

Program EVALPLOT  
(Version 2015-2)

by

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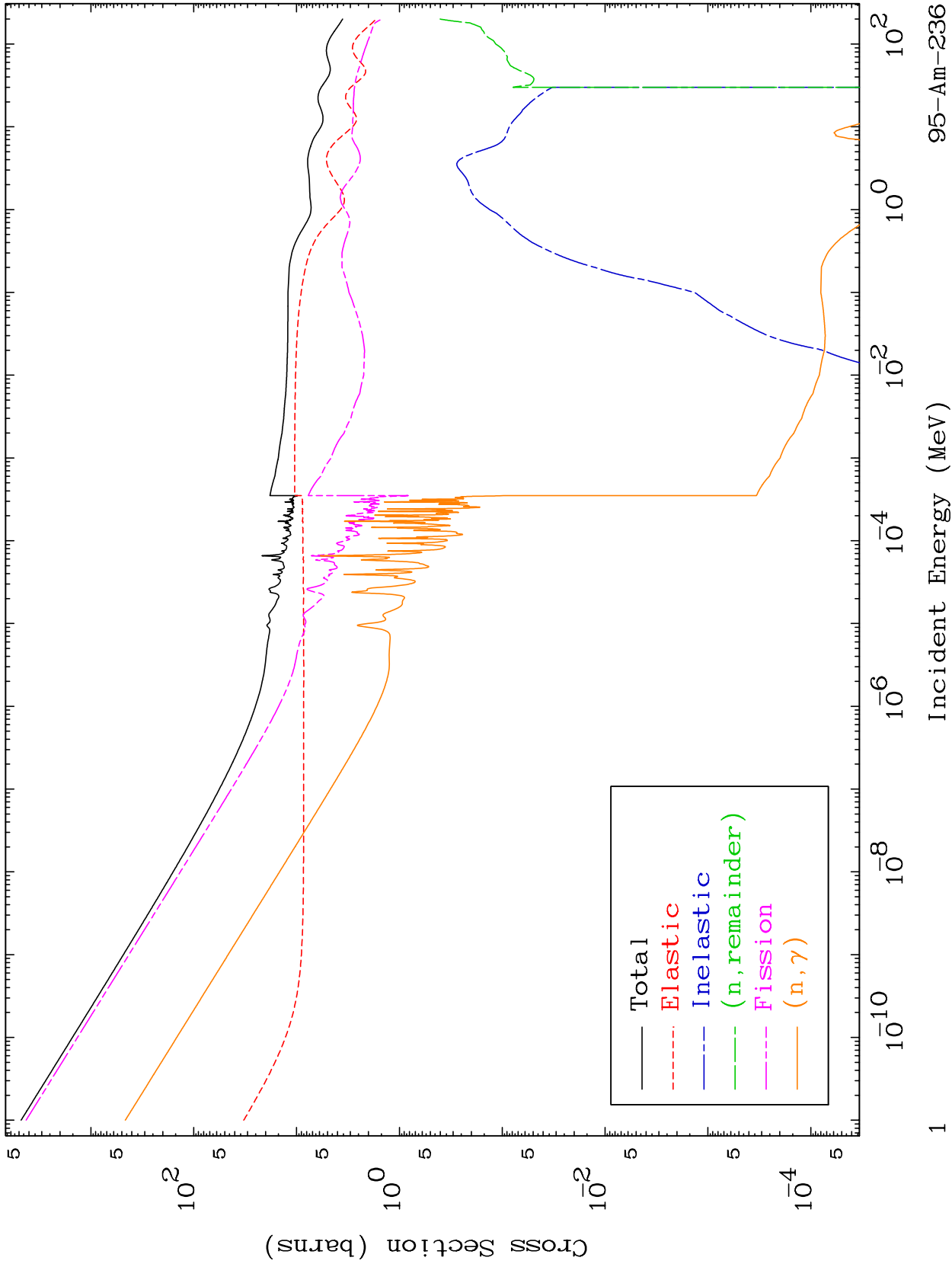
Press Mouse Button to Start

