

Program EVALPLOT
(Version 2015-2)

by

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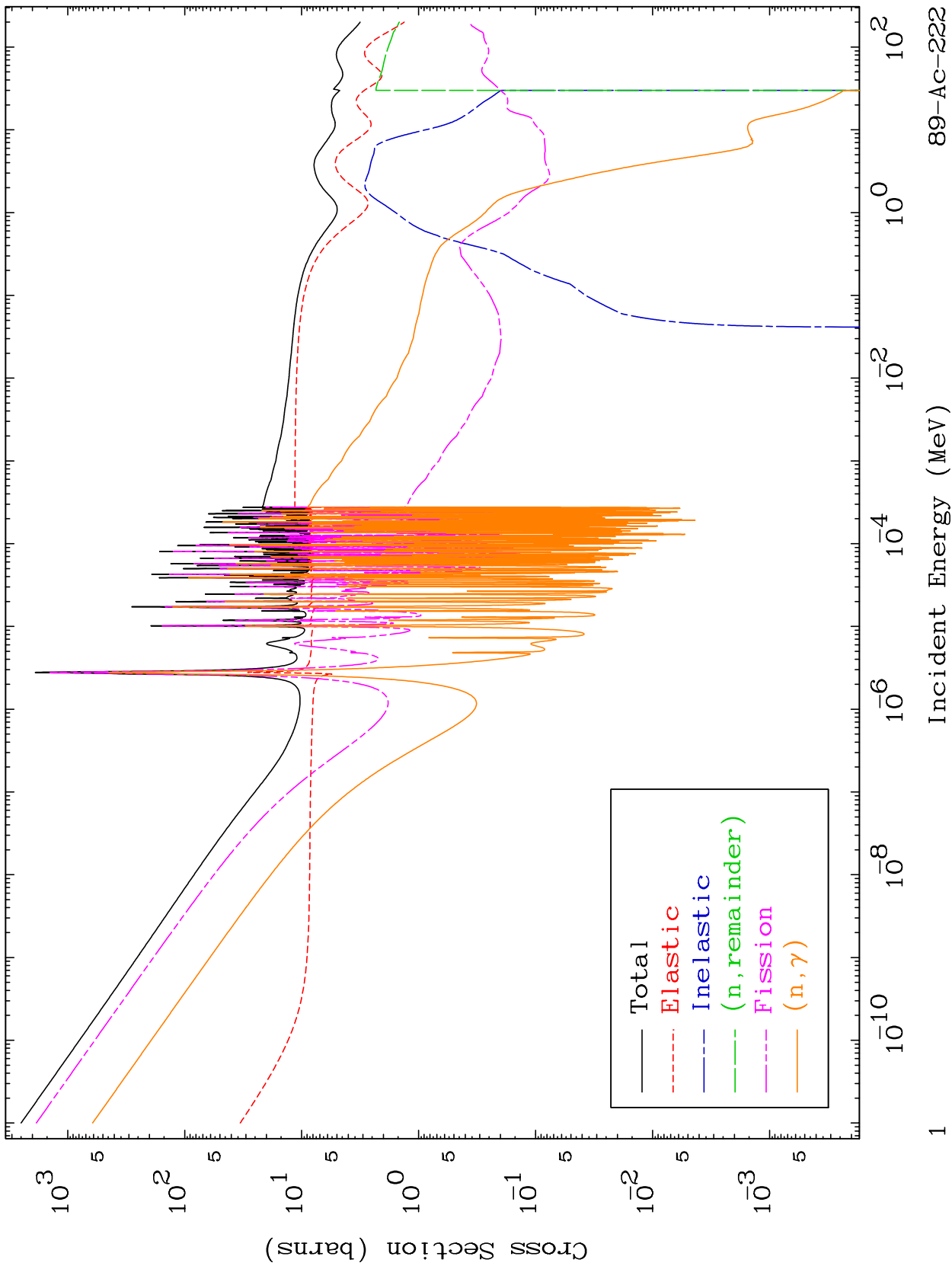
Press Mouse Button to Start

