

Program EVALPLOT
(Version 2015-2)

by

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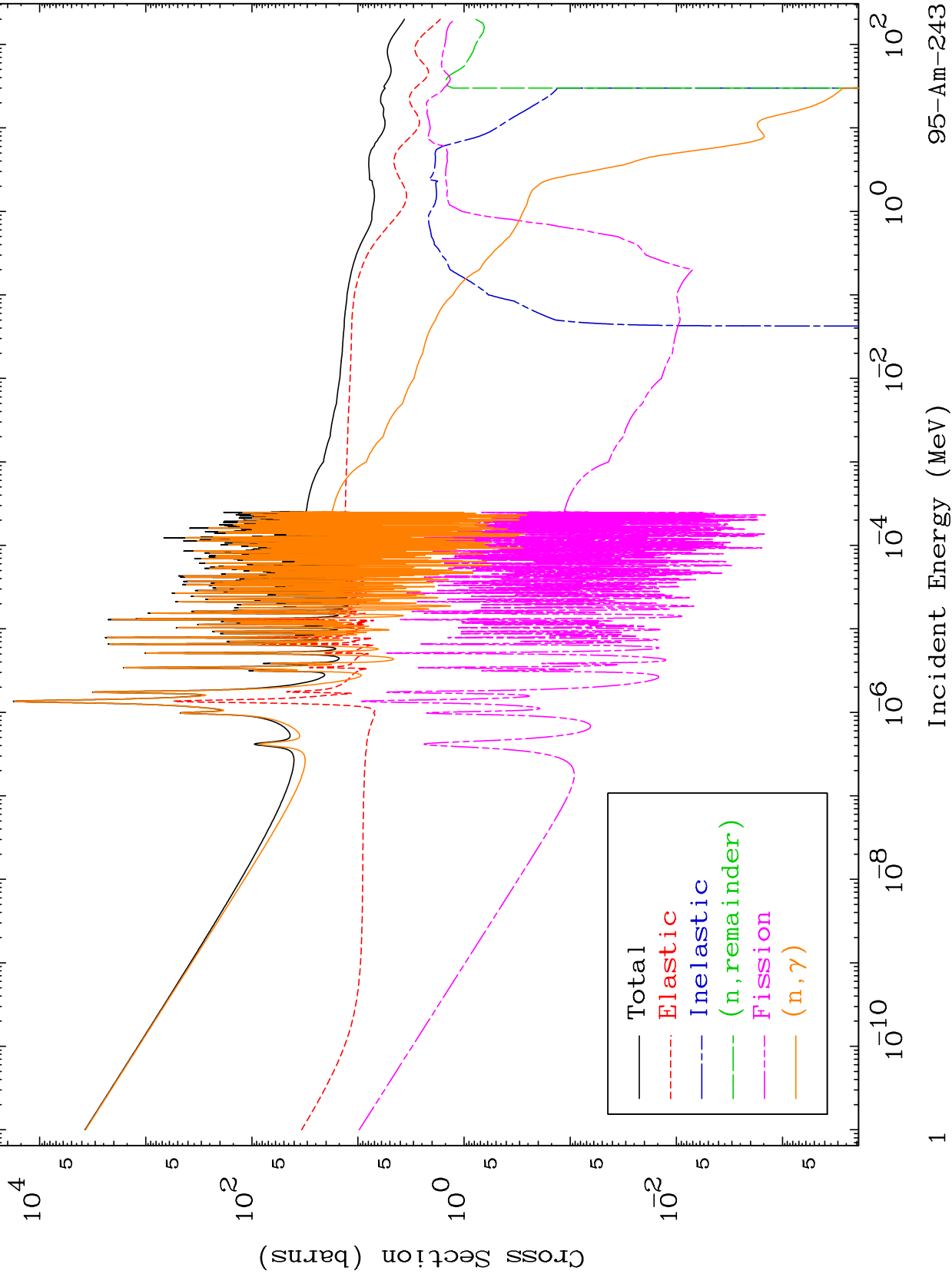
E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

MAT 9549

Major
293 Kelvin Cross Sections

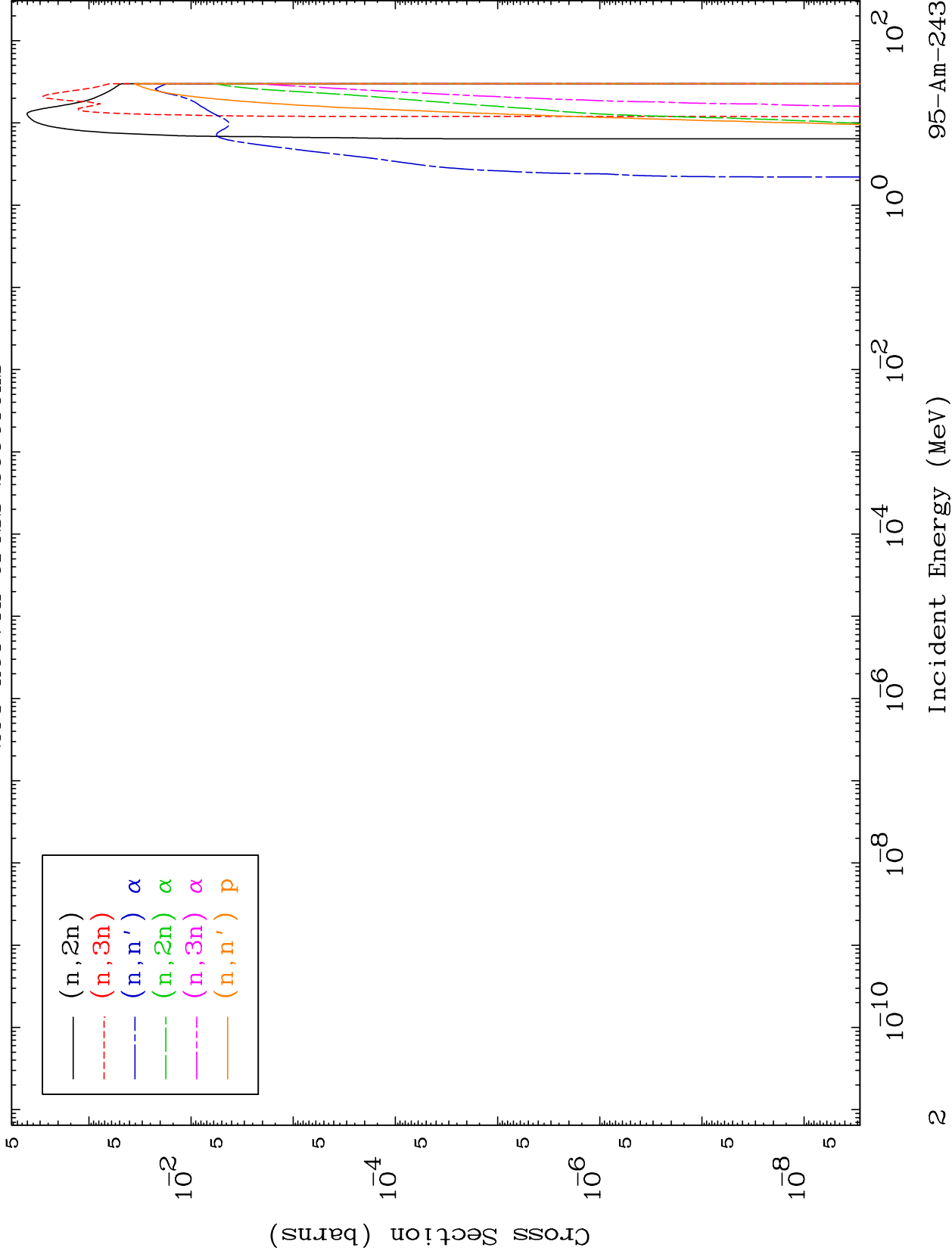
95-Am-243



MAT 9549

Neutron Production
293 Kelvin Cross Sections

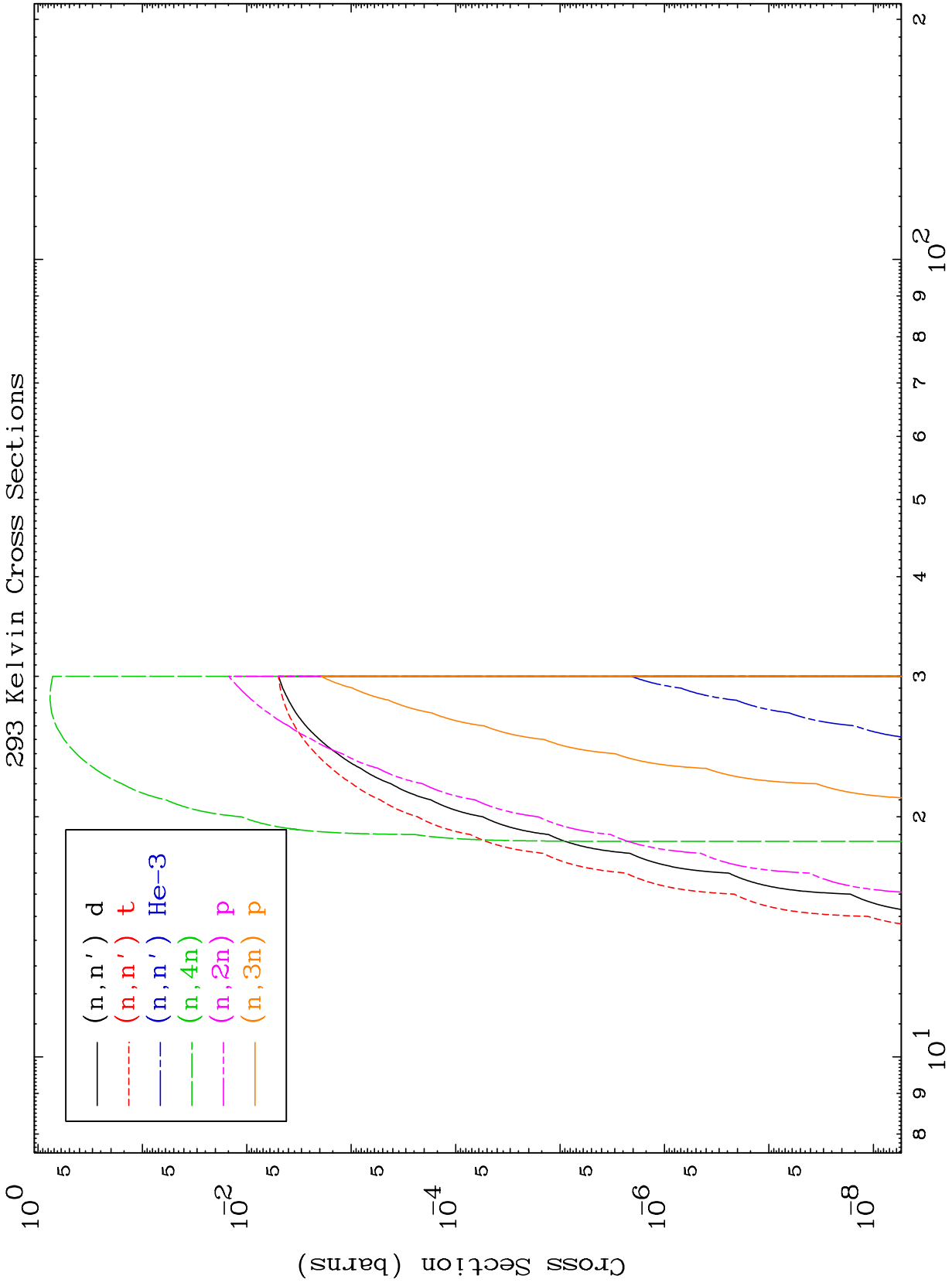
95-Am-243



MAT 9549

Neutron Production
293 Kelvin Cross Sections

95-Am-243



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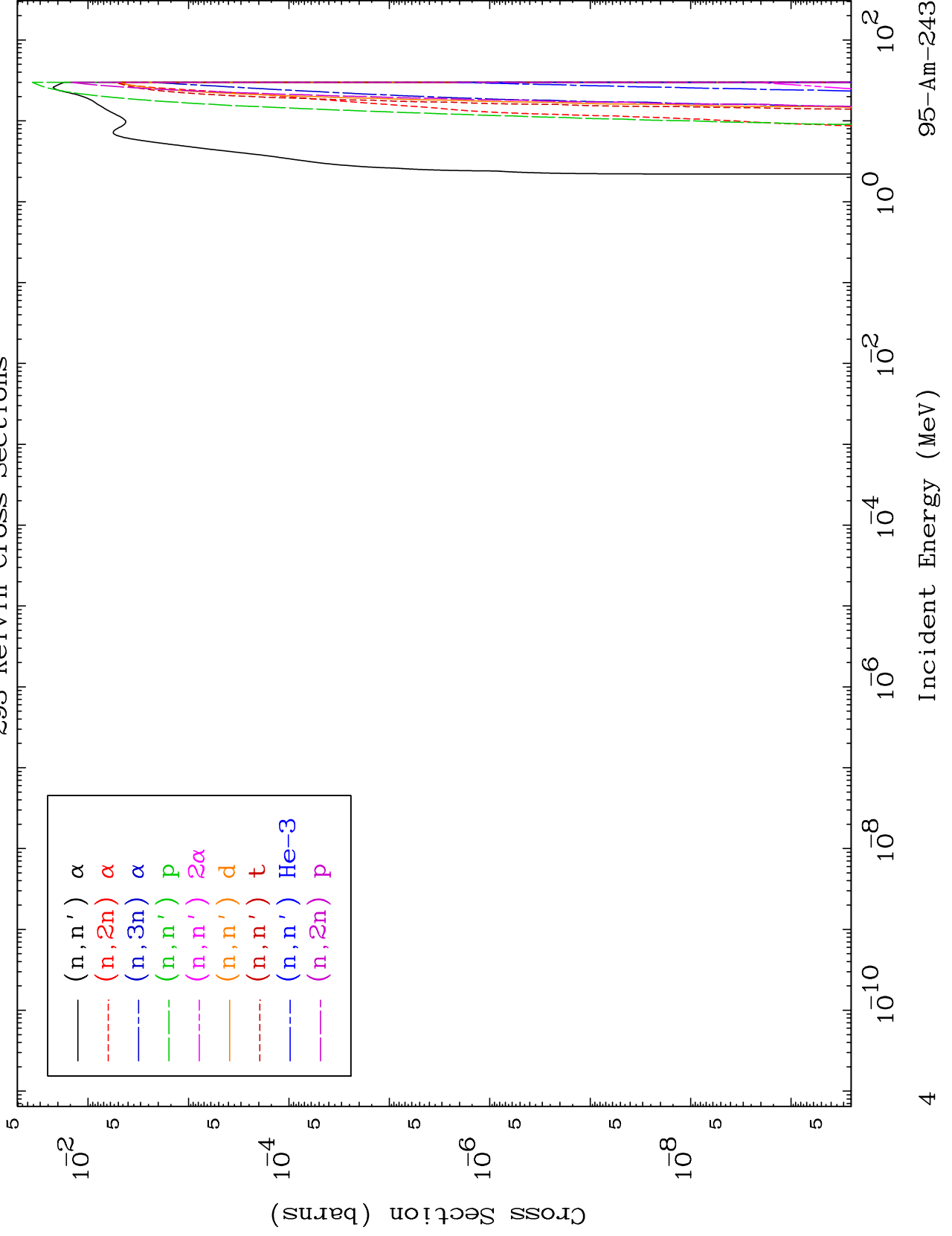
Incident Energy (MeV)

