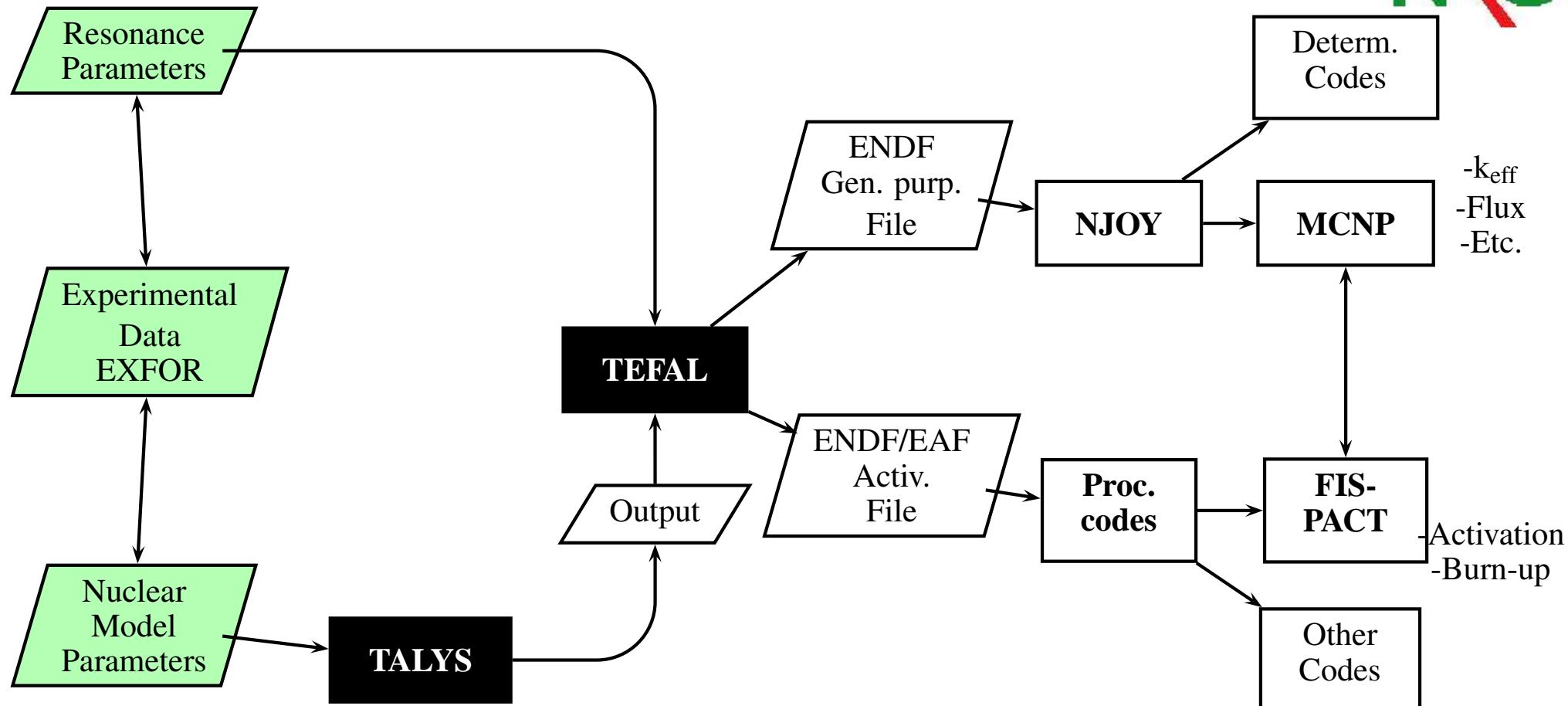
*(None of the existing libraries fulfill these requirements.)*

## Long-term goal:

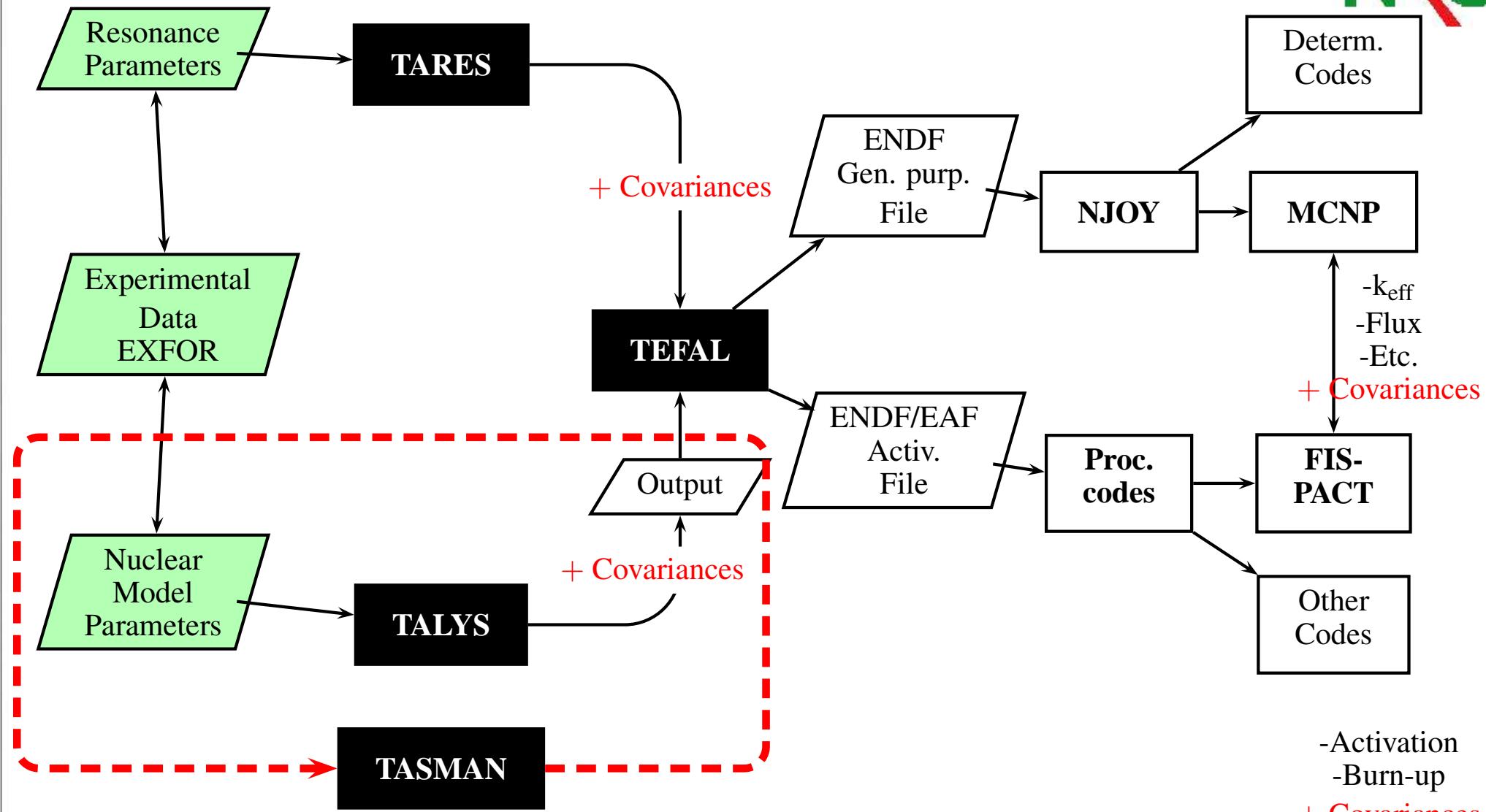
Generate nuclear data on the flight  $\implies$  no need of libraries anymore.