MAT 9529

Major

293 Kelvin Cross Sections

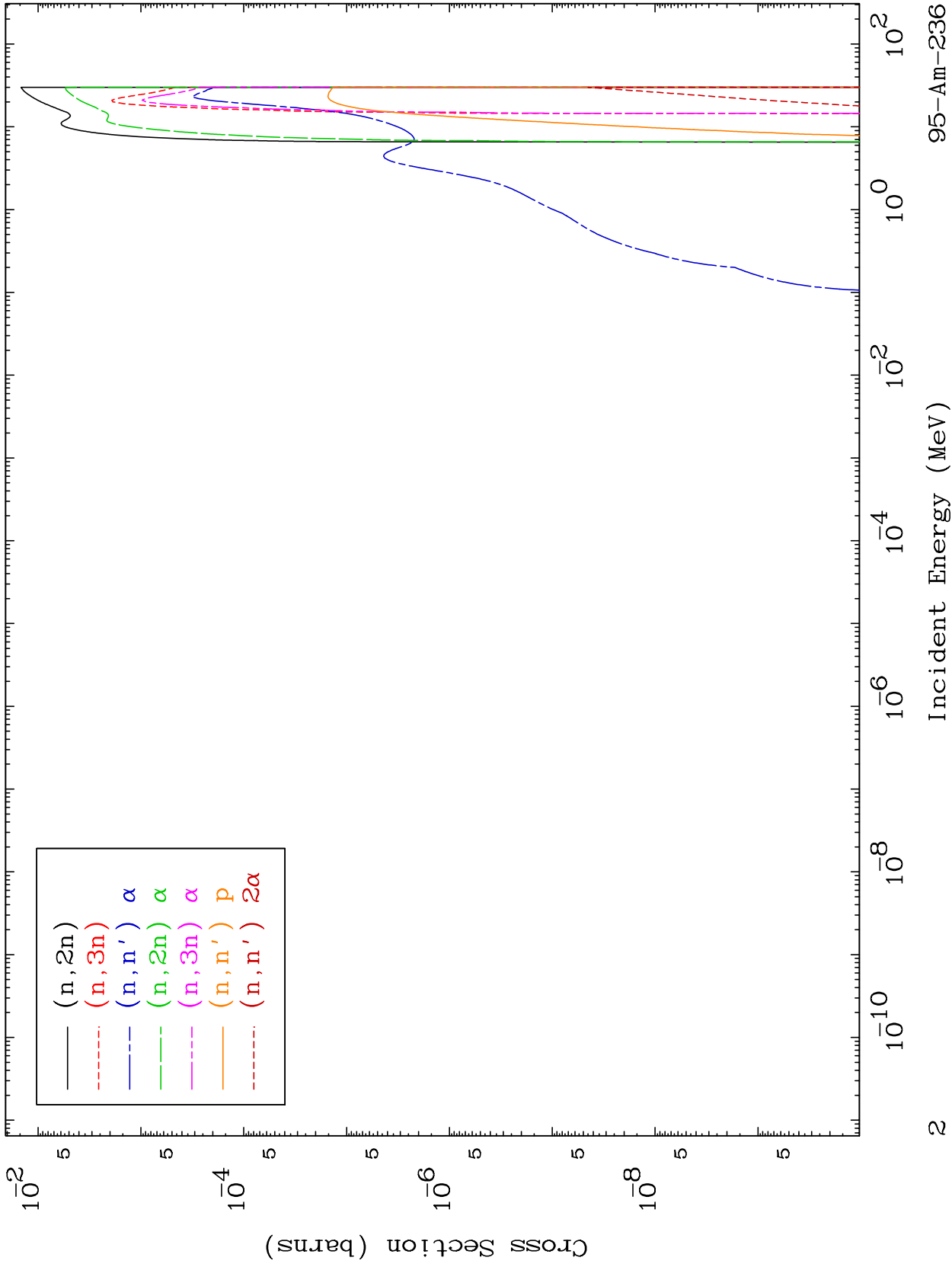
95-Am-236



MAT 9529

Neutron Production  
293 Kelvin Cross Sections

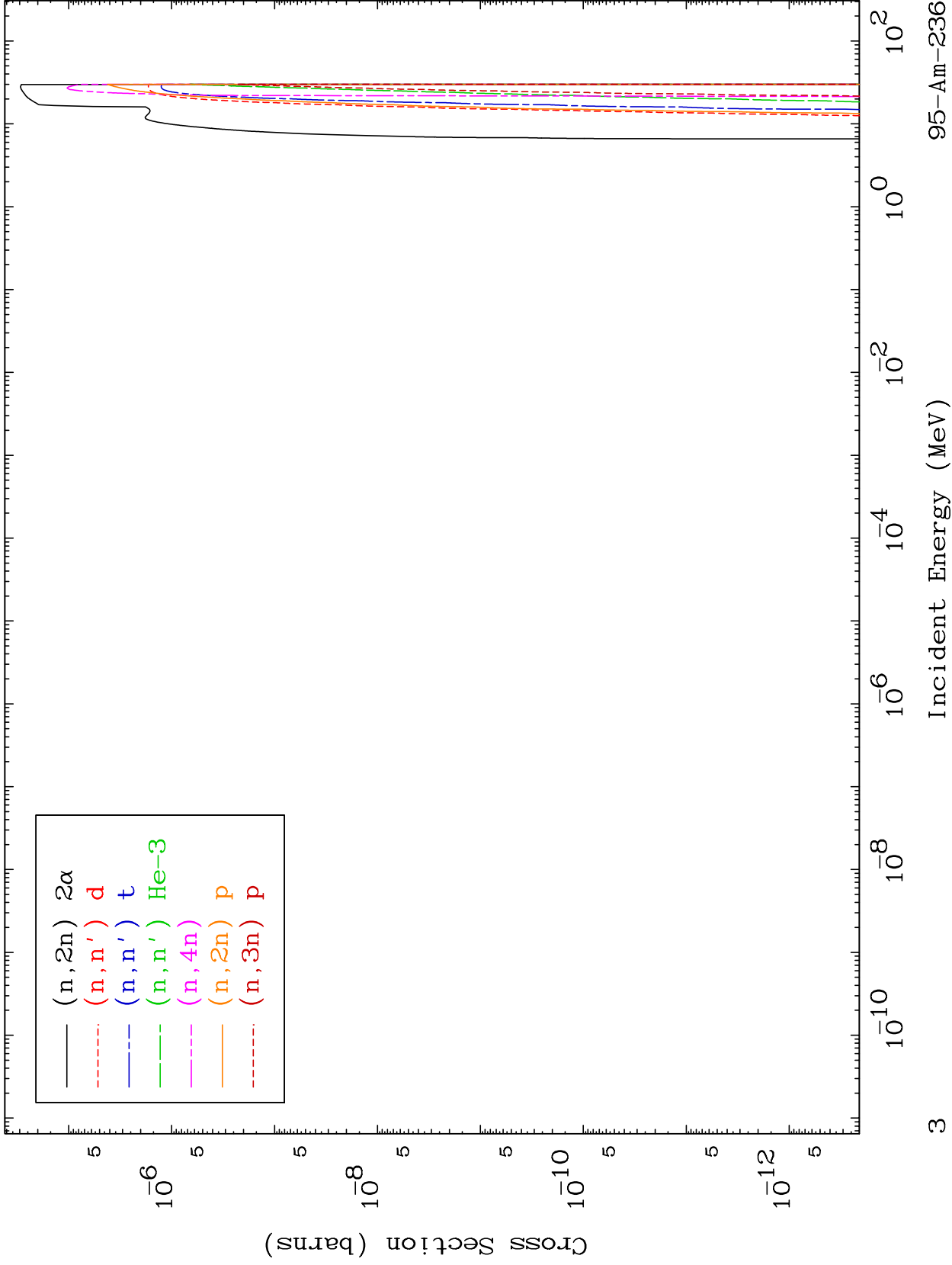
95-Am-236



MAT 9529

Neutron Production  
293 Kelvin Cross Sections

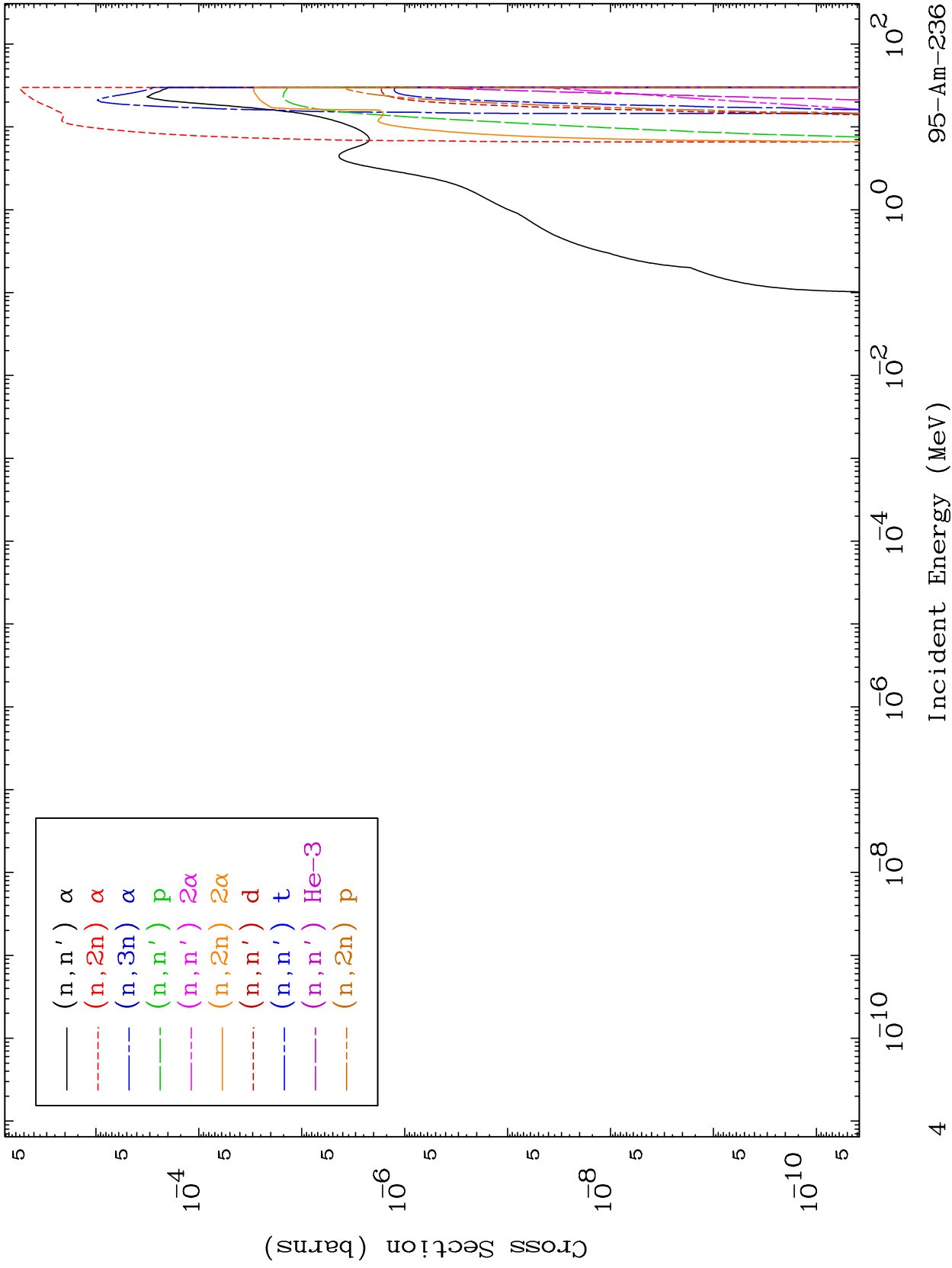
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Charged Particle  
293 Kelvin Cross Sections

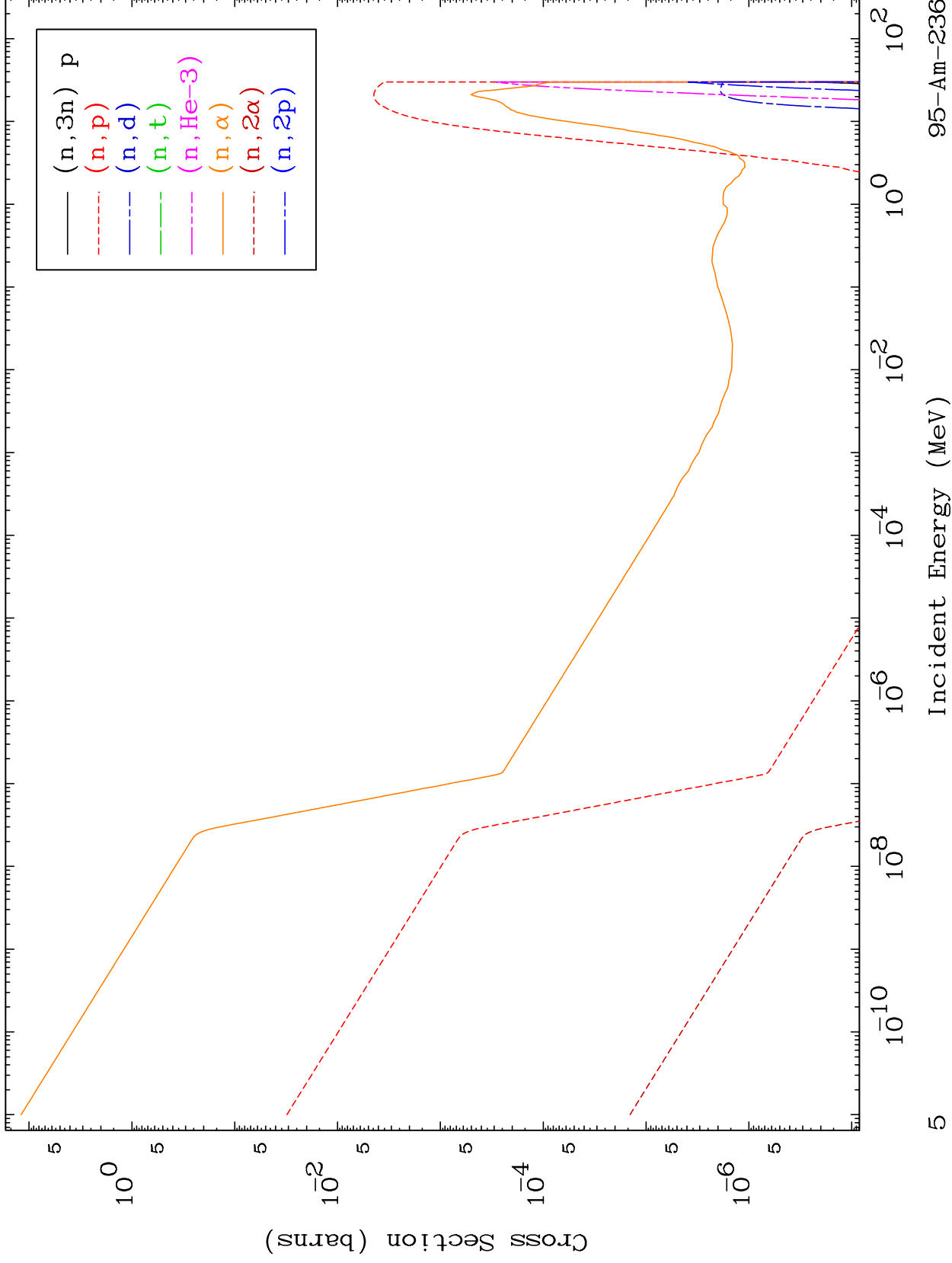
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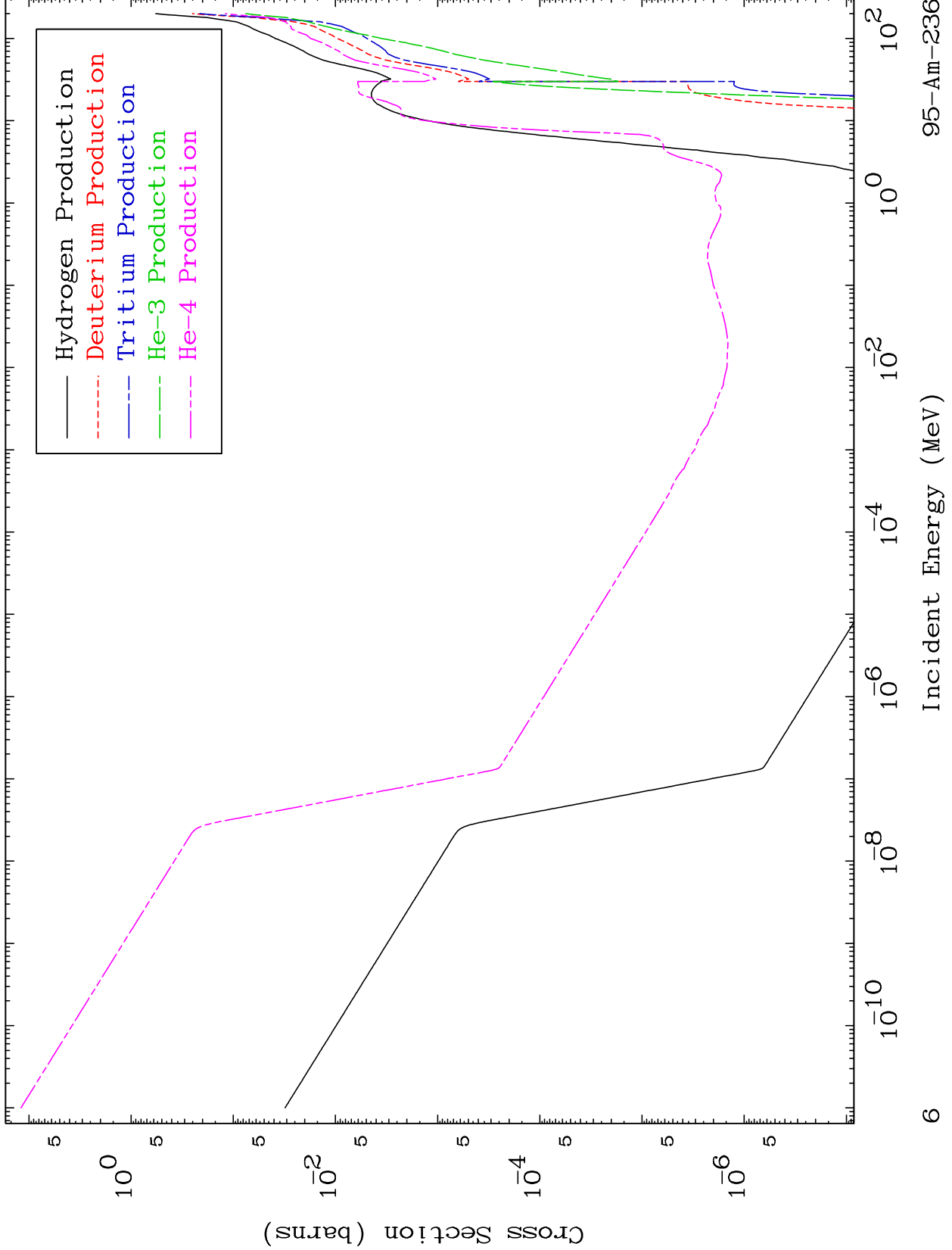
Charged Particle  
293 Kelvin Cross Sections

