

Program EVALPLOT  
(Version 2015-2)

by

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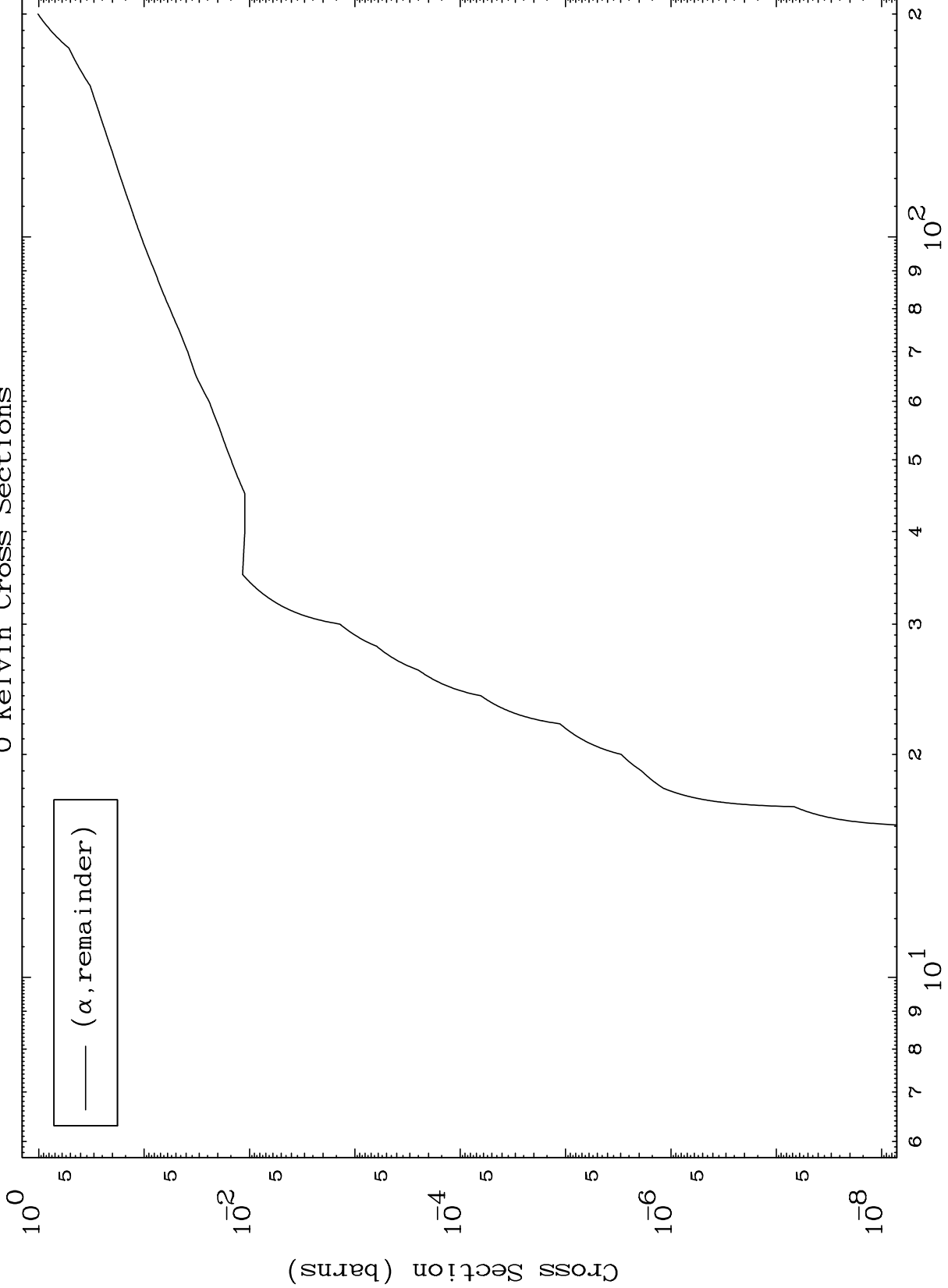
Press Mouse Button to Start

MAT 9713

$\alpha$  Major

97-Bk-236

0 Kelvin Cross Sections

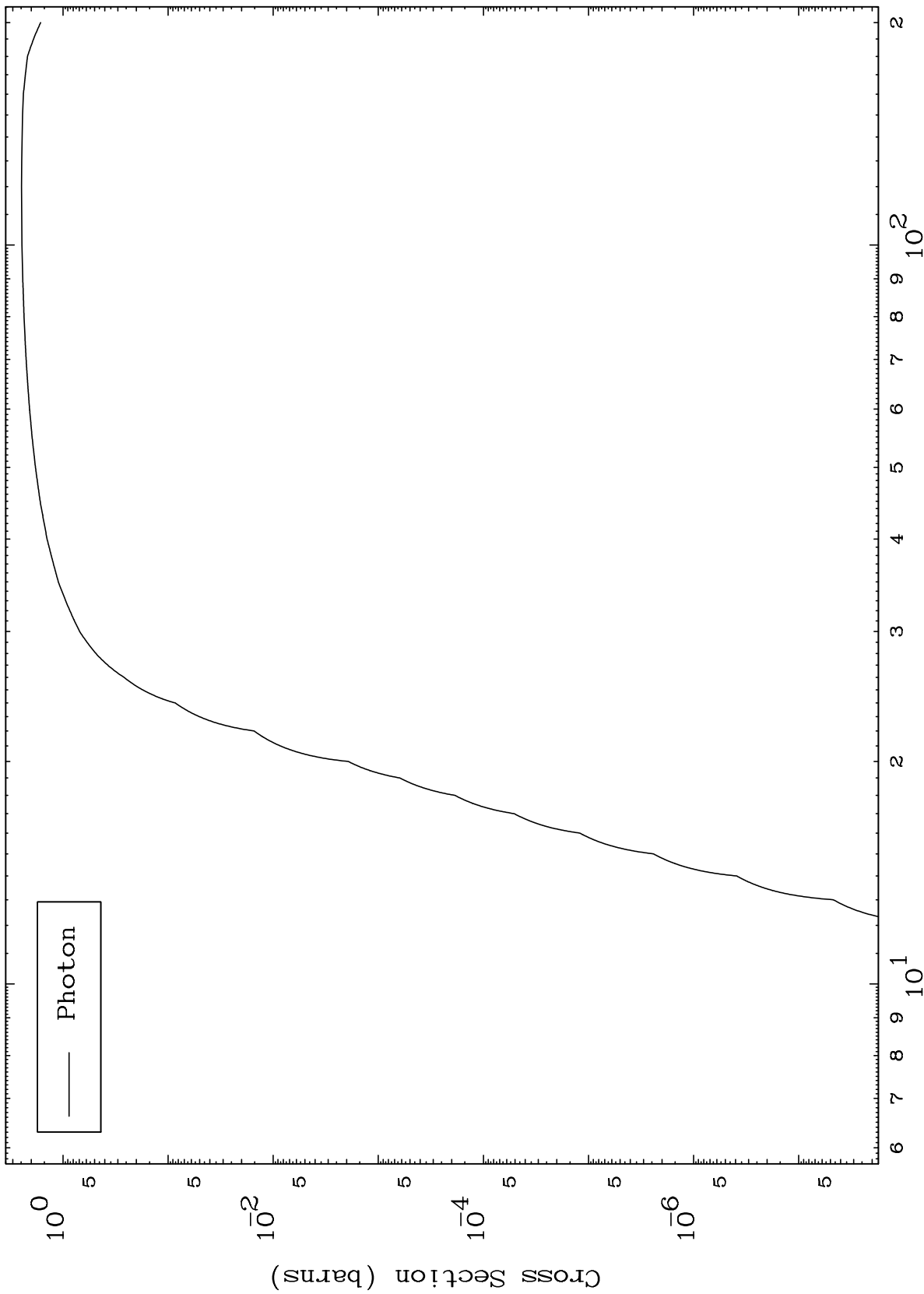


( $\alpha$ , remainder)

MAT 9713

97-Bk-236

$\alpha$  Fission  
Radionuclide Production Cross Section



97-Bk-236

Incident Energy (MeV)

2