

Program EVALPLOT
(Version 2015-2)

by

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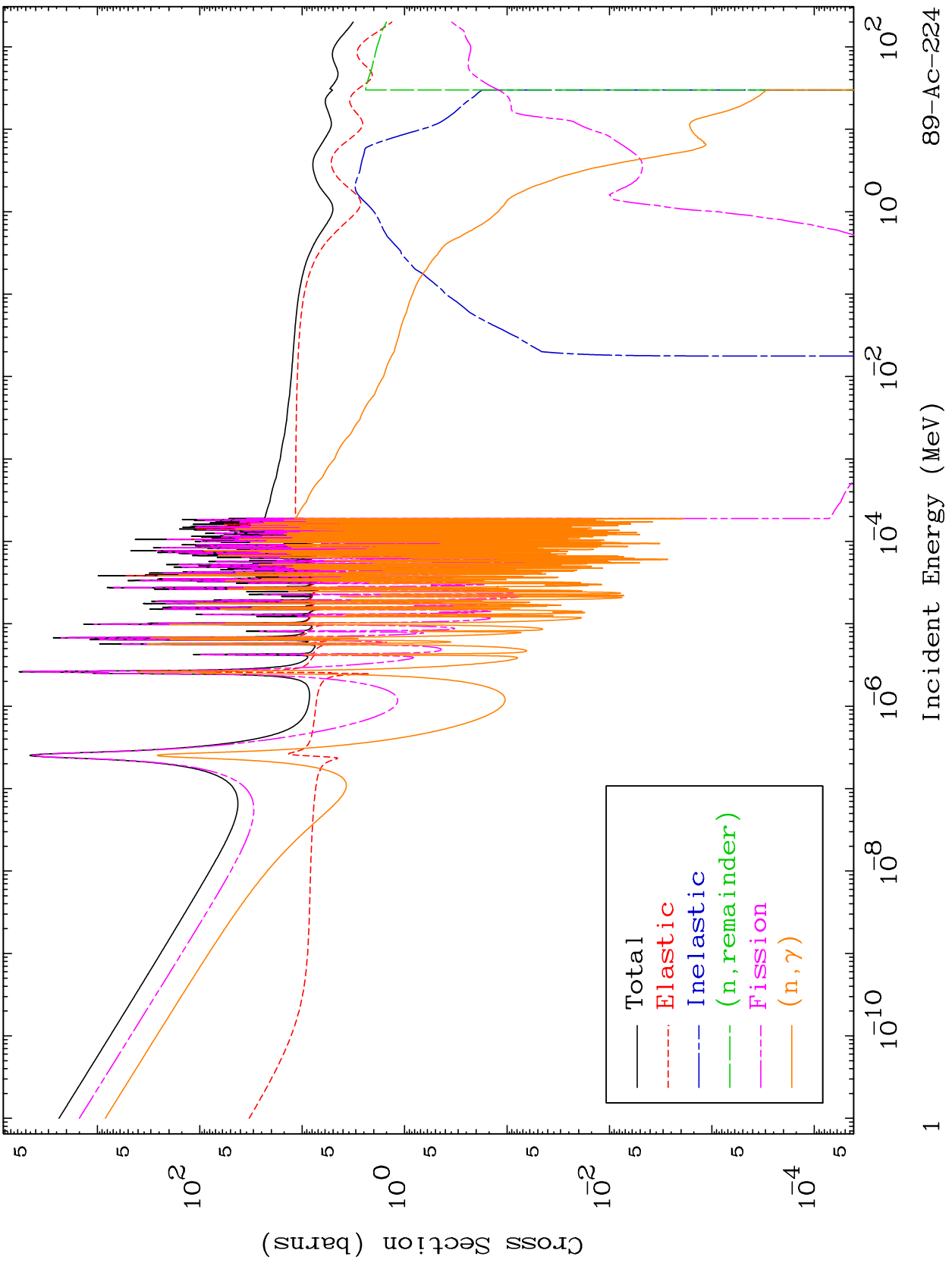
Press Mouse Button to Start

