

Program EVALPLOT  
(Version 2015-2)

by

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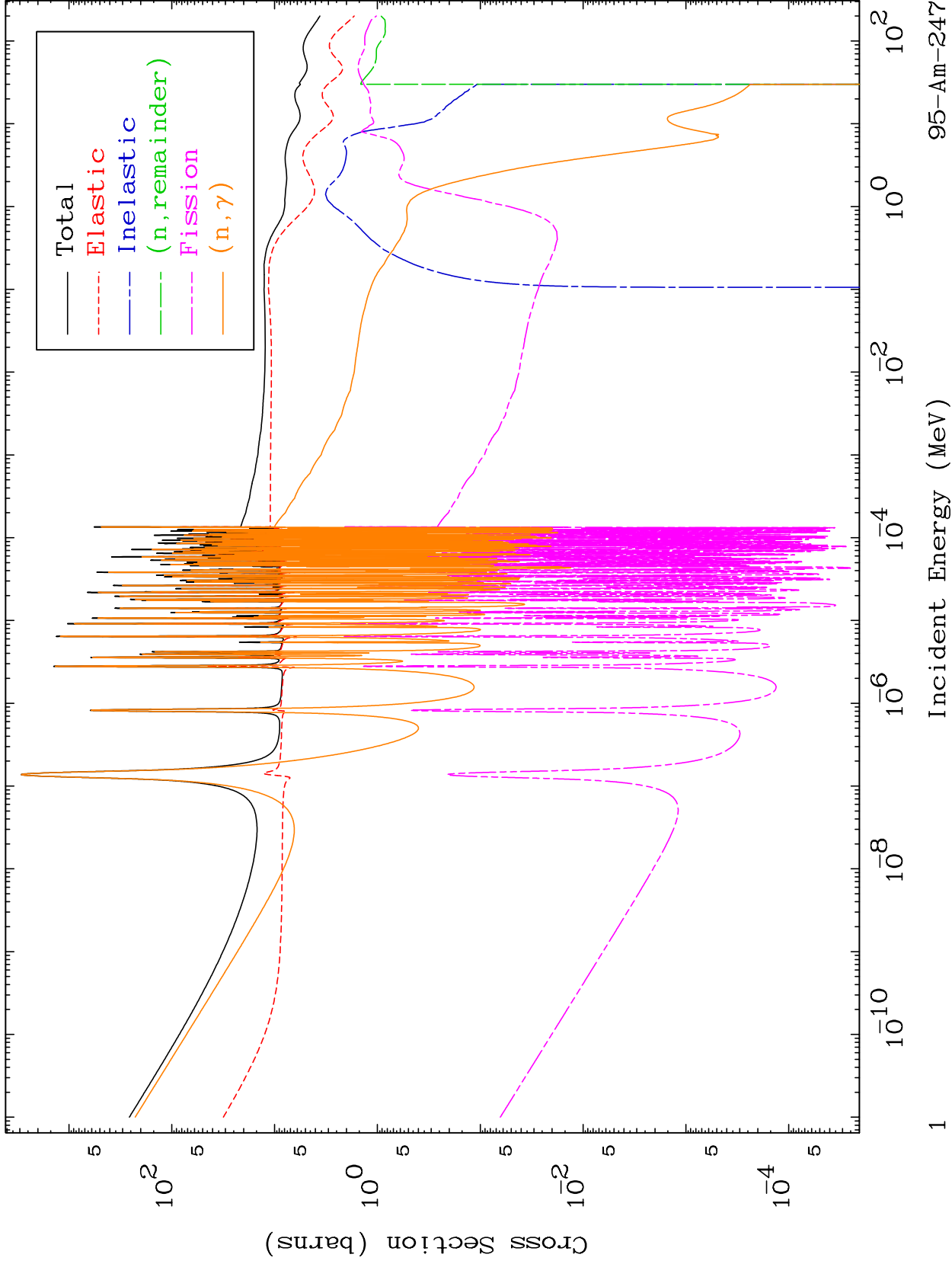
Press Mouse Button to Start

