

TMC vs. perturbation

and other applications

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All slides can be found at:

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ftp://ftp.nrg.eu/pub/www/talys/bib_rochman/presentation.html.

Introduction to nuclear data uncertainties

General comments:

- I uncertainties are not errors (and vice versa),
- I uncertainties should now be given with all data (seems obvious ?),
- III they are related to risks, quality of work, money, perception, fear, safety...

Uncertainty \rightleftharpoons safety \rightleftharpoons professionalism

- ₩ True uncertainties do not exist ! They are the reflection of our knowledge and methods.
- I All the above for covariances
- Image: The importance of nuclear data uncertainties should be checked. If believed negligible, please prove it !

Backbone of our methodology: REPRODUCIBILITY





TMC for nuclear data uncertainty propagation, what else ?

- \bigcirc + No covariance matrices (no 2 Gb files) but every possible cross correlation included,
- \bigcirc + No approximation but true probability distribution,
- \bigcirc + Only essential info for an evaluation is stored, (
 - + No perturbation code necessary, only "essential" codes,
 - + Feedback to model parameters,
 - + Full reactor core calculation and transient,
- \bigcirc + Also applicable to fission yields, thermal scattering, pseudo-fission products, all isotopes (...just everything),
- (+ Other variants: AREVA (NUDUNA), GRS (XSUSA), CIEMAT (ACAB), PSI (NUSS), CNRS Grenoble..., based on covariance files,
- (\cdot) + Many spin-offs (TENDL covariances, sensitivity, adjustment...)

 $(\dot{})$ + QA.

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- Needs discipline to reproduce,
- Memory and computer time (not human time),
- Need mentality change.



For each random ENDF file, the benchmark calculation is performed with MCNP. At the end of the *n* calculations, *n* different k_{eff} values are obtained.

 $\sigma_{\text{total}}^2 = \sigma_{\text{statistics}}^2 + \sigma_{\text{nuclear data}}^2$



Example with ²³⁸U: Monte Carlo calculations



Examples with ⁶³Cu(n,2n) and ⁶⁵Cu(n,el)





TMC versus Perturbation method

- ① Obtain uncertainties on large-scale models due to nuclear data uncertainties
- ② Systematic approach, reliable and reproducable

Solution (1): Total Monte Carlo



Solution (2): Perturbation method

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 \implies MCNP+ Perturbation cards+covariance files

TMC versus Perturbation: Results

Comparison TMC-Perturbation methods for a few k_{eff} benchmarks. The ratio in the last column is "TMC over Perturbation".

		Total Monte Carlo	Perturbation	Ratio
Benchmark	Isotopes	Uncertainty	Uncertainty	
		due to nuclear	due to nuclear	
		data (pcm)	data (pcm)	
hst39-6	¹⁹ F	330	290	1.16
hmf7-34	¹⁹ F	350	290	1.21
ict3-132	90 Zr	190	150	1.29
hmf57-1	²⁰⁸ Pb	500	410	1.22
pmf2	²³⁹ Pu	840	720	1.16
pmf2	²⁴⁰ Pu	790	650	1.21

Results: Details of the TMC-Perturbation methods for ^{239,240}Pu k_{eff} benchmarks

	pn	nf2 ²³⁹ Pu	pmf2 ²⁴⁰ Pu			
	$\Delta \mathbf{k}$	K _{eff} (pcm)	Δk_{eff} (pcm)			
	TMC	Perturbation	TMC	Perturbation		
Total	840	720	790	650		
MF1	400	-	370	-		
(n,inl)	170	140	70	50		
(n,el)	250	240	30	40		
(n,γ)	100	100	30	30		
(n,f)	720	660	730	640		
MF4	20	_	20	_		
MF5	50	_	30	_		
MF6	50	_	30	_		

Considered data in TMC (or fast TMC)



Several hundreds of random ENDF files for transport + depletion

- 3 Major actinides: ²³⁵U, ²³⁸U, ²³⁹Pu,
- Light elements: lighter than oxygen,
- 2 Thermal scattering data: H in H_2O , D in D_2O
- All Fission yields (e.g. ^{234,235,236,238}U, ^{239,240,241}Pu, ²³⁷Np, ^{241,243}Am, ^{243,244}Cm),
- All Minor actinides (e.g. ^{234,236,237}U, ²³⁷Np, ^{238,240,241,242}Pu, Am, Cm),
- All fission products (e.g. from Ge to Er), and decay data,

(fast) TMC can be applied to any input data, propagating uncertainties to any outputs



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TMC was already applied to

- criticality-safety, shielding, pincell/assembly burn-up, activation,
- PWR, BWR, Gen-IV systems,
- UO₂, MOX fuels,
 - MCNP, SERPENT, FISPACT, DRAGON, PANTHER, RELAP-5

Comparison of Δk_{∞} for assemblies and full core (SERPENT)



TMC applied to PWR assembly burn-up calculations with DRAGON N



Example of TMC for the impact of the matrix fuel



PWR MOX assembly uncertainties



- 1 MOX assembly surrounded by UO₂ assemblies,
- Burn-up calculated with SERPENT,
- All major nuclear data taken into account.