95-Am-243

MAT 9549

Charged Particle
293 Kelvin Cross Sections

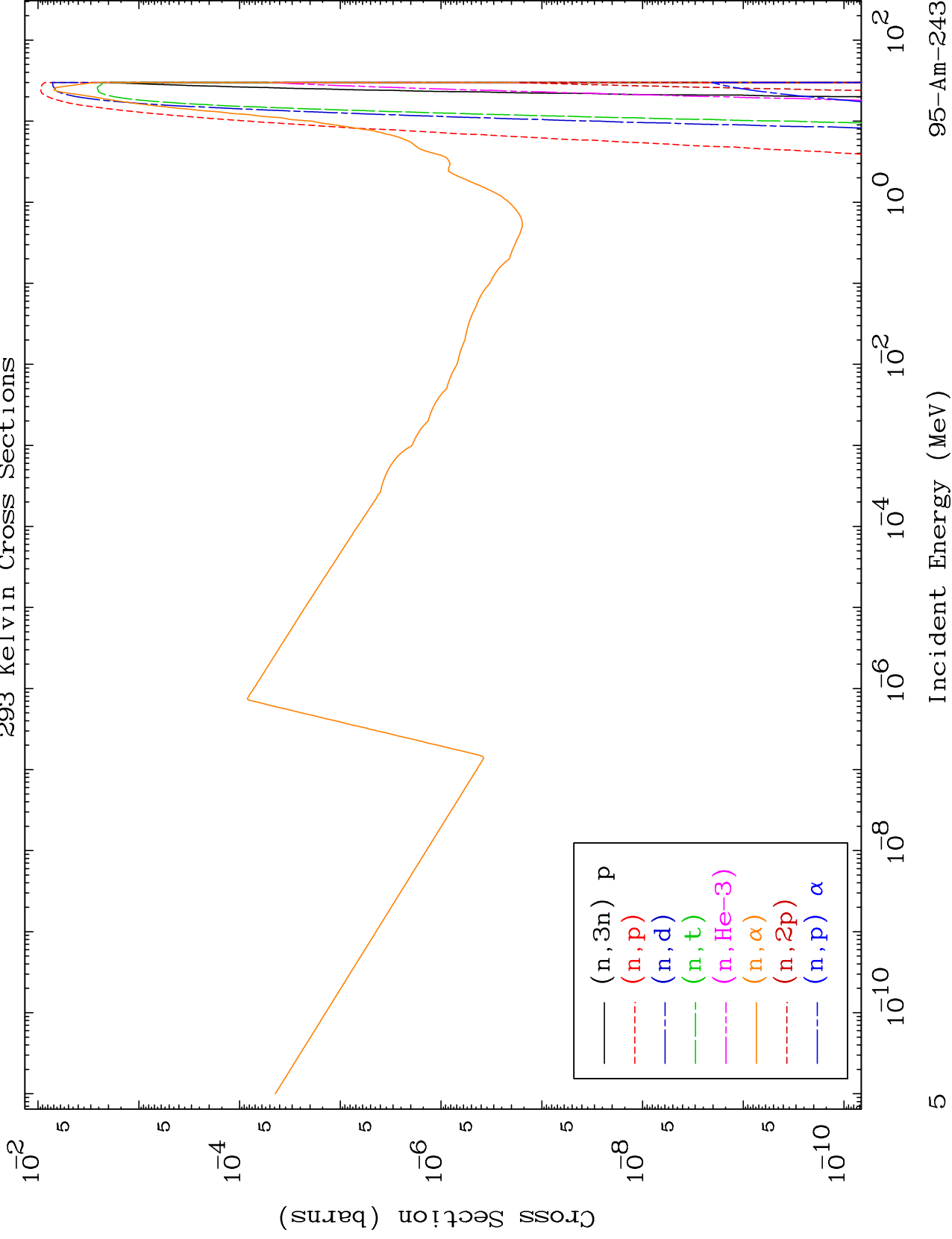
95-Am-243



MAT 9549

Charged Particle
293 Kelvin Cross Sections

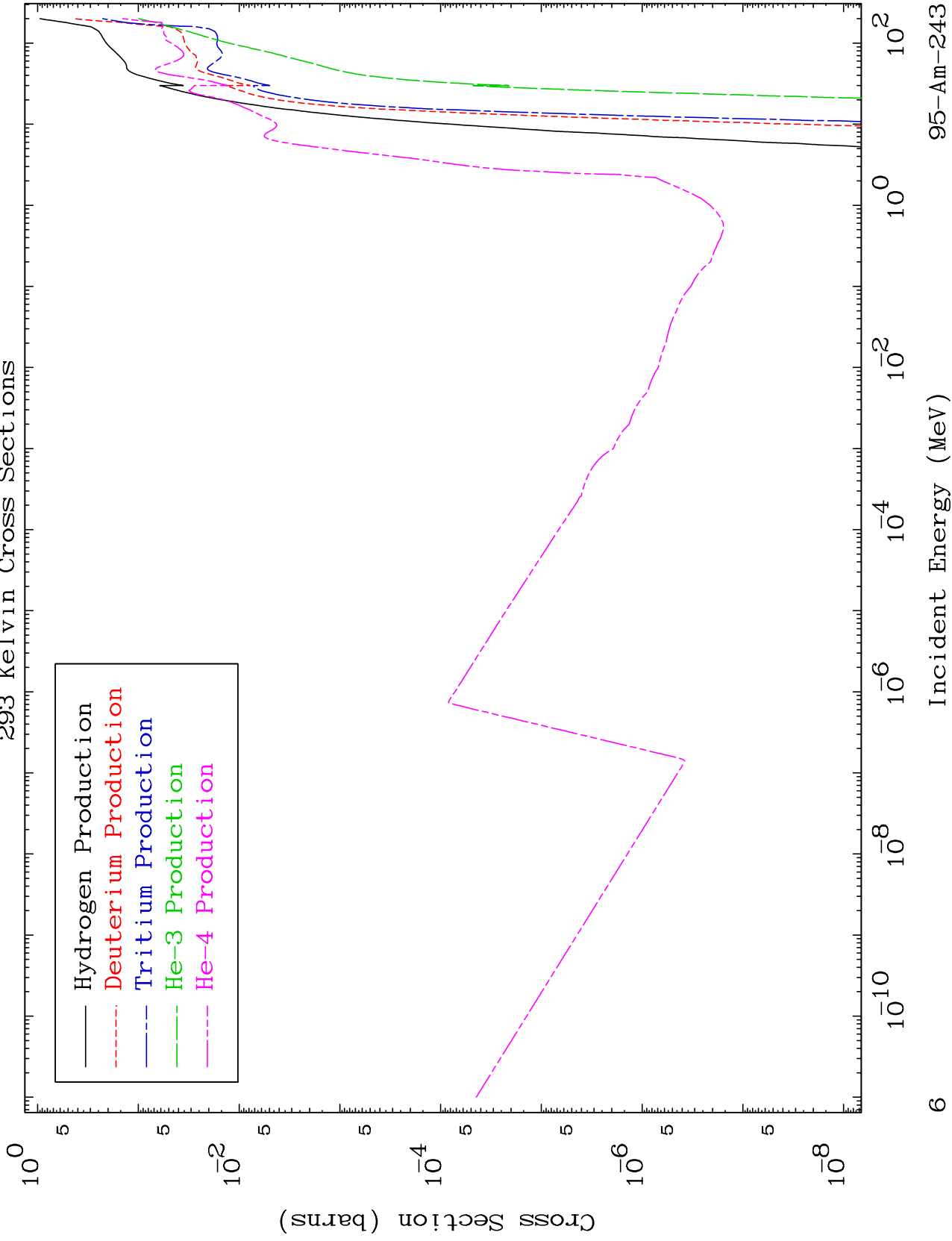
95-Am-243



MAT 9549

Particle Production
293 Kelvin Cross Sections

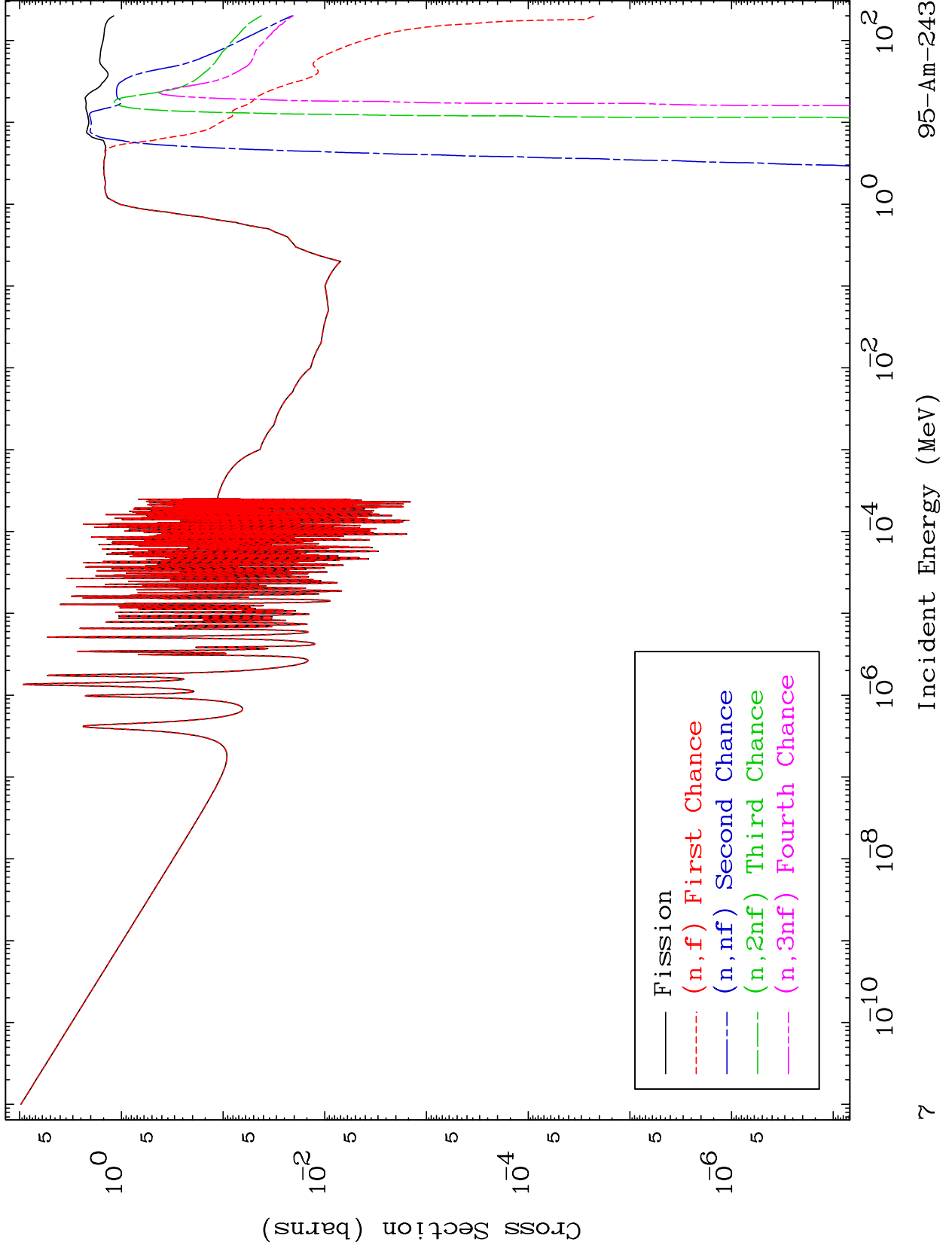
95-Am-243



MAT 9549

Fission
293 Kelvin Cross Sections

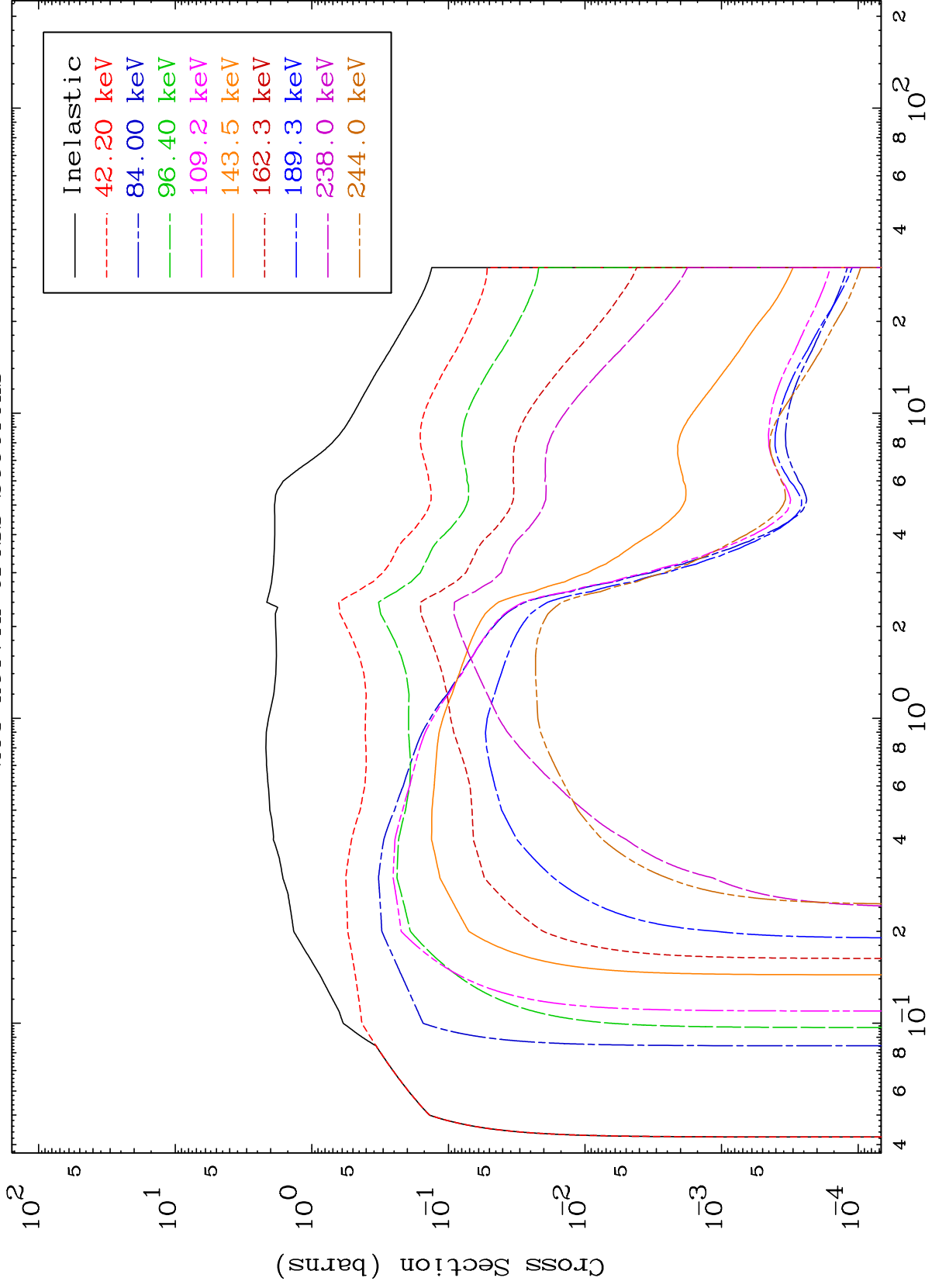
95-Am-243