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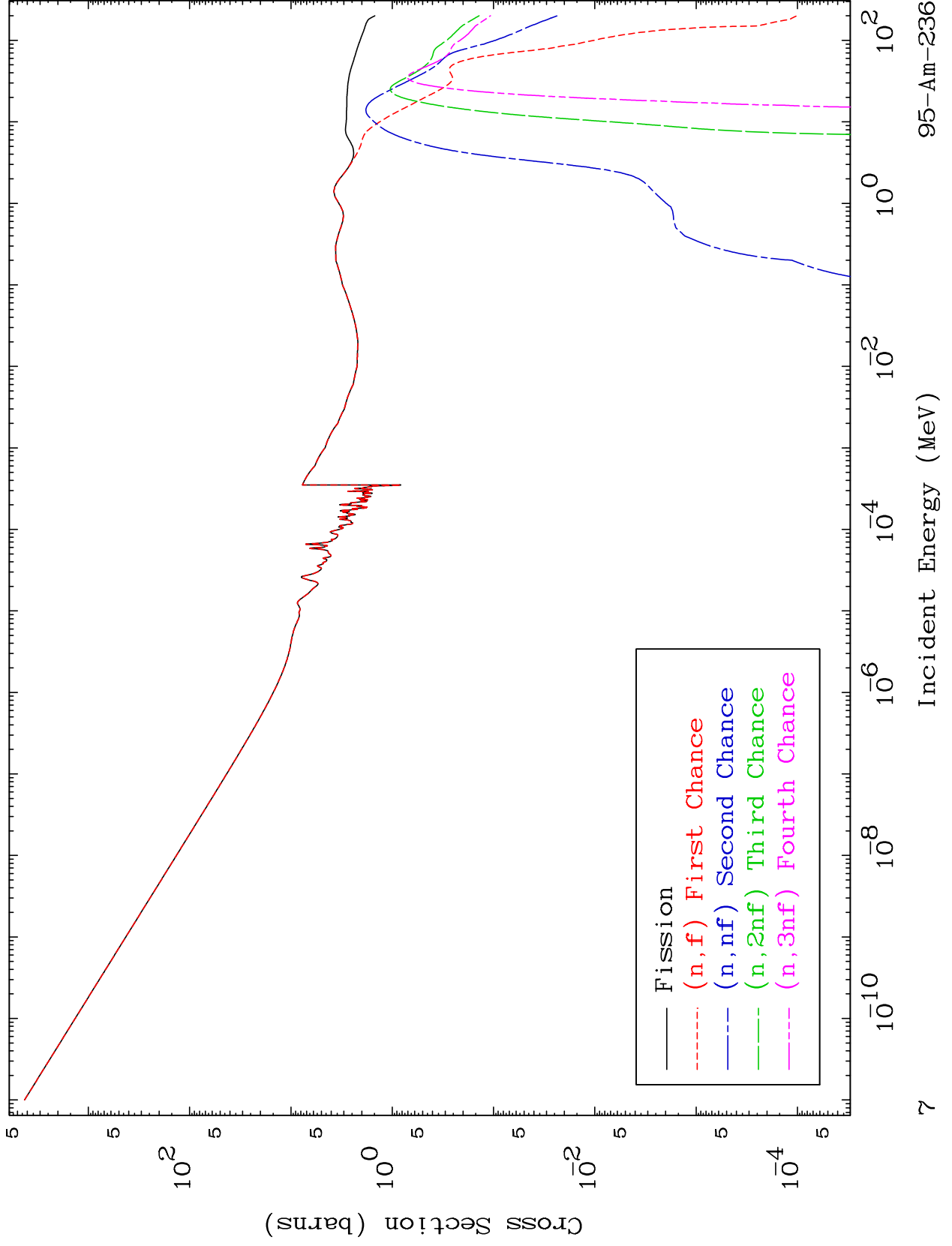
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Fission  
293 Kelvin Cross Sections

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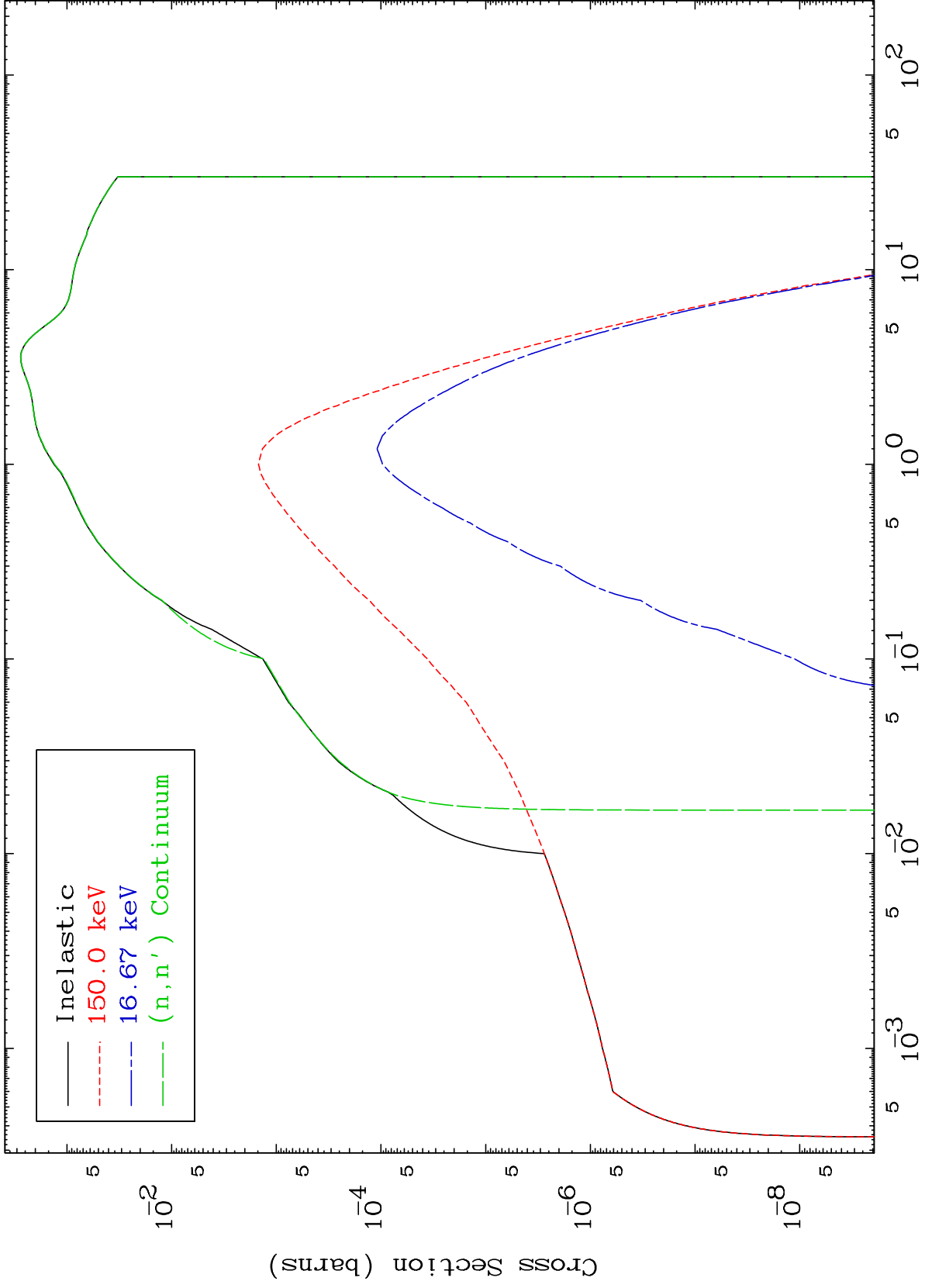




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293 Kelvin Cross Sections  
(n,n') Level

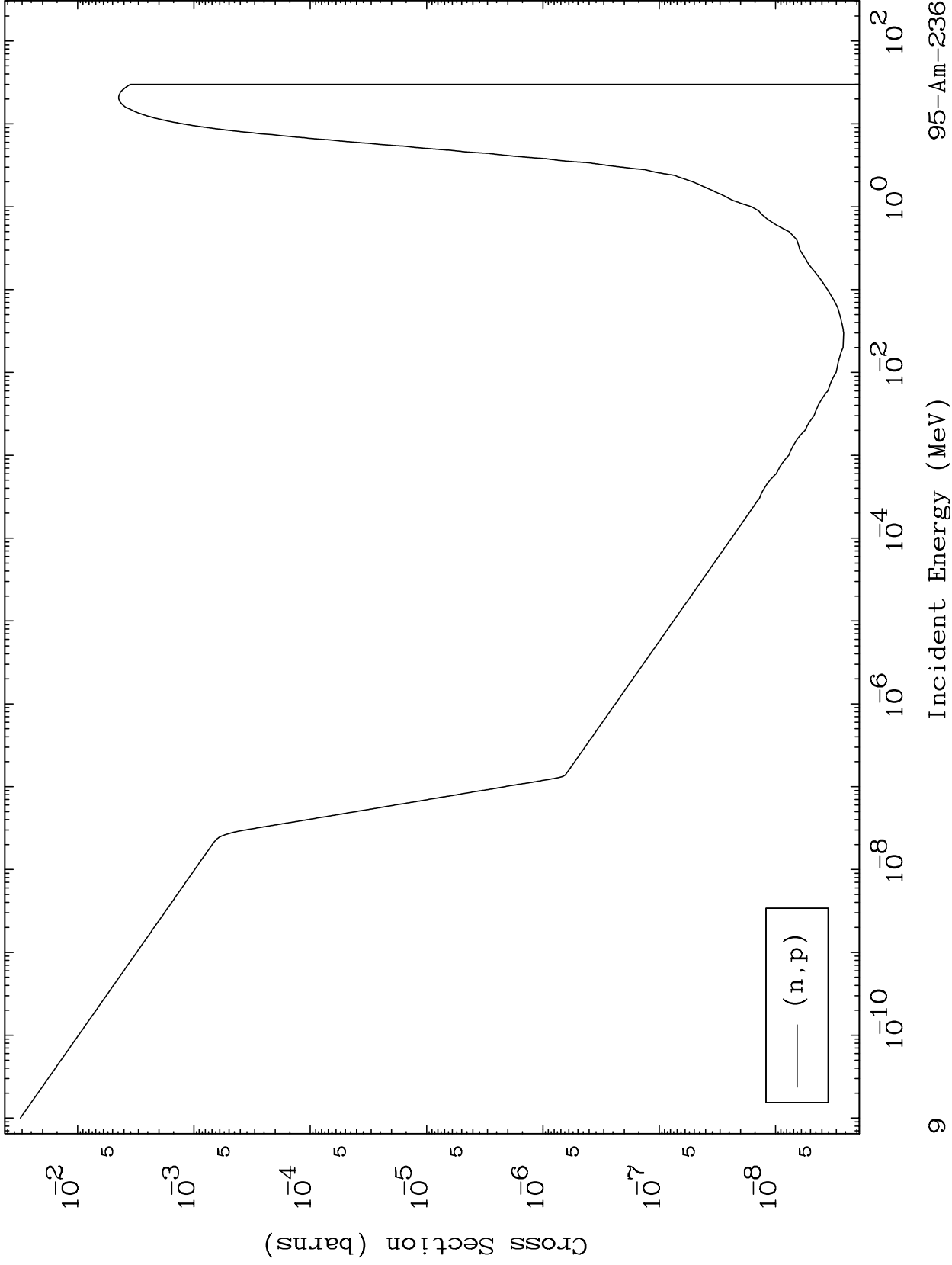
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(n,p) Levels  
293 Kelvin Cross Sections

