

Program EVALPLOT  
(Version 2015-2)

by

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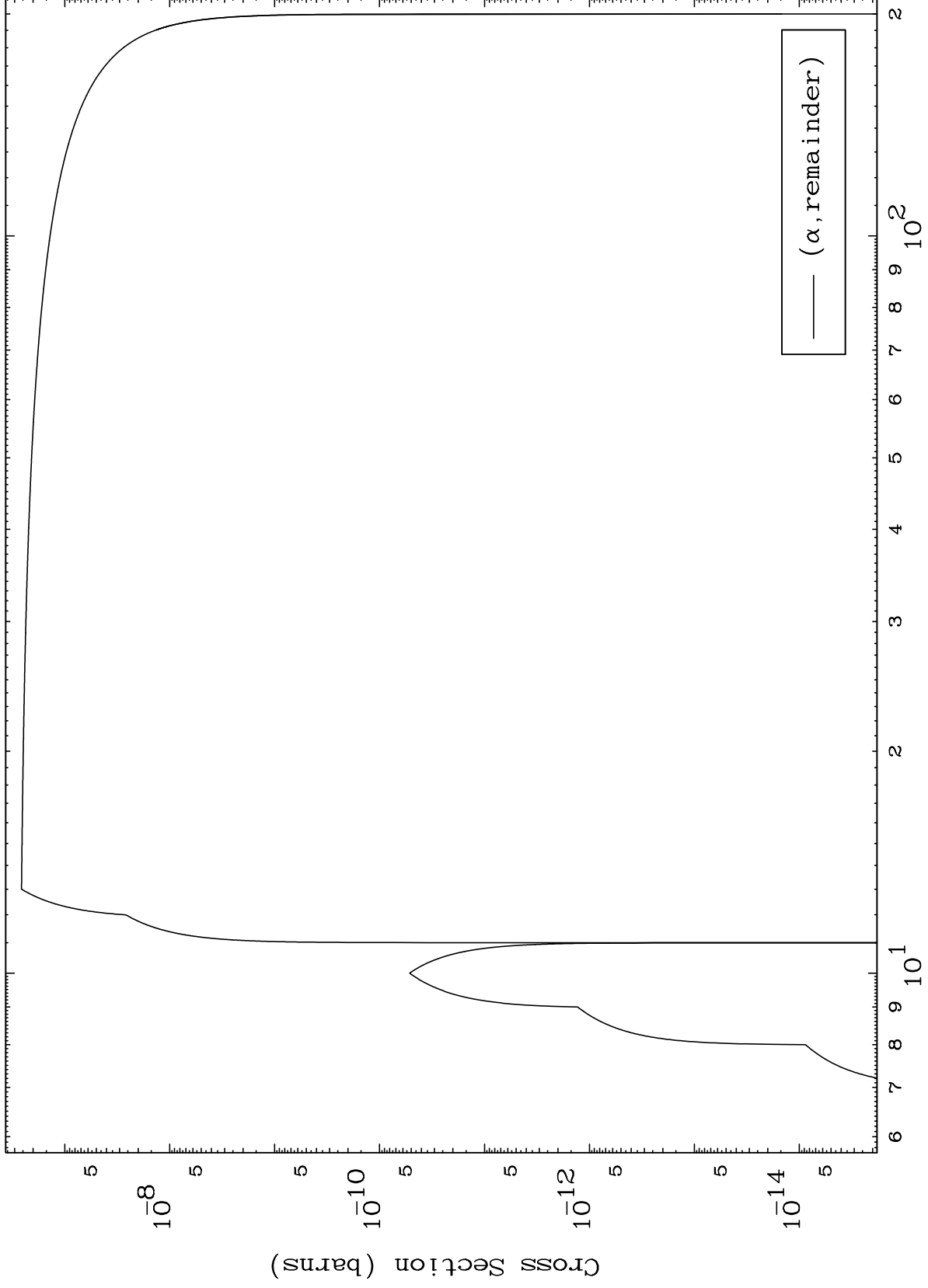
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Press Mouse Button to Start