MAT 9549

(n,n') Level
293 Kelvin Cross Sections

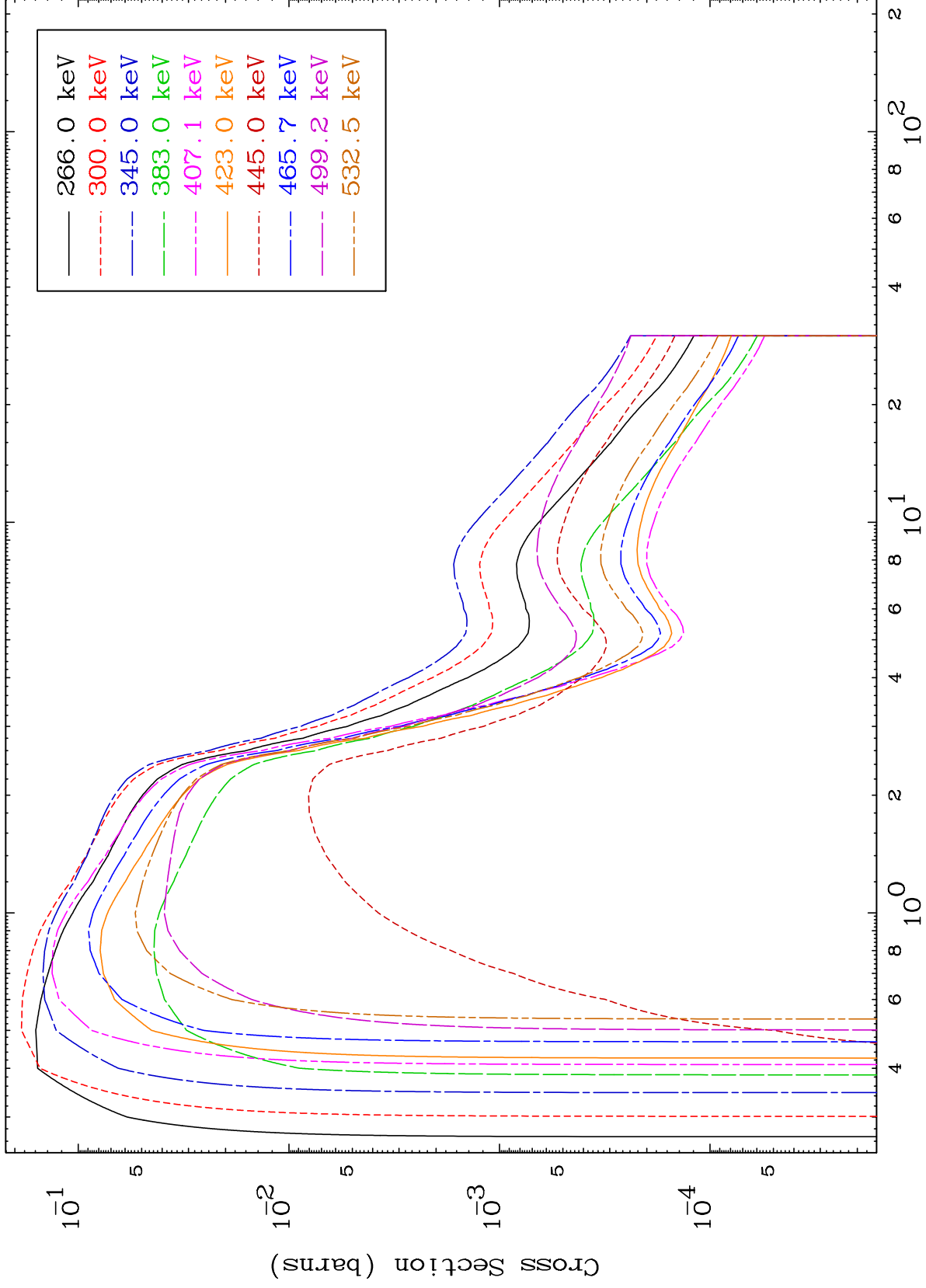
95-Am-243



MAT 9549

(n,n') Level
293 Kelvin Cross Sections

95-Am-243



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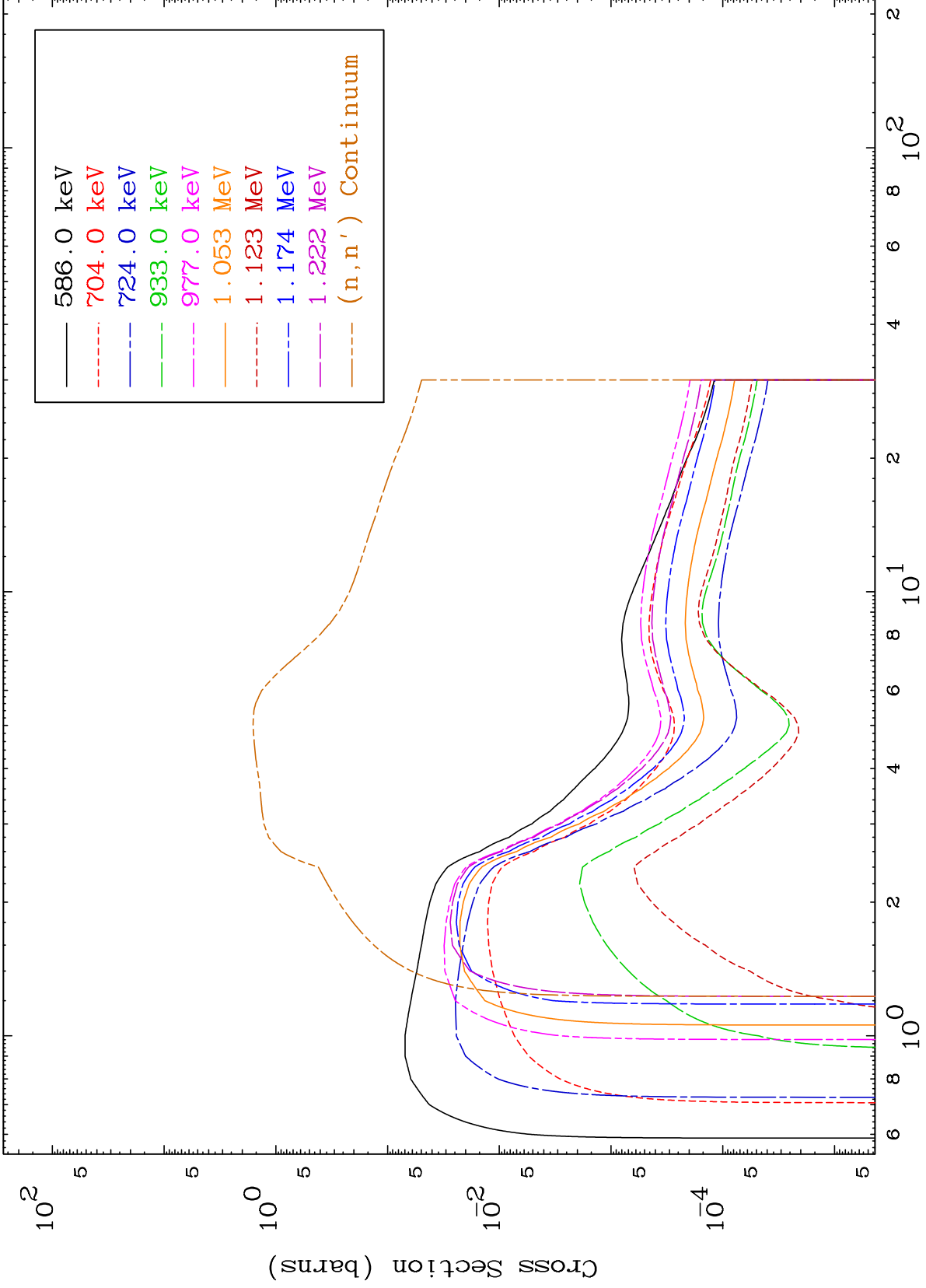
Incident Energy (MeV)

95-Am-243

MAT 9549

(n,n') Level
293 Kelvin Cross Sections

95-Am-243



10

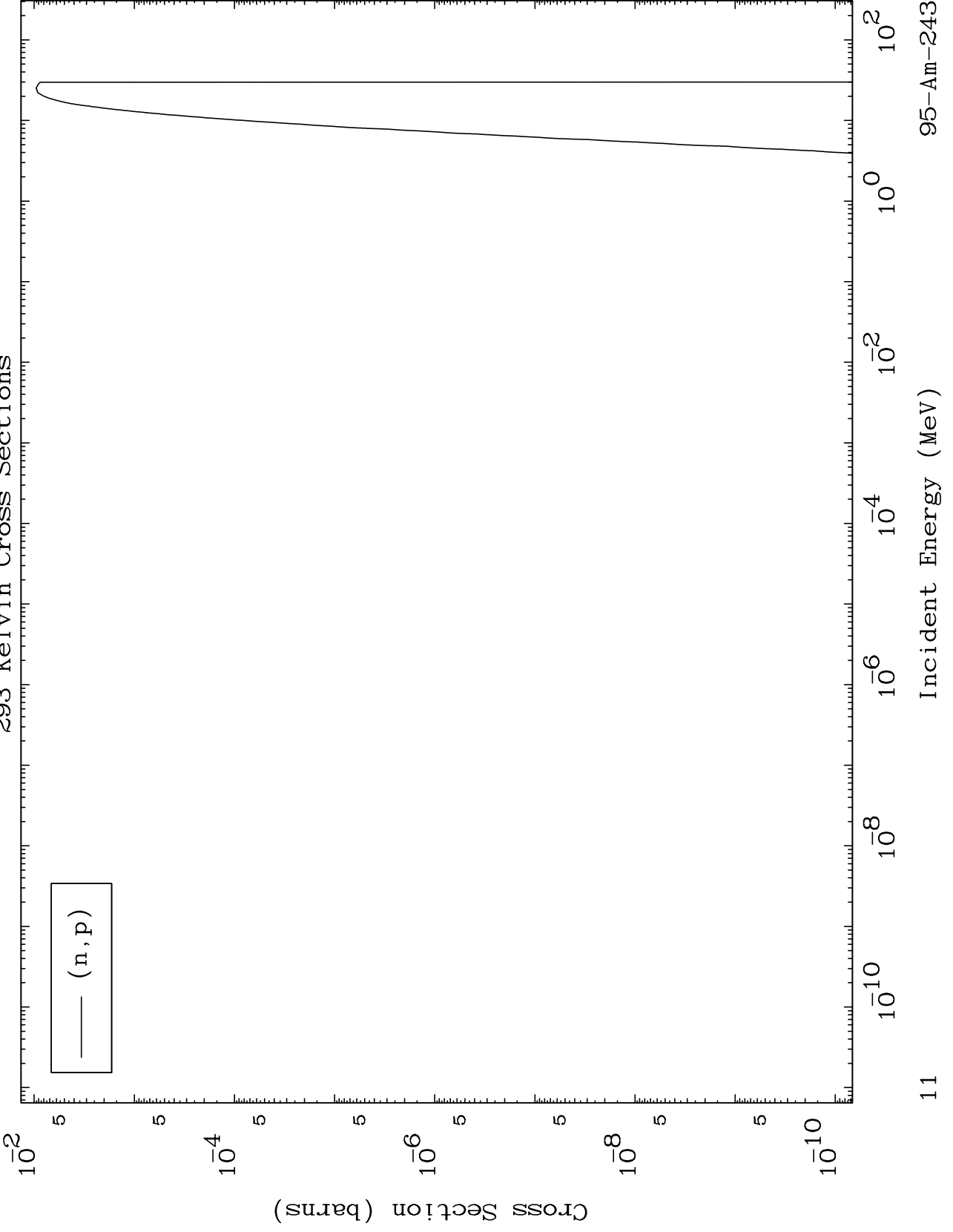
Incident Energy (MeV)