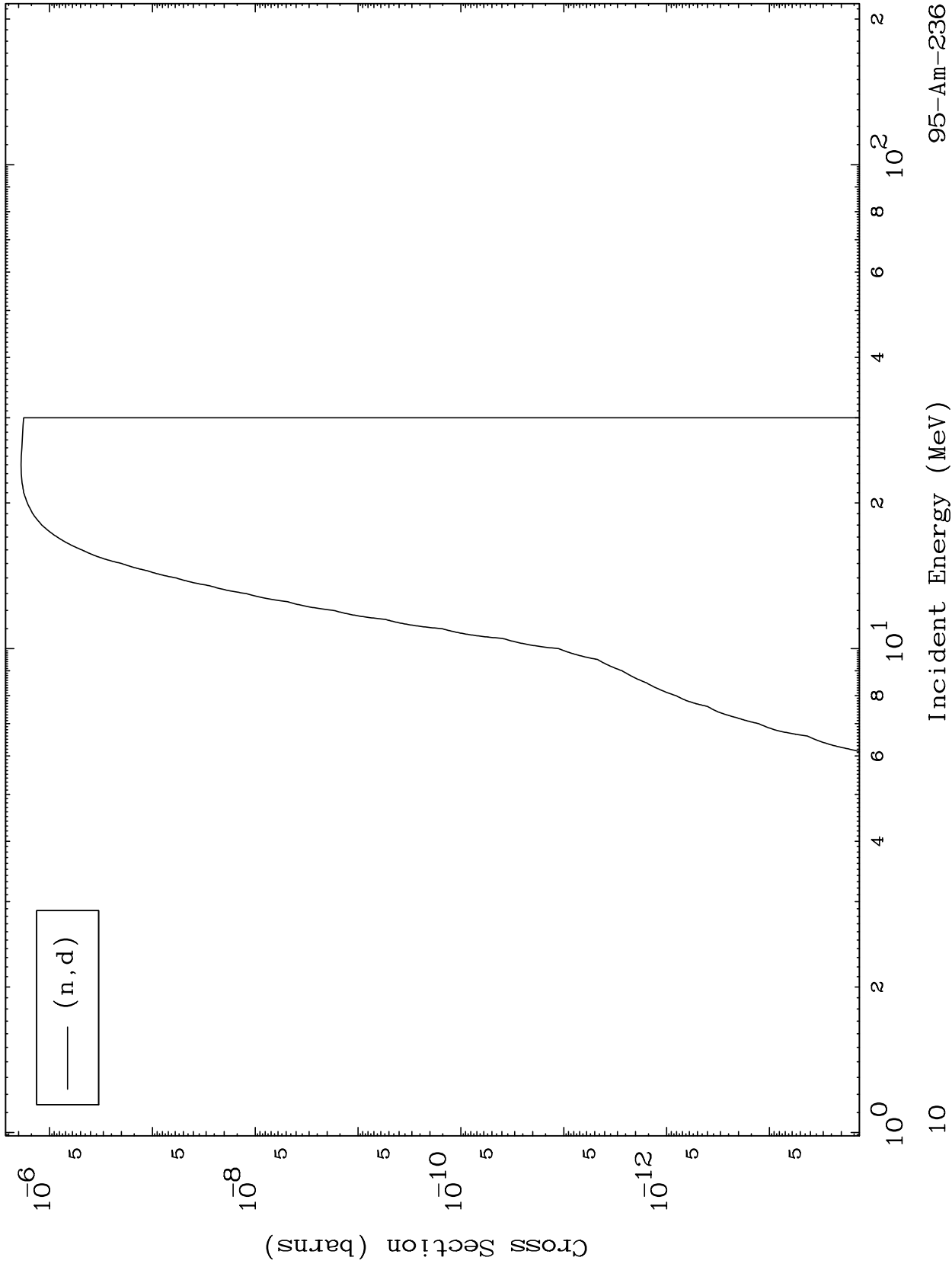
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(n,d) Levels  
293 Kelvin Cross Sections

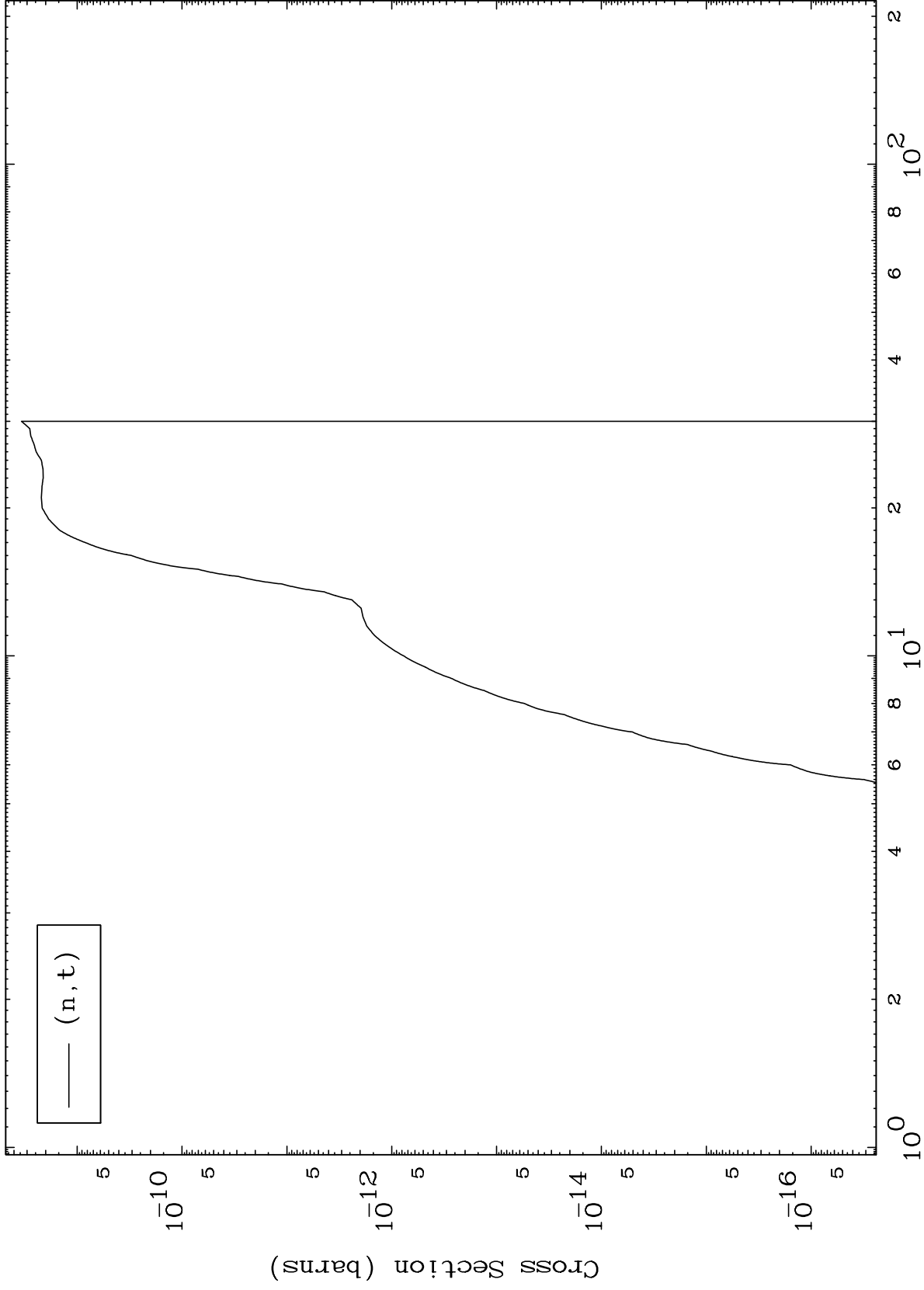
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(n,t) Levels  
293 Kelvin Cross Sections

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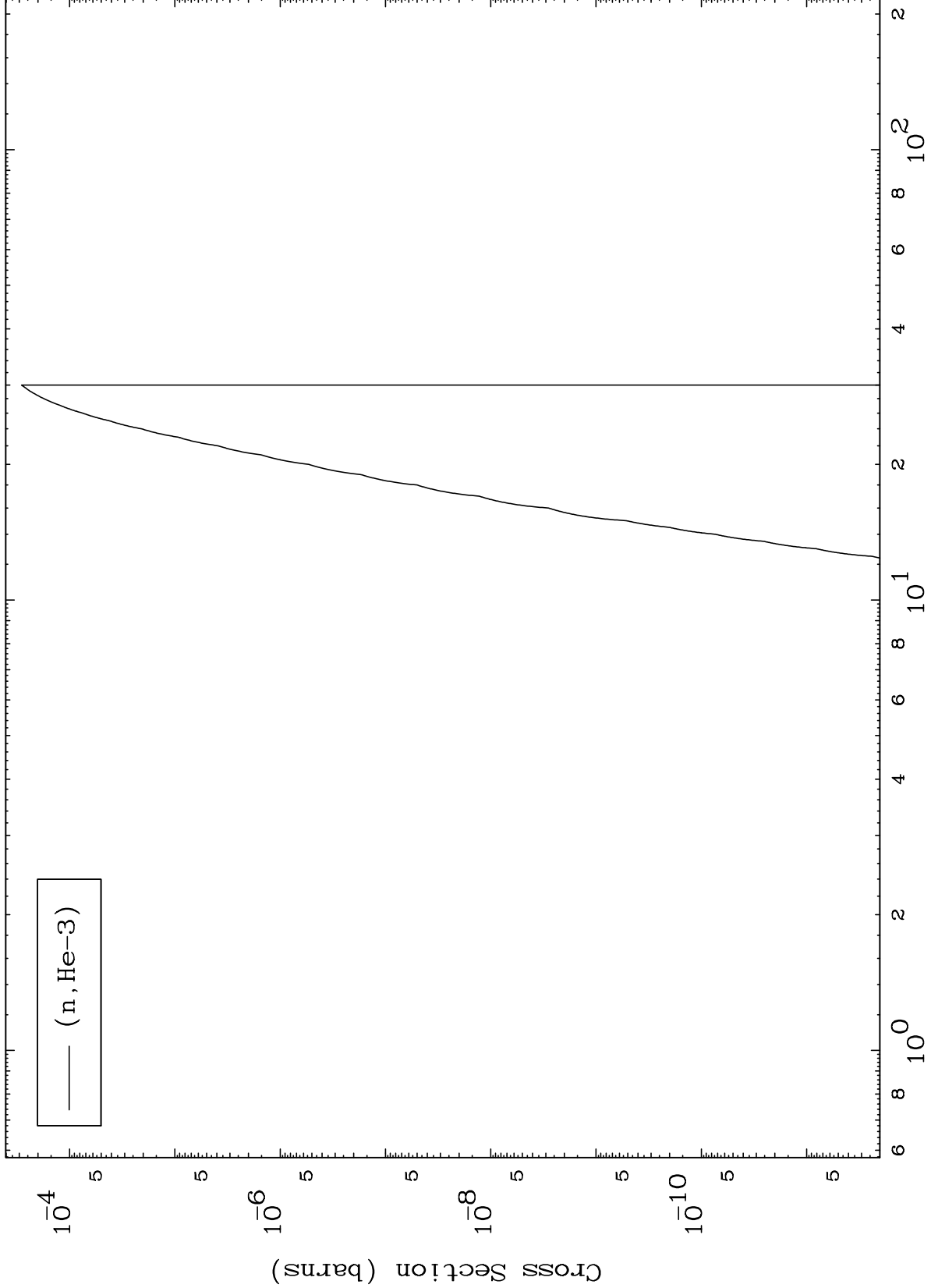
Incident Energy (MeV)