MAT 8916

Major

293 Kelvin Cross Sections

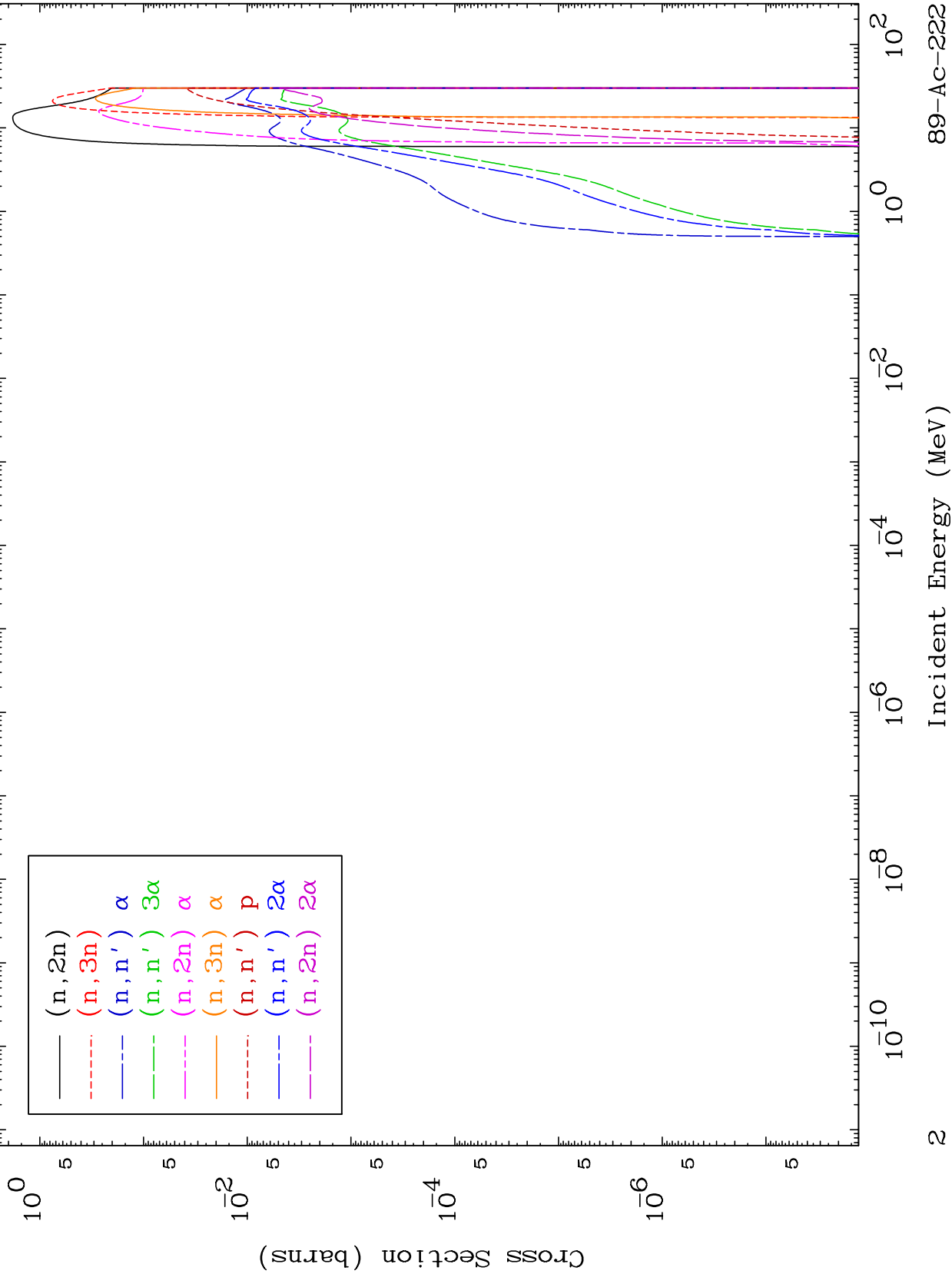
89-Ac-222



MAT 8916

Neutron Production
293 Kelvin Cross Sections

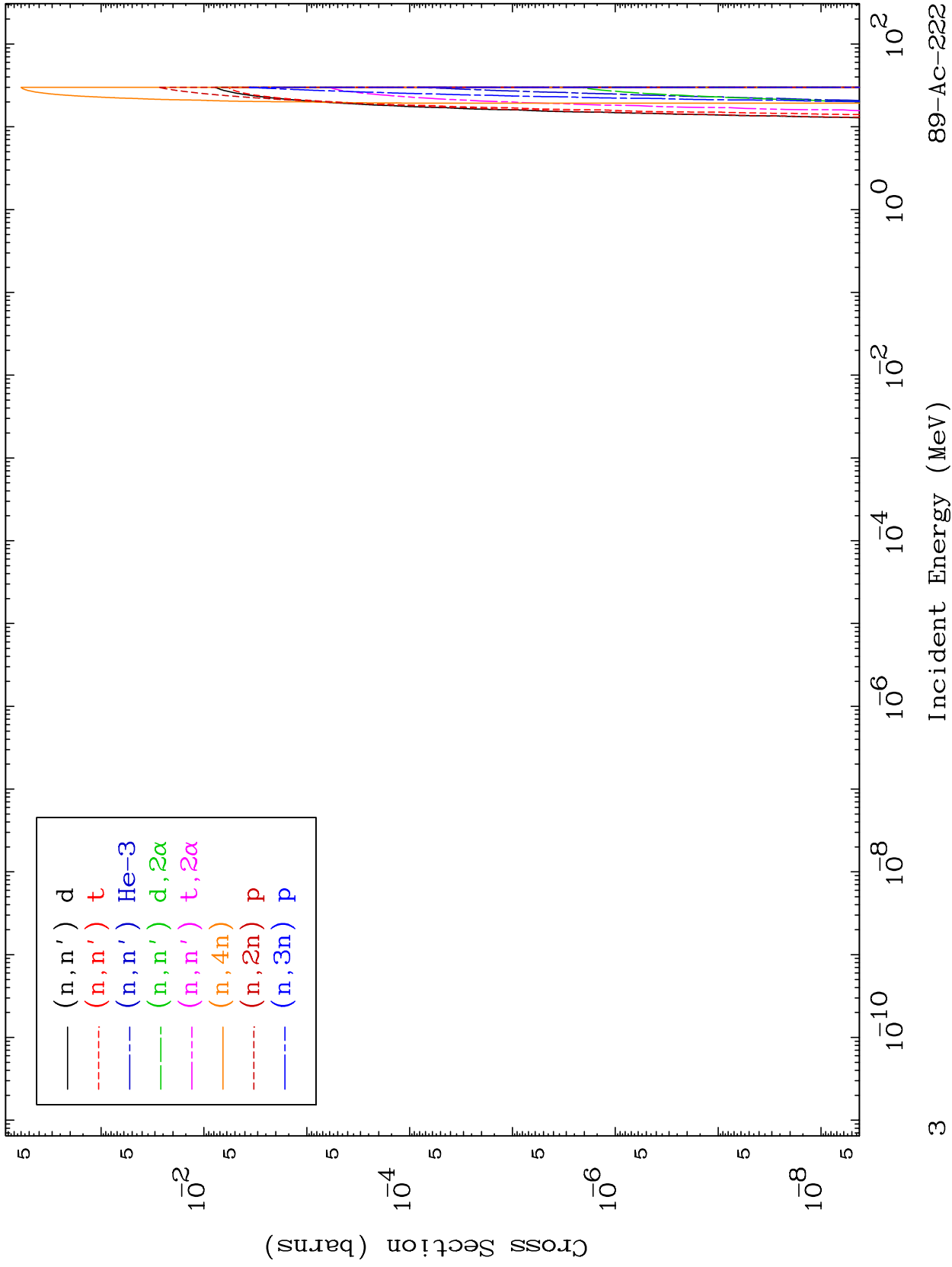
89-Ac-222



MAT 8916

Neutron Production
293 Kelvin Cross Sections

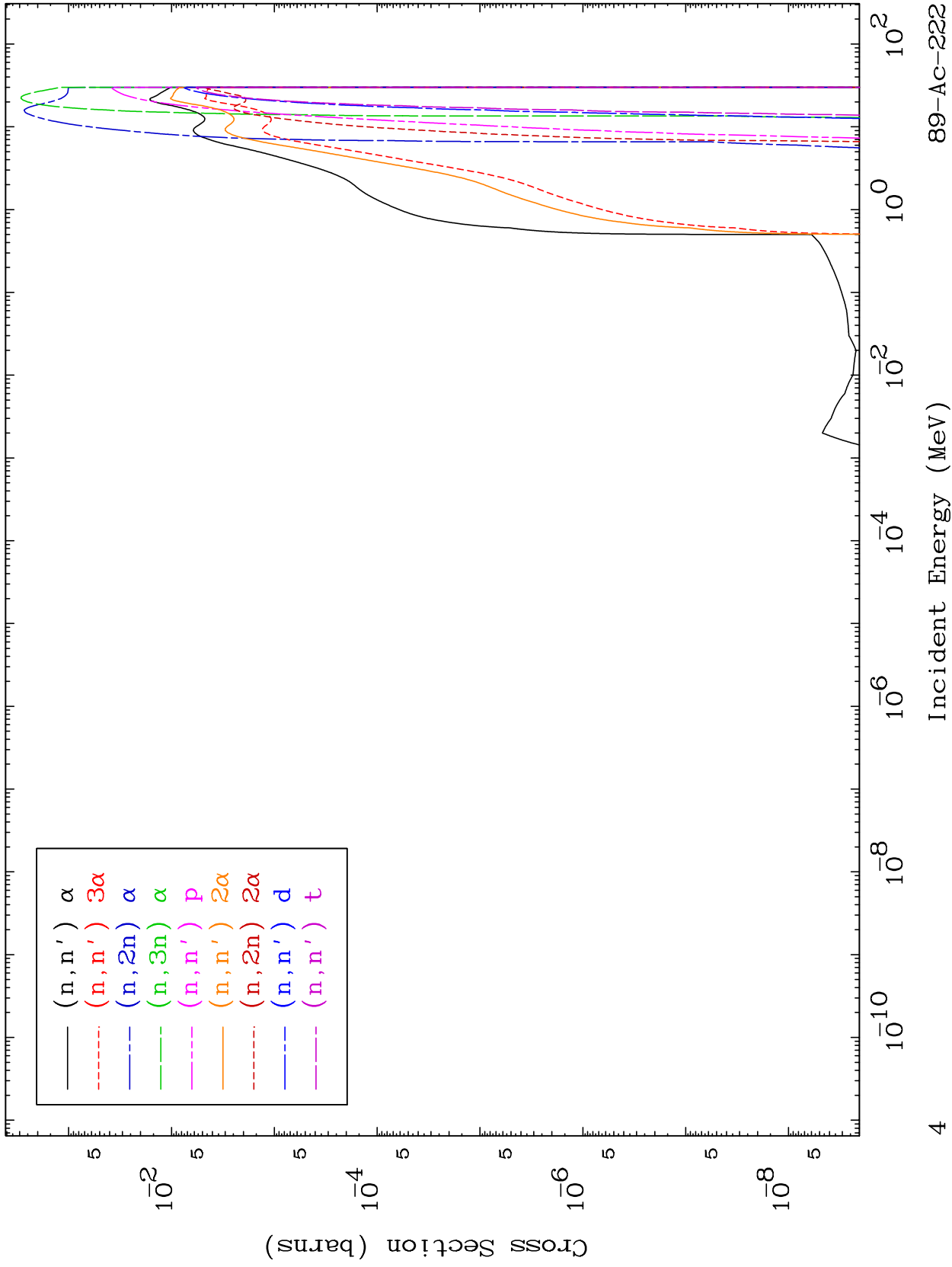
89-Ac-222



MAT 8916

Charged Particle
293 Kelvin Cross Sections

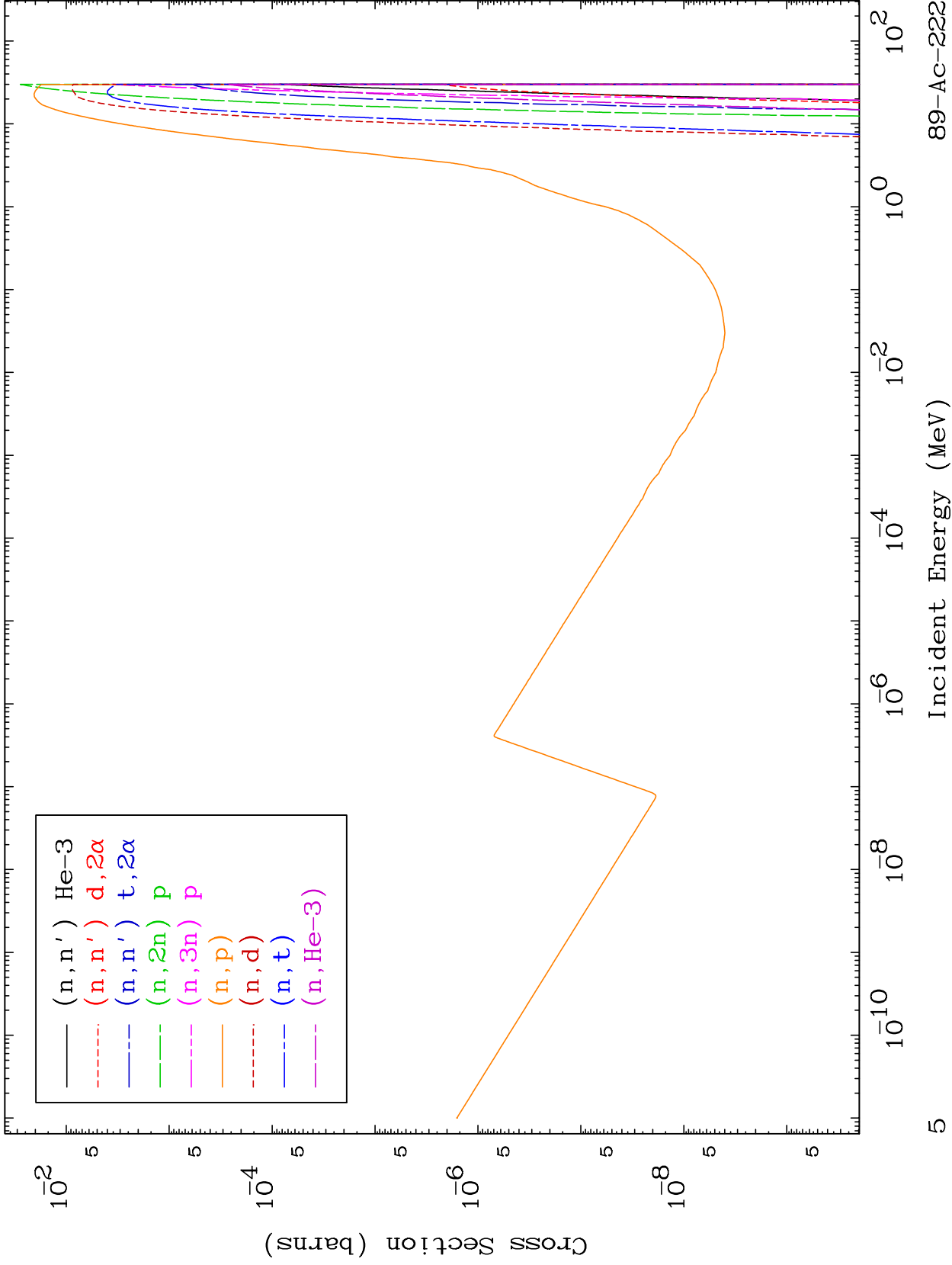
89-Ac-222



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Charged Particle
293 Kelvin Cross Sections

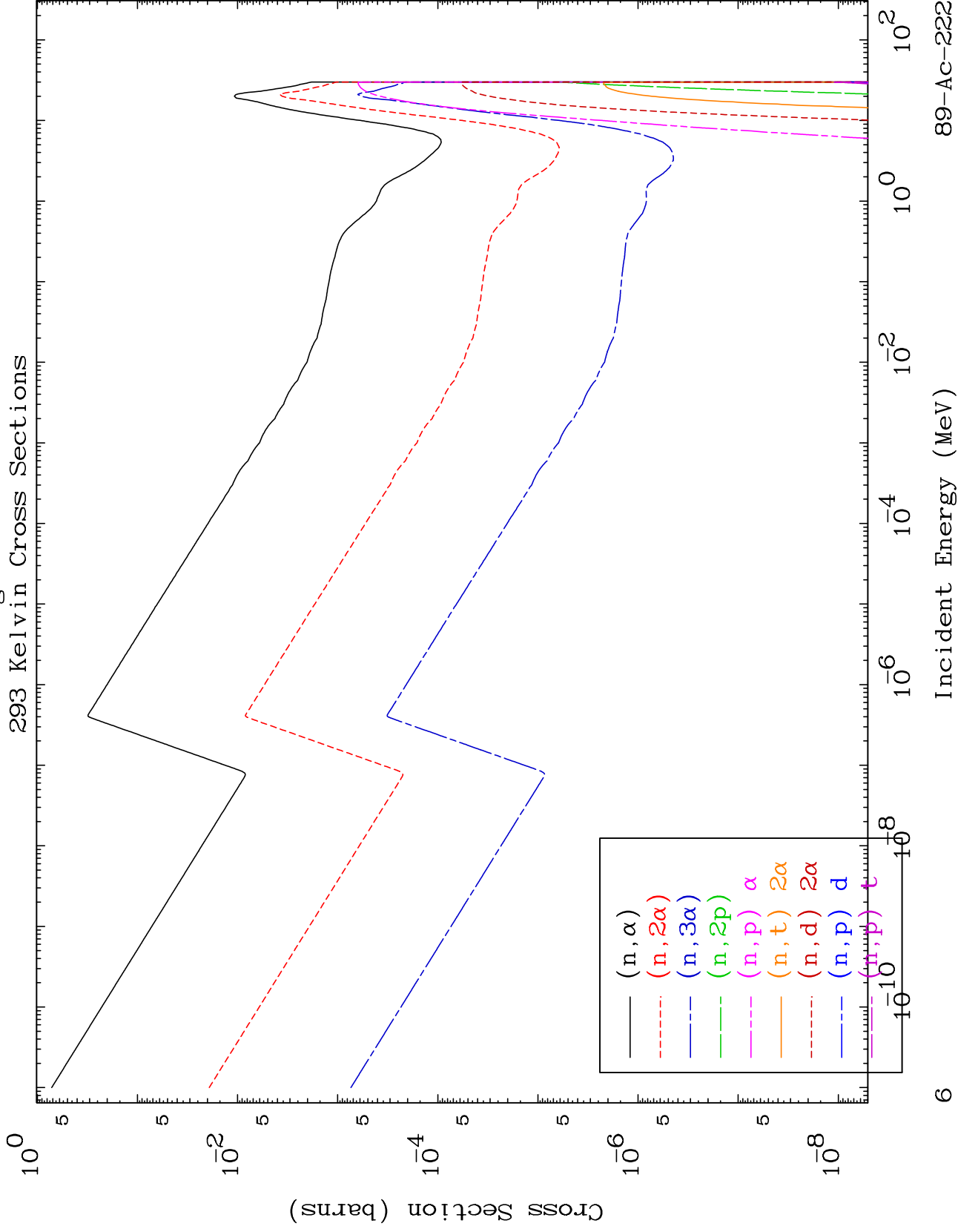
89-Ac-222



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Charged Particle
293 Kelvin Cross Sections

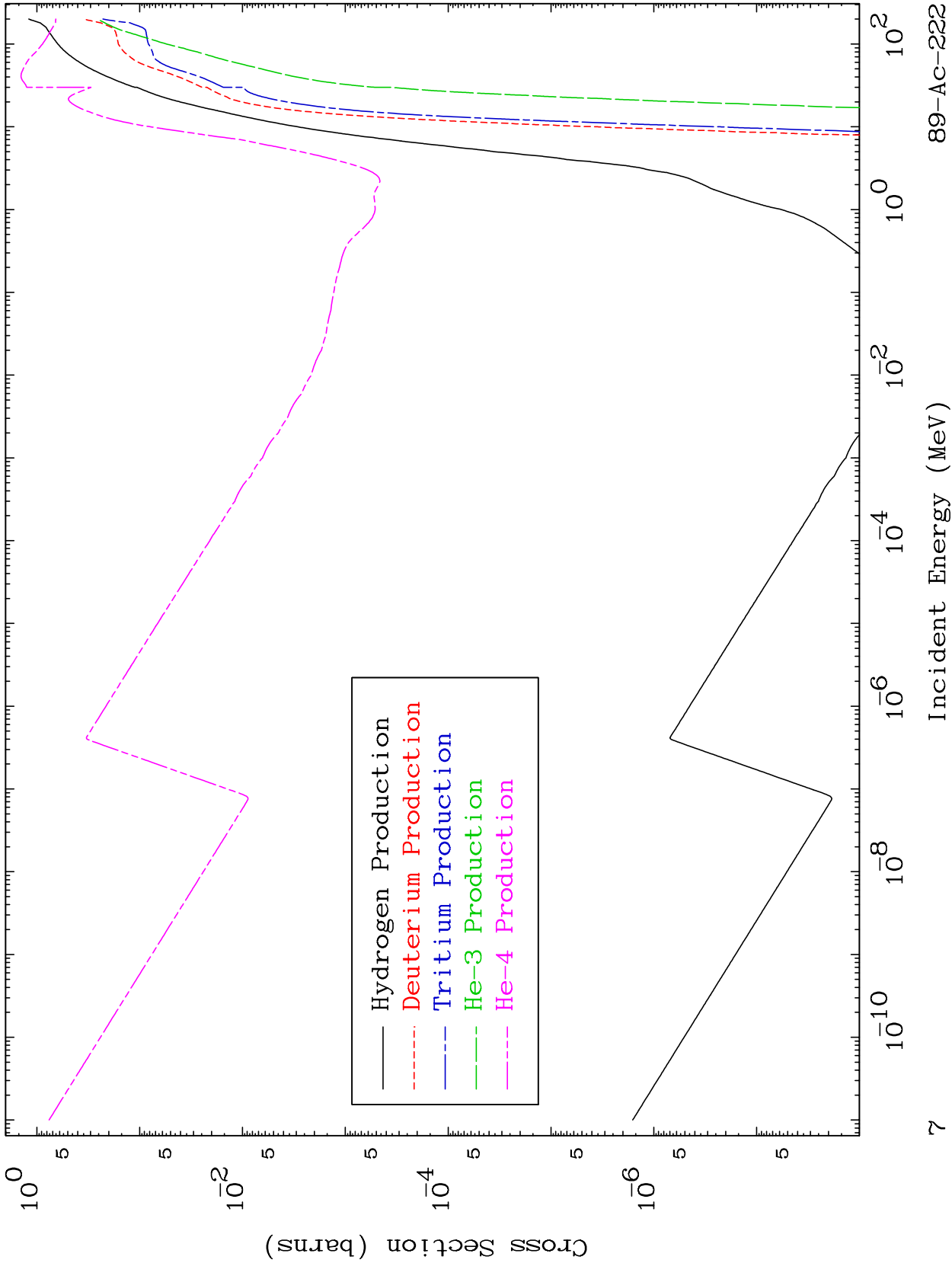
89-Ac-222



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Particle Production
293 Kelvin Cross Sections