95-Am-243

MAT 9549

(n,p) Levels
293 Kelvin Cross Sections

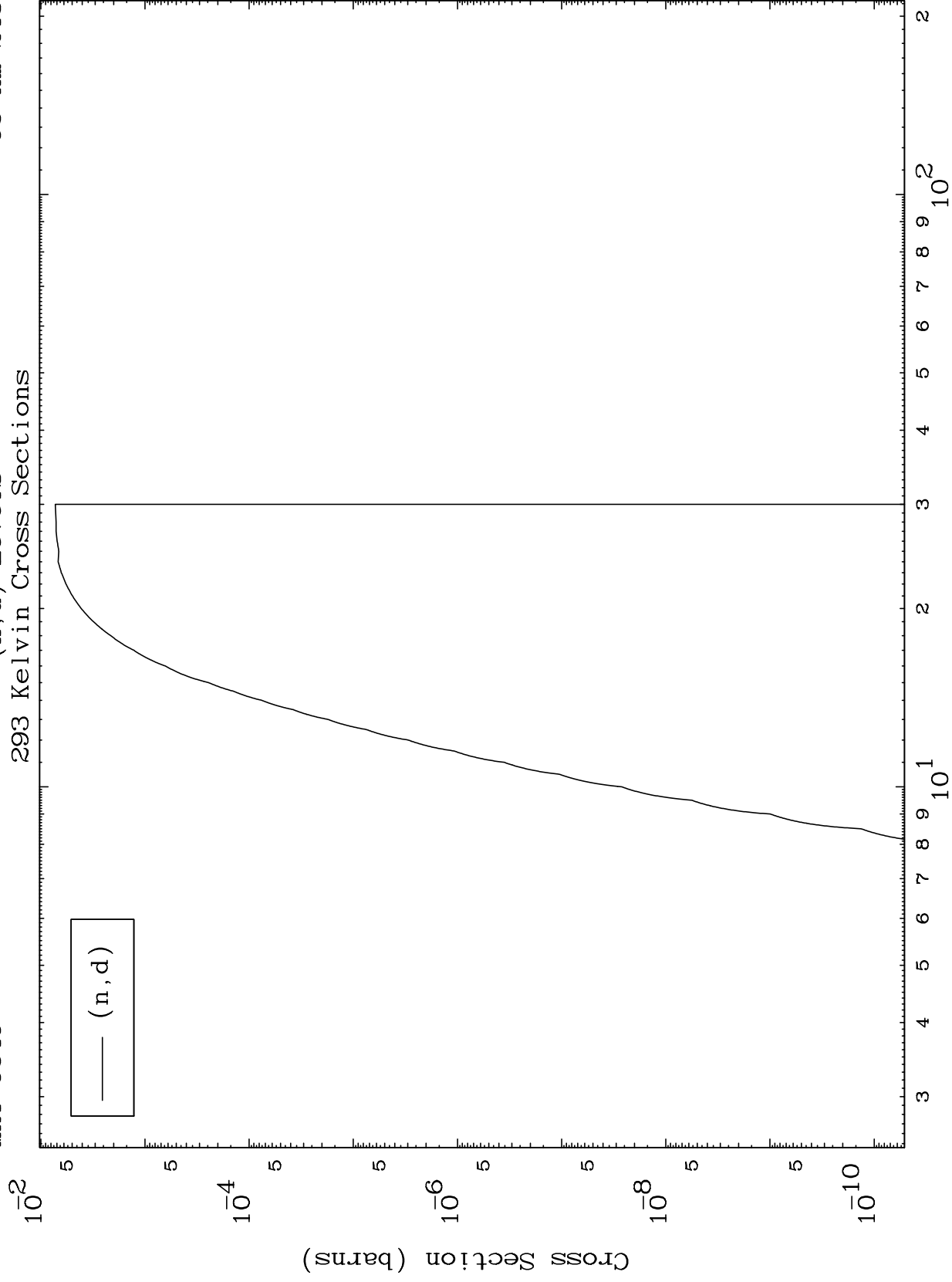
95-Am-243



MAT 9549

(n,d) Levels
293 Kelvin Cross Sections

95-Am-243



12

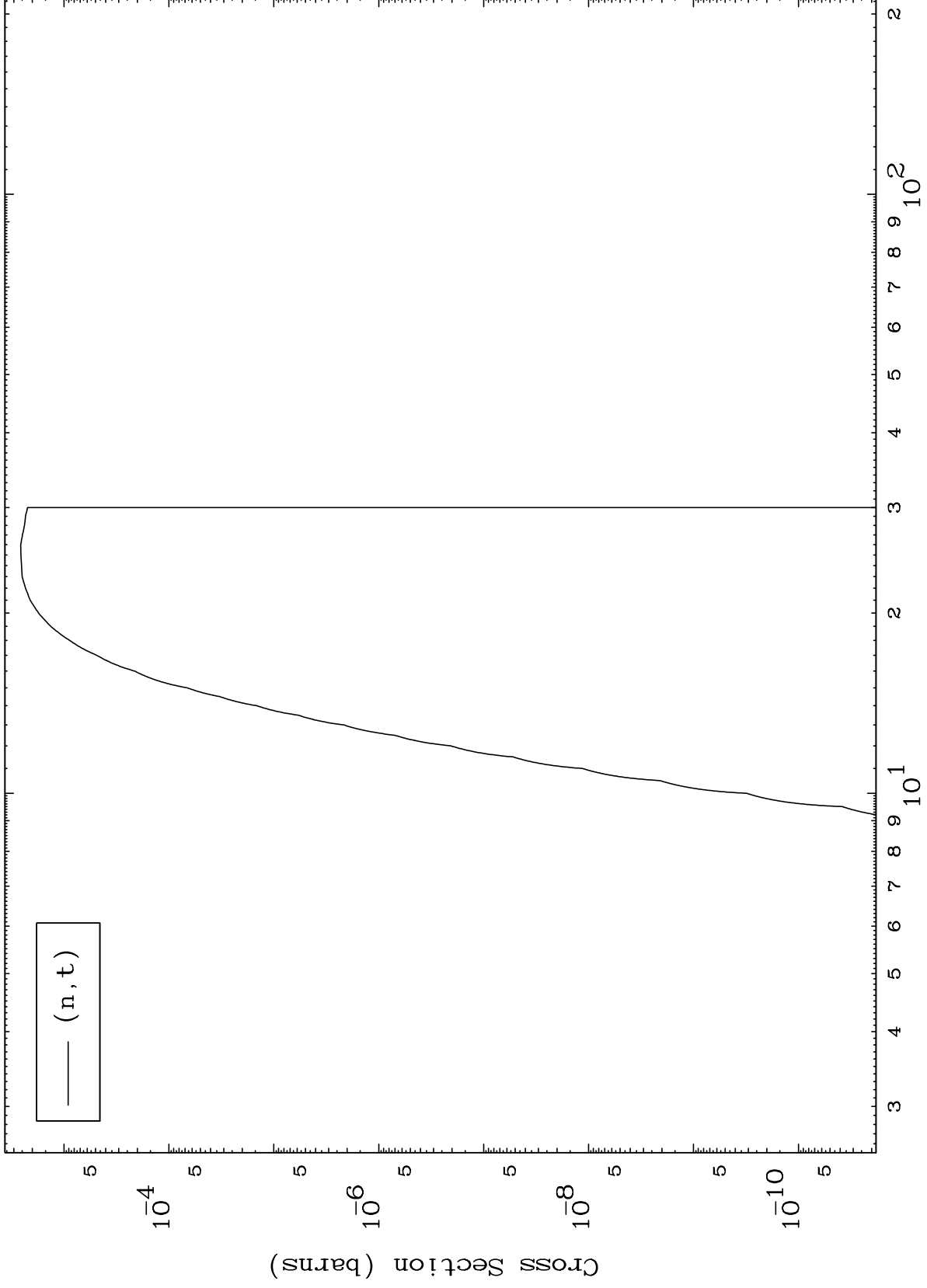
Incident Energy (MeV)

95-Am-243

MAT 9549

(n,t) Levels
293 Kelvin Cross Sections

95-Am-243



13

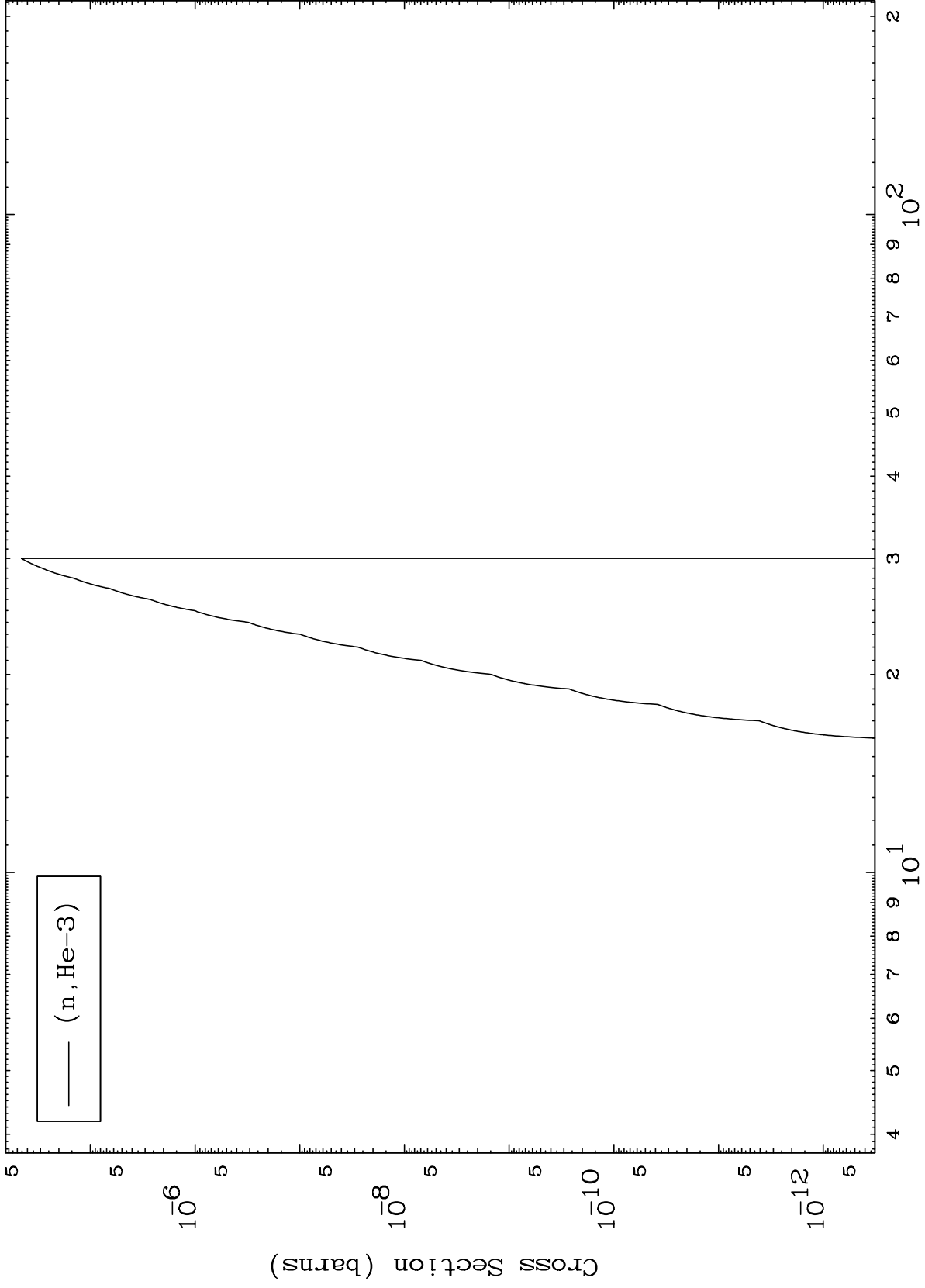
Incident Energy (MeV)

95-Am-243

MAT 9549

(n,He3) Levels
293 Kelvin Cross Sections

95-Am-243



14

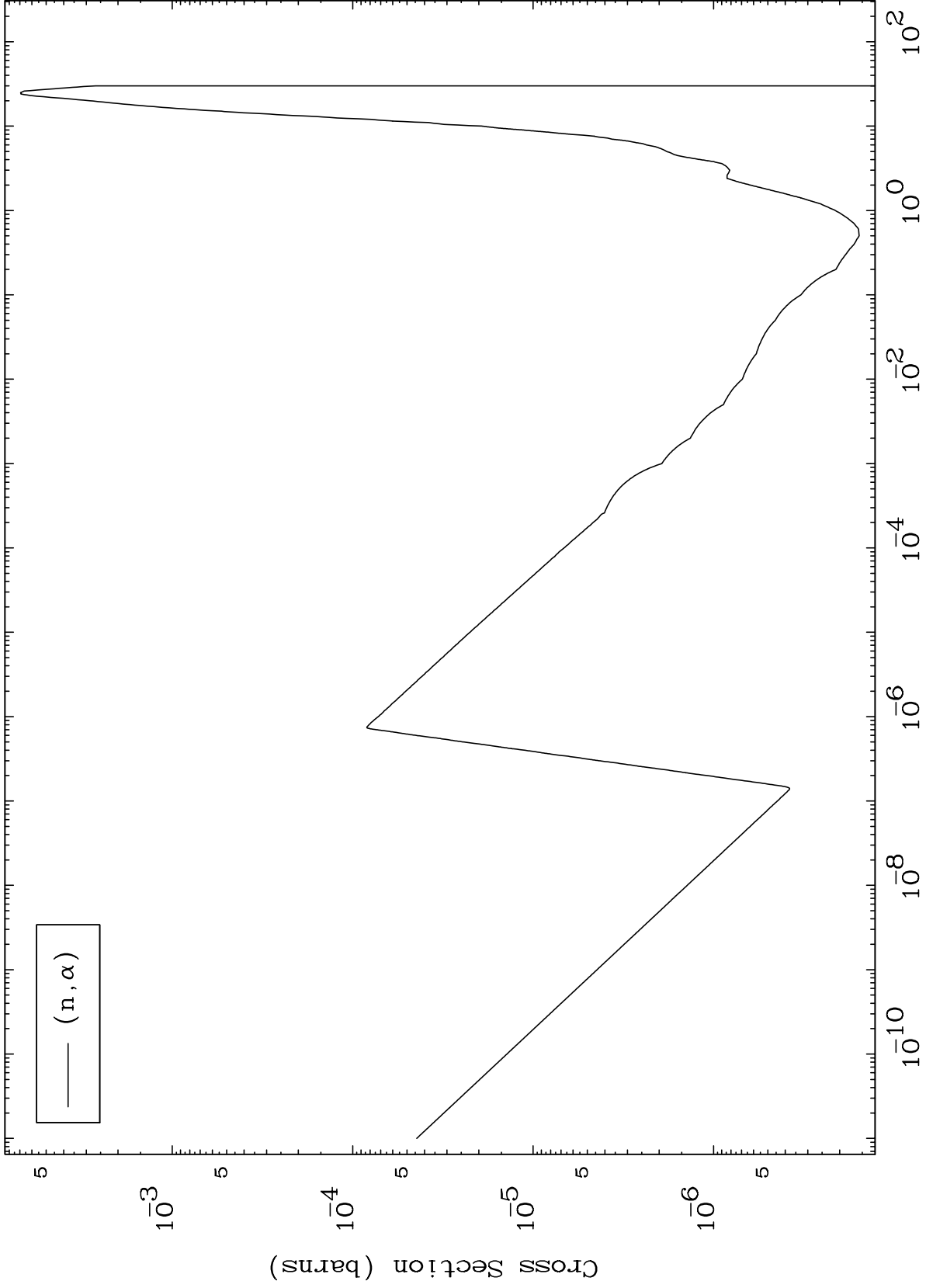
Incident Energy (MeV)