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(n,He3) Levels  
293 Kelvin Cross Sections

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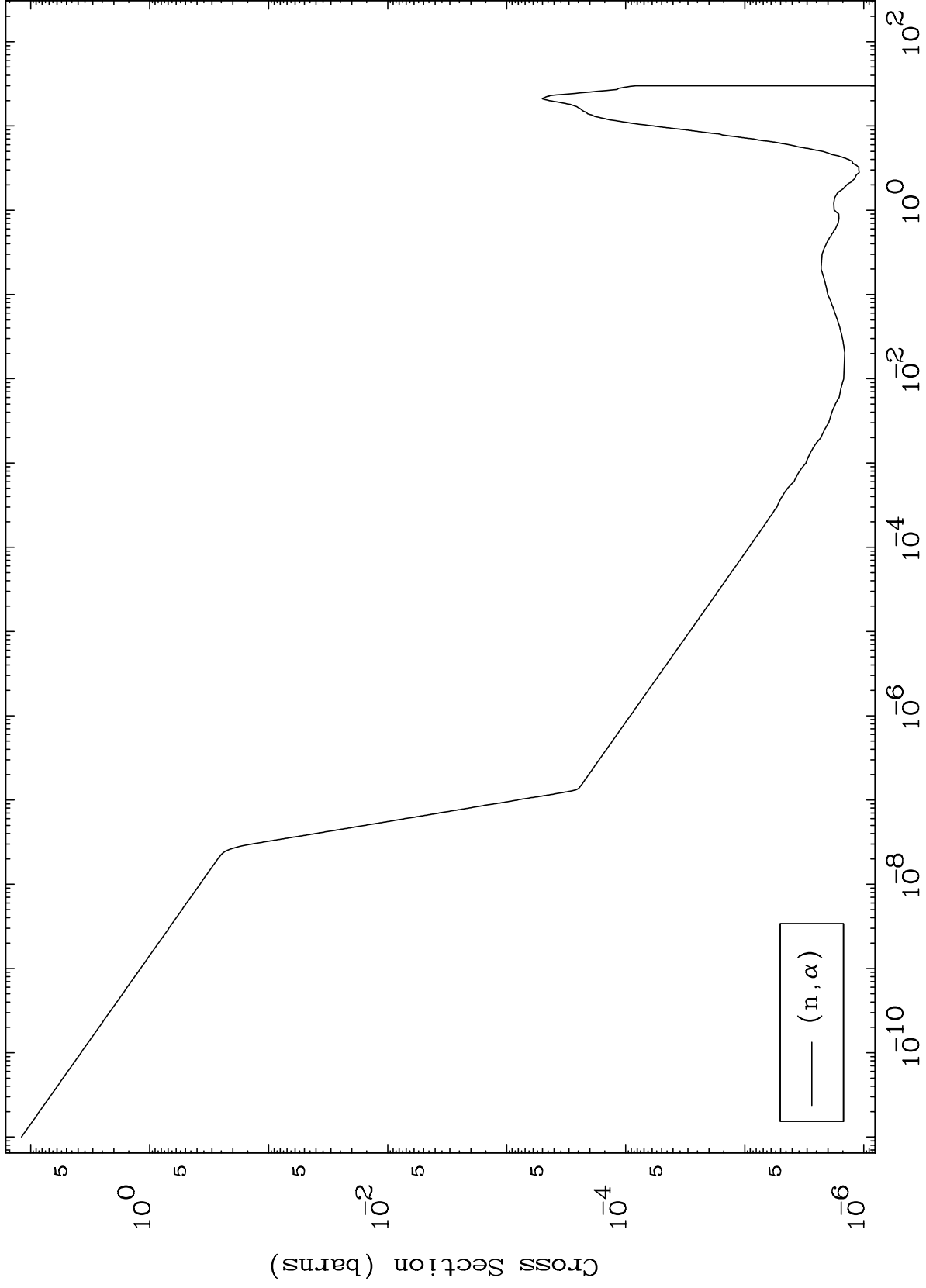
Incident Energy (MeV)

95-Am-236

MAT 9529

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

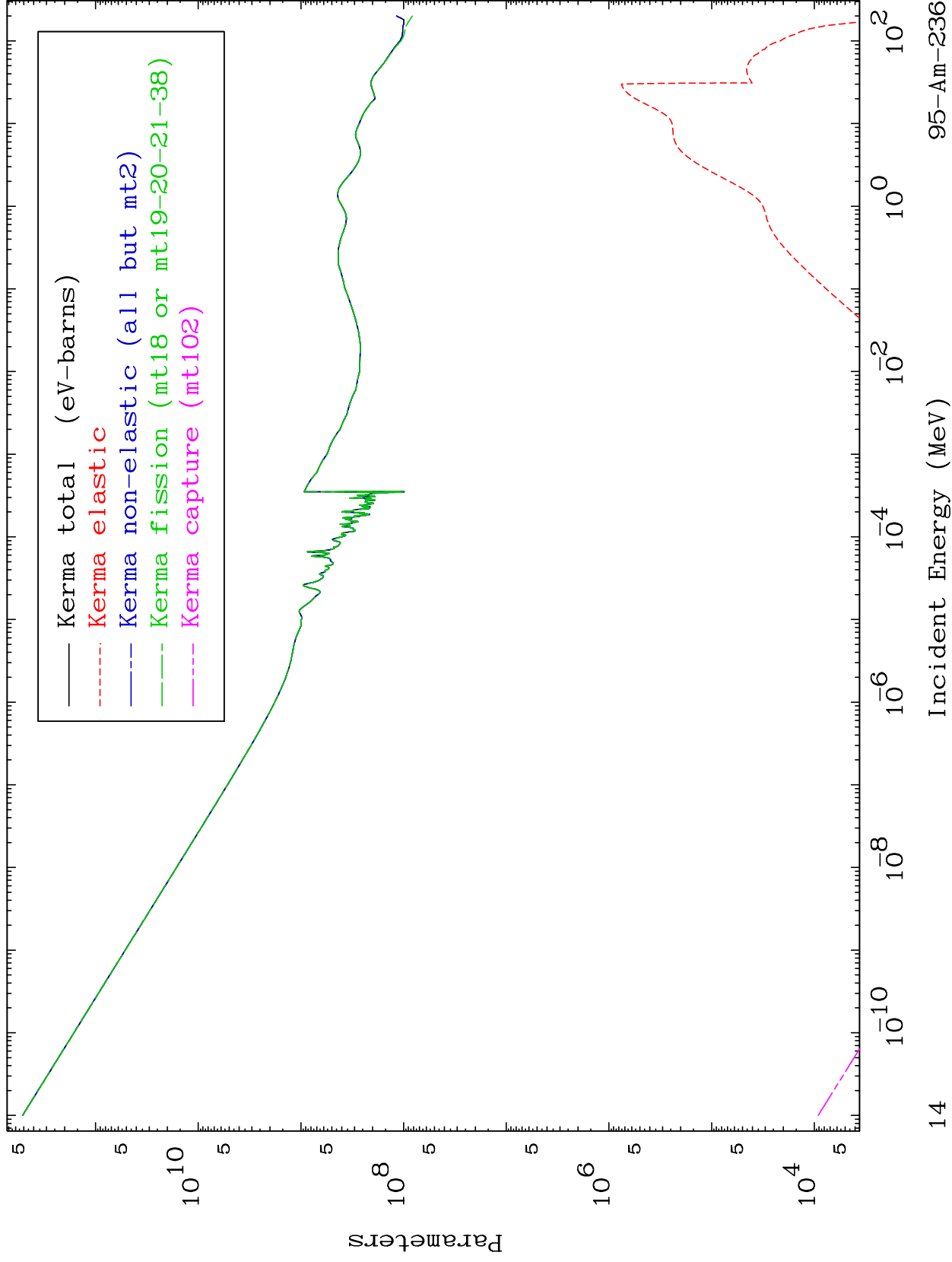
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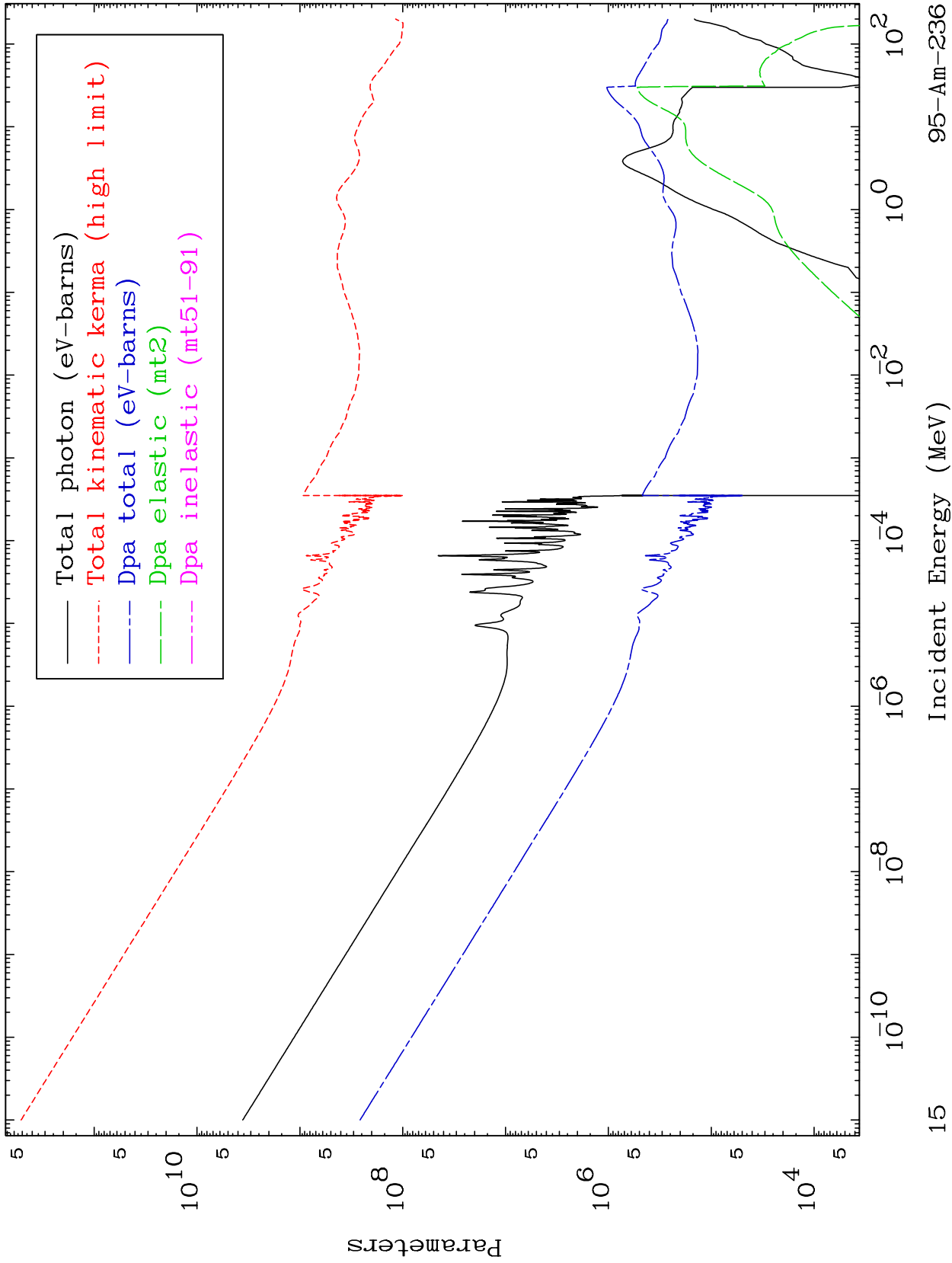