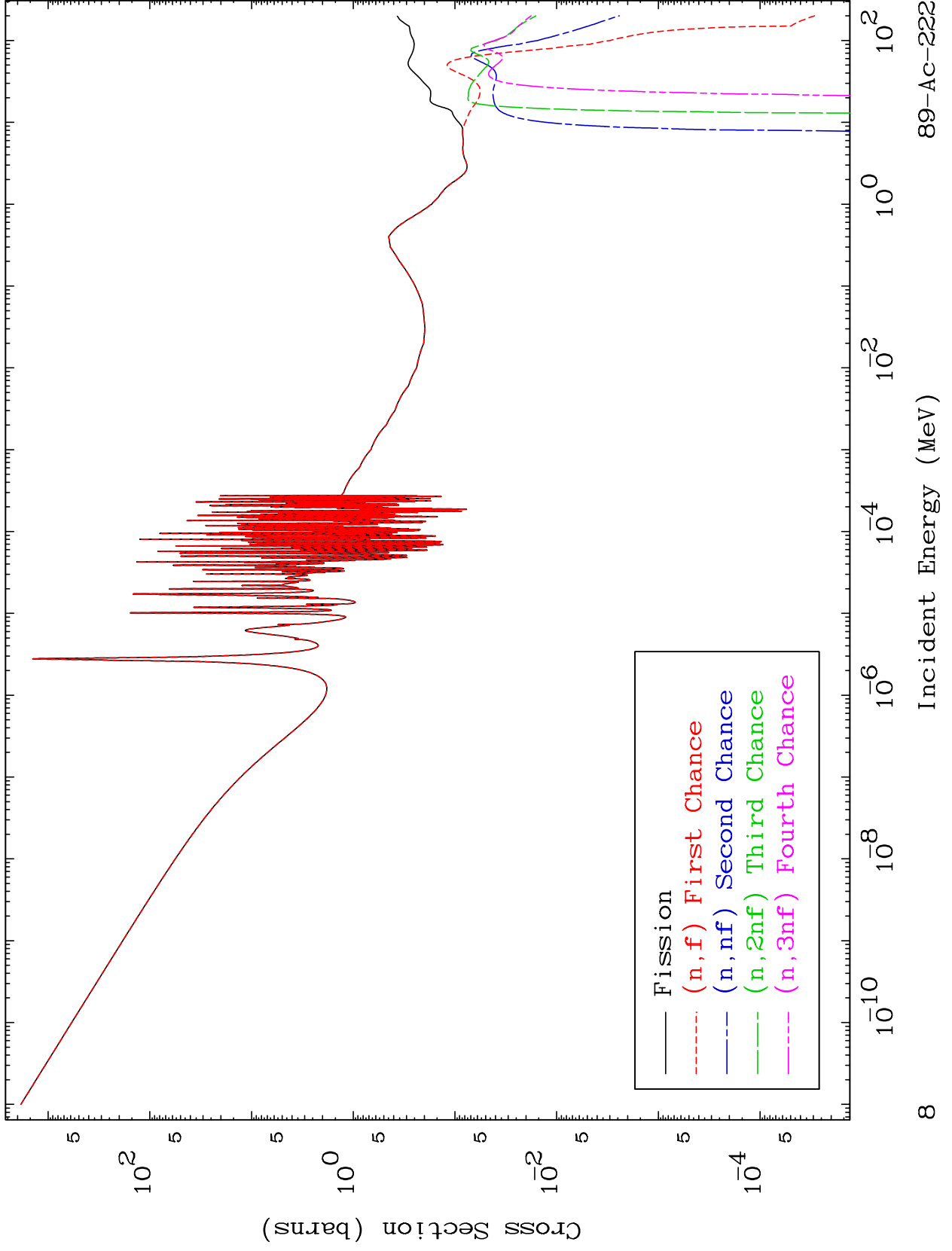
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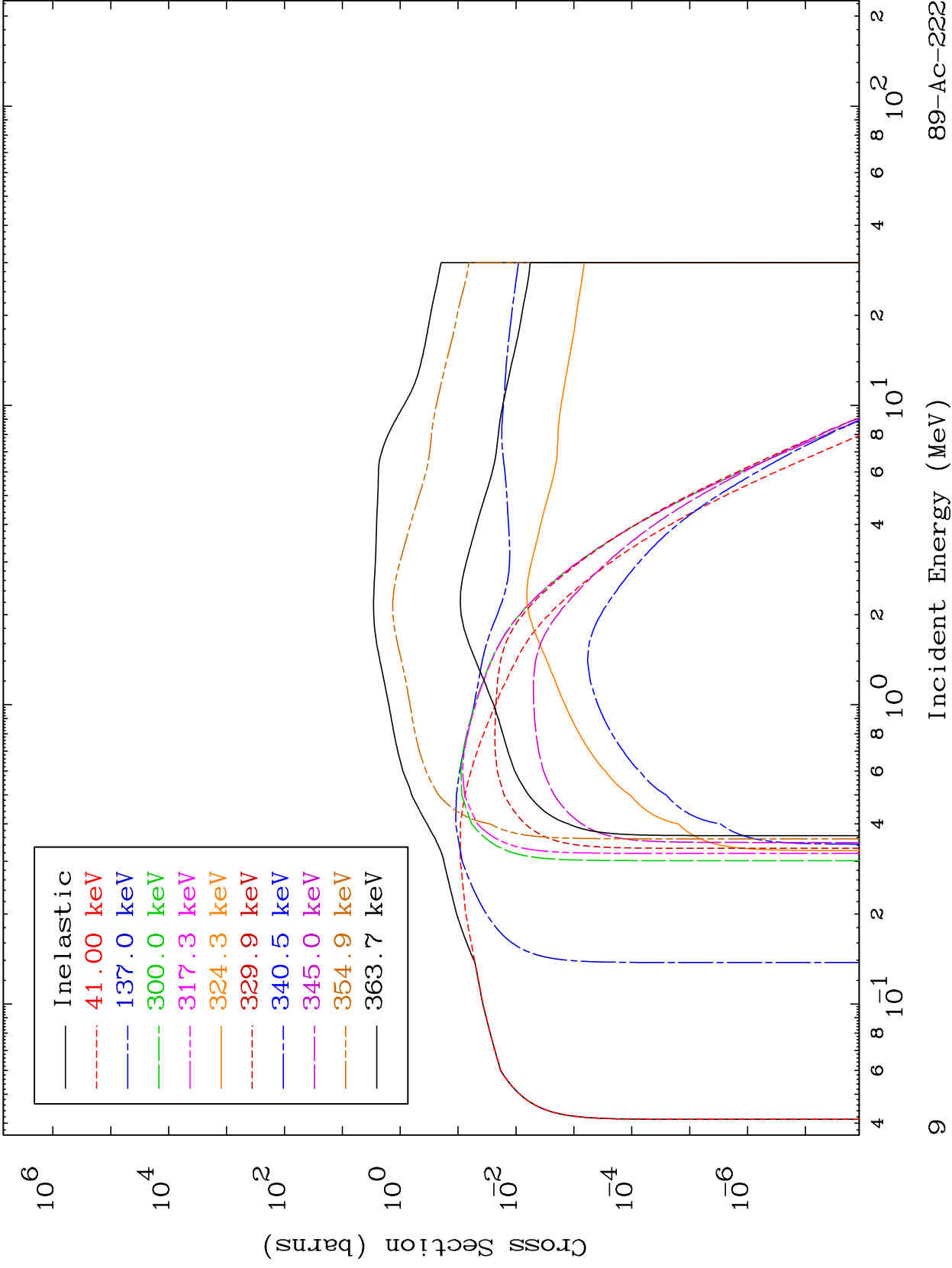


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Fission
293 Kelvin Cross Sections

89-Ac-222



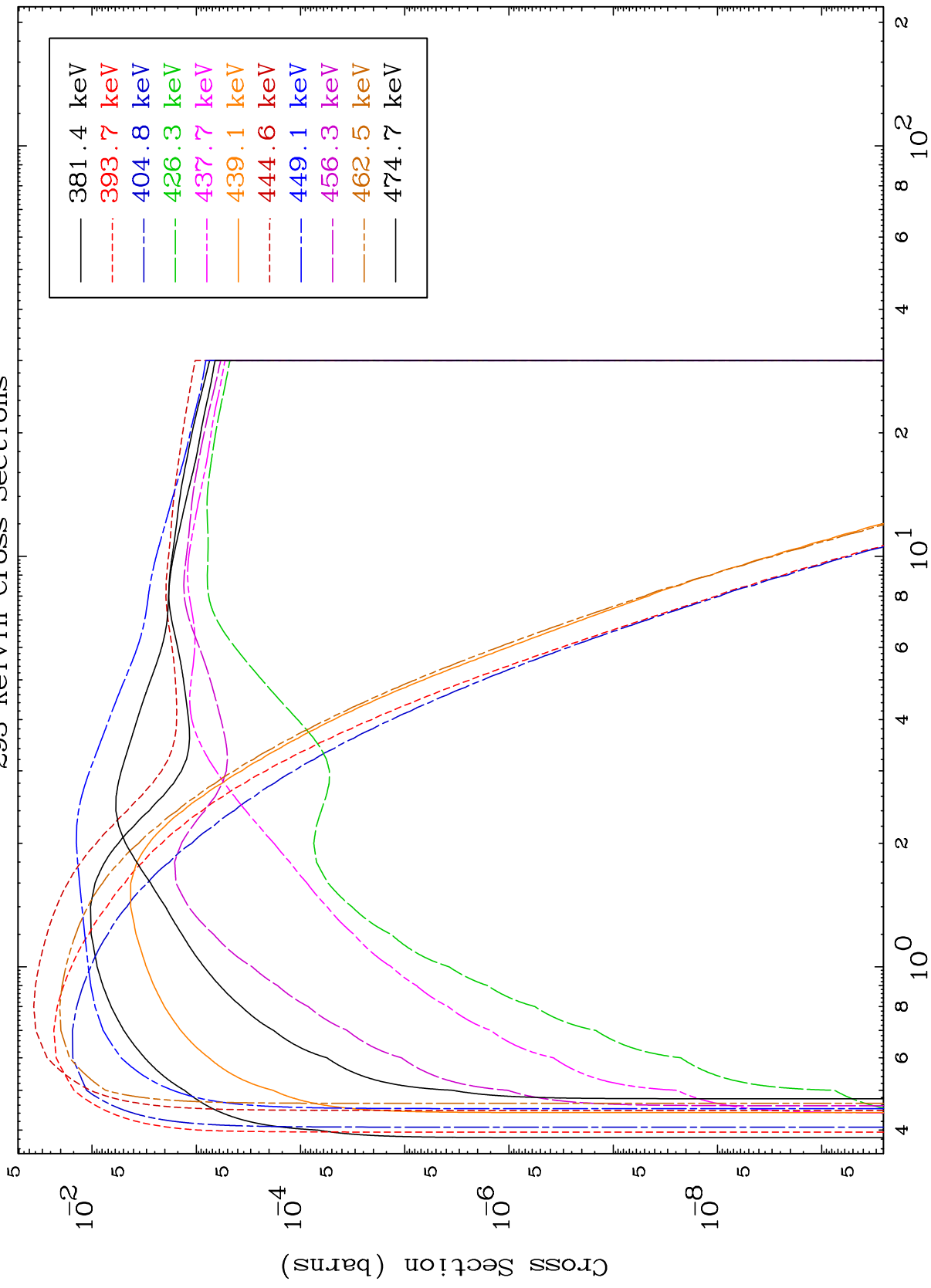


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(n,n') Level

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293 Kelvin Cross Sections

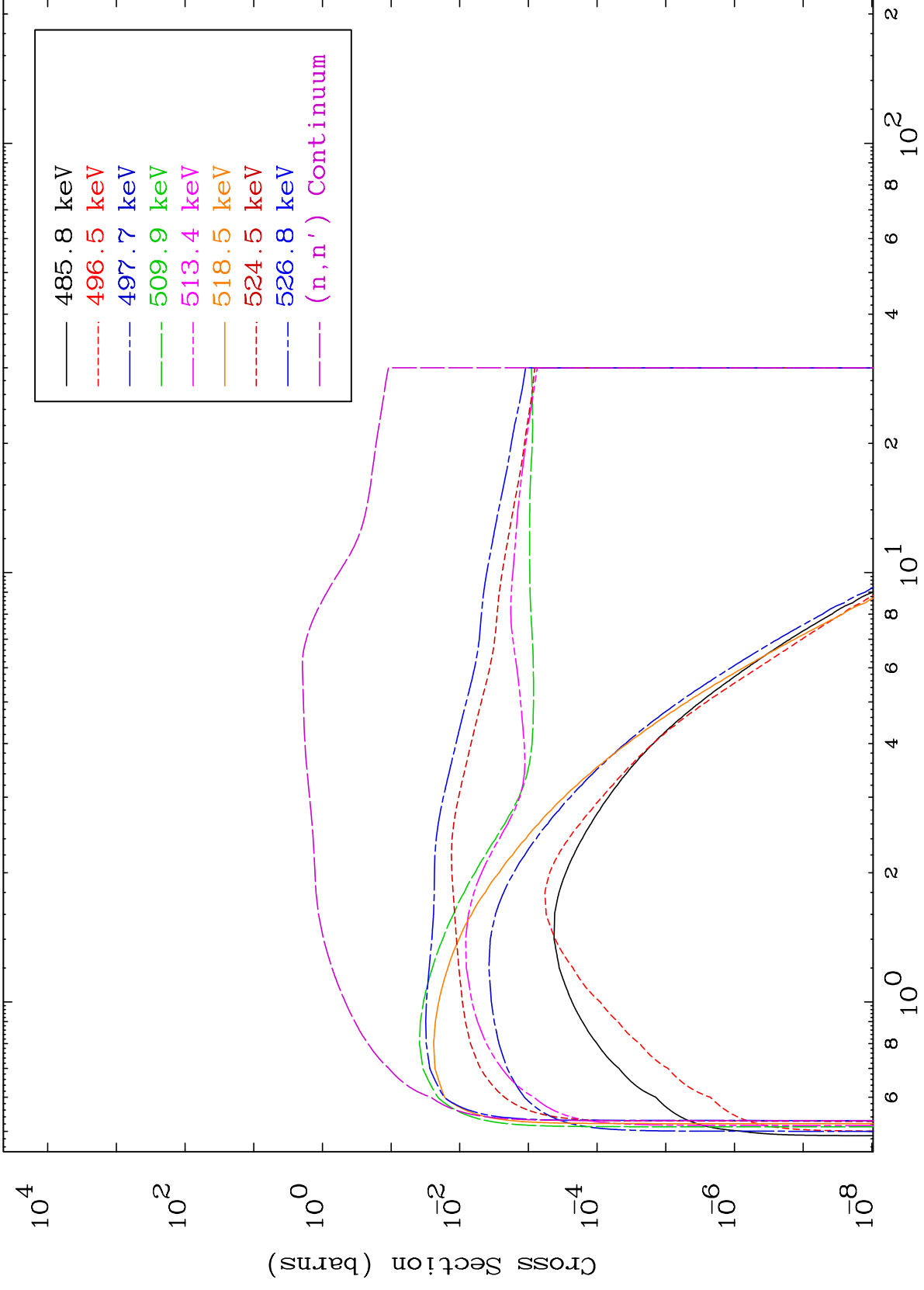


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Incident Energy (MeV)

