

Program EVALPLOT
(Version 2015-2)

by

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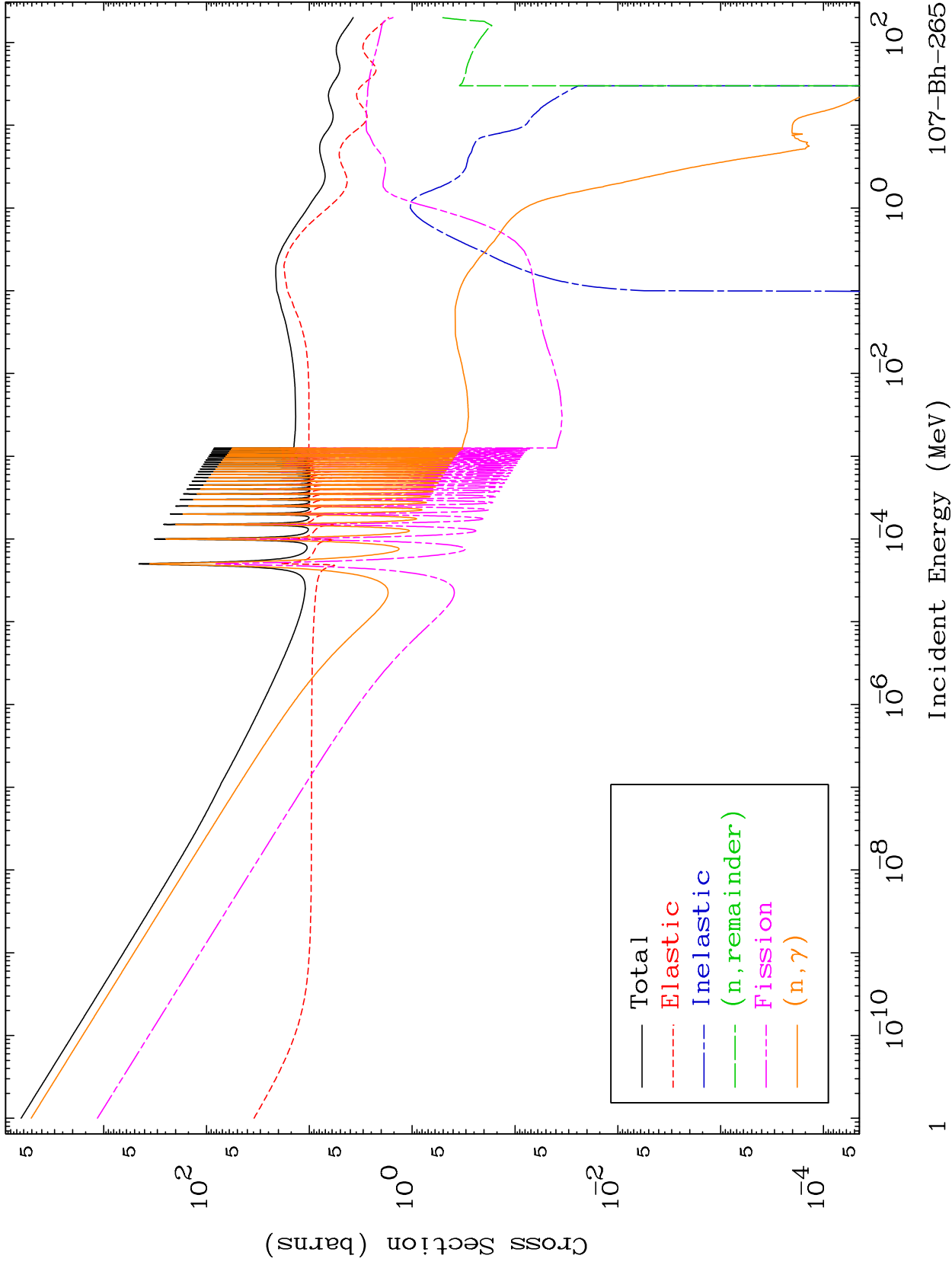
Press Mouse Button to Start