MAT 8922

Major

293 Kelvin Cross Sections

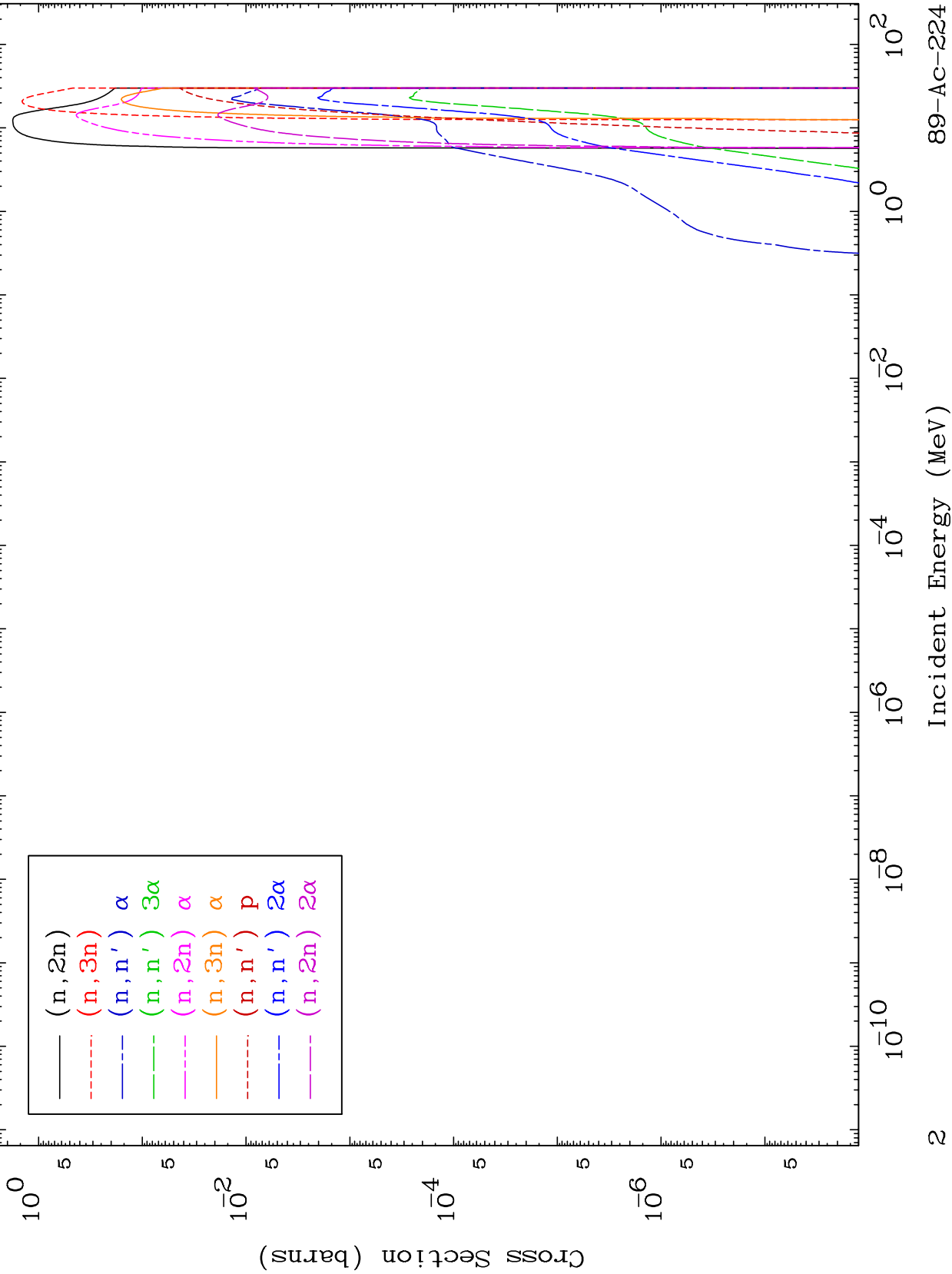
89-Ac-224



MAT 8922

Neutron Production
293 Kelvin Cross Sections

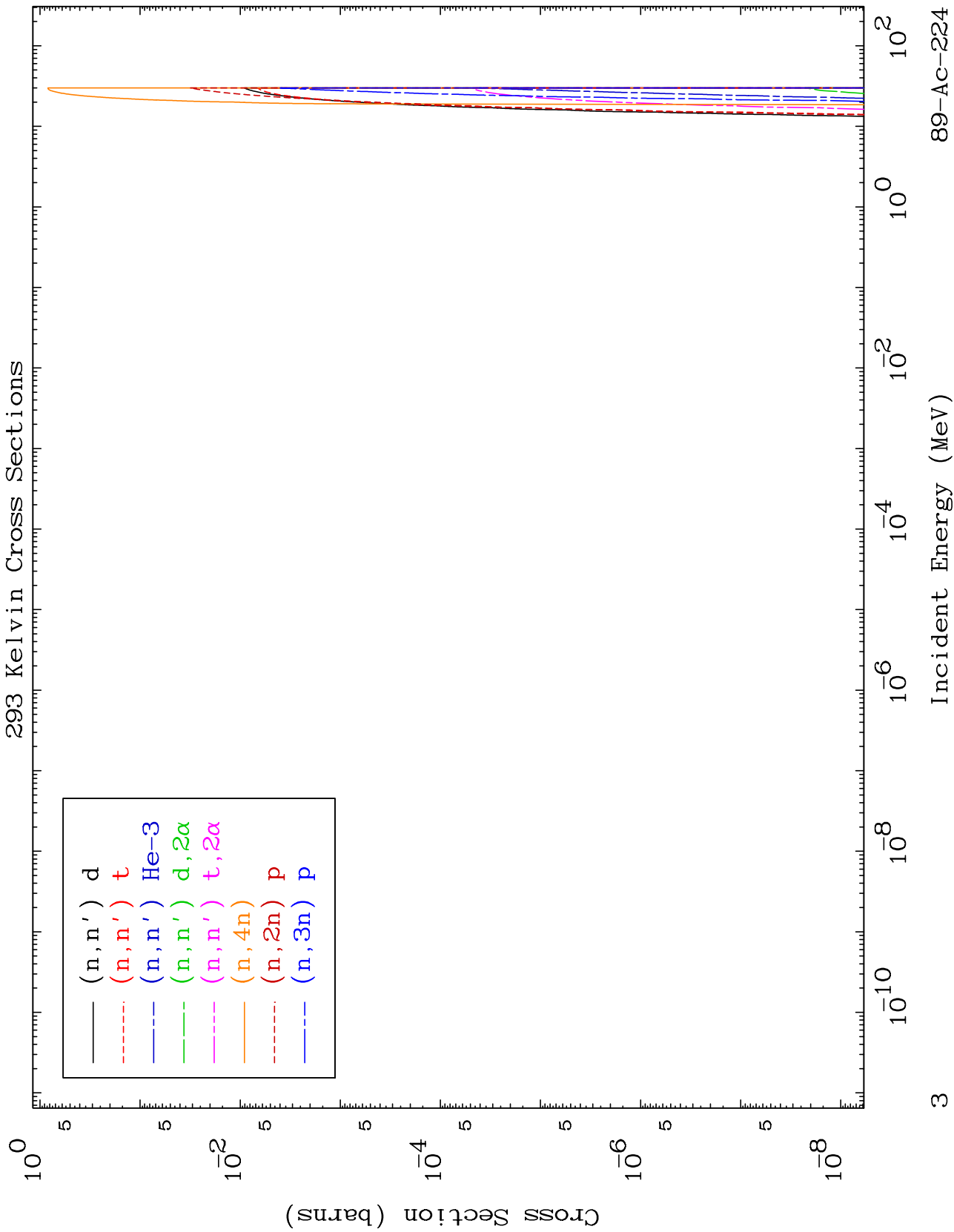
89-Ac-224



MAT 8922

Neutron Production
293 Kelvin Cross Sections

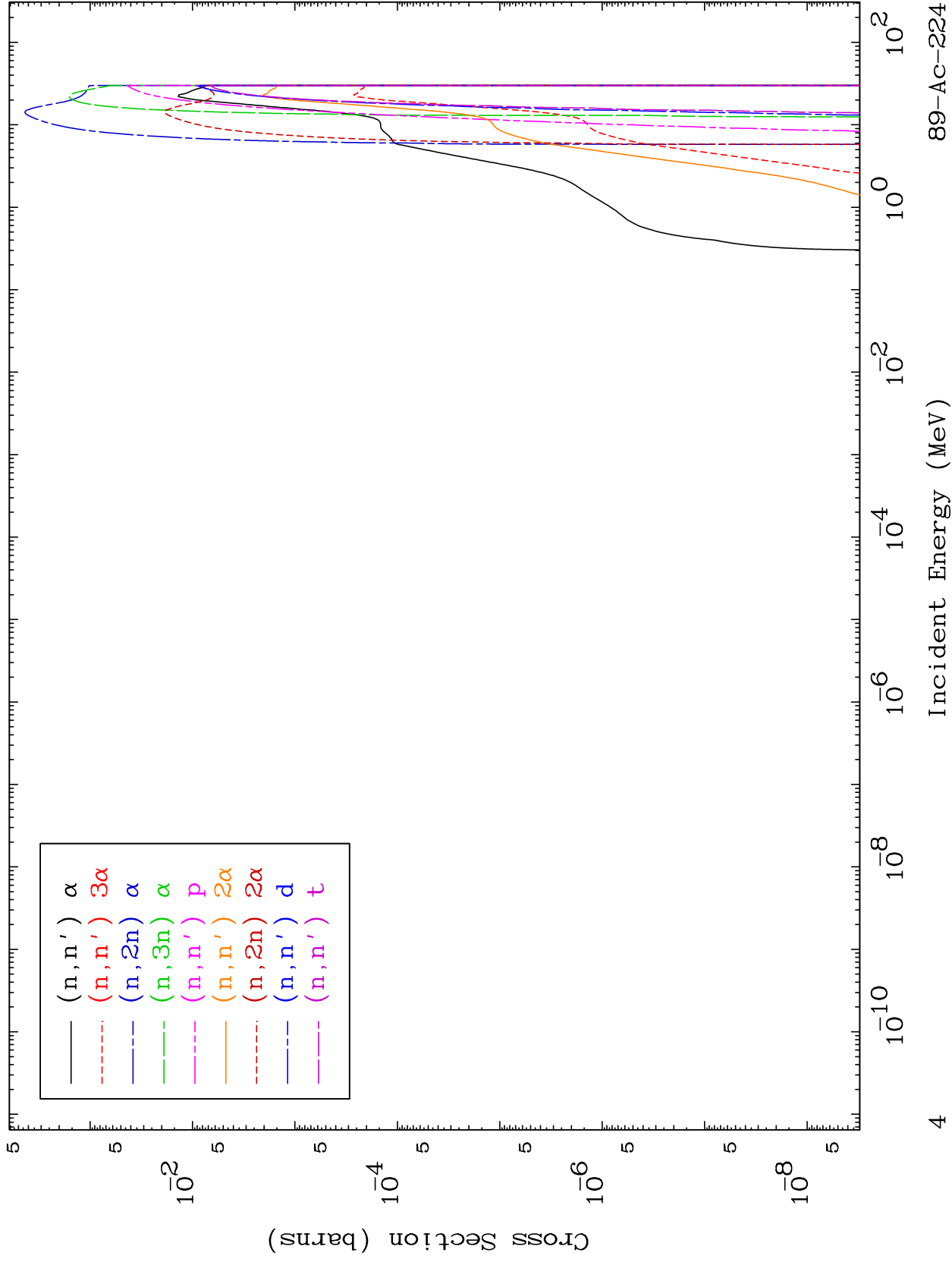
89-Ac-224

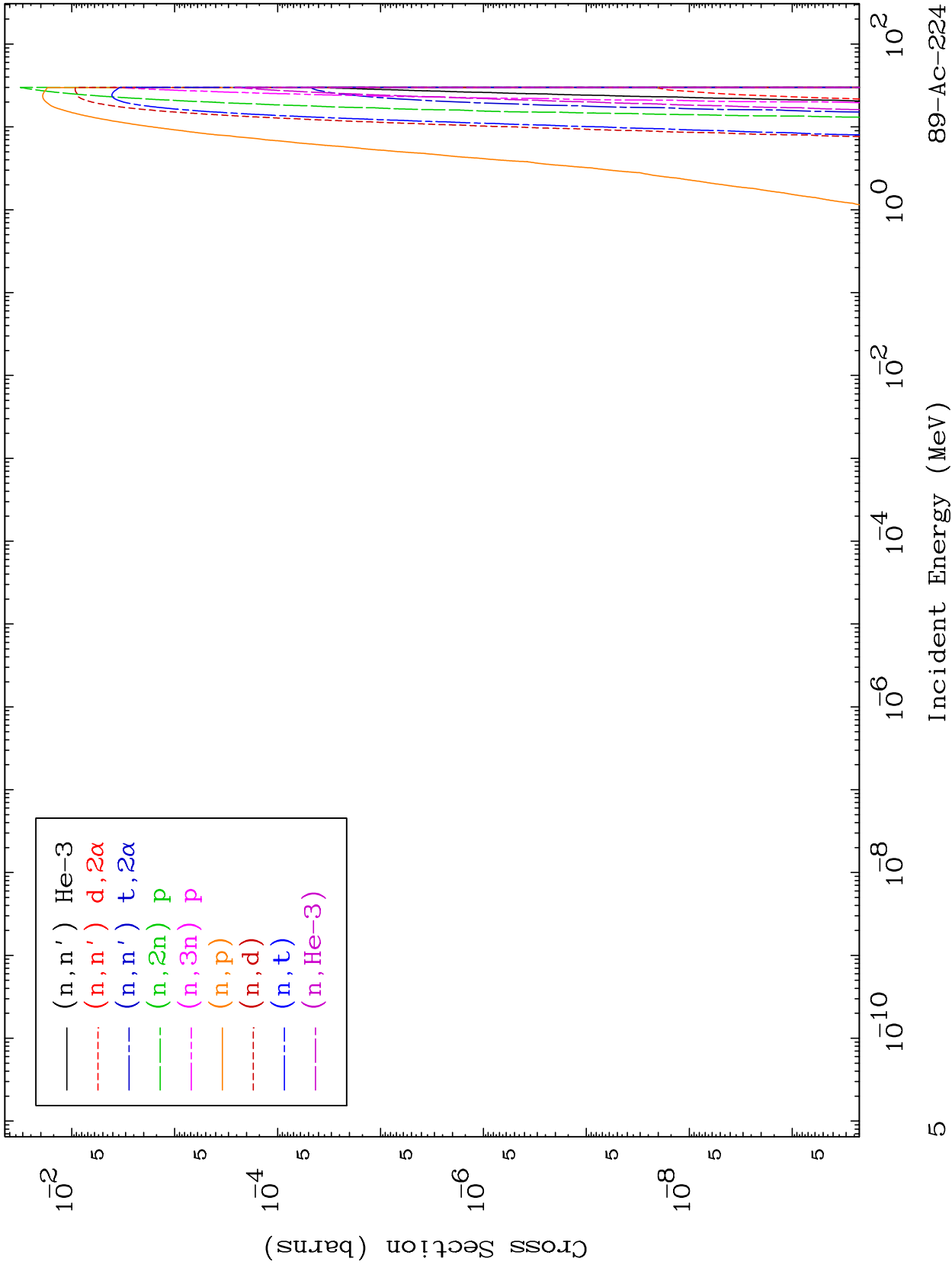


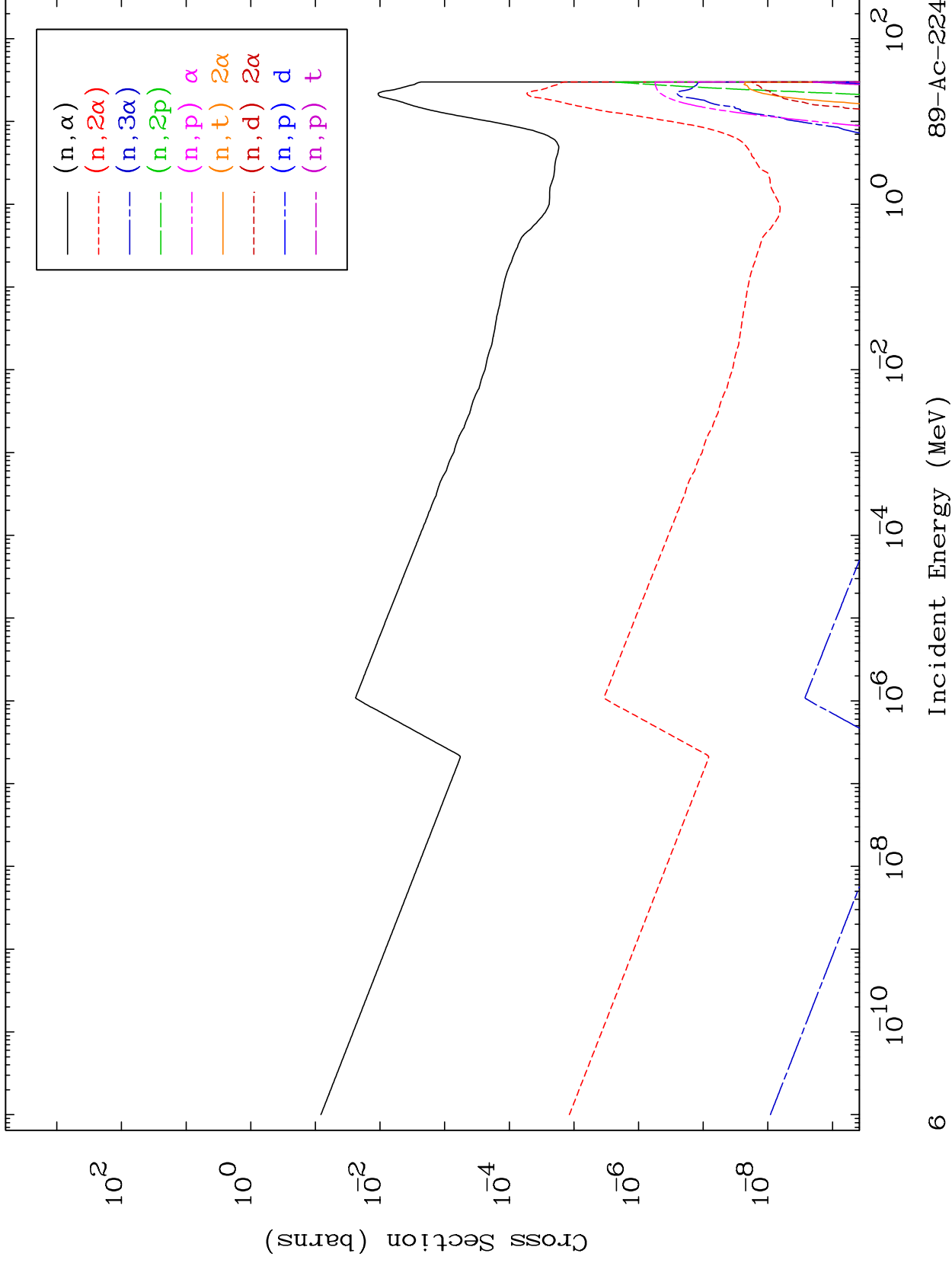
MAT 8922

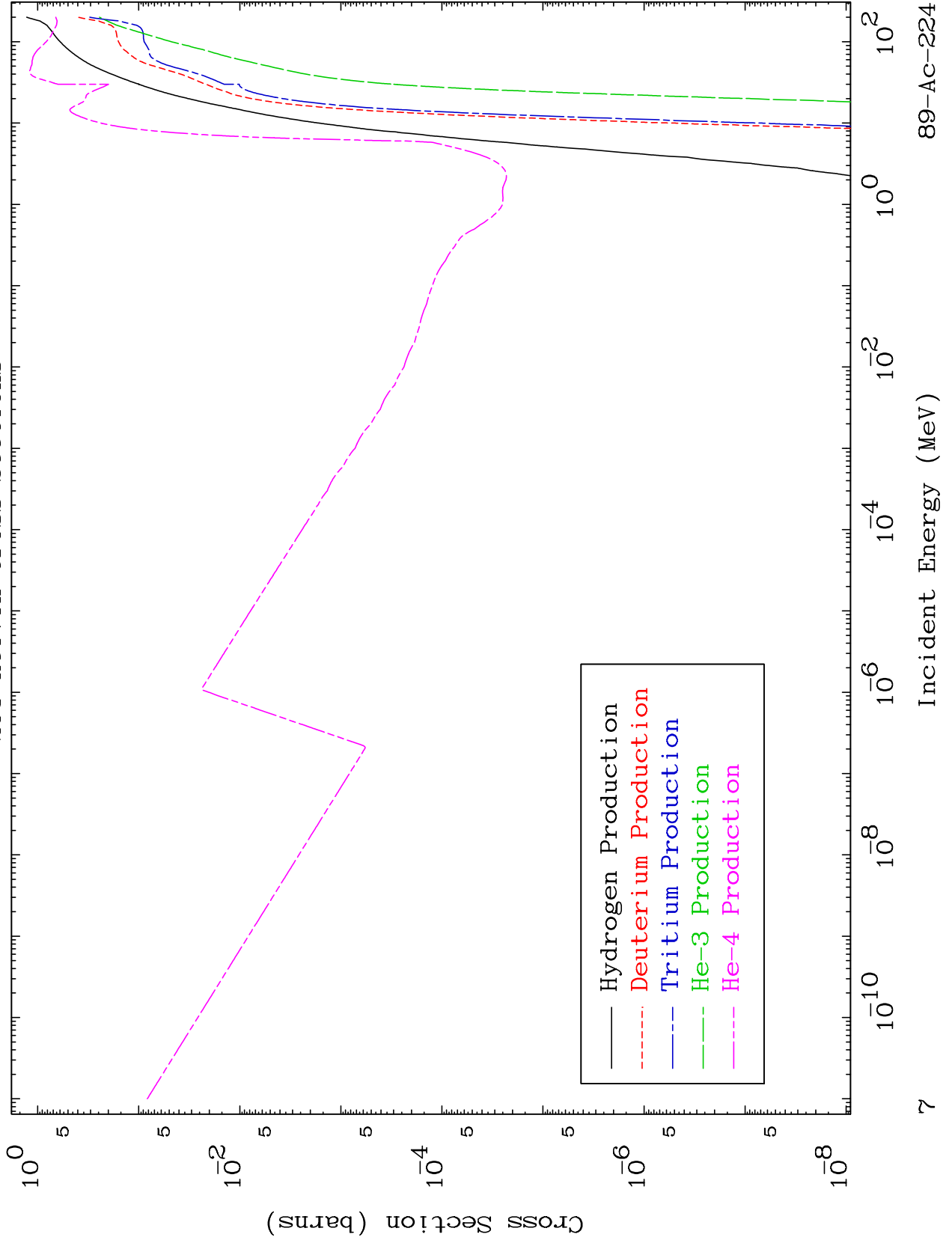
Charged Particle
293 Kelvin Cross Sections