MAT 9561

Major

293 Kelvin Cross Sections

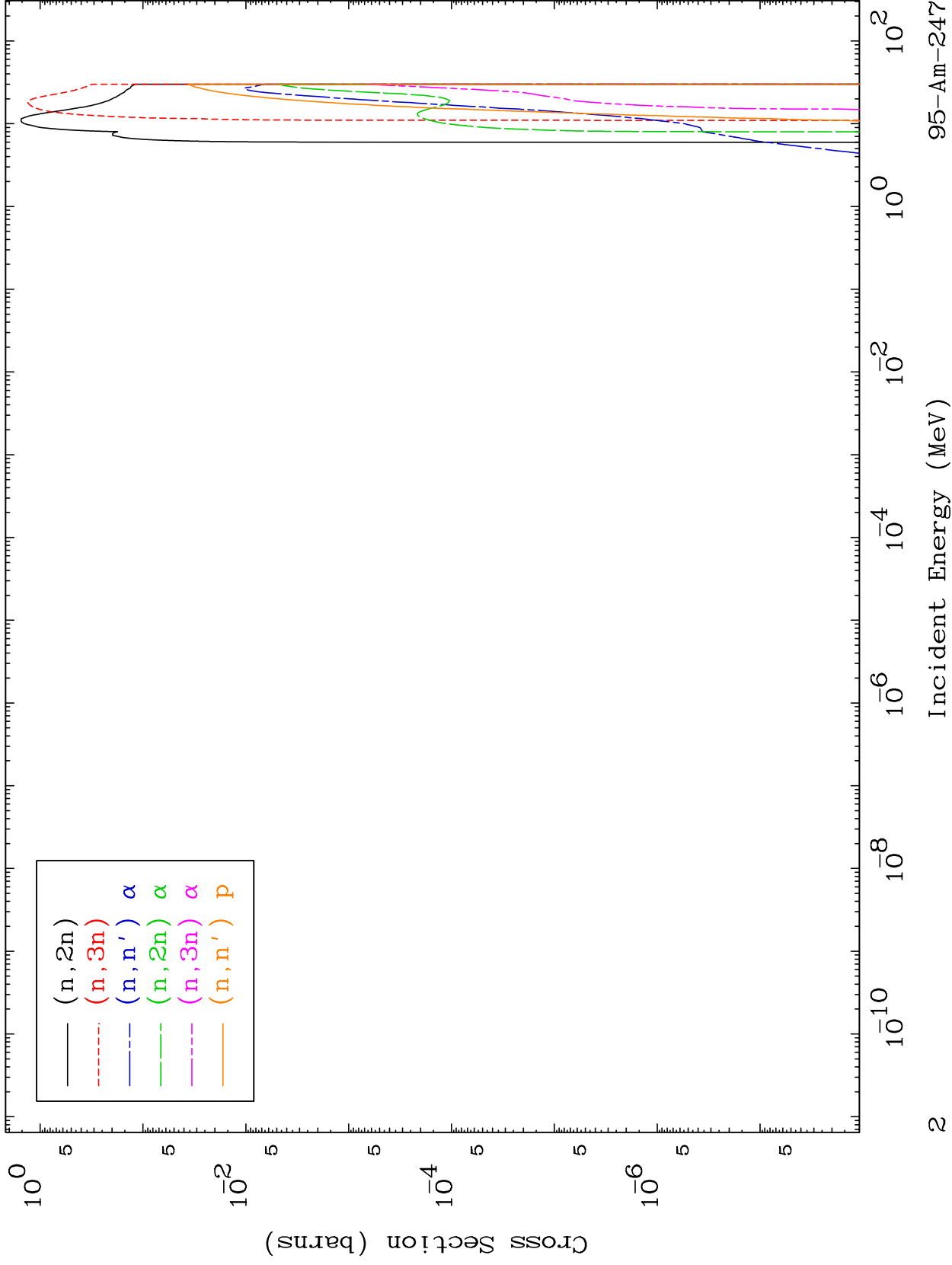
95-Am-247

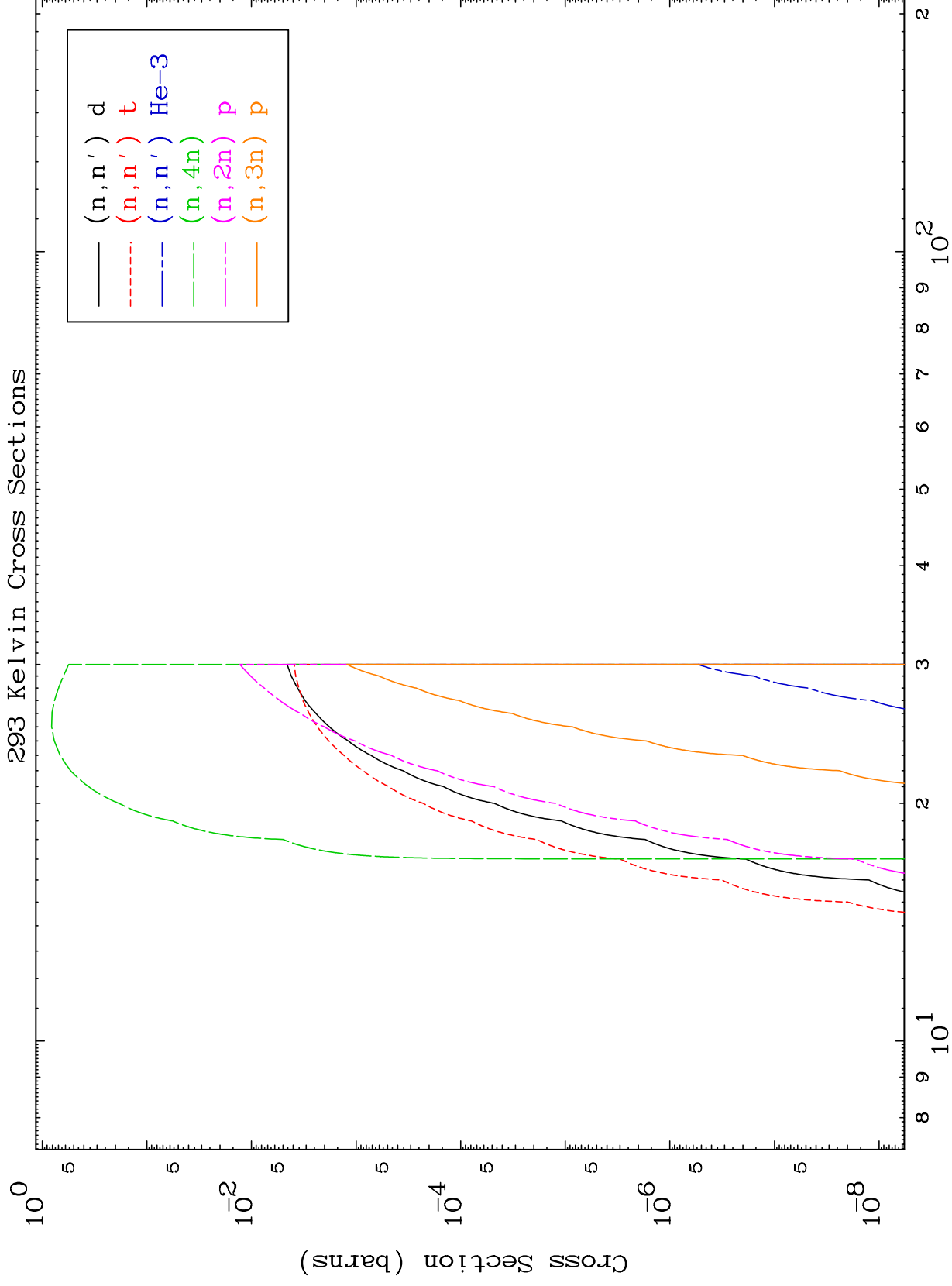


MAT 9561

Neutron Production  
293 Kelvin Cross Sections

95-Am-247

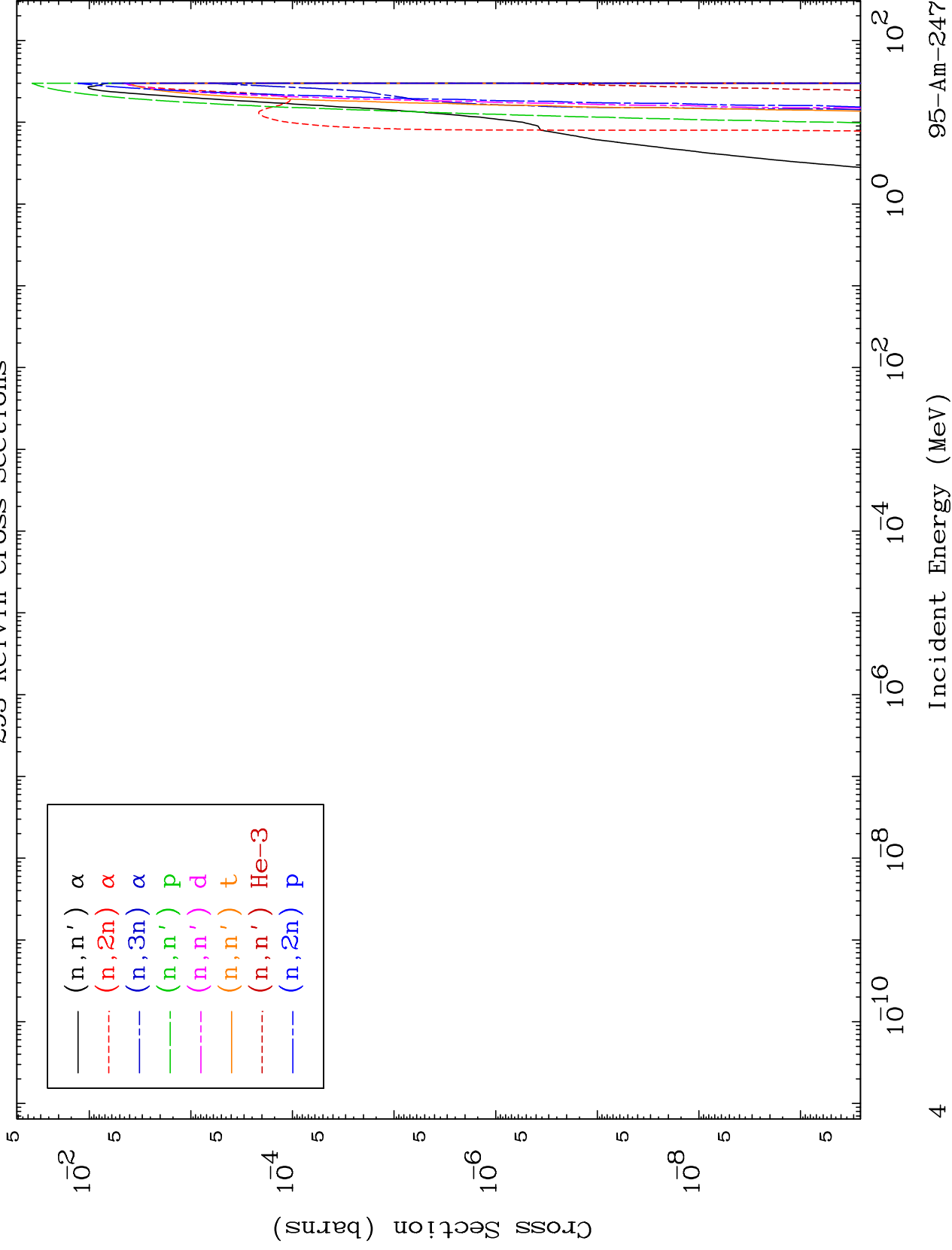


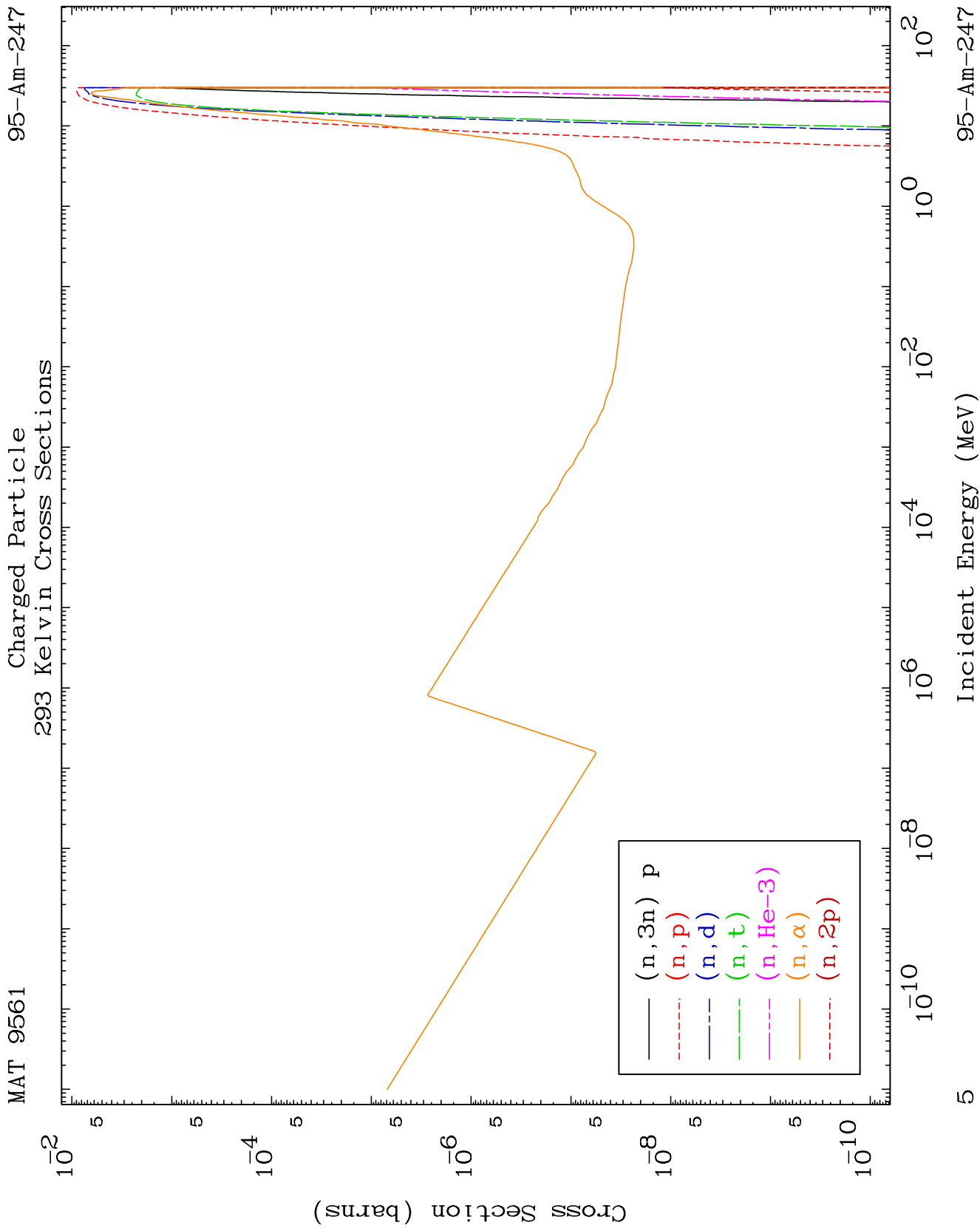


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Charged Particle  
293 Kelvin Cross Sections

95-Am-247

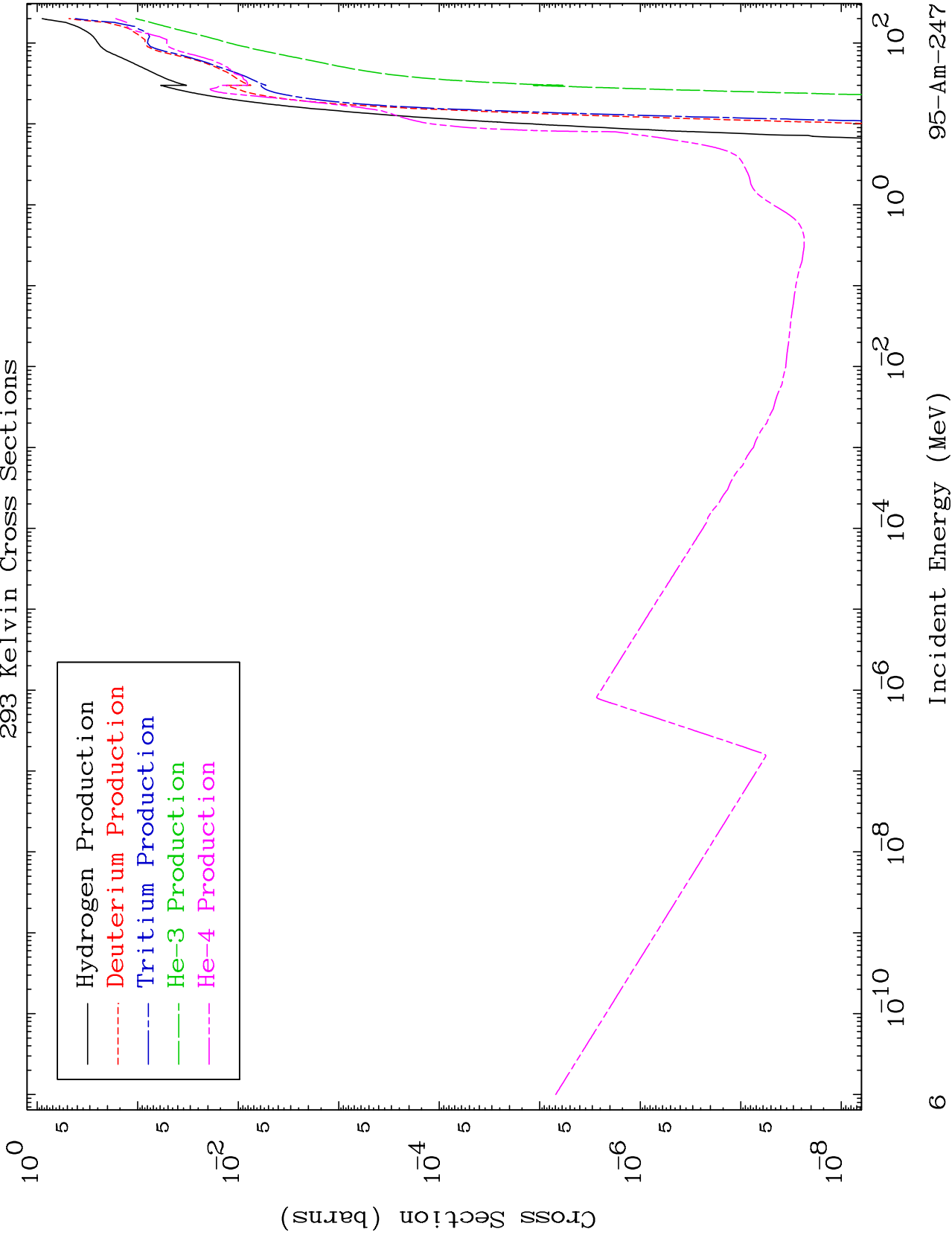




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Particle Production  
293 Kelvin Cross Sections

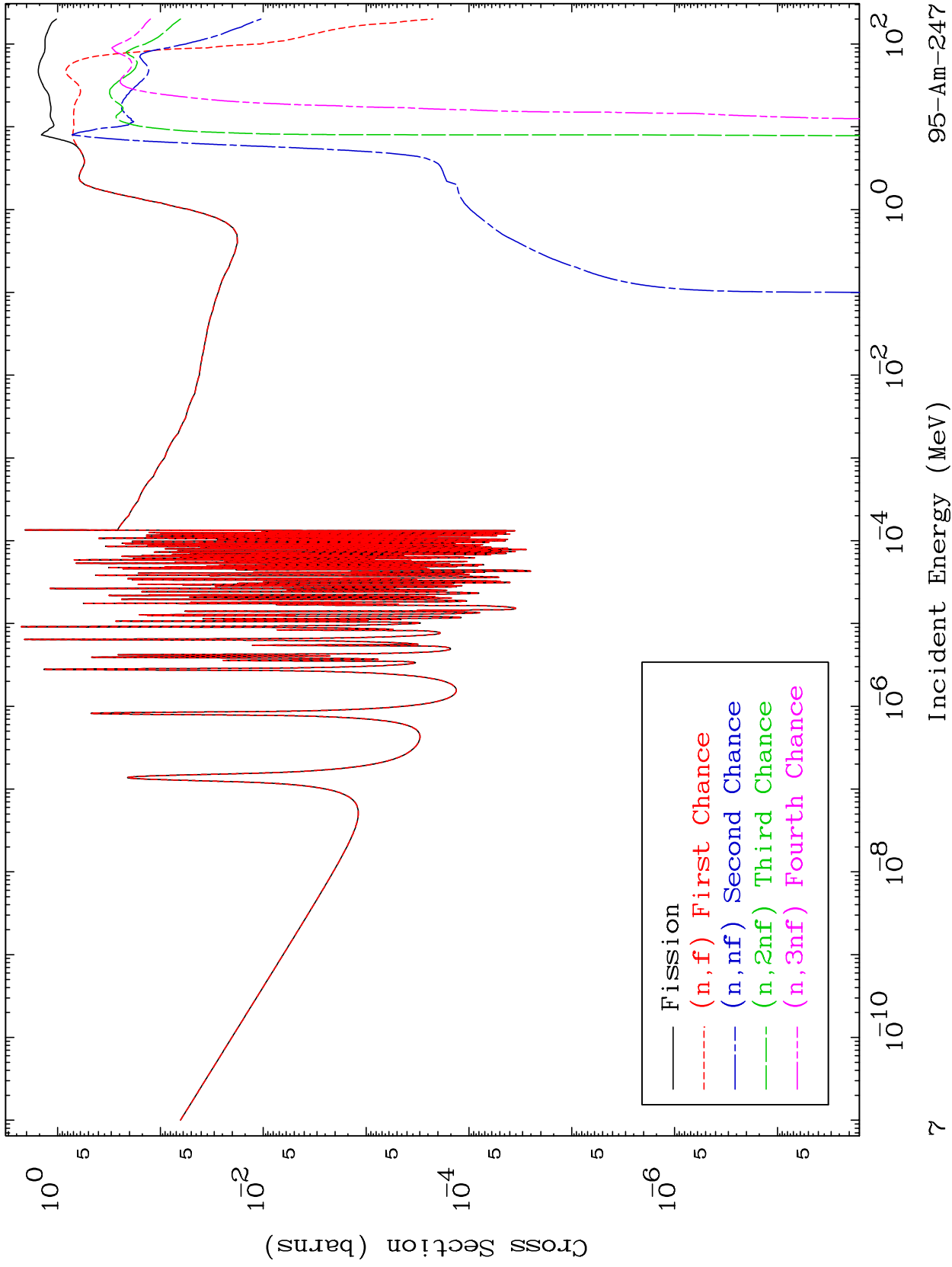
95-Am-247



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Fission  
293 Kelvin Cross Sections

