

Program EVALPLOT
(Version 2015-2)

by

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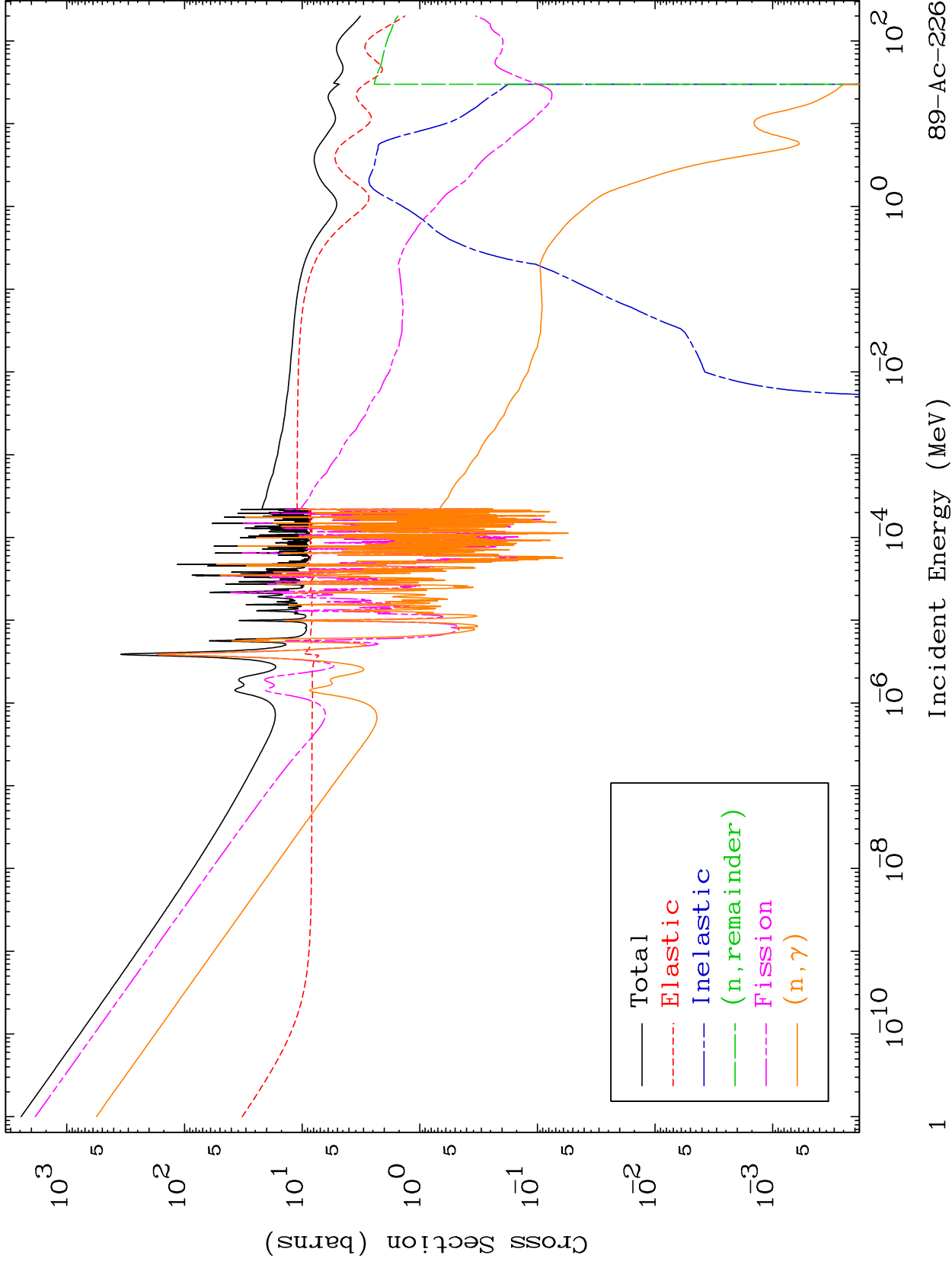
E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

MAT 8928

Major
293 Kelvin Cross Sections

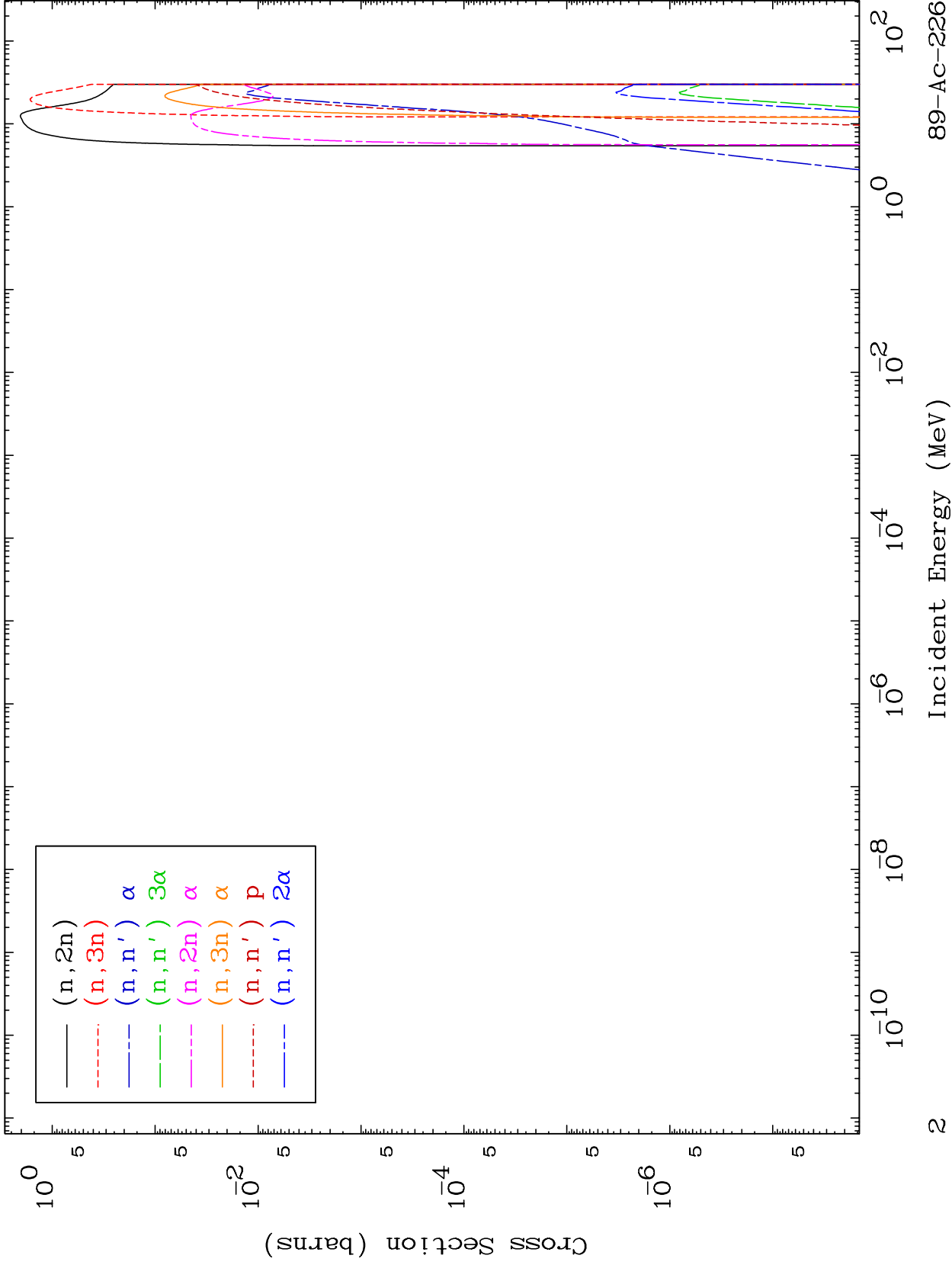
89-Ac-226



MAT 8928

Neutron Production
293 Kelvin Cross Sections

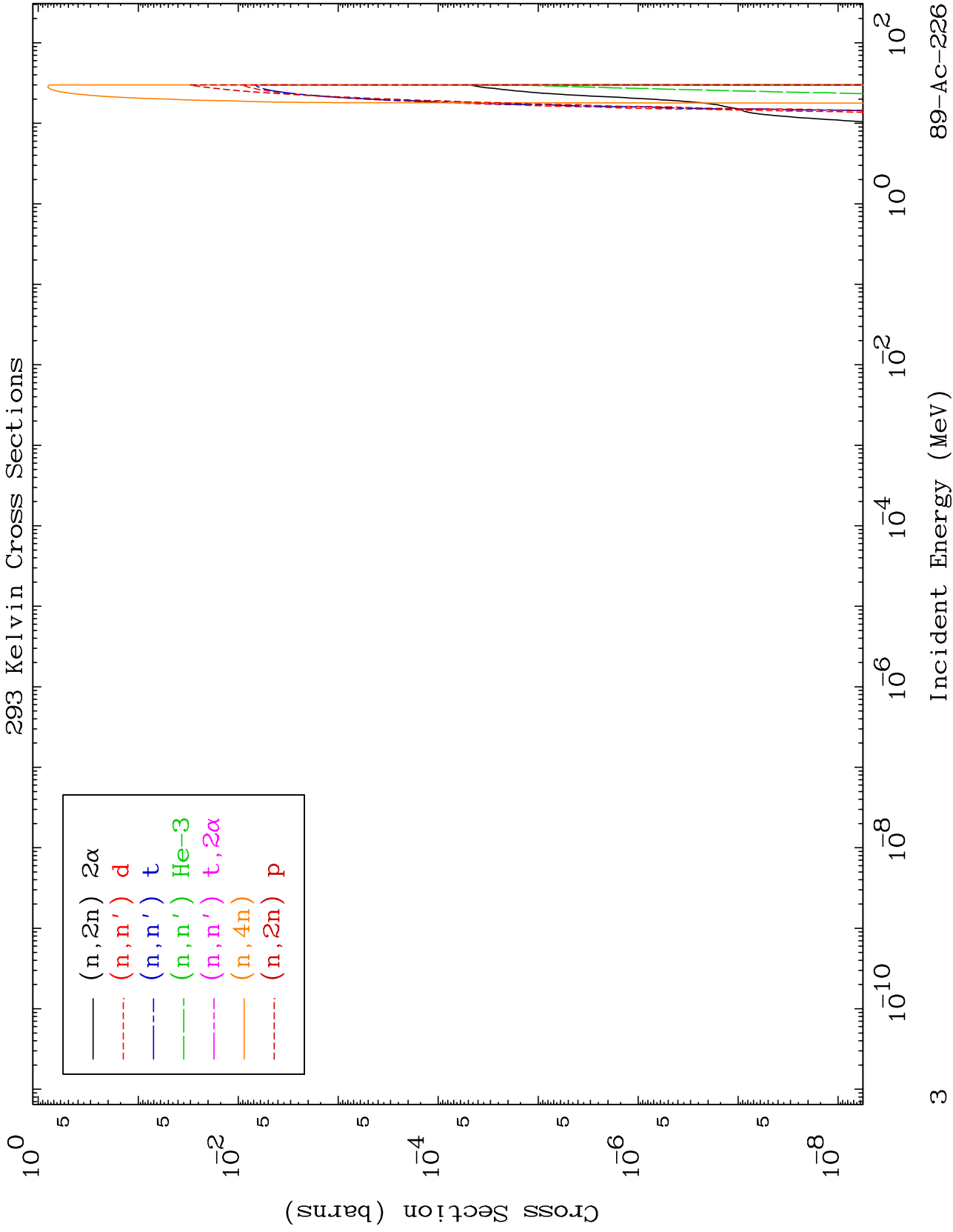
89-Ac-226



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Neutron Production
293 Kelvin Cross Sections

