

Program EVALPLOT
(Version 2015-2)

by

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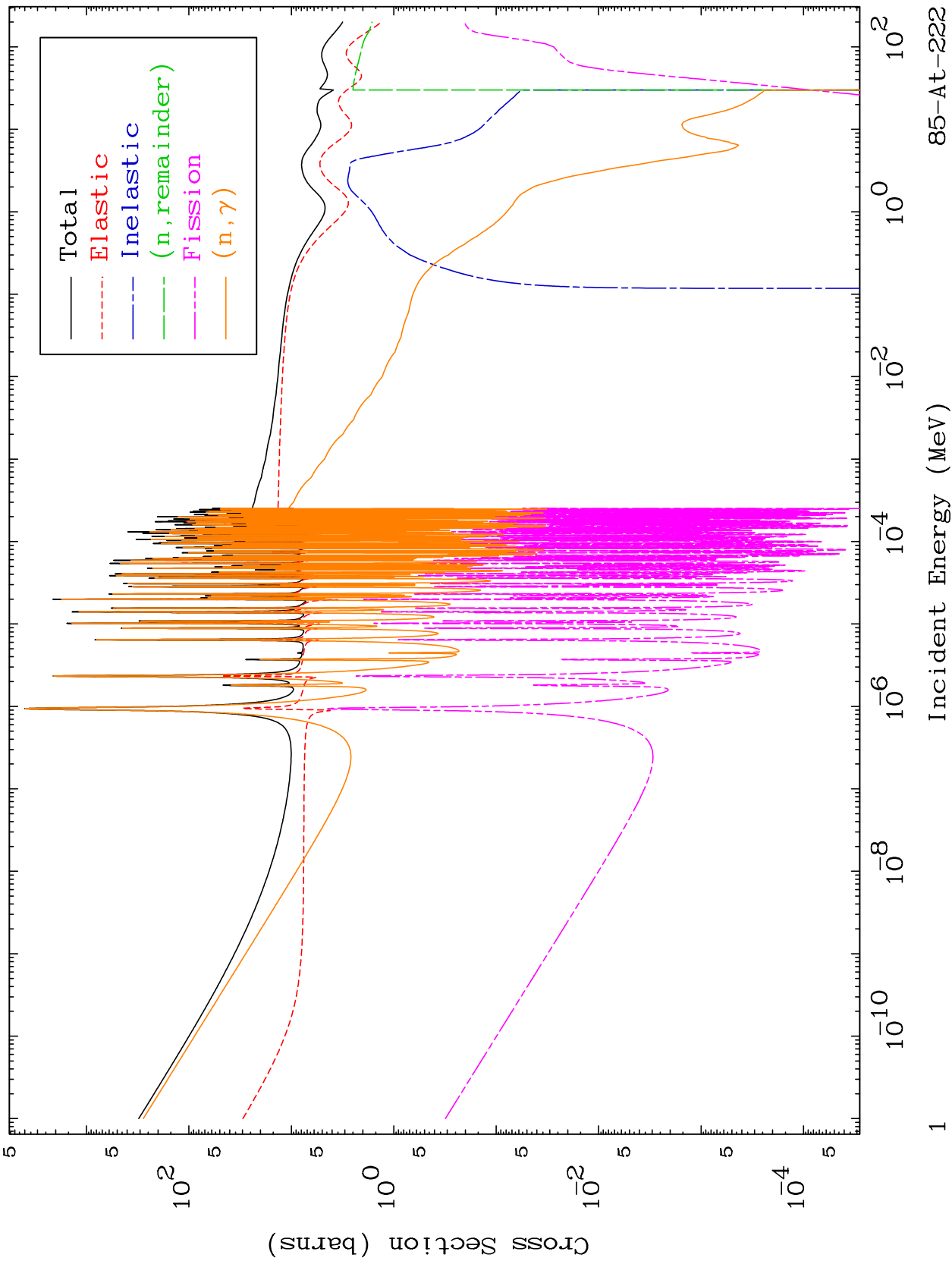
Press Mouse Button to Start