89-Ac-224





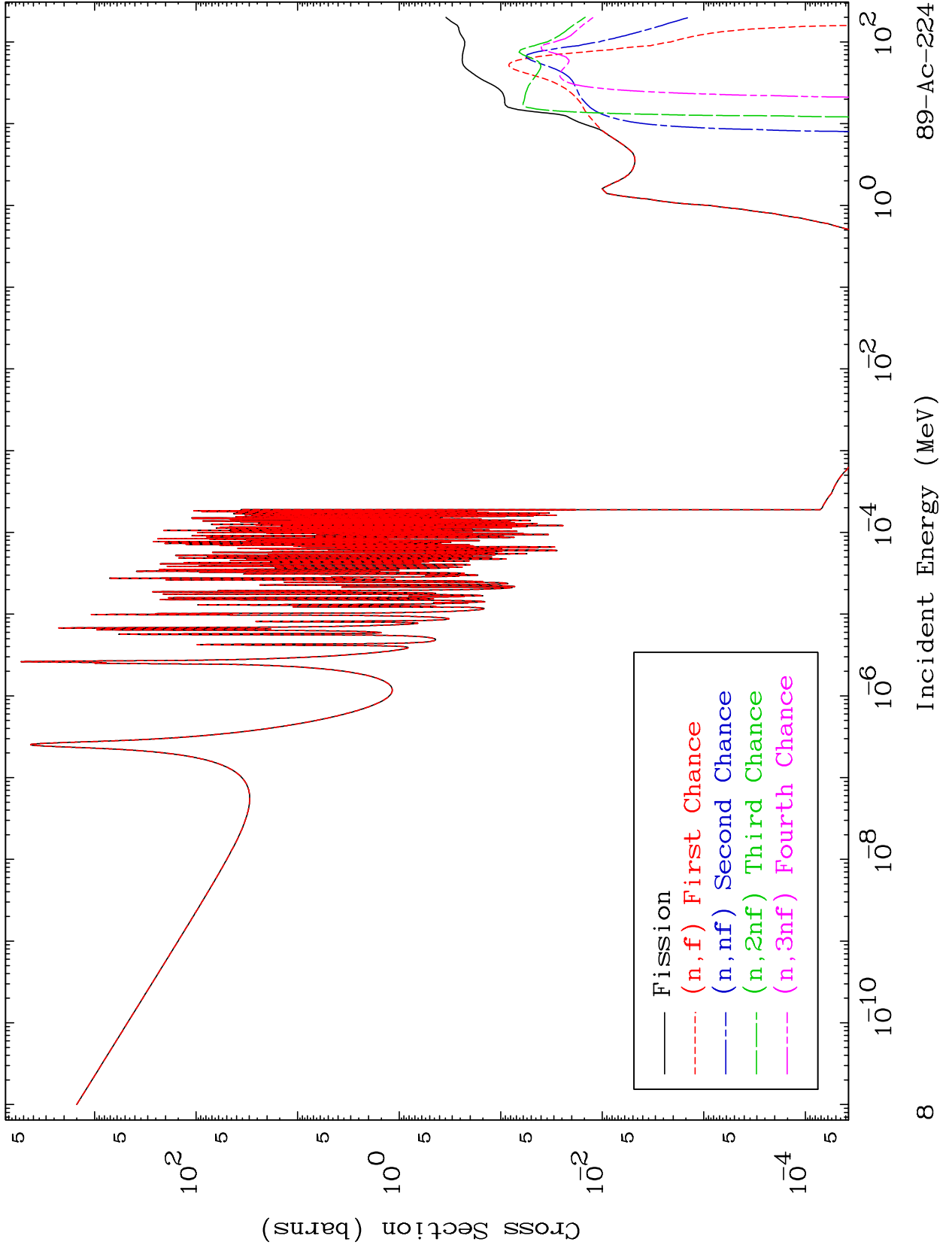


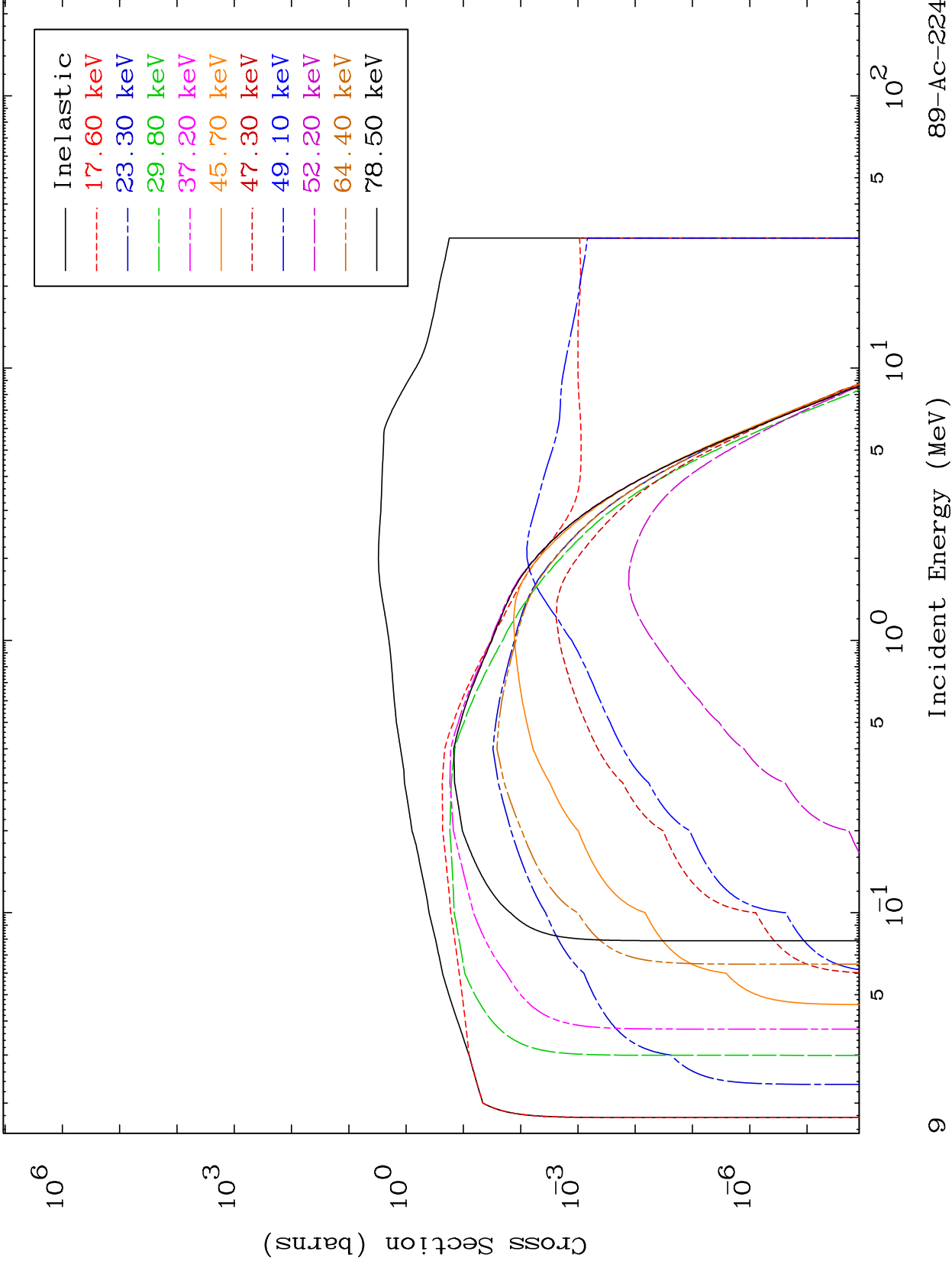


MAT 8922

Fission
293 Kelvin Cross Sections

89-Ac-224

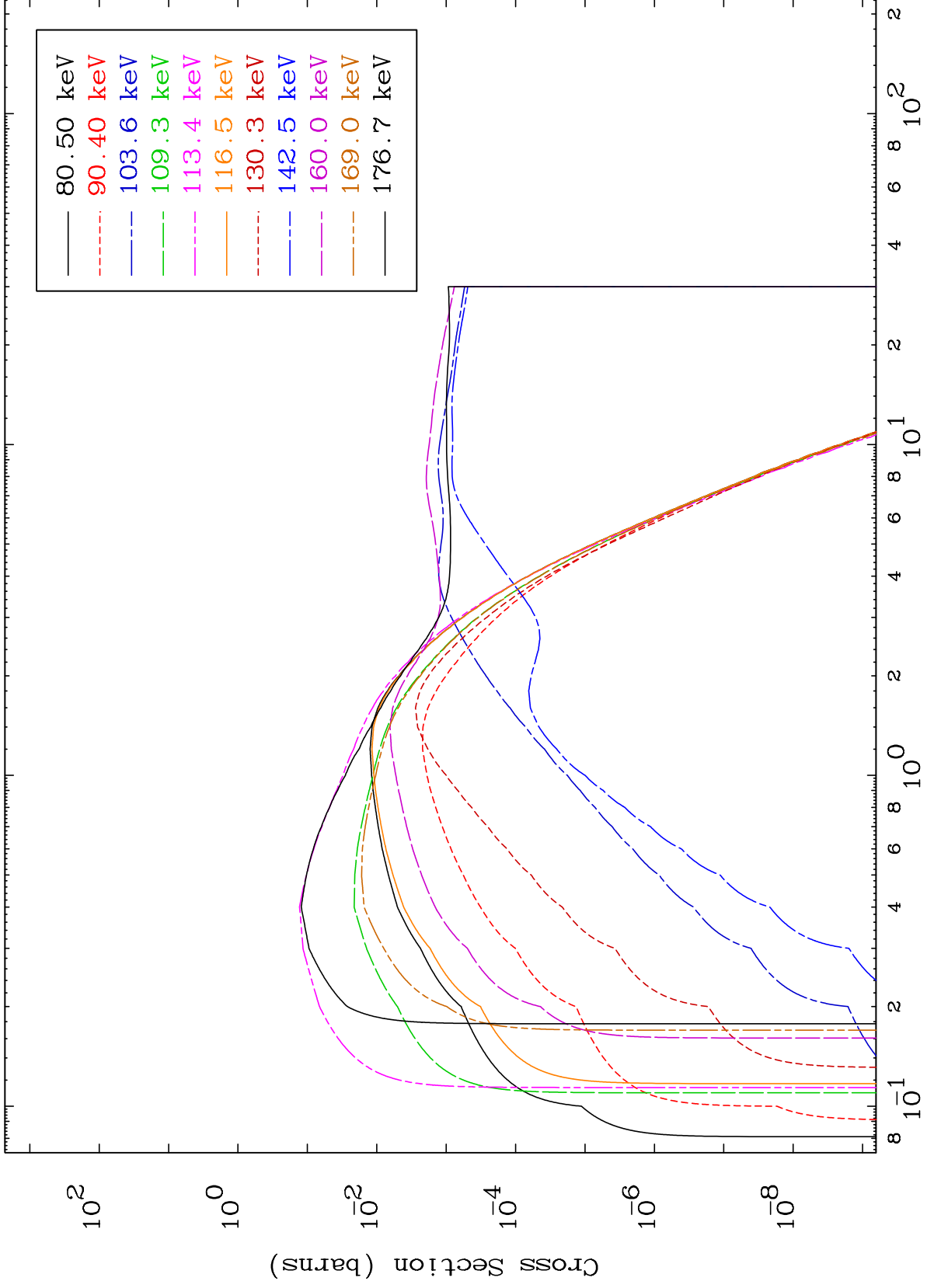




MAT 8922

(n,n') Level
293 Kelvin Cross Sections

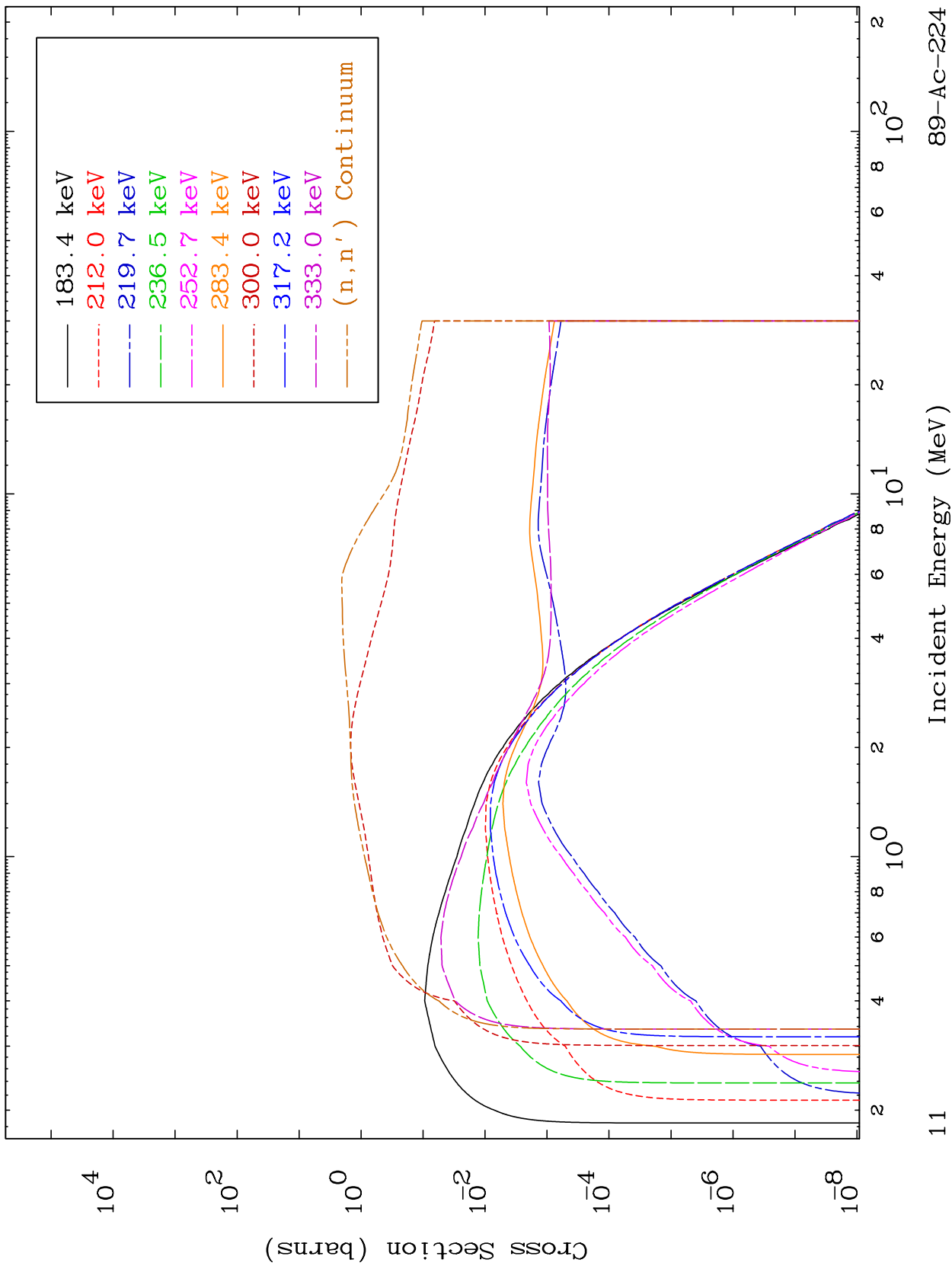
89-Ac-224



89-Ac-224

Incident Energy (MeV)