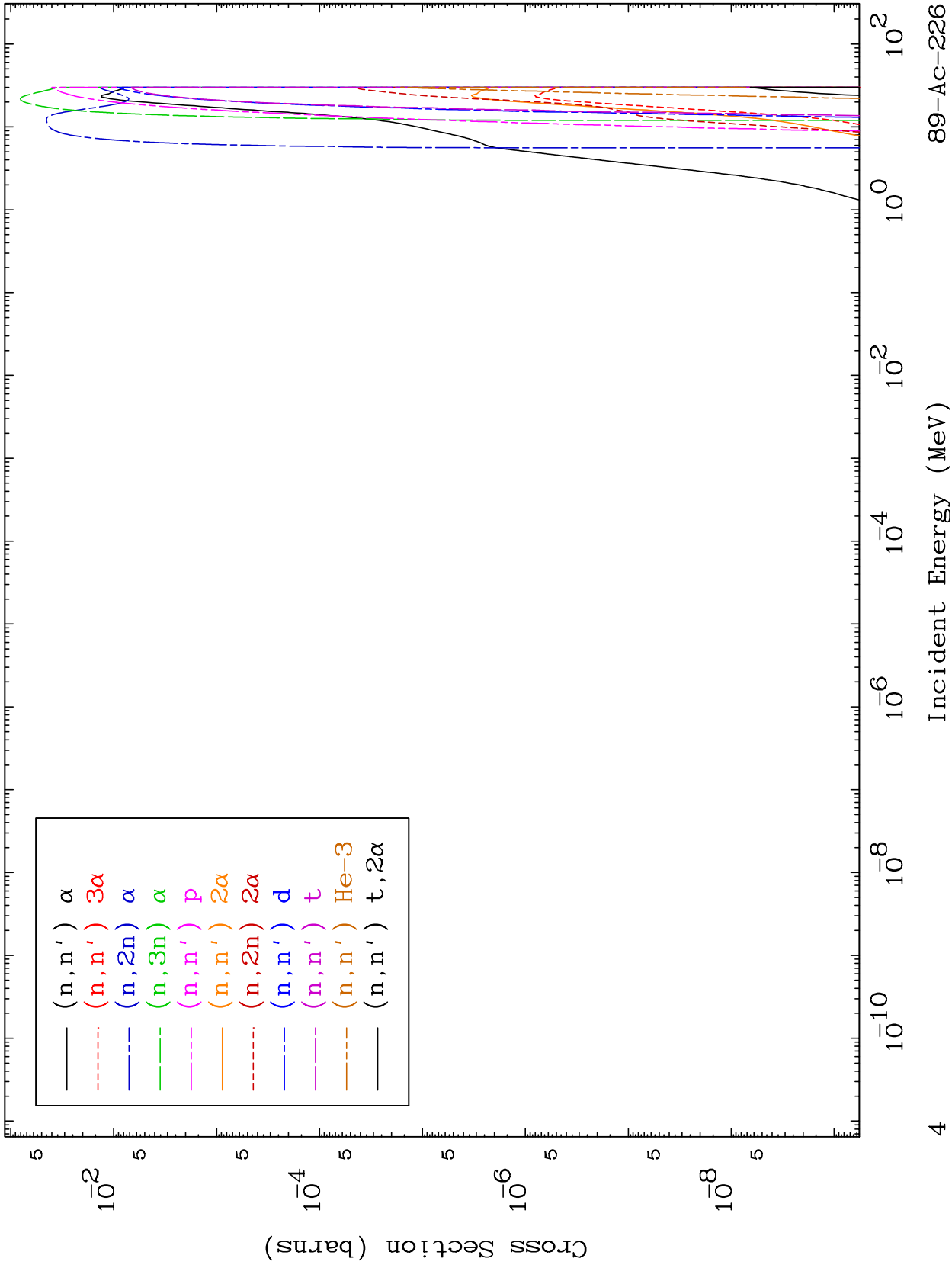
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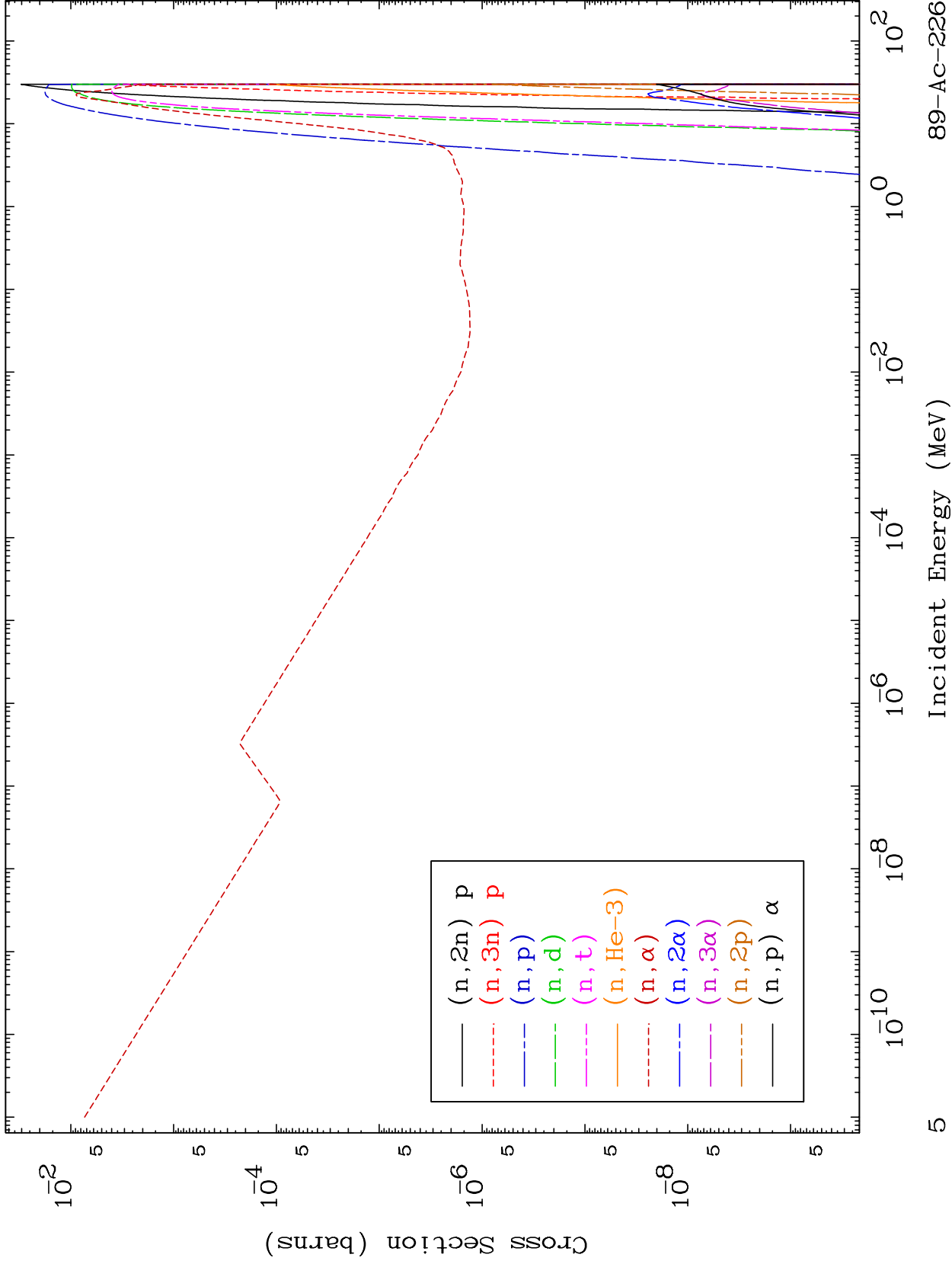


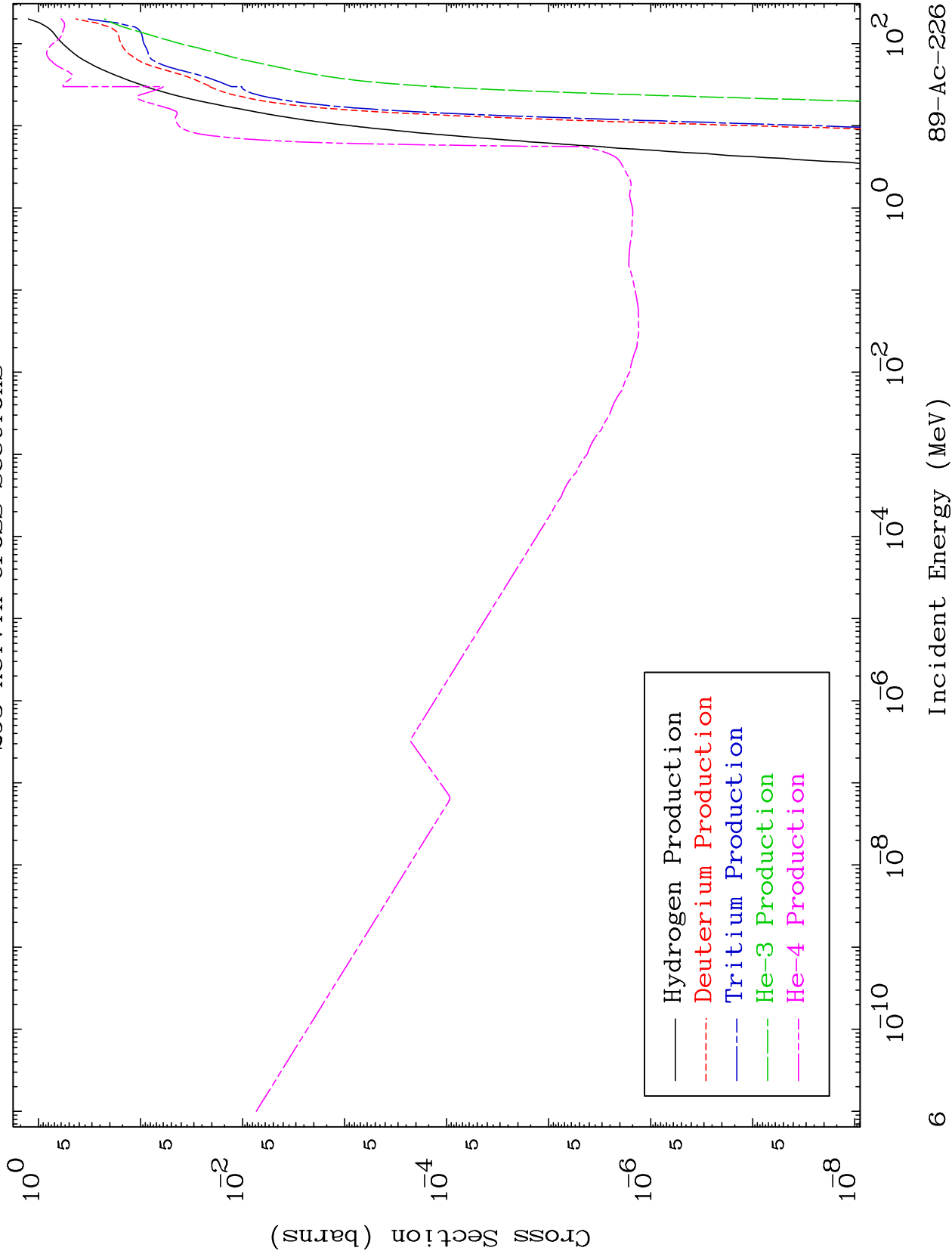
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Charged Particle
293 Kelvin Cross Sections

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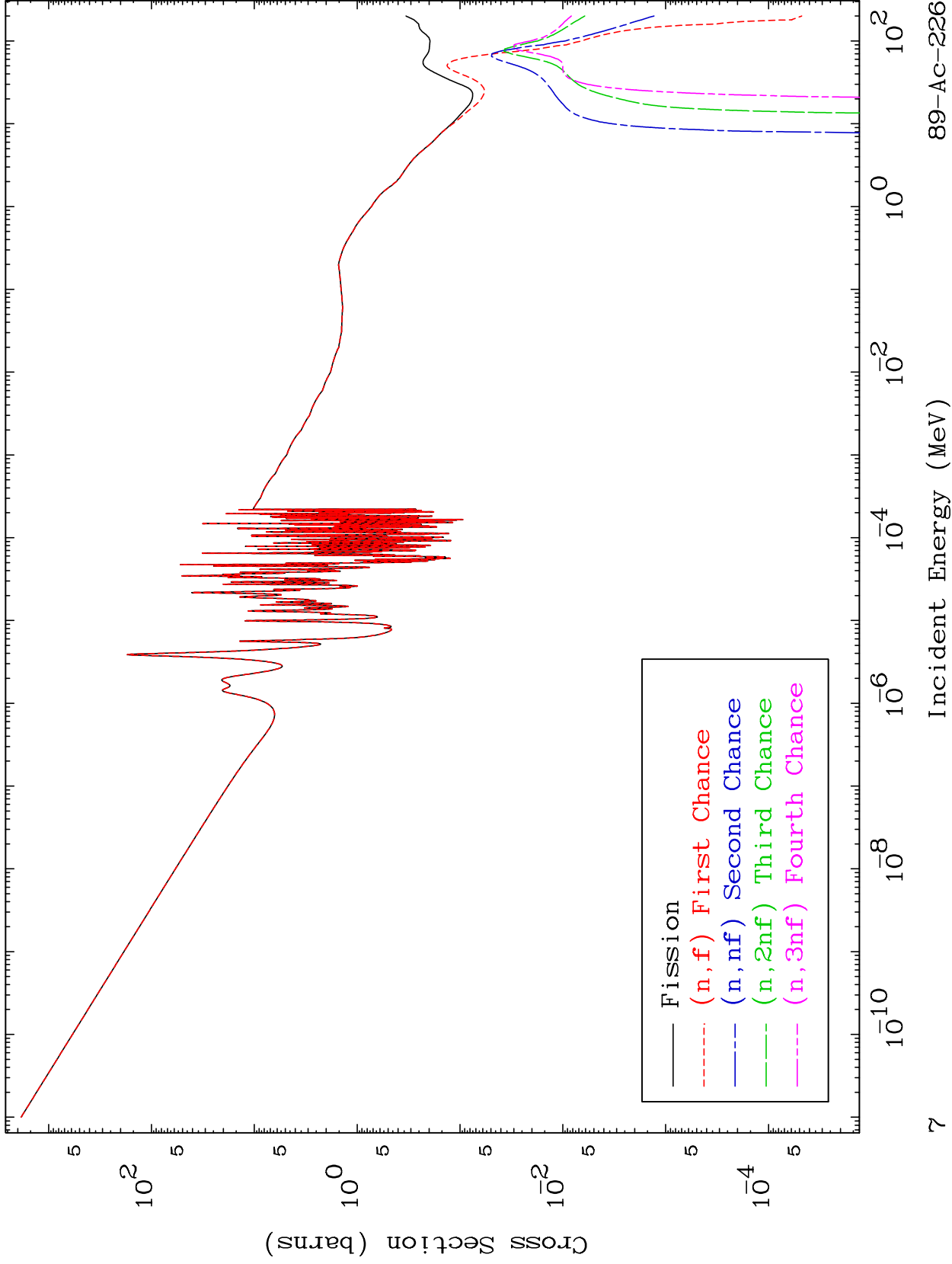