MAT 765

Major

293 Kelvin Cross Sections

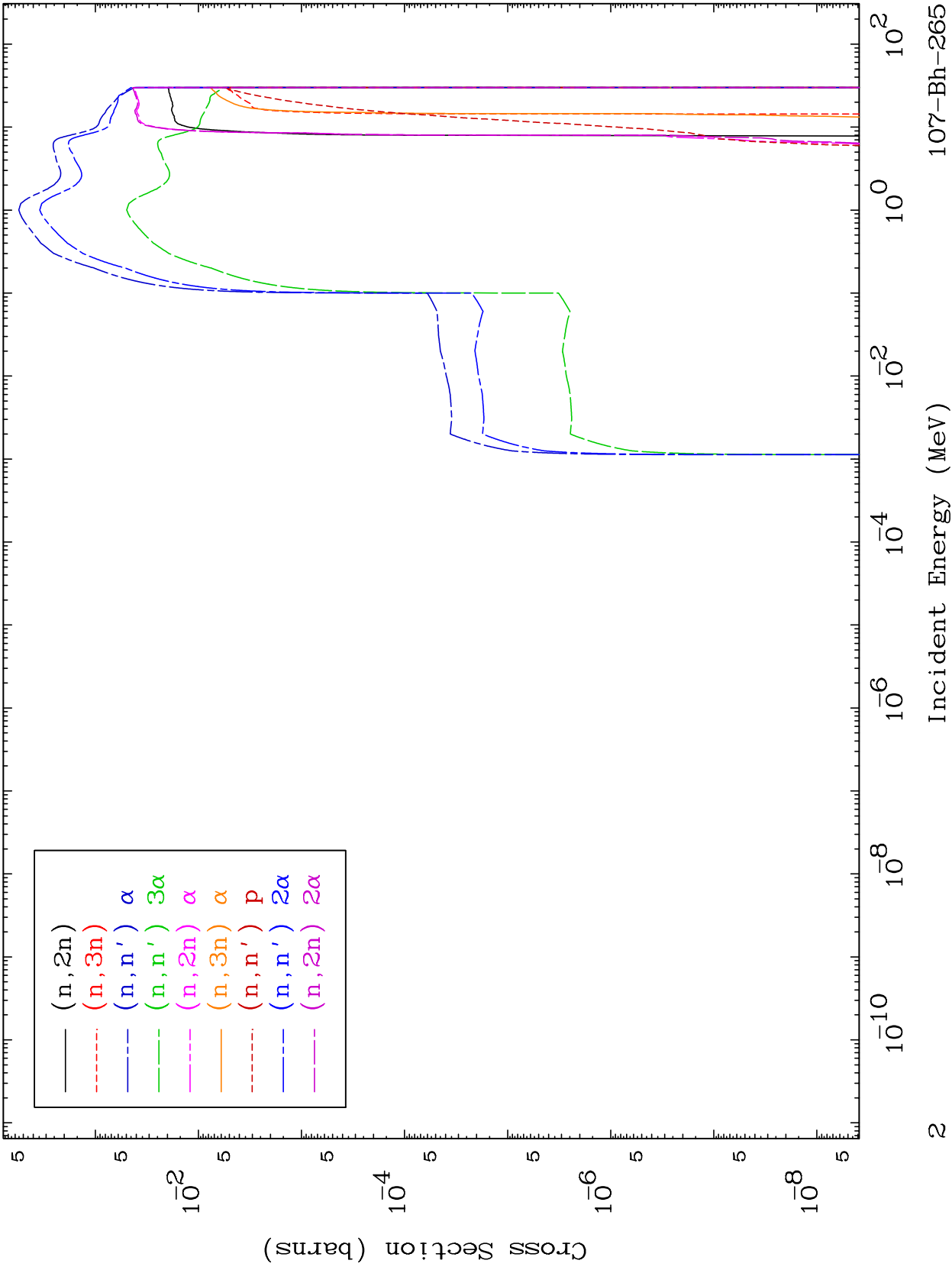
107-Bh-265



MAT 765

Neutron Production