MAT 8582

Major

293 Kelvin Cross Sections

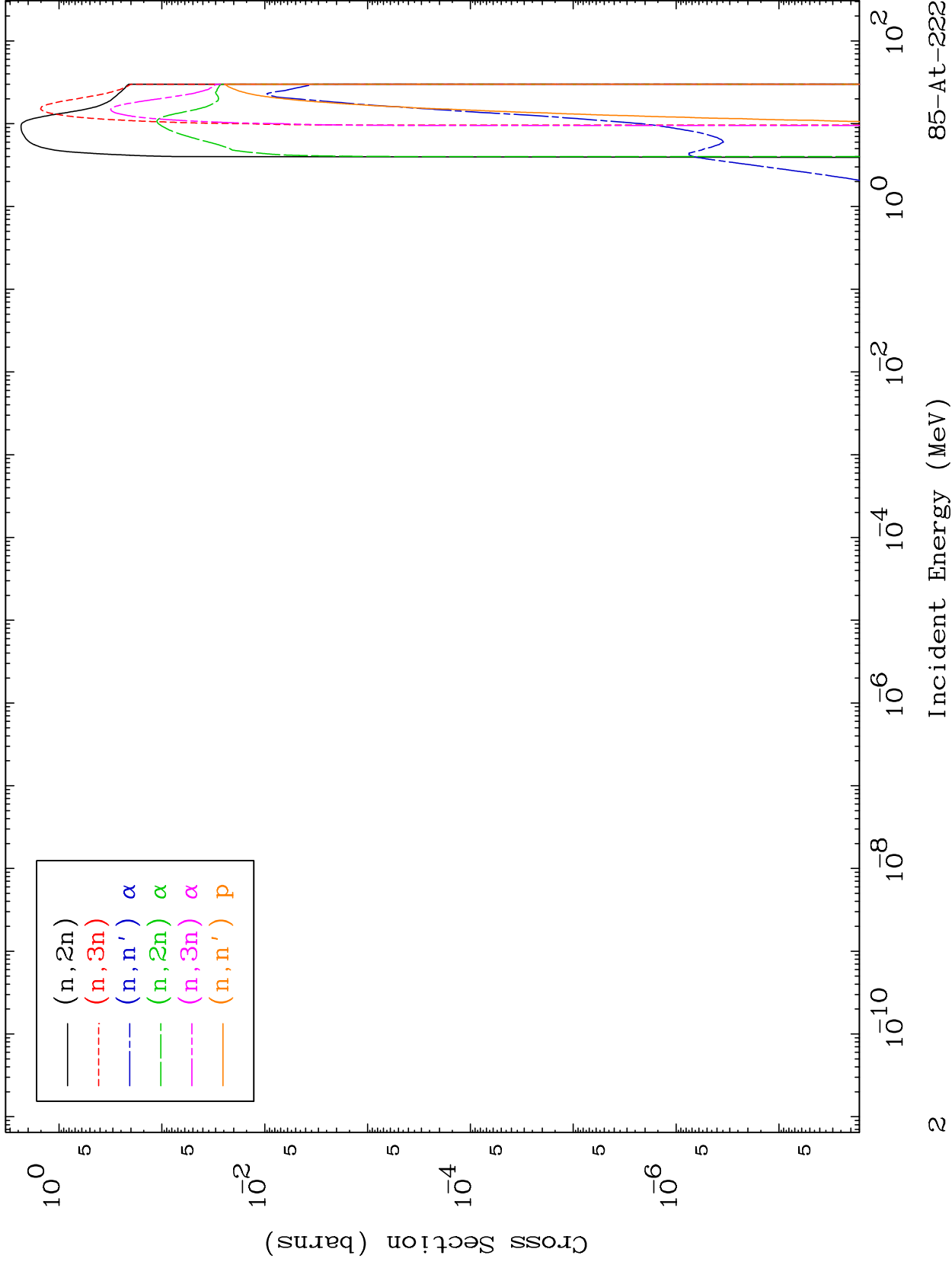
85-At-222



MAT 8582

Neutron Production
293 Kelvin Cross Sections

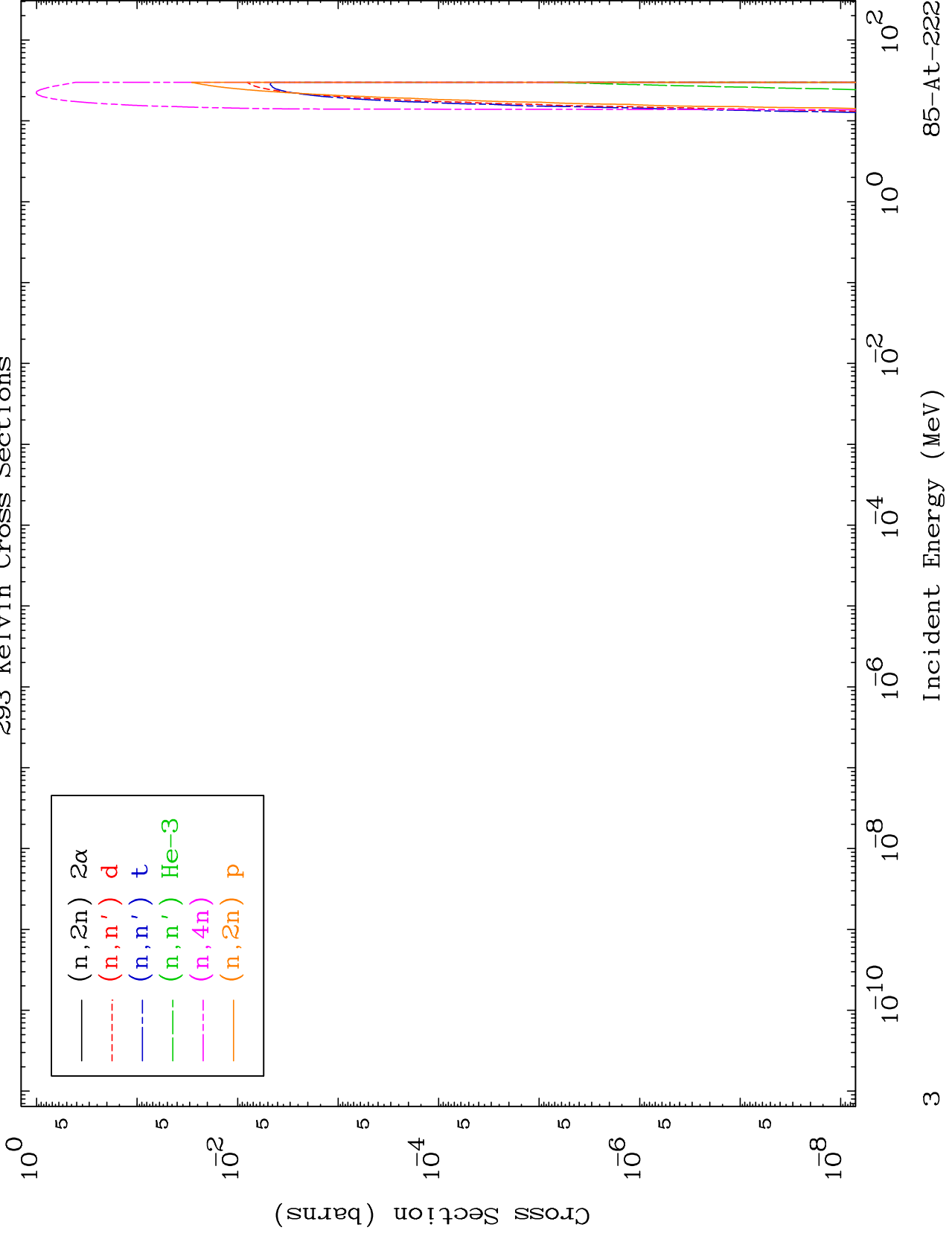
85-At-222



MAT 8582

Neutron Production
293 Kelvin Cross Sections

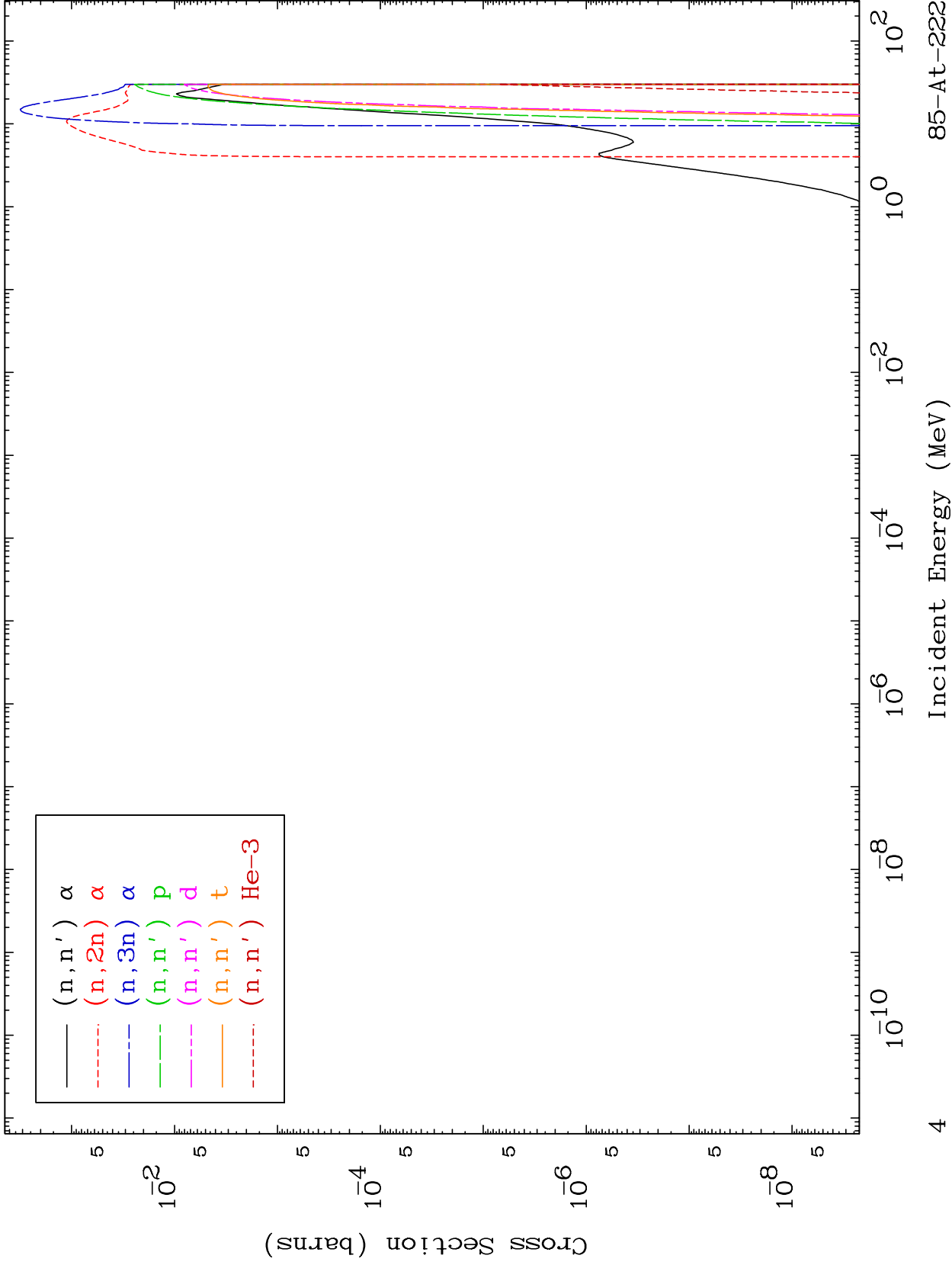
85-At-222

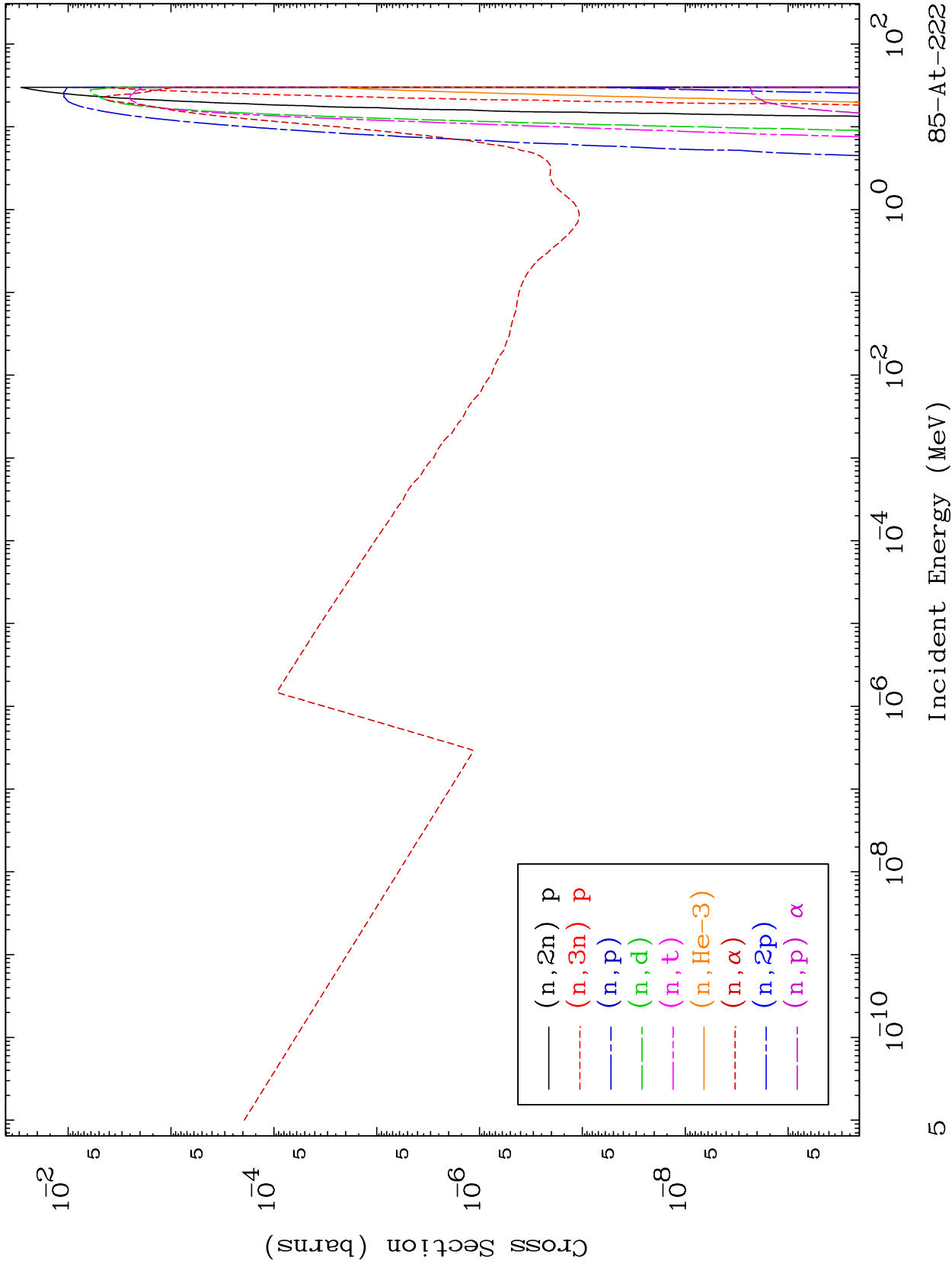


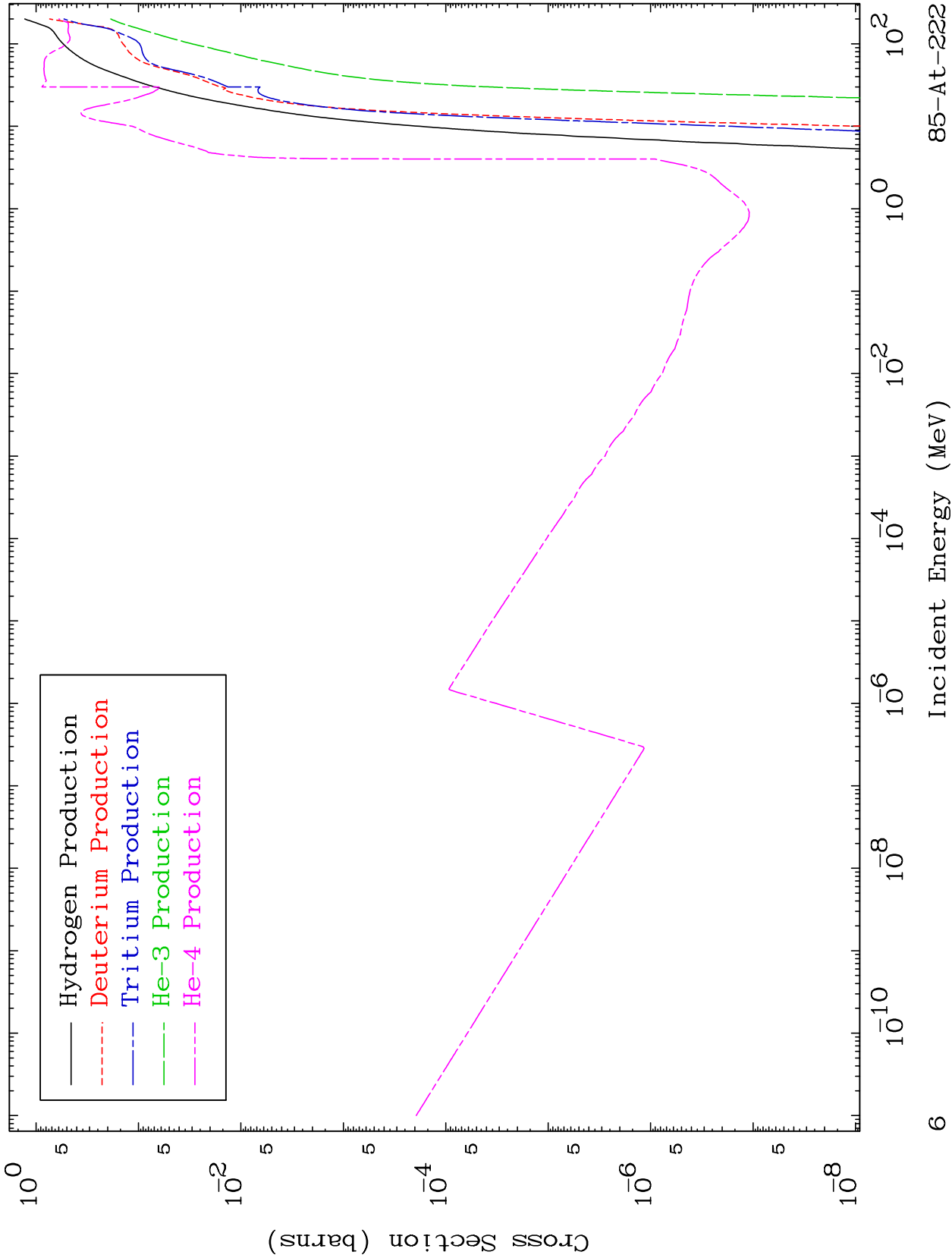
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Charged Particle
293 Kelvin Cross Sections