95-Am-247

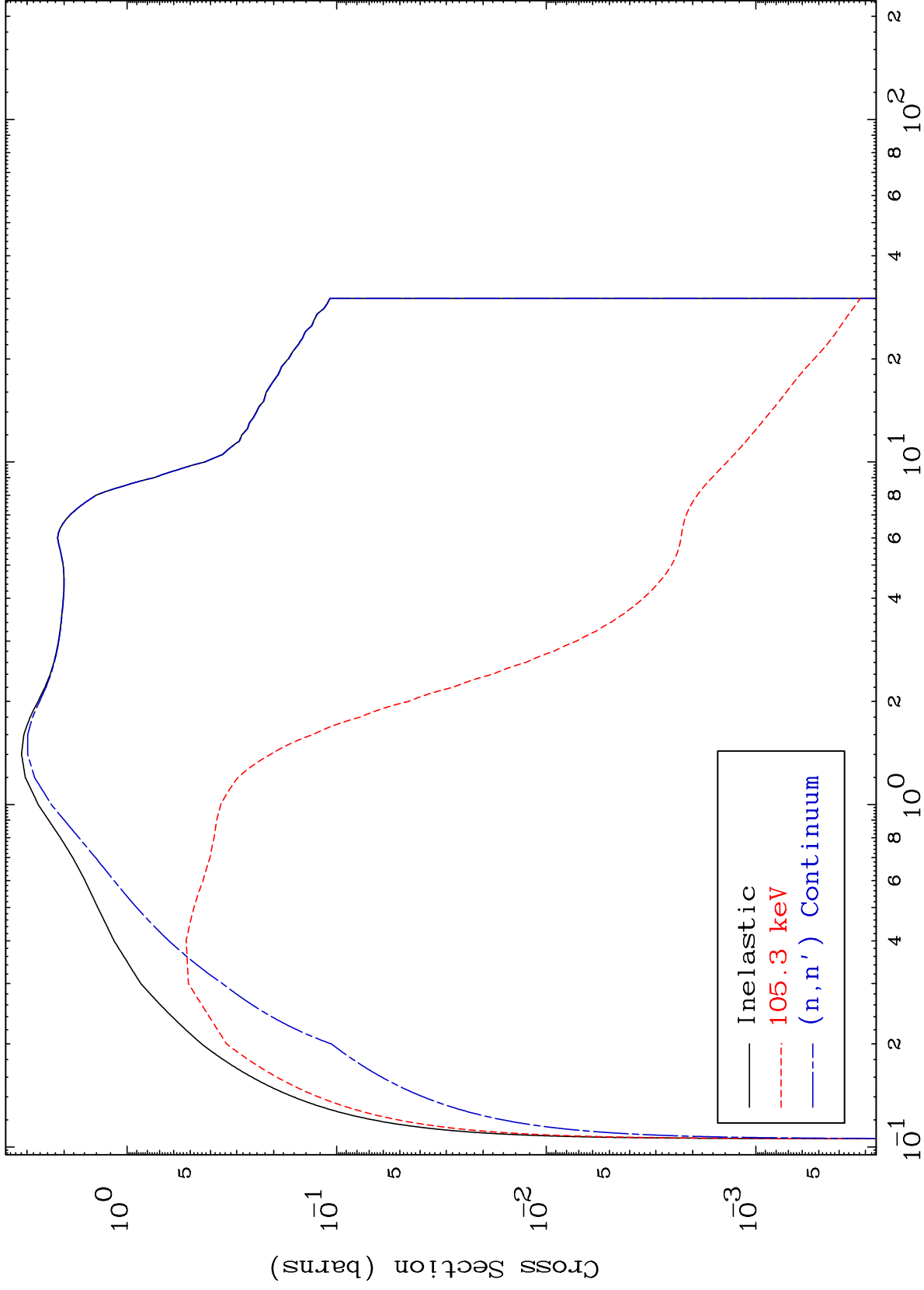




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(n,n') Level  
293 Kelvin Cross Sections

95-Am-247



95-Am-247

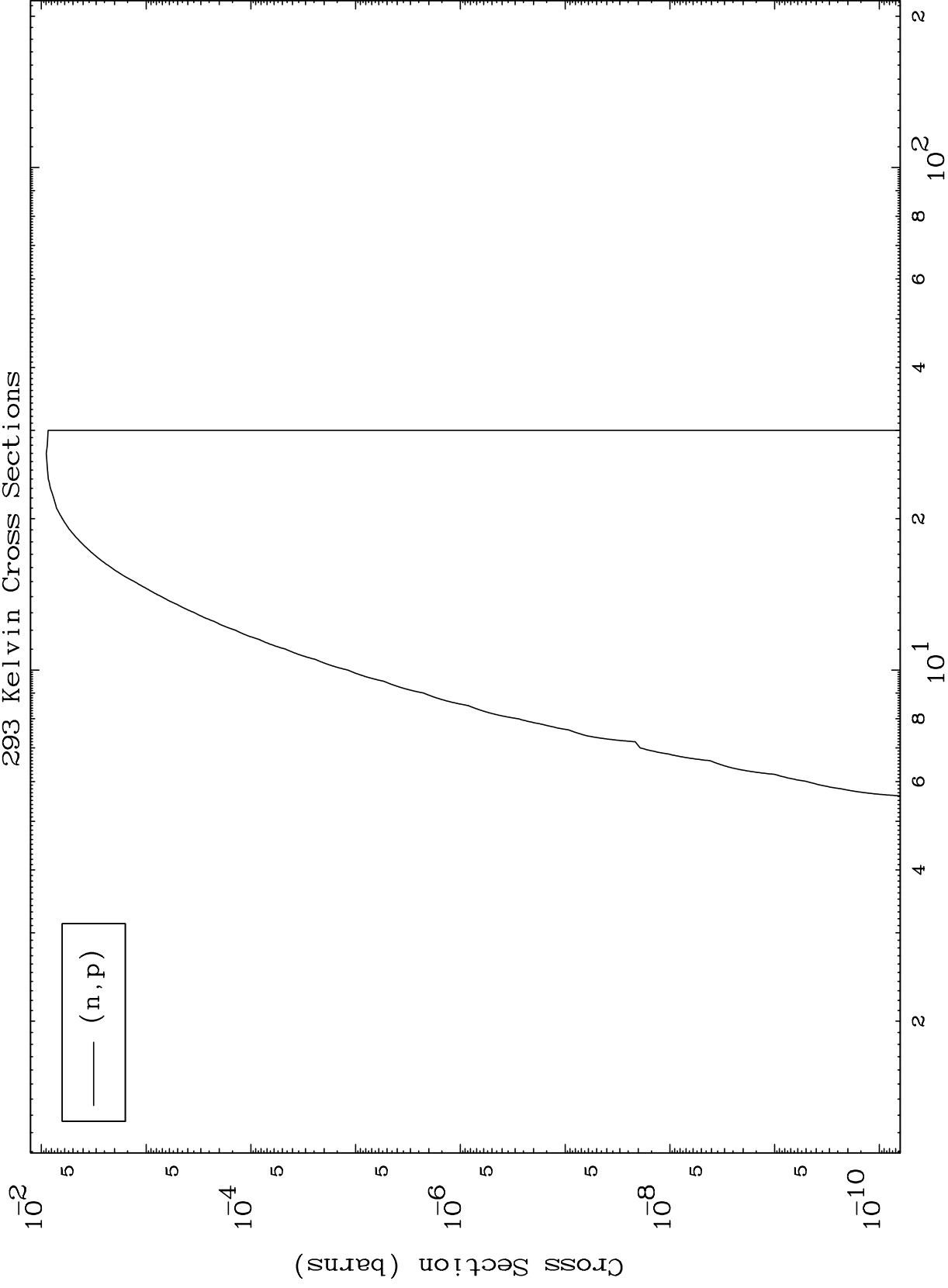
Incident Energy (MeV)

8

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(n,p) Levels  
293 Kelvin Cross Sections

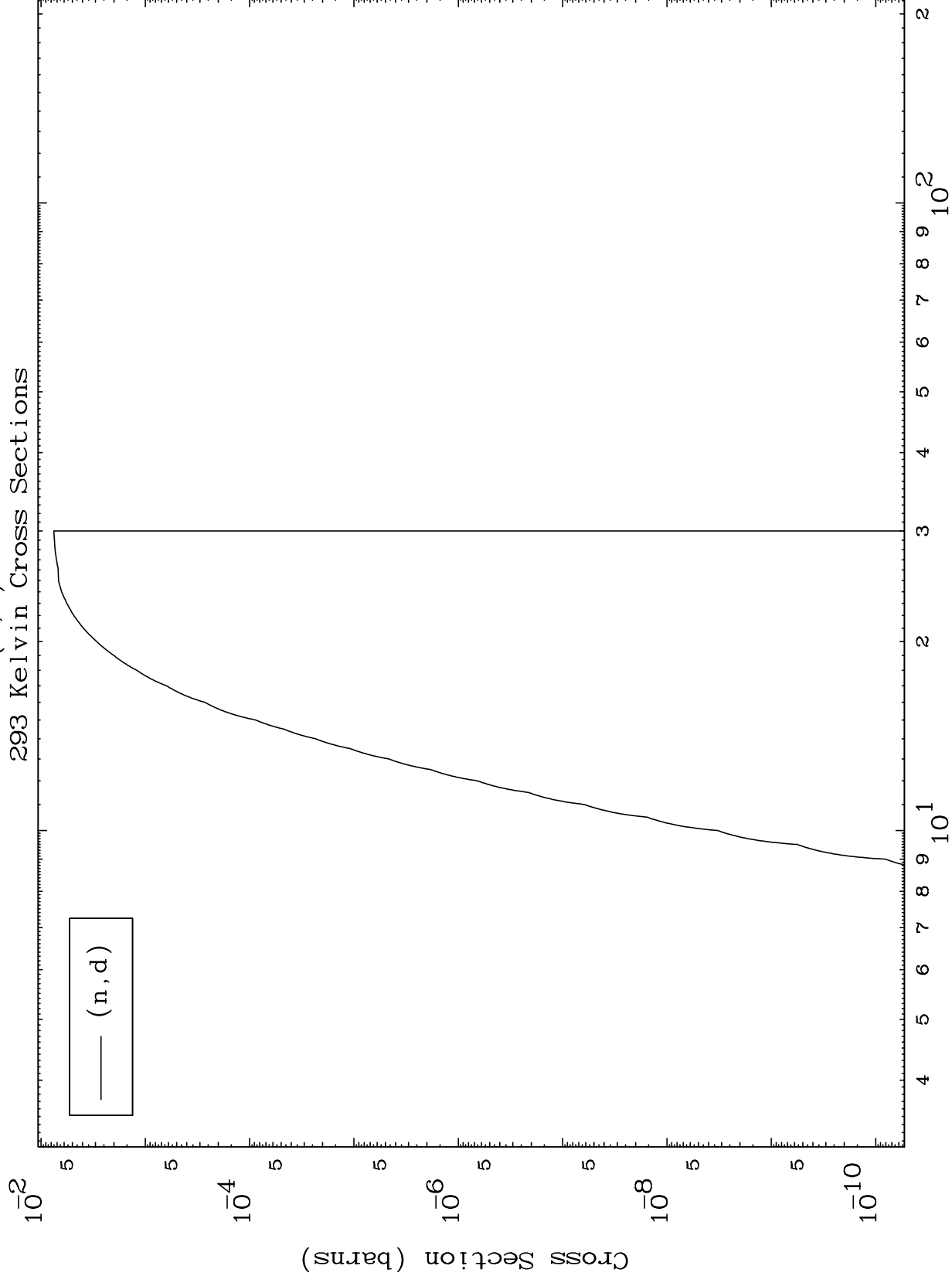
95-Am-247



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(n,d) Levels  
293 Kelvin Cross Sections