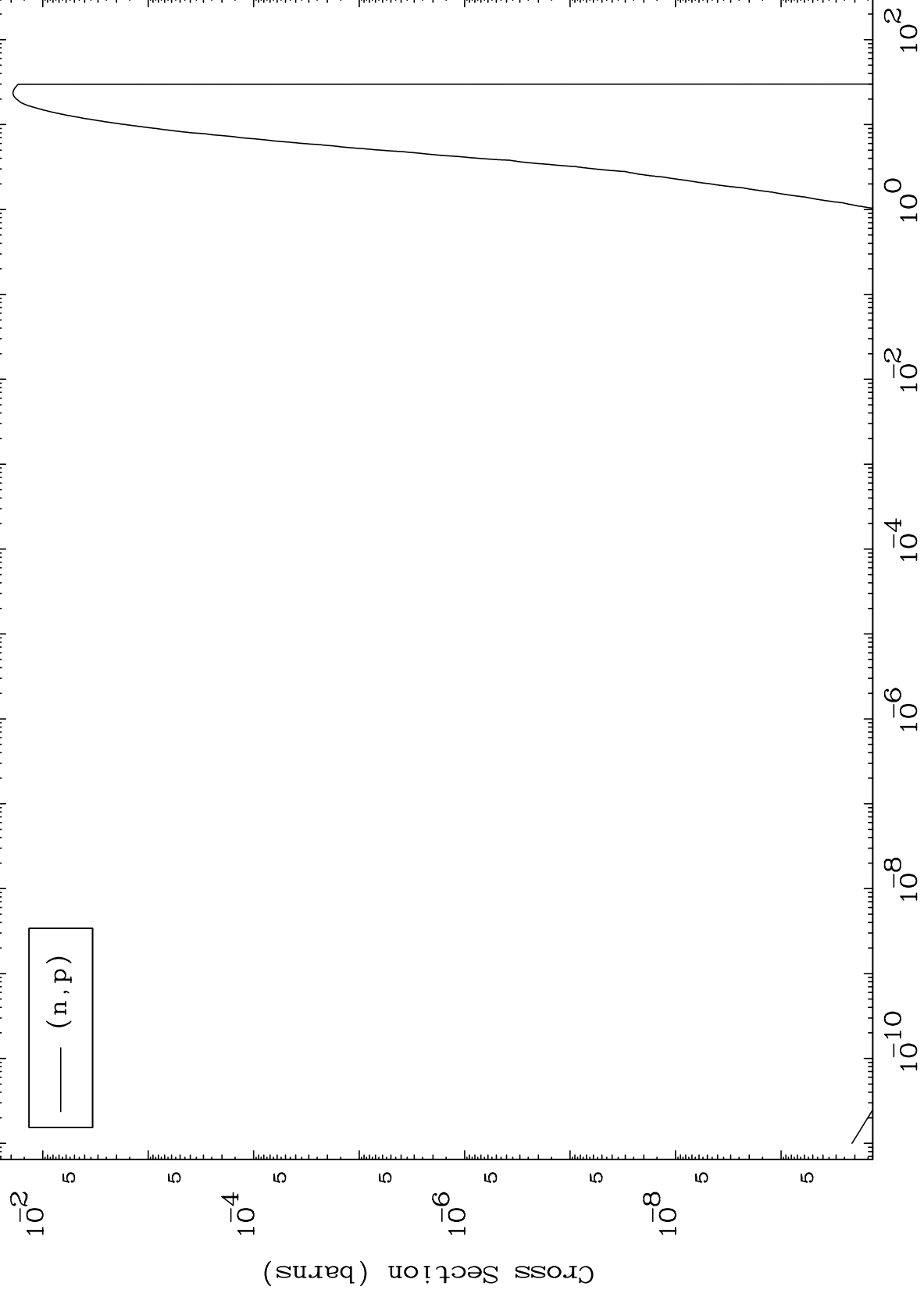
10



MAT 89222

(n,p) Levels
293 Kelvin Cross Sections

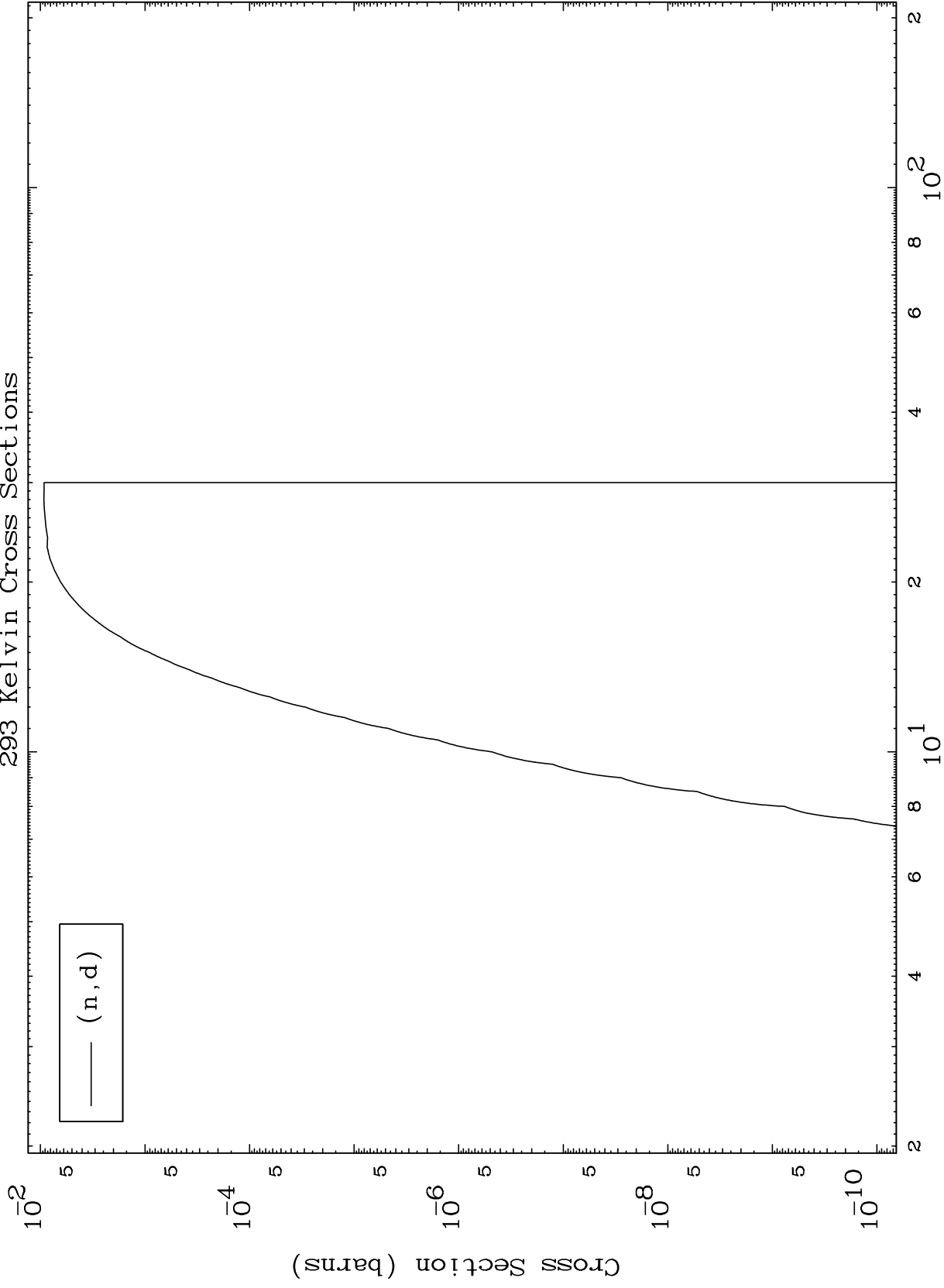
89-Ac-224



MAT 8922

(n,d) Levels
293 Kelvin Cross Sections

89-Ac-224



89-Ac-224

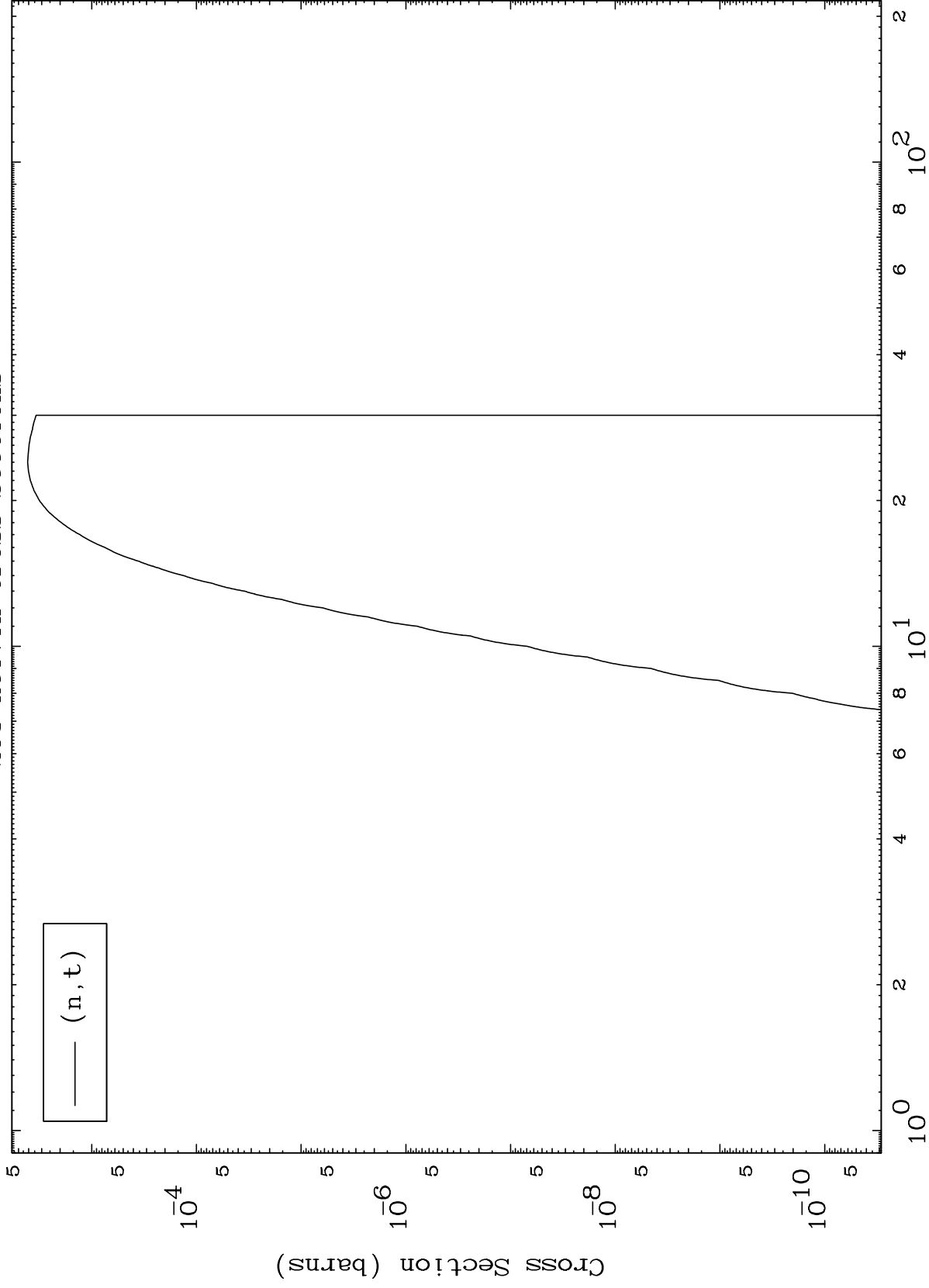
Incident Energy (MeV)

13

MAT 8922

(n,t) Levels
293 Kelvin Cross Sections

89-Ac-224



Incident Energy (MeV)