89-Ac-222

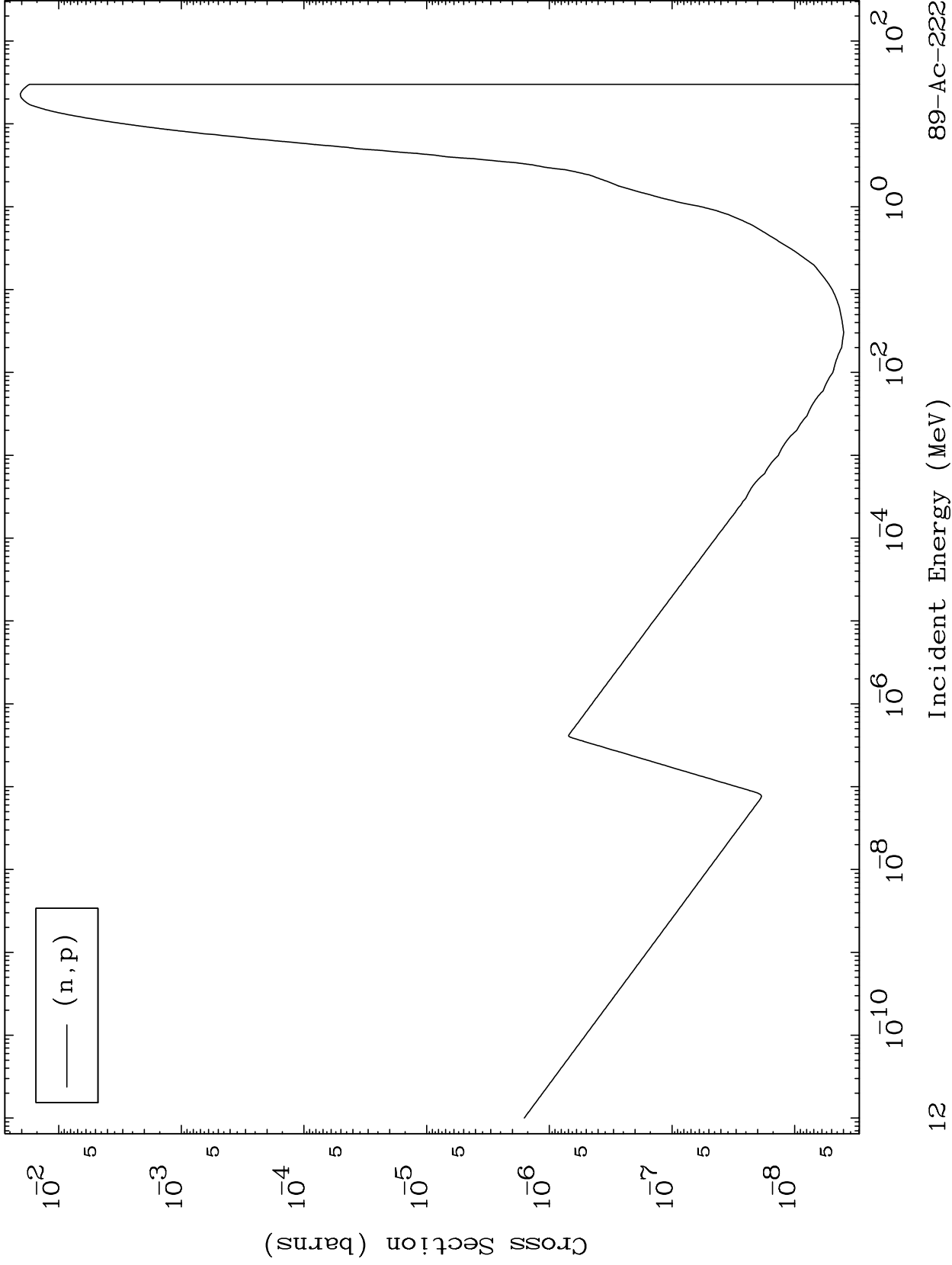
293 Kelvin Cross Sections



MAT 8916

(n,p) Levels
293 Kelvin Cross Sections

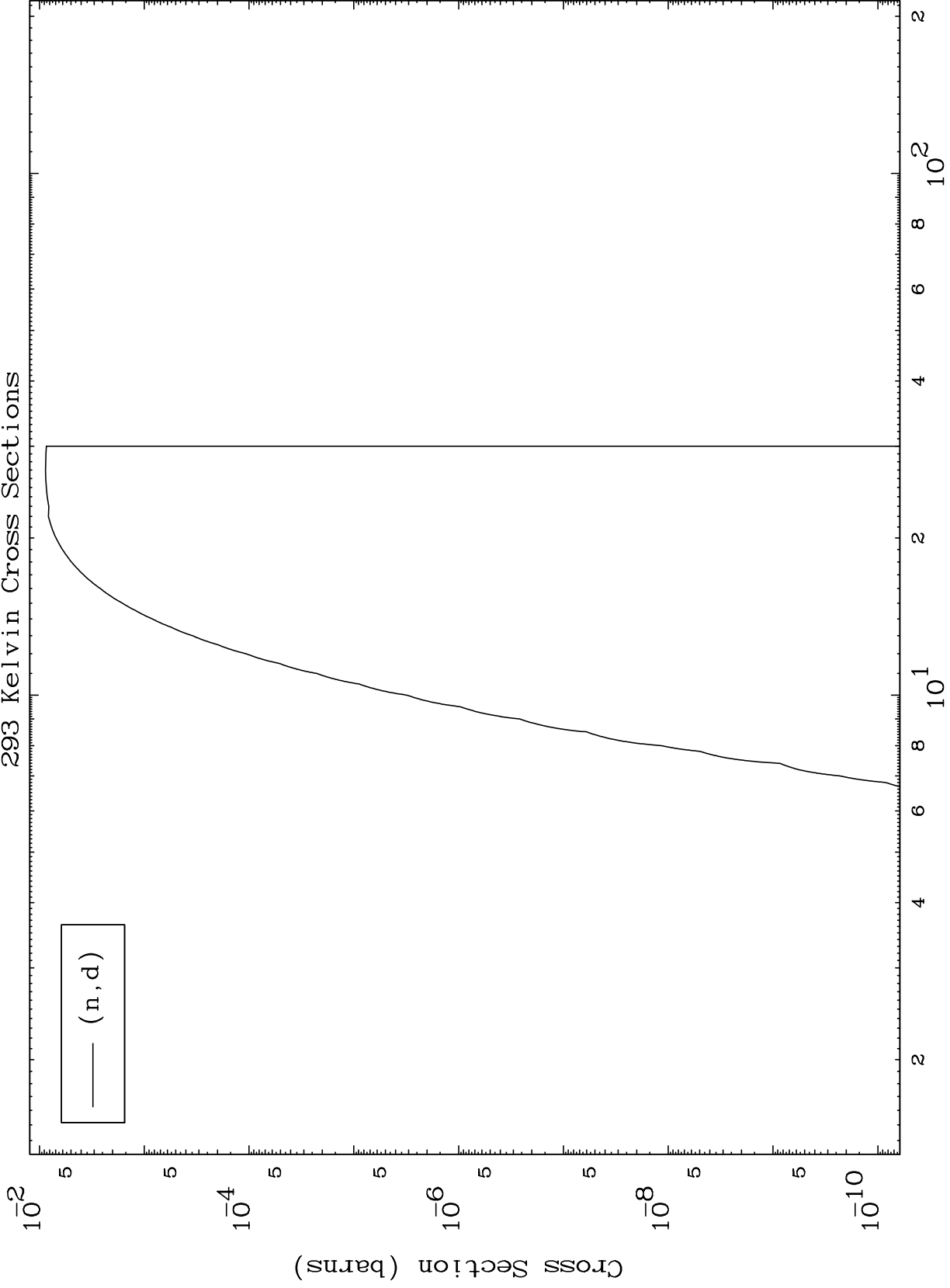
89-Ac-222



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(n,d) Levels
293 Kelvin Cross Sections

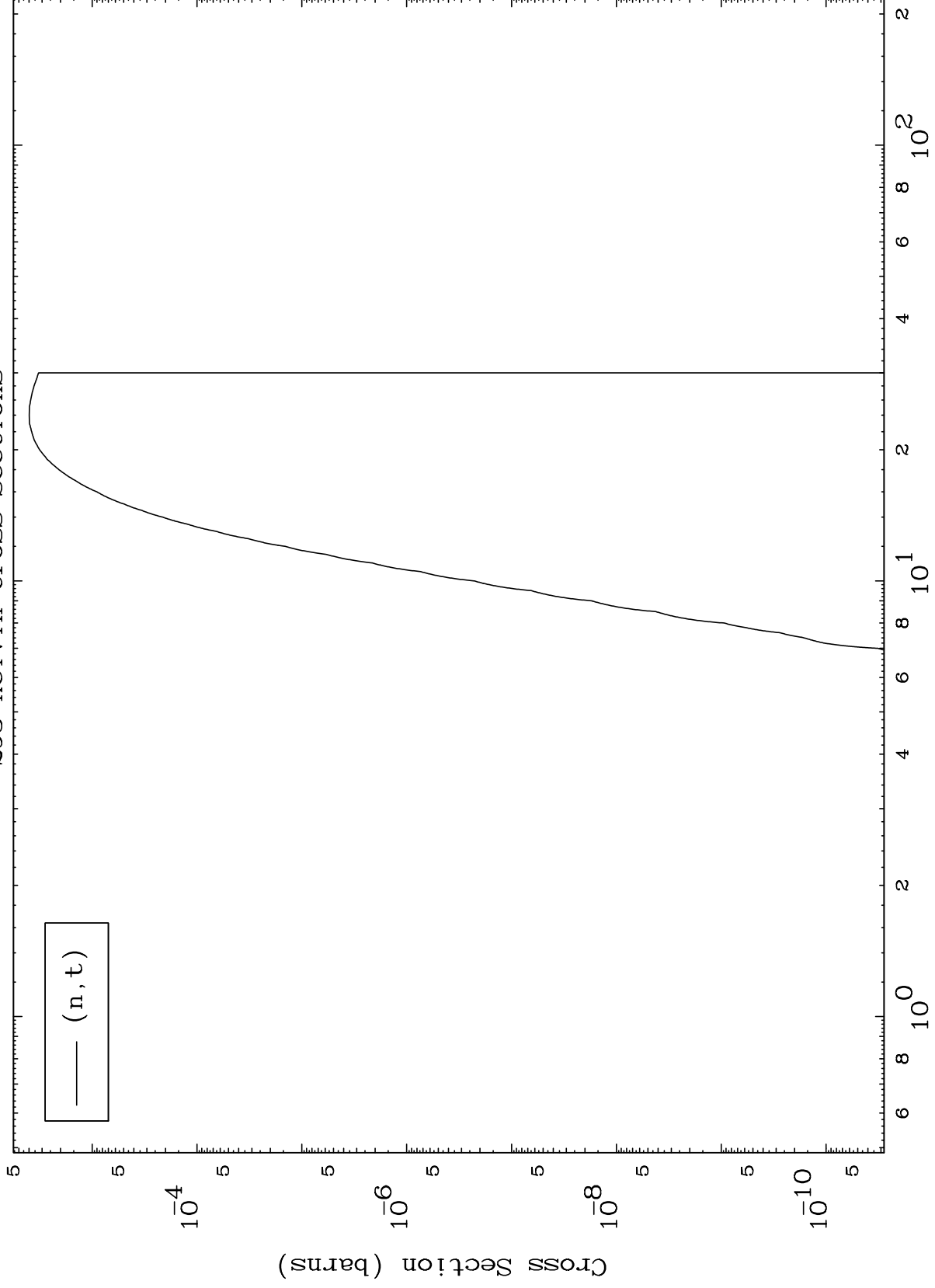
89-Ac-222



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(n,t) Levels
293 Kelvin Cross Sections

89-Ac-222



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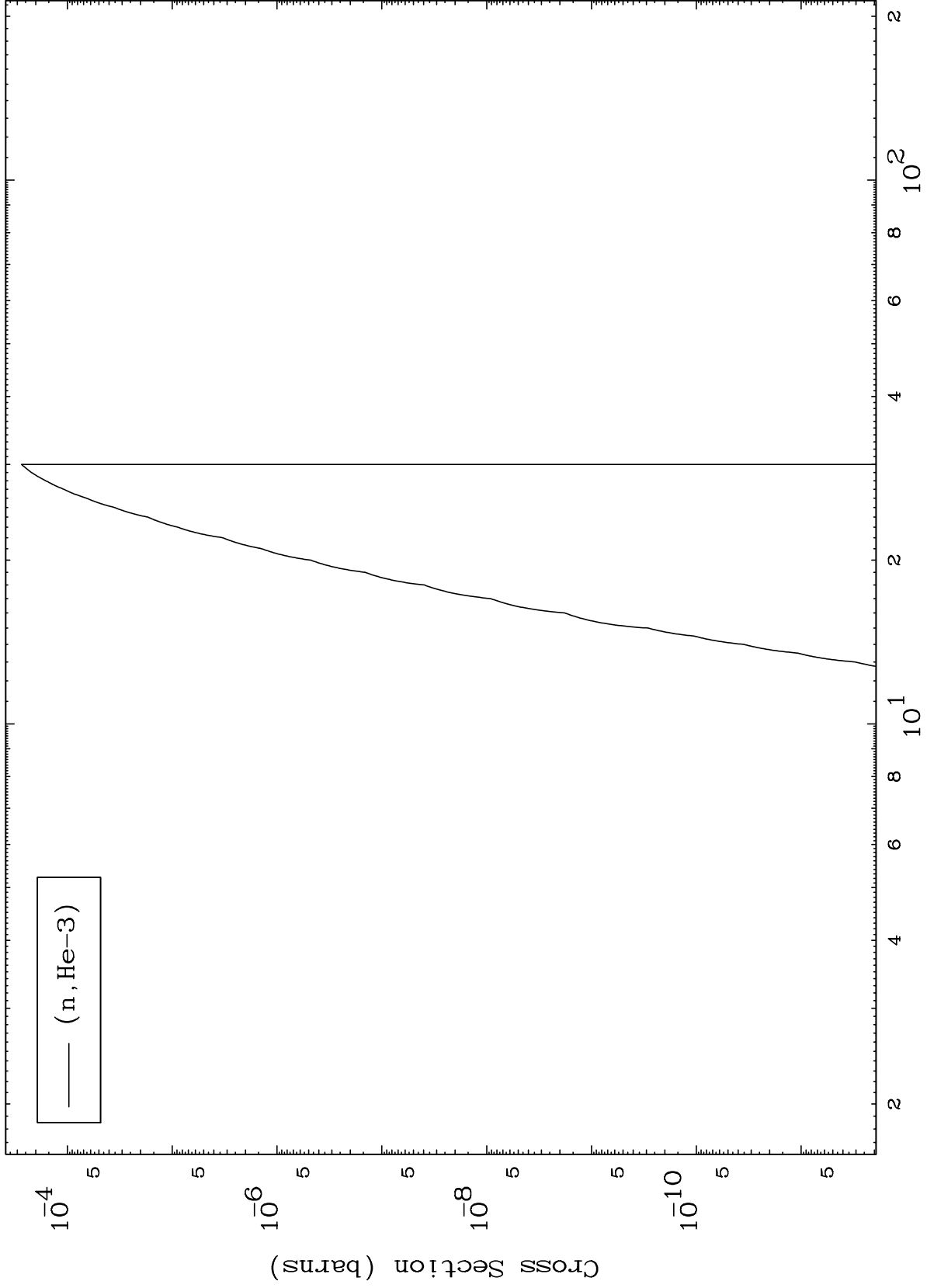
Incident Energy (MeV)