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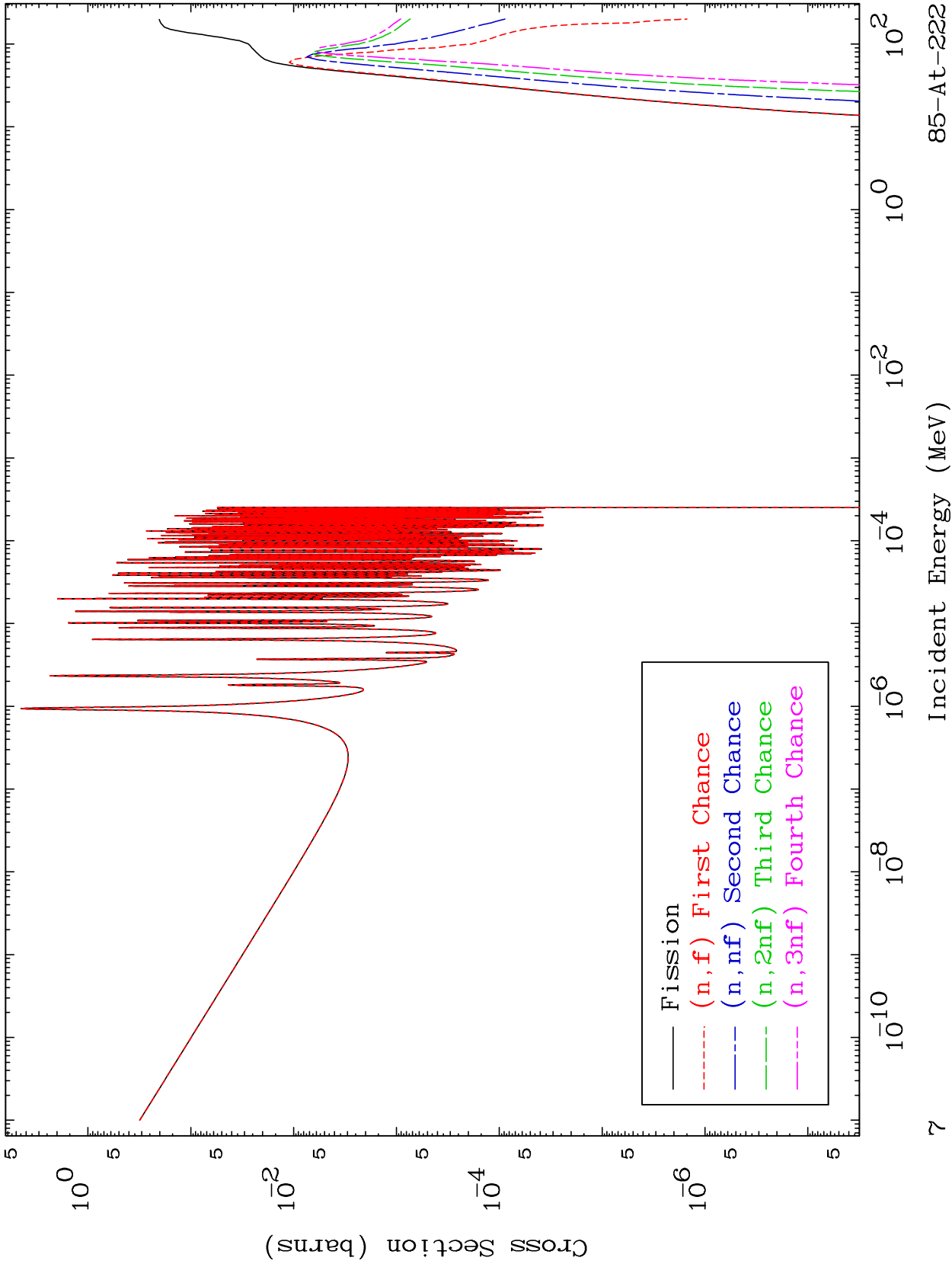


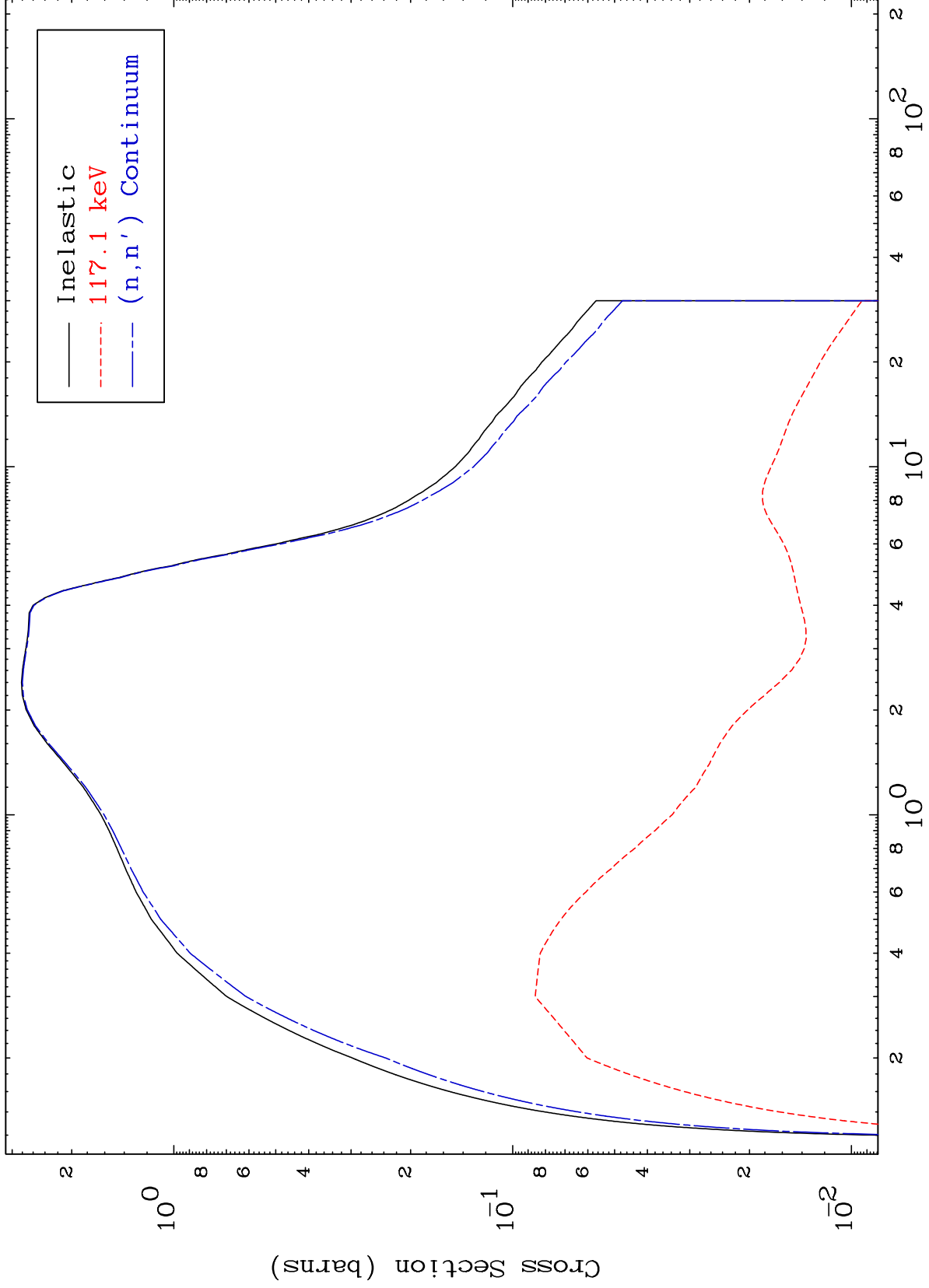


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Fission
293 Kelvin Cross Sections

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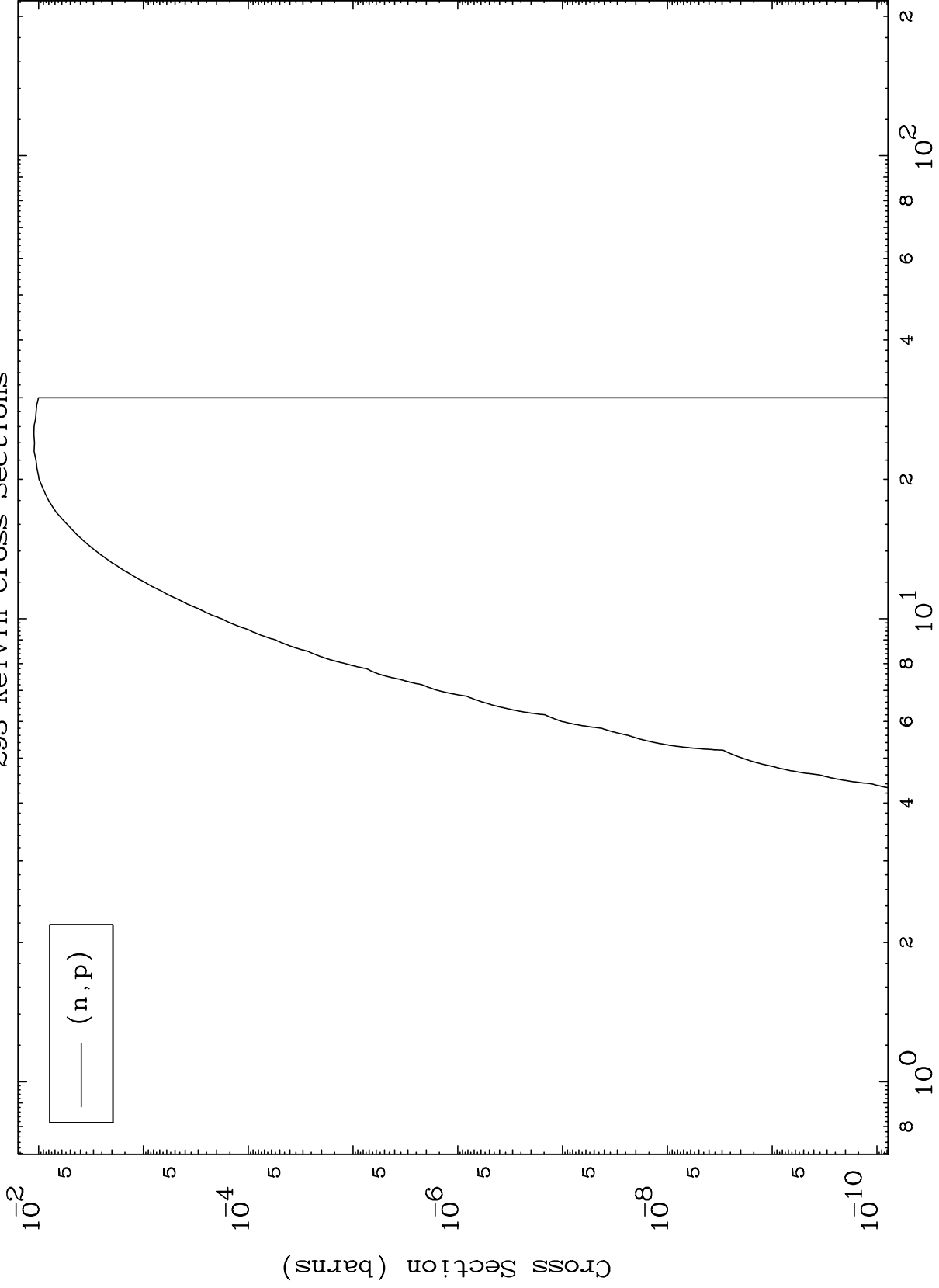