*(See last slide.)*

# Concept: Standard nuclear data scheme



# Concept: Nuclear Data Scheme with covariances



Monte Carlo: 1000 TALYS runs

-Activation  
-Burn-up  
+ Covariances

# Content 1- TENDL-2009

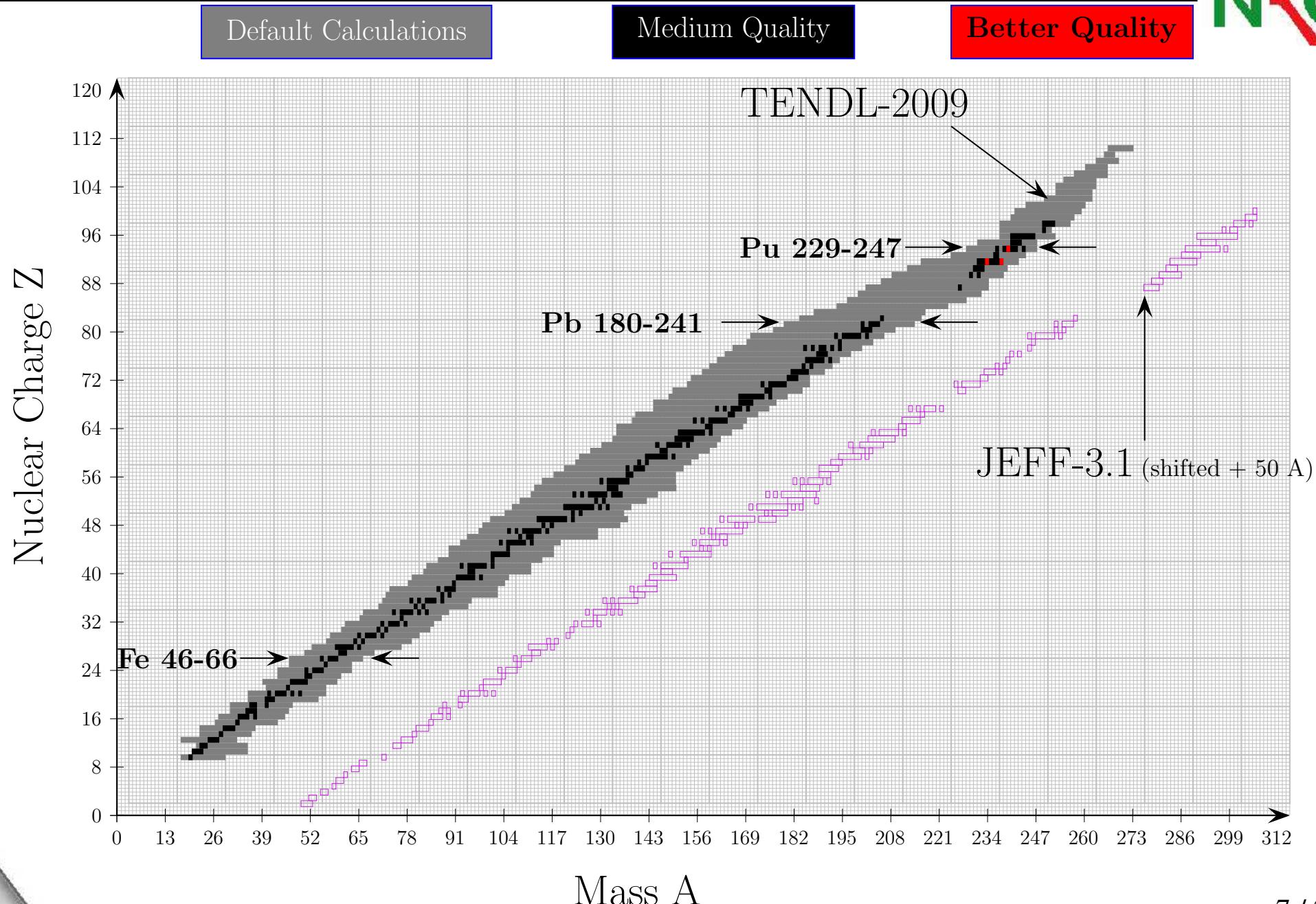


- Neutrons: ENDF files, plots, ACE files
- Protons: ENDF files, ACE files
- Alphas: ENDF files, ACE files
- ...

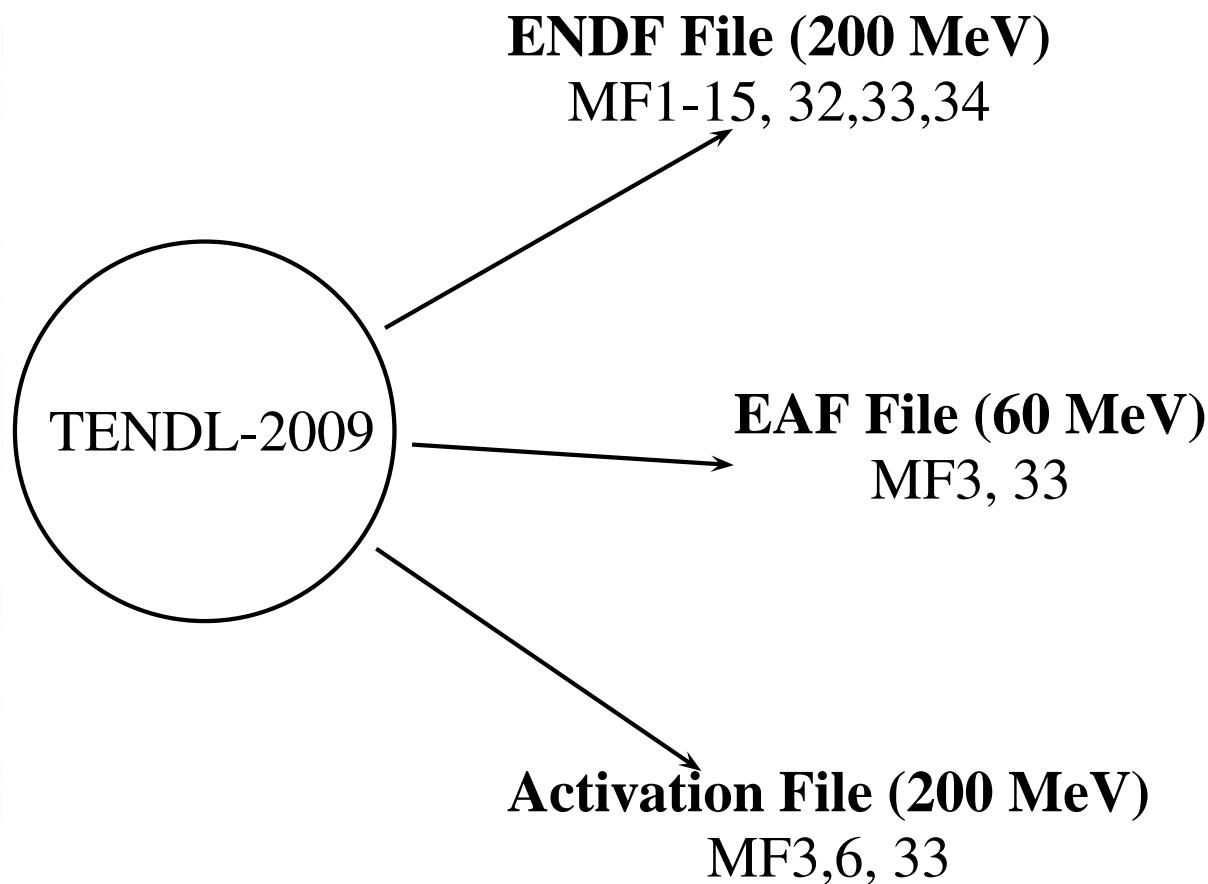
	Neutron	Proton	Deuteron	Triton	Alpha	Helium3	Photon
TENDL-2009	>1000	>1000	>1000	>1000	>1000	>1000	>1000
TENDL-2008	348	344	336	339	342	338	327
(JEFF-3.1)	381	26					
(ENDF/B-VII.0)	393	48	5	3			163

# Content 2- TENDL-2009 Neutron library: $^{19}\text{F}$ to $^{281}\text{Ds}$ ( $t_{1/2} > 1 \text{ sec}$ )

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## Content 3- TENDL-2009 Transport, Activation and EAF files



## Content 4- TENDL-2009 Neutron library: from MF-1 to MF-34



Content of a typical file up to **200 MeV** (out of  $> 1000$  files):

- ☞ **MF-1:** Description + fission parameters
- ☞ **MF-2:** Resonance parameters (Reich-Moore or Multi-level Breit Wigner)
- ☞ **MF-3:** Cross sections (n,tot), (n,el), (n,non), (n,inl<sub>i</sub>), ..., (n, $\gamma$ ), (n,p<sub>i</sub>), (n, $\alpha$ <sub>i</sub>)
- ☞ **MF-4:** Elastic angular distribution (Legendre Polynomials)
- ☞ **MF-5:** Fission neutron spectrum
- ☞ **MF-6:** Double differential distributions and spectra for (n,2n), ..., (n, $\alpha$ <sub>i</sub>)
- ☞ **MF- 8-10:** Isomeric cross sections
- ☞ **MF- 12-15:** Gamma yields, angular distributions and spectra
- ☞ **MF- 32-33-34:** Resonance parameter covariances, cross section covariances, elastic angular distribution covariances

## Content 5- Available files

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- ① Tabular angular distributions
- ② Tabular Gamma-ray intensities
- ③ Tabular partial cross sections to discrete levels
- ④ Tabular residual cross sections
- ⑤ Tabular cross sections
- ⑥ ENDF files
- ⑦ EAF cross section and variance files
- ⑧ Processed ACE files (with NJOY)
- ⑨ Processed AMPX files (with PUFF)
- ⑩ Processed covariances (tabular and plots)