89-Ac-222

MAT 8916

(n,He3) Levels
293 Kelvin Cross Sections

89-Ac-222



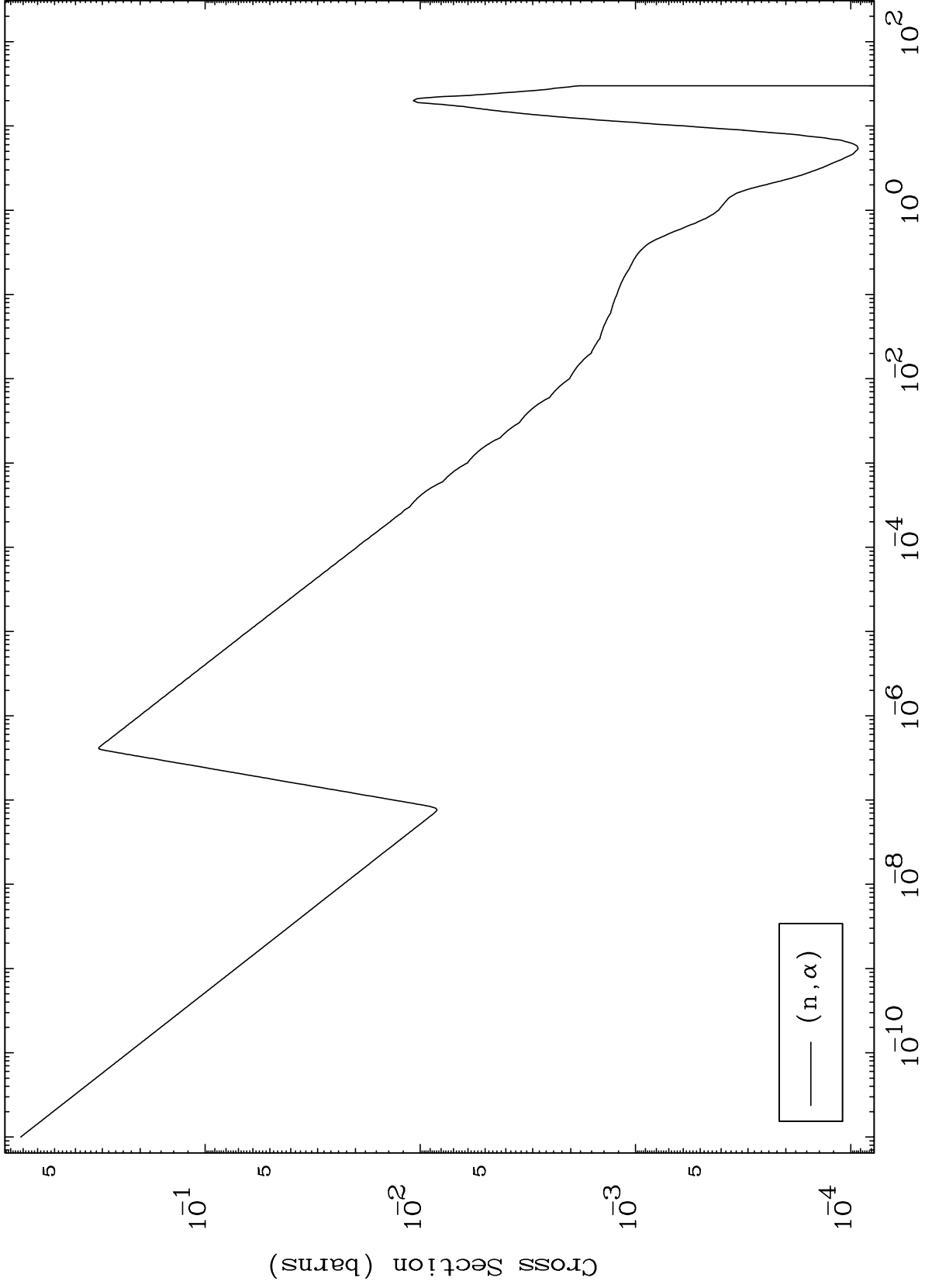
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(n,α) Levels
293 Kelvin Cross Sections

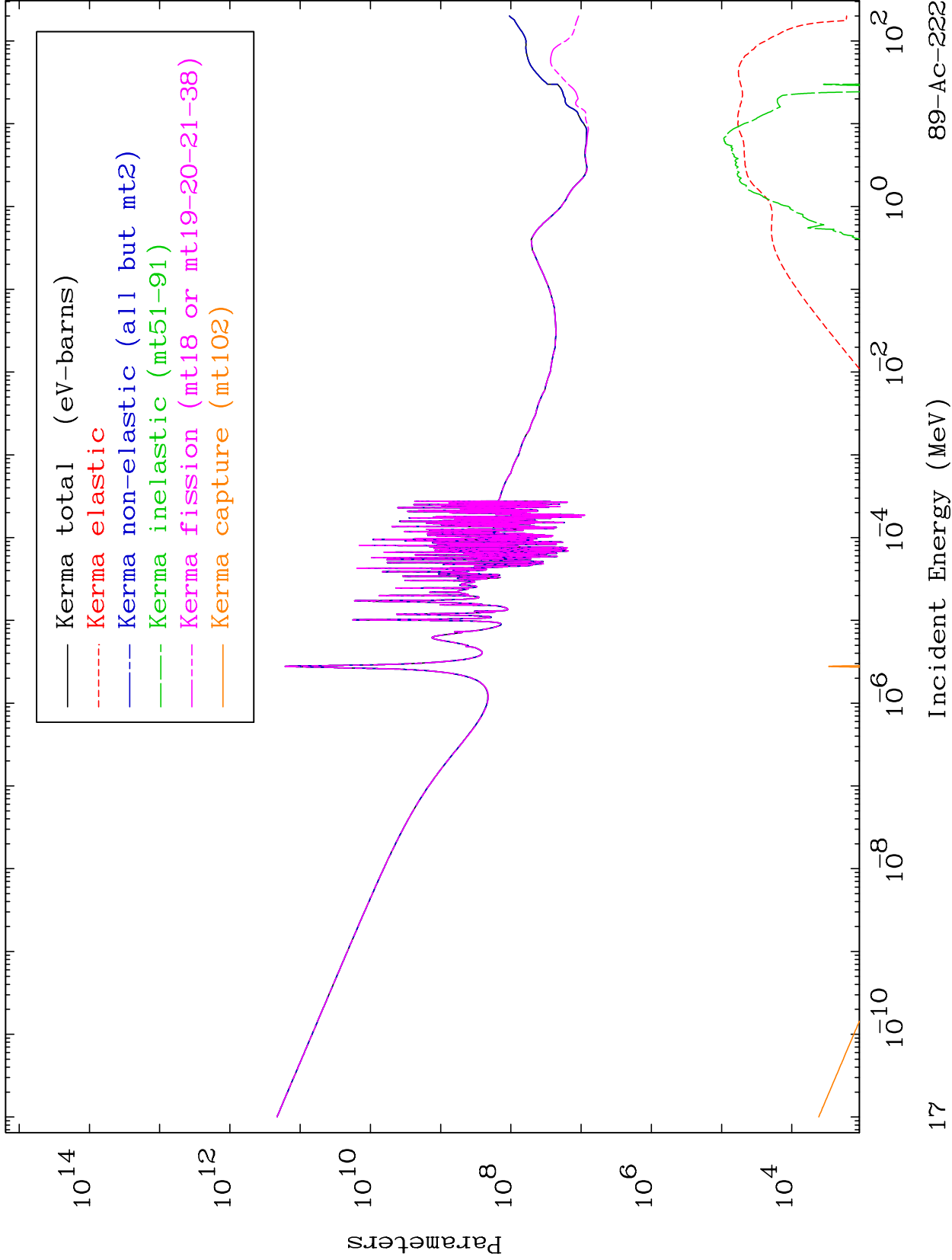
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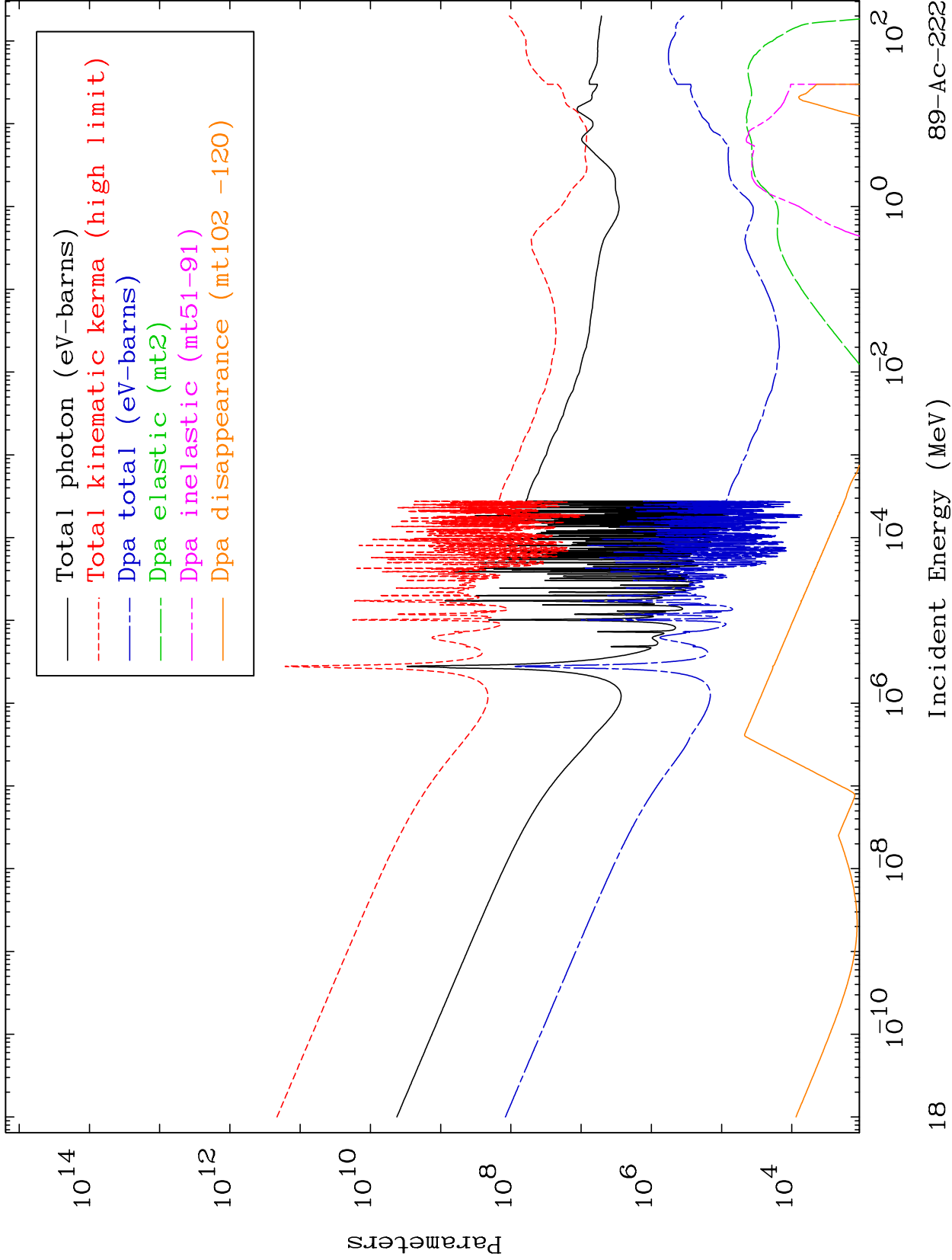


16

Incident Energy (MeV)

89-Ac-222

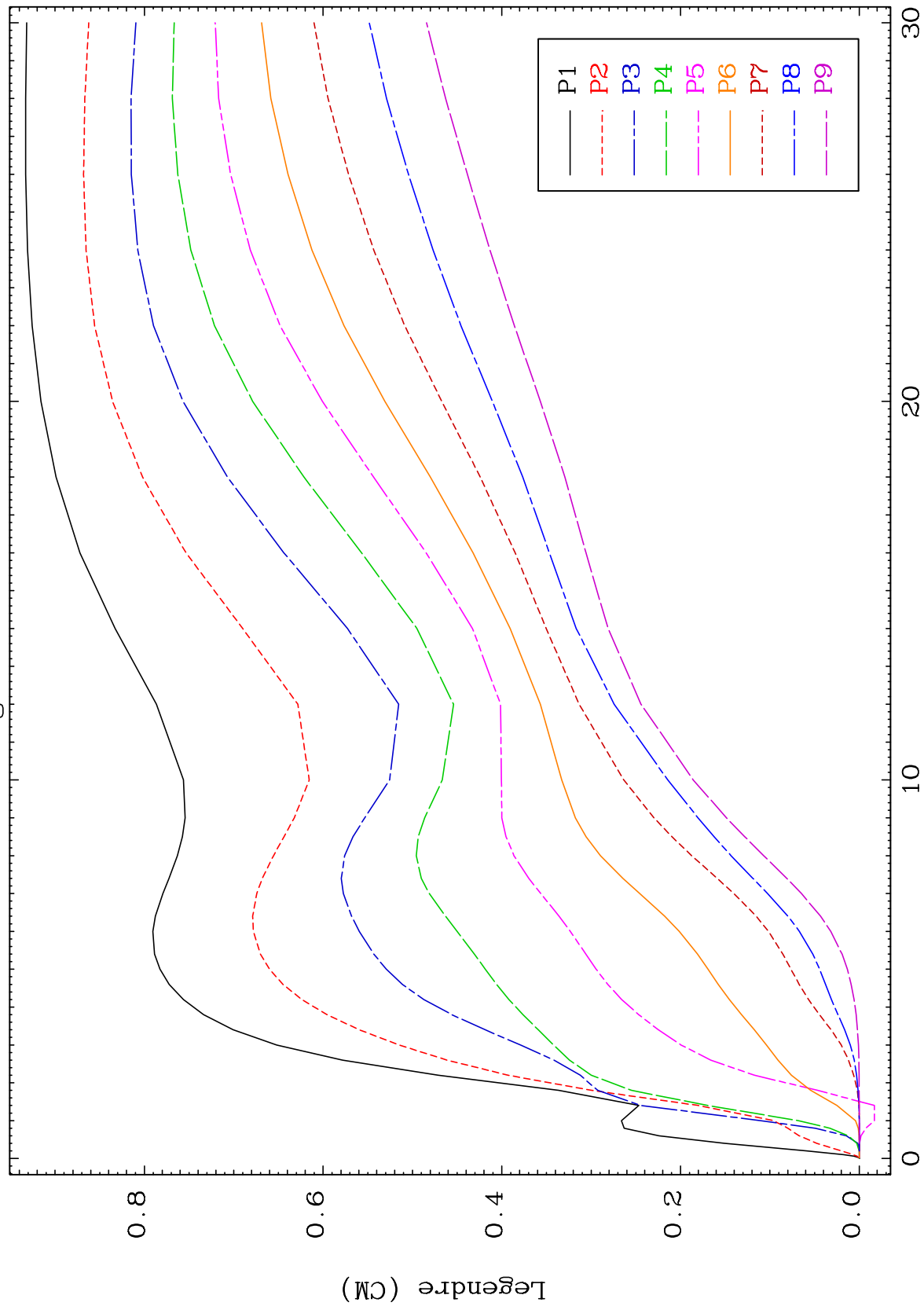




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Elastic Legendre Coefficients