95-Am-247



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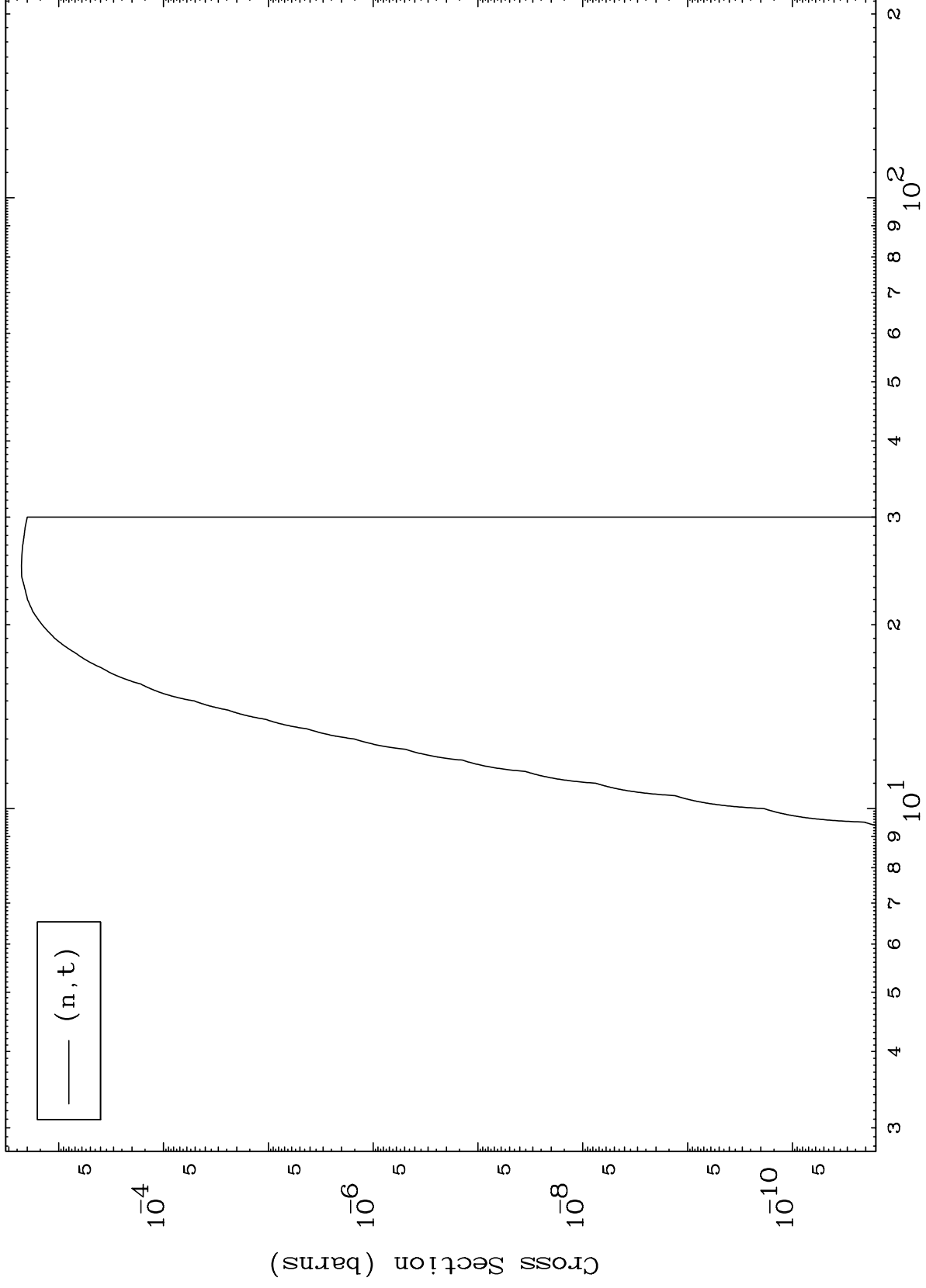
Incident Energy (MeV)

95-Am-247

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(n,t) Levels  
293 Kelvin Cross Sections

95-Am-247



(n,t)

Incident Energy (MeV)

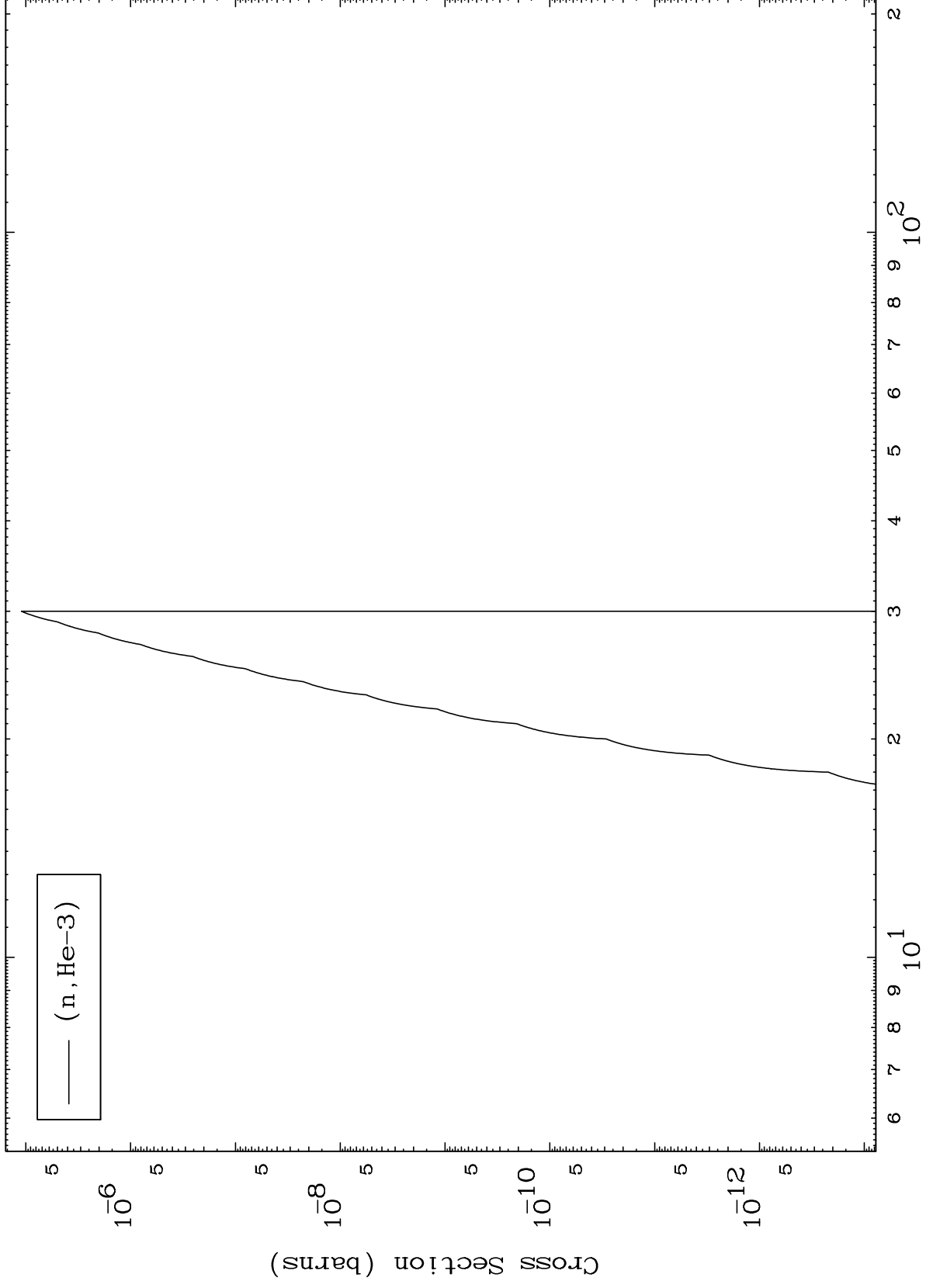
95-Am-247

11

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(n,He3) Levels  
293 Kelvin Cross Sections

95-Am-247



12

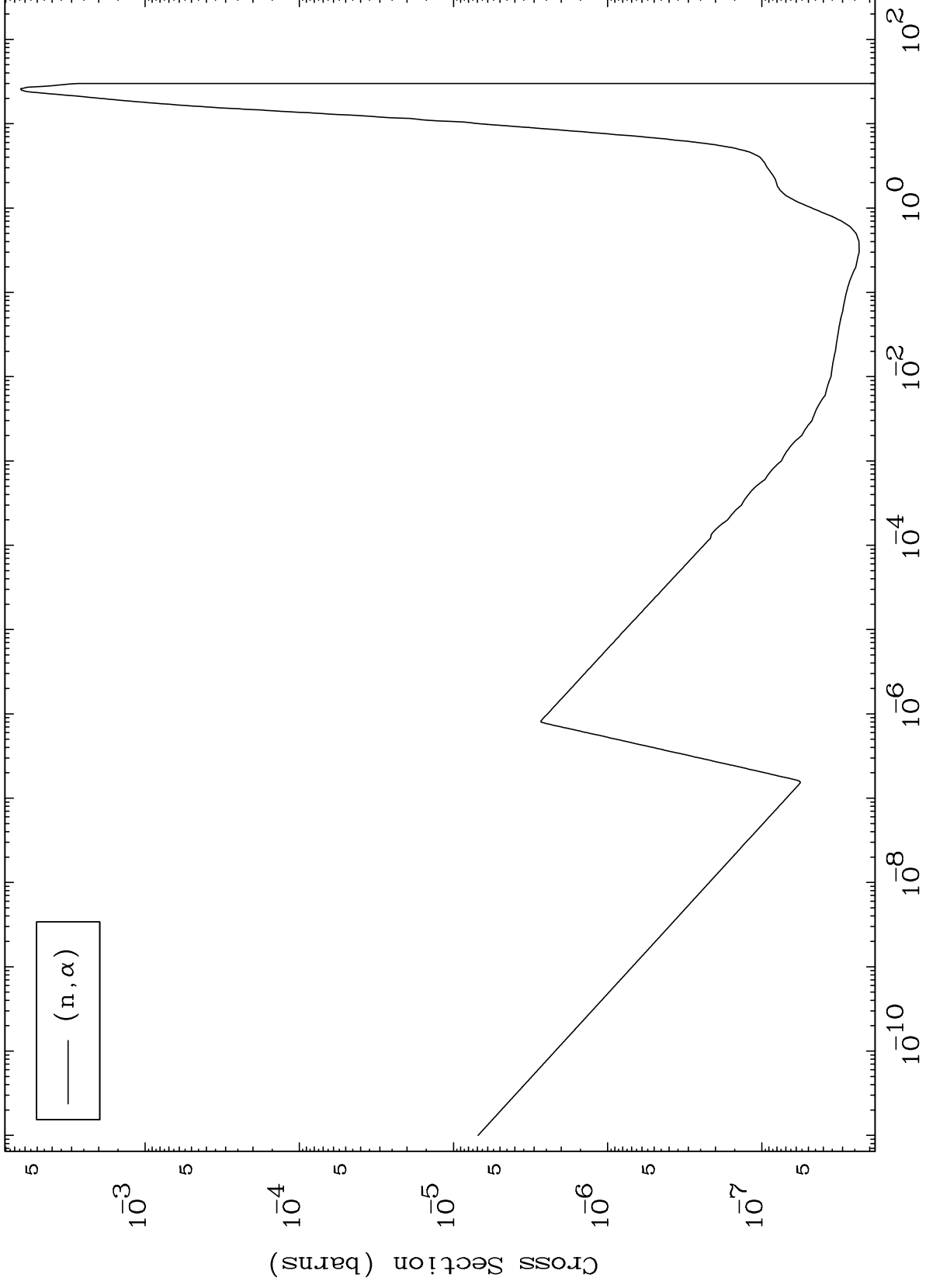
Incident Energy (MeV)