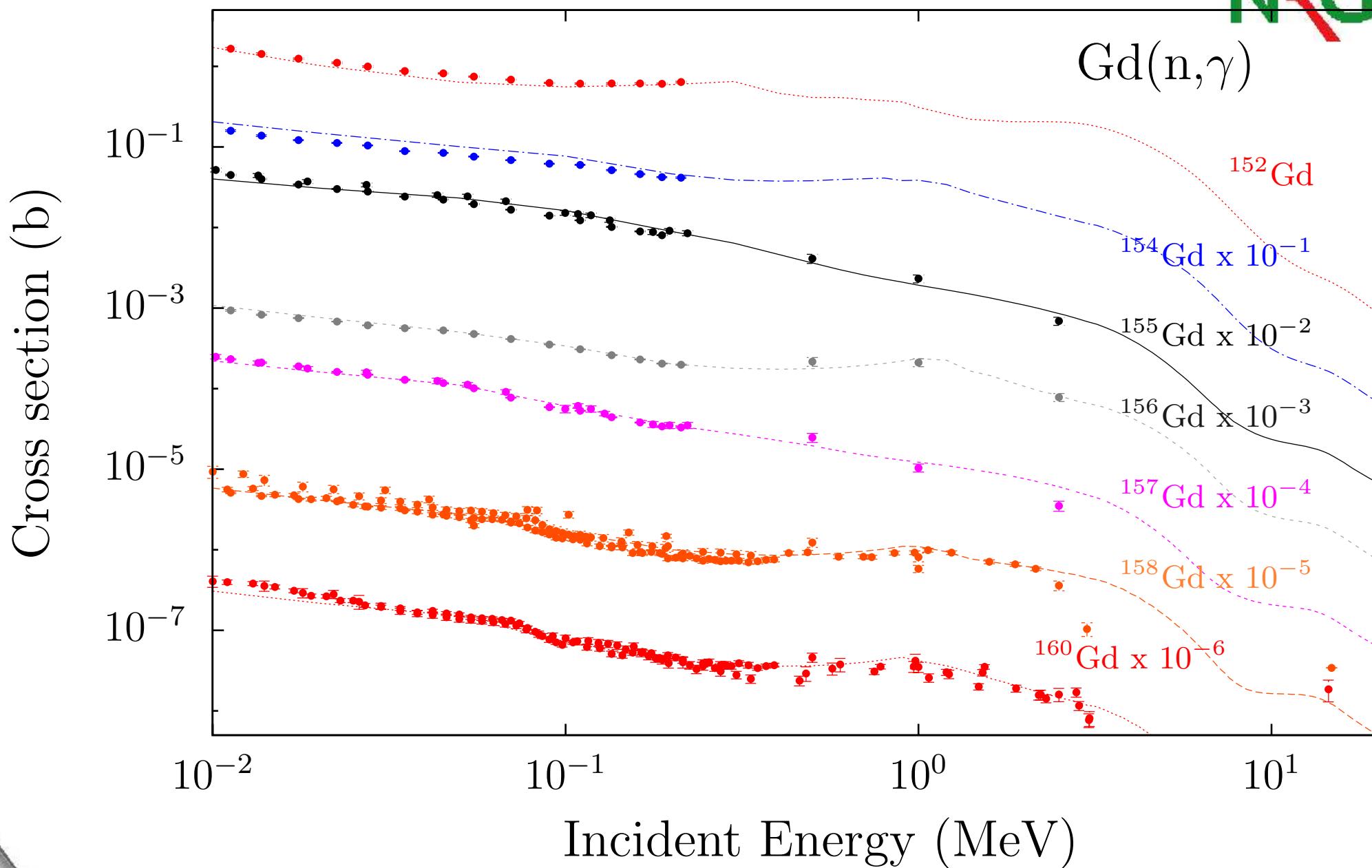
## Content 6- TENDL-2009: Actinides



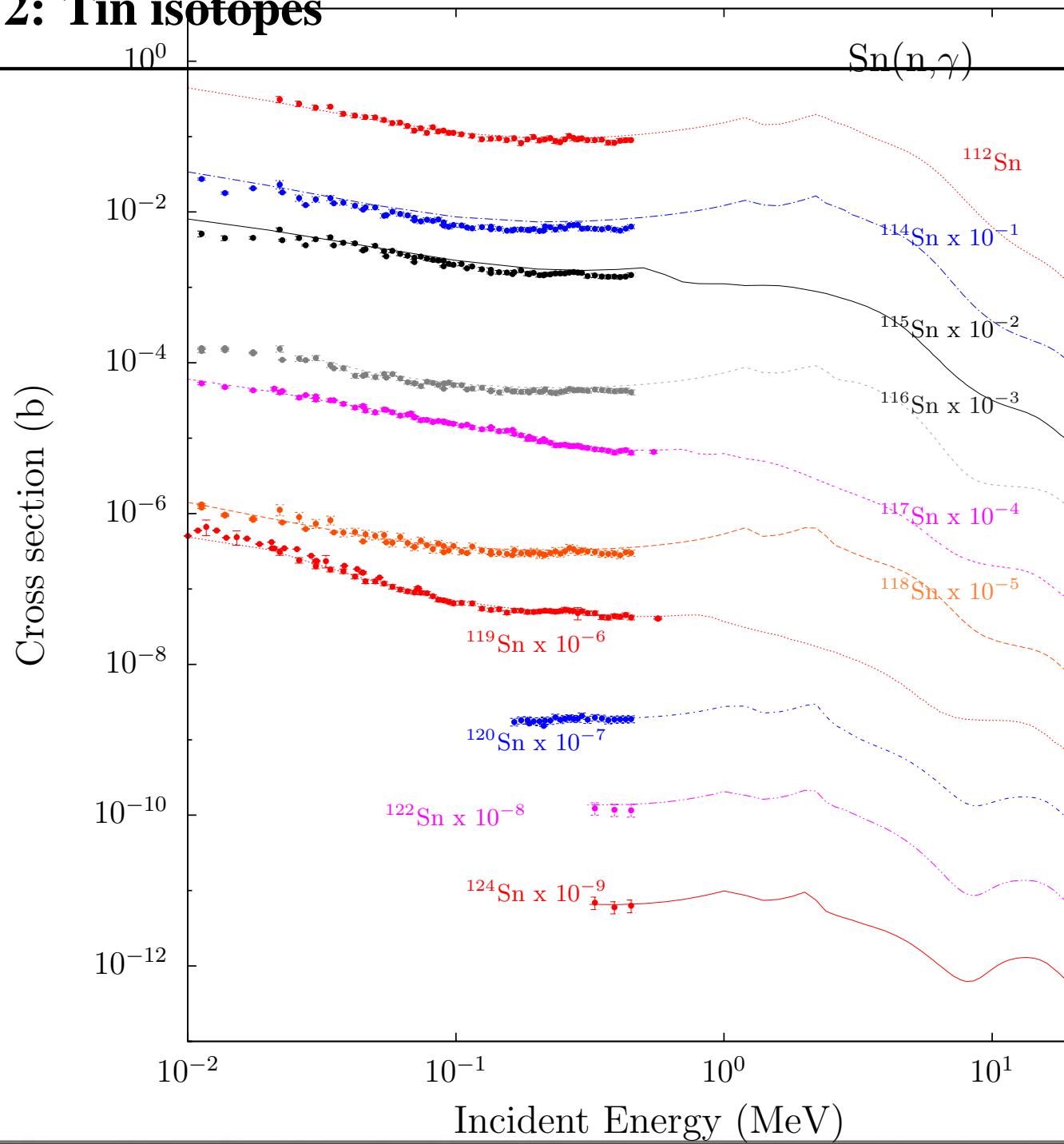
Element	Isotopes	Element	Isotopes
Th	208-227, <a href="#">228-232</a> , 233-239	Pa	213-230, <a href="#">231,233</a> , 234-239
U	222-232, <a href="#">233-238</a> , 239-242	Np	225-236, <a href="#">237</a> , 238-244
Pu	229-237, <a href="#">238-244</a> , 245-247	Am	232-238, <a href="#">239-243</a> , 245-247
Cm	238-242, <a href="#">243-248</a> , 249-252	Bk	238-248, <a href="#">249</a> , 250, 251
Cf	237-248, <a href="#">249</a> , 250-256	Es	241-253, <a href="#">254</a> , 255-257
Fm	242-257	Md	245-260
No...Ds			

⇒ 36 "medium" or "**better**" quality evaluations and 406 "default" quality evaluations (with isomers)