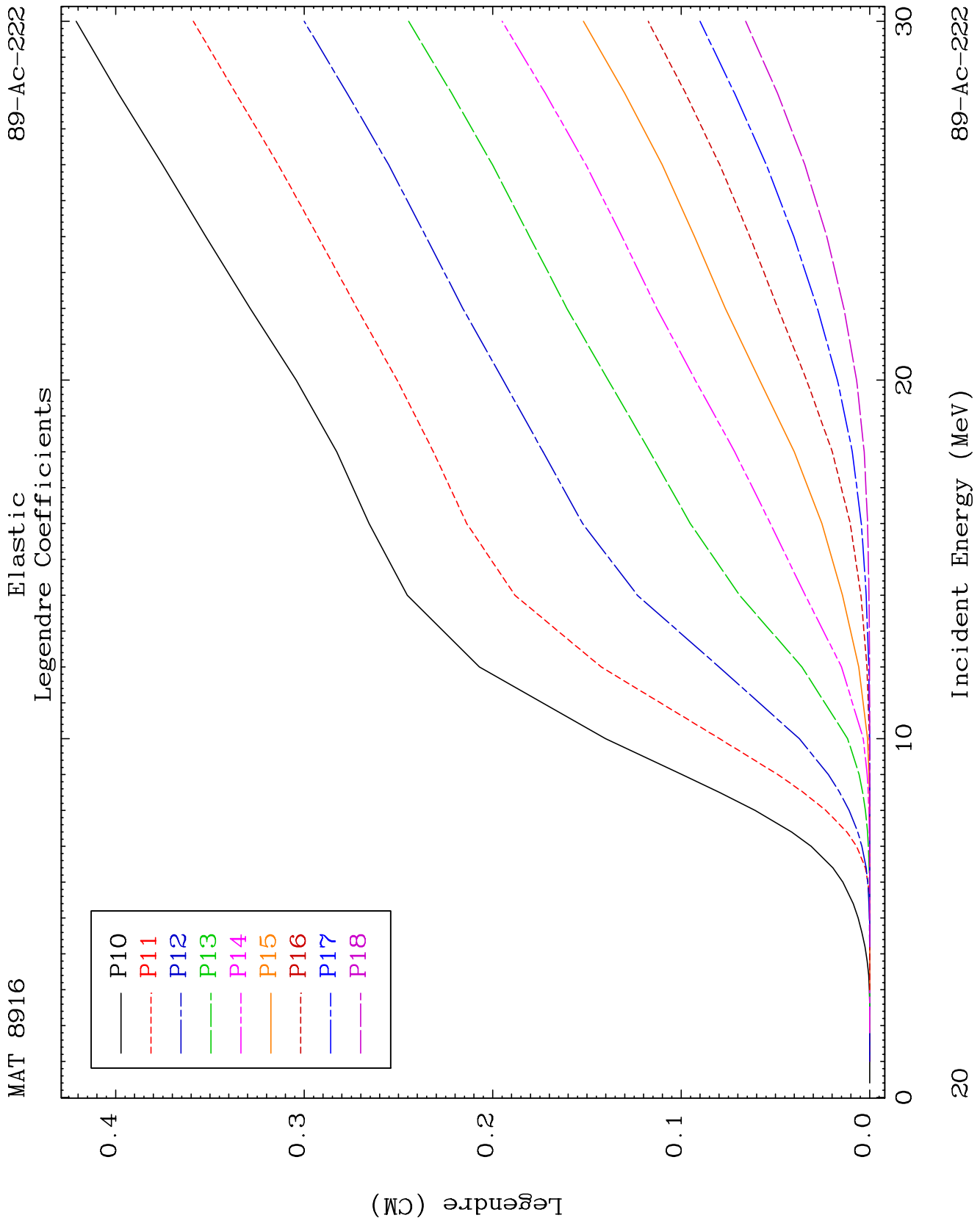
89-Ac-222



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Incident Energy (MeV)

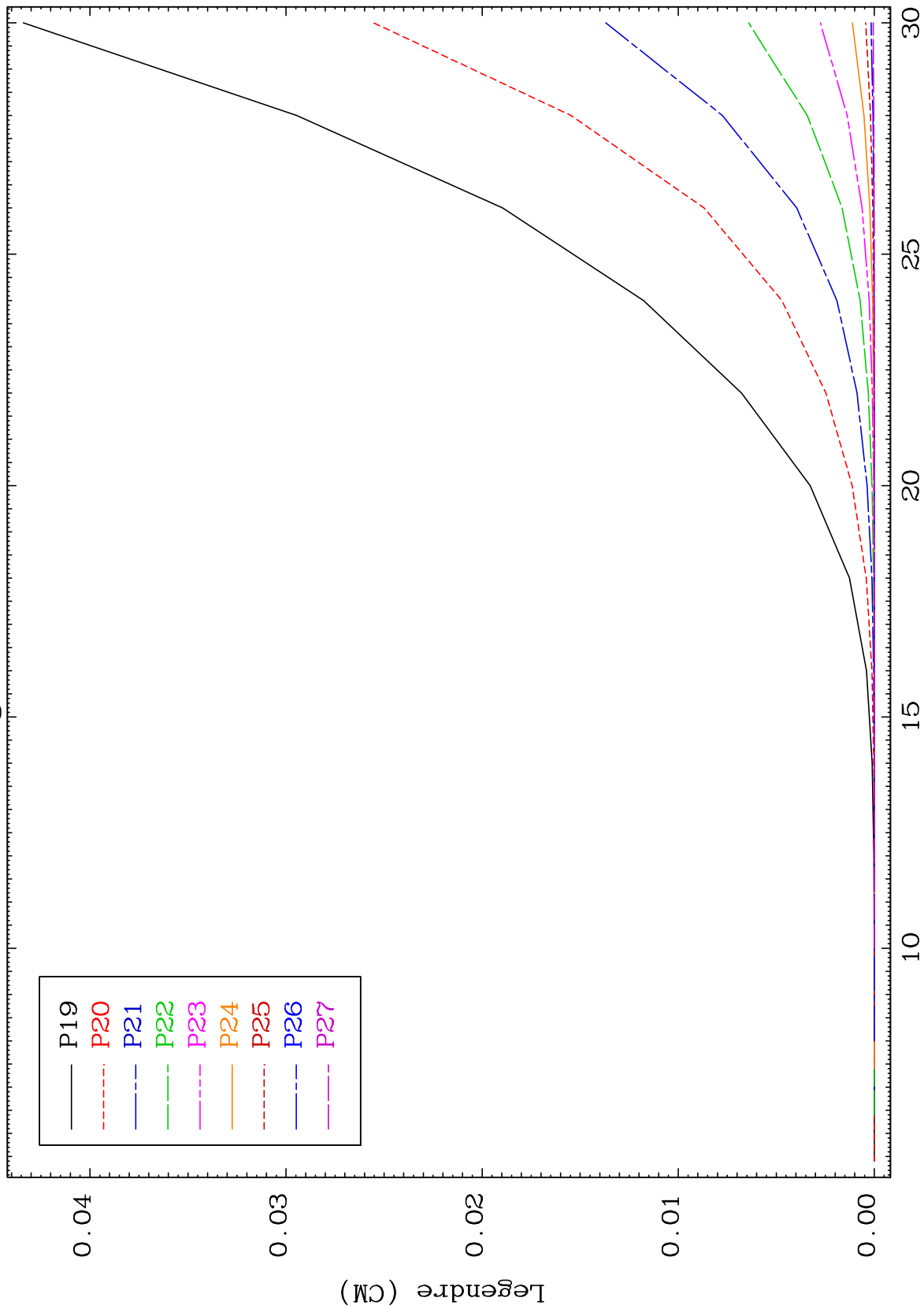
89-Ac-222



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Elastic Legendre Coefficients

89-Ac-222



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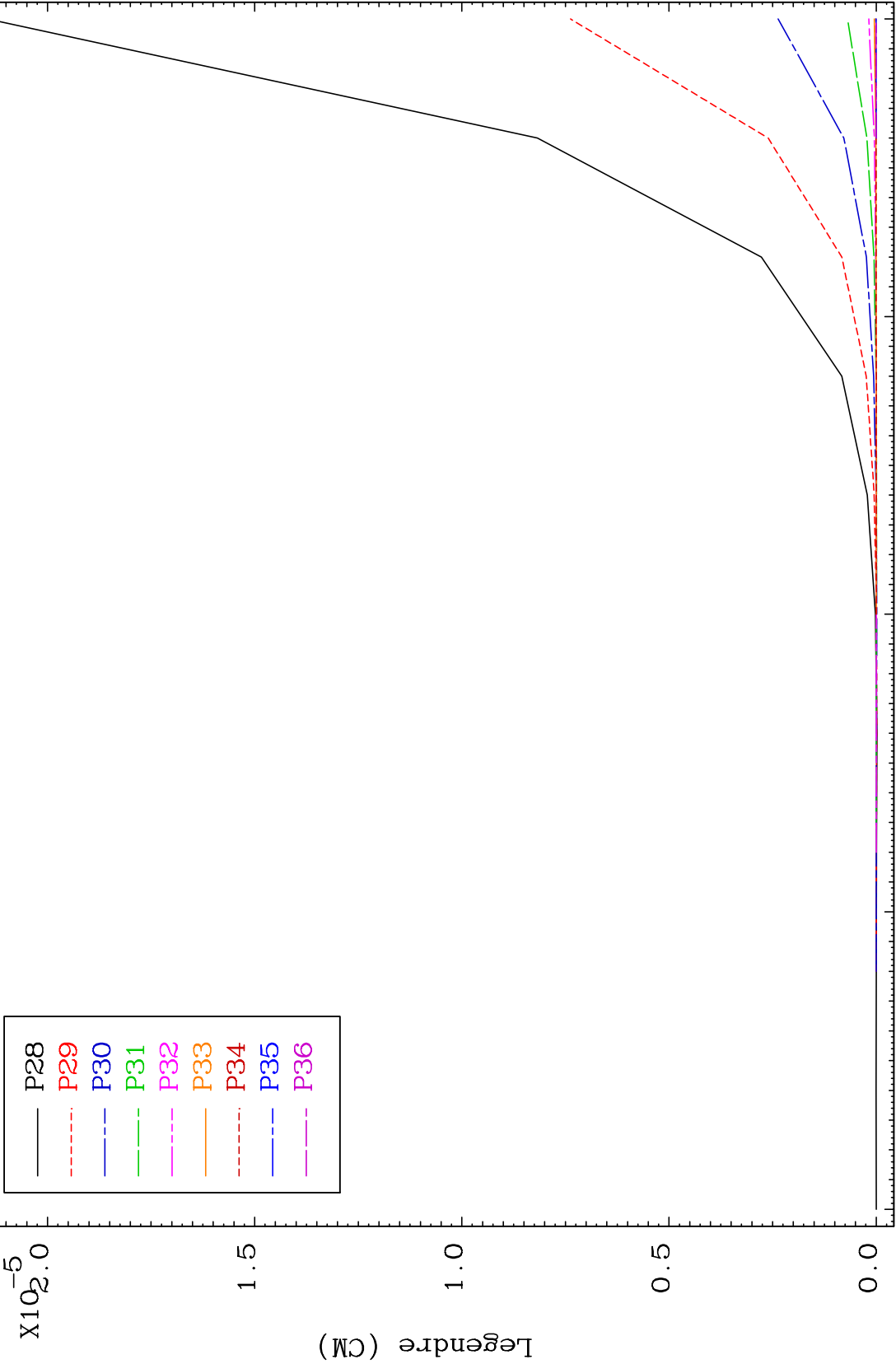
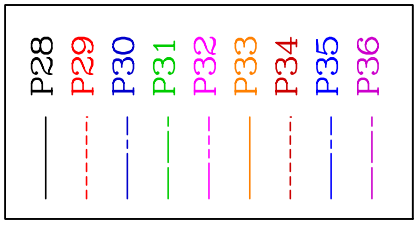
Incident Energy (MeV)

89-Ac-222

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Elastic Legendre Coefficients