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(n,p) Levels
293 Kelvin Cross Sections

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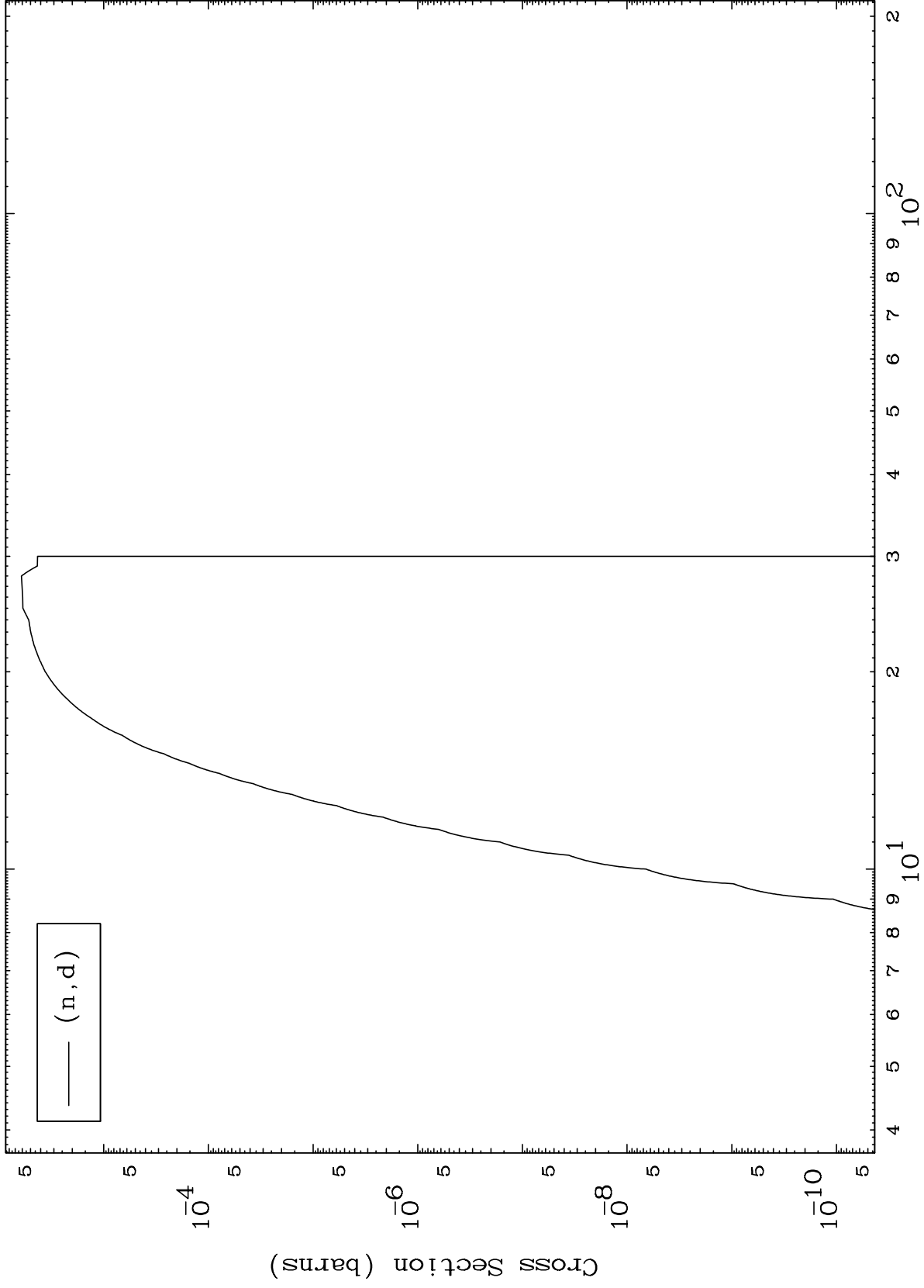
Incident Energy (MeV)

85-At-222

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(n,d) Levels
293 Kelvin Cross Sections

85-At-222



10

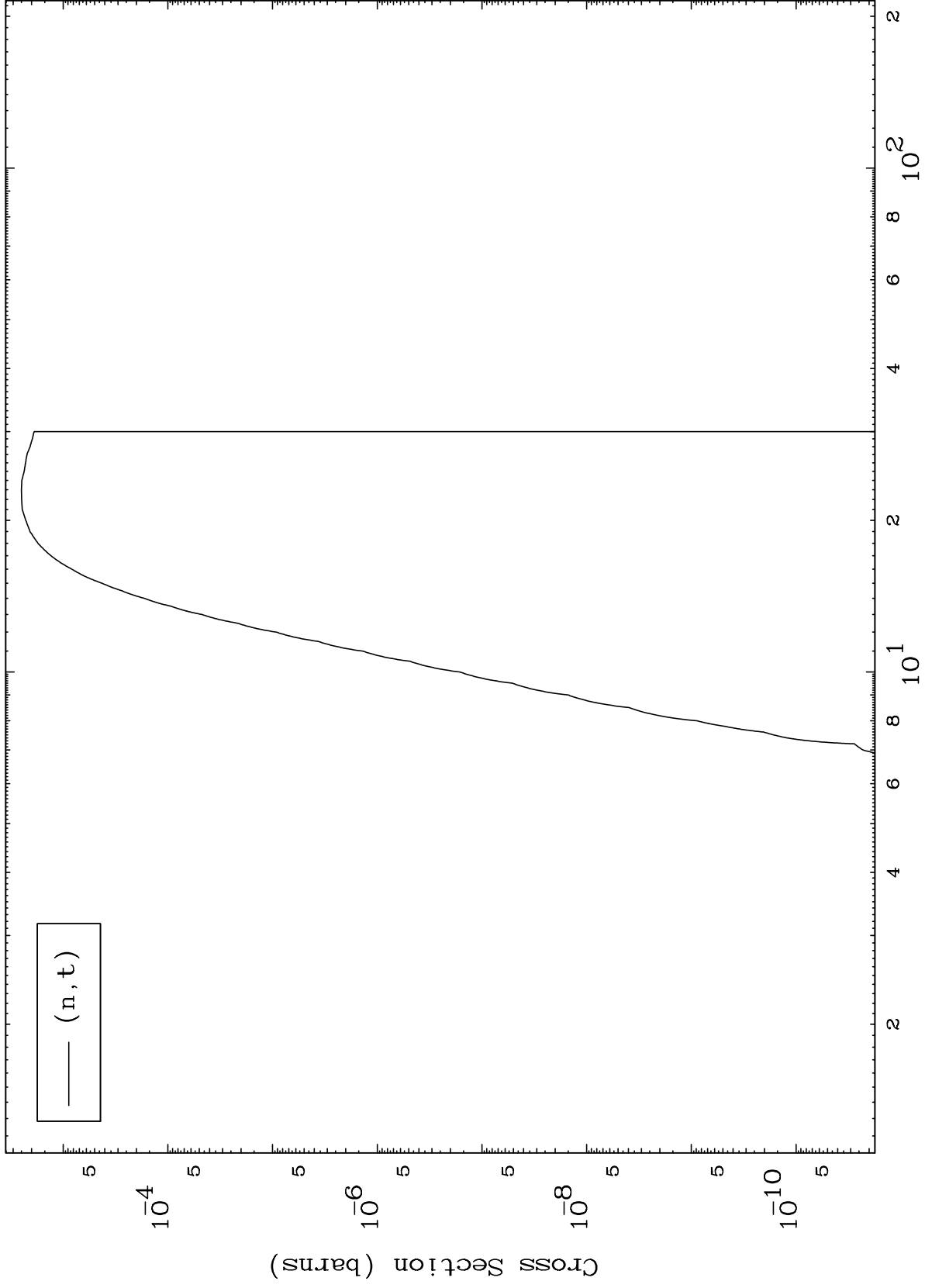
Incident Energy (MeV)

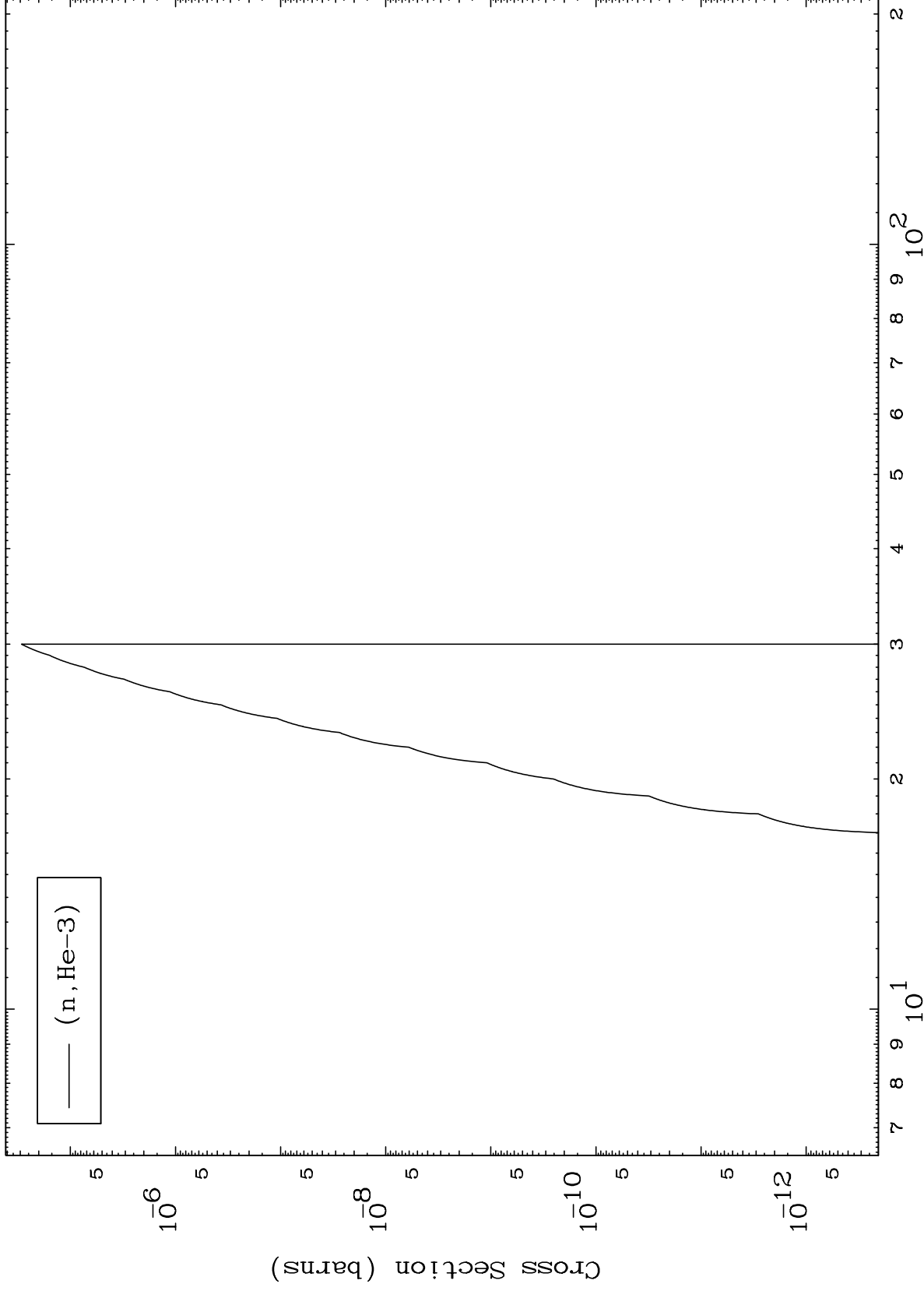
85-At-222

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(n,t) Levels
293 Kelvin Cross Sections