293 Kelvin Cross Sections

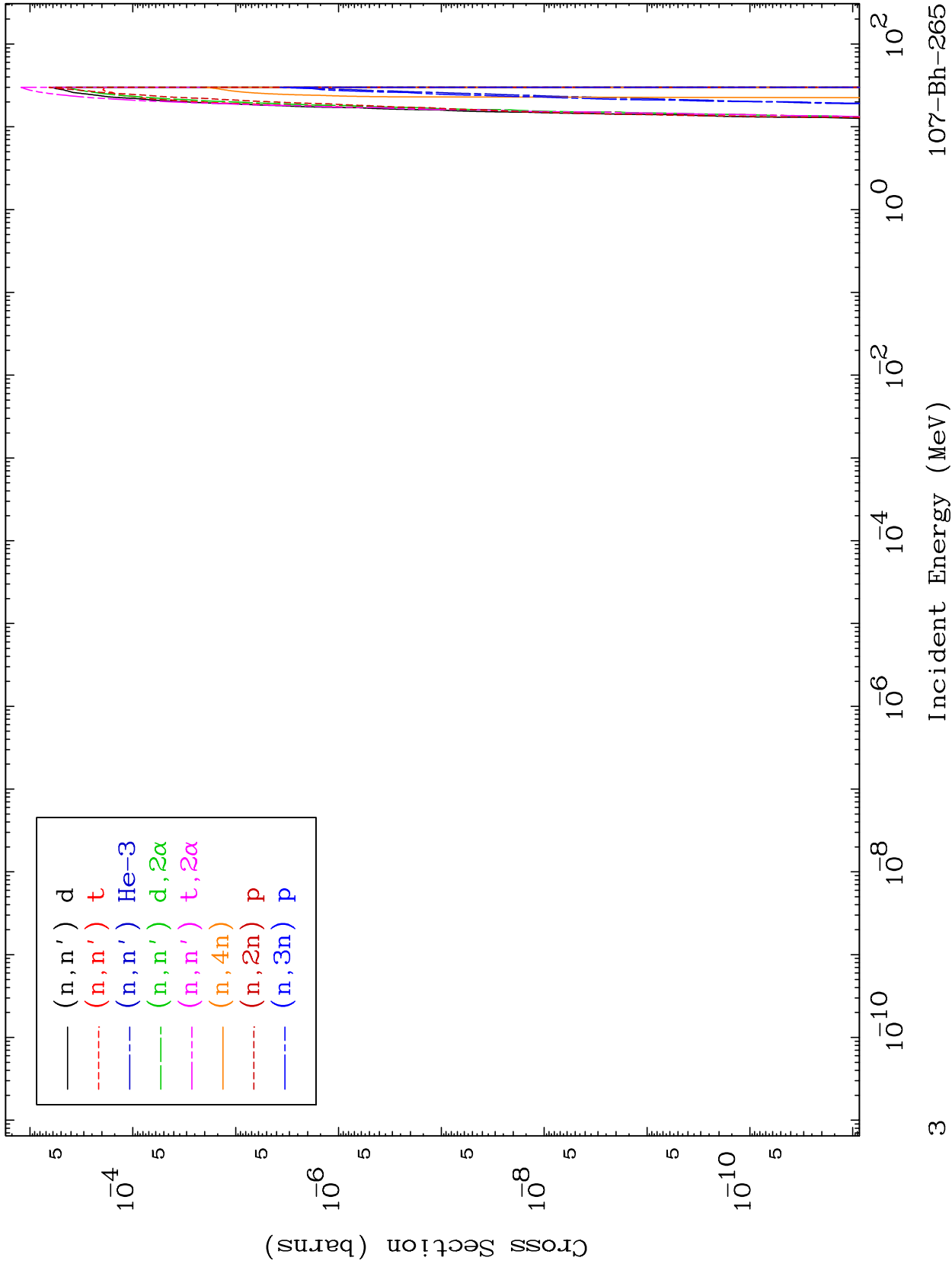
107-Bh-265



MAT 765

Neutron Production
293 Kelvin Cross Sections