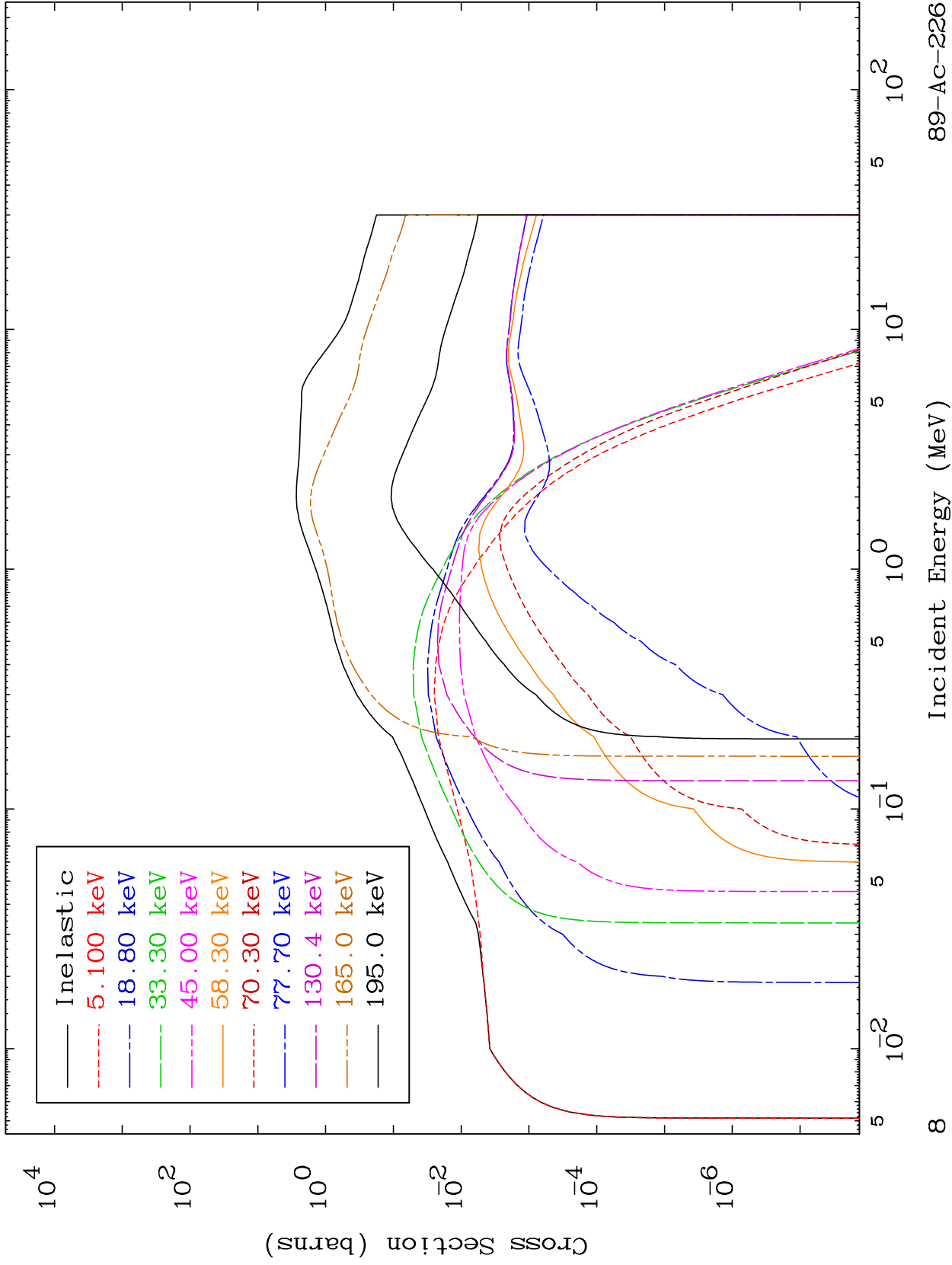


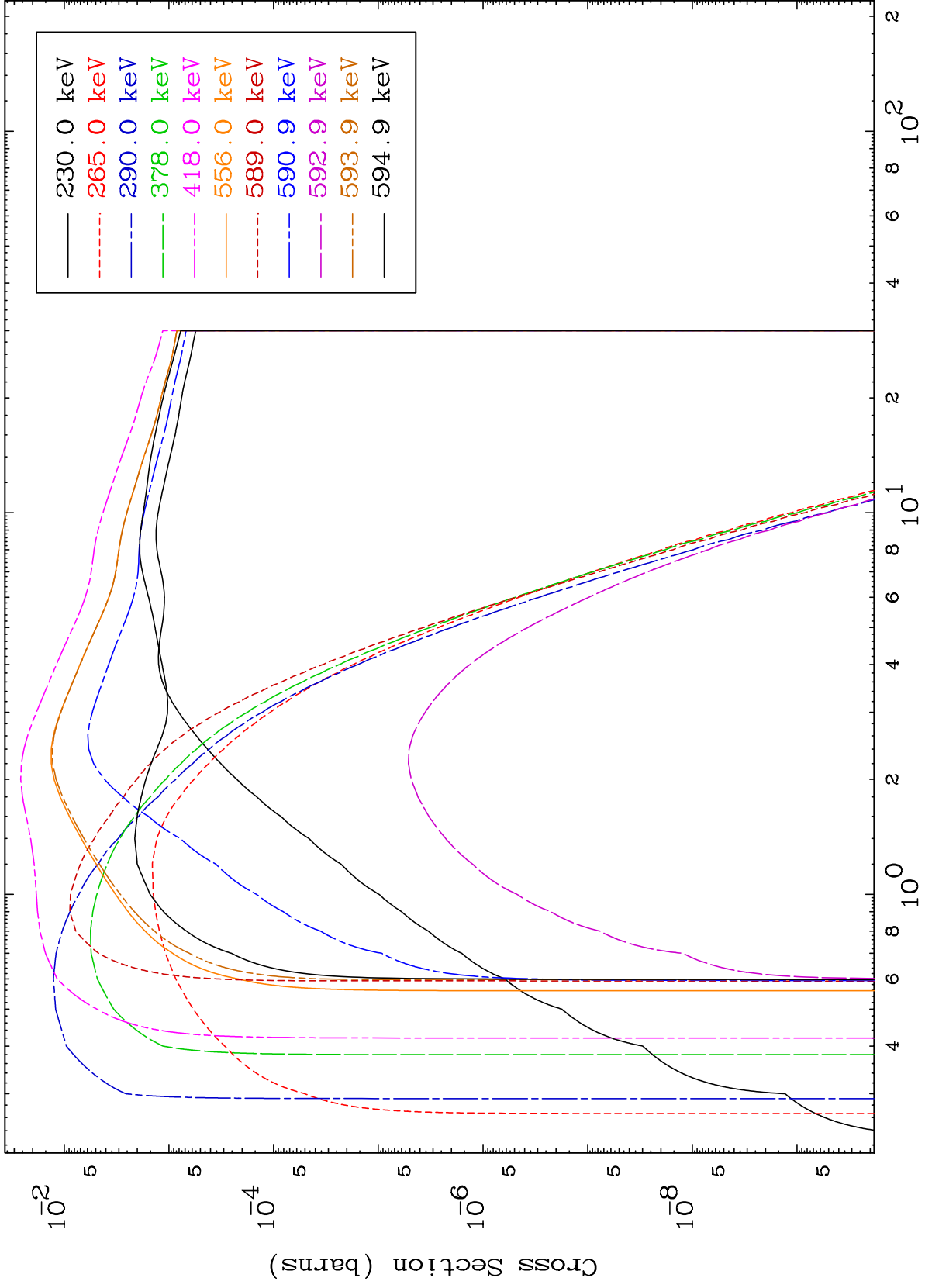
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Fission
293 Kelvin Cross Sections

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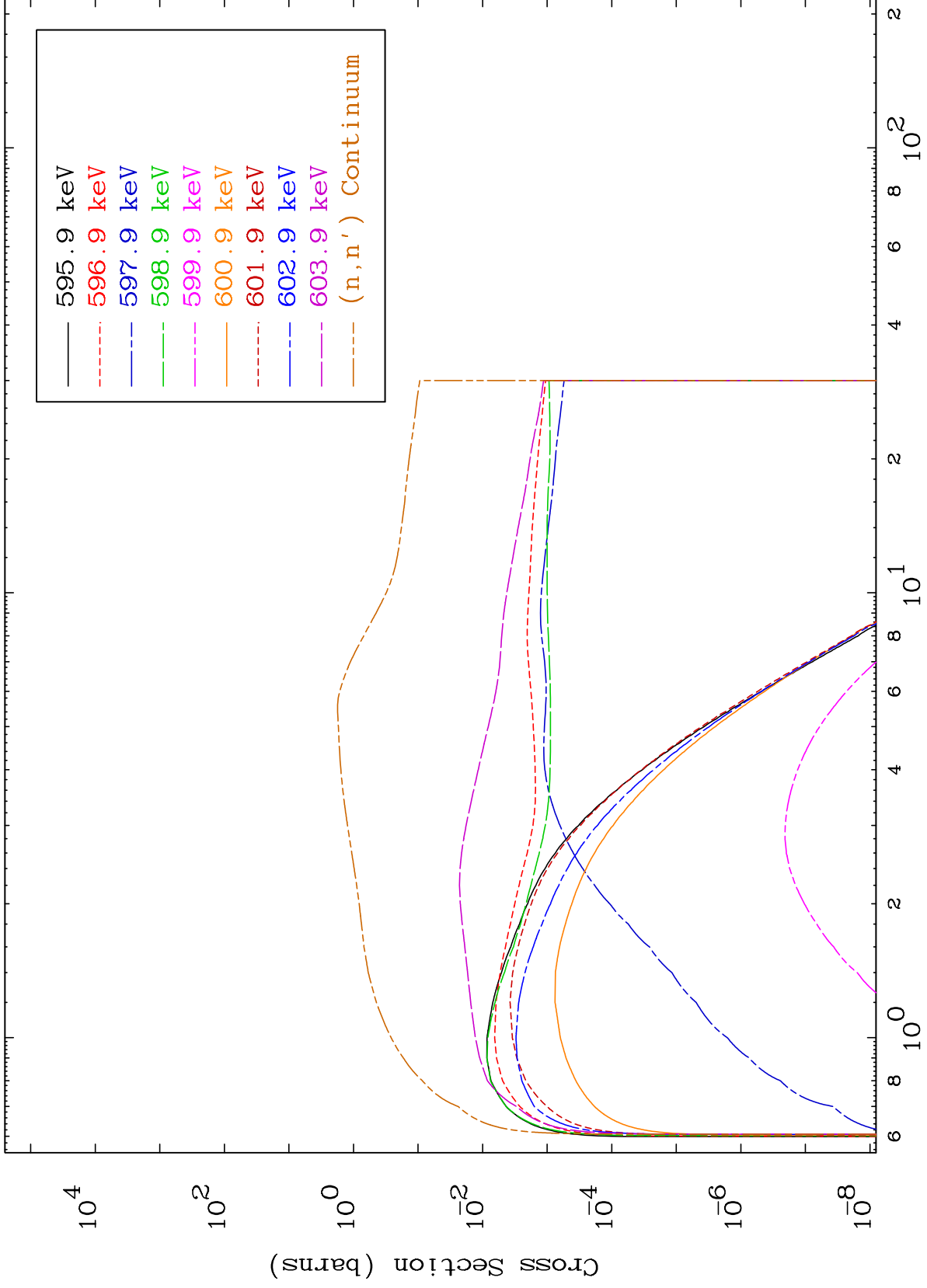




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(n,n') Level
293 Kelvin Cross Sections