85-At-222

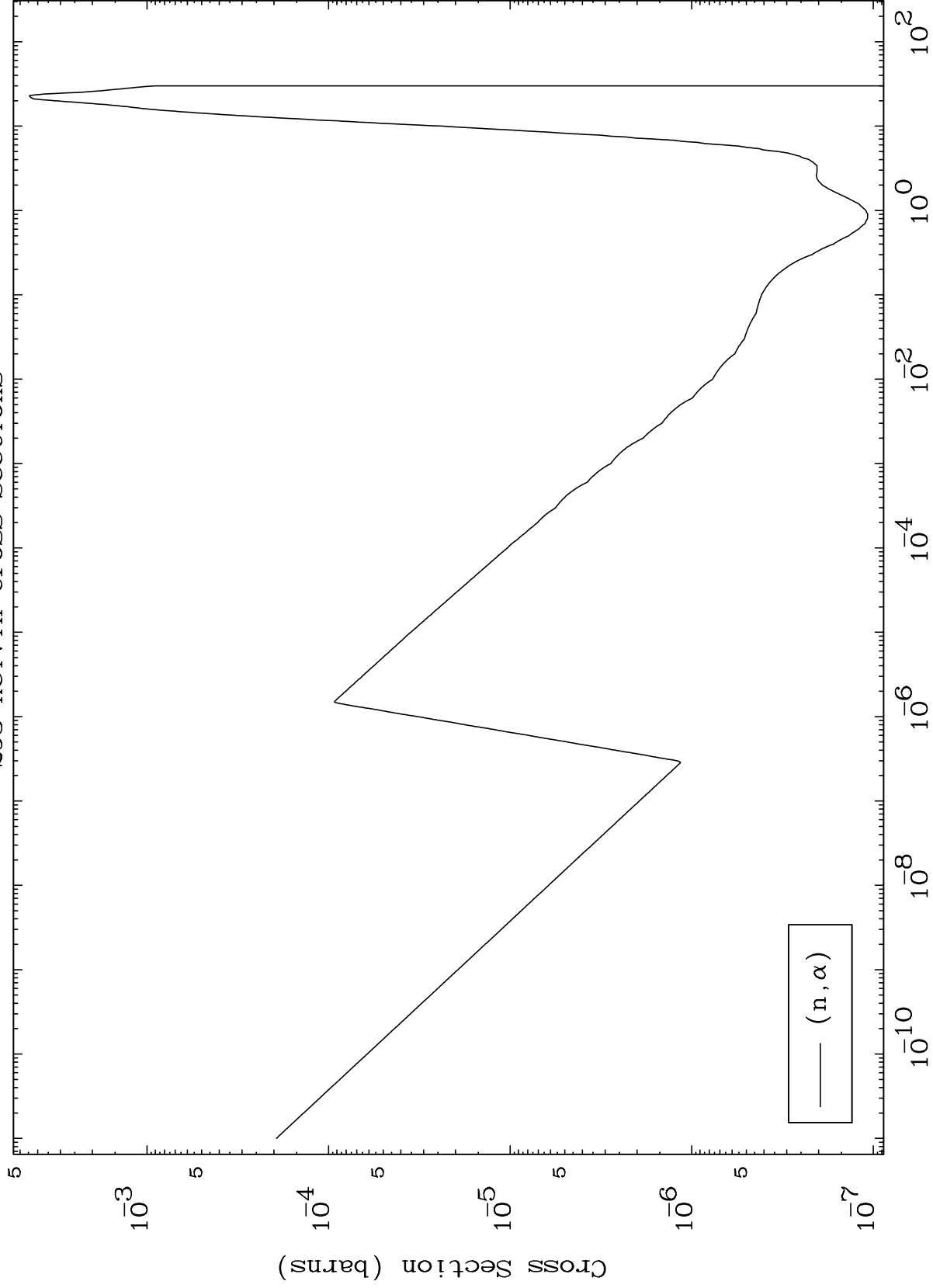




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(n,α) Levels
293 Kelvin Cross Sections

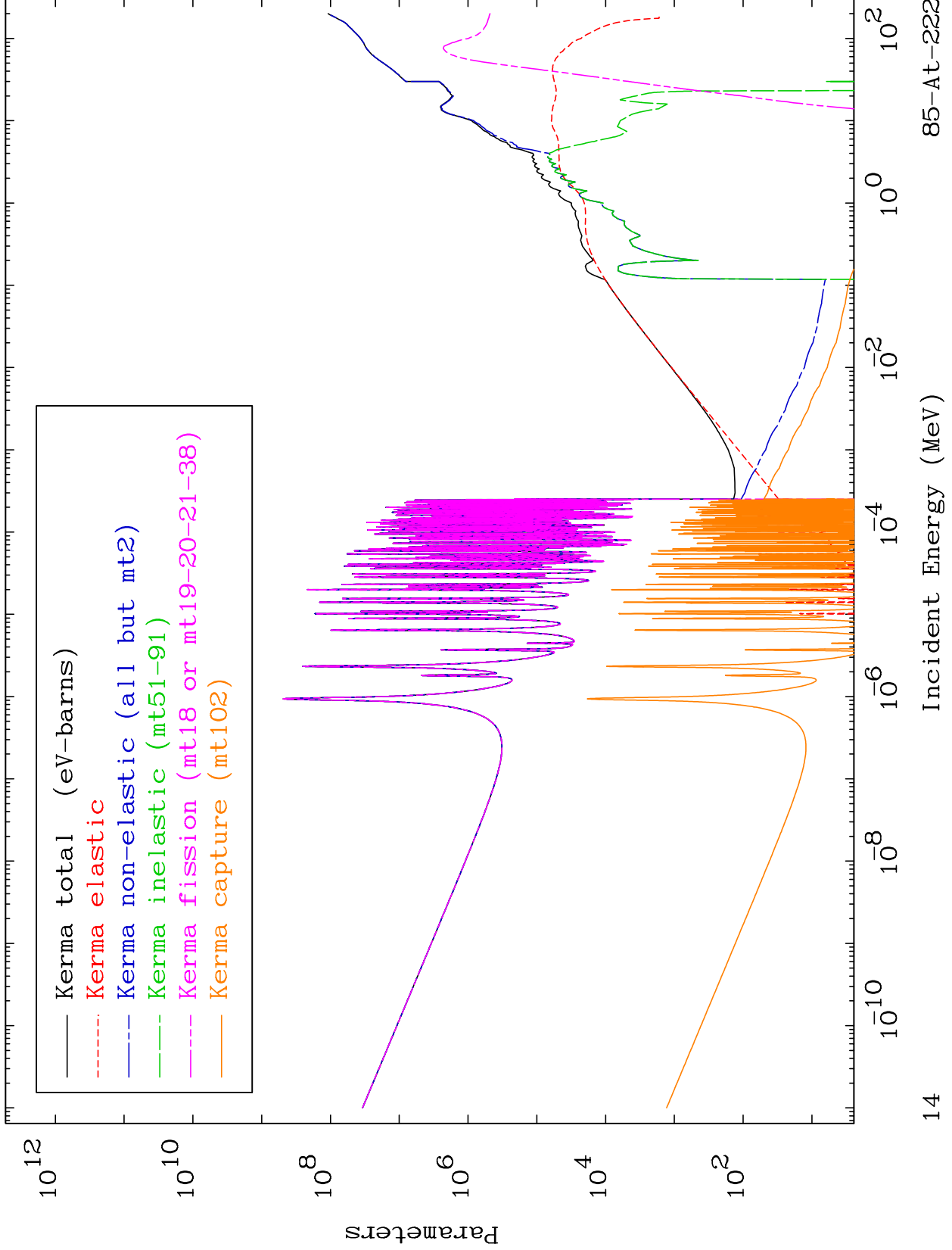
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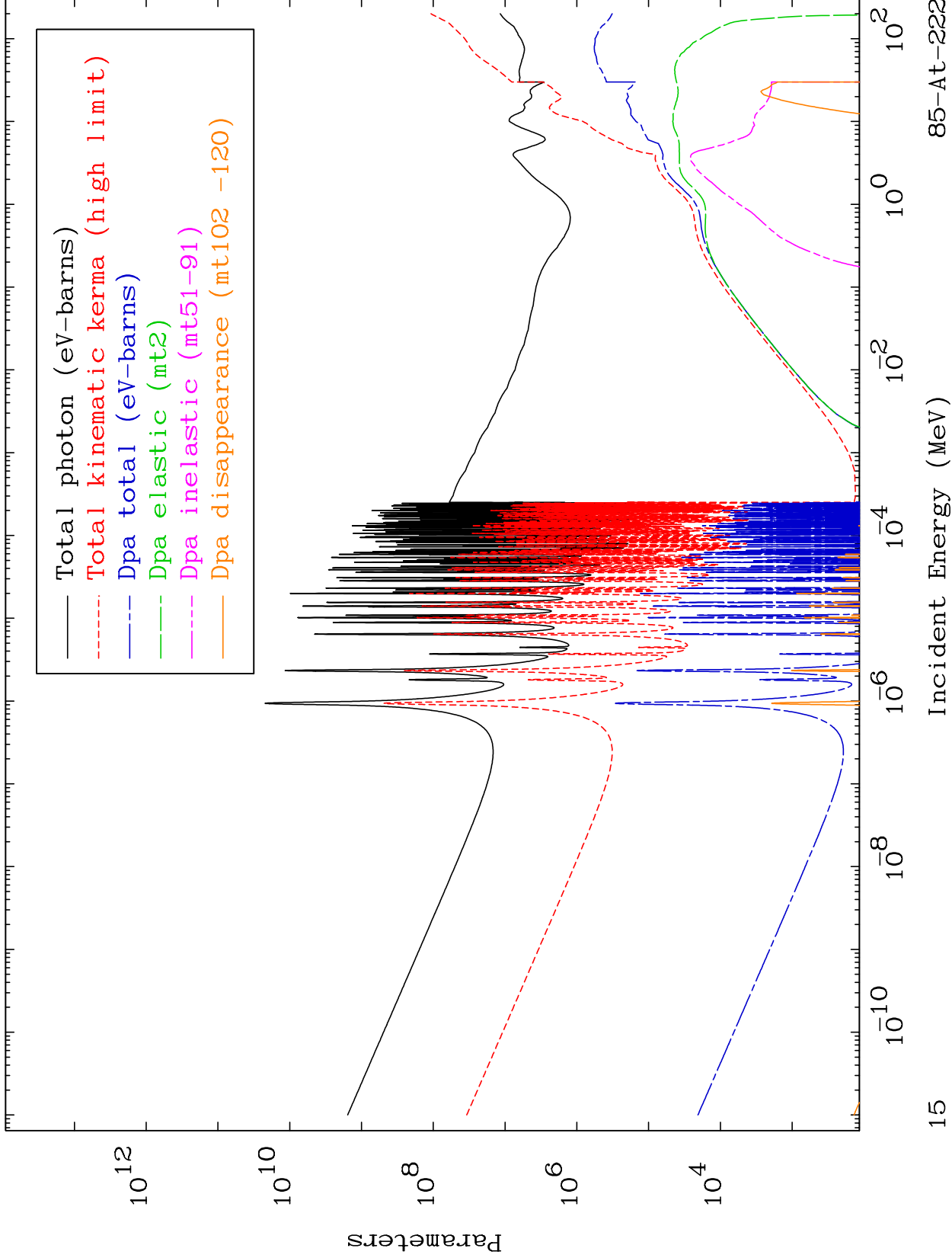


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Incident Energy (MeV)