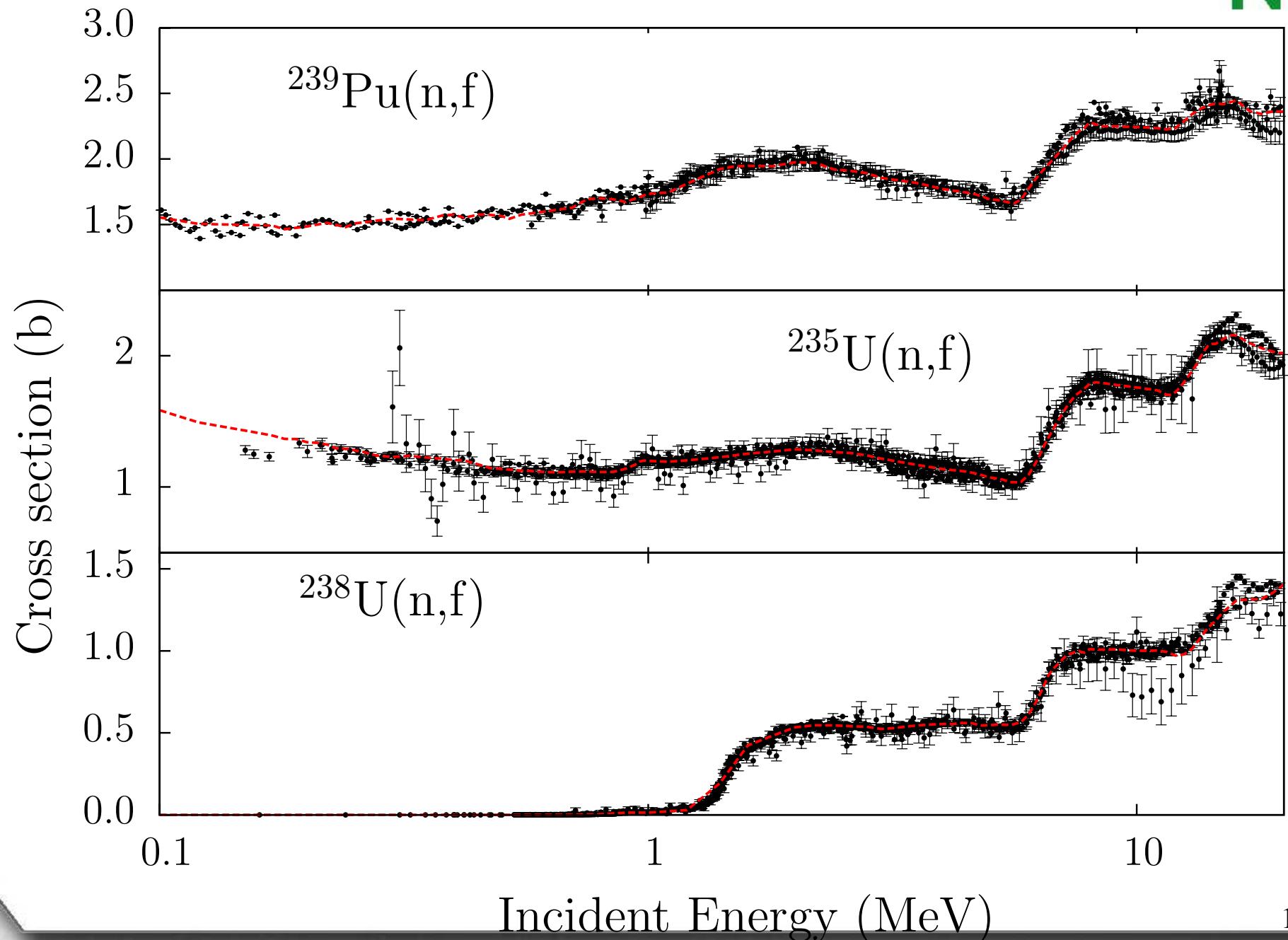
# Examples 1: Gadolinium isotopes



## Examples 2: Tin isotopes



## Examples 3: Some actinides



# Covariance generation with TALYS via Monte Carlo



Let  $\vec{\mathbf{p}}$  be the vector of the  $L$  adjustable nuclear model parameters that are relevant to the problem under consideration, *i.e.*

$$\vec{\mathbf{p}} = \{p_1, \dots, p_l, \dots, p_L\} \implies p_l^{(k)} = p_l^{(0)} \pm \Delta p_l, \quad l = 1, L$$

The basis of our method is to let TALYS perform many calculations:

$$\vec{\sigma}^{(k)} = T(\vec{\mathbf{p}}^{(k)})$$

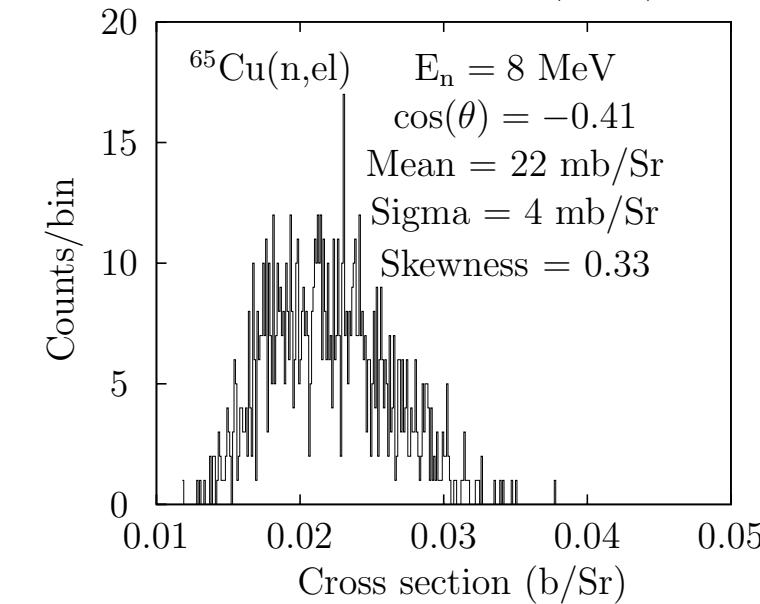
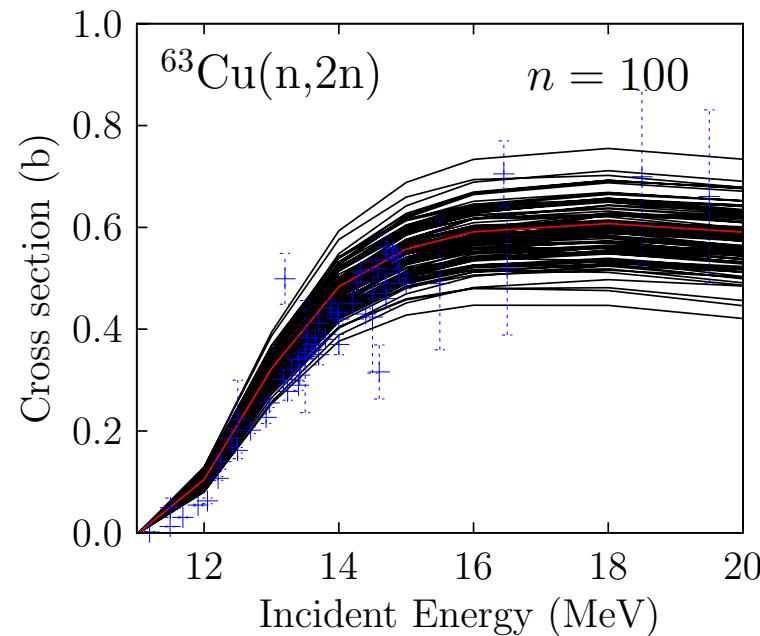
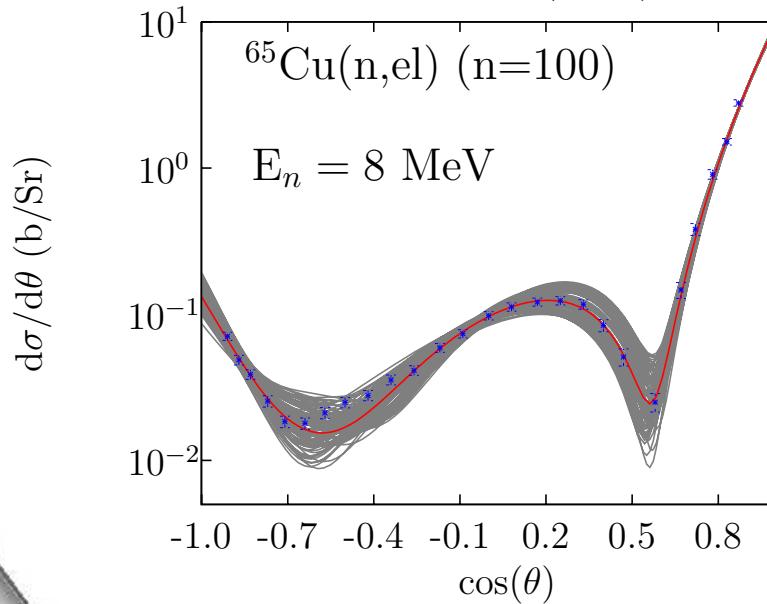
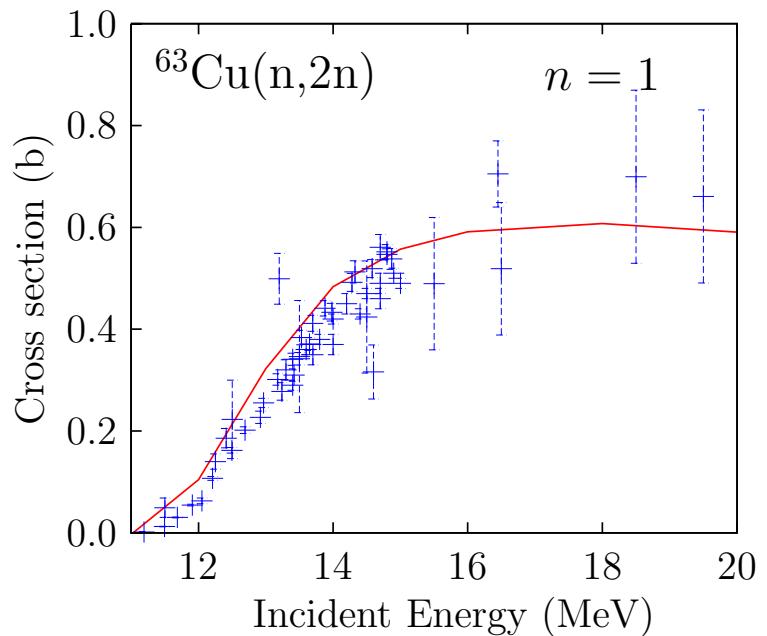
The average covariance matrix for cross sections is given by