95-Am-243

MAT 9549

(n, α) Levels
293 Kelvin Cross Sections

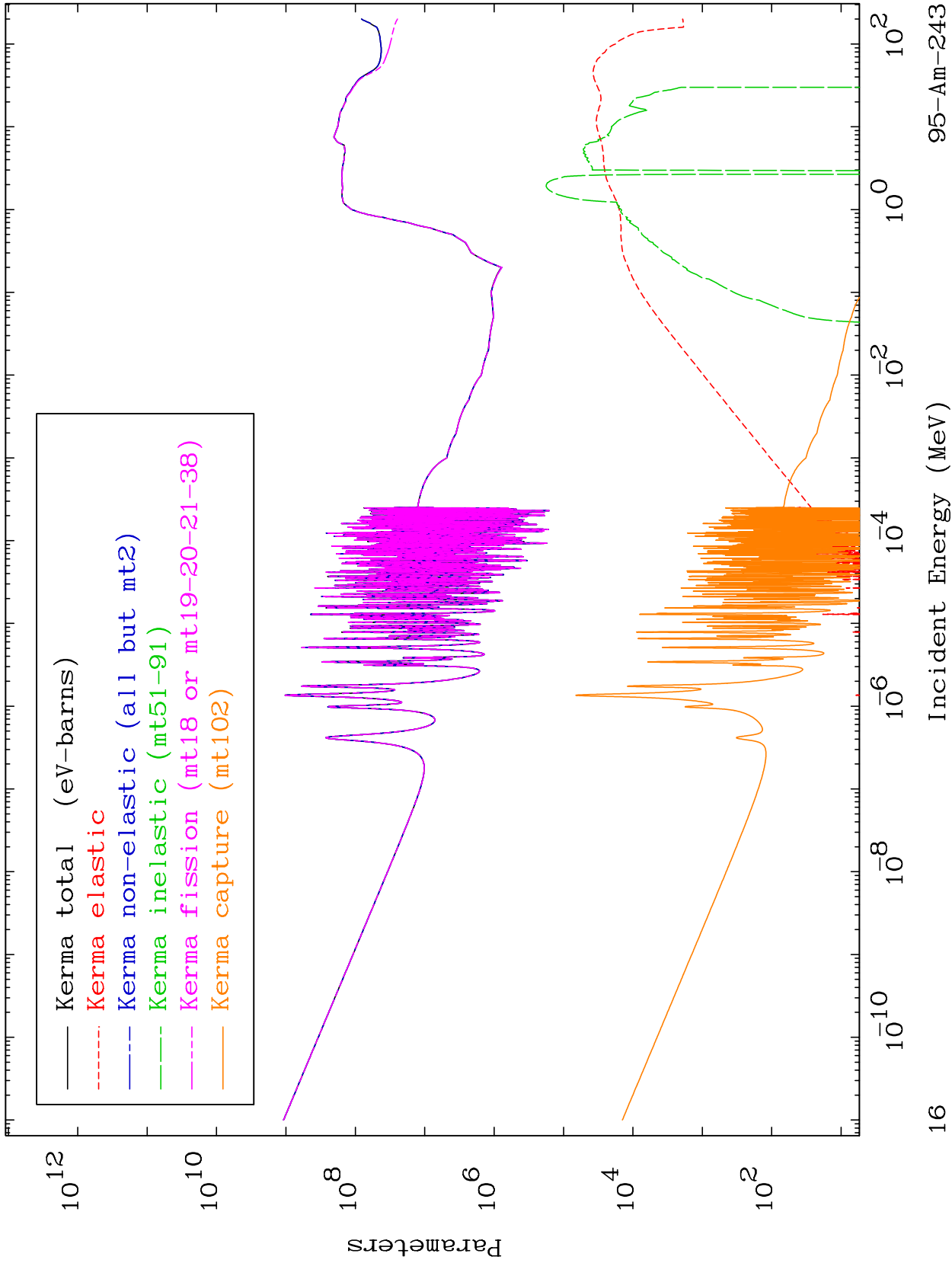
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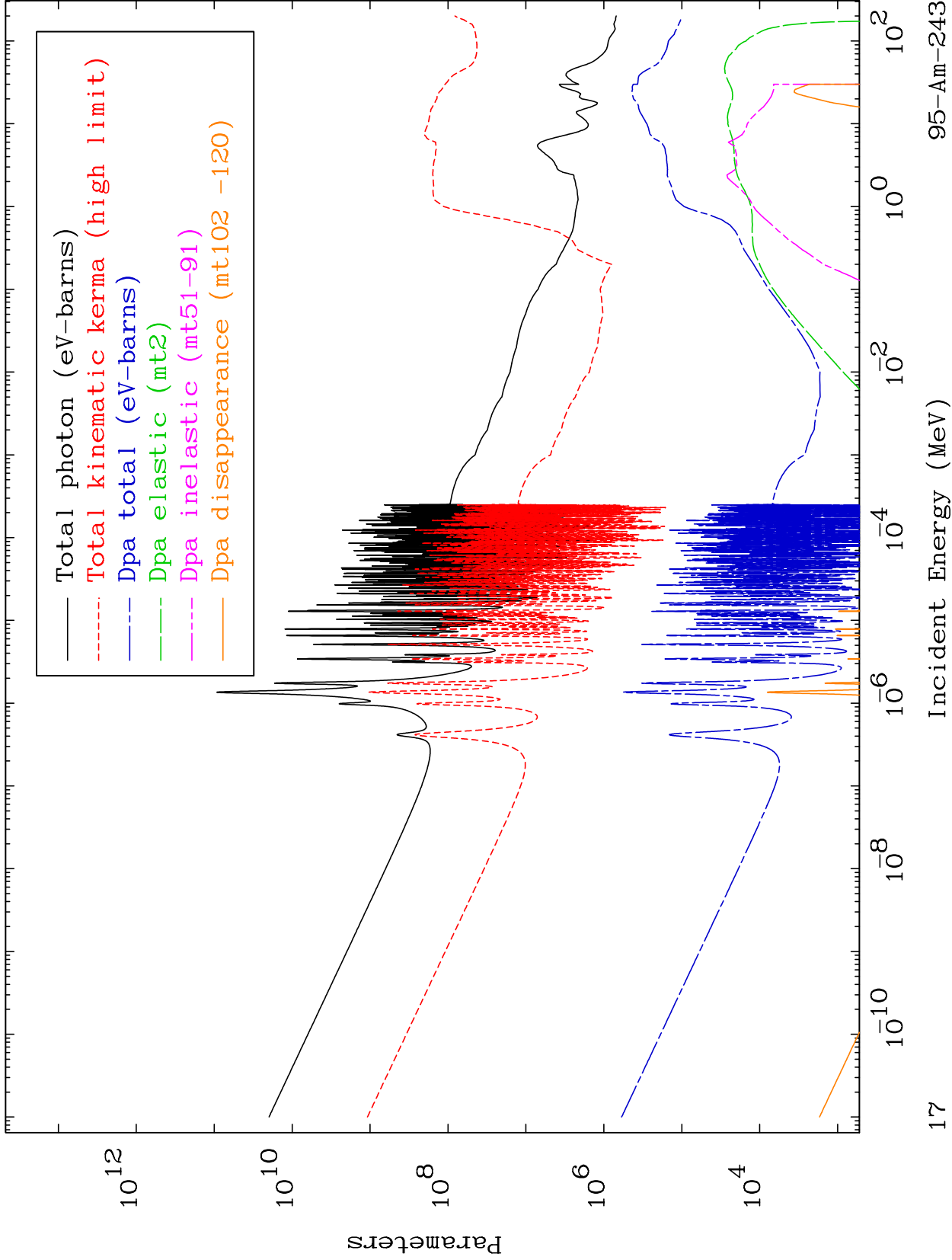


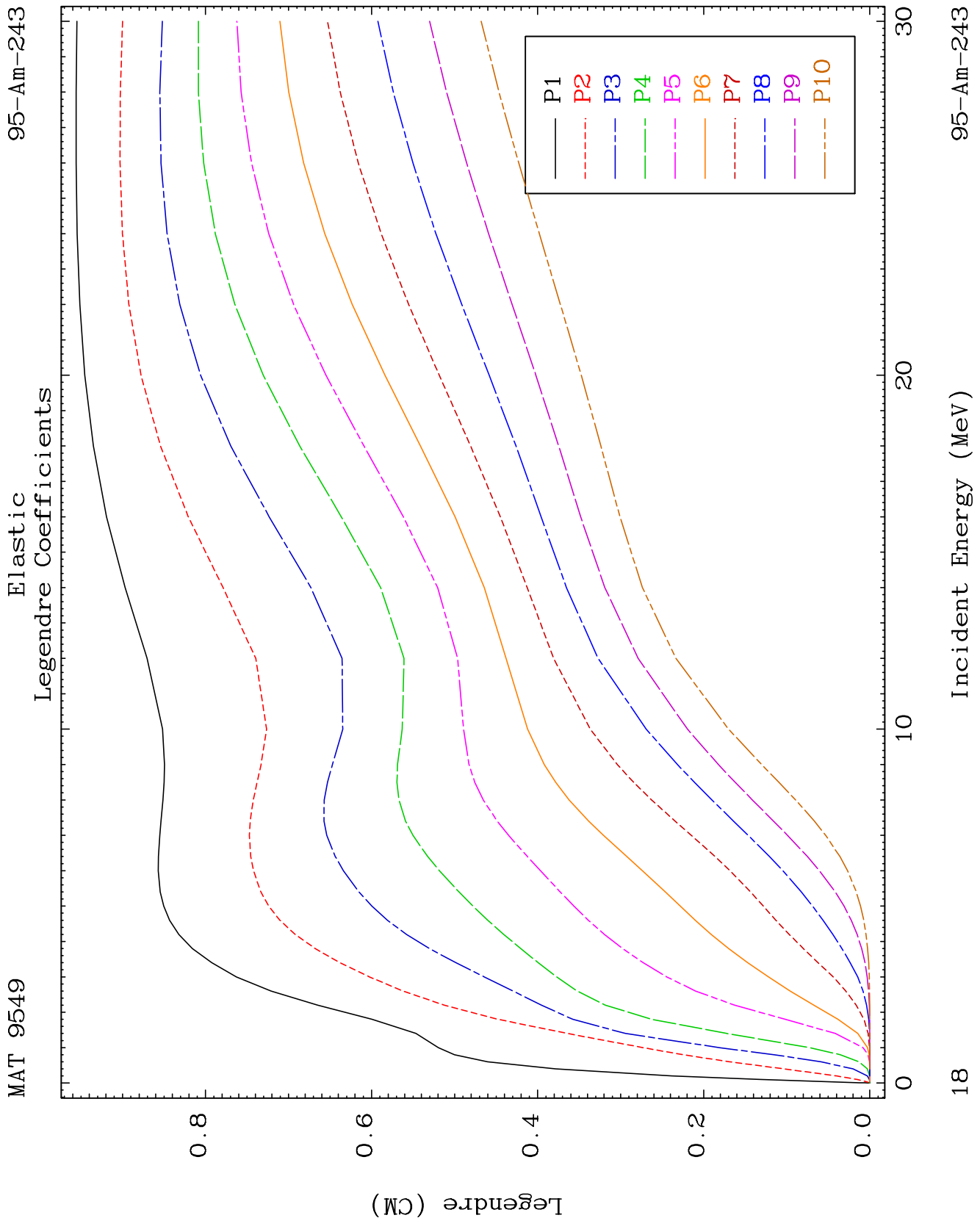
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Incident Energy (MeV)

95-Am-243



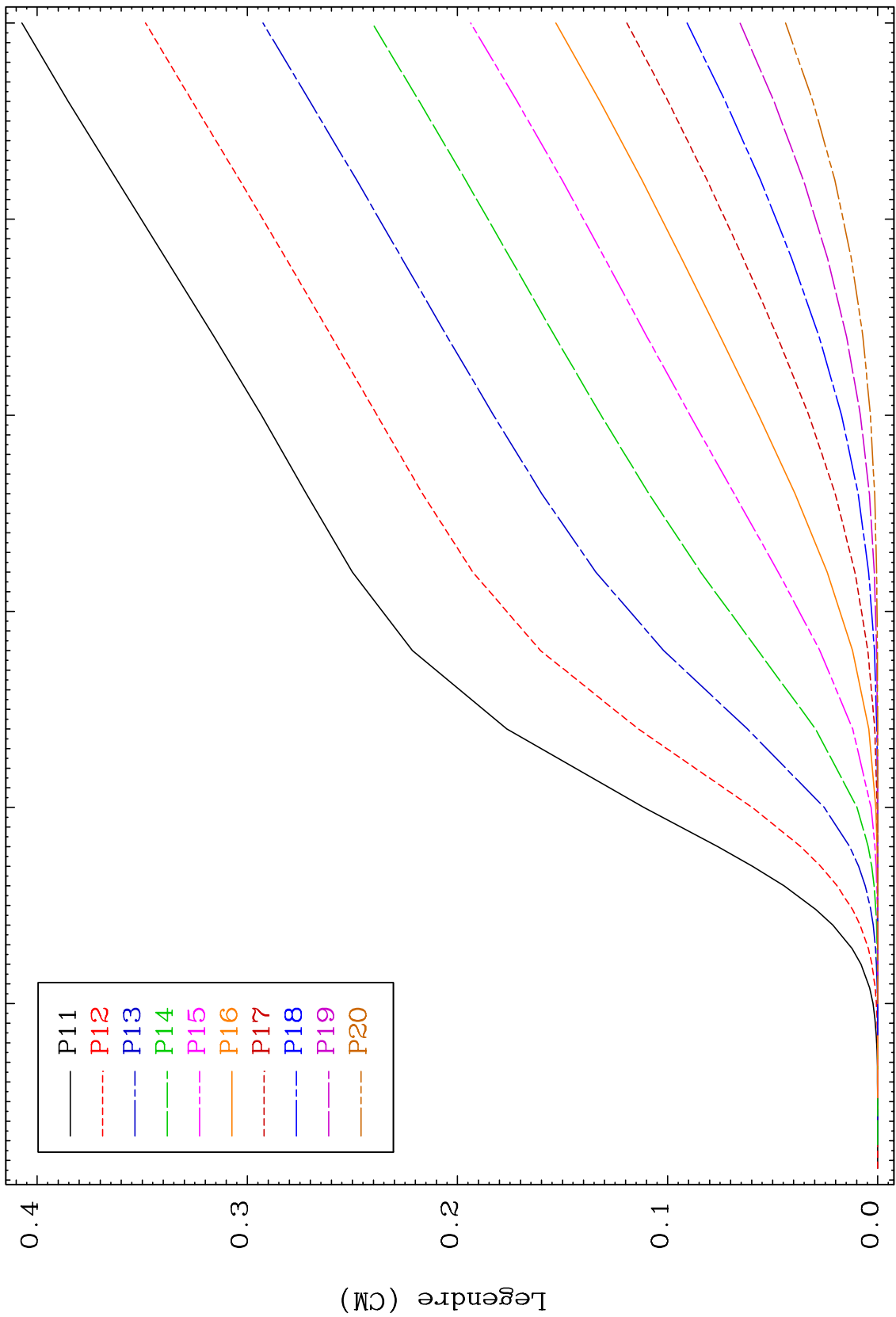




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Elastic Legendre Coefficients

95-Am-243



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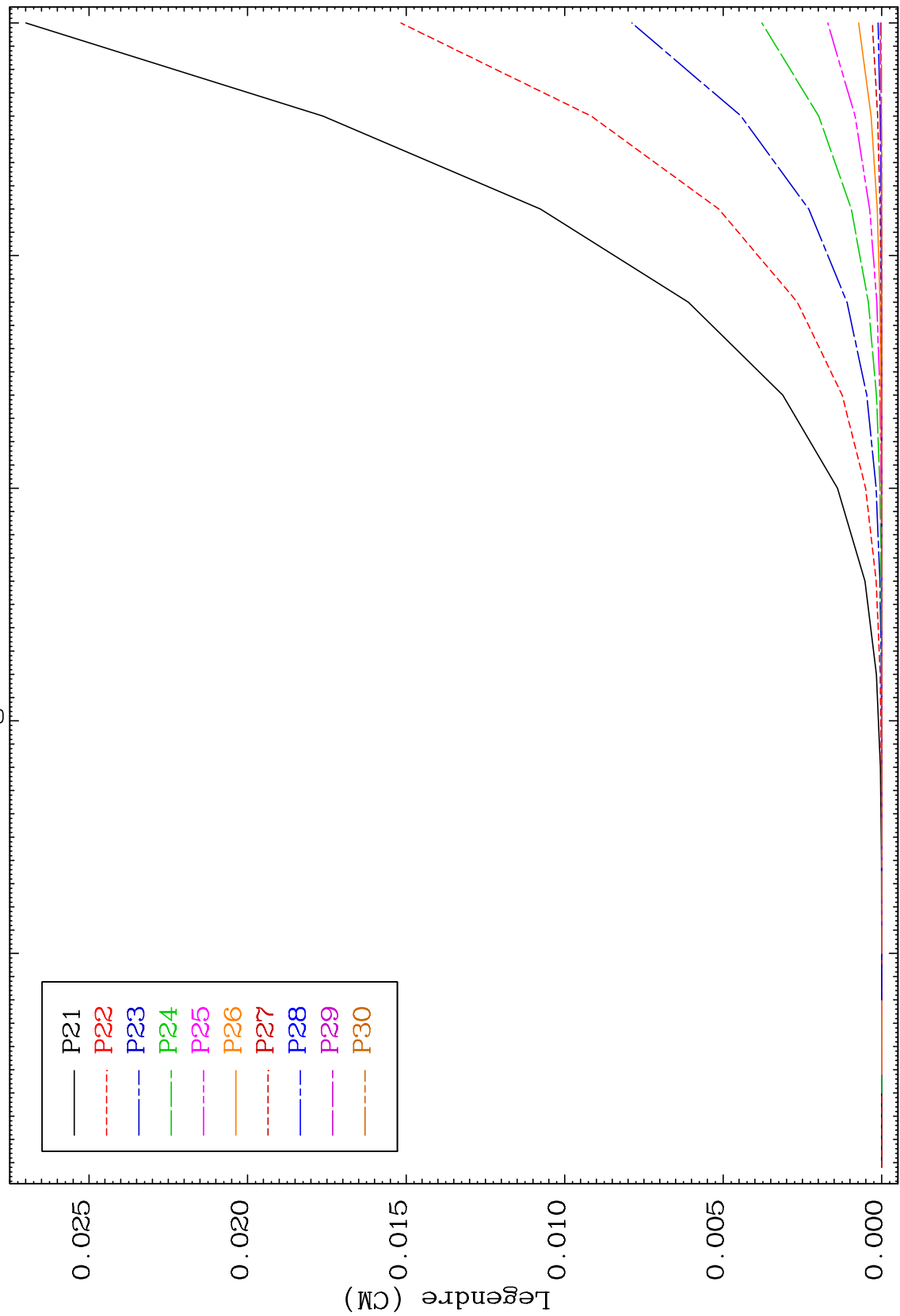
Incident Energy (MeV)