Effect of H in H₂O for a full core PWR (courtesy of O. Cabellos, UPM, Spain)

Method: TMC applied to COBAYA (3D multigroup core calculations) + SIMULA (coupled neutronic-thermohydraulics 3D core calculations)

		NDU TSII	J STR UPM	IALE	S				IJ	I.2 P\	NR p	roble	em de	escri	ption
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2	13	11	15	2	16	6	20	7					F OFA	3.24	0
3	4	15	3	21	8	22	19						E OFA	3.24	0
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There is a solution with Monte Carlo codes.



If a single calculation takes *m* histories (σ_{stat} small enough), then repeat it *n* times with *m/n* histories, random nuclear data and random seeds.

 $\sigma_{\text{total}}^2 = \sigma_{\text{statistics}}^2 + \sigma_{\text{nuclear data}}^2$ still holds.



2013: fast TMC method... If a single calculation takes *m* histories (σ_{stat} small enough), then repeat it *n* times with *m/n* histories, random nuclear data and random seeds. $\sigma_{\text{total}}^2 = \sigma_{\text{statistics}}^2 + \sigma_{\text{nuclear data}}^2$ still holds. run 0 ENDF/B-VII.1 seed s₀ *m* histories T sec. $k \pm \sigma_{\text{stat}}$

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2013: fa	st TMC metho	d			
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n runs				T sec.			

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n runs		$\left\{ \begin{array}{c} \sigma_{total}^2 \\ \end{array} \right.$	$=\frac{1}{n-1}\sum_{i=1}^{n}$	T sec. $\sum_{i=1}^{n} (k_i - \overline{k})^2 \checkmark$	
					D. Rochman $-22/26$

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•		•			•
run n	nuclear data <i>n</i>	seed s_n	m/n hist.	T/n sec.	$\mathbf{k}_n \pm \mathbf{\sigma}_n$
n runs		$\begin{cases} \sigma_{total}^{2} \\ \sigma_{statistic}^{2} \end{cases}$	$= \frac{1}{n-1} \sum_{i=1}^{n} \sum_{i=1}^{n} \mathbf{o}_{i}$	T sec. $\int_{1}^{2} (k_i - \overline{k})^2 \ll$ $\int_{1}^{2} (k_i - \overline{k})^2 \ll$	
-					D. Rochman – 22 / 26

The fast TMC method

- \odot as fast as S/U methods (1-2 × longer than 1 single calculation *without* uncertainties),
- \odot tested on criticality & shielding benchmarks, burn-up (k_{eff} and inventory),



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MCNP model: 241 fuel assemblies, with 264 fuel pins each



 \implies 357 × 357 × 100 regions (1.26 × 1.26 × 3.66 cm³): 12.7 million cells

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 $\implies 357 \times 357 \times 100 \text{ regions } (1.26 \times 1.26 \times 3.66 \text{ cm}^3): 12.7 \text{ million cells}$ Uncertainty on generated local pin power (tally f7) due to ²³⁵U, ²³⁸U, ²³⁹Pu and H in H₂O thermal scattering in each cell ?

Fast TMC method

1 normal calculation without nuclear data uncertainty takes $n = 2 \times 10^{11}$ histories ($\sigma_{\text{statistics}} = 0.25$ % at the center, 500 weeks on 1 cpu)

 \implies TMC: 500 random runs of $n = 2 \times 10^{11}$ histories (500 weeks for each)

 \implies fast TMC: 500 random runs of $n/500 = 4 \times 10^8$ histories (1 week for each)

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Other outcomes of the NRG approach (not detailed here)

- ✗ Sensitivity,
- ✤ Nuclear data adjustment,
- * Include fast TMC in Serpent ? $_{0.0}$



D. Rochman – 25 / 26

Conclusions

- (fast) TMC is a powerful tool for uncertainty propagation,
- All types of nuclear data impact can be assessed,
- ➡ Most direct way to propagate uncertainties,
- Better QA, better modern use of computers,
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fast TMC:

If we can do a calculation once, we can also get nuclear data uncertainties at the same time