85-At-222

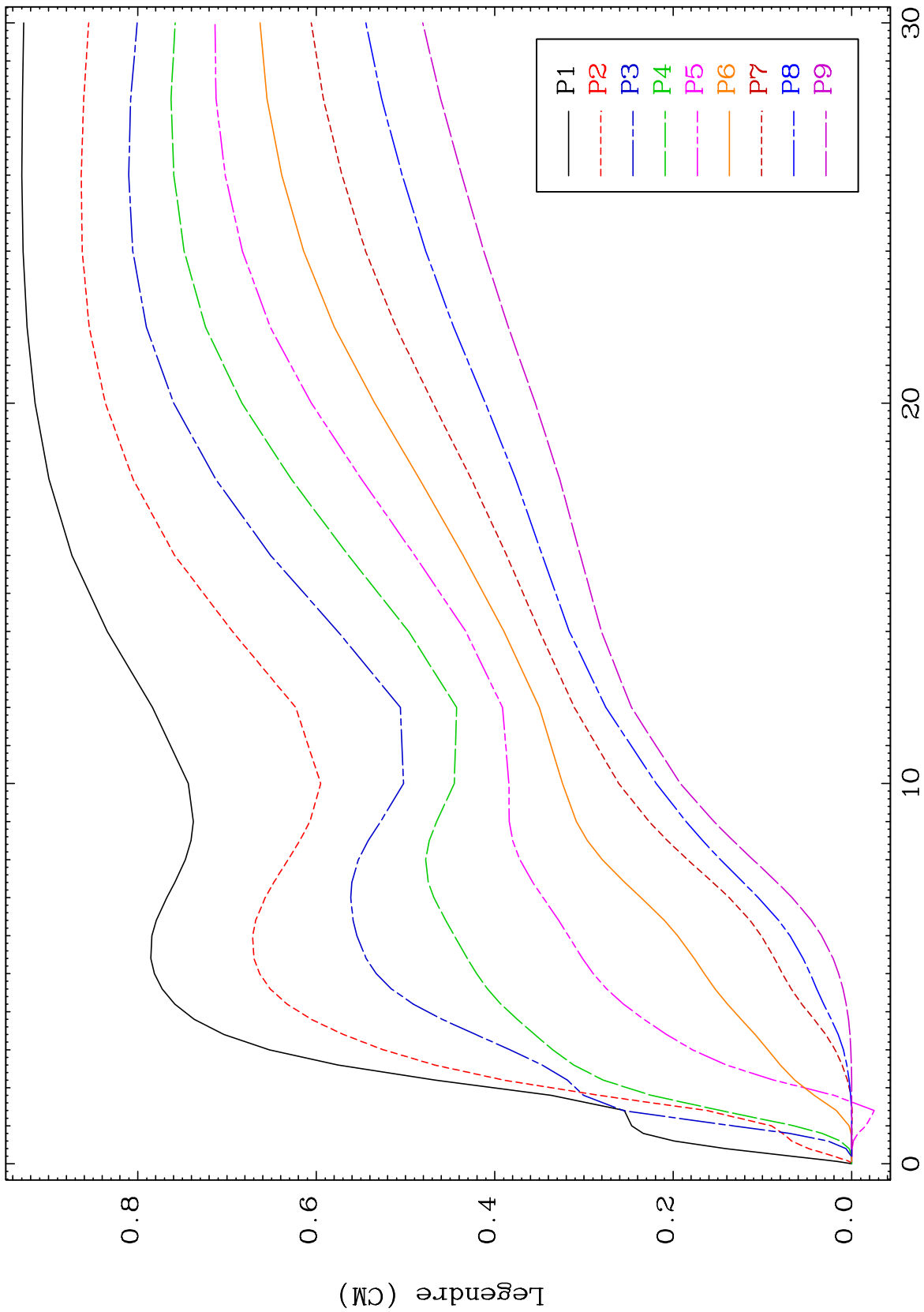




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Elastic Legendre Coefficients



85-At-222

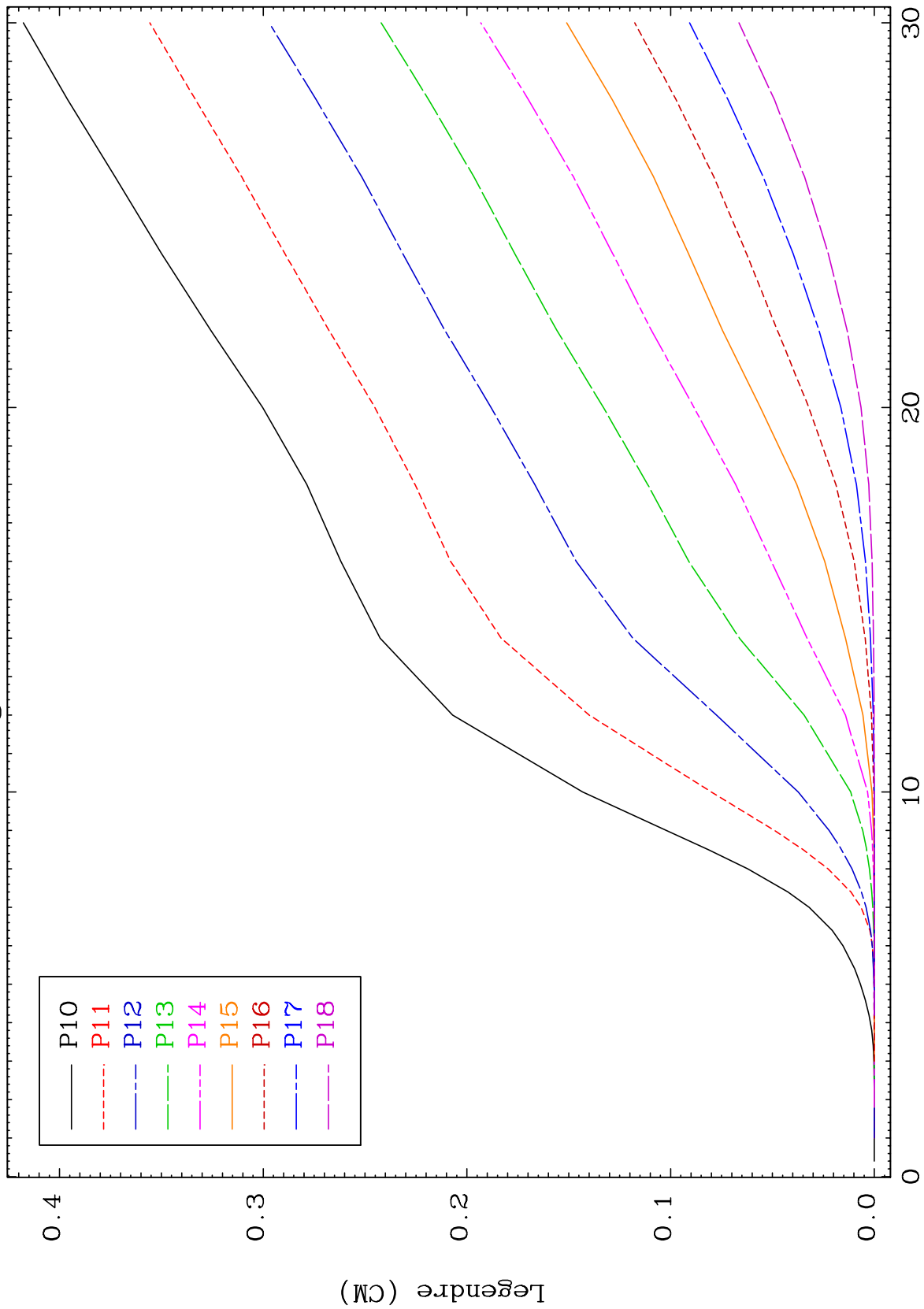
Incident Energy (MeV)

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Elastic Legendre Coefficients

85-At-222



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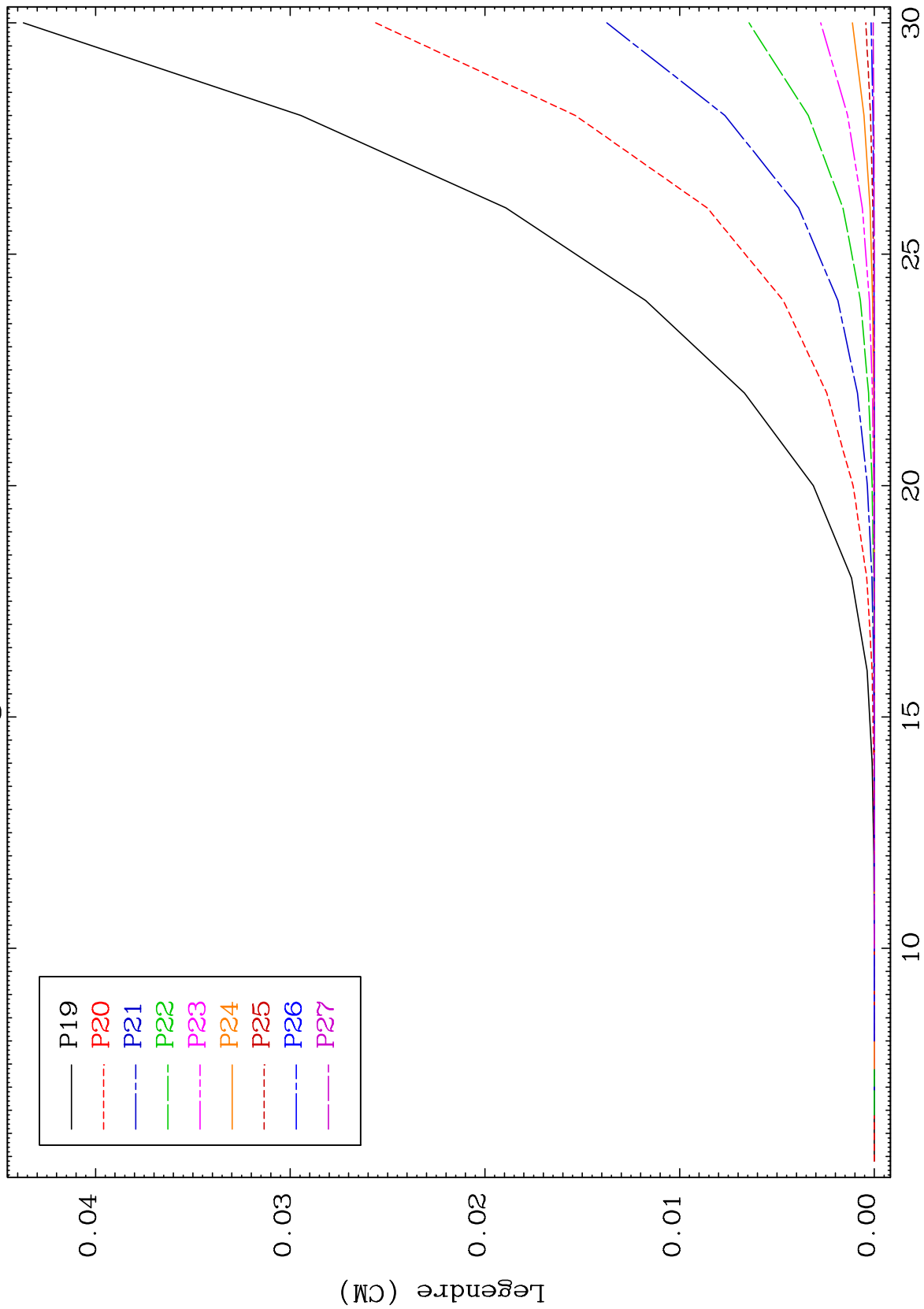
Incident Energy (MeV)

85-At-222

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Elastic Legendre Coefficients