95-Am-243

MAT 9549

Elastic Legendre Coefficients

95-Am-243



95-Am-243

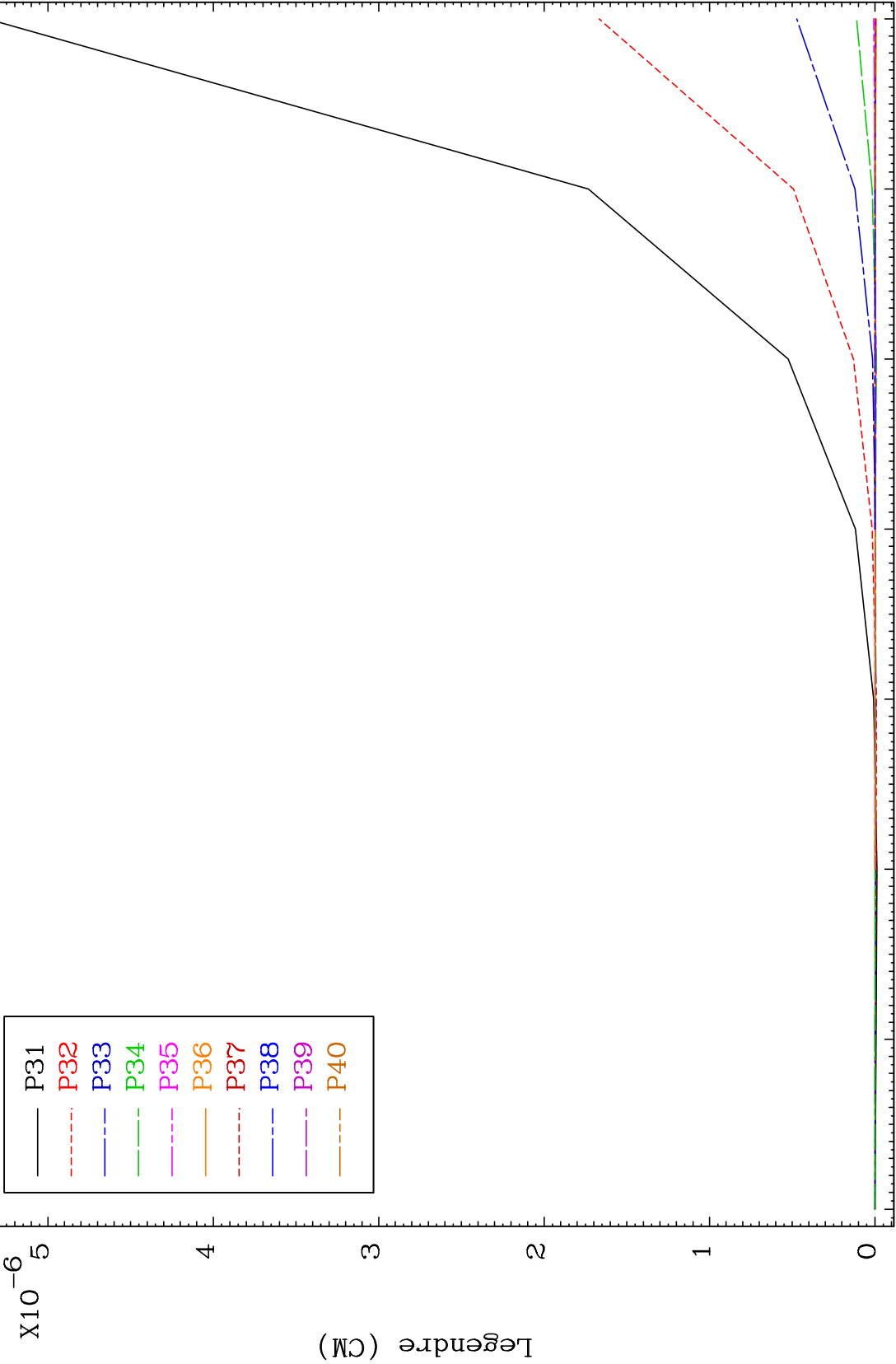
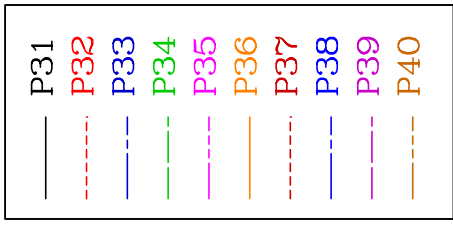
Incident Energy (MeV)

20

MAT 9549

Elastic Legendre Coefficients

95-Am-243

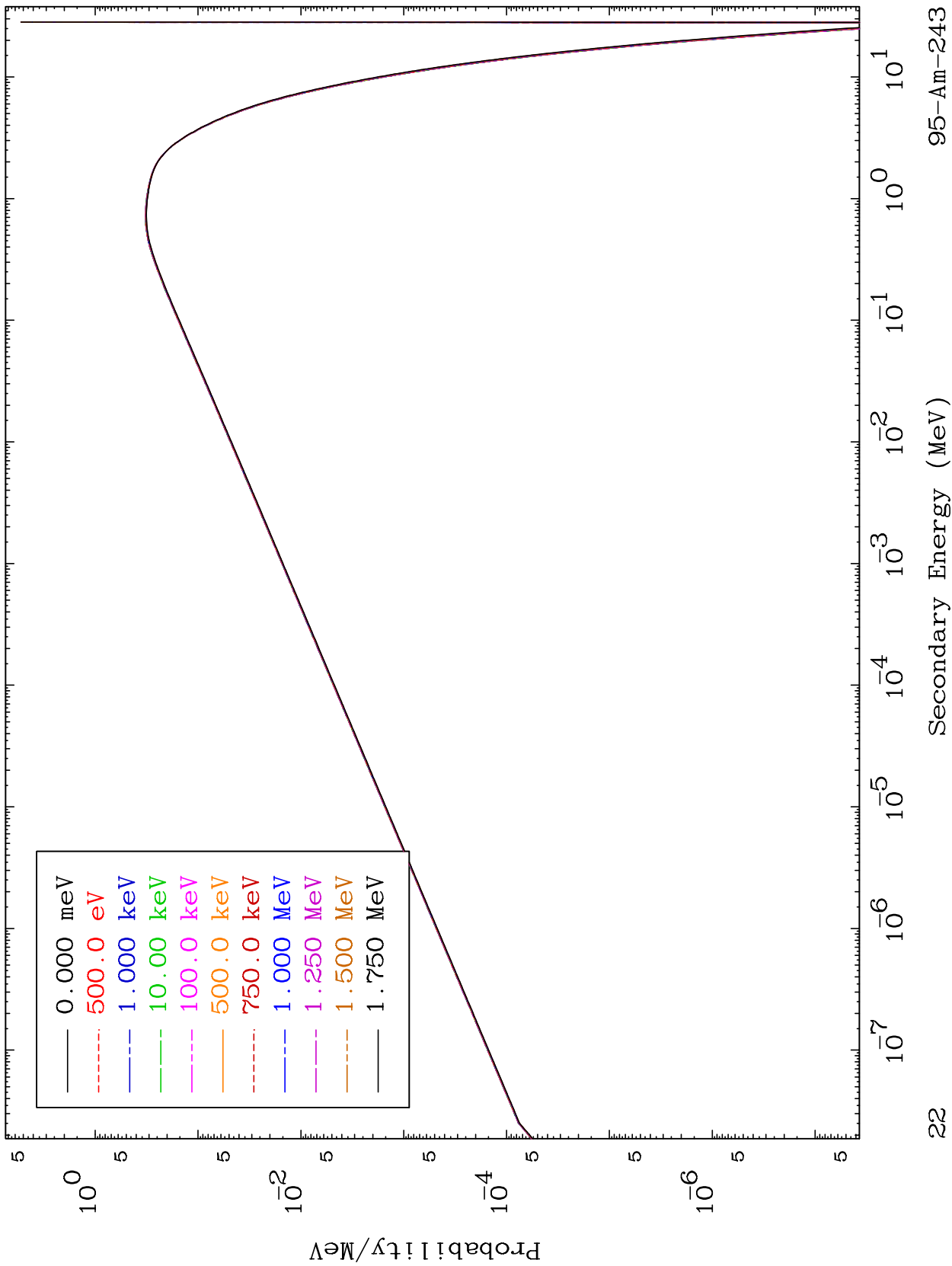


21

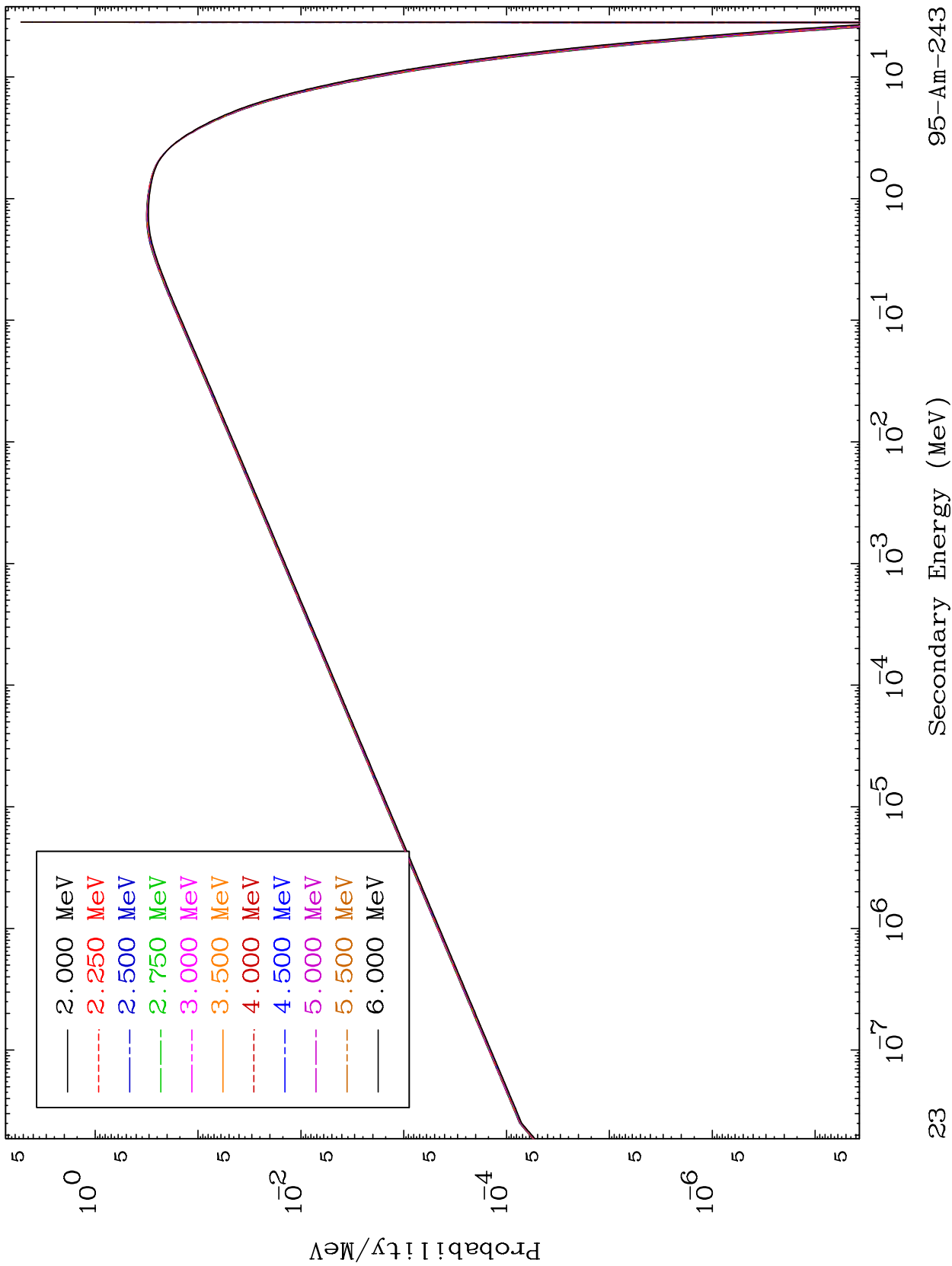
Incident Energy (MeV)