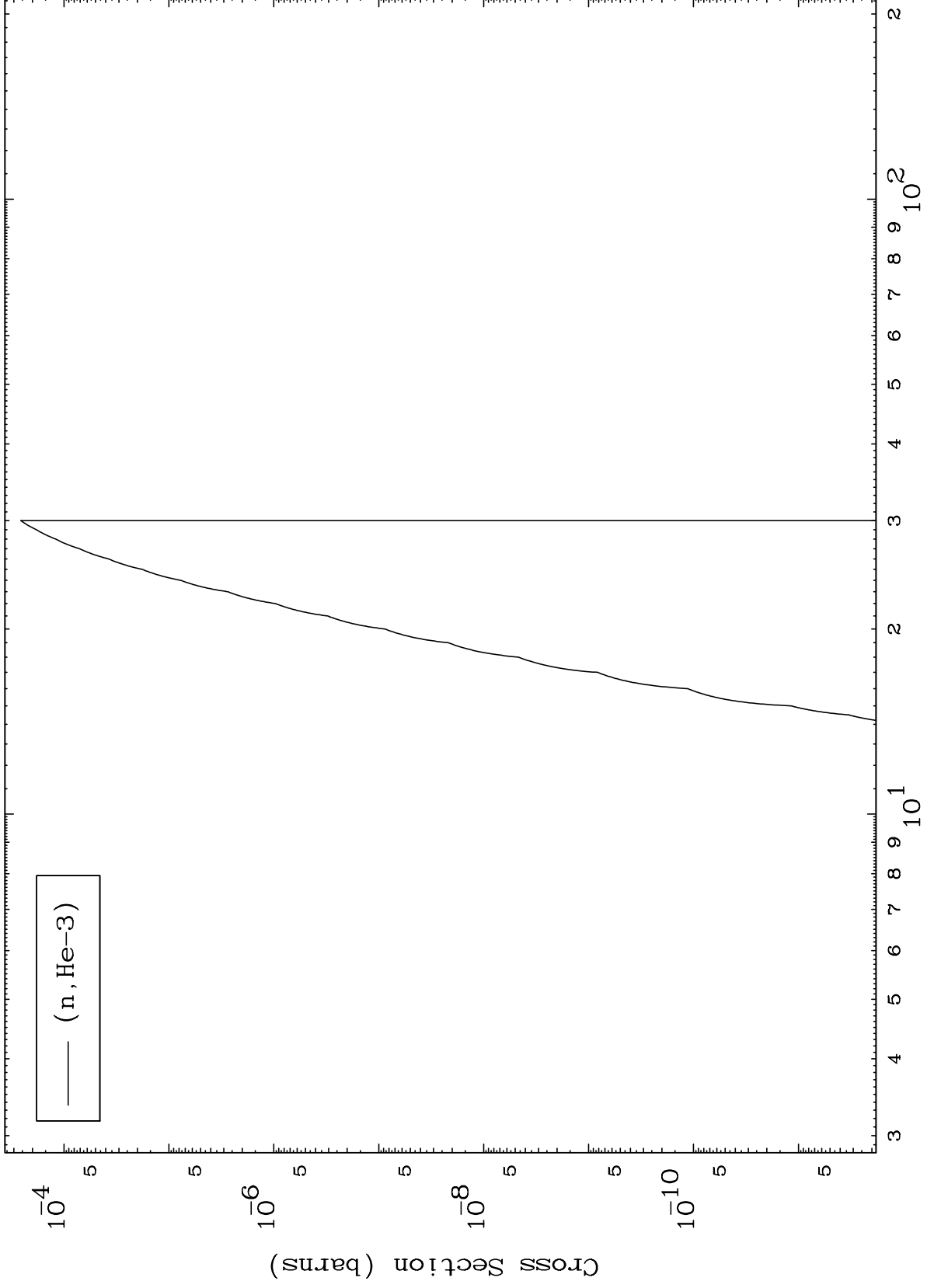
89-Ac-224

14

MAT 8922

(n,He3) Levels
293 Kelvin Cross Sections

89-Ac-224



15

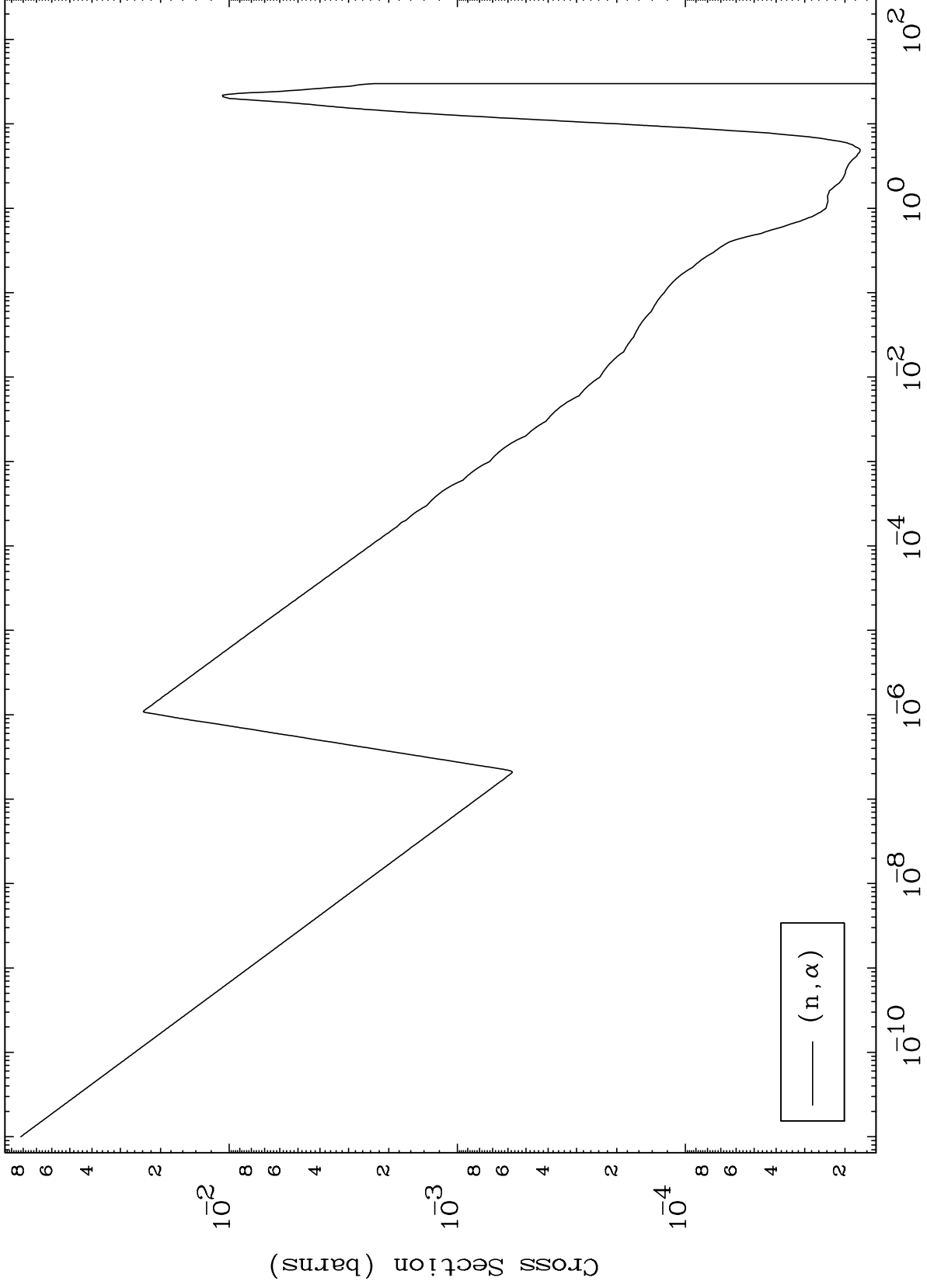
Incident Energy (MeV)

89-Ac-224

MAT 89222

(n, α) Levels
293 Kelvin Cross Sections

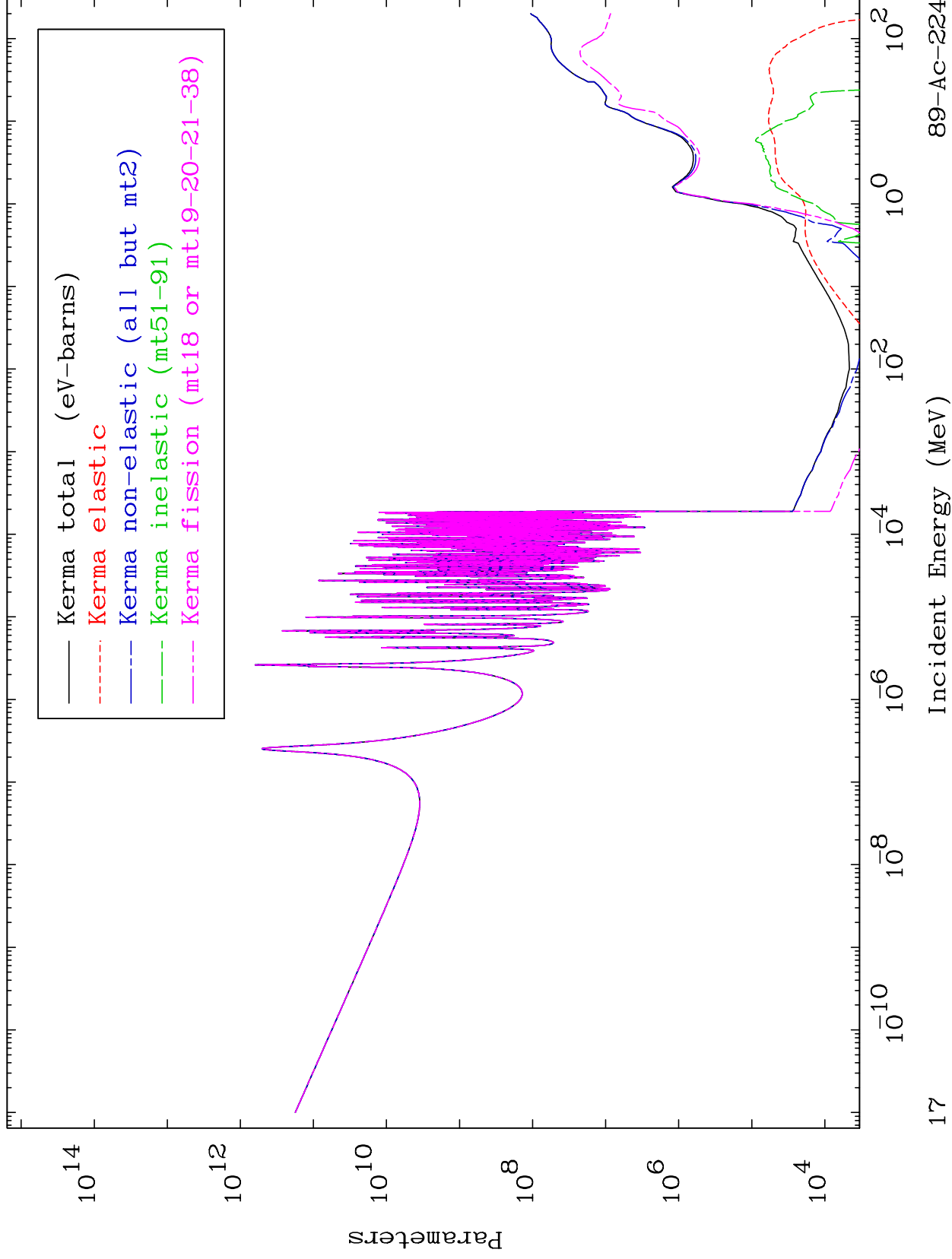
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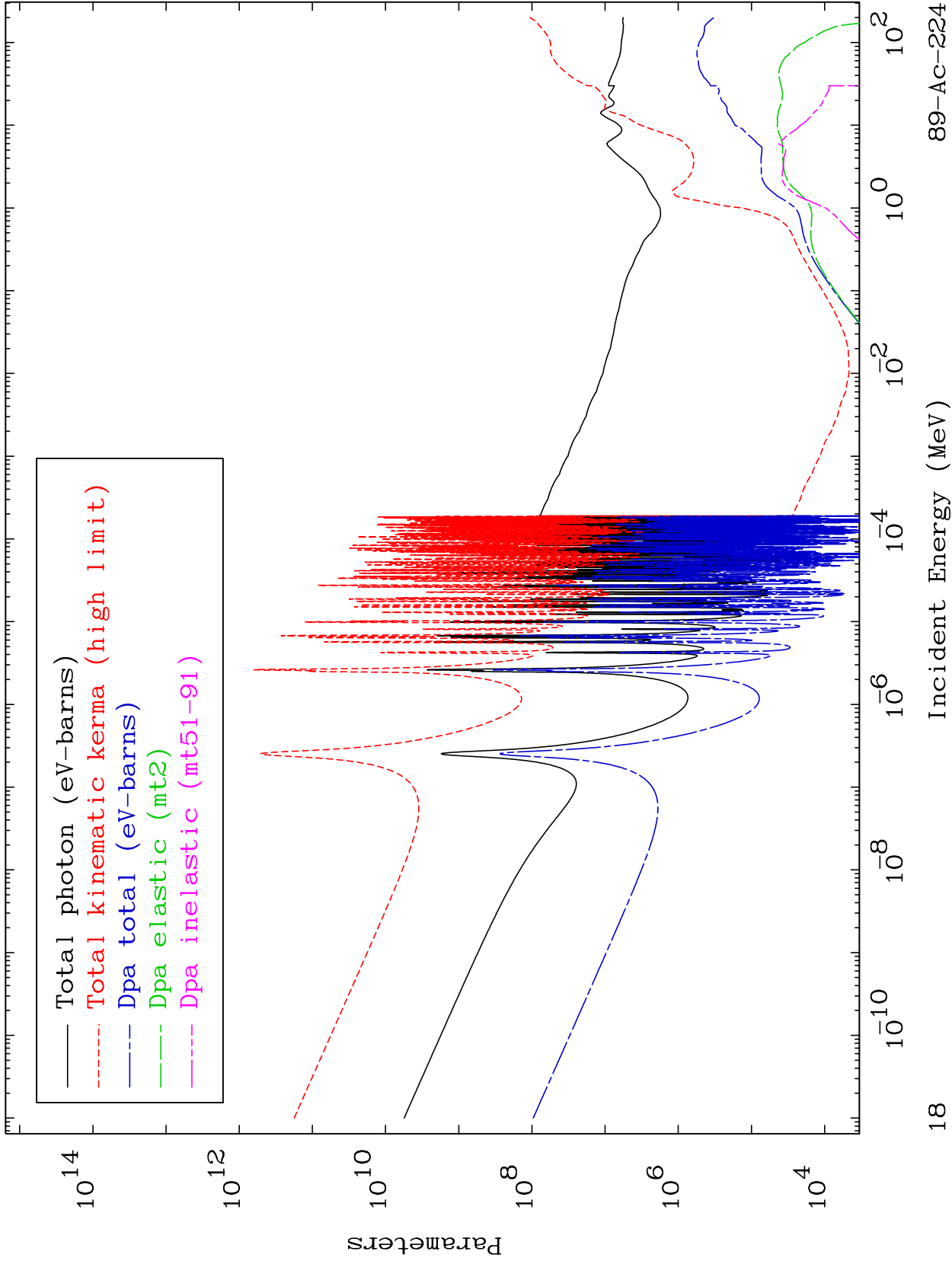