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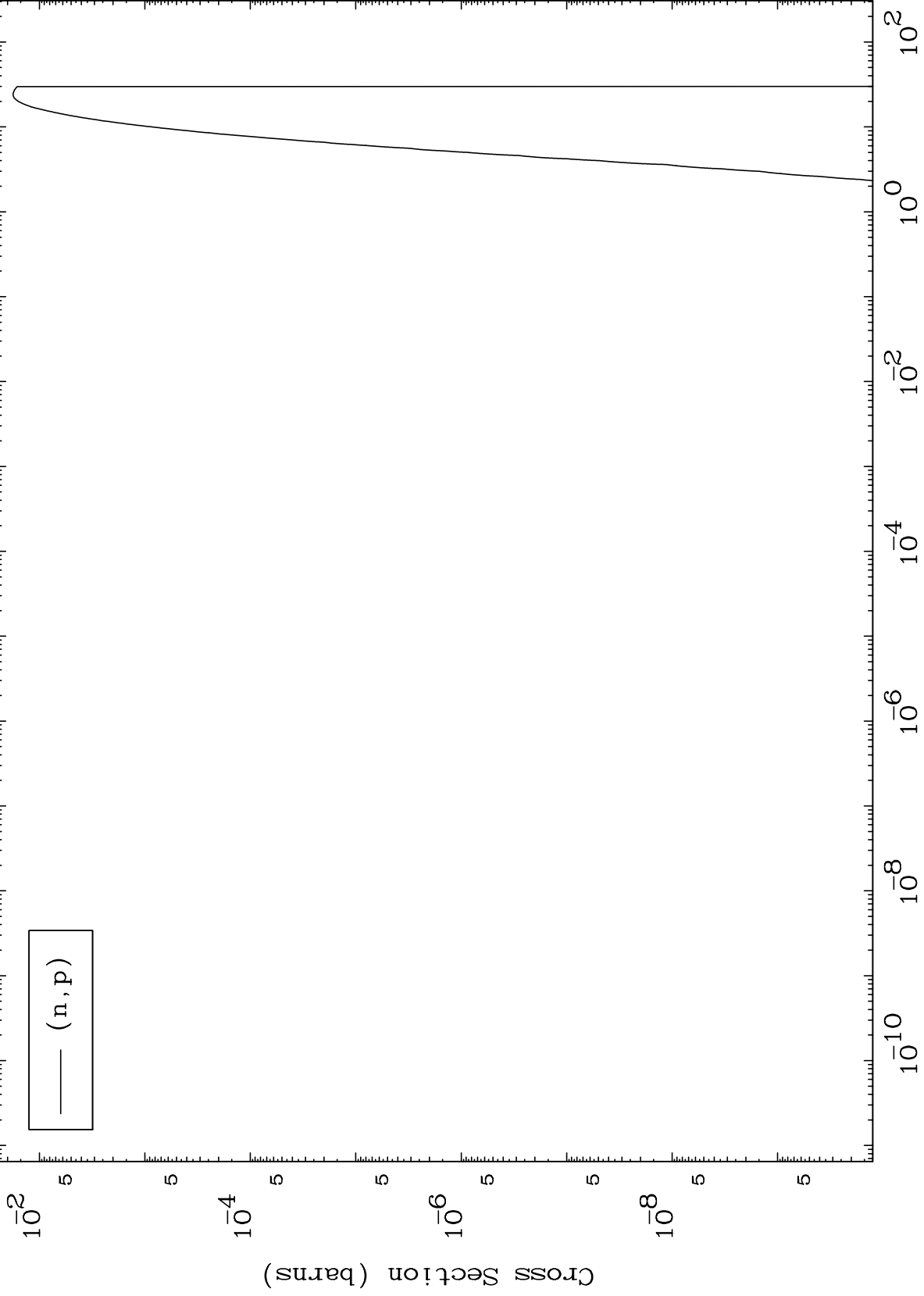
Incident Energy (MeV)

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(n,p) Levels
293 Kelvin Cross Sections

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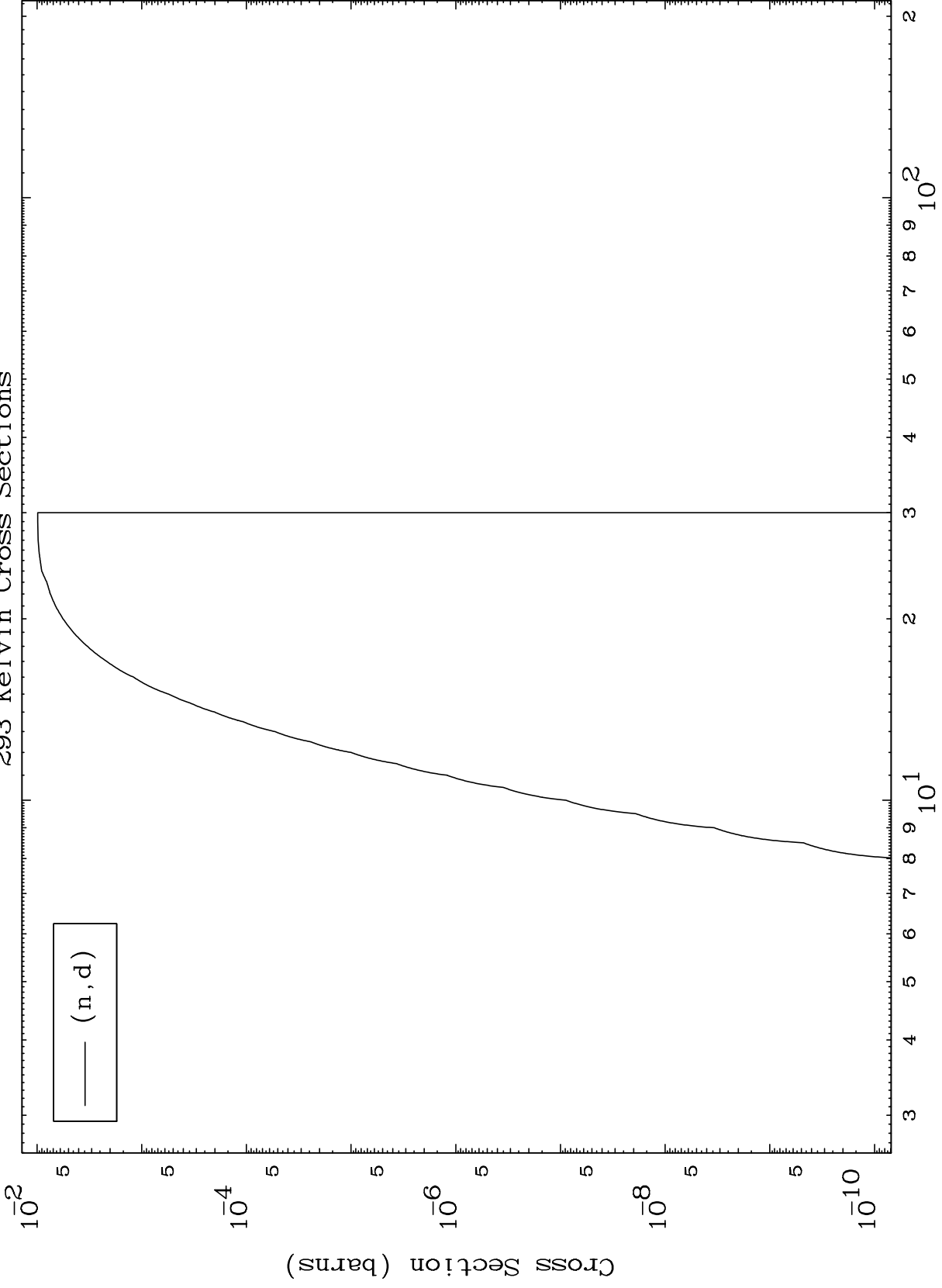
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Incident Energy (MeV)

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(n,d) Levels
293 Kelvin Cross Sections

89-Ac-226



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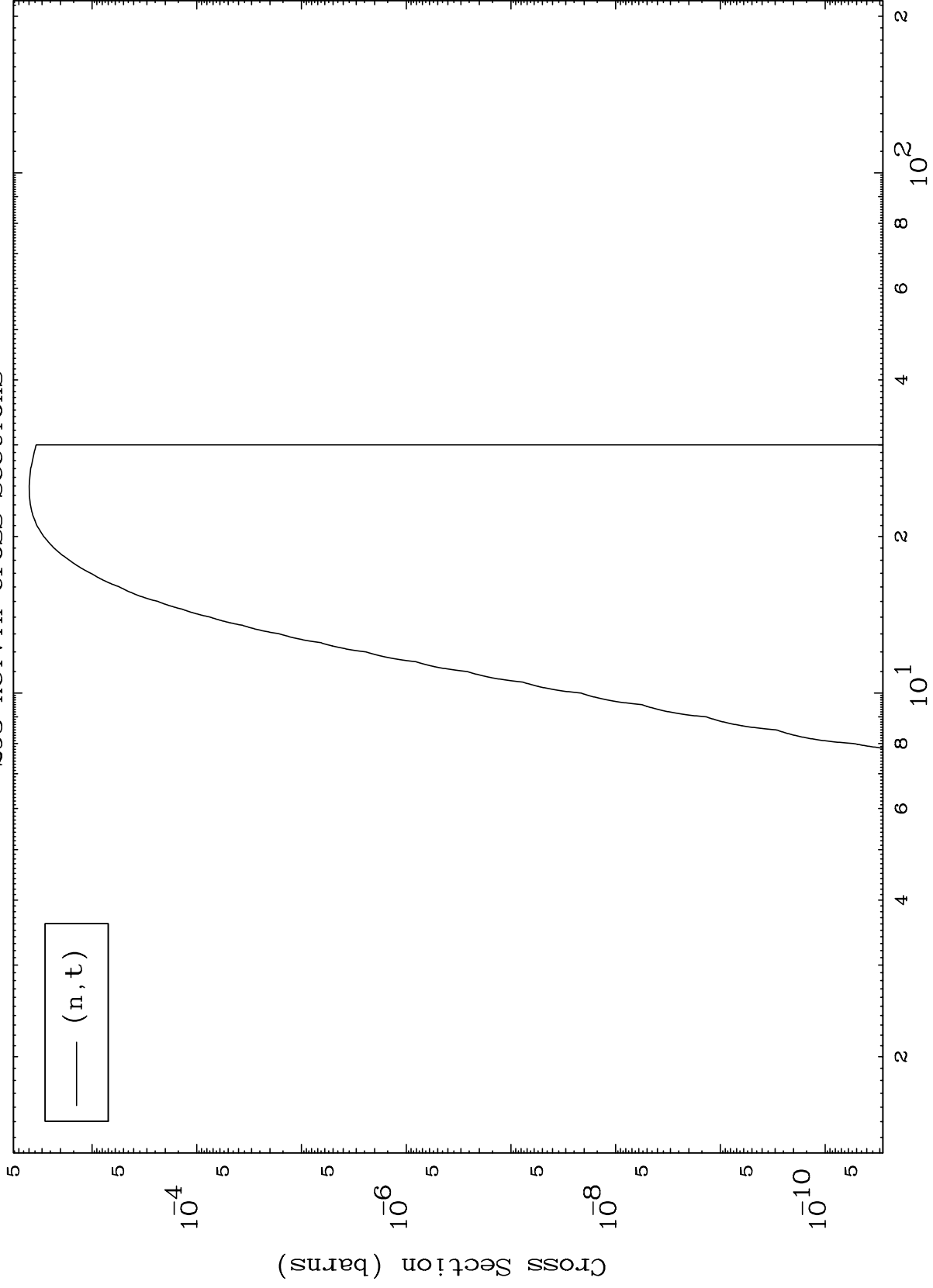
Incident Energy (MeV)

89-Ac-226

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(n,t) Levels
293 Kelvin Cross Sections