95-Am-243

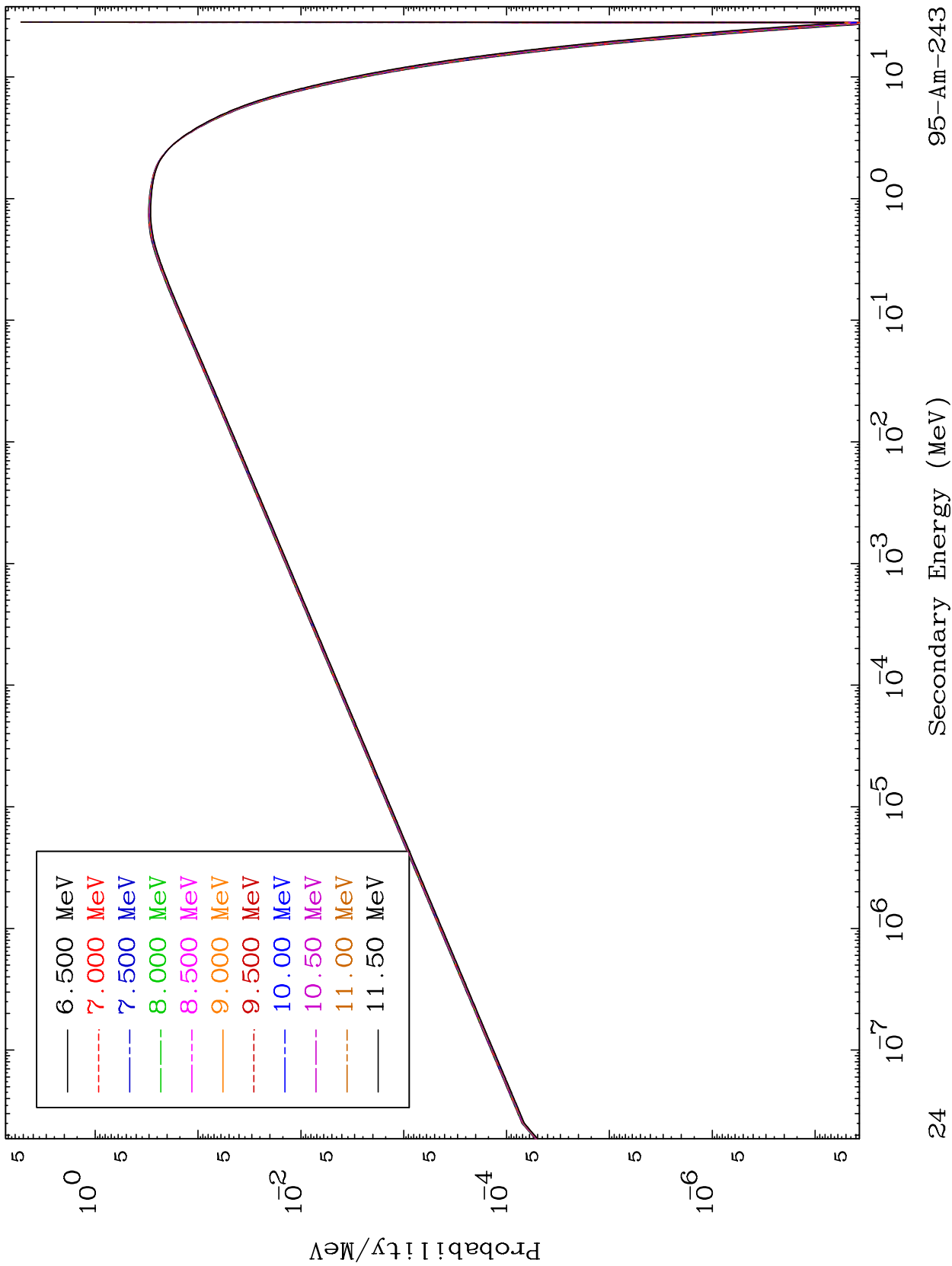
Fission Energy Distribution



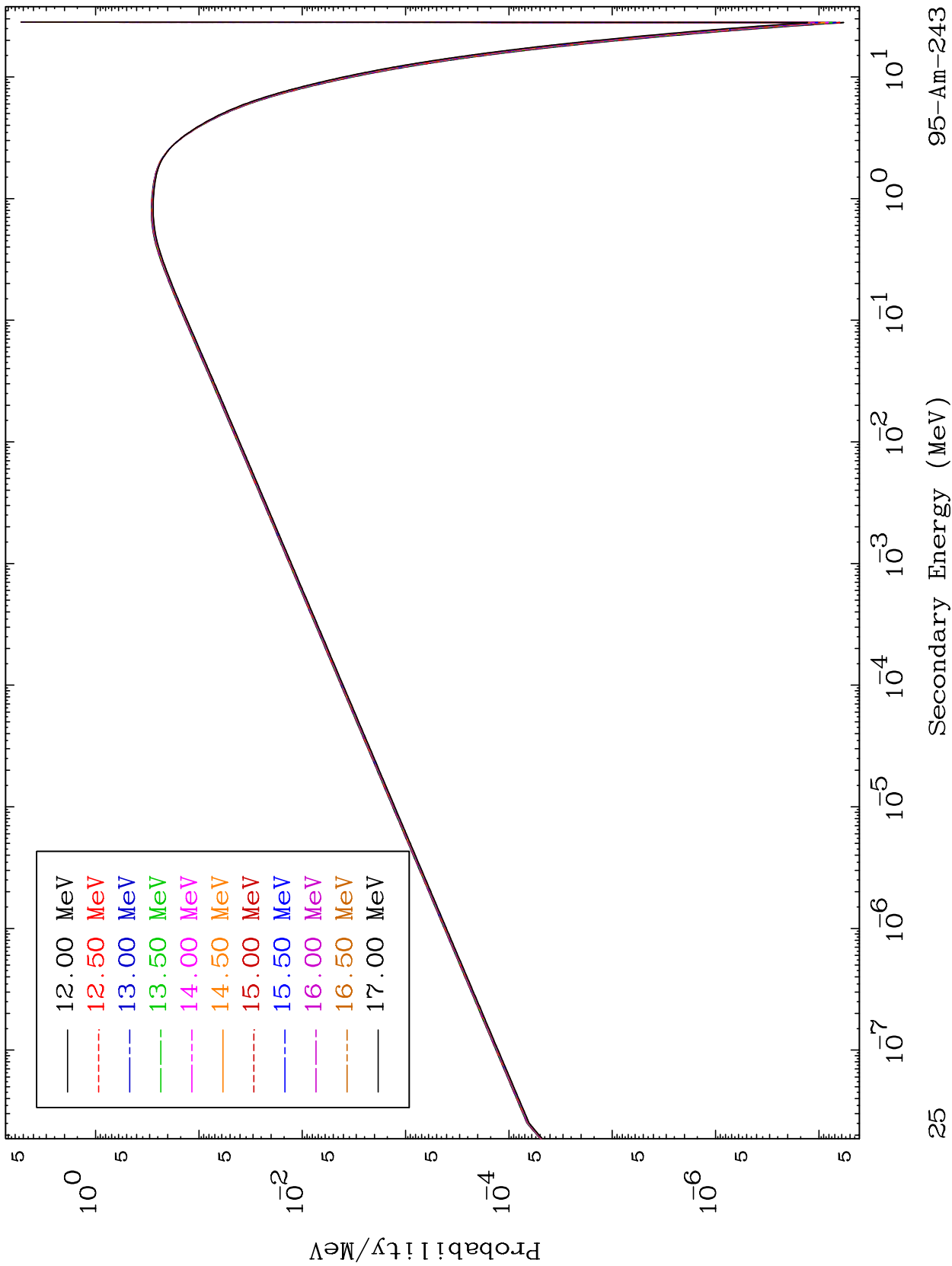
Fission Energy Distribution



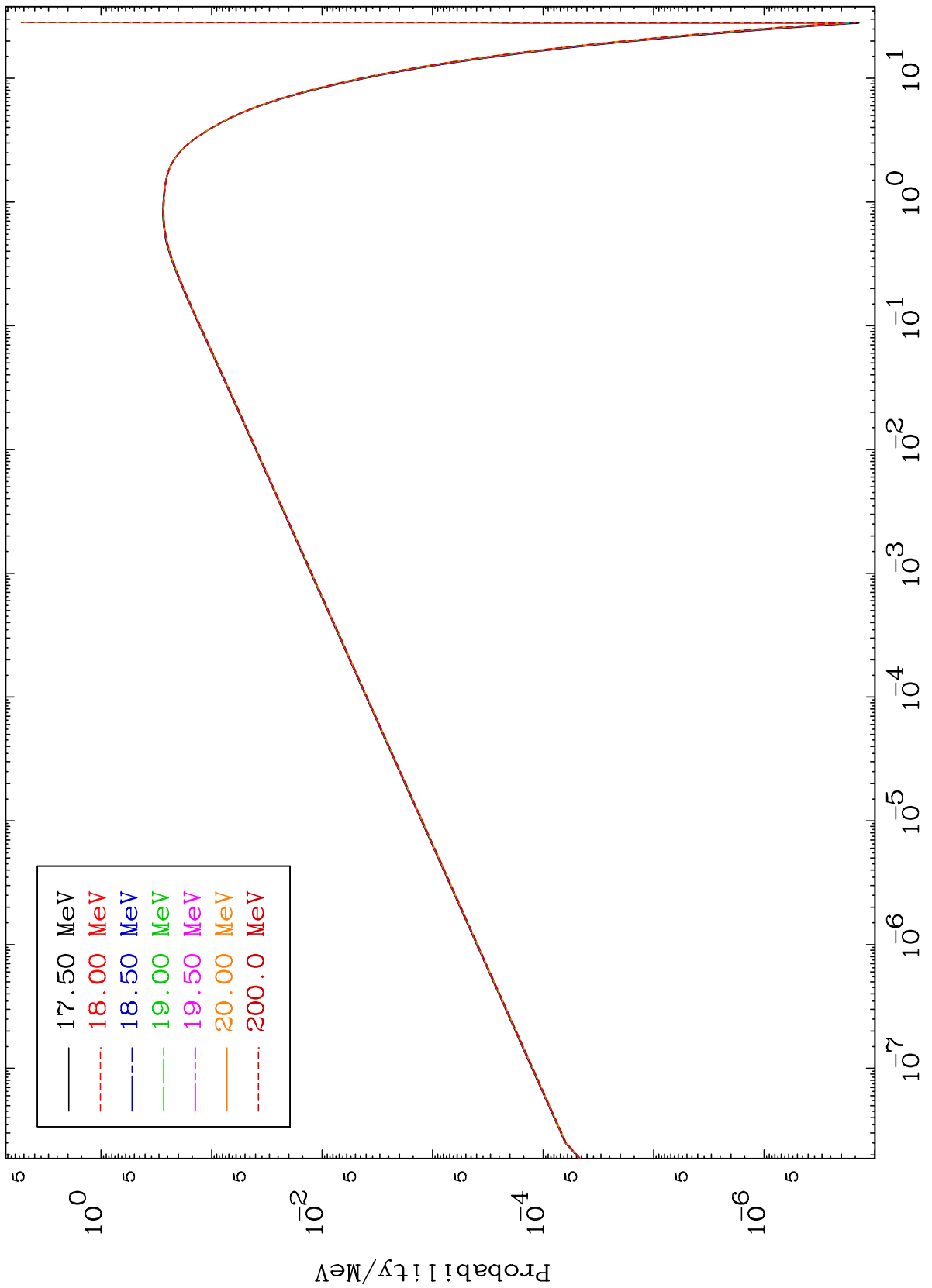
Fission Energy Distribution



Fission
Energy Distribution



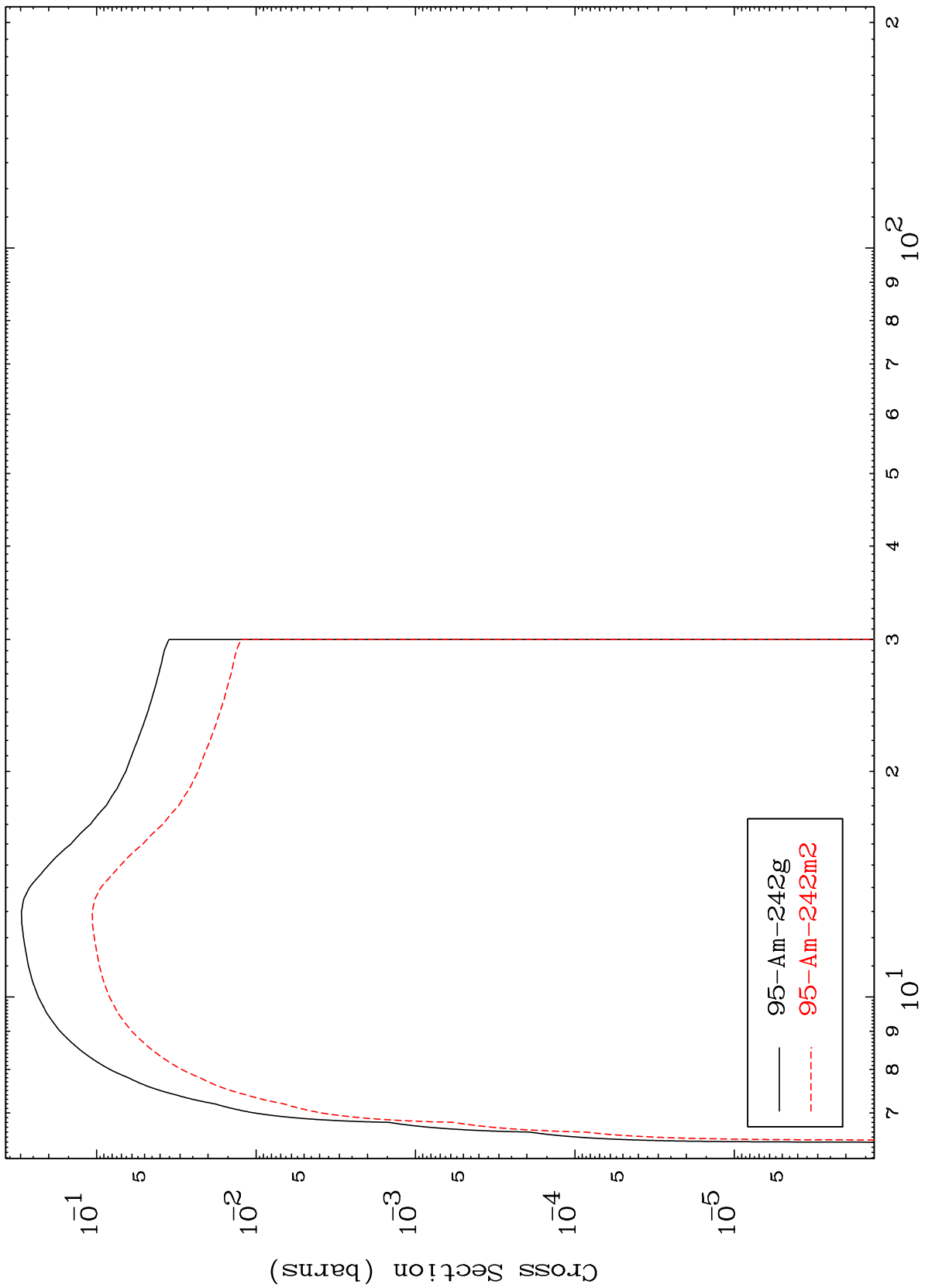
Fission
Energy Distribution



MAT 9549

95-Am-243

(n,2n)
Radionuclide Production Cross Section



95-Am-243

Incident Energy (MeV)