MAT 9240

$\alpha$  Major  
0 Kelvin Cross Sections

92-U -239



1

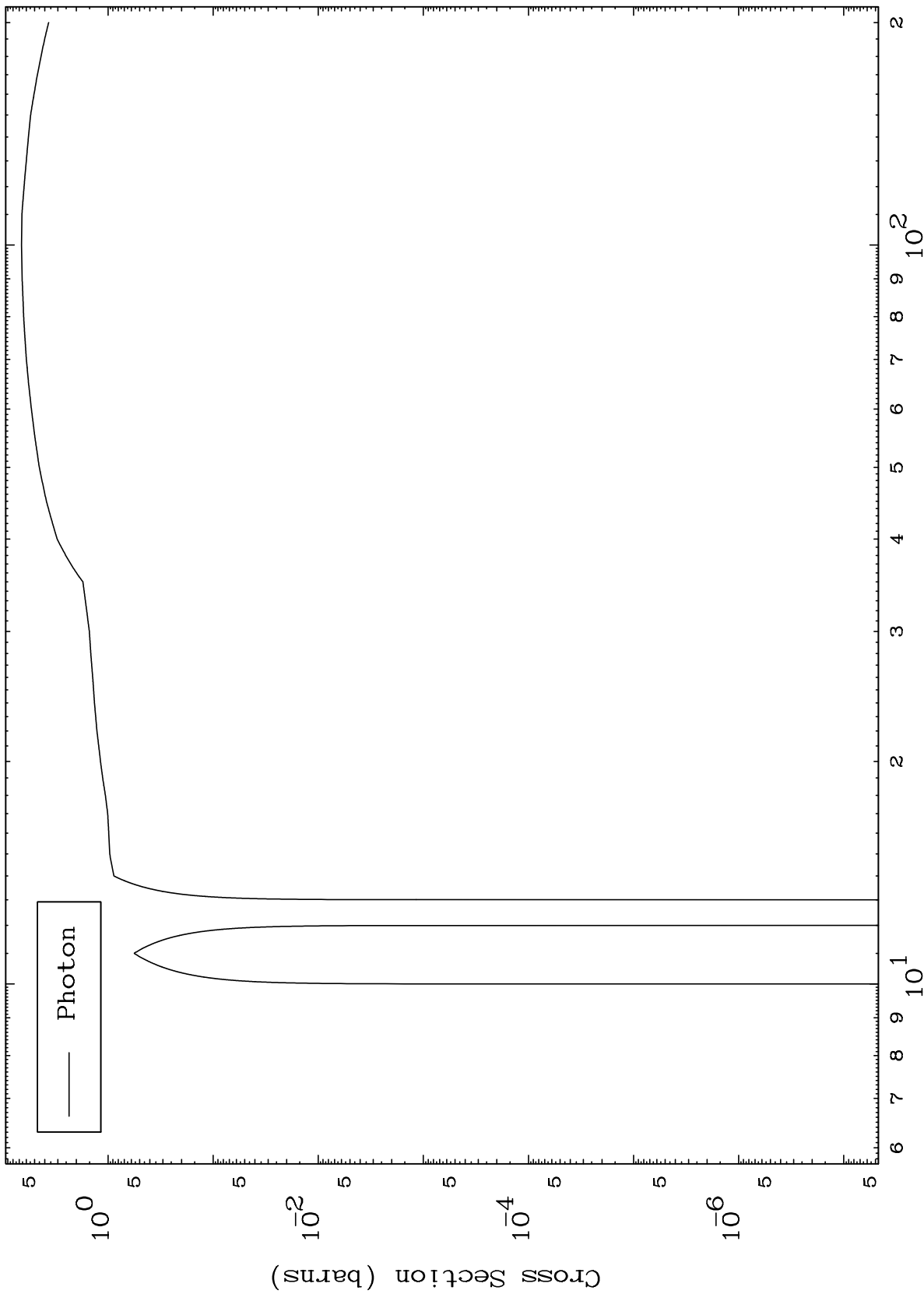
Incident Energy (MeV)

92-U -239

MAT 9240

92-U -239

$\alpha$  Fission  
Radionuclide Production Cross Section



92-U -239

Incident Energy (MeV)

2