95-Am-247

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

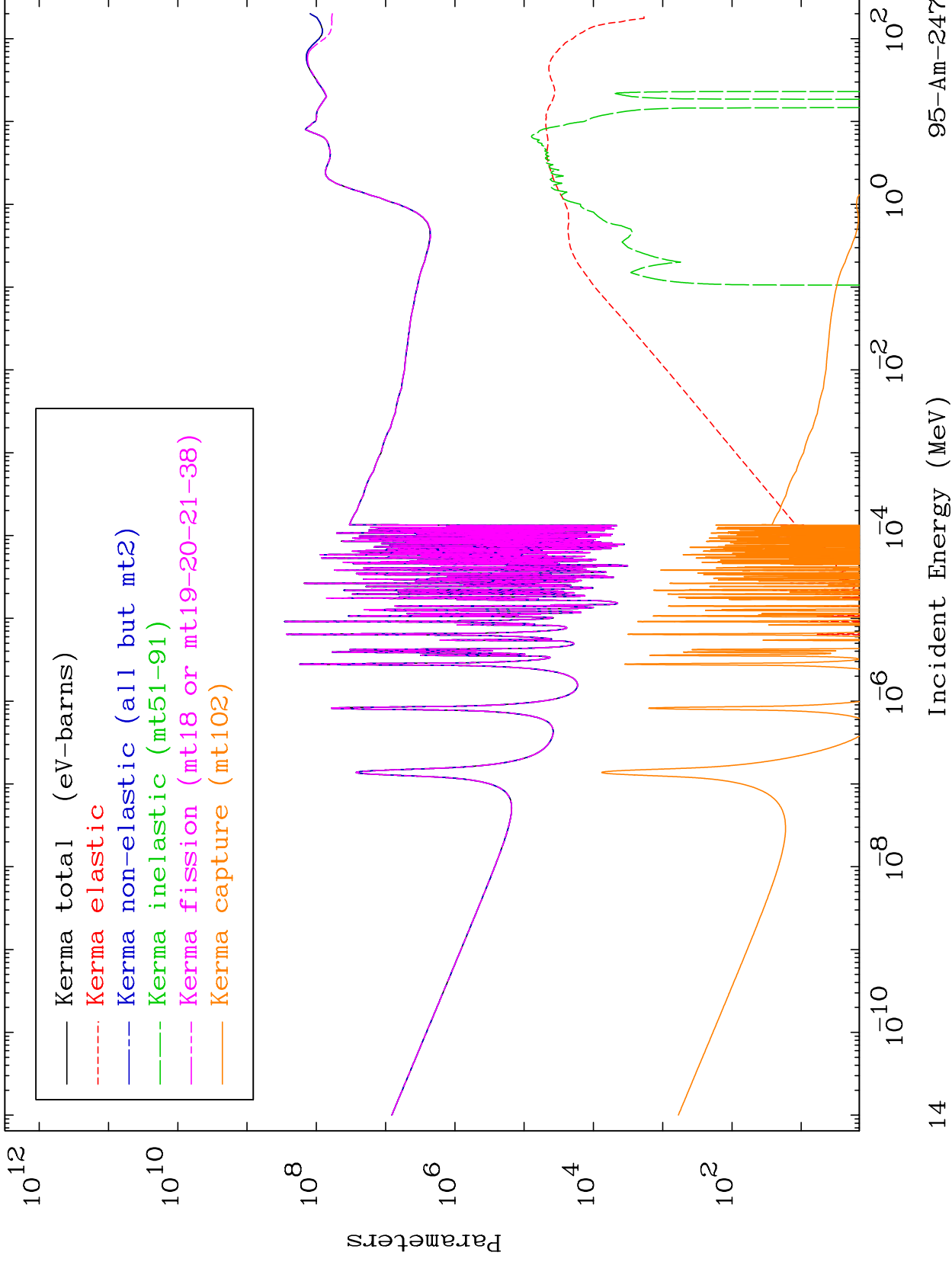
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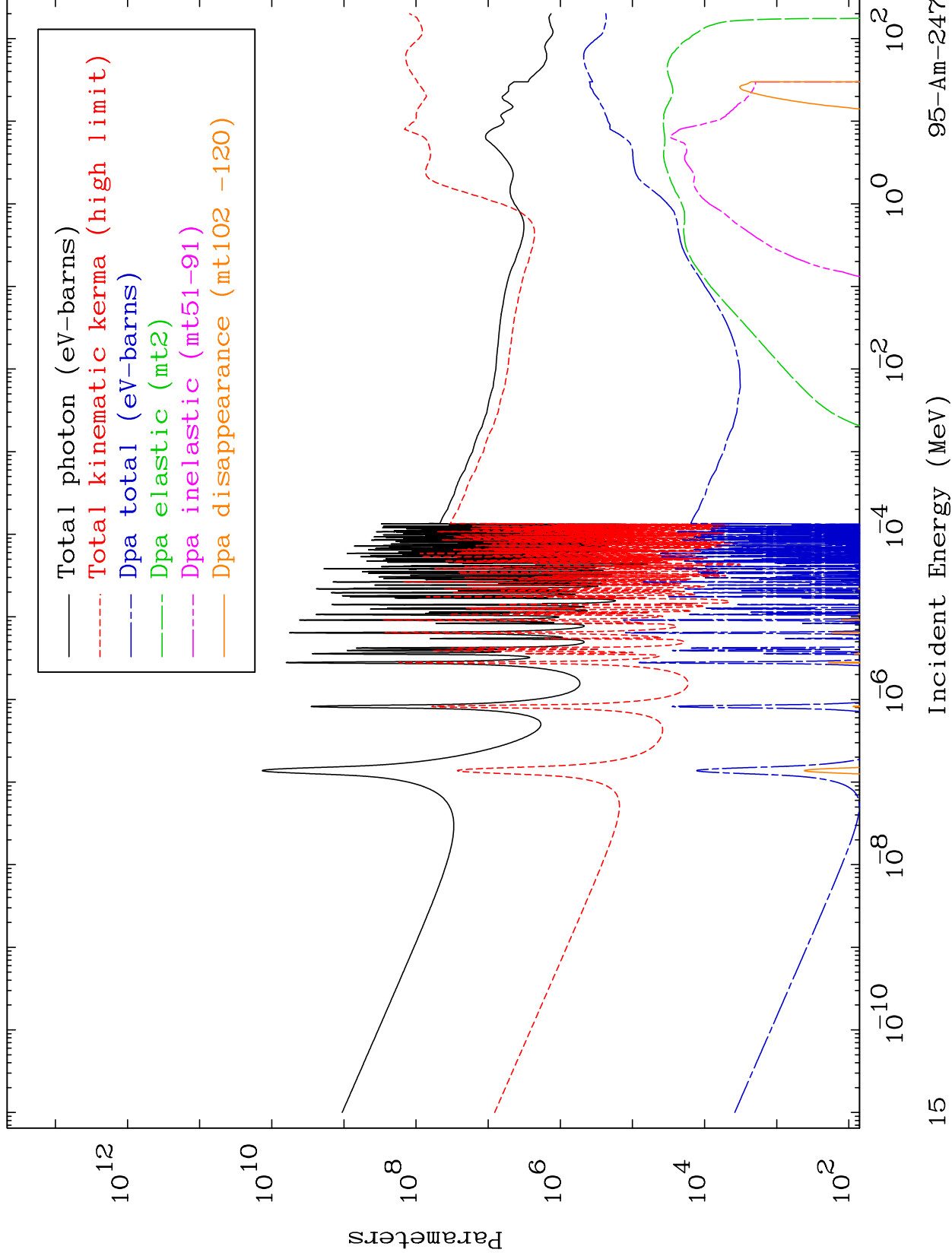


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Incident Energy (MeV)