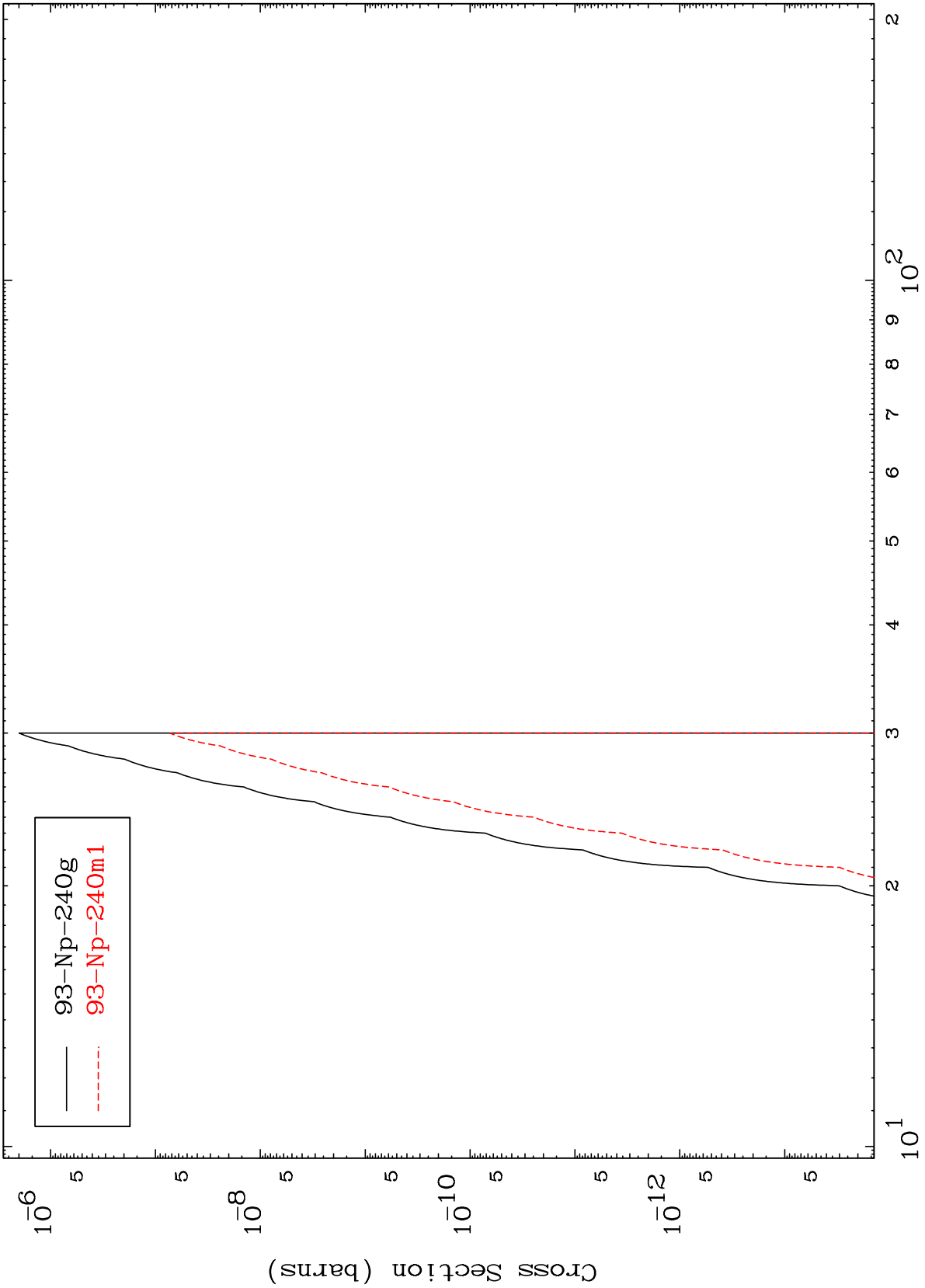
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(n,n') He-3

95-Am-243

Radionuclide Production Cross Section



— $^{93}\text{Np-240g}$
- - - $^{93}\text{Np-240m1}$

Incident Energy (MeV)

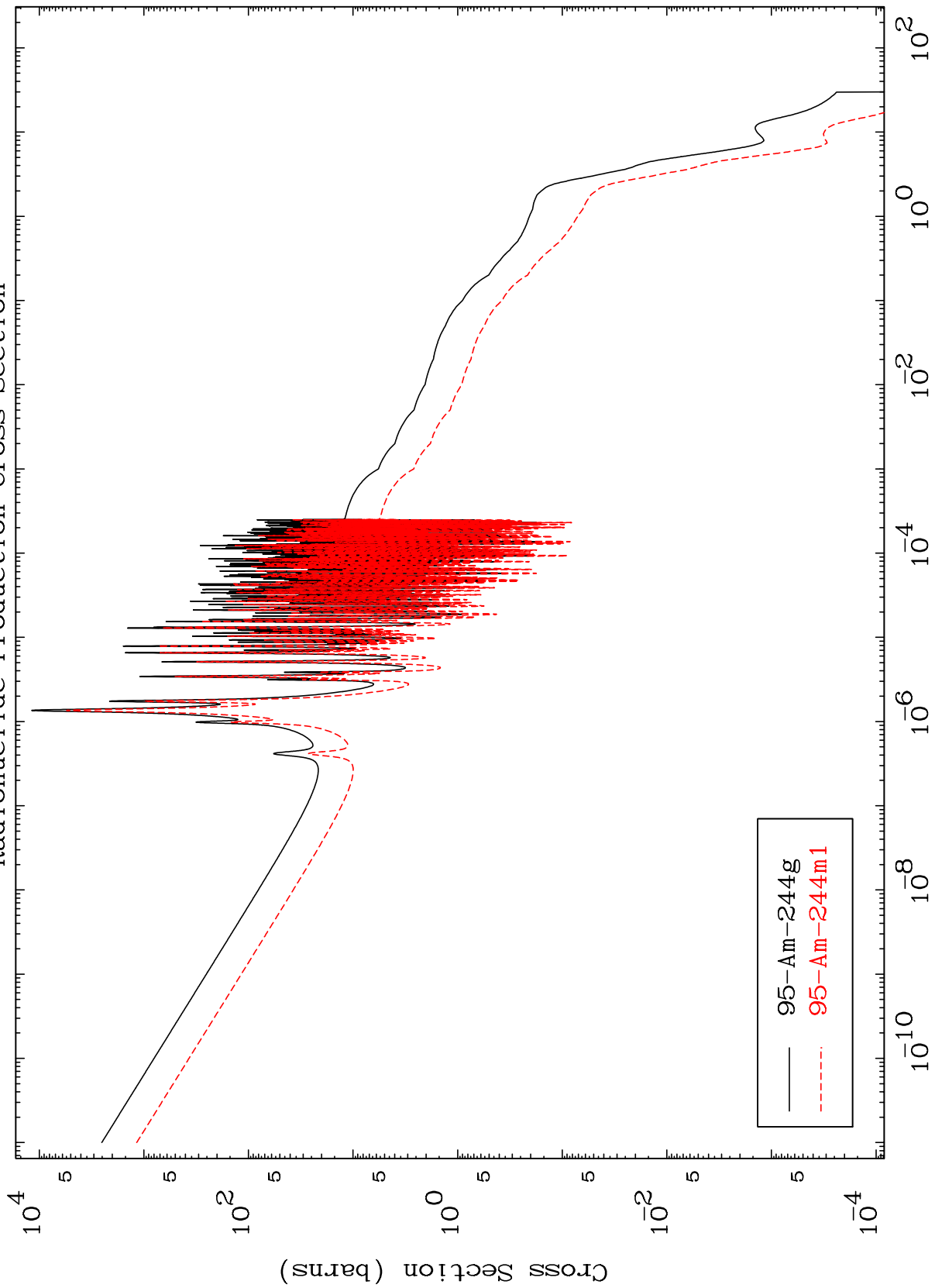
95-Am-243

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MAT 9549

95-Am-243

(n, γ)
Radionuclide Production Cross Section



95-Am-243

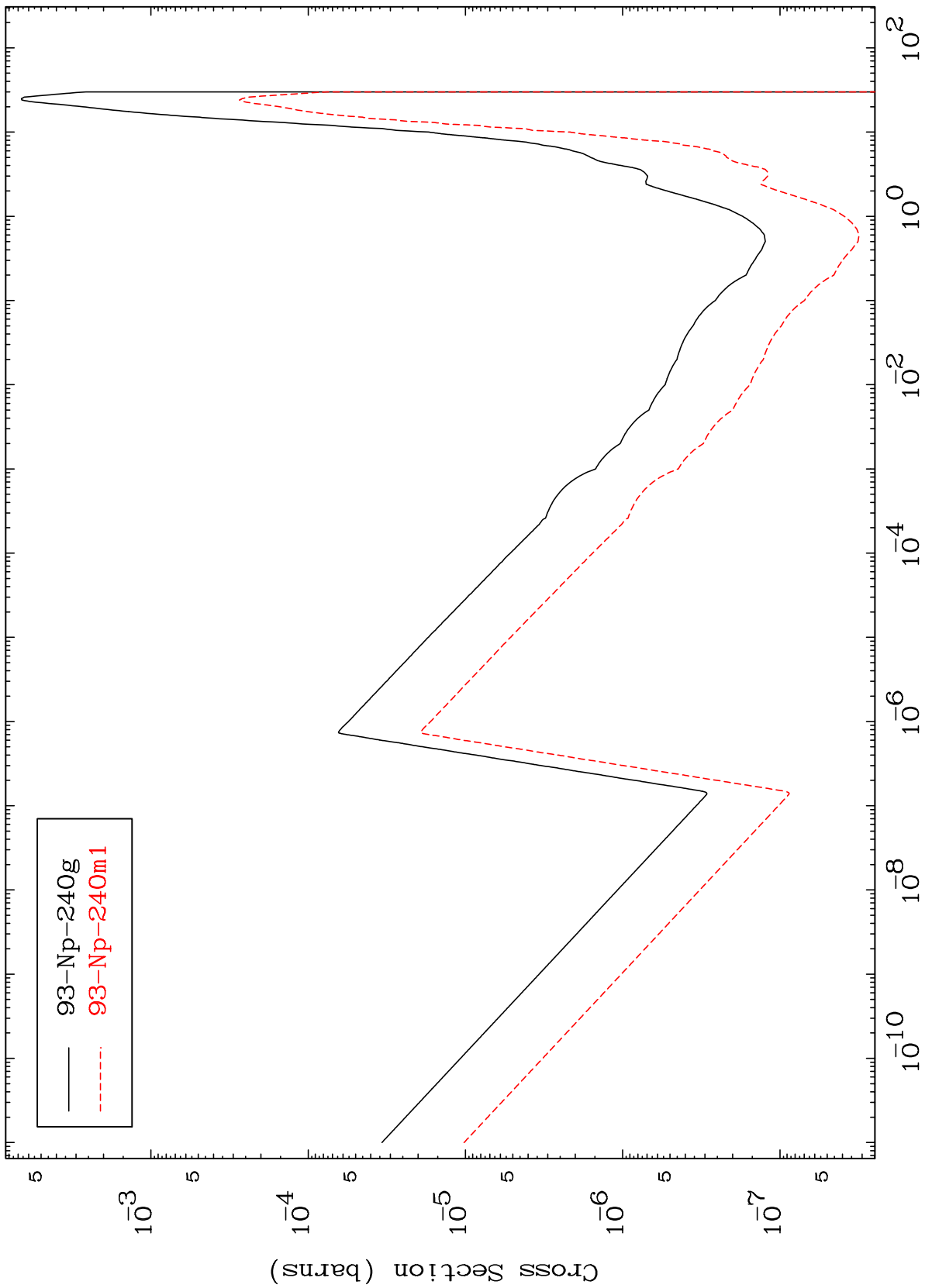
Incident Energy (MeV)

29

MAT 9549

95-Am-243

(n, α)
Radionuclide Production Cross Section



95-Am-243

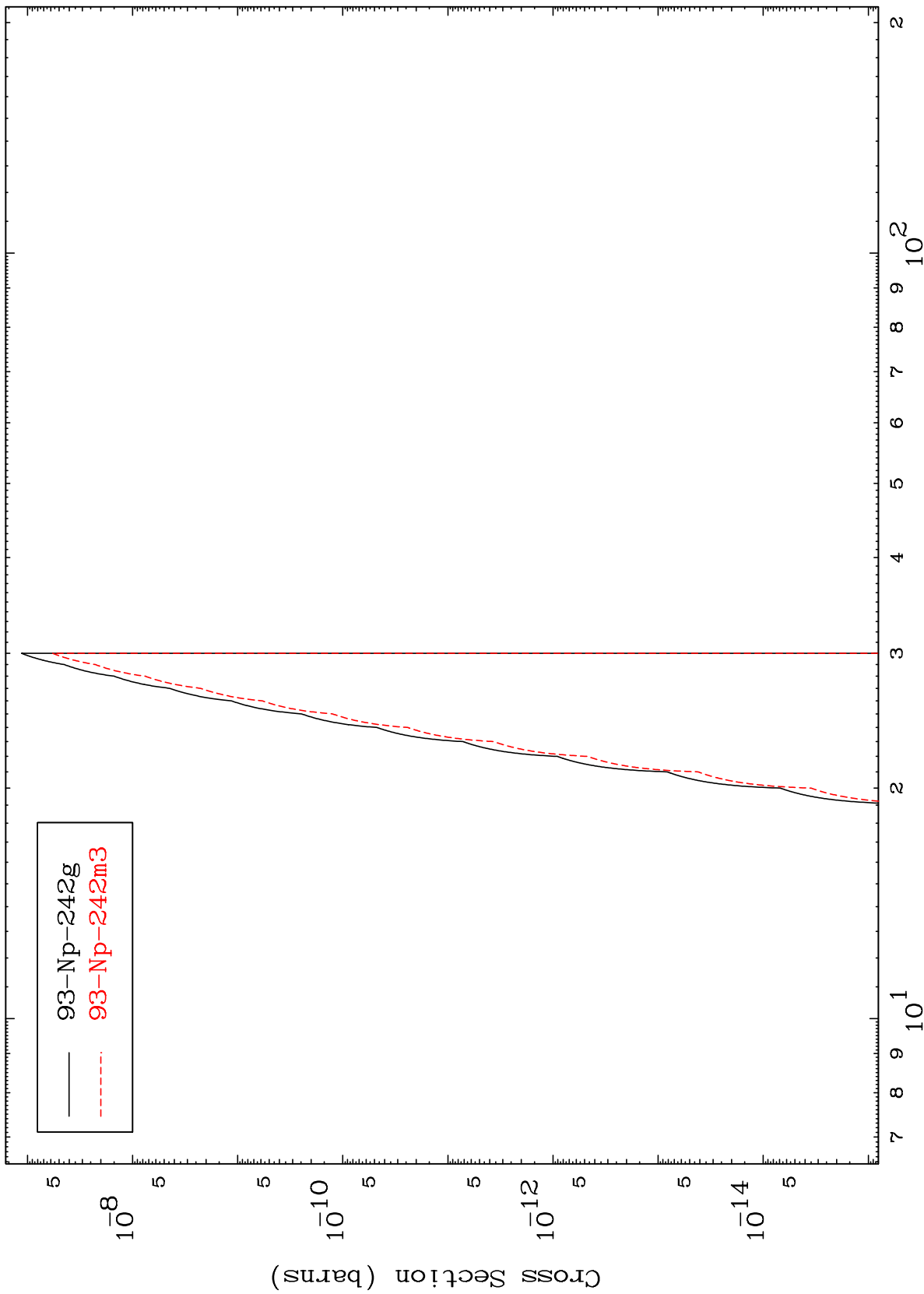
Incident Energy (MeV)

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MAT 9549

95-Am-243

(n,2p)
Radionuclide Production Cross Section



— 93-Np-242g
- - - 93-Np-242m3

95-Am-243

Incident Energy (MeV)

31