85-At-222



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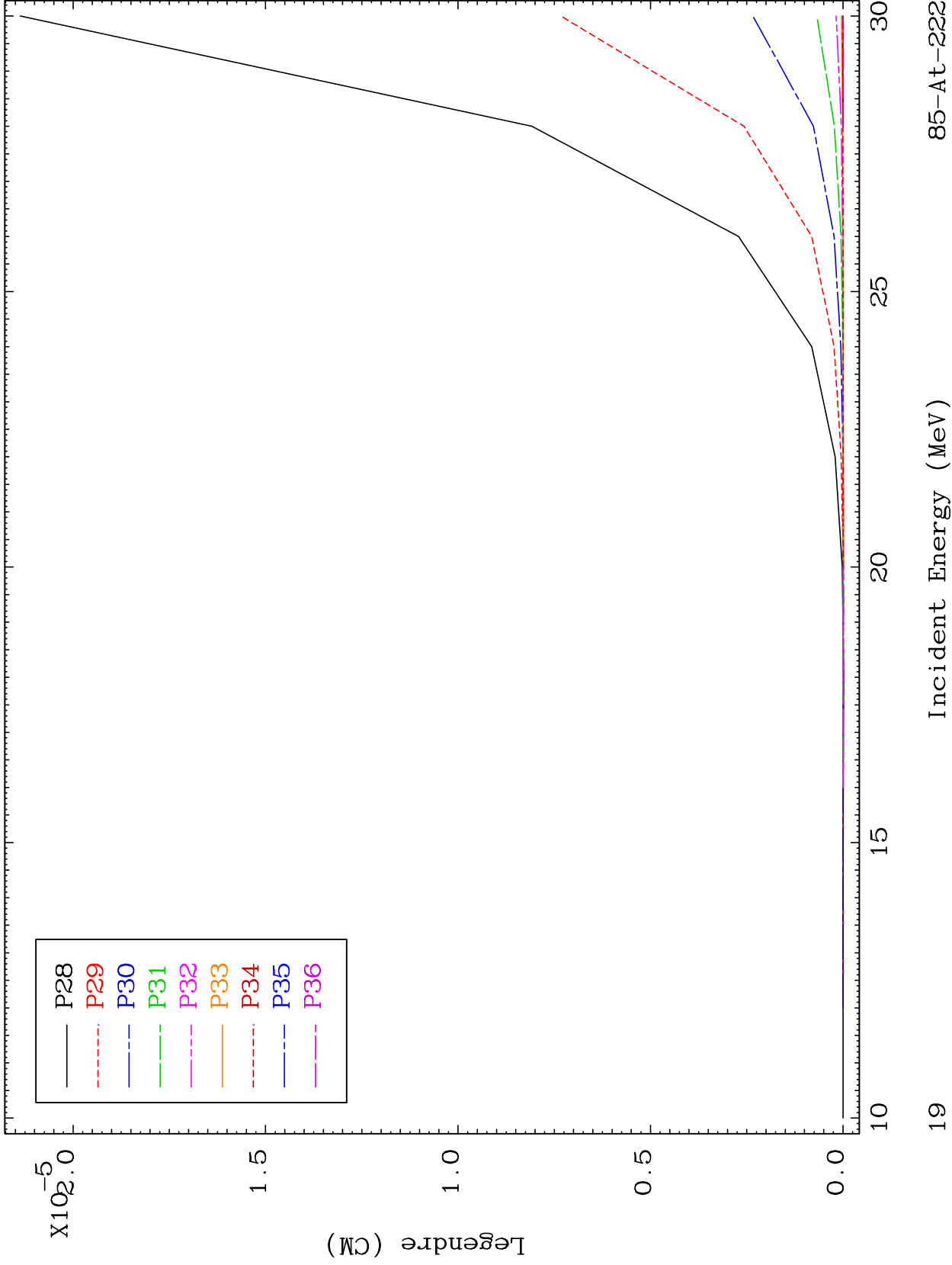
Incident Energy (MeV)