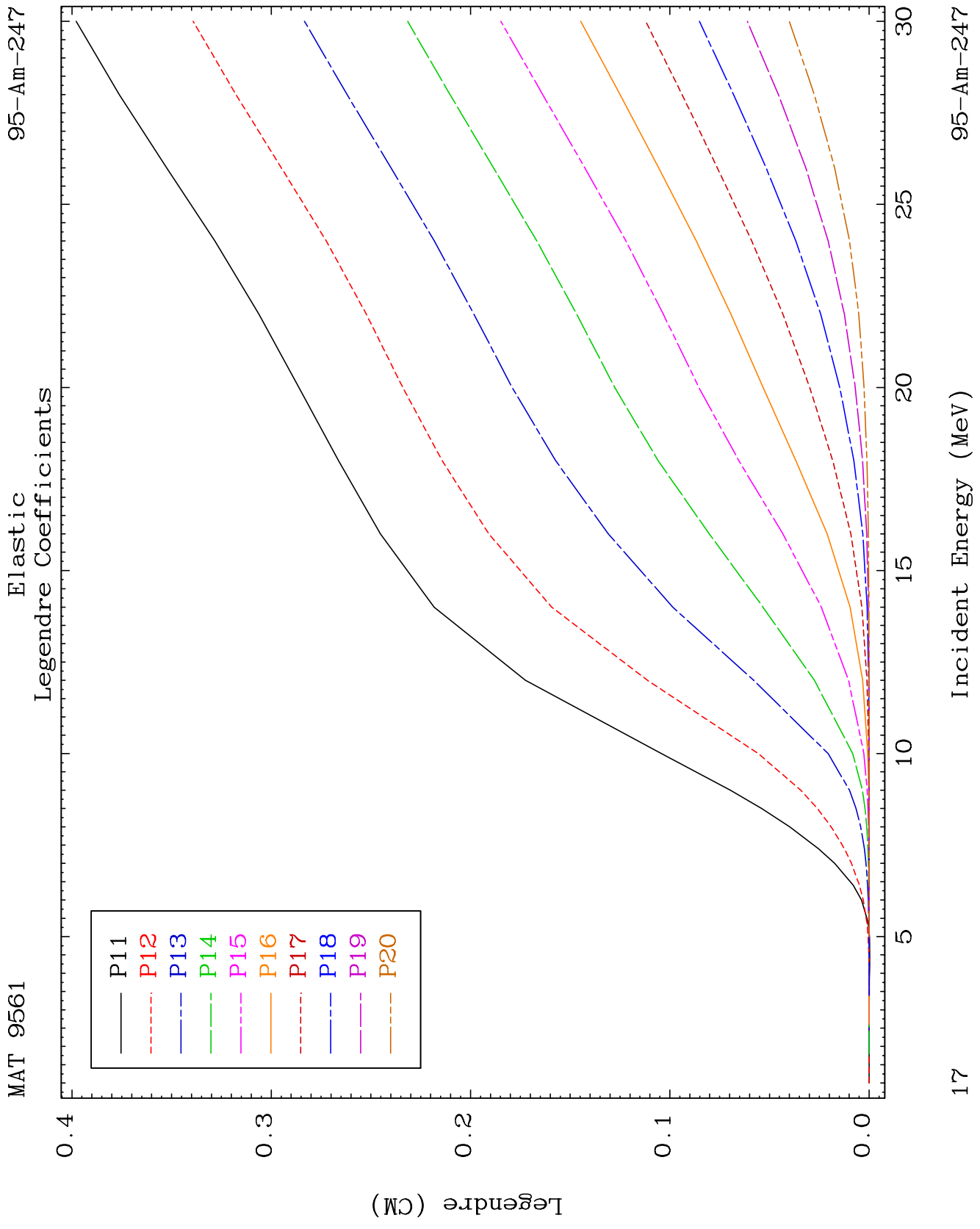
95-Am-247







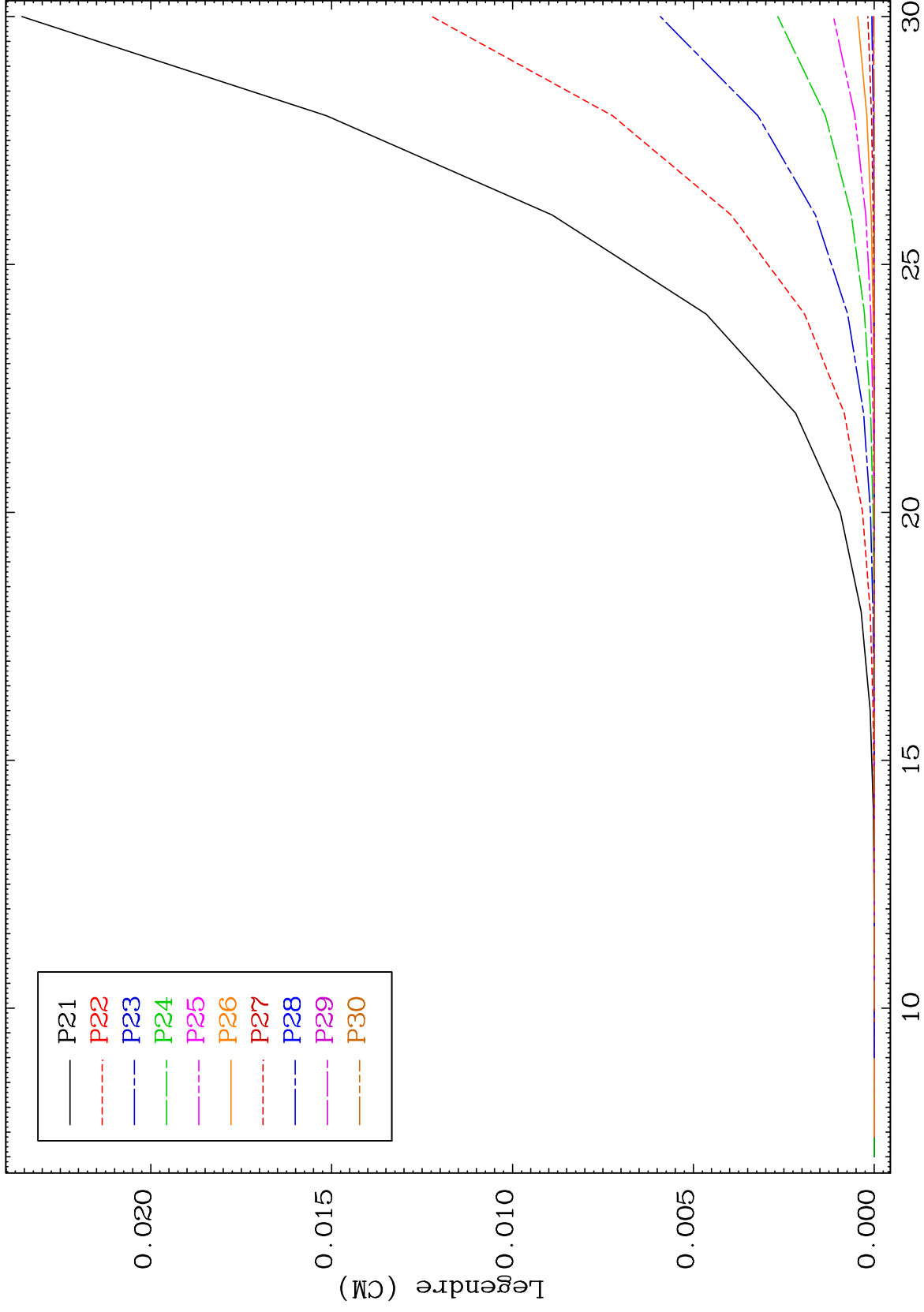




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### Elastic Legendre Coefficients

95-Am-247



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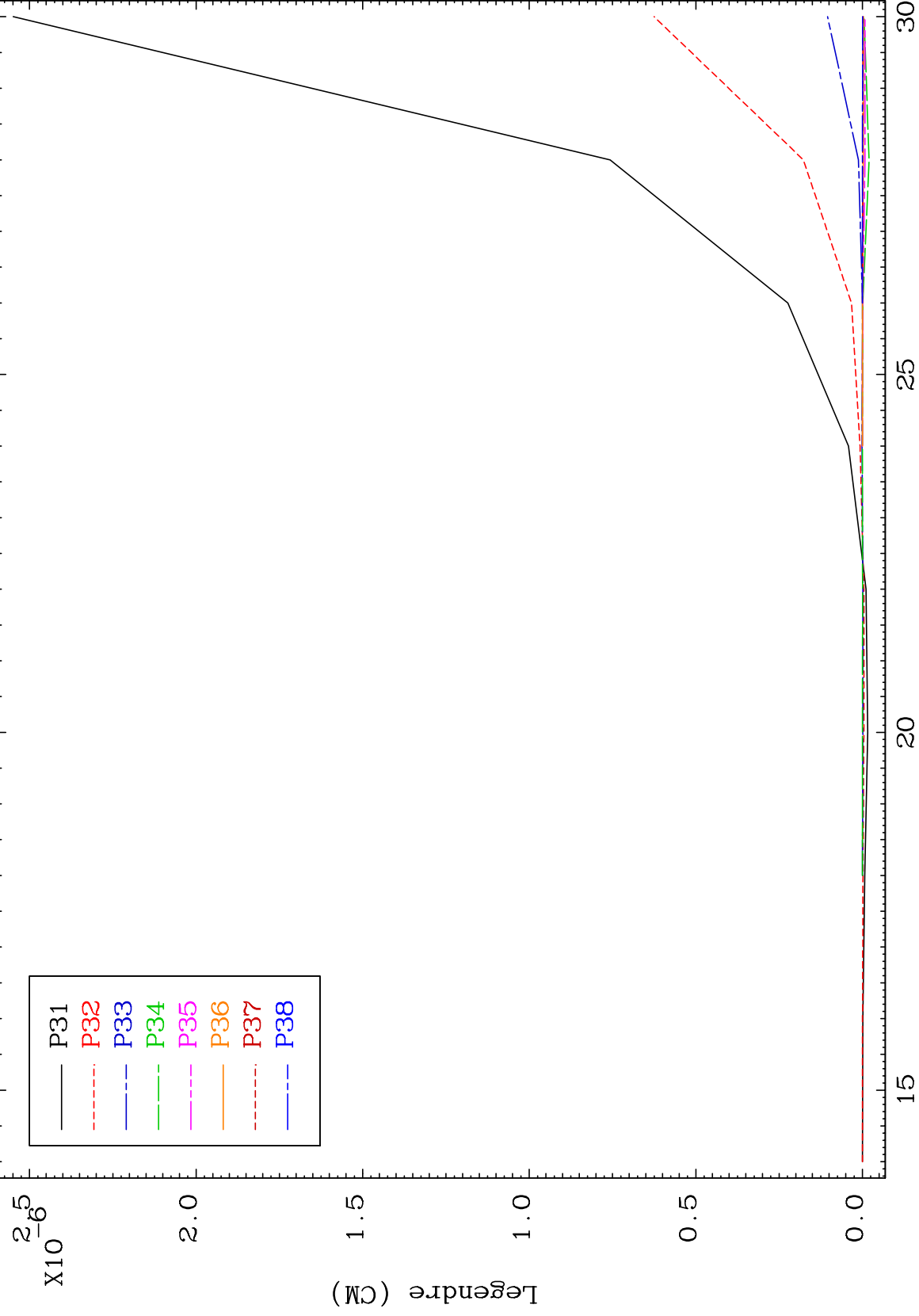
Incident Energy (MeV)

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Elastic  
Legendre Coefficients

95-Am-247

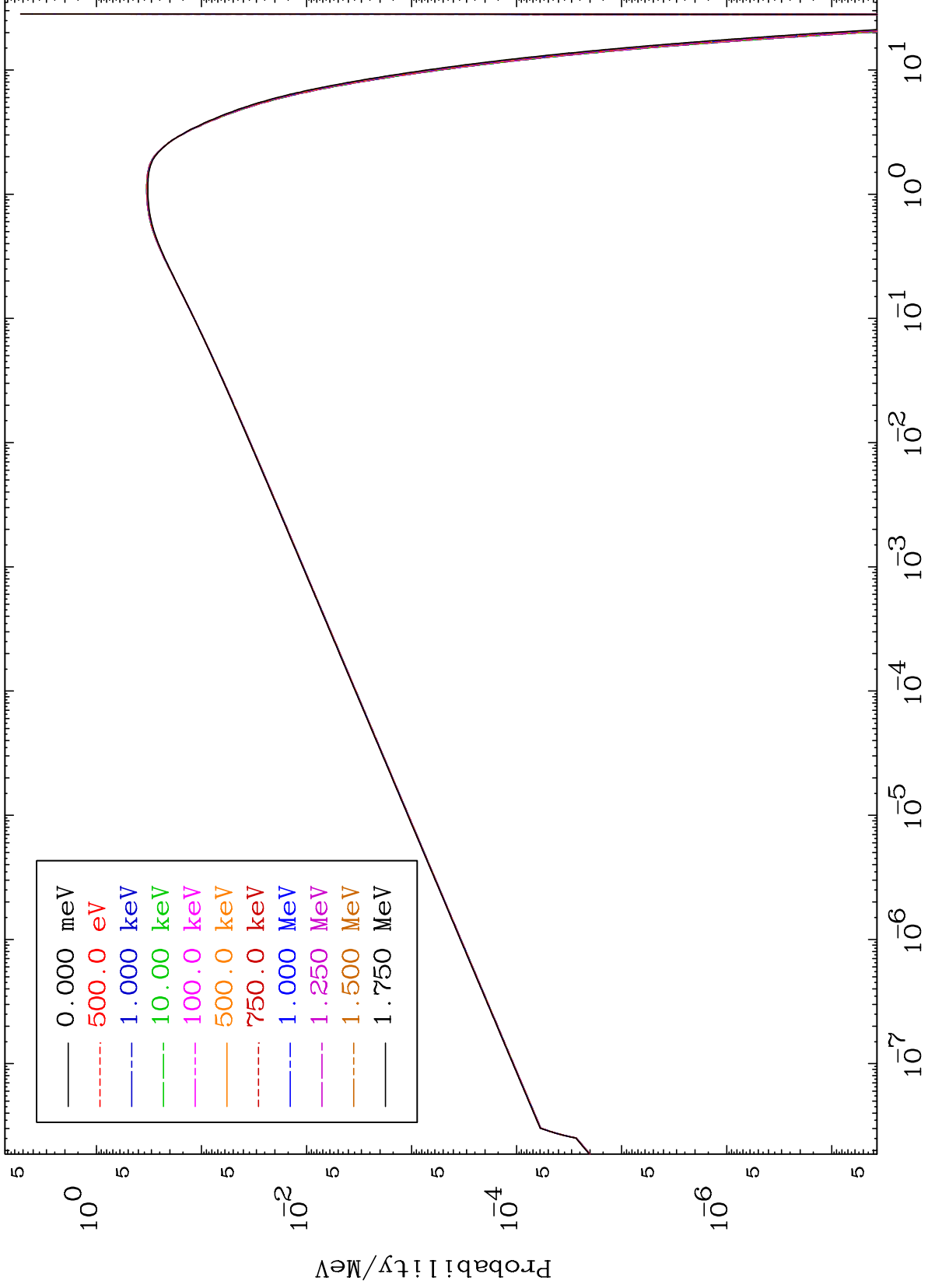


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Incident Energy (MeV)