$$V_{ij} = \frac{1}{K} \sum_{k=1}^K (\sigma_i^{(k)} - \bar{\sigma}_i)(\sigma_j^{(k)} - \bar{\sigma}_j), \quad i, j = 1, N,$$

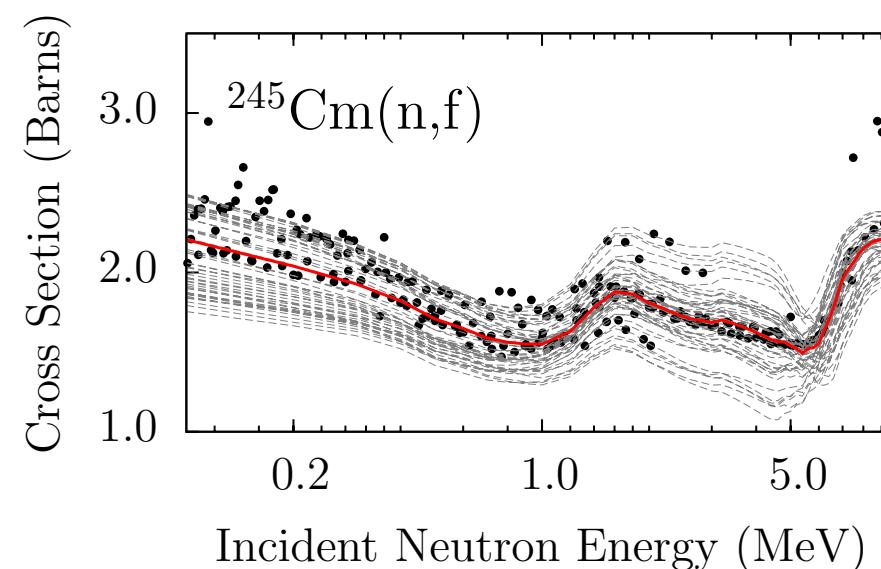
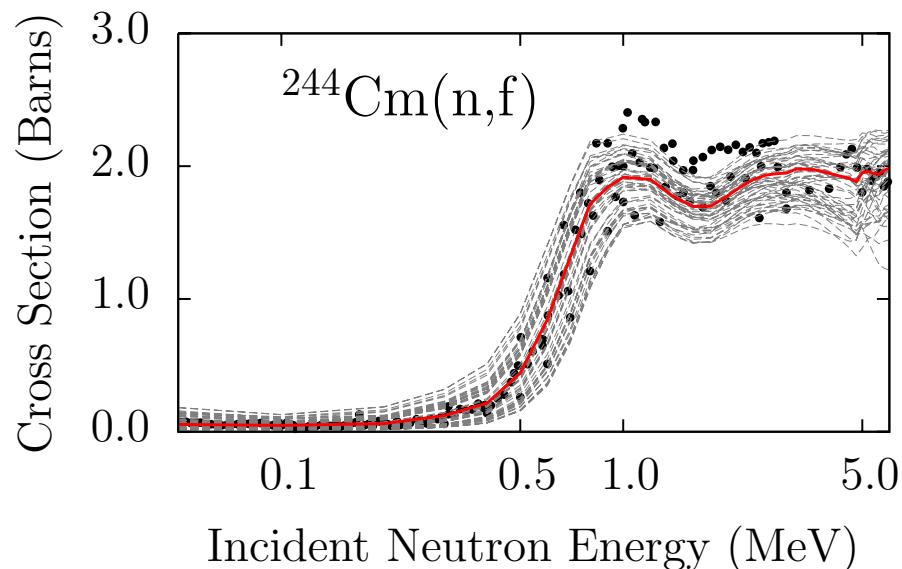
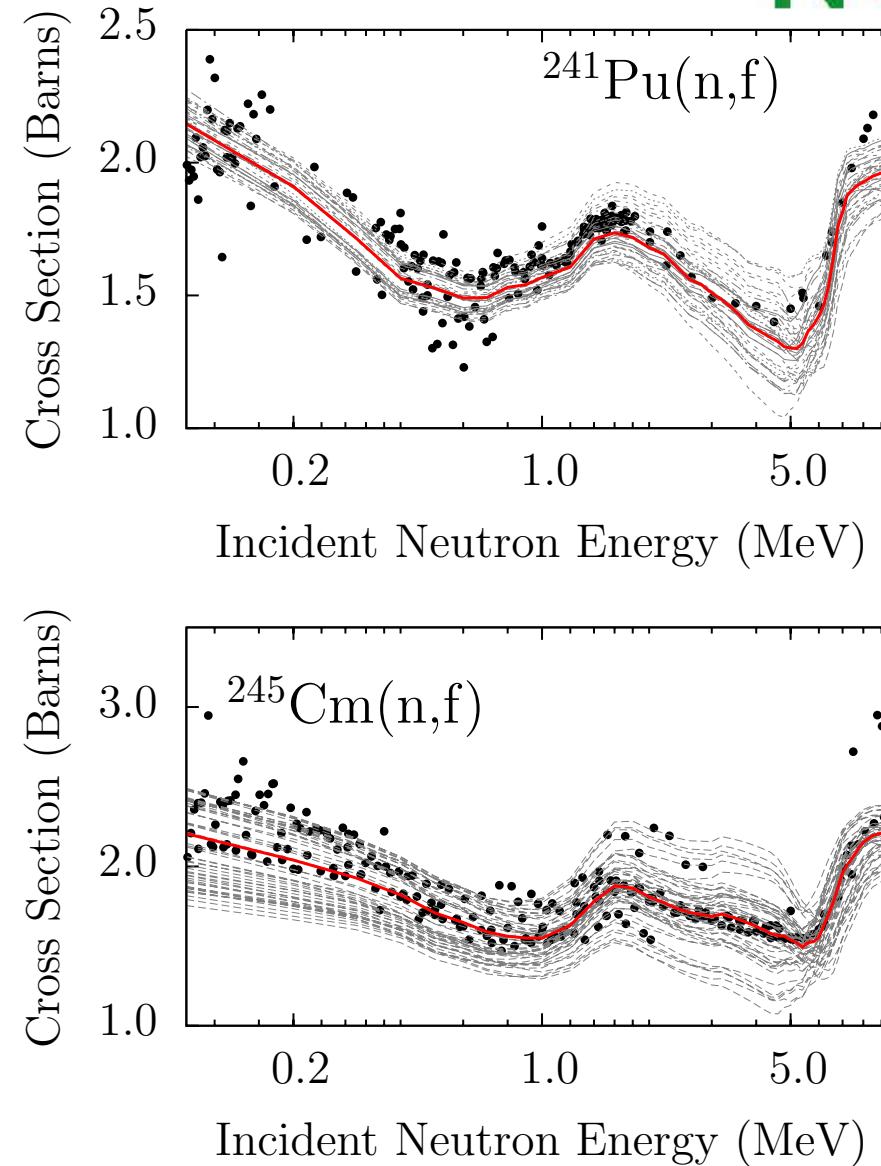
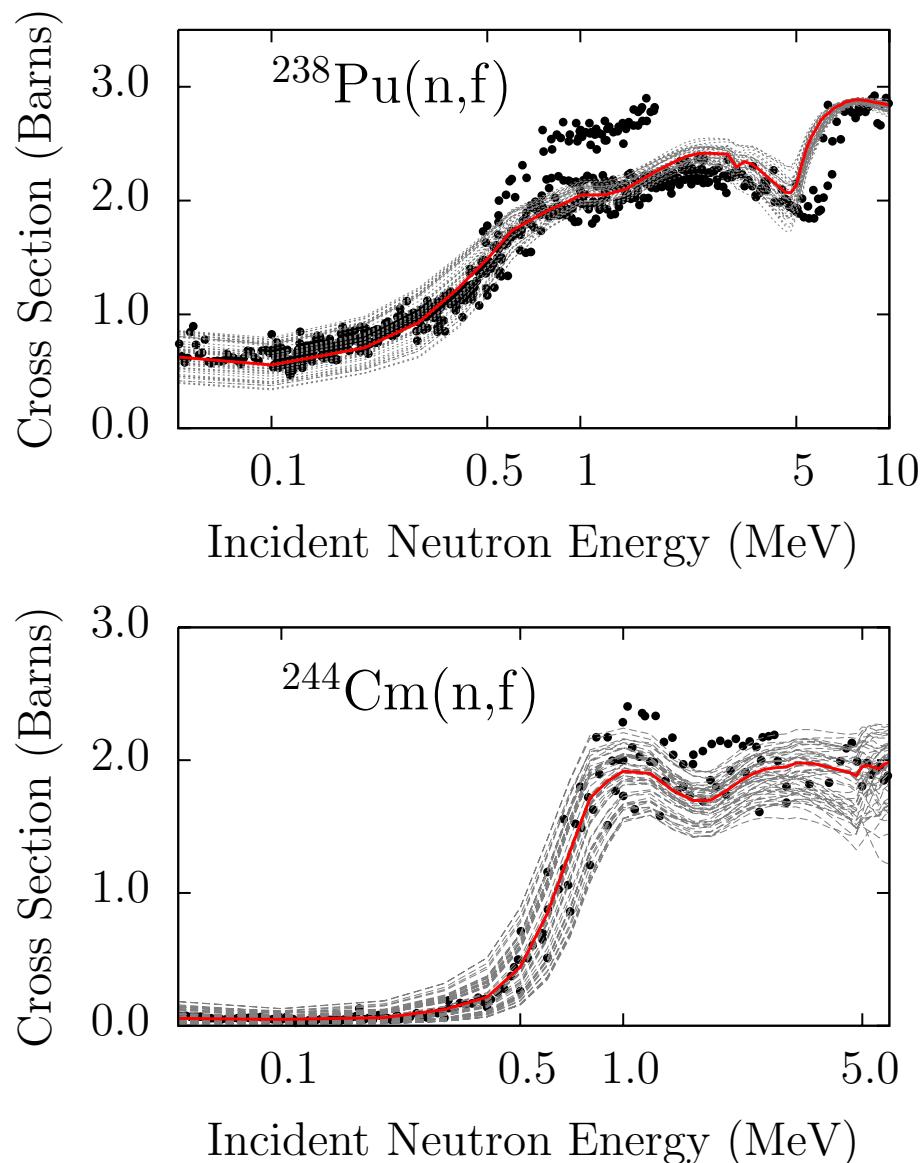
where  $K$  is the total number of TALYS runs needed for statistical convergence.  
The average calculated cross sections are