16

Incident Energy (MeV)

89-Ac-224

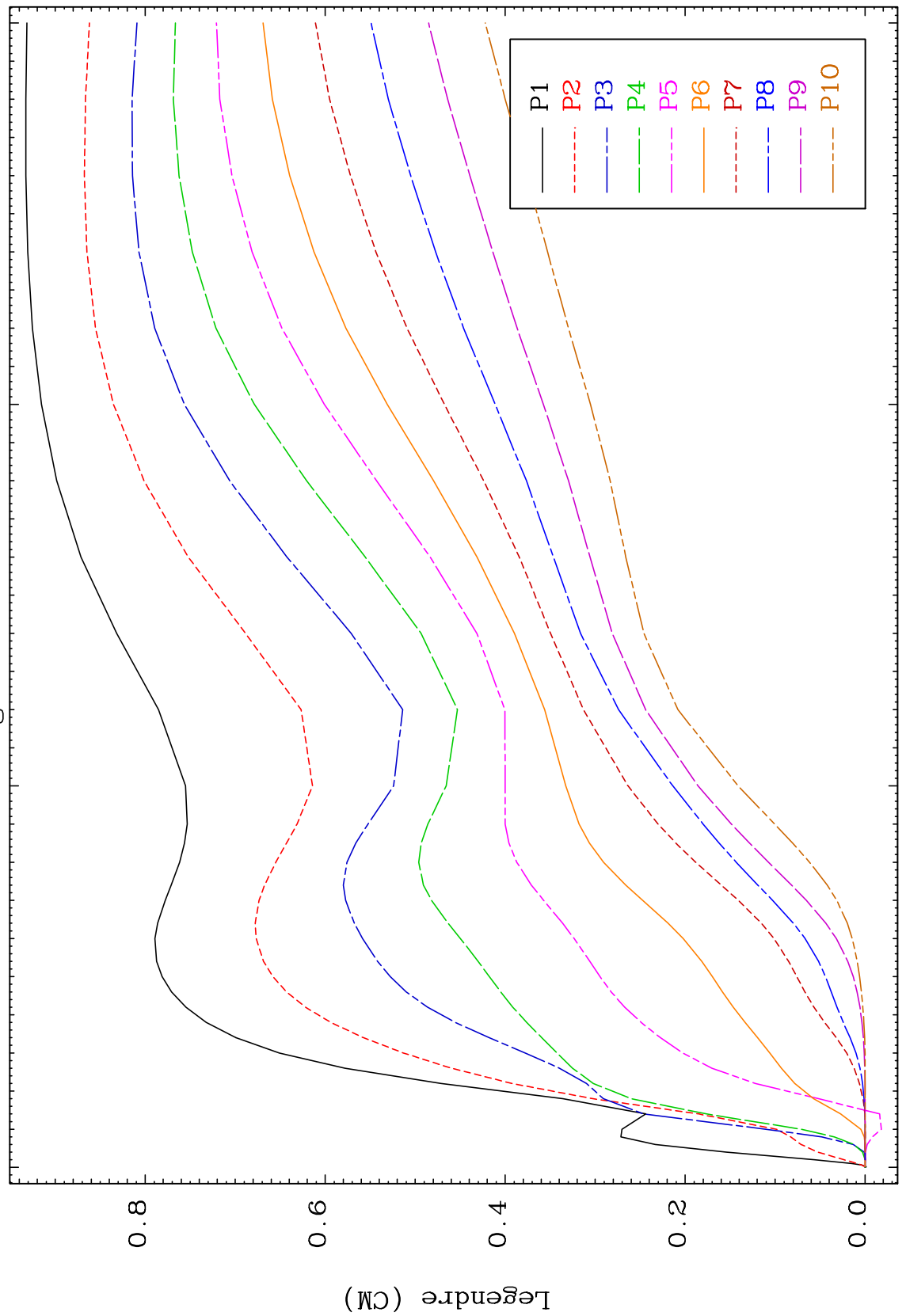




MAT 8922

89-Ac-224

Elastic Legendre Coefficients



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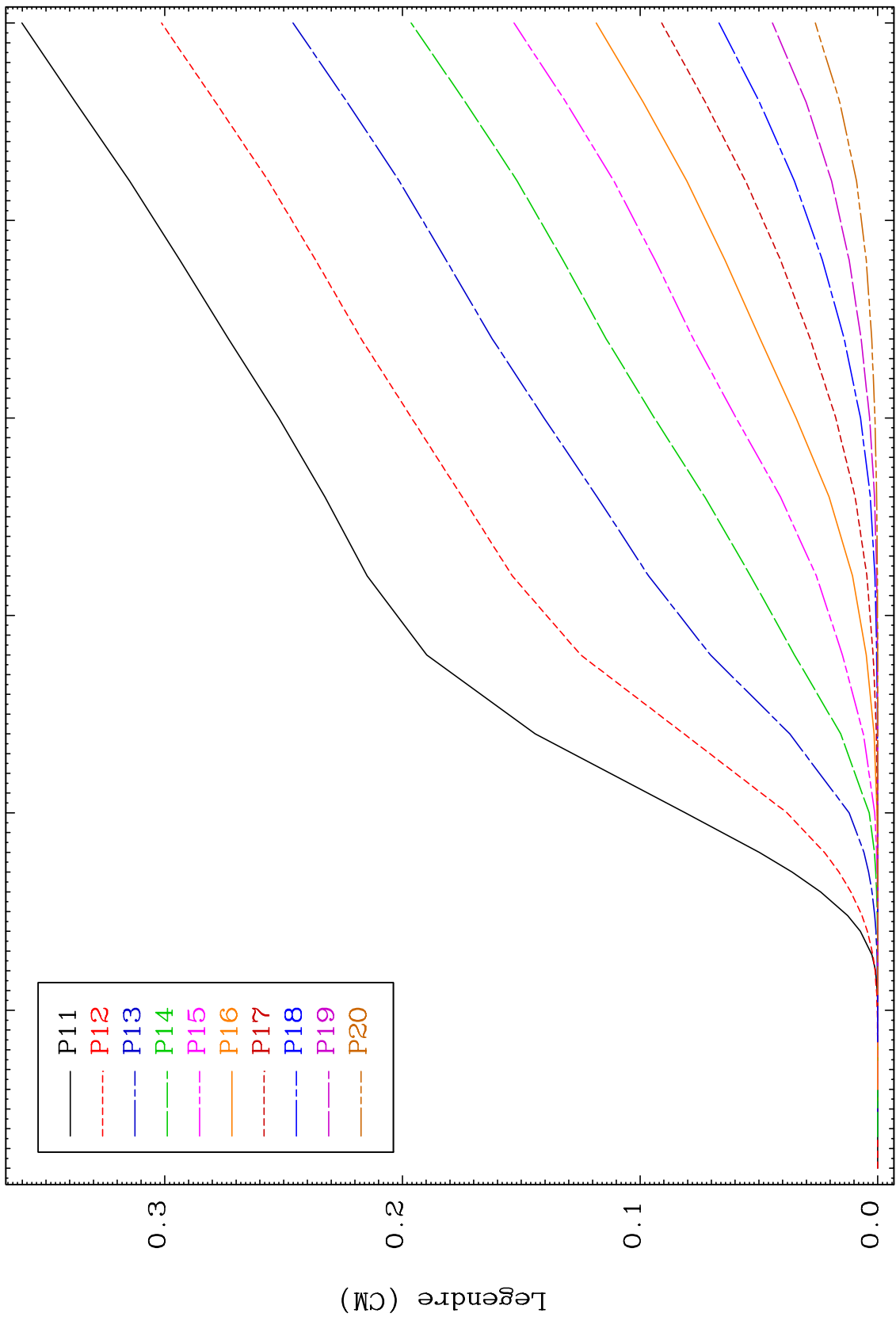
Incident Energy (MeV)

89-Ac-224

MAT 8922

Elastic Legendre Coefficients

89-Ac-224



20

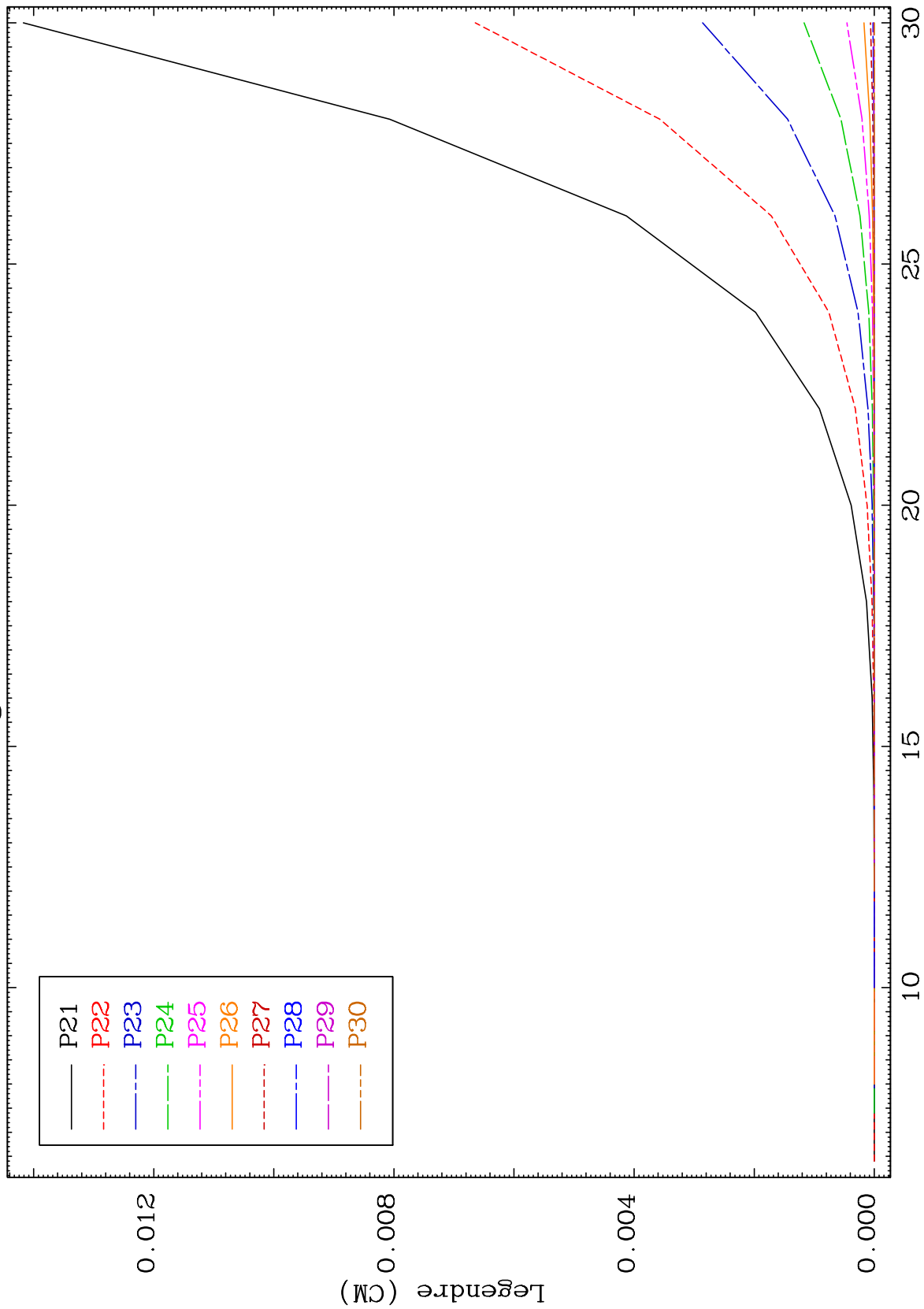
Incident Energy (MeV)