95-Am-247

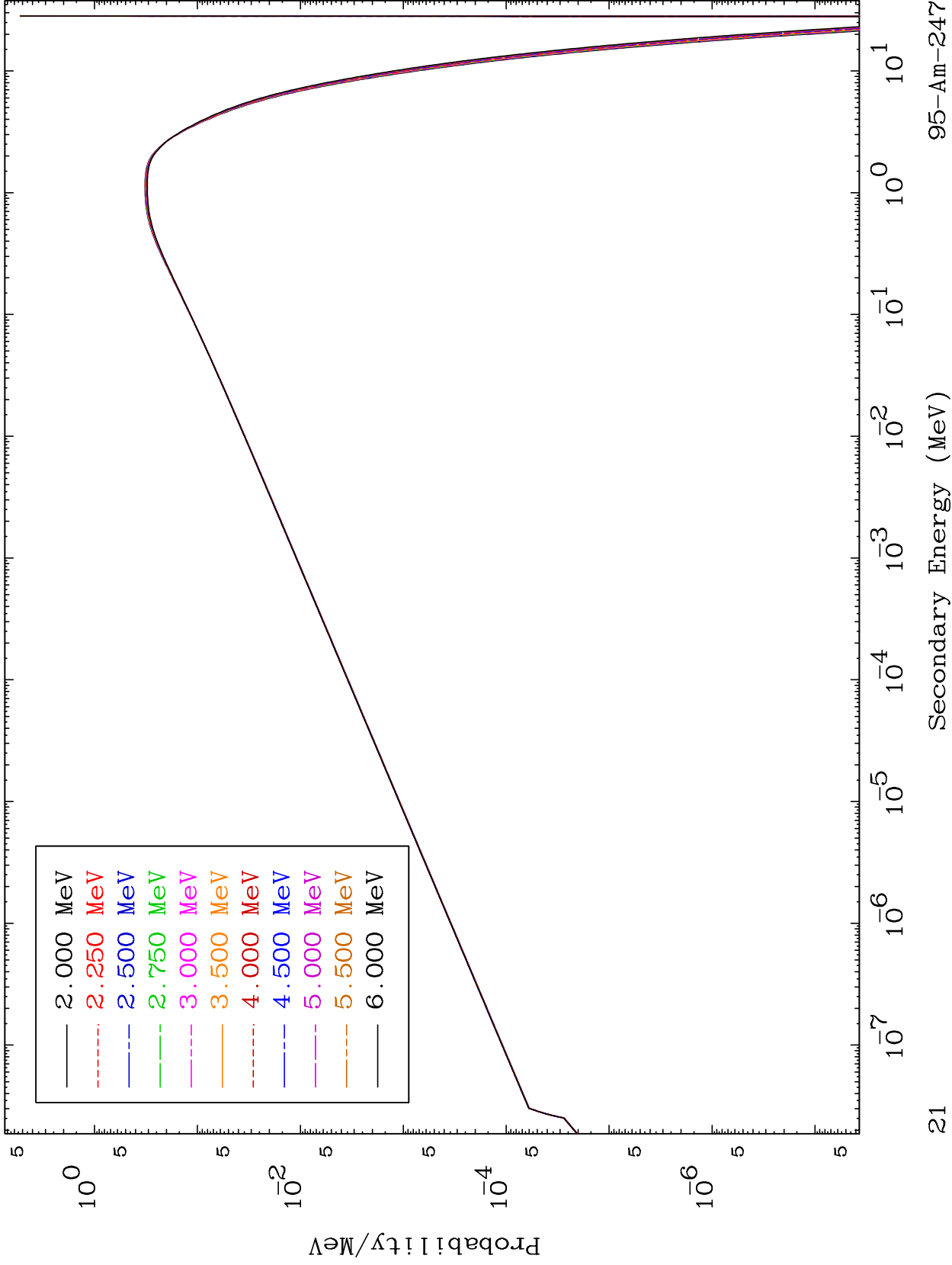
Fission Energy Distribution



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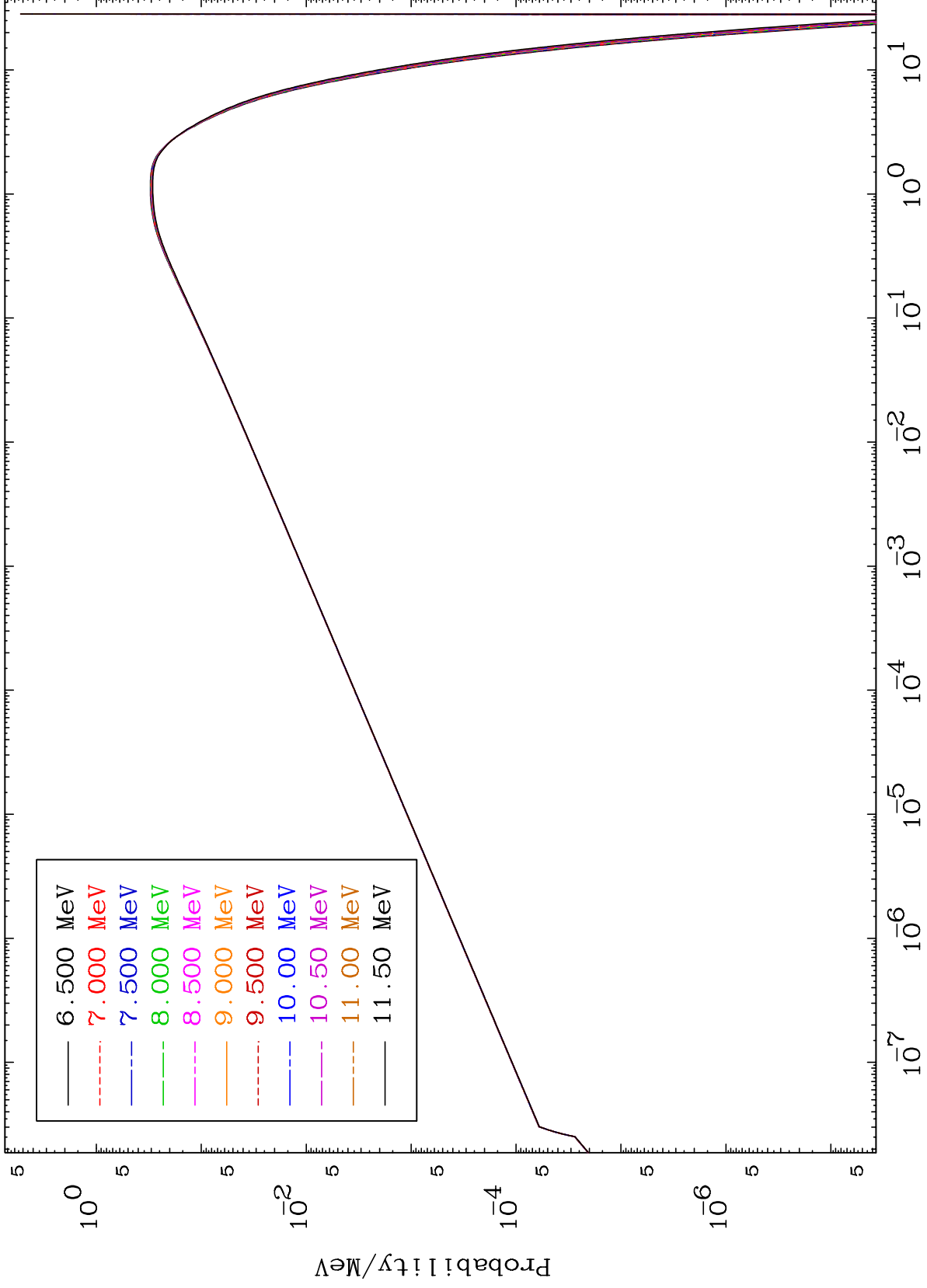
### Fission Energy Distribution

95-Am-247



95-Am-247

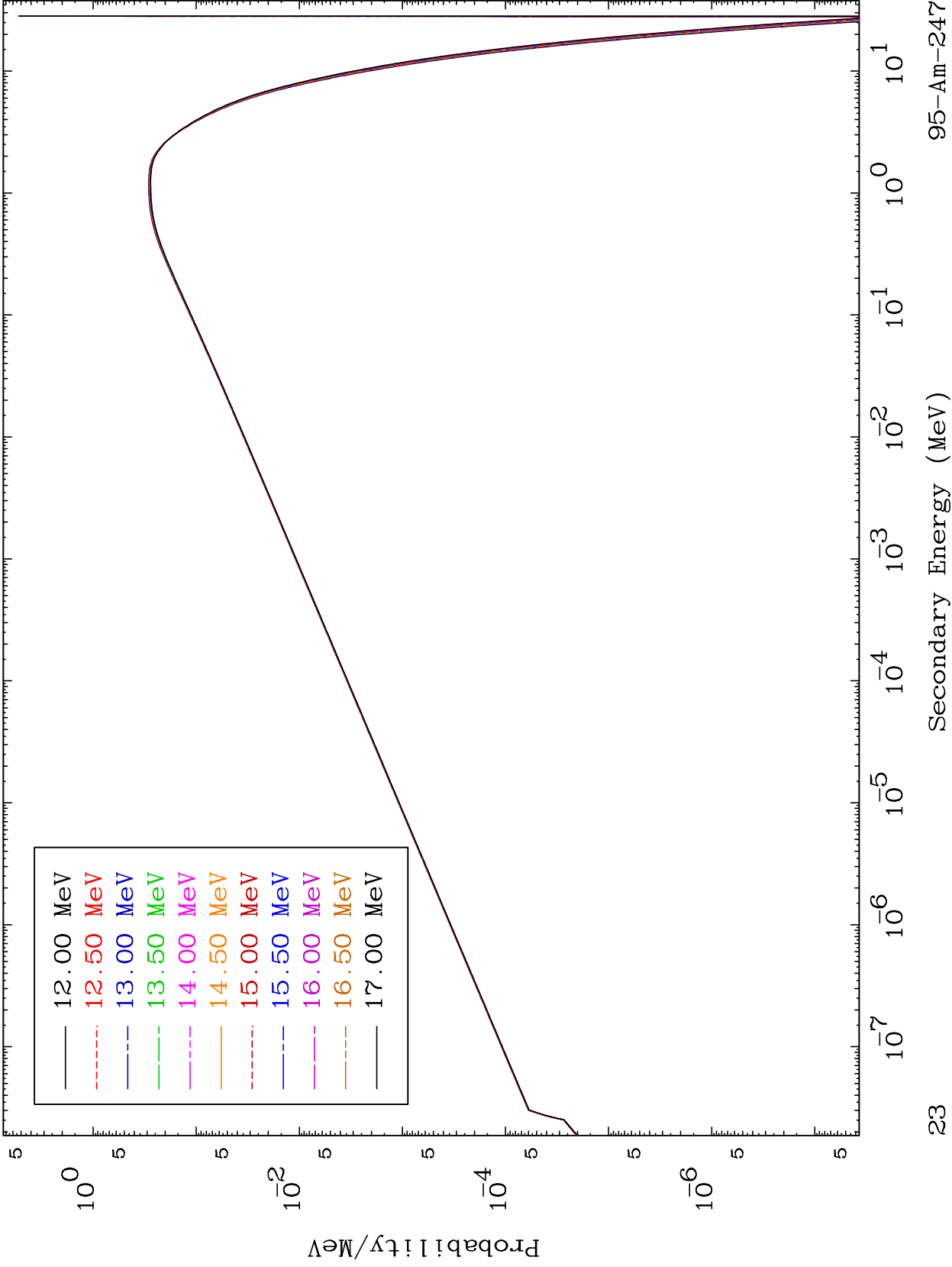
Fission  
Energy Distribution



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### Fission Energy Distribution

95-Am-247

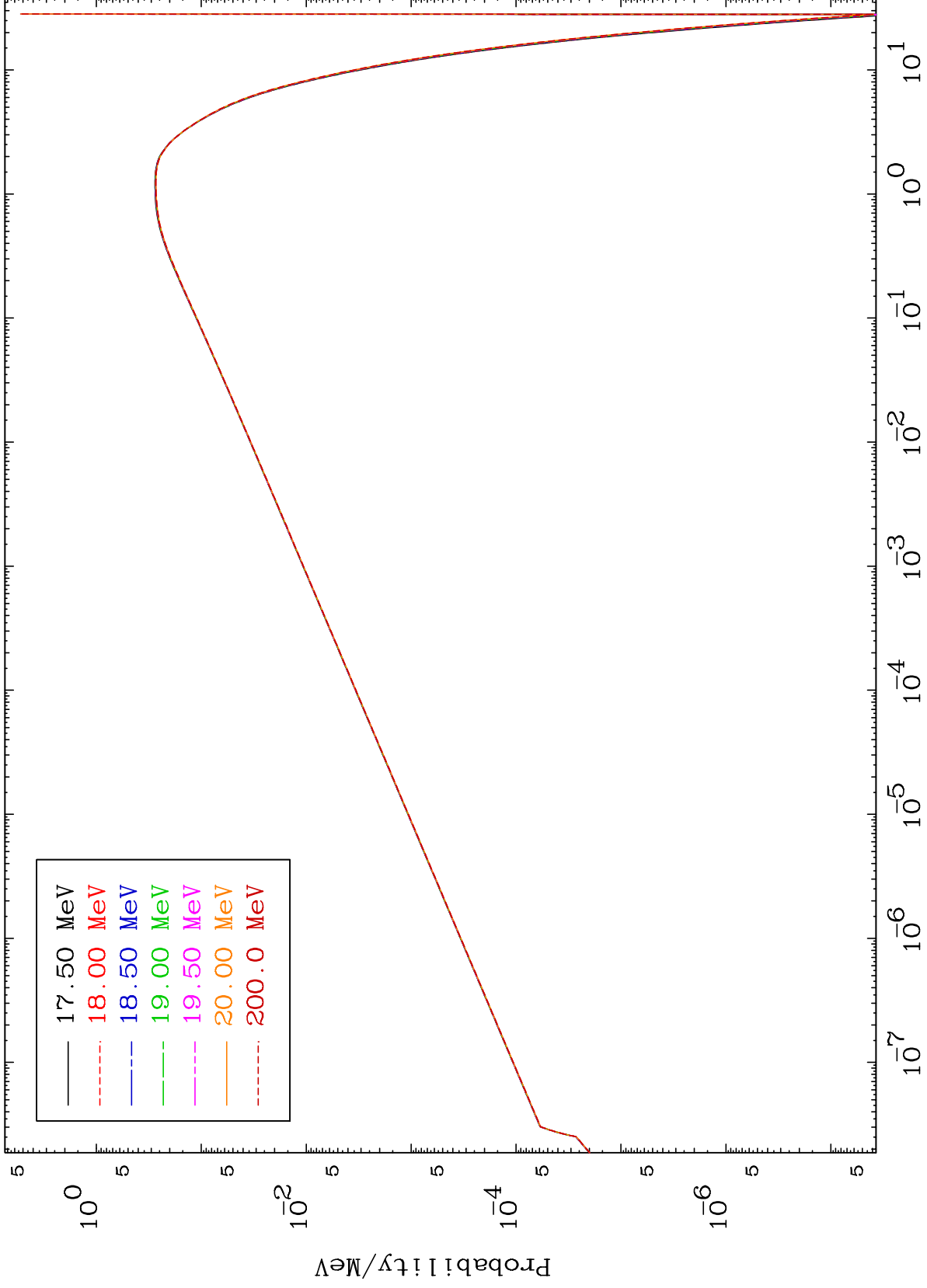




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### Fission Energy Distribution