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Incident Energy (MeV)

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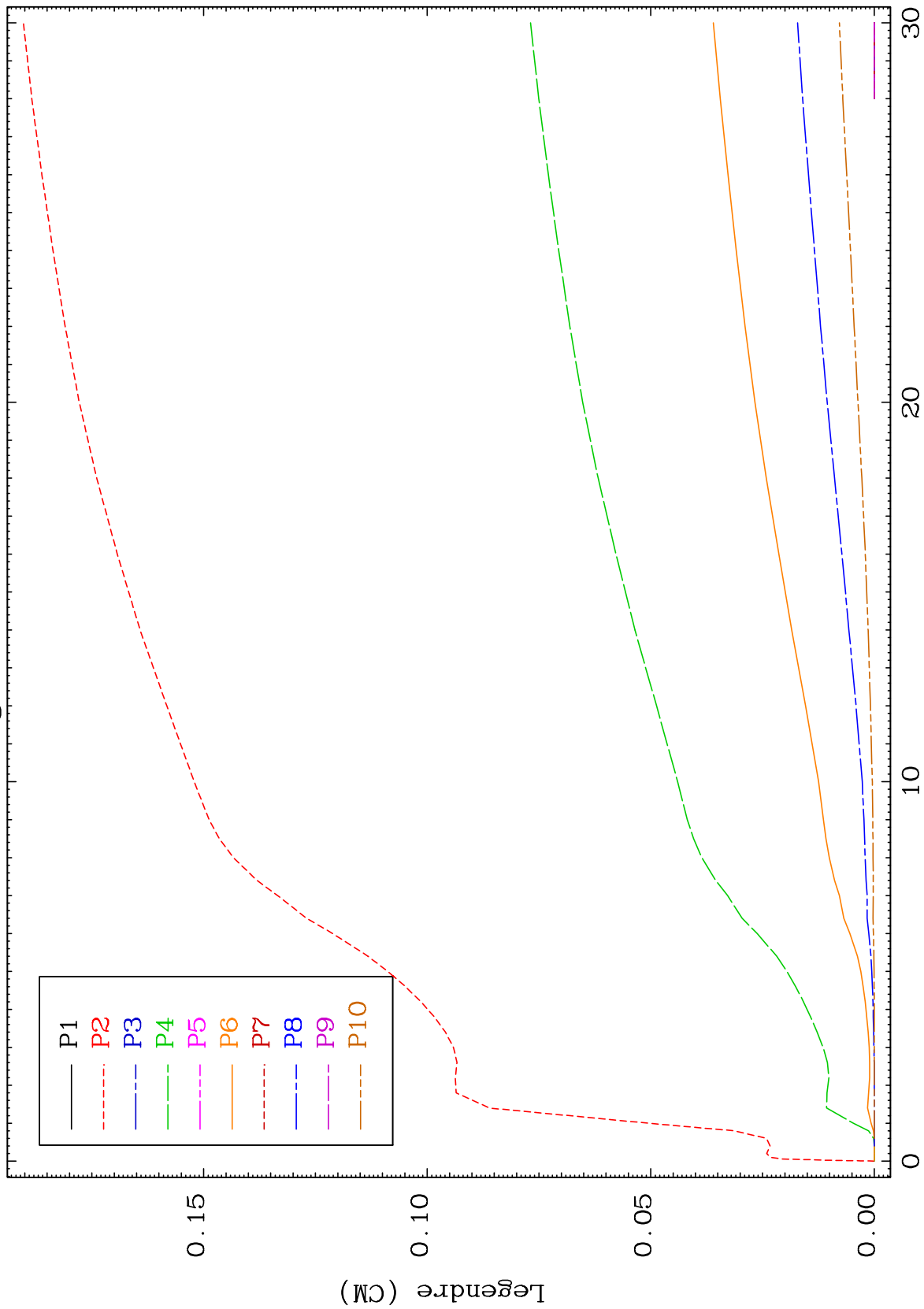




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Elastic Legendre Coefficients

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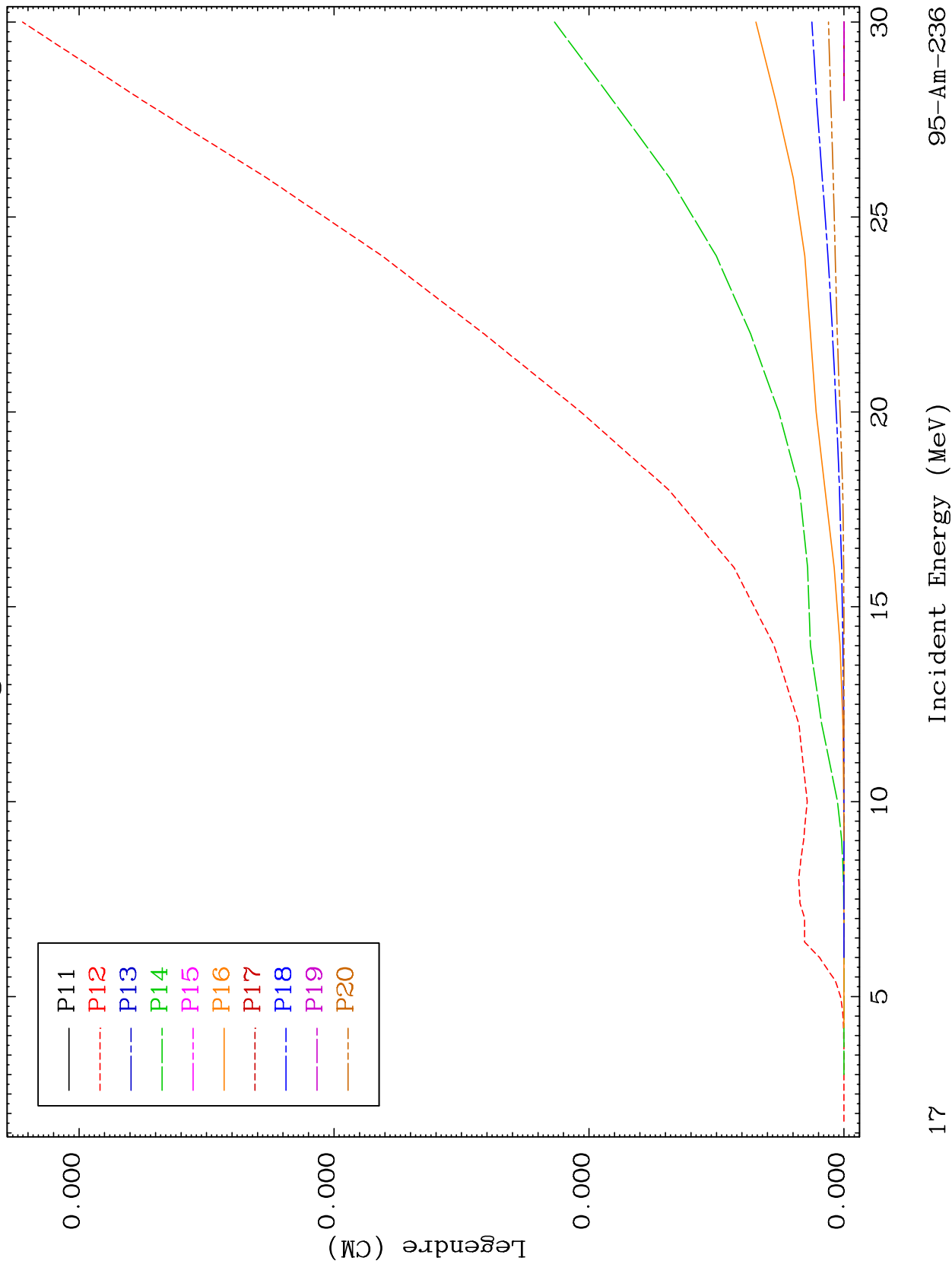
Incident Energy (MeV)

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### Elastic Legendre Coefficients

95-Am-236



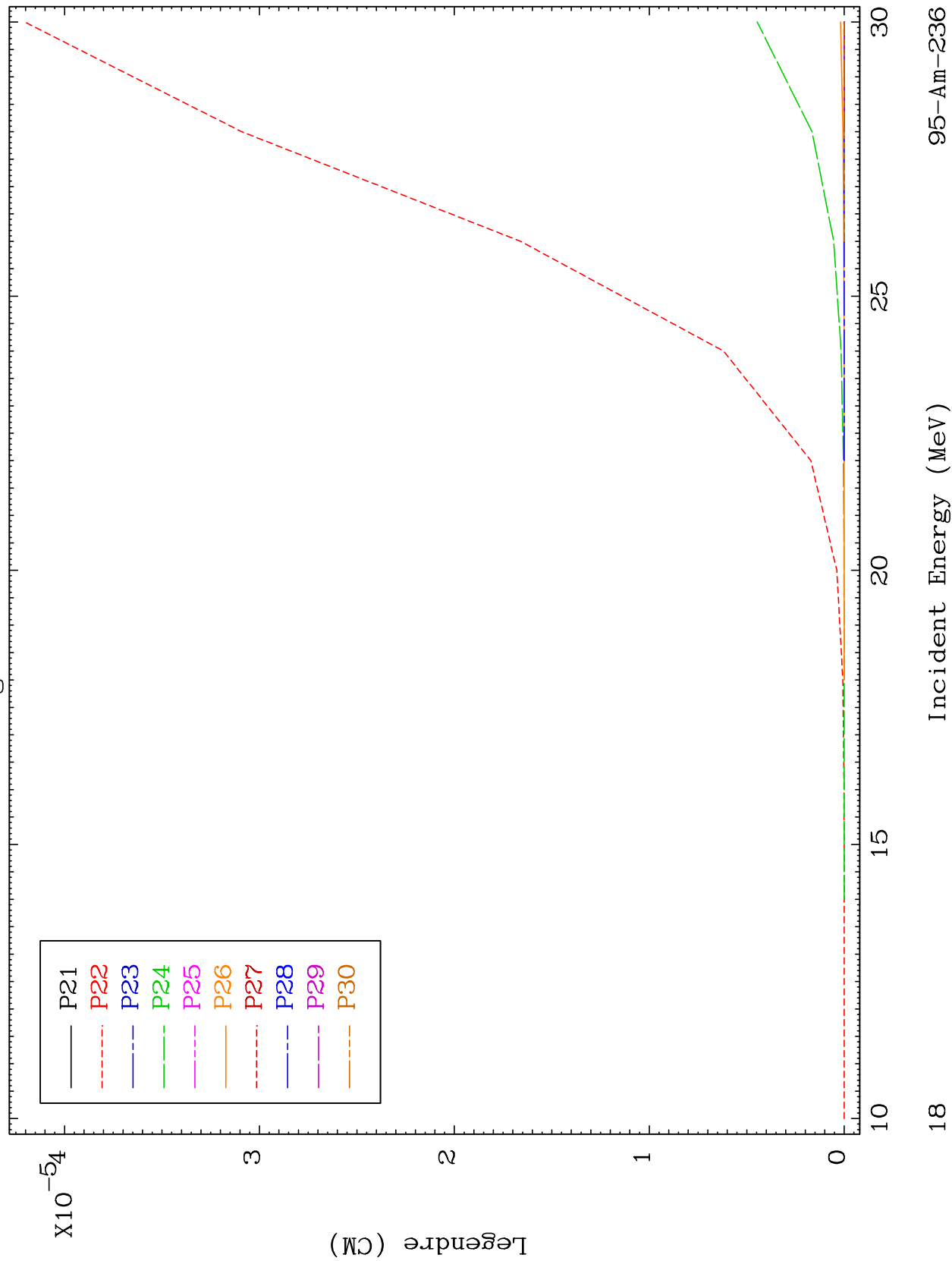
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### Elastic Legendre Coefficients