89-Ac-224

MAT 8922

Elastic Legendre Coefficients

89-Ac-224



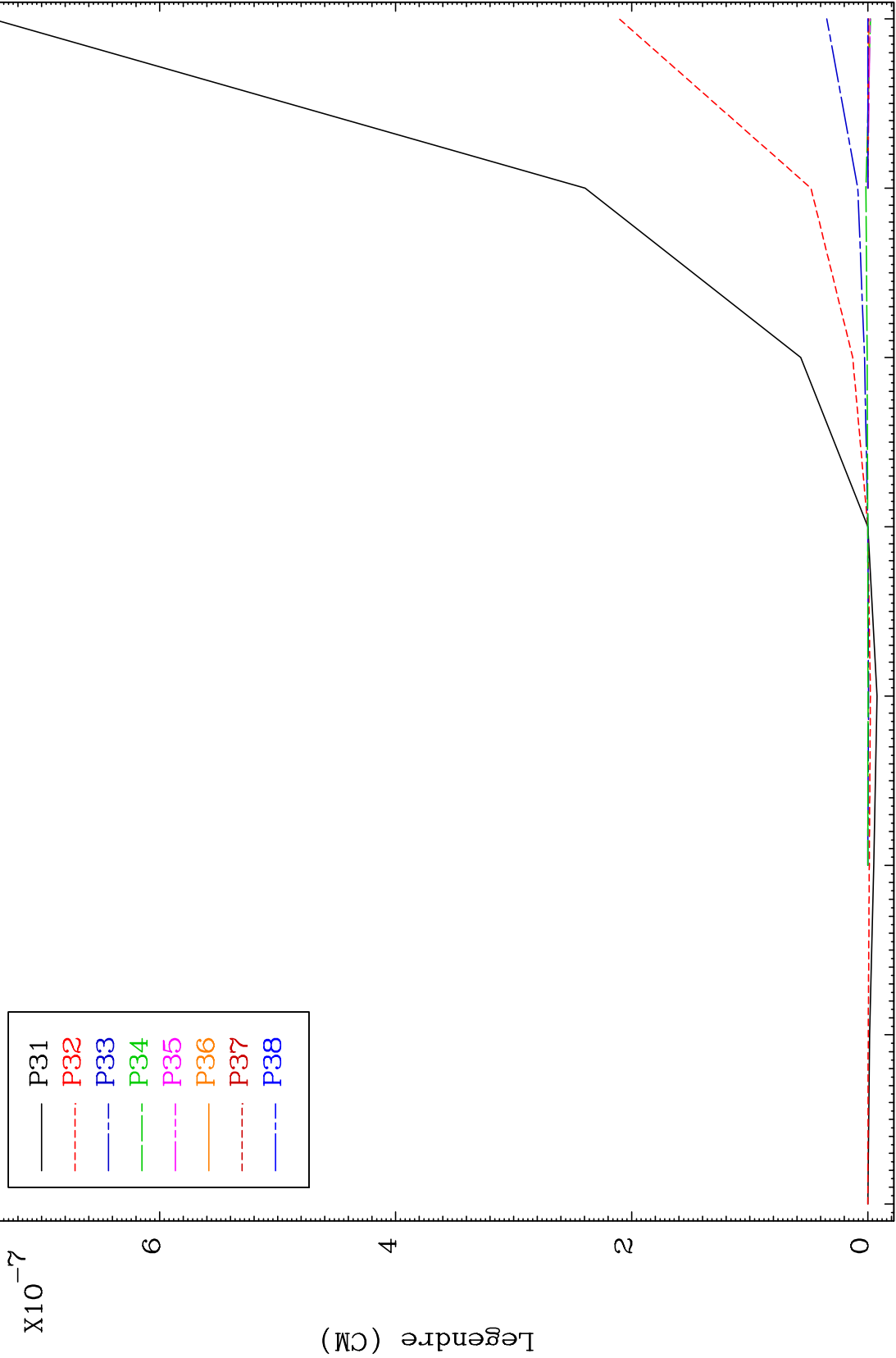
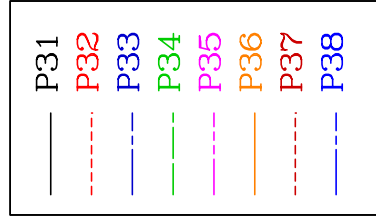
21

89-Ac-224

MAT 8922

Elastic Legendre Coefficients

89-Ac-224

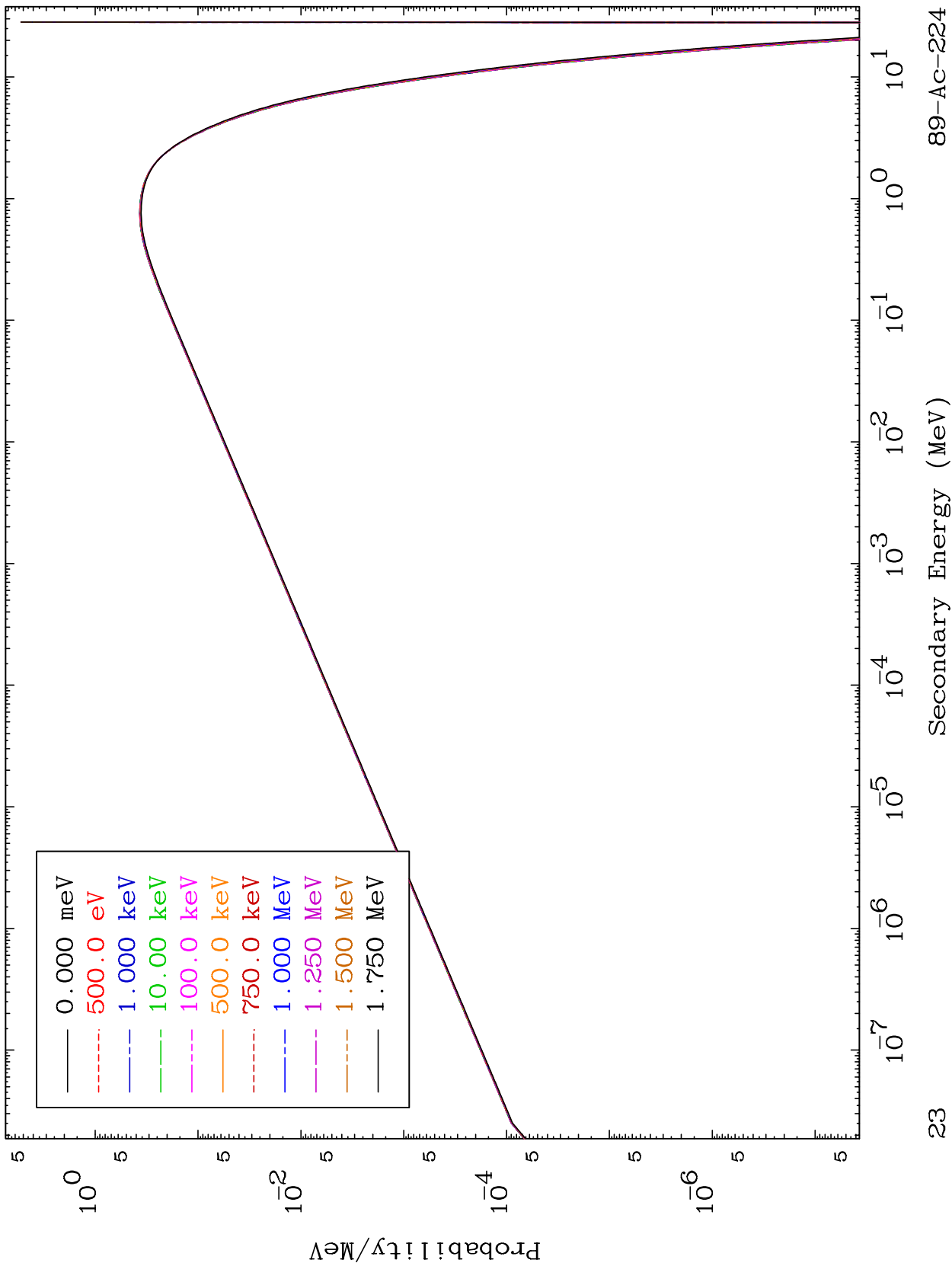


22

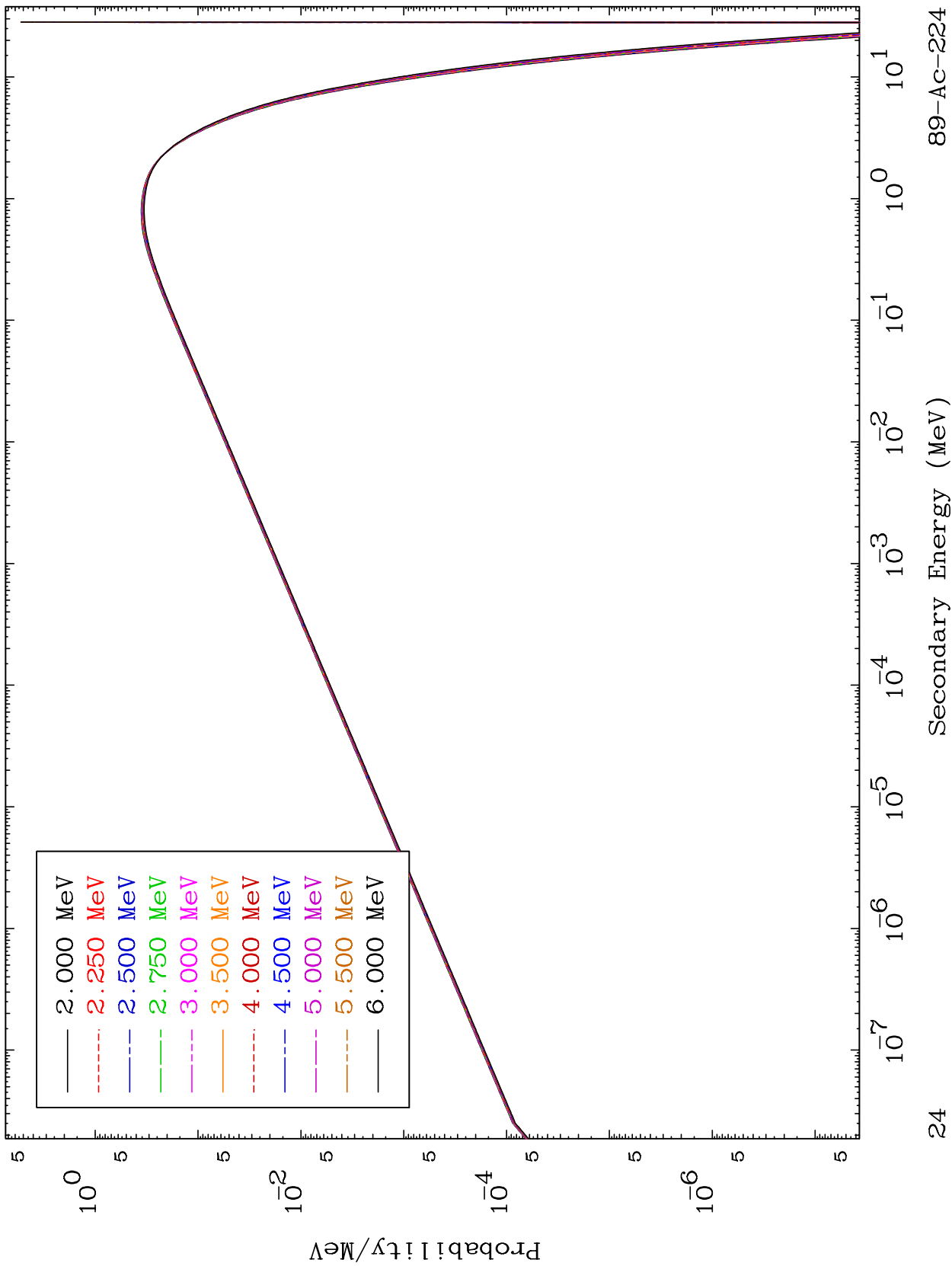
Incident Energy (MeV)