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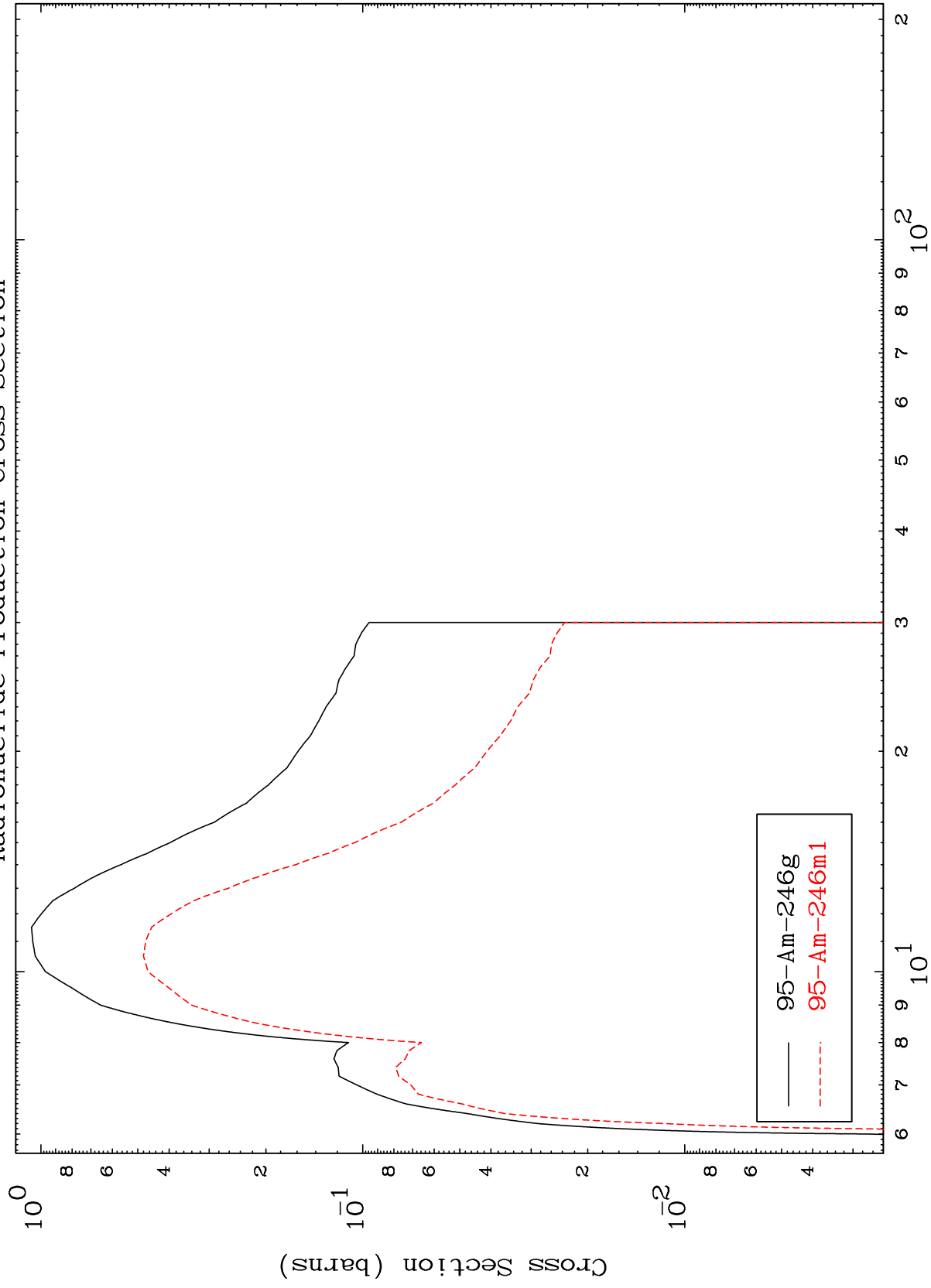
Secondary Energy (MeV)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

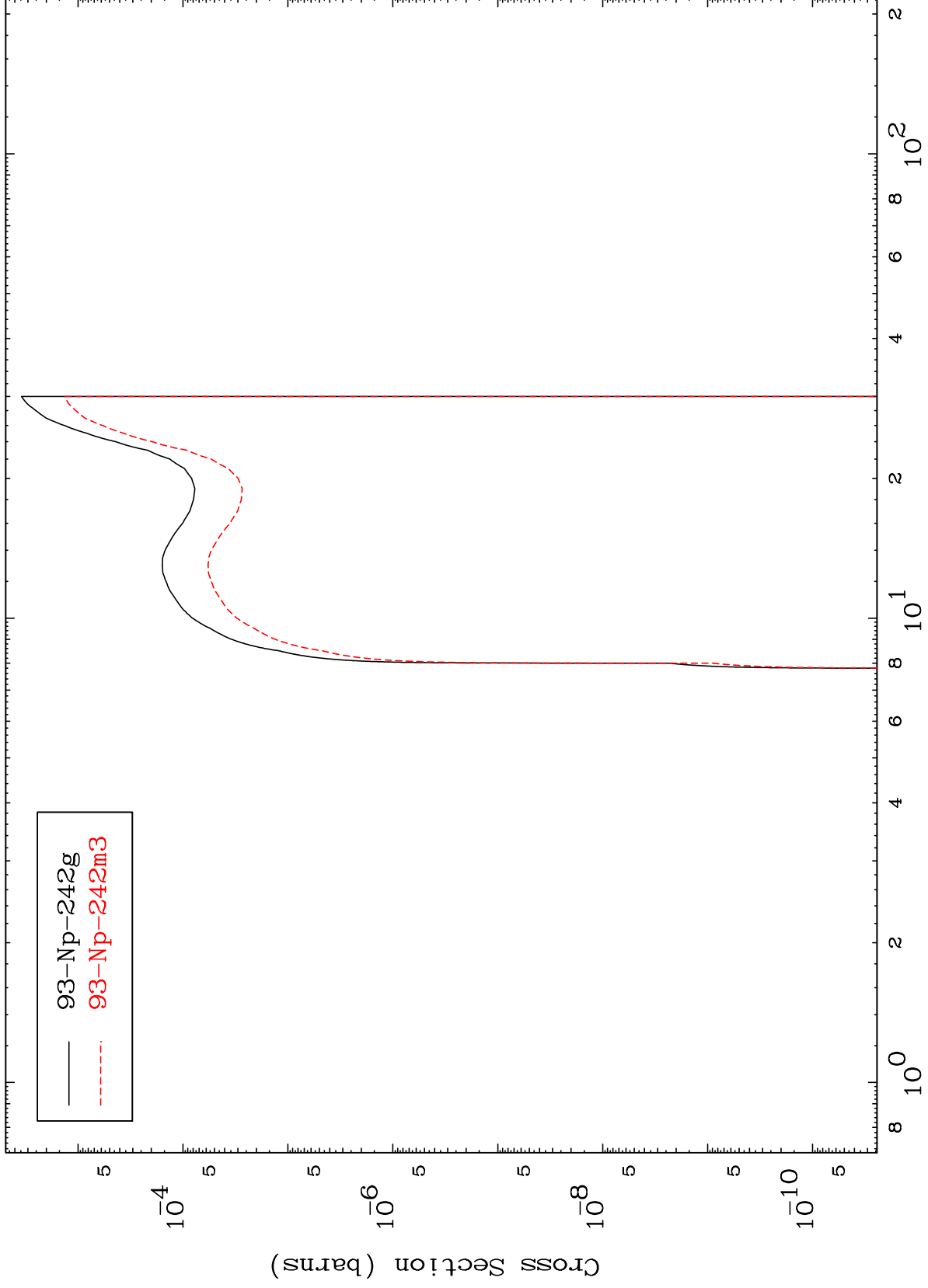
25

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(n,2n)  $\alpha$

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Radionuclide Production Cross Section



26

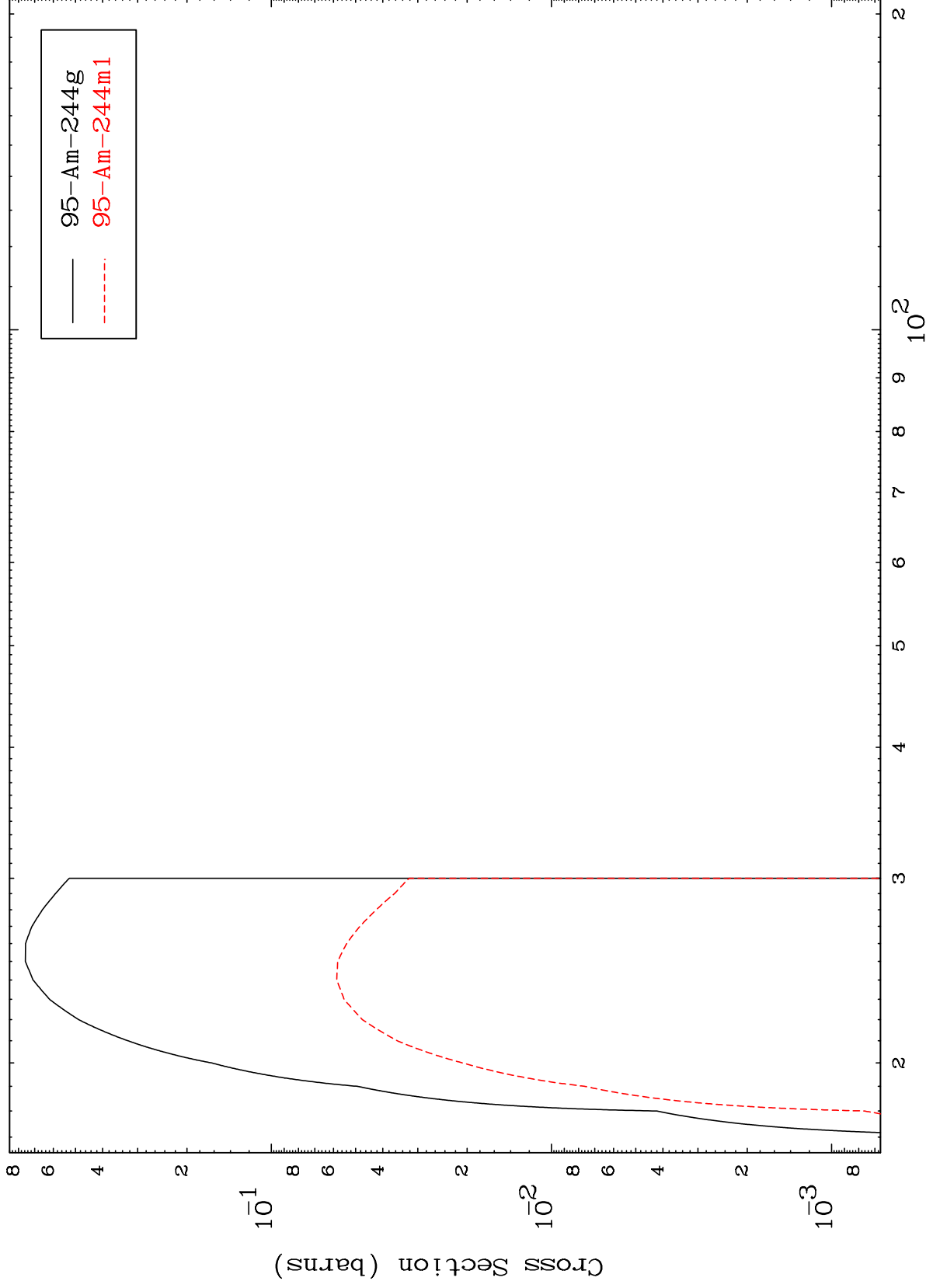
Incident Energy (MeV)

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(n,4n)  
Radionuclide Production Cross Section



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Incident Energy (MeV)

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