$$\bar{\sigma}_i = \frac{1}{K} \sum_{k=1}^K \sigma_i^{(k)}, \quad i = 1, N,$$

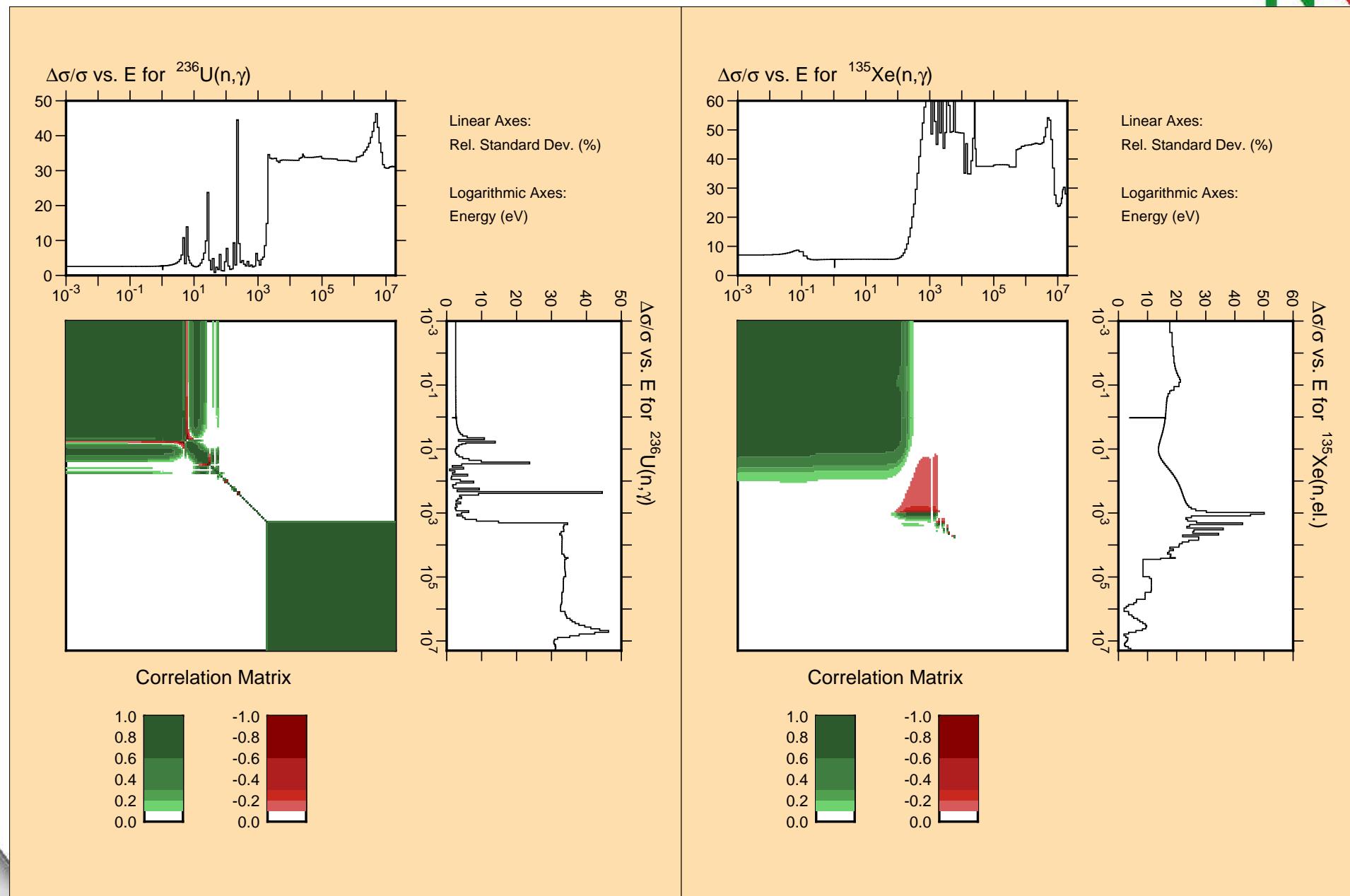
# Covariances: example



# Covariances: example



# Covariances: example



# TENDL-2009: validation



Table 1: Number of benchmarks per main ICSBEP category used in the validation of TENDL-2009 $\beta$ .