89-Ac-224

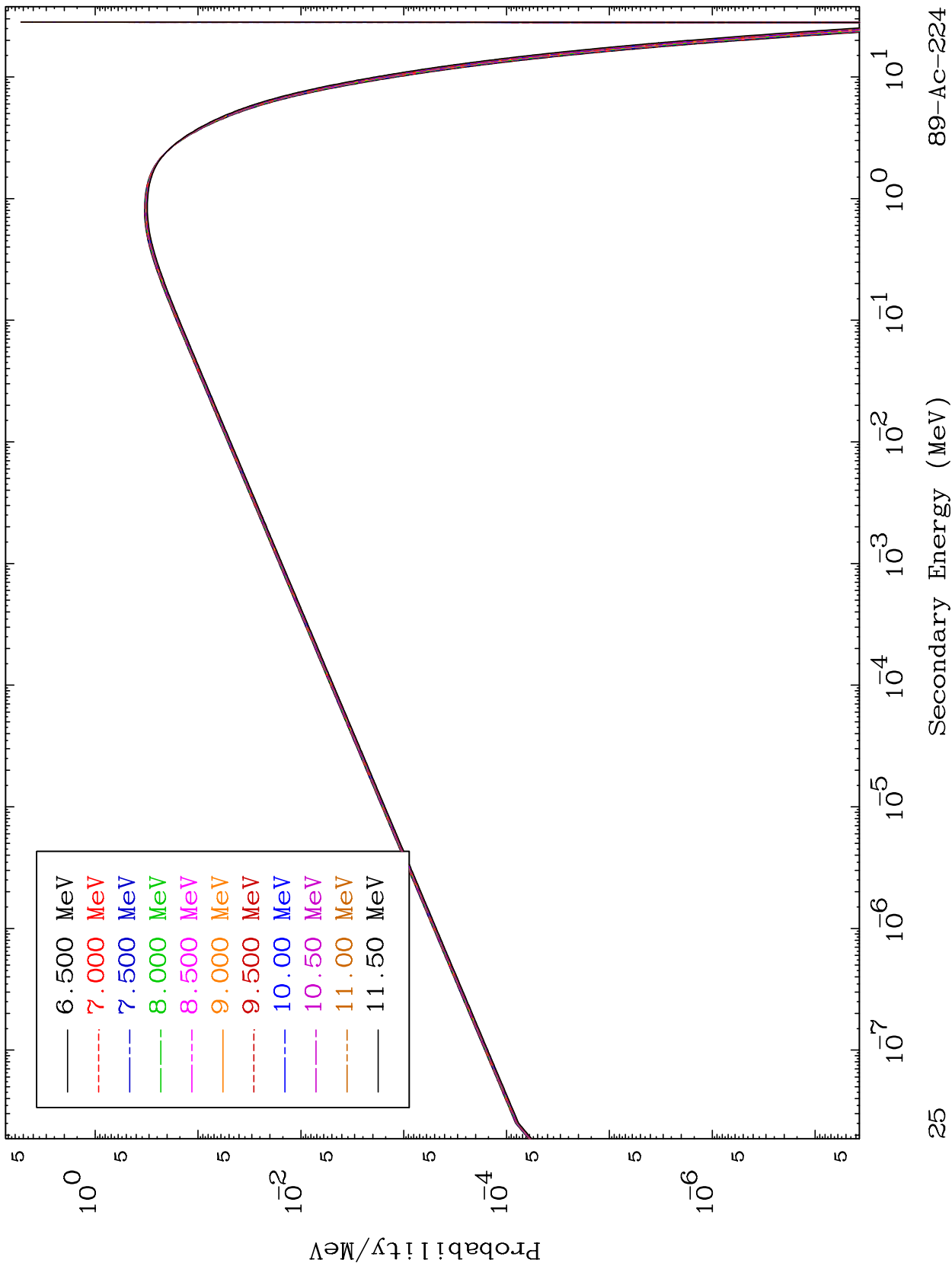
Fission Energy Distribution



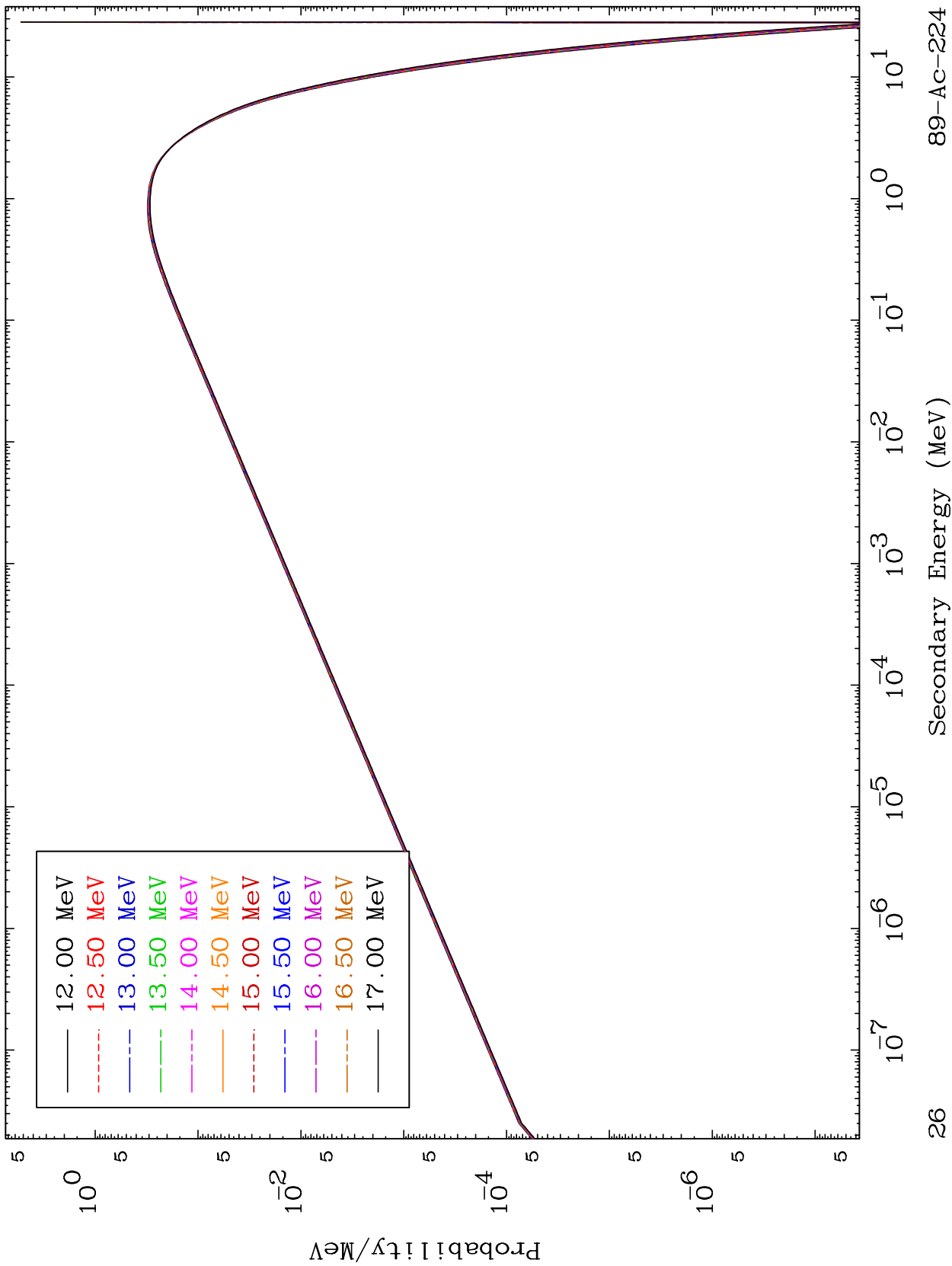
Fission Energy Distribution



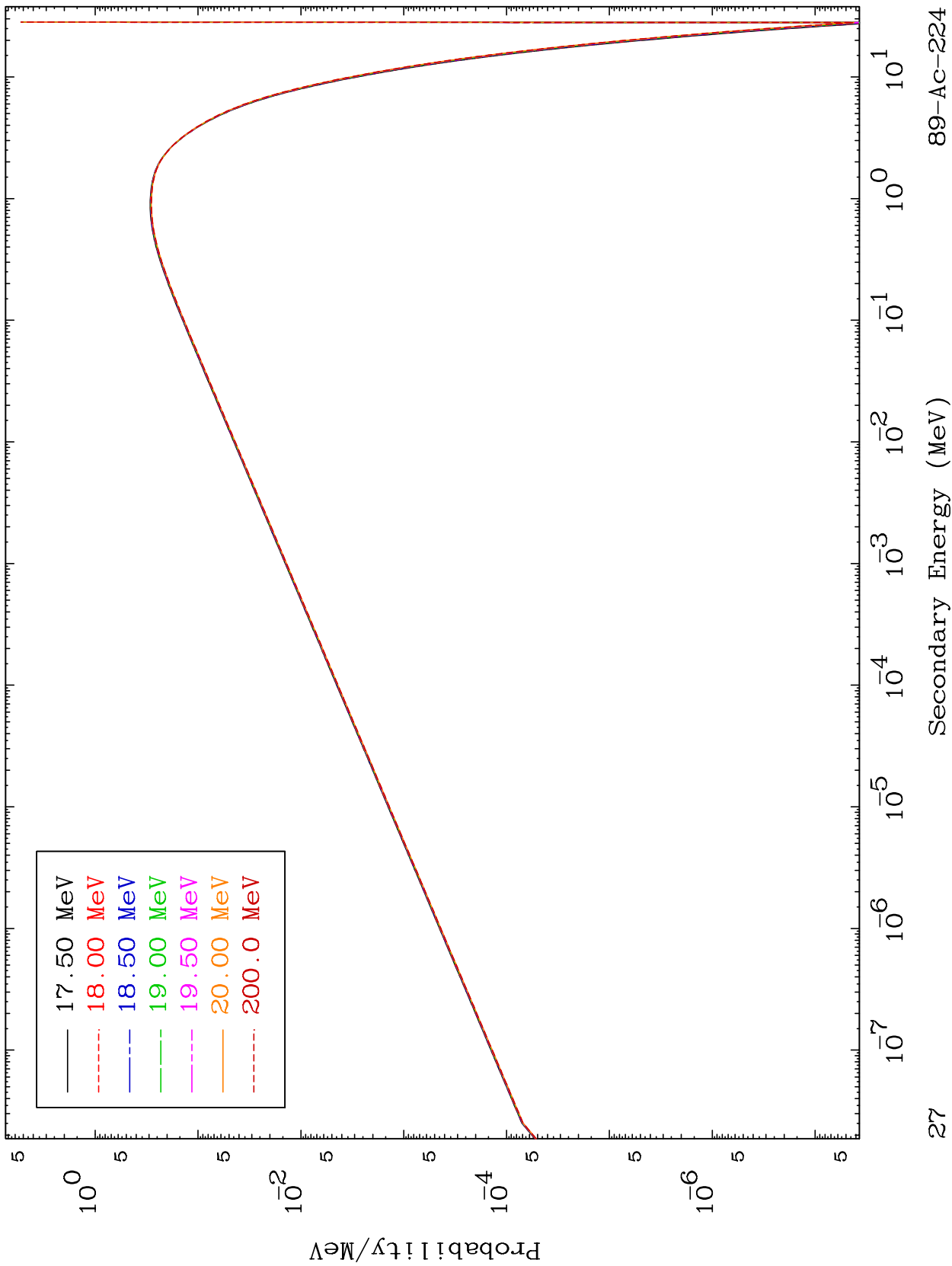
Fission Energy Distribution



Fission Energy Distribution



Fission
Energy Distribution

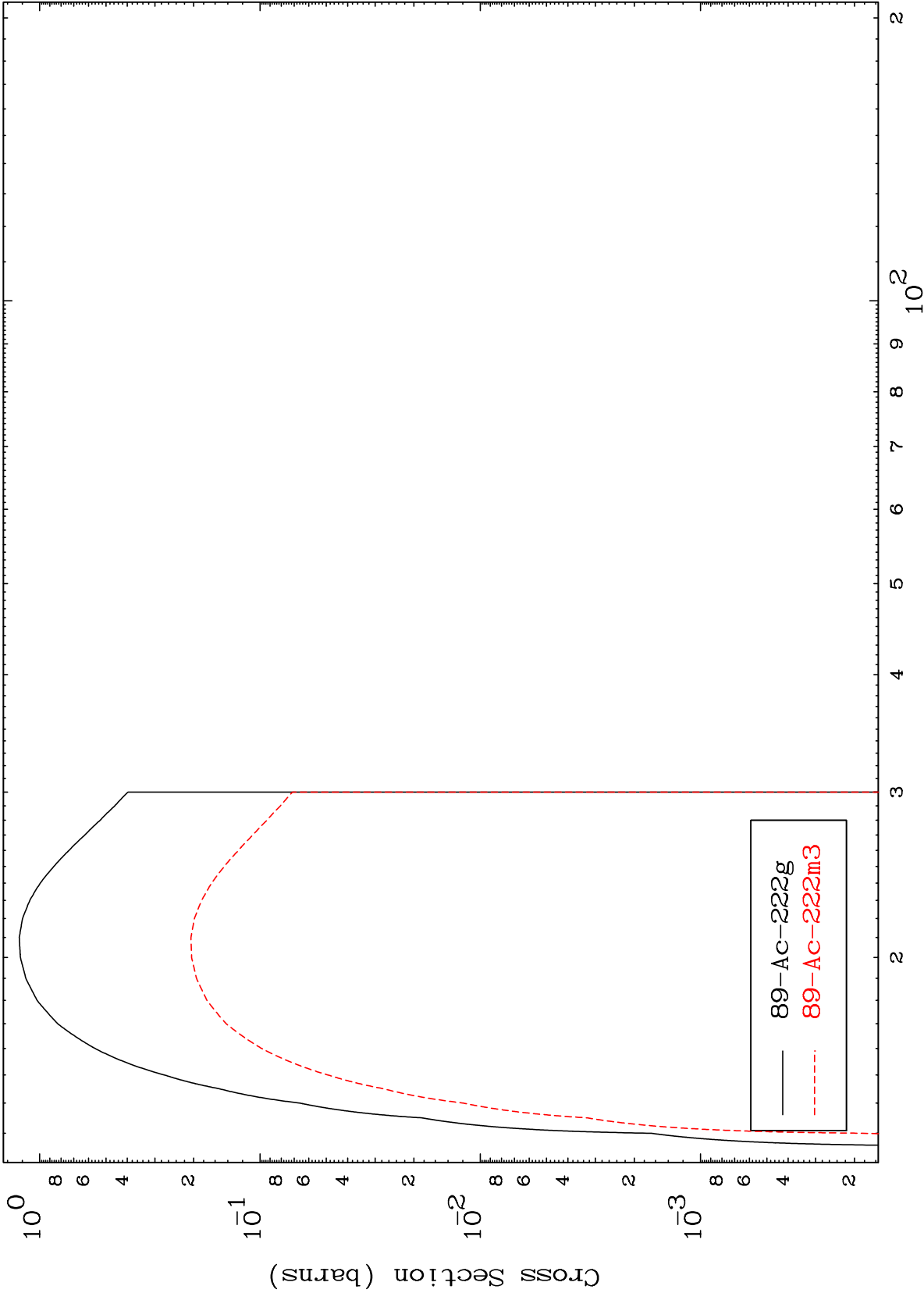


MAT 8922

(n,3n)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

89-Ac-224

MAT 8922

(n,n') 3 α

89-Ac-224

Radionuclide Production Cross Section