89-Ac-226



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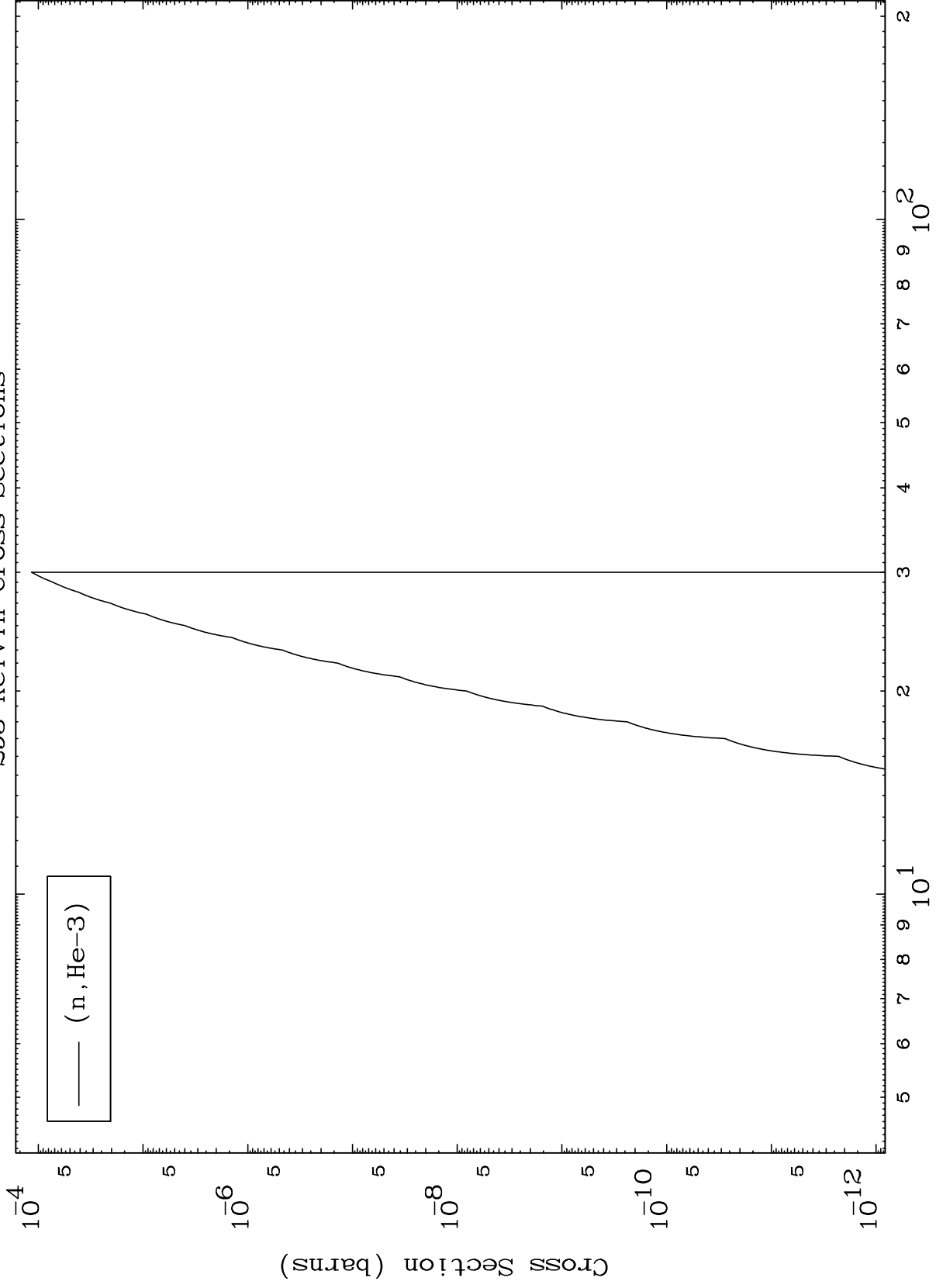
Incident Energy (MeV)

89-Ac-226

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(n,He3) Levels
293 Kelvin Cross Sections

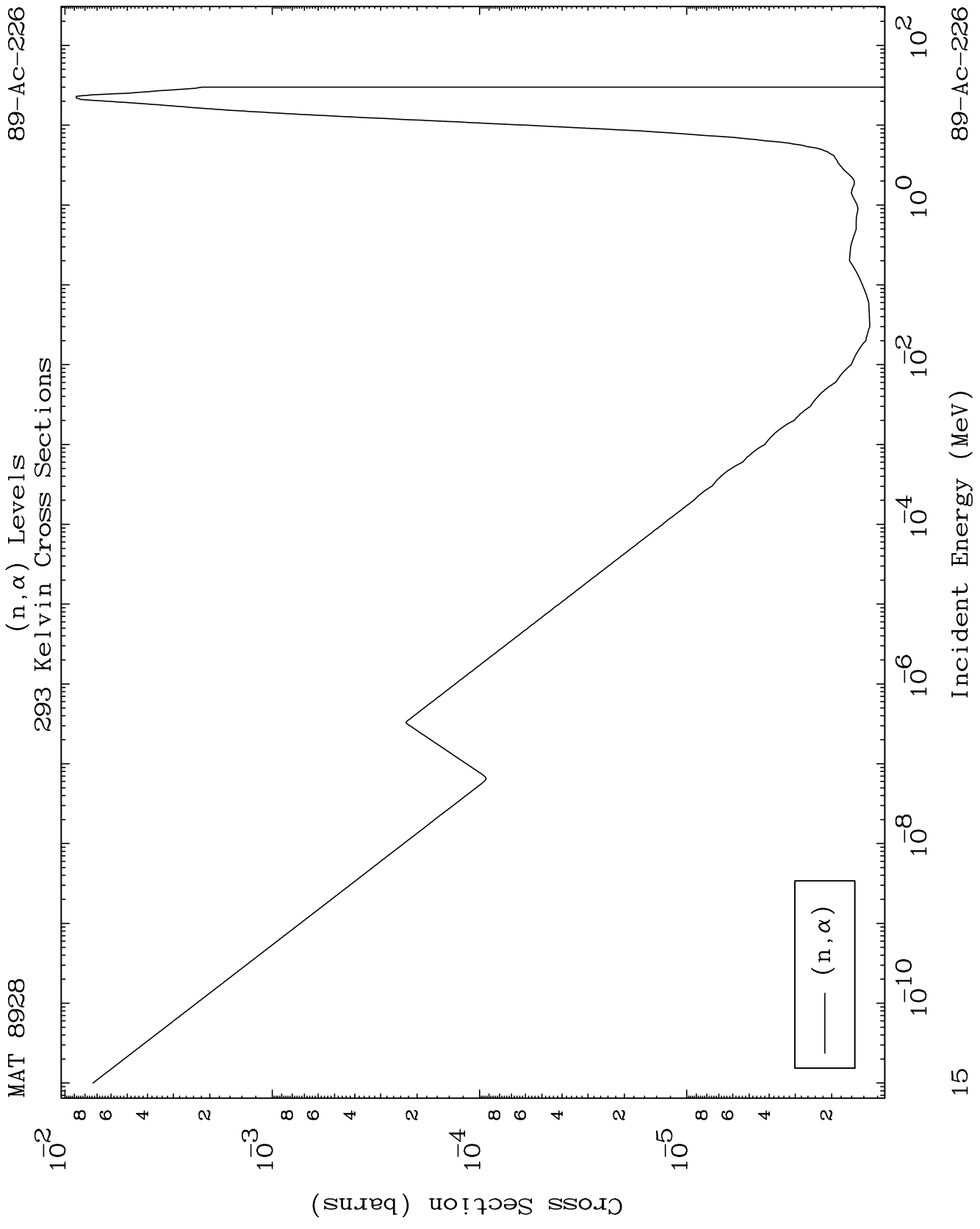
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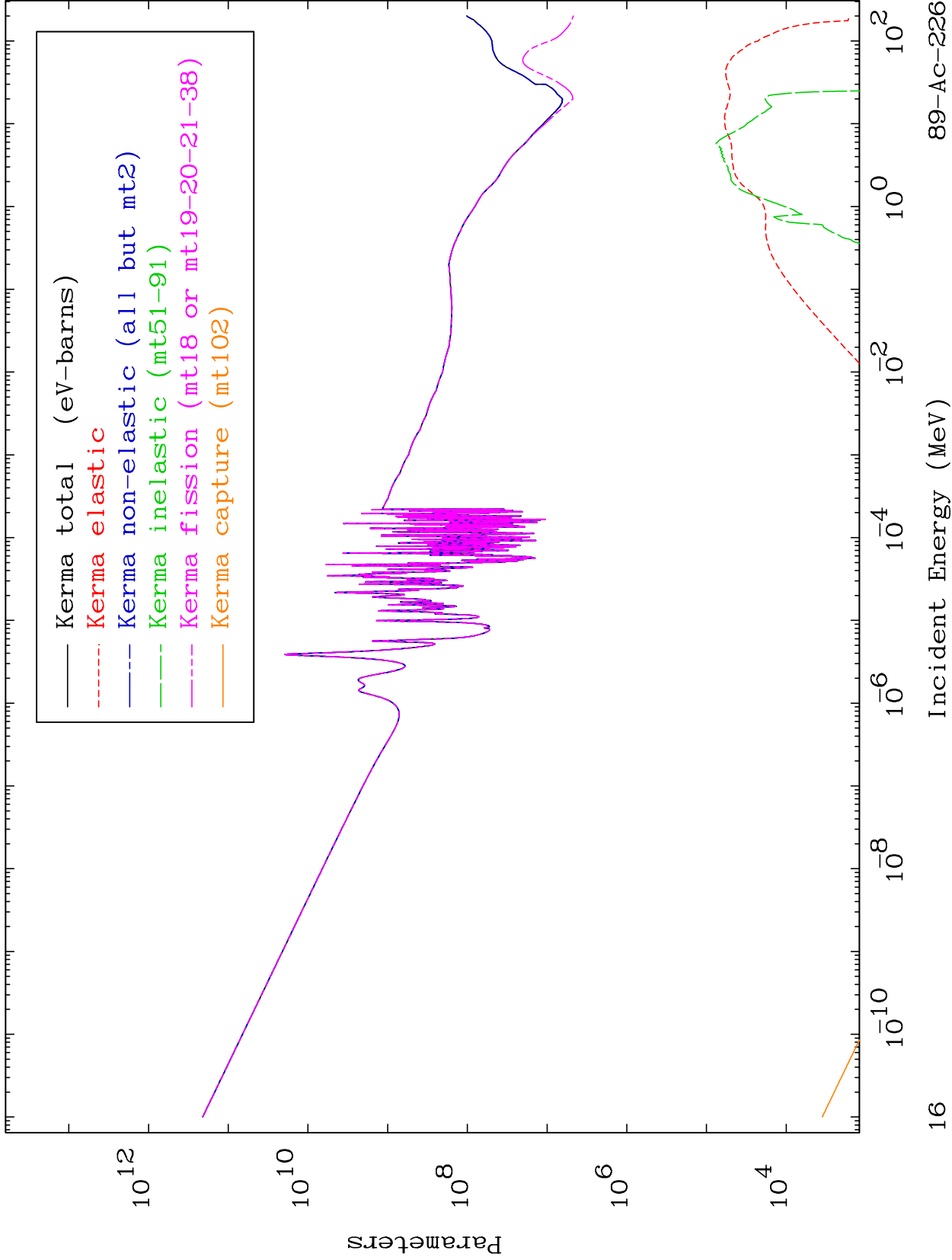


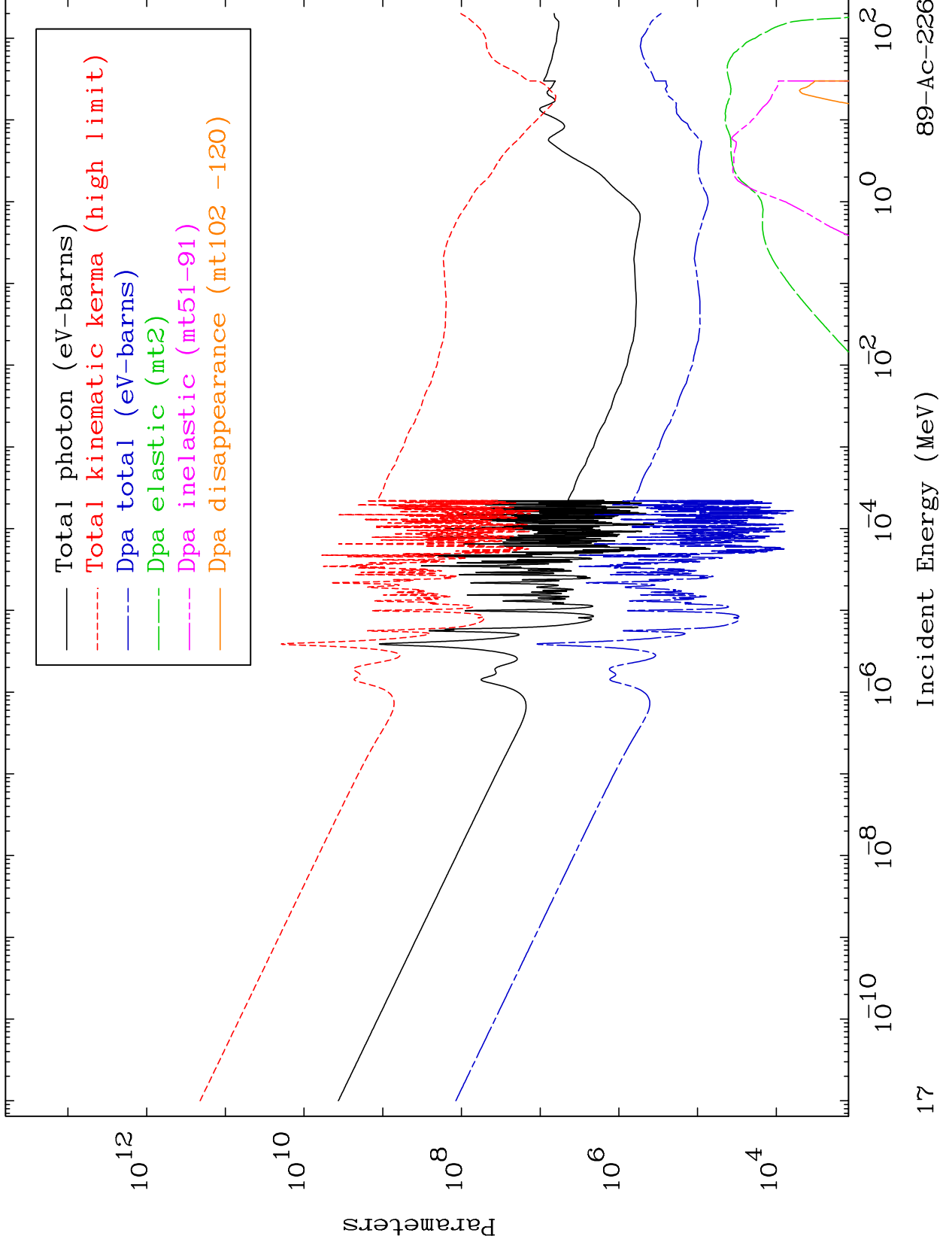
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Incident Energy (MeV)