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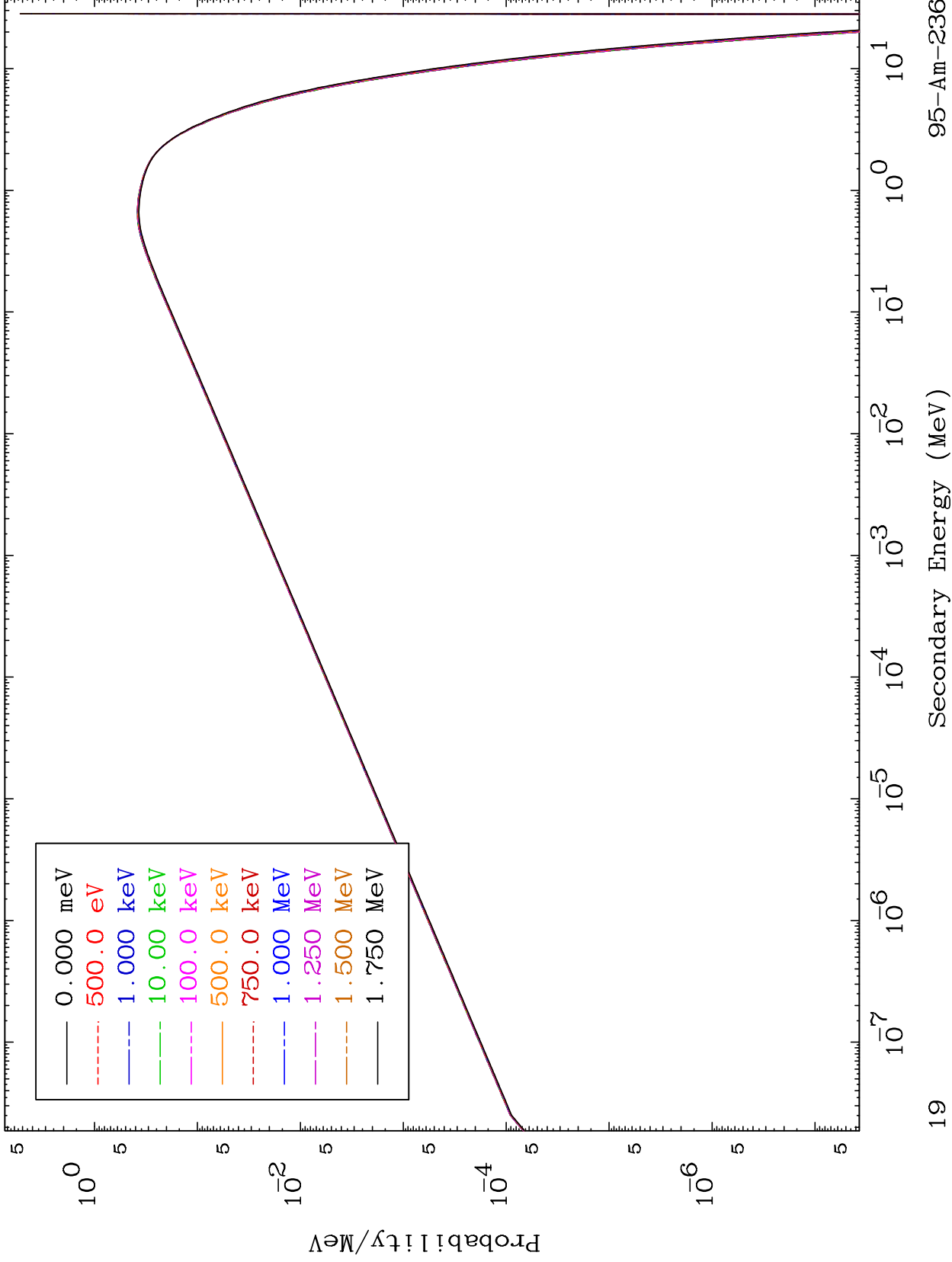


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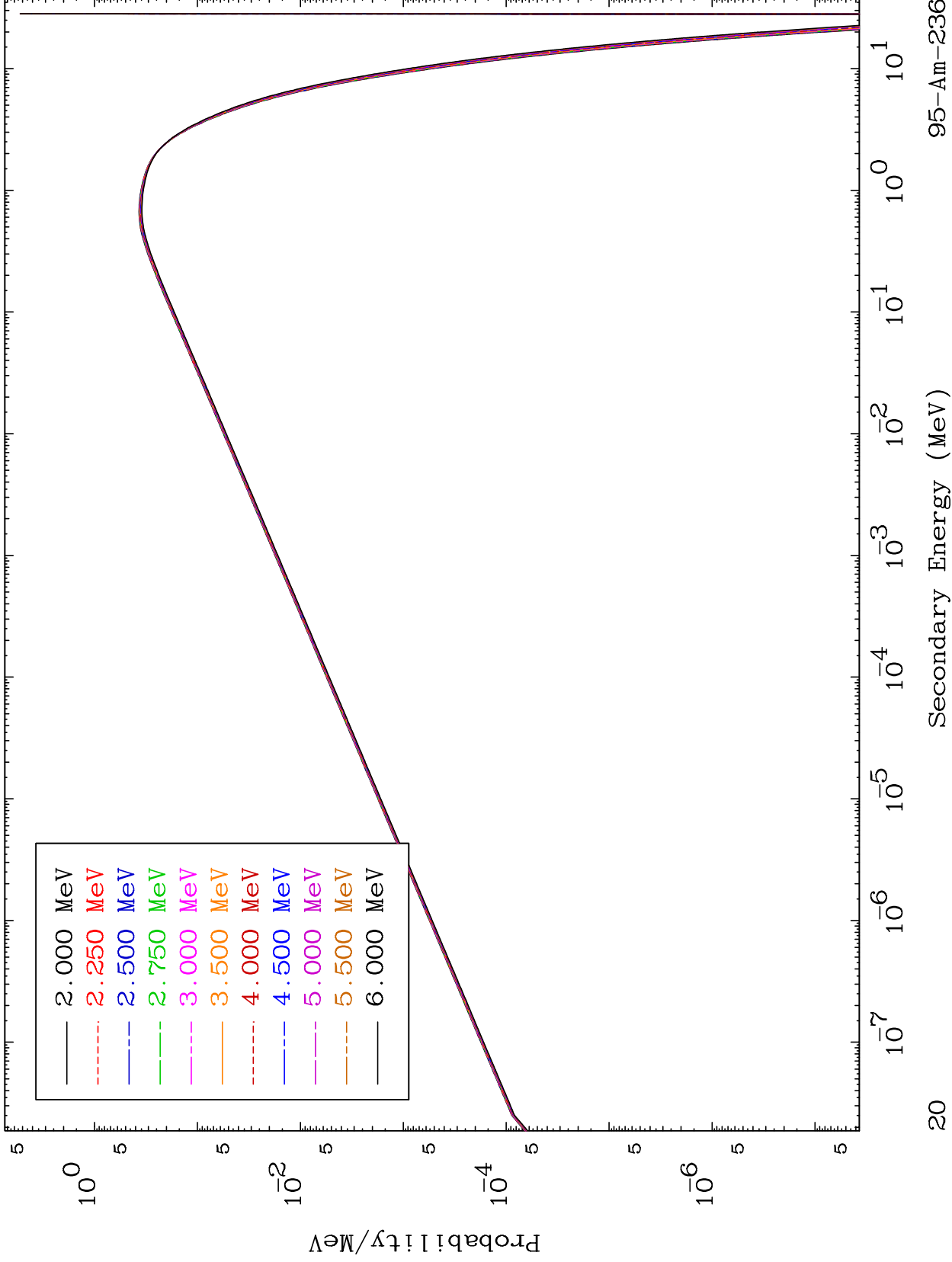
Incident Energy (MeV)

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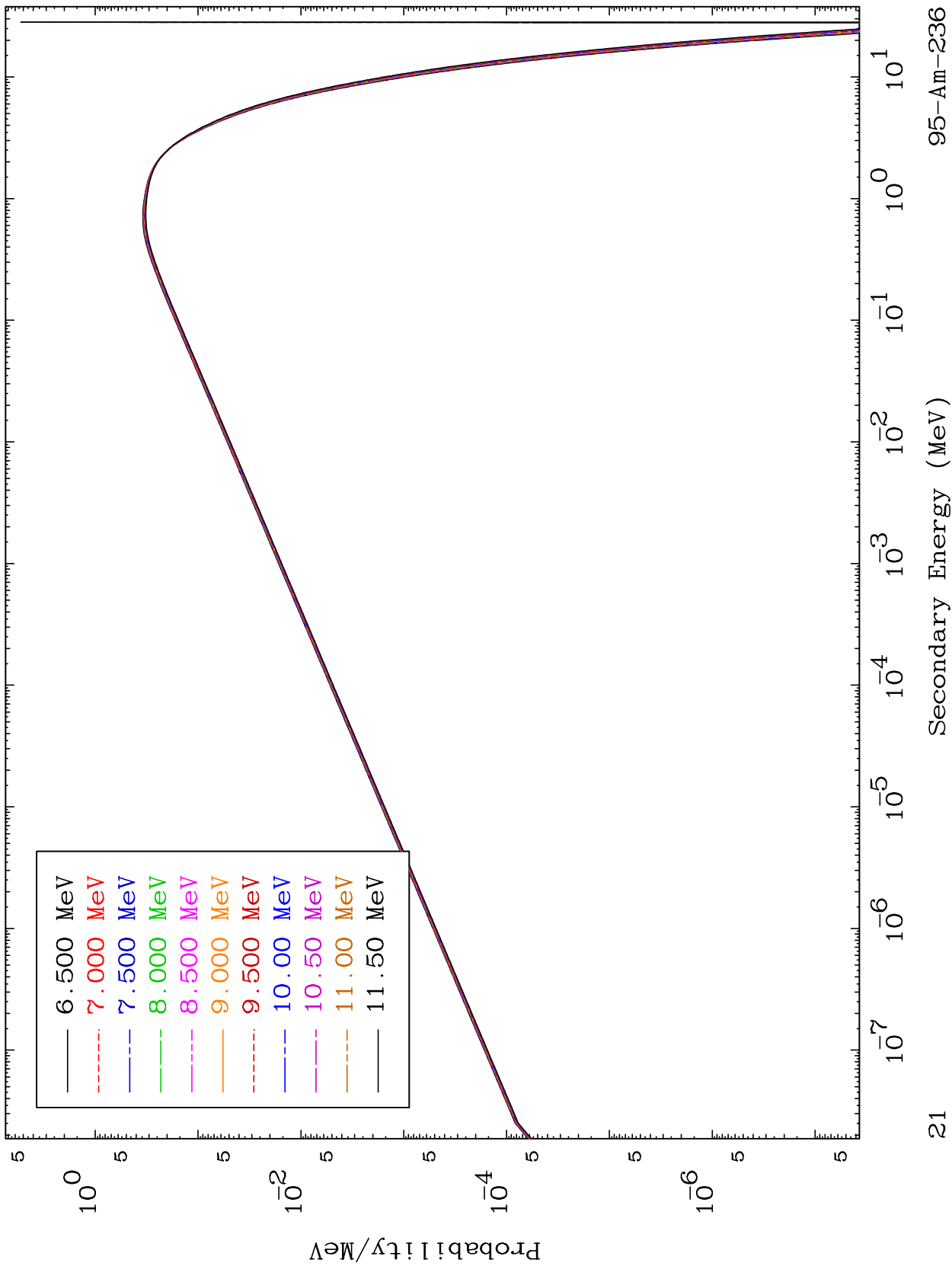
### Fission Energy Distribution