85-At-222

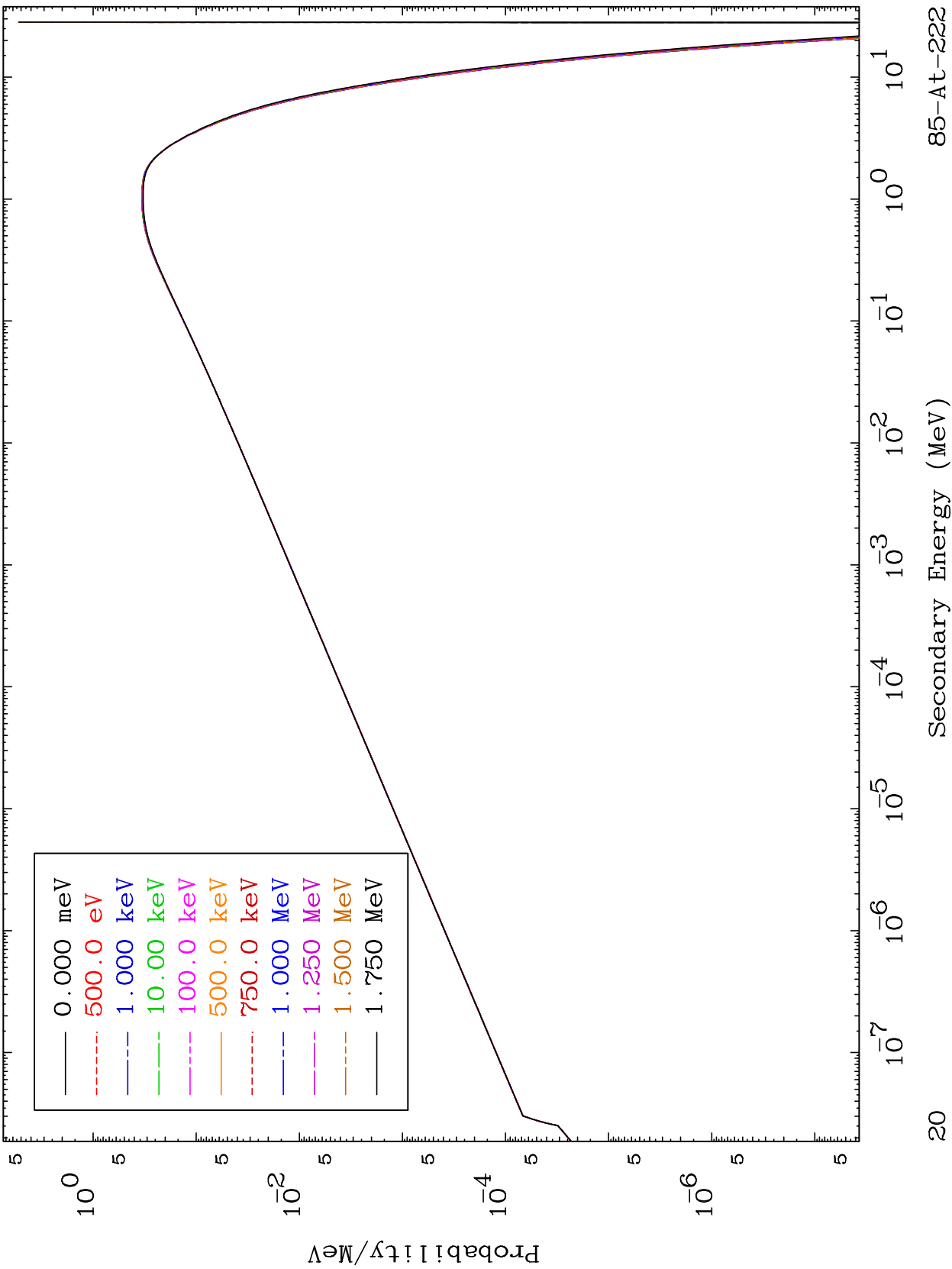
MAT 8582

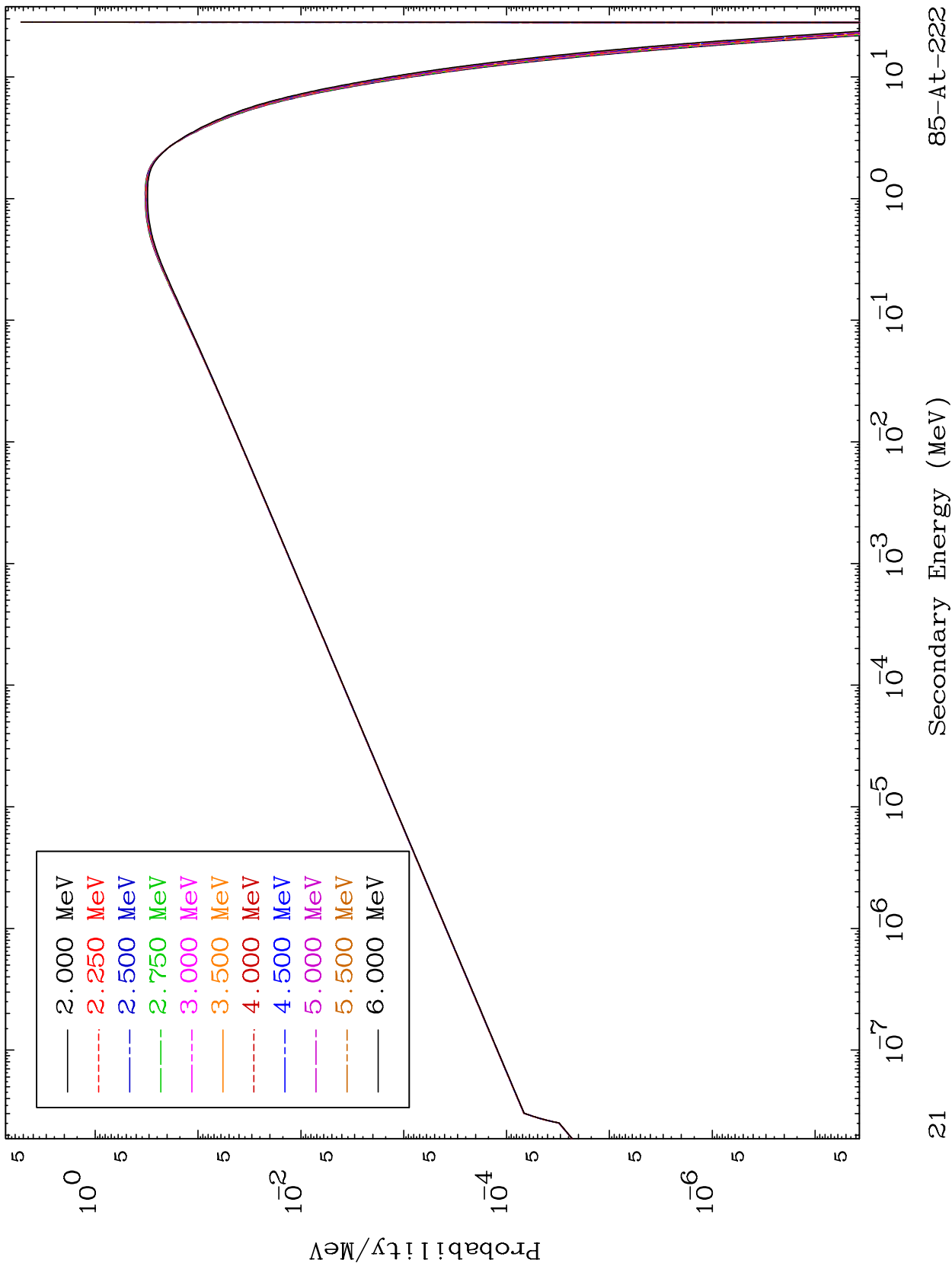
Elastic Legendre Coefficients

85-At-222

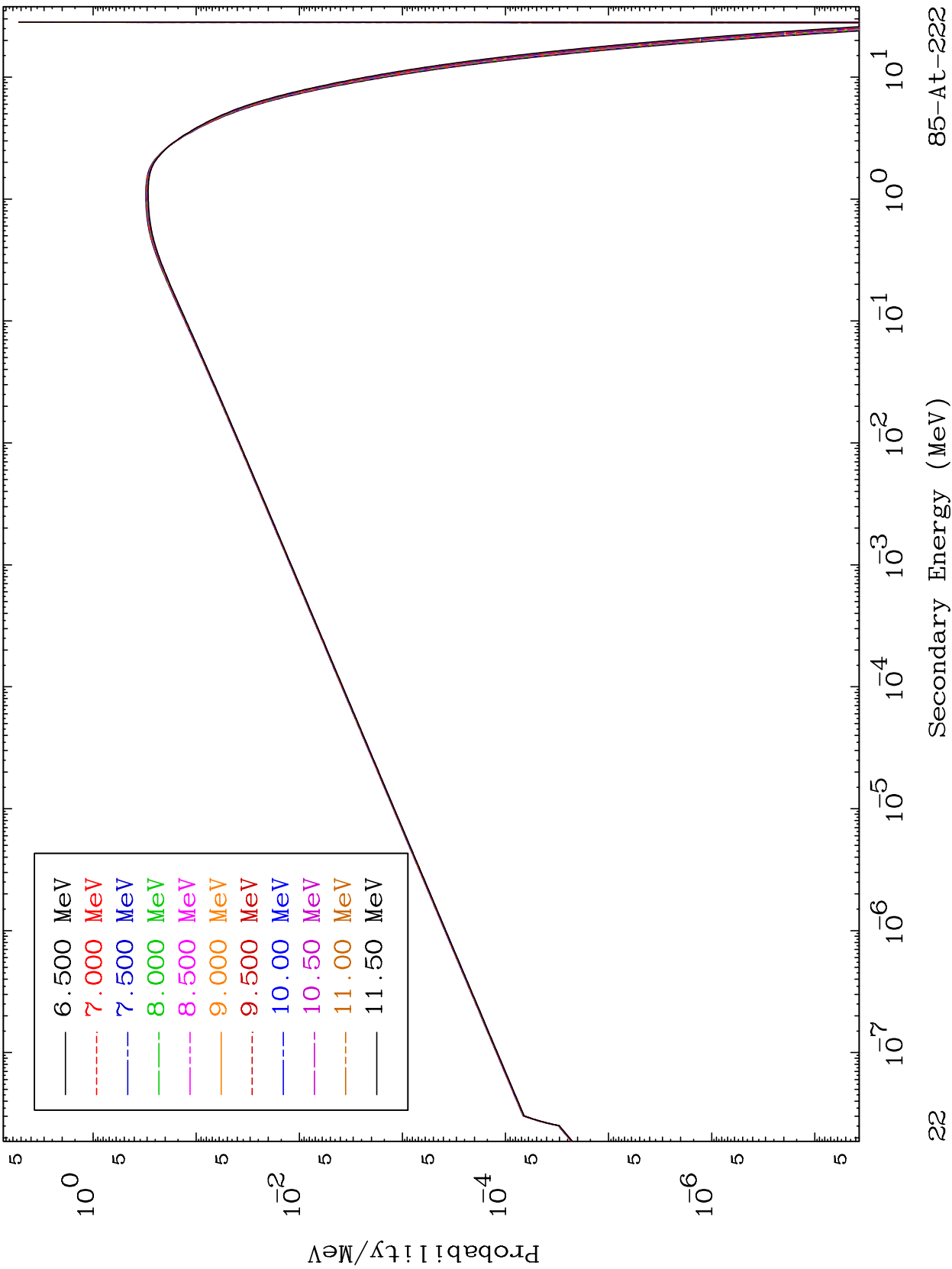


Fission Energy Distribution

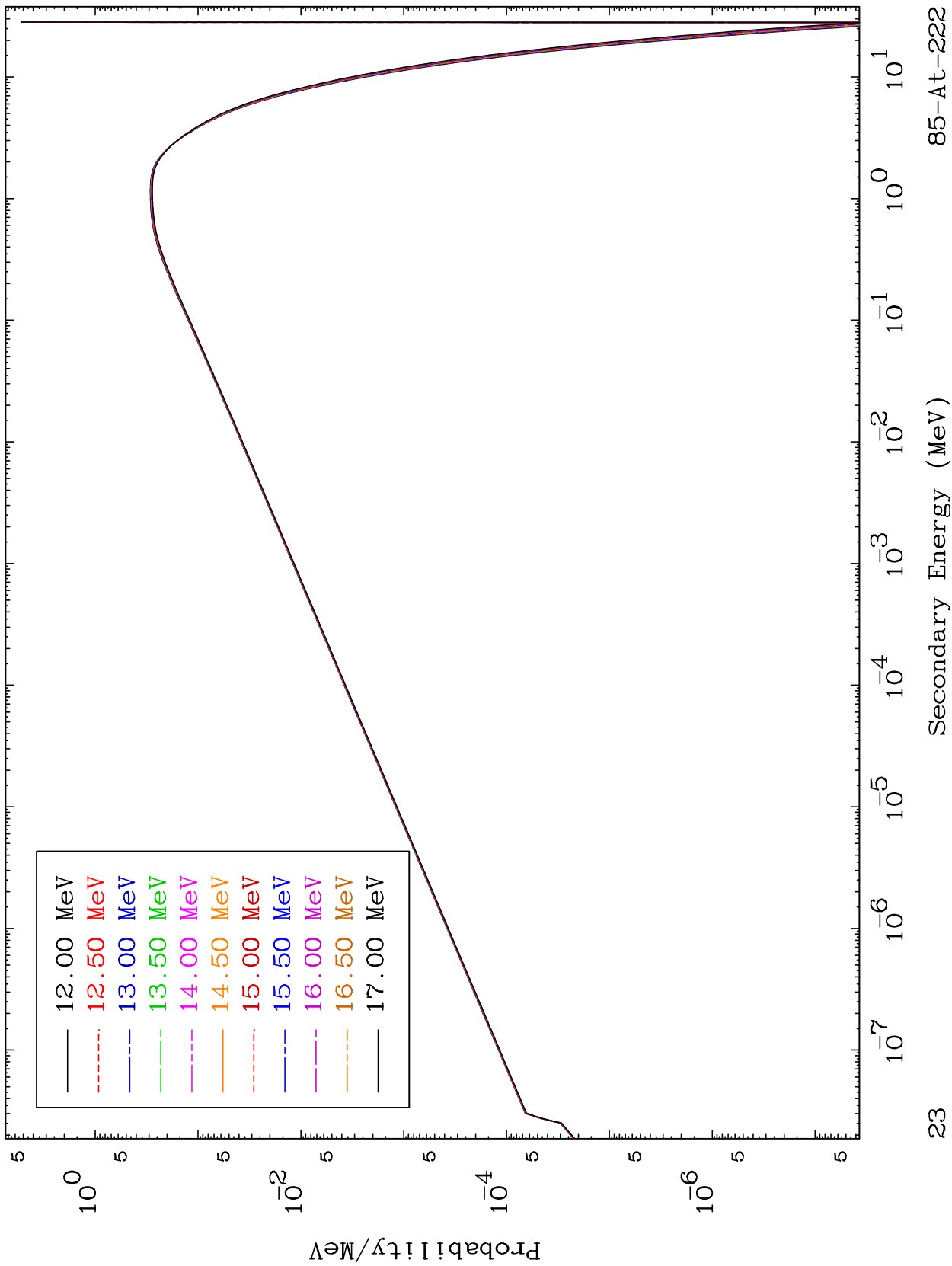




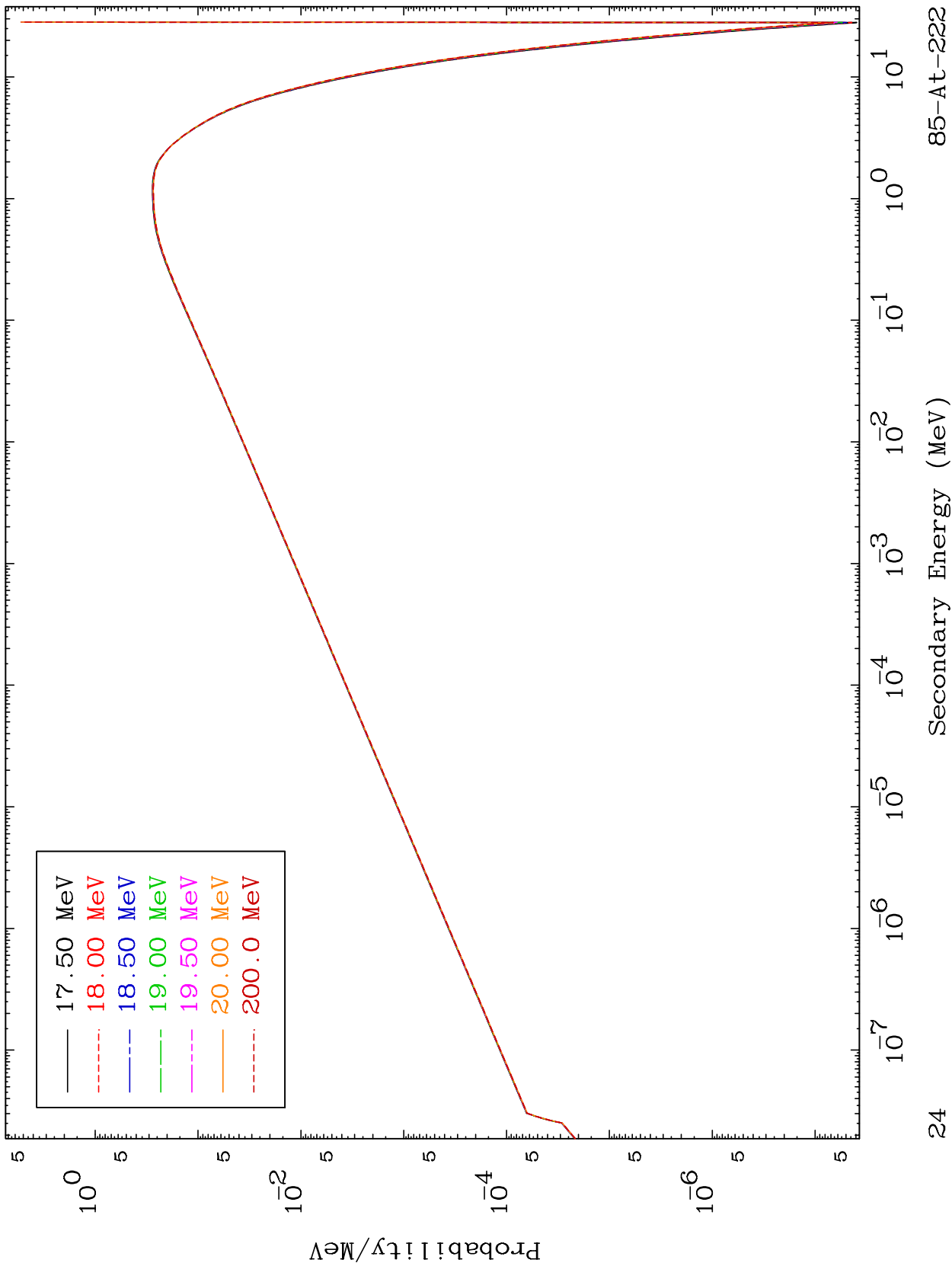
Fission Energy Distribution



Fission Energy Distribution



Fission
Energy Distribution



Radionuclide Production Cross Section