Category	Energy Range	Number	Category	Energy Range	Number
LEU	Thermal	285	$^{239}\text{Pu}$	Thermal	105
HEU	Thermal	122	$^{239}\text{Pu}$	Fast	6
HEU	Fast	64	$^{239}\text{Pu}$	Mixed	6
HEU	Inter	9	Other	Thermal	48
HEU	Mixed	5	Other	Fast	19
$^{233}\text{U}$	Thermal	13	$^{233}\text{U}$	Fast	4
Total		686			

# Validation: global comparison of 686 crit-safety benchmarks



As a large number of results are available, a global normalised  $\chi^2_{\text{N,library}}$  is calculated over all  $k_{\text{eff}}$ s to facilitate comparison between libraries. The following formula is used for each library ( $C$  being the calculated value and  $E$  the value of the benchmark):

$$\chi^2_{\text{library}} = \sum_i^{\text{benchmarks}} \frac{(C_i^{\text{library}} - E_i)^2}{E_i}$$

Then all  $\chi^2_{\text{library}}$  for the five libraries are normalized to  $\chi^2_{\text{ENDF/B-VII.0}}$ :

$$\chi^2_{\text{Normalized,library}} = \frac{\chi^2_{\text{library}}}{\chi^2_{\text{ENDF/B-VII.0}}}$$

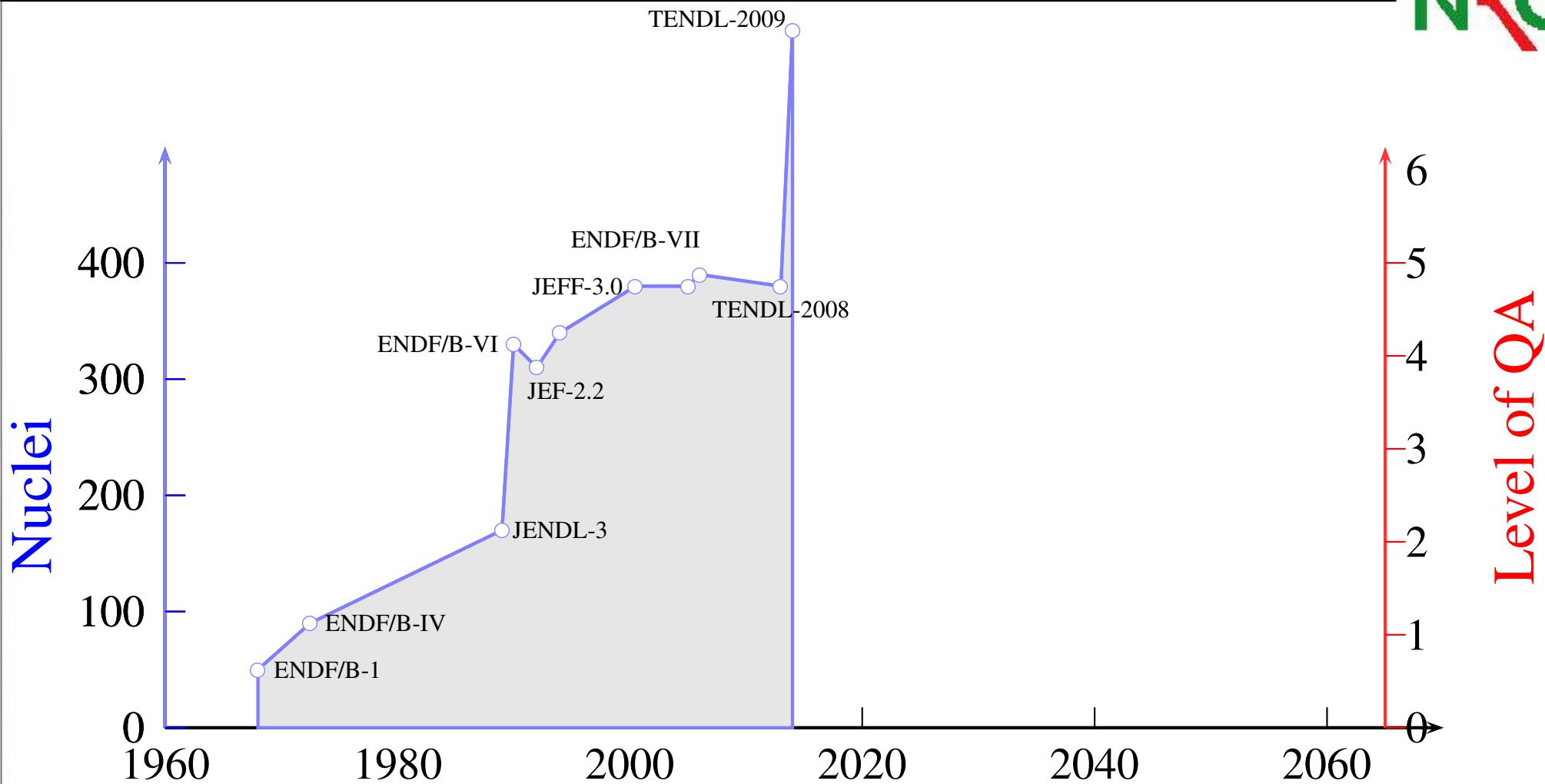
$$\text{library} = \begin{pmatrix} \text{ENDF/B - VII.0} \\ \text{TENDL - 2009}\beta \\ \text{JEFF - 3.1} \\ \text{JENDL - 3.3} \\ \text{ENDF/B - VI.8} \end{pmatrix} \quad (1)$$

# Validation: global $\chi^2$ comparison of 686 crit-safety benchmarks

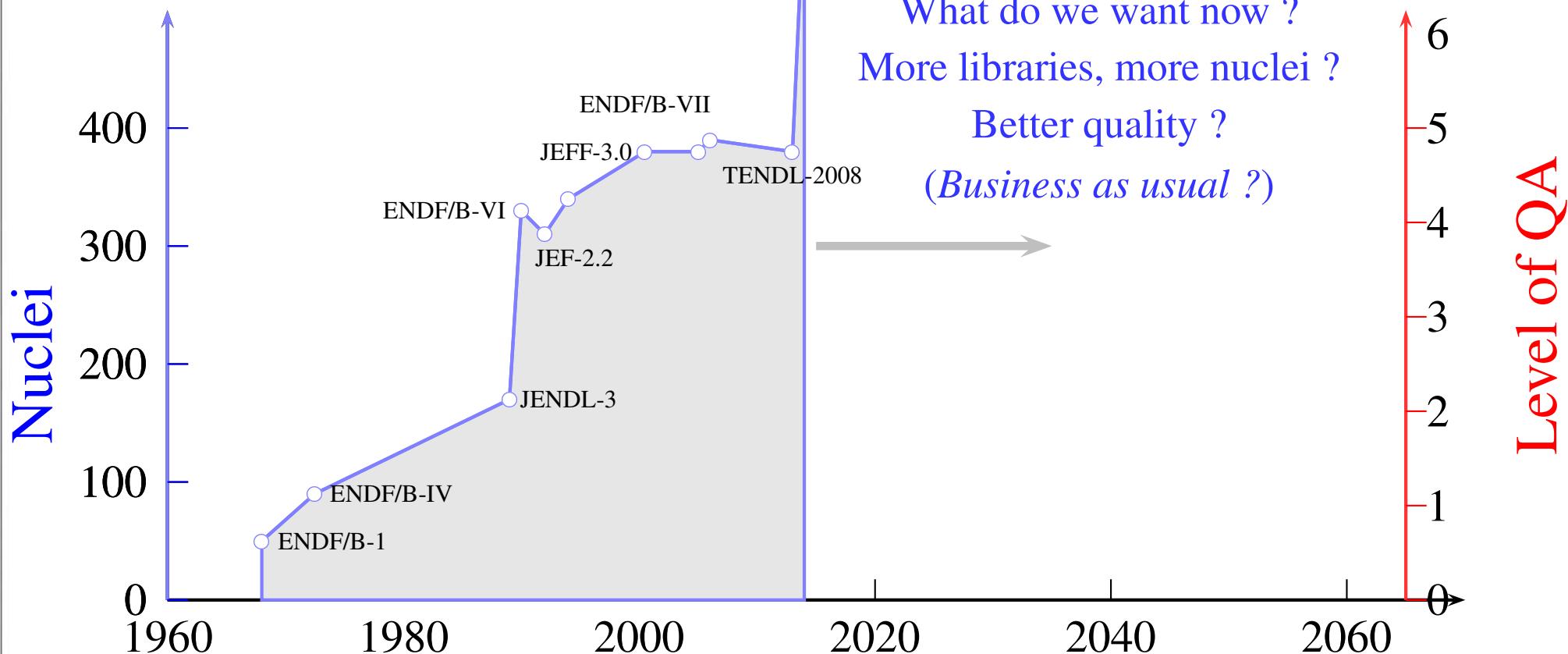


Library	Global 686	Normalized $\chi^2_N$				
		LEU 285	HEU 200	$^{239}\text{Pu}$ 117	Other 67	$^{233}\text{U}$ 17
ENDF/B-VII.0	1.00	1.00	1.00	1.00	<b>1.00</b>	1.00
ENDF/B-VI.8	1.19	1.68	1.24	0.88	1.12	2.52
JEFF-3.1	1.13	<b>0.91</b>	1.39	<b>0.85</b>	1.04	1.68
JENDL-3.3	1.30	1.04	1.42	1.39	1.12	<b>0.42</b>
TENDL-2009 $\beta$	<b>0.98</b>	1.05	<b>0.97</b>	0.93	1.01	0.90