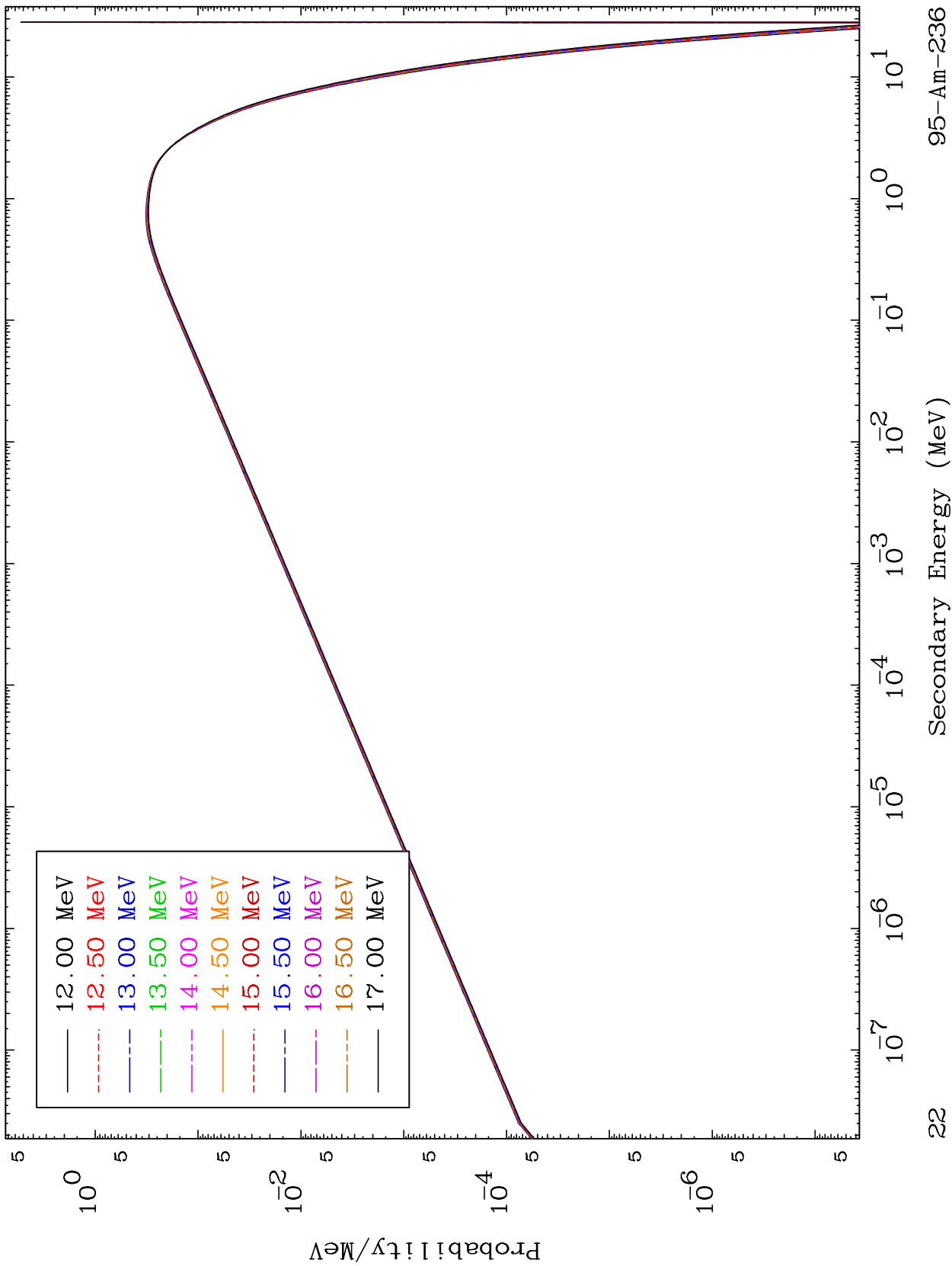
### Fission Energy Distribution



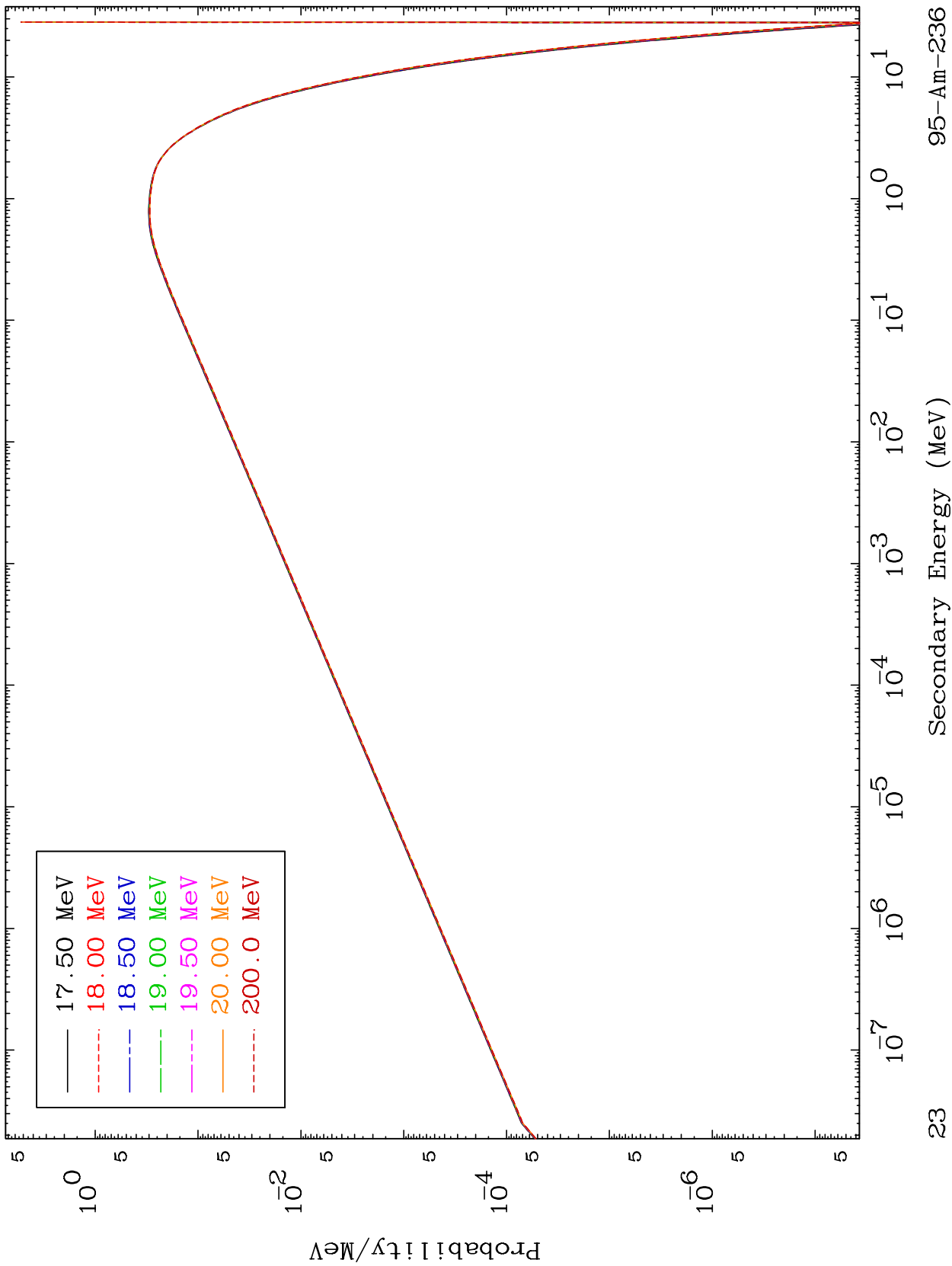
### Fission Energy Distribution



### Fission Energy Distribution



Fission  
Energy Distribution

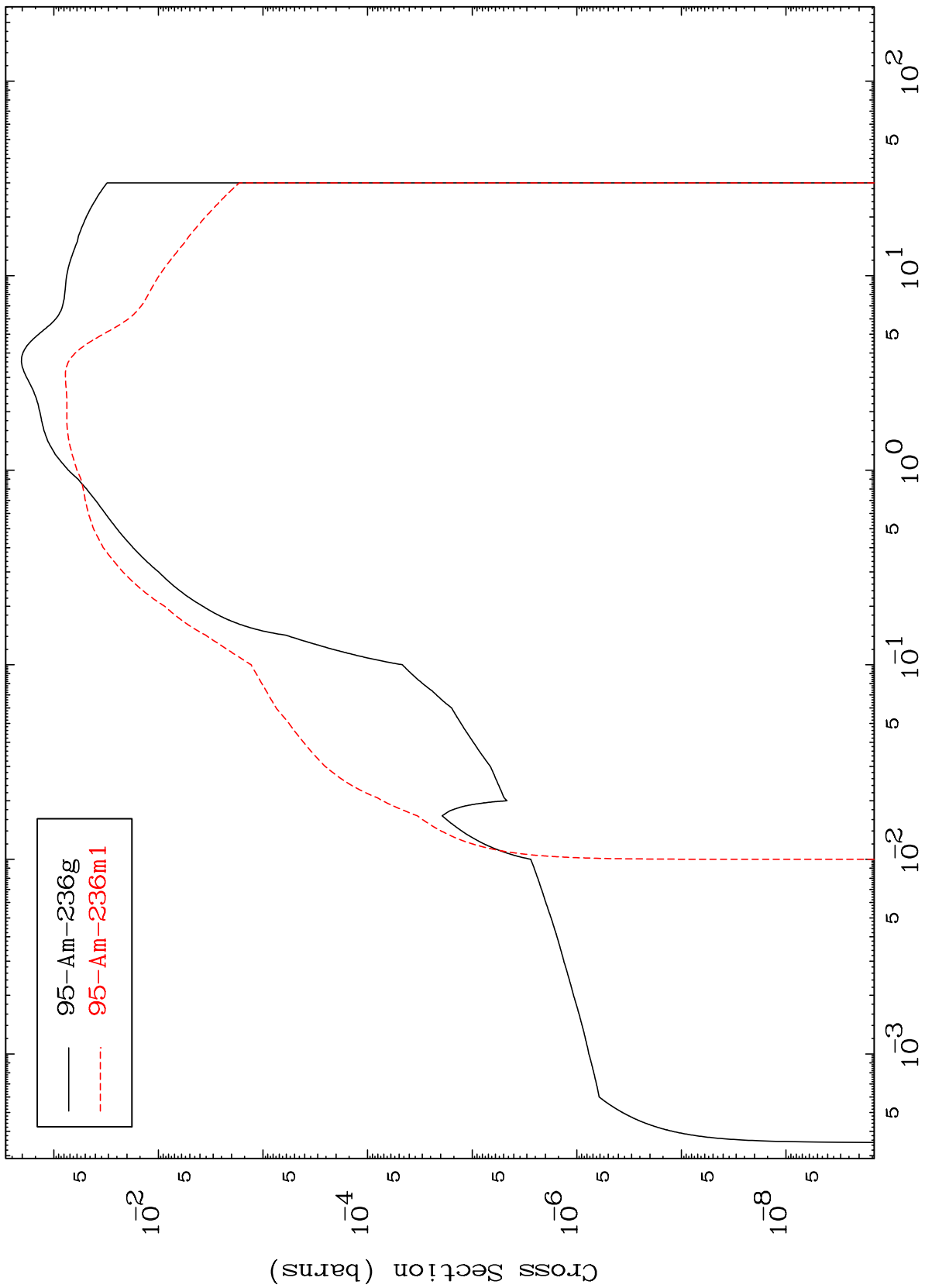




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Inelastic  
Radionuclide Production Cross Section



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Incident Energy (MeV)

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