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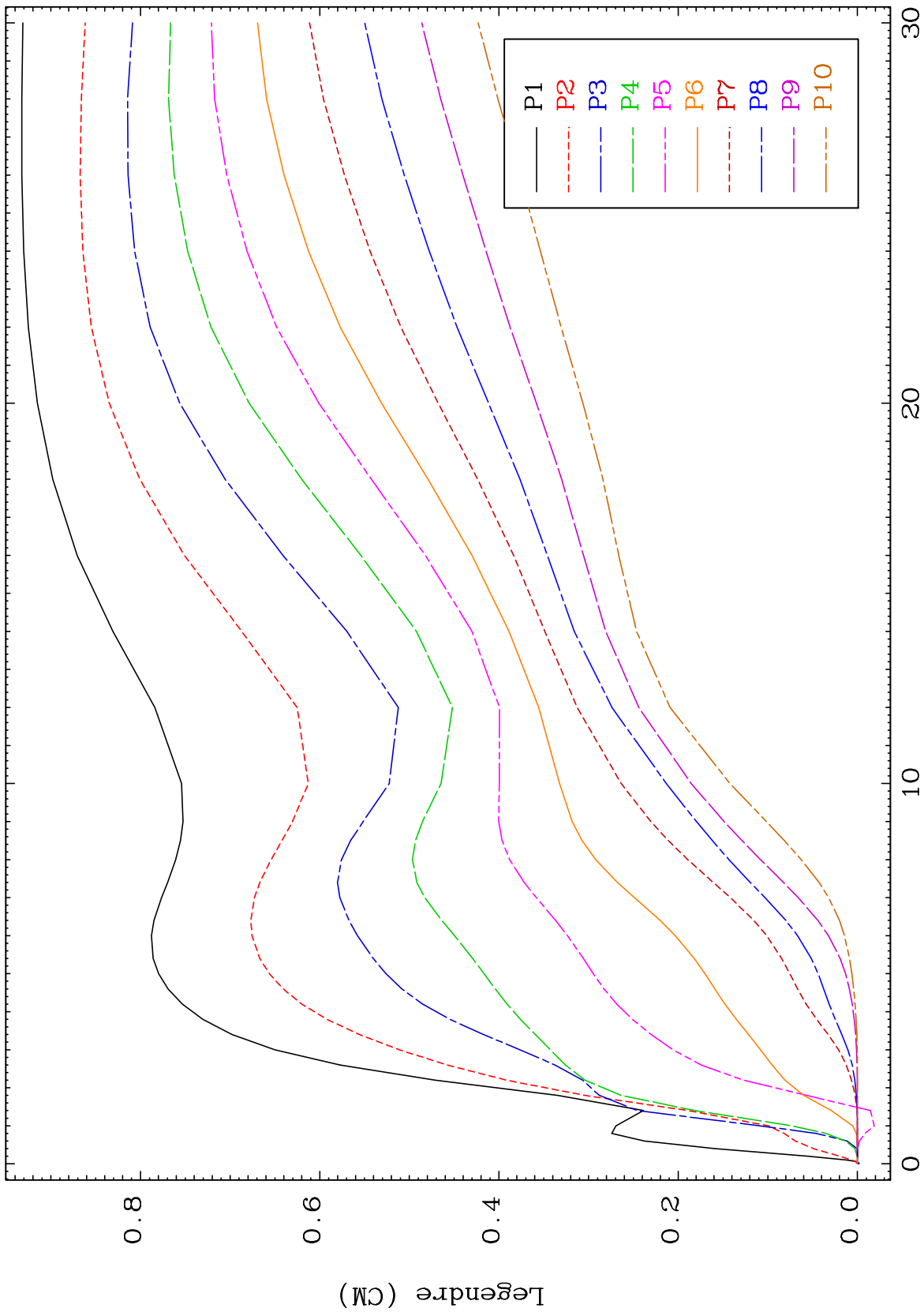




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Elastic Legendre Coefficients

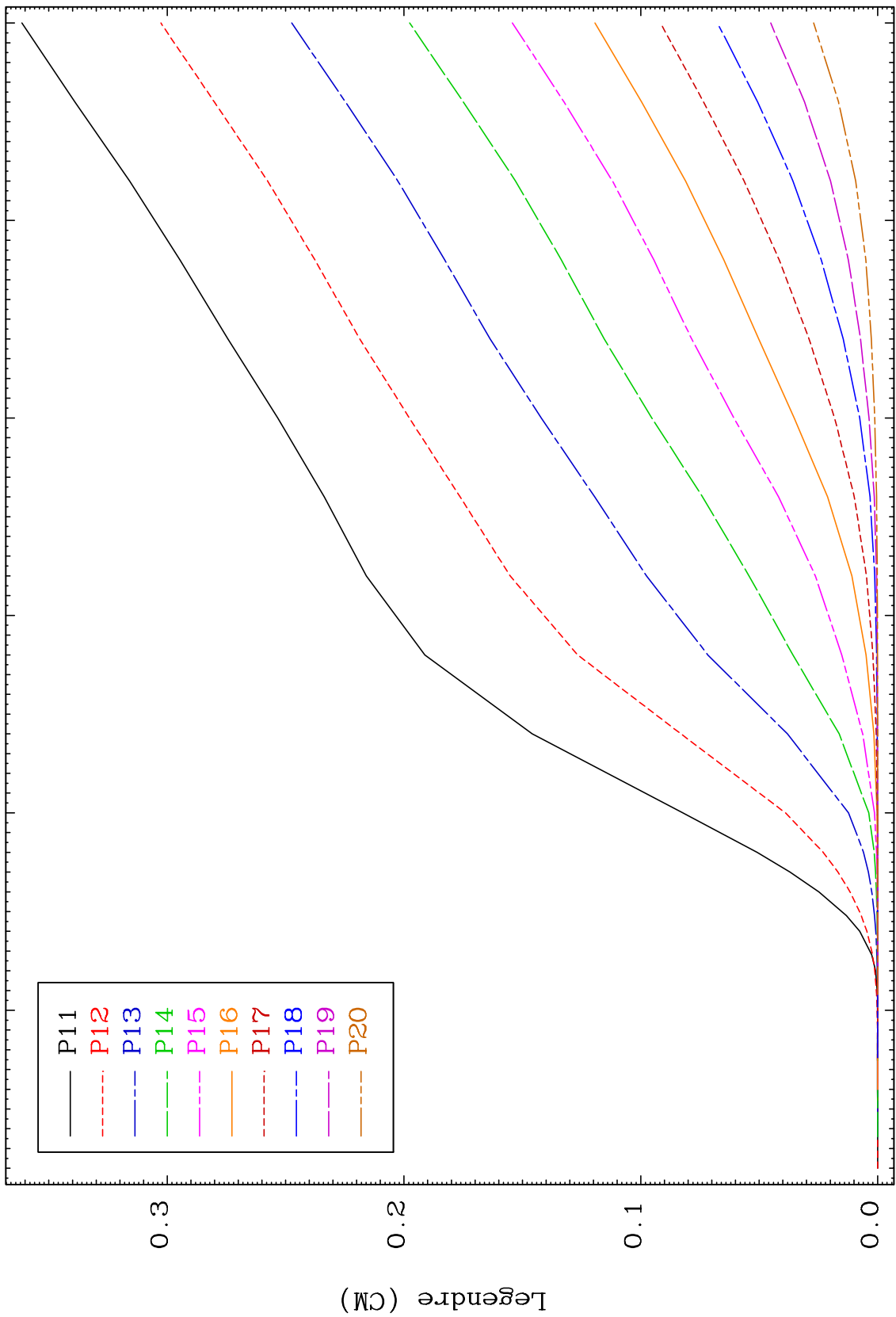


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Incident Energy (MeV)

18

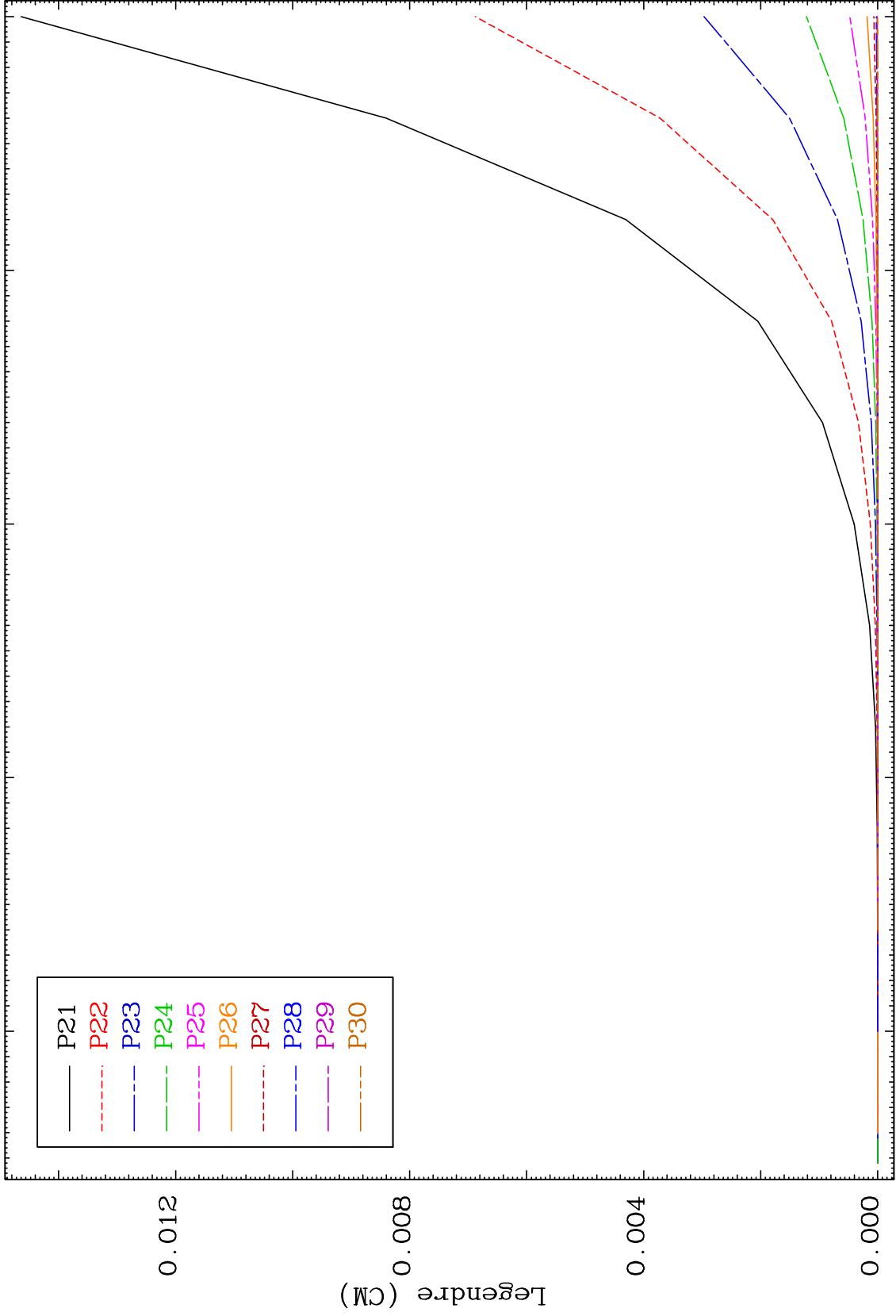
Elastic Legendre Coefficients



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Elastic Legendre Coefficients