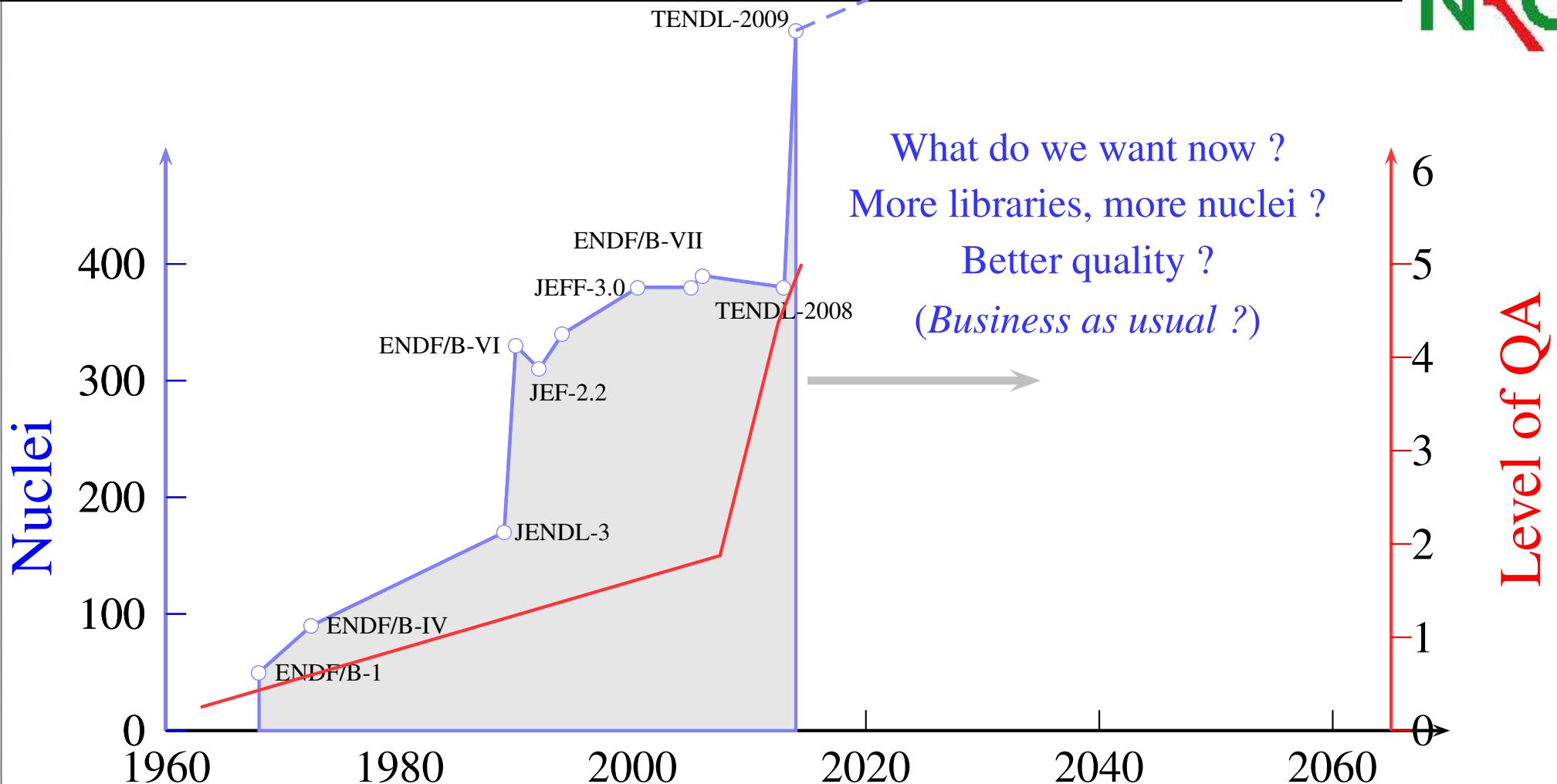
# Back to the motivations: where are we ?



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QA 0: All manual work

QA 1: Manual/auto + Checkr, Fizcon, Psyche

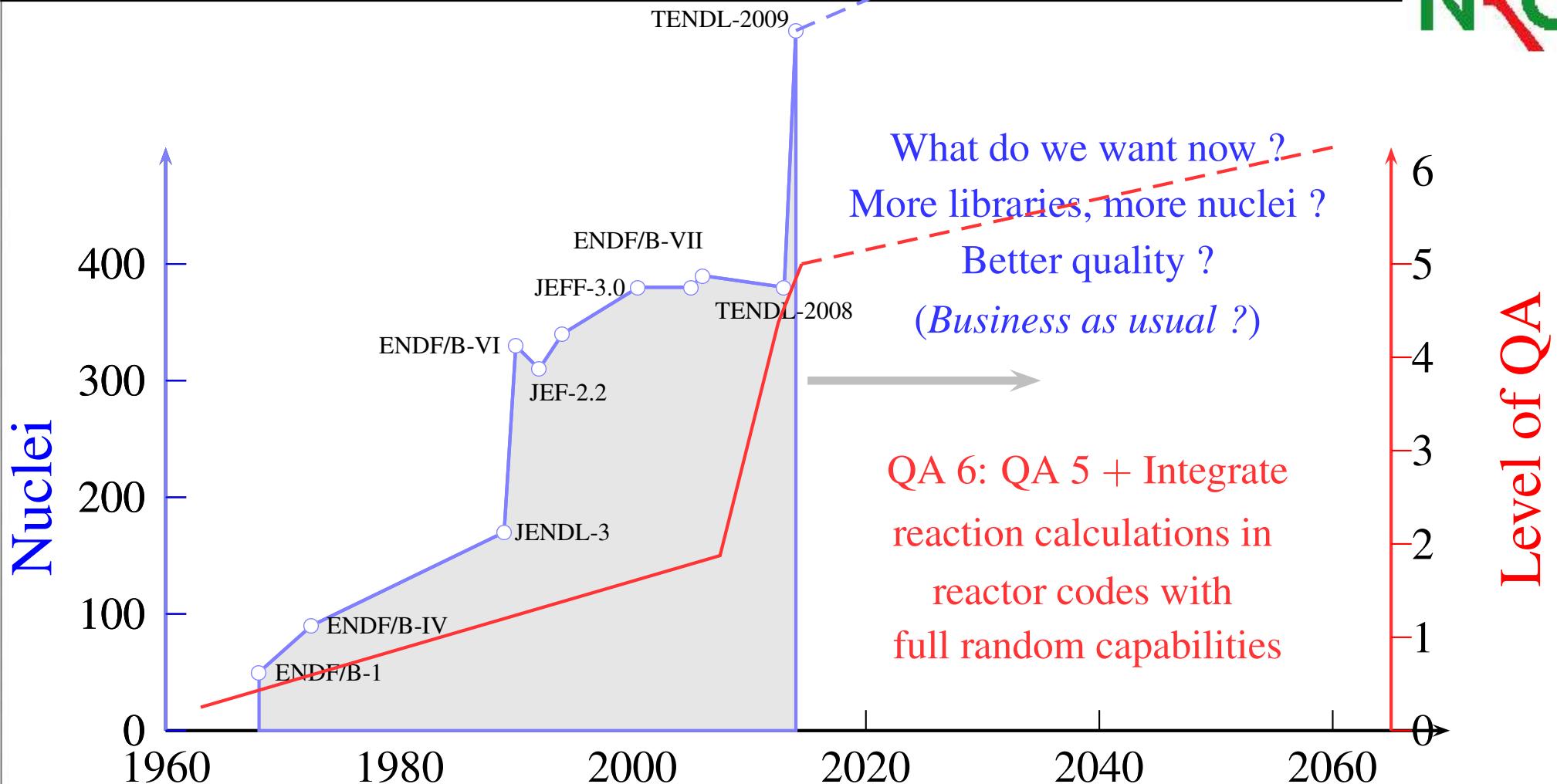
QA 2: QA 1 + Njoy, MCNP

QA 3: QA 2 + other codes (Puff, Eranos...)

QA 4: QA 3 + completeness, reproducibility

QA 5: QA 4 + full automation

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# Conclusions and Future improvements



- 👉 Consistent, complete data files with automatic updates for all actinides, projectiles, energies reaction channels and quantities,
- 👉 Extensive set of covariance files,
- 👉 Reproducible with TALYS-1.2 (released with the library and reference papers)
- 👉 Best TALYS and resonance parameters fitting

Release date: End of  
November 2009 !

- 👉 Improve global model and uncertainties
- 👉 More extensive validation (shielding...)
- 👉 Addition of original URR