89-Ac-222

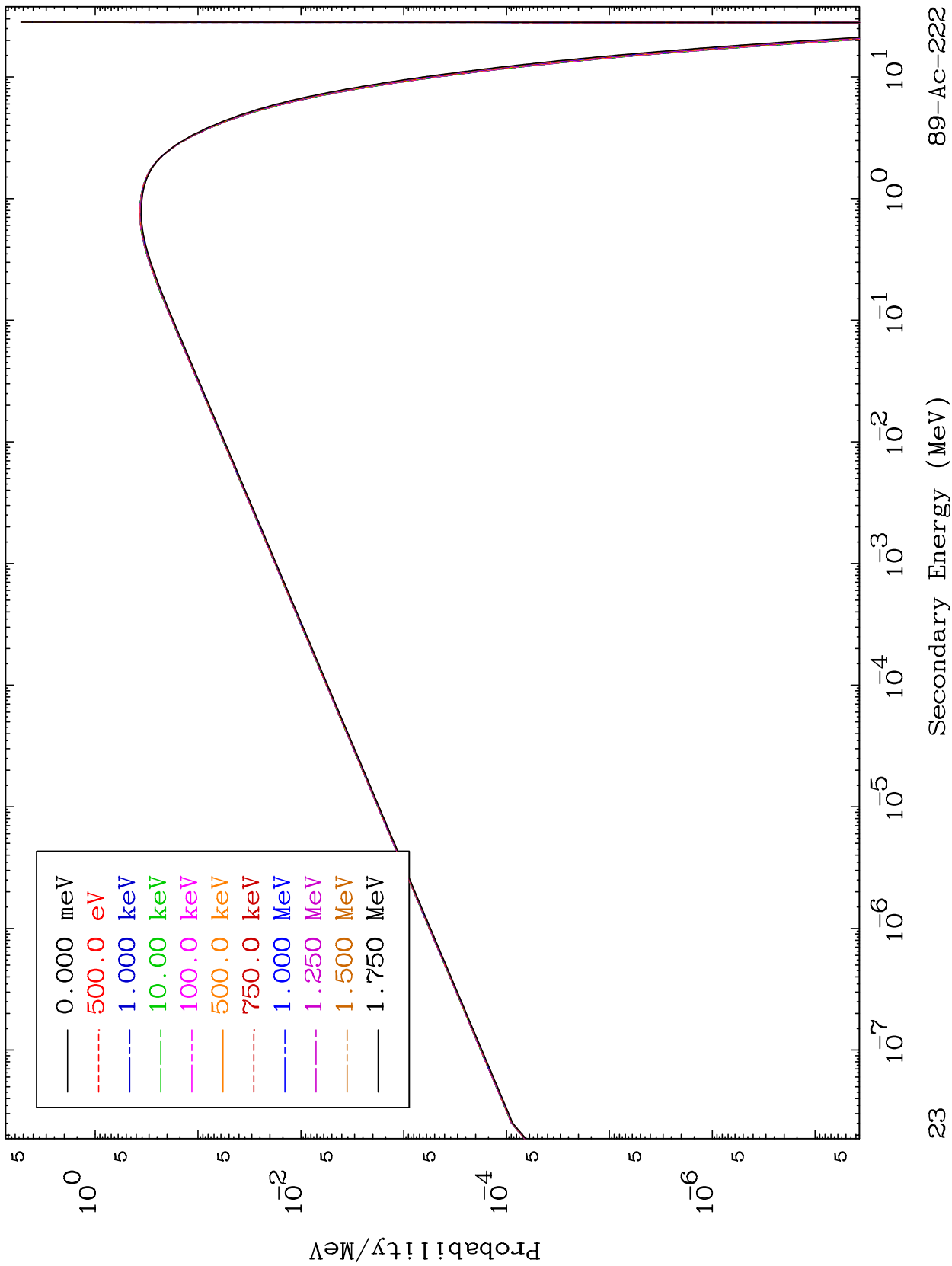


22

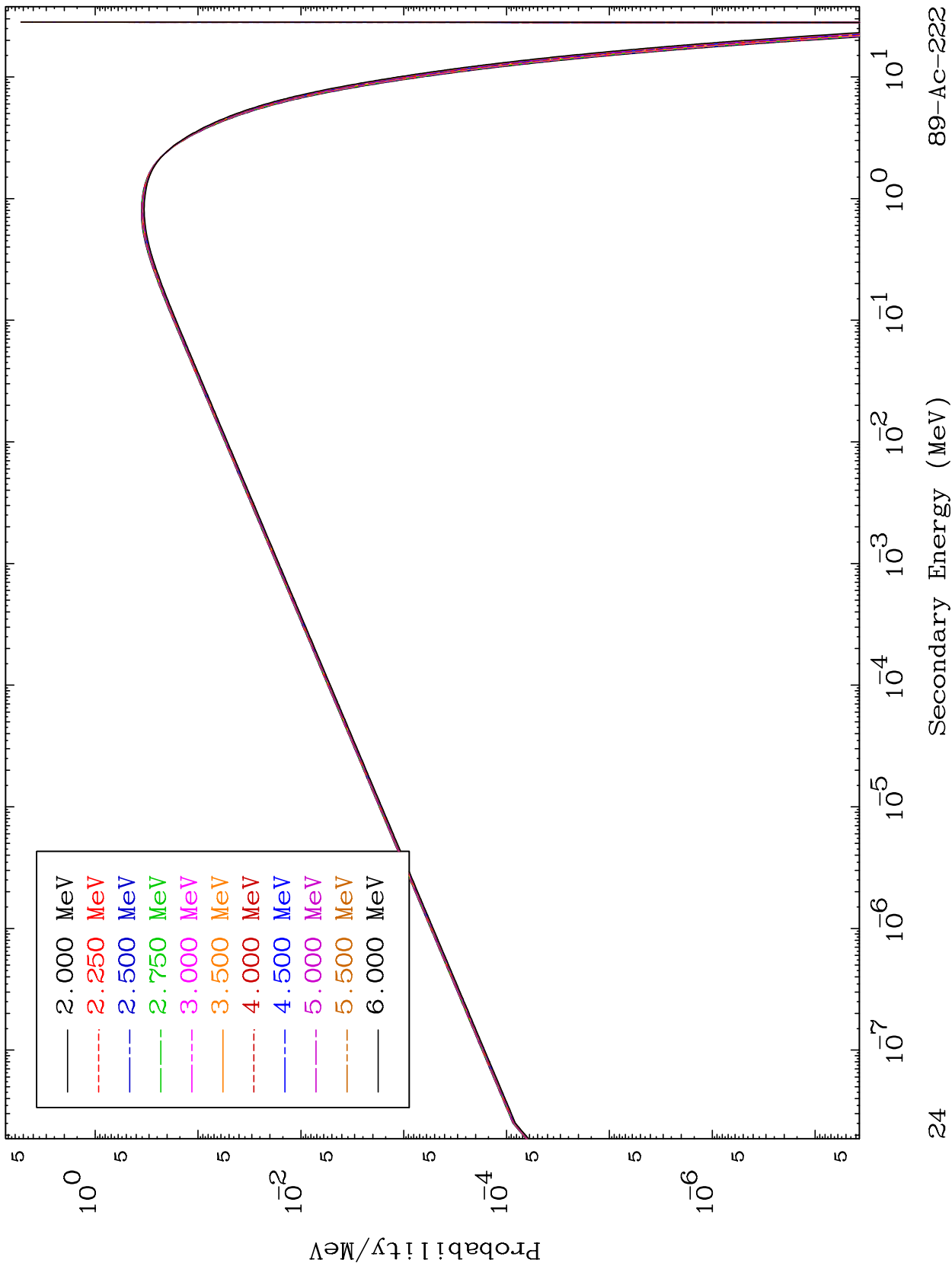
Incident Energy (MeV)