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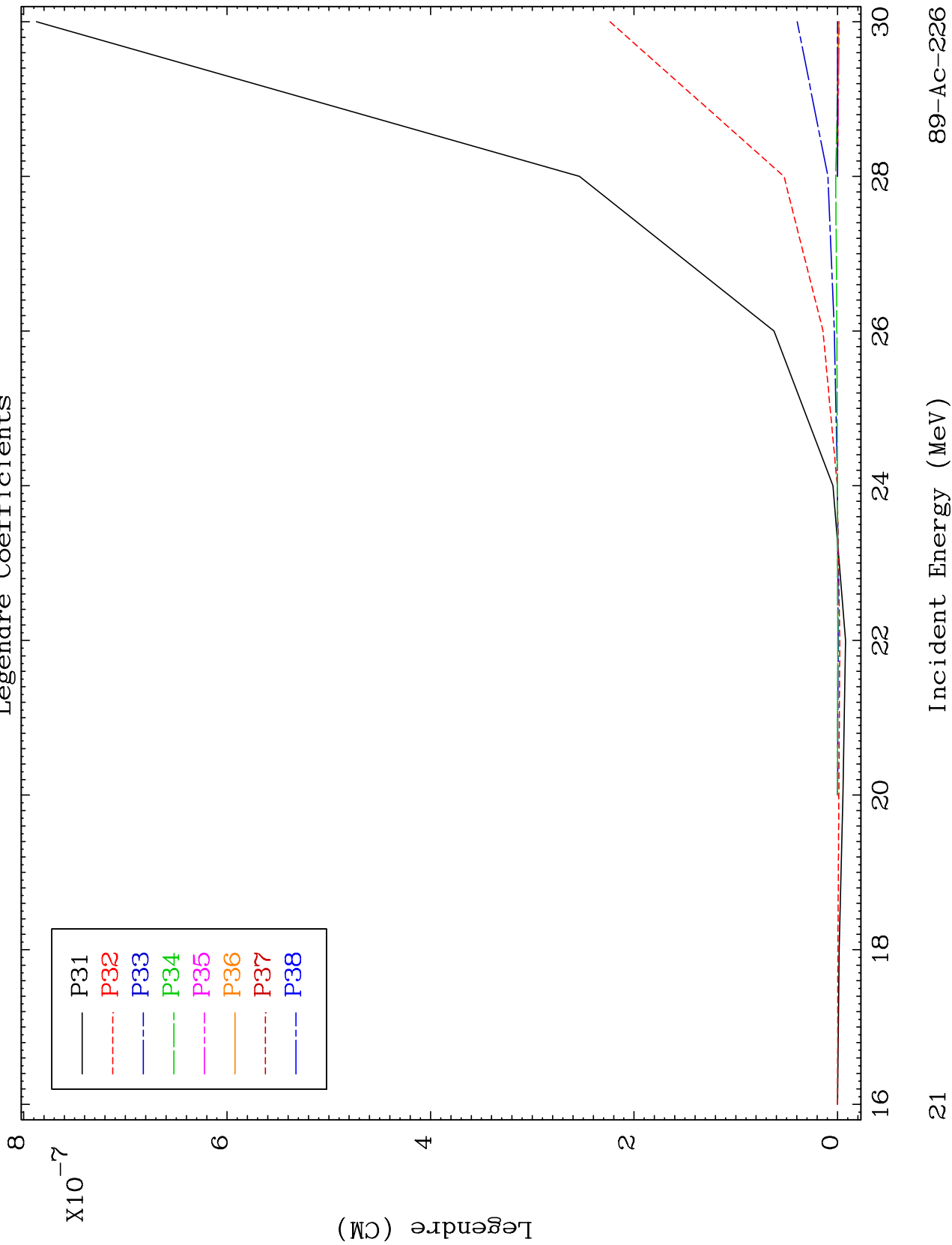
Incident Energy (MeV)

20

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Elastic Legendre Coefficients

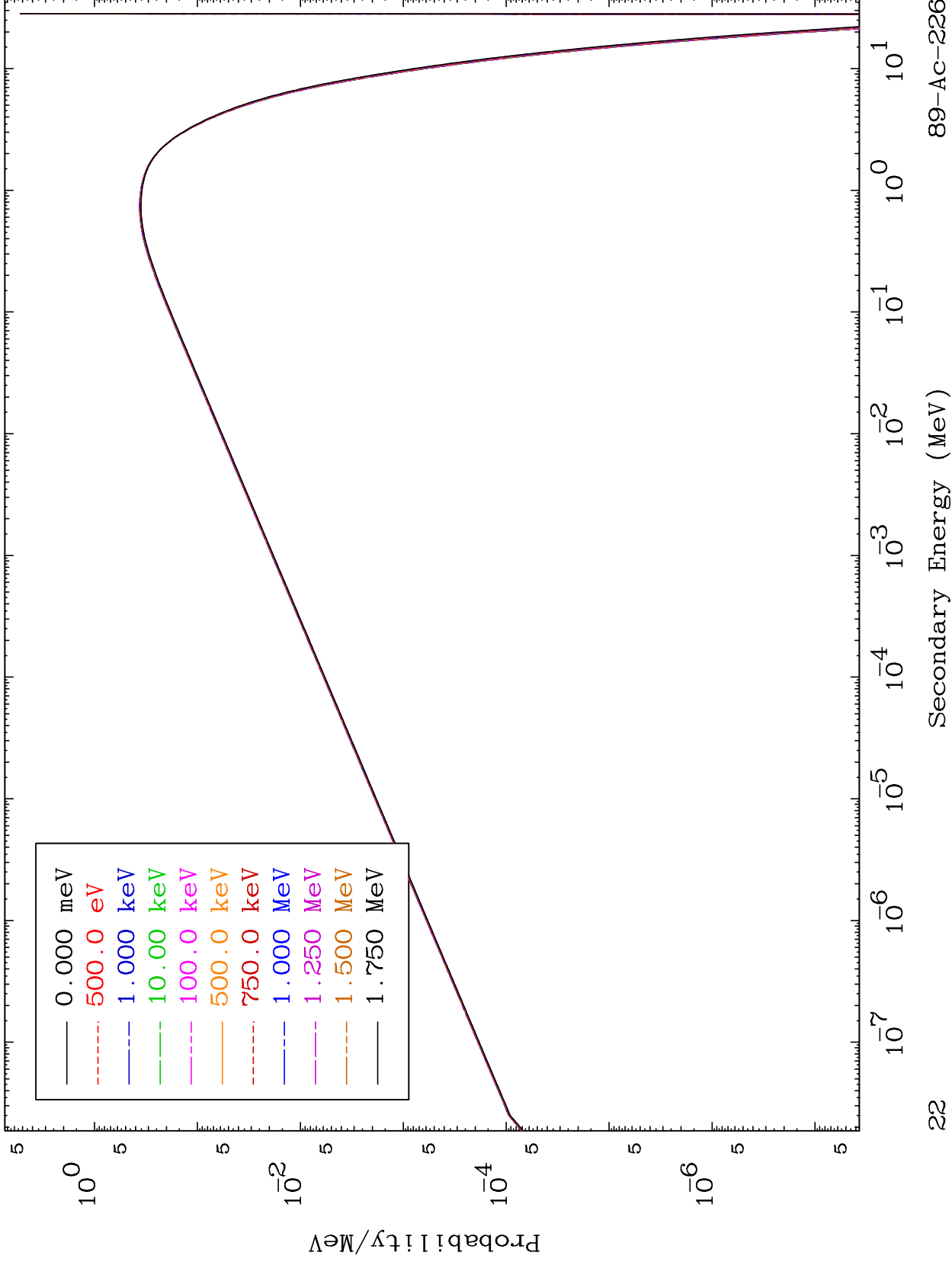
89-Ac-226



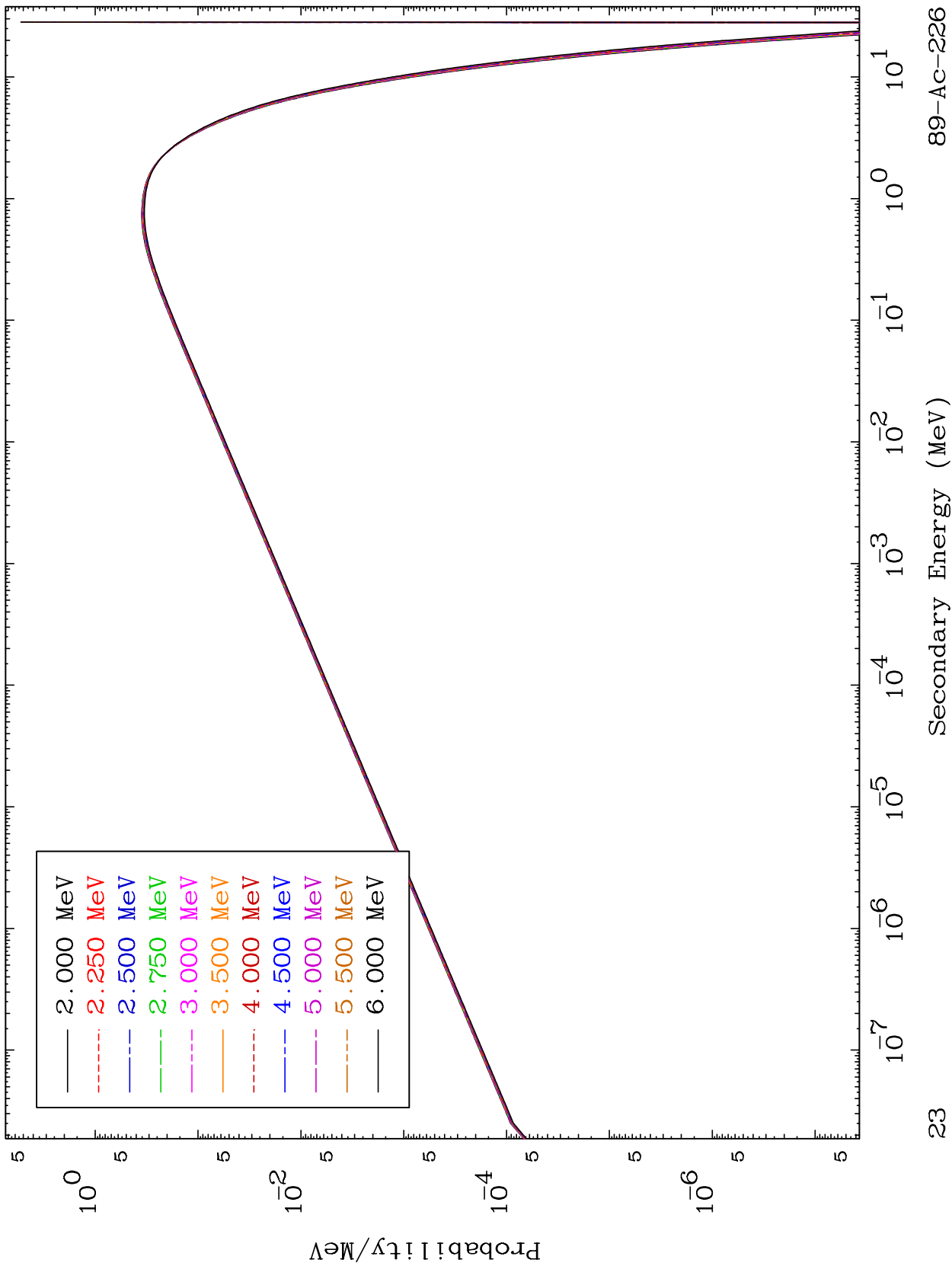
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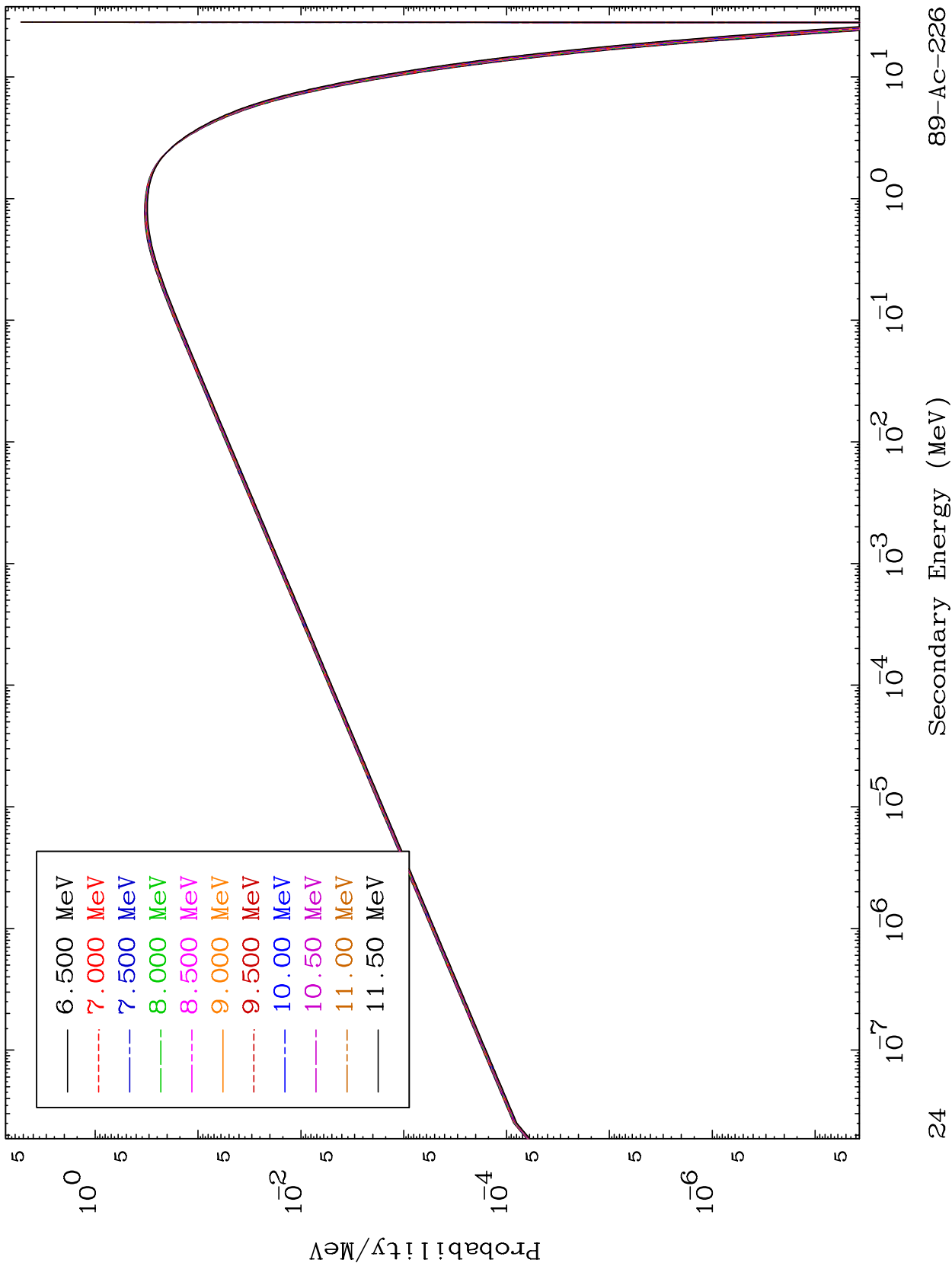
Fission Energy Distribution

