

Program EVALPLOT  
(Version 2015-2)

by

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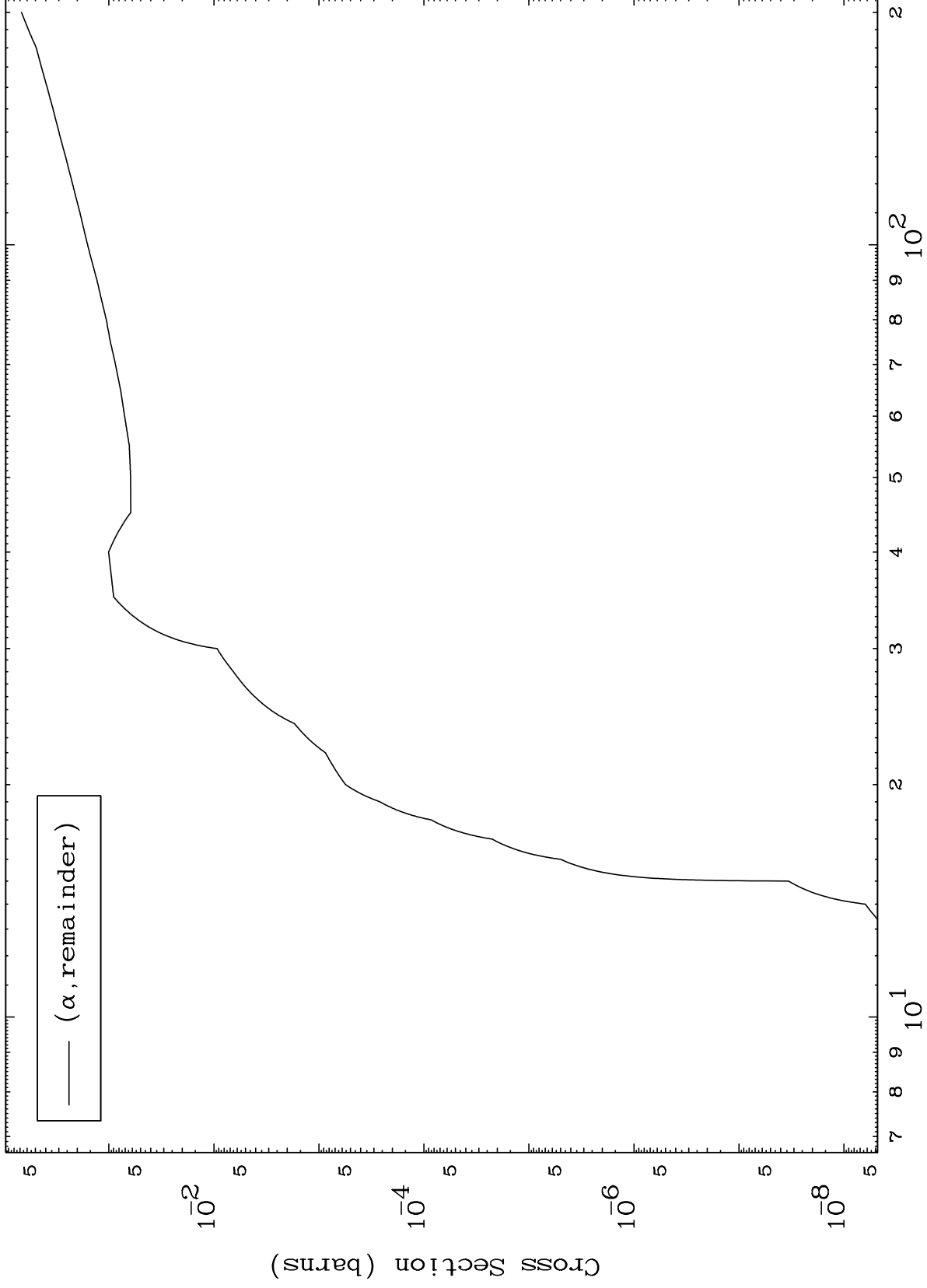
Press Mouse Button to Start