89-Ac-222

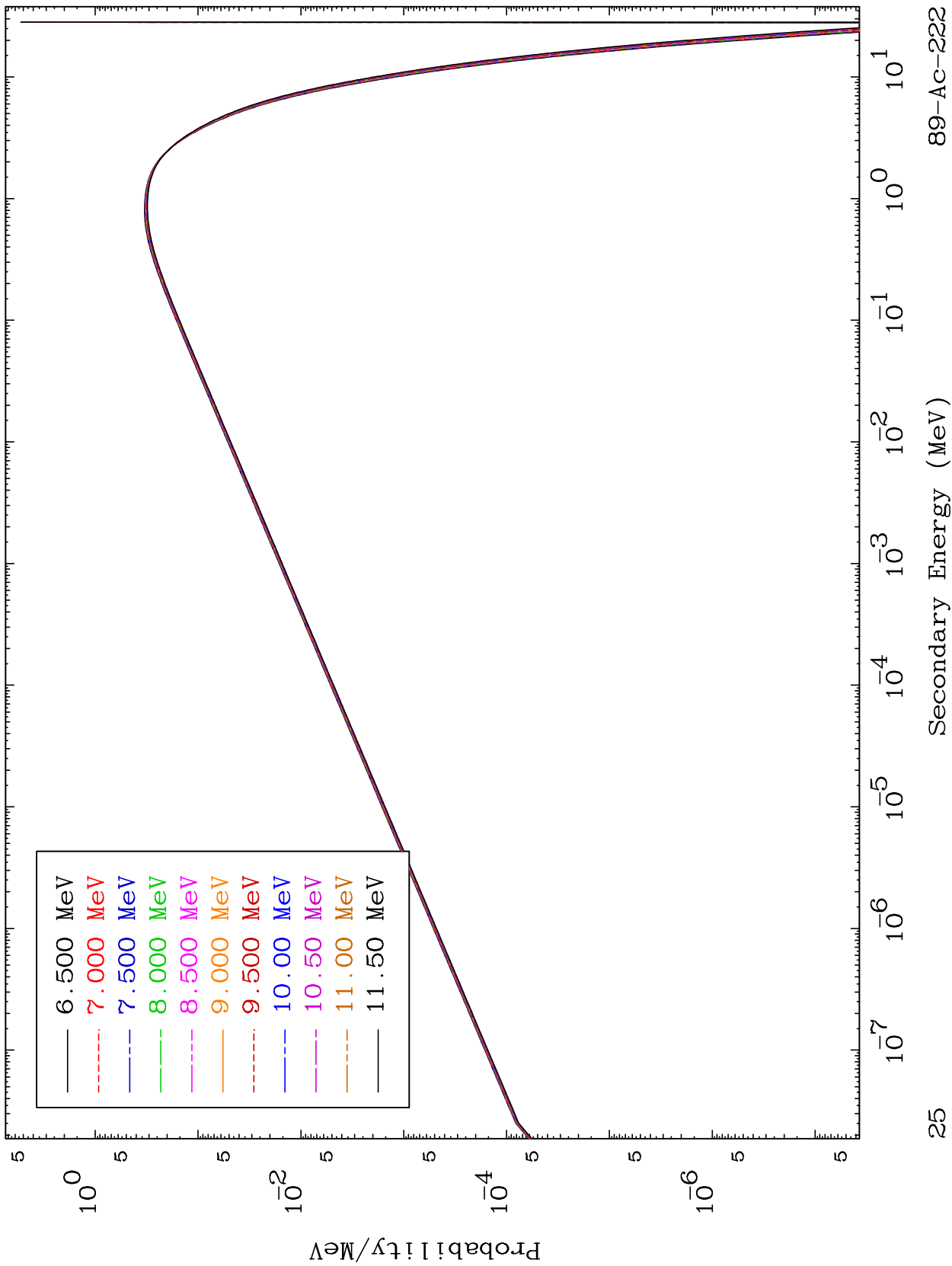
Fission Energy Distribution



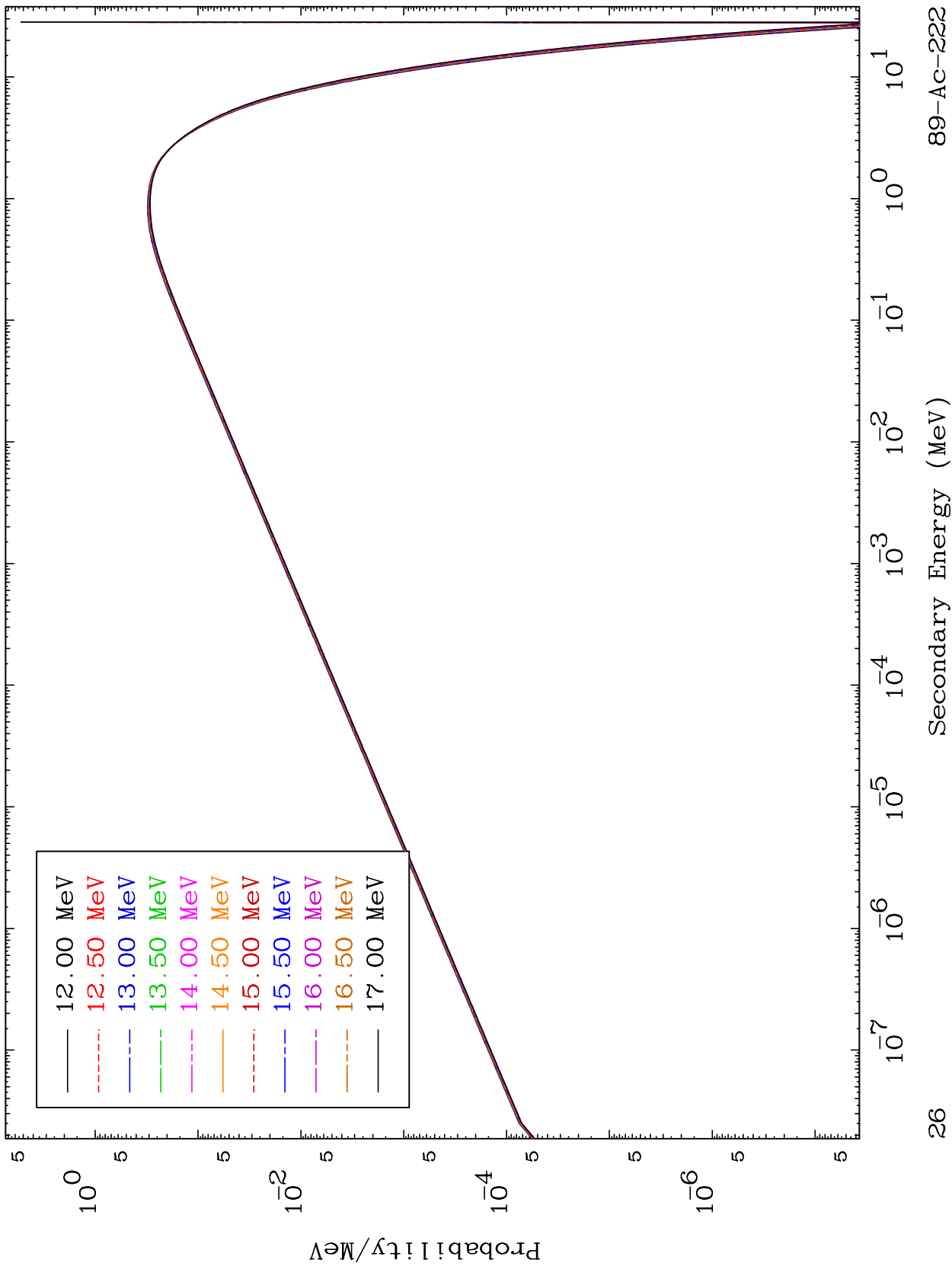
Fission Energy Distribution



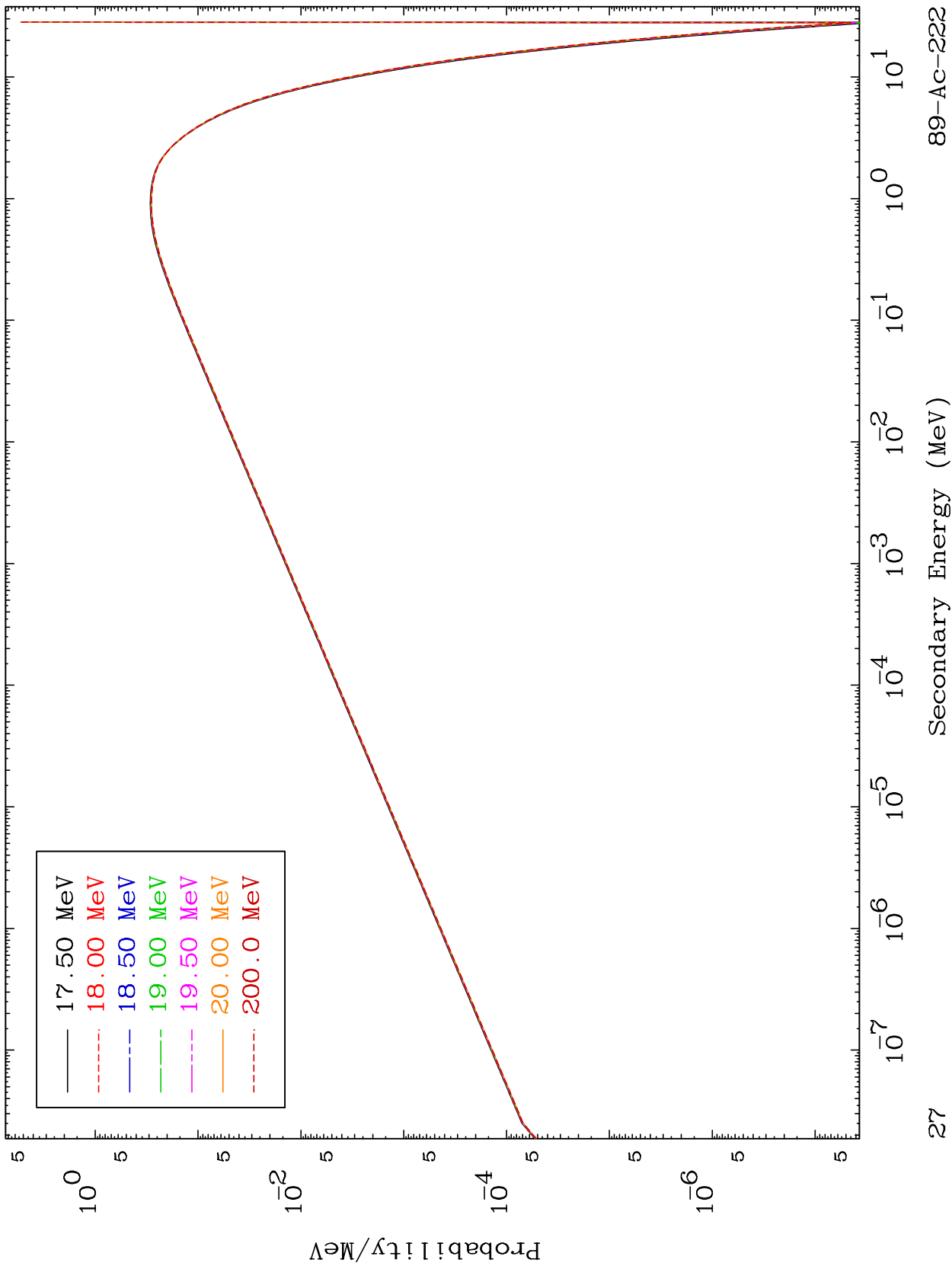
Fission Energy Distribution



Fission
Energy Distribution



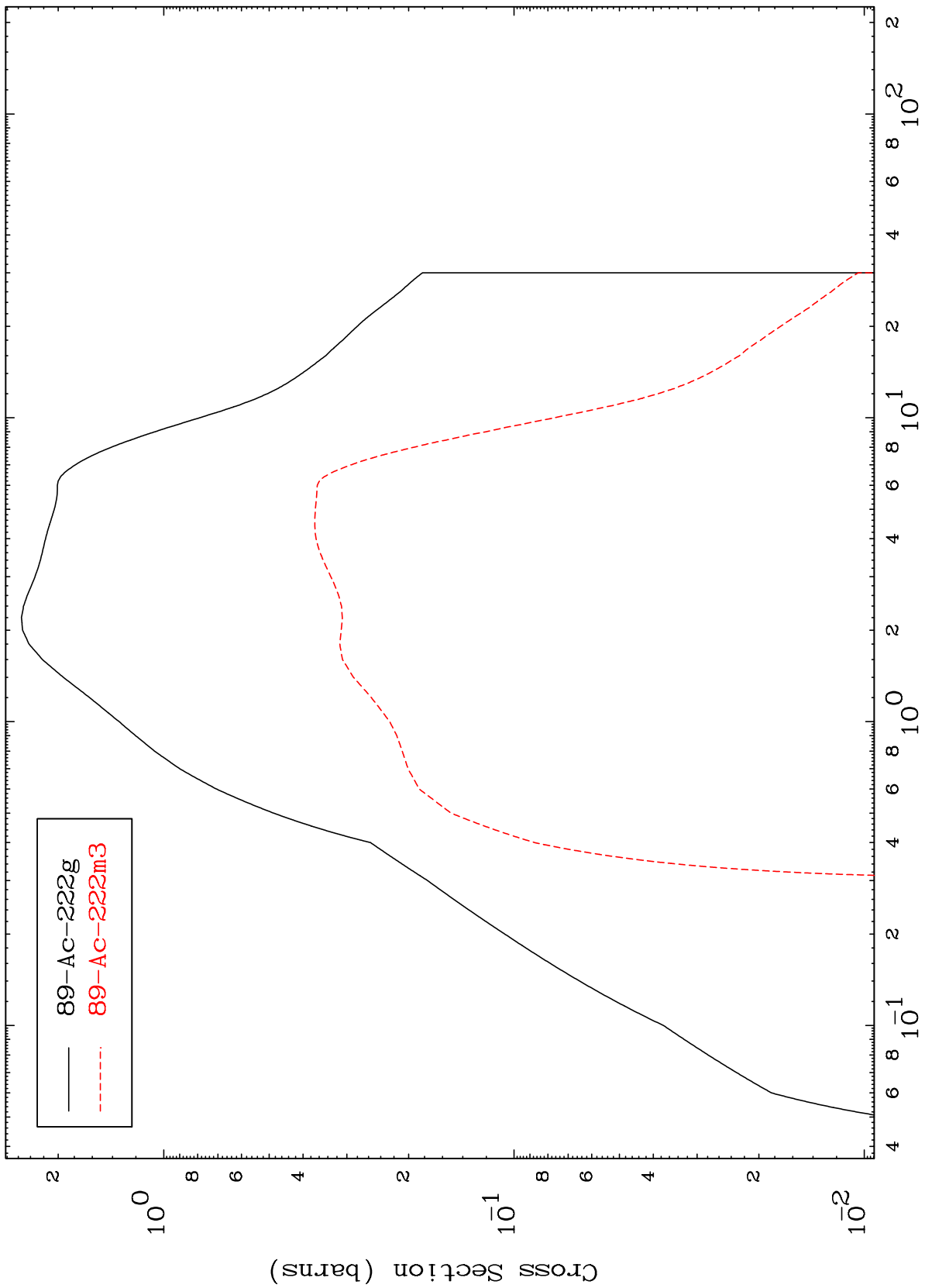
Fission
Energy Distribution



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Inelastic
Radionuclide Production Cross Section



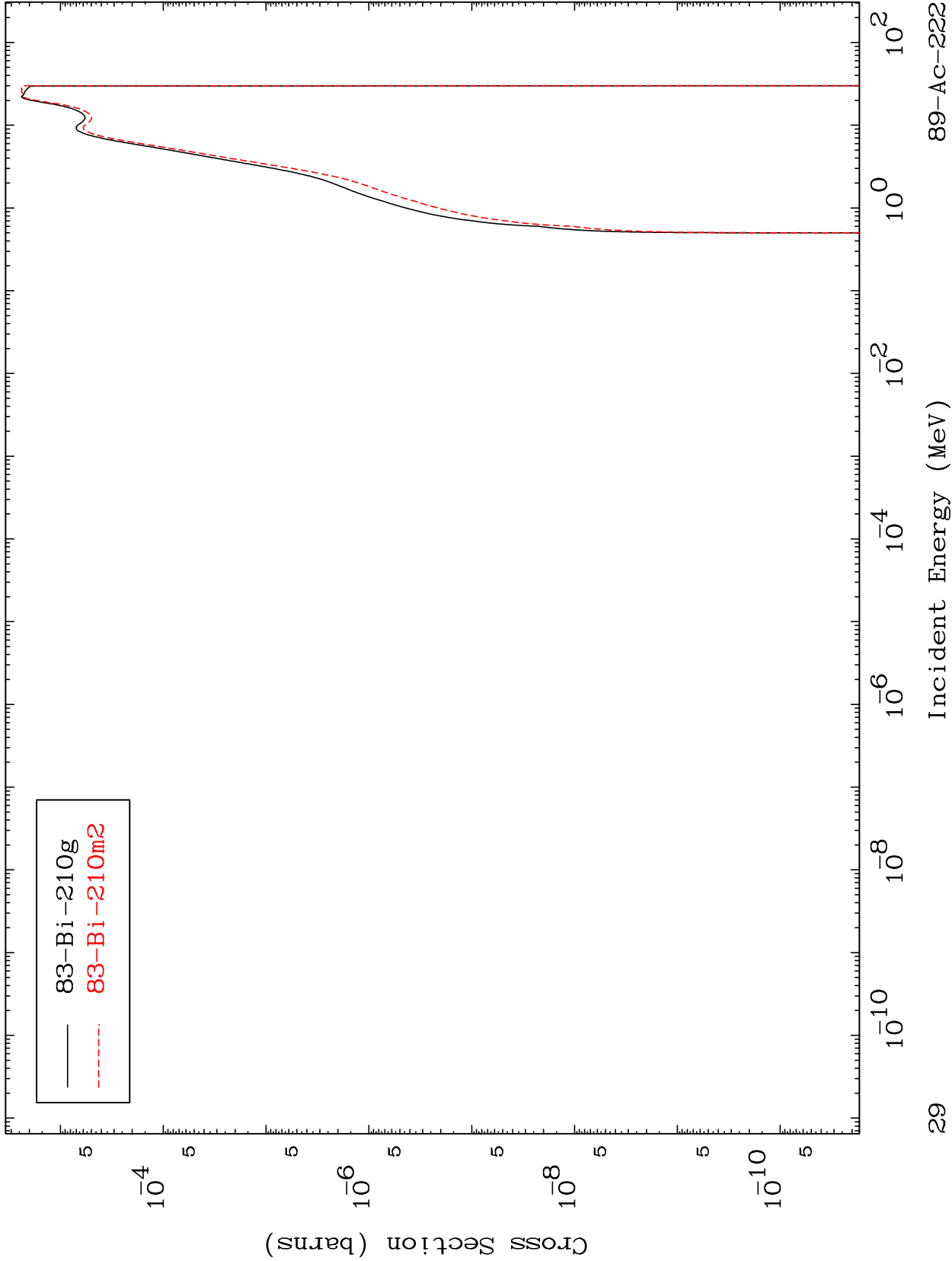
89-Ac-222g
89-Ac-222m3

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(n,n') 3α

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Radionuclide Production Cross Section

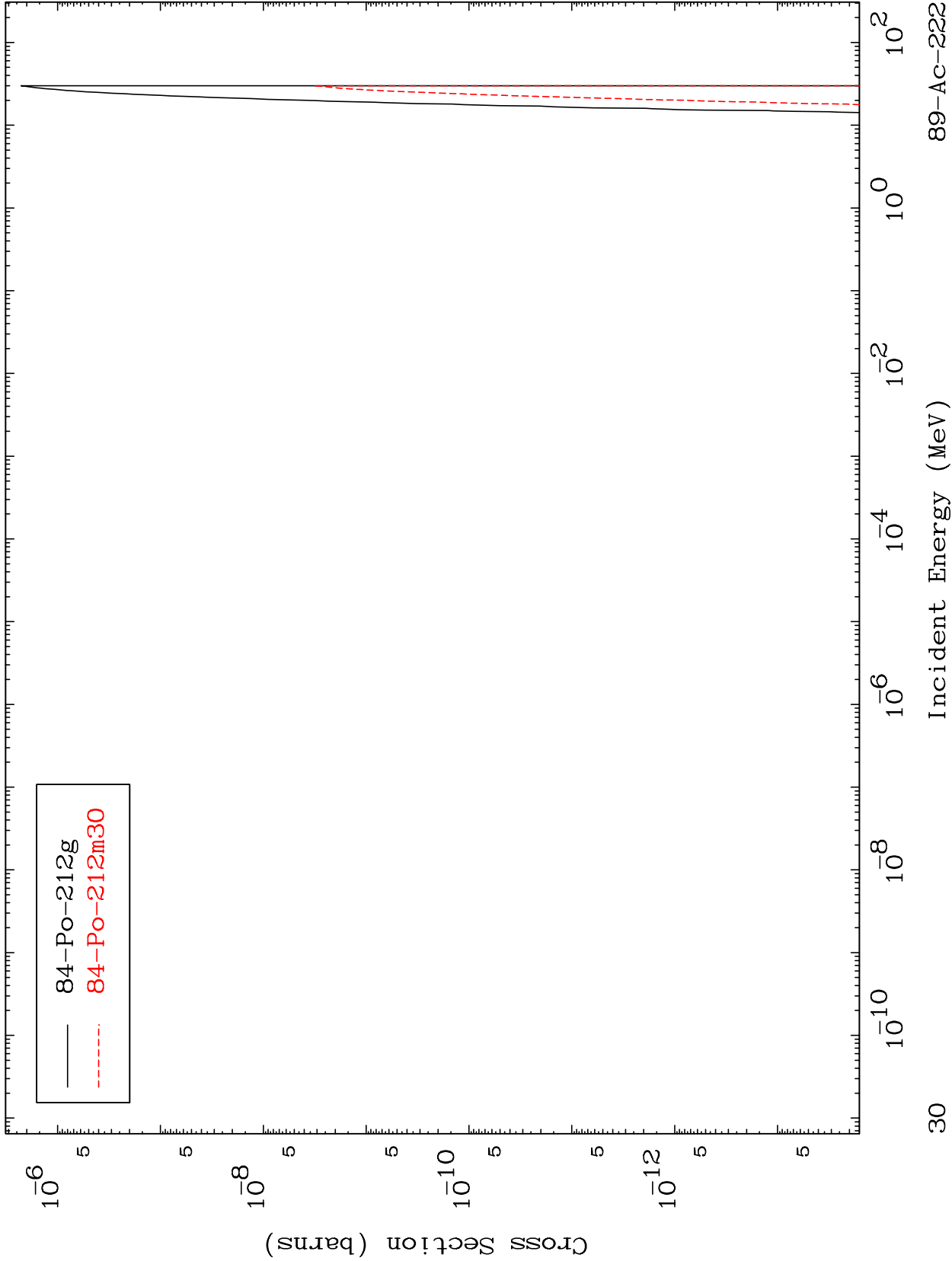


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(n,n') d,2 α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

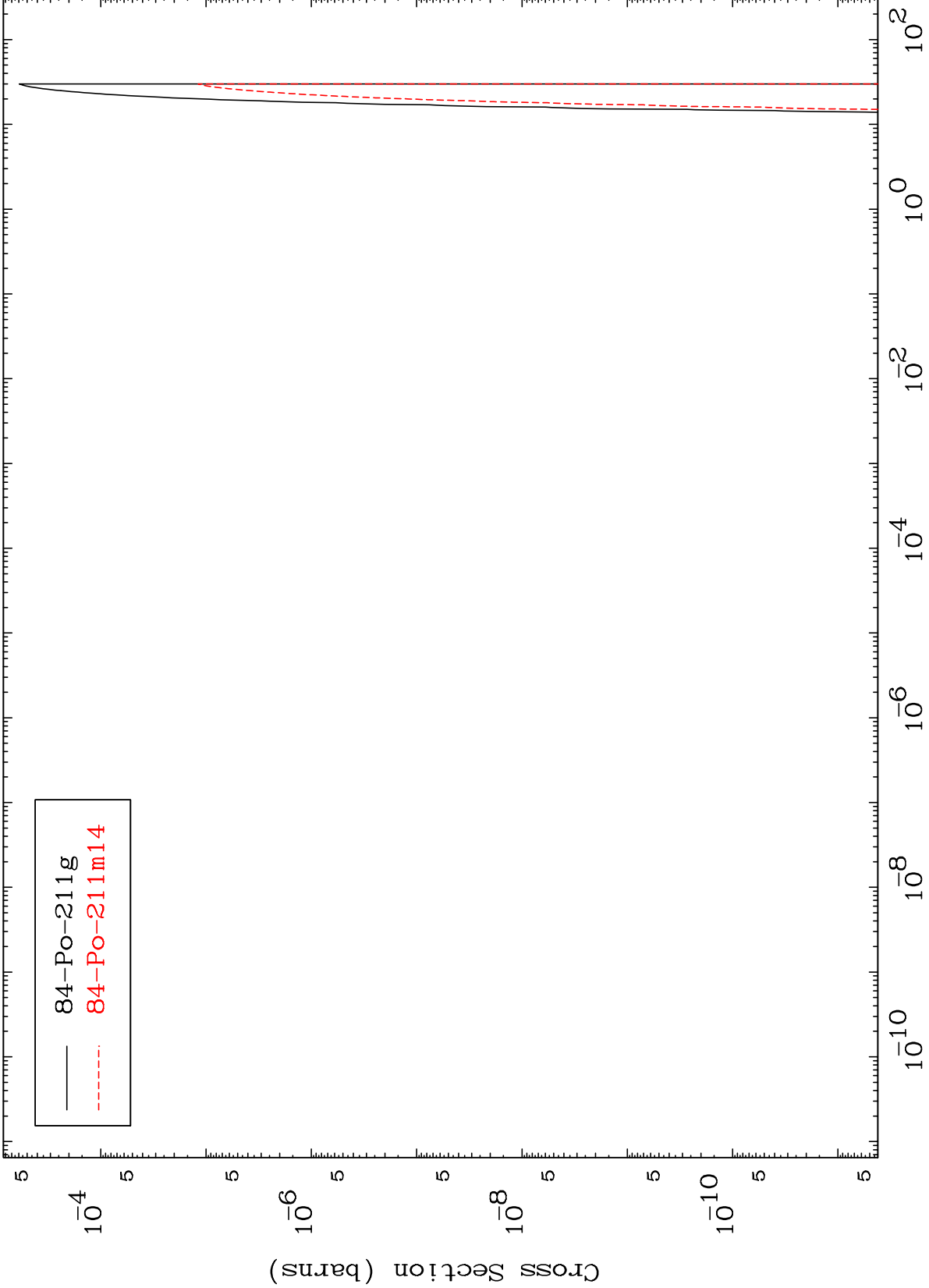
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(n,n') t,2 α

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Radionuclide Production Cross Section



MAT 8916

(n,t) 2 α

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Radionuclide Production Cross Section