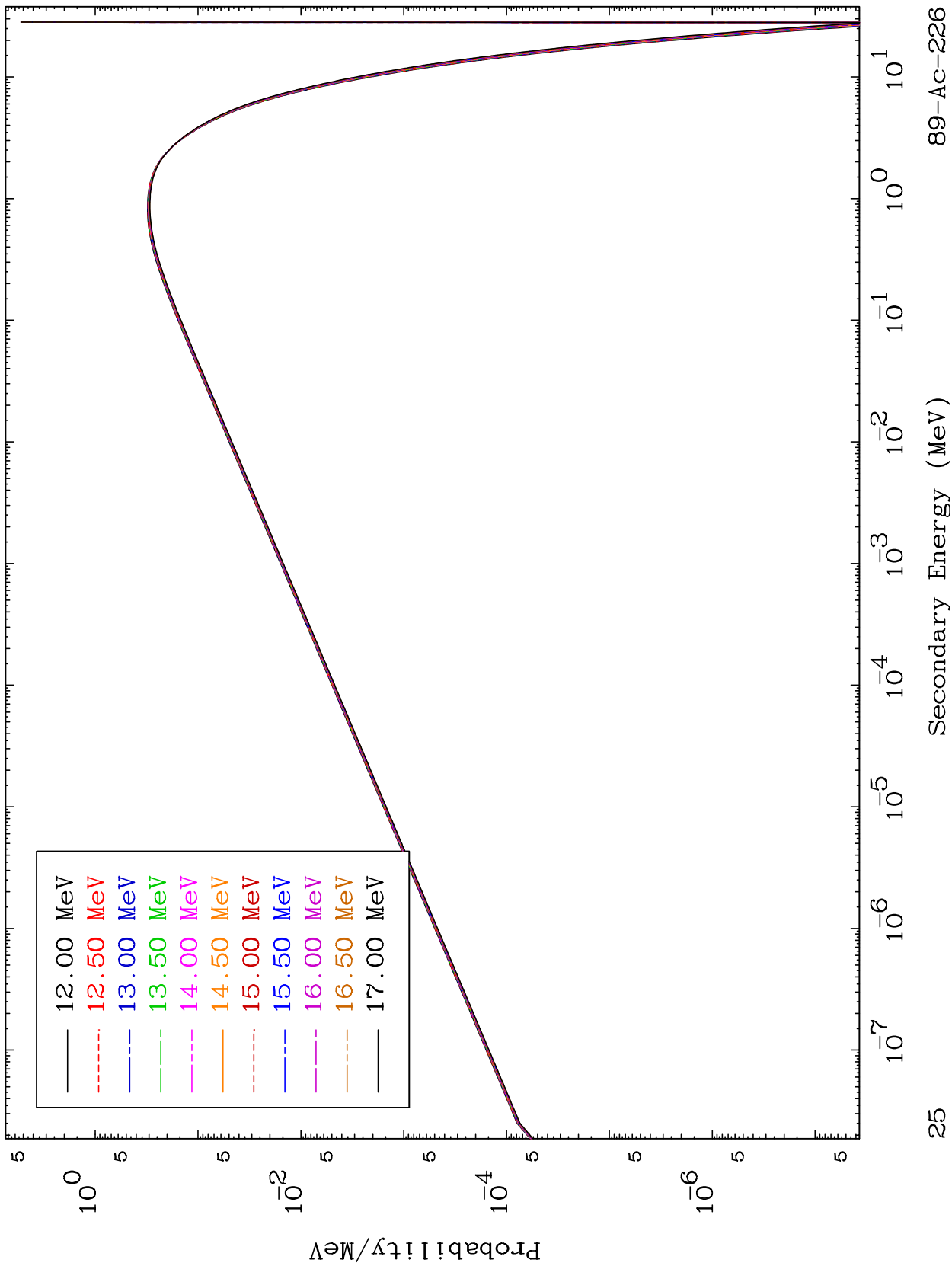


Fission Energy Distribution

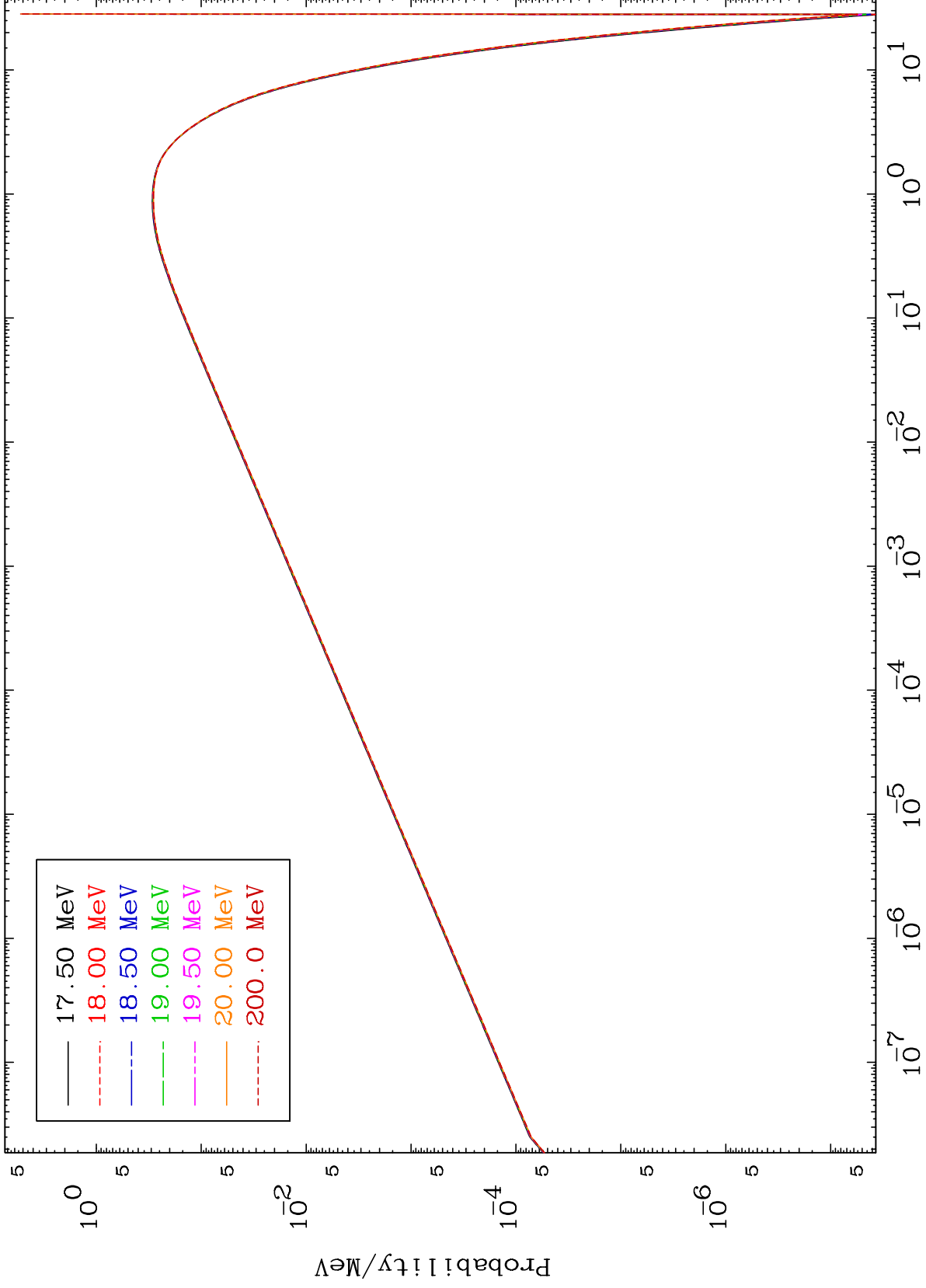


Fission Energy Distribution





Fission
Energy Distribution

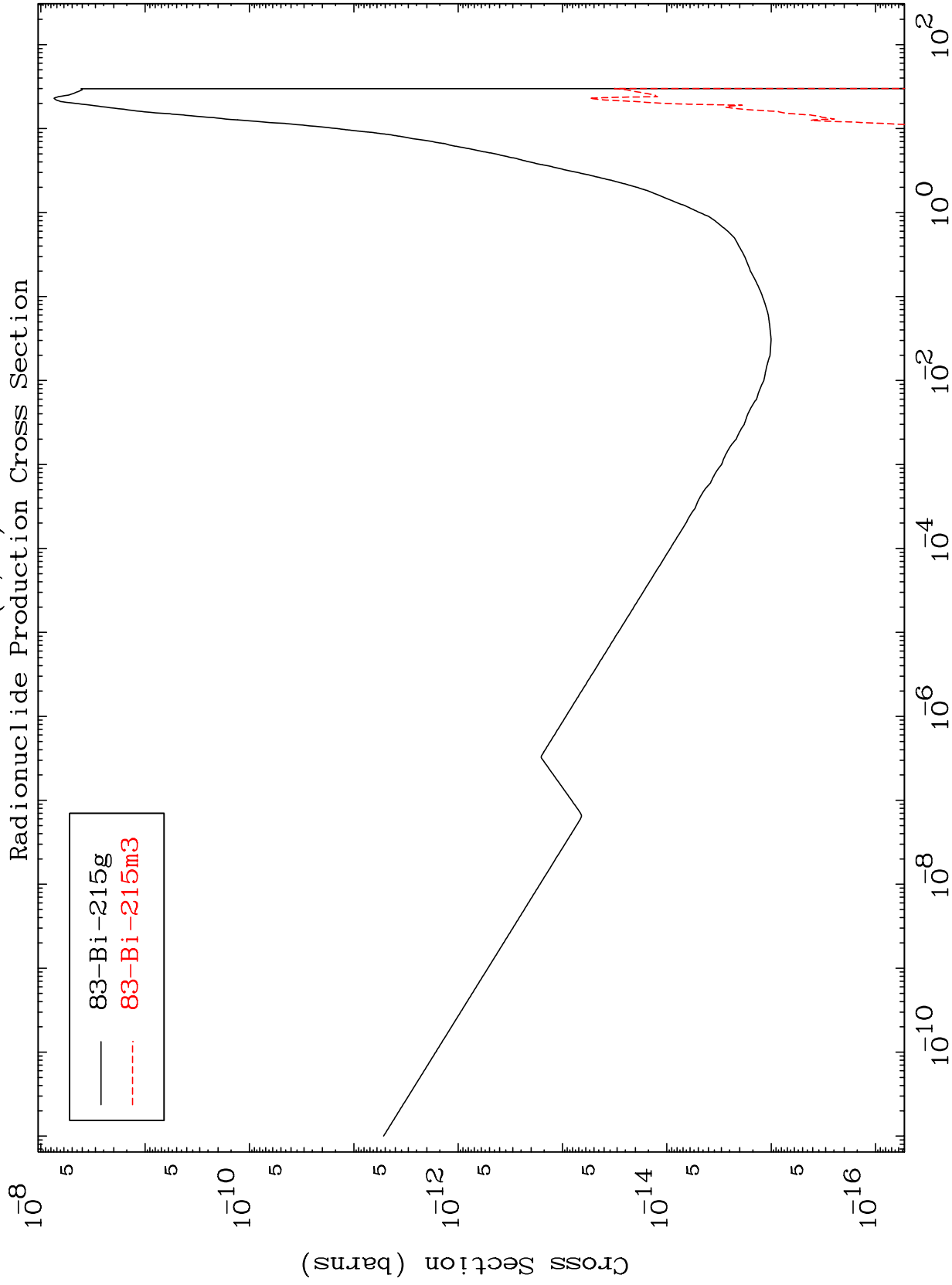


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(n, 3α)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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