MAT 9979

0 Kelvin

$\alpha$  Major  
Cross Sections

102-No-261



1

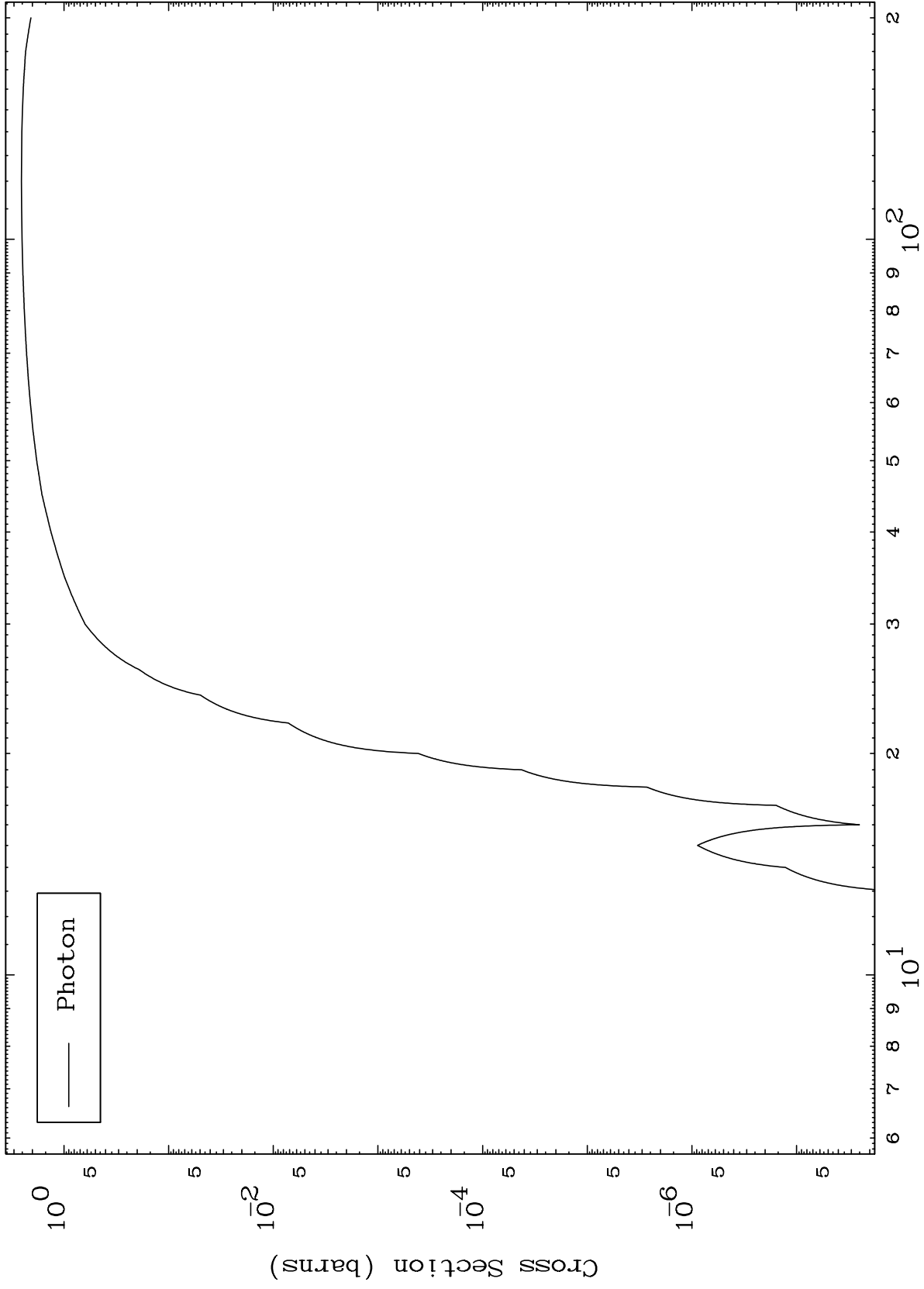
Incident Energy (MeV)

102-No-261

MAT 9979

102-No-261

$\alpha$  Fission  
Radionuclide Production Cross Section



102-No-261

Incident Energy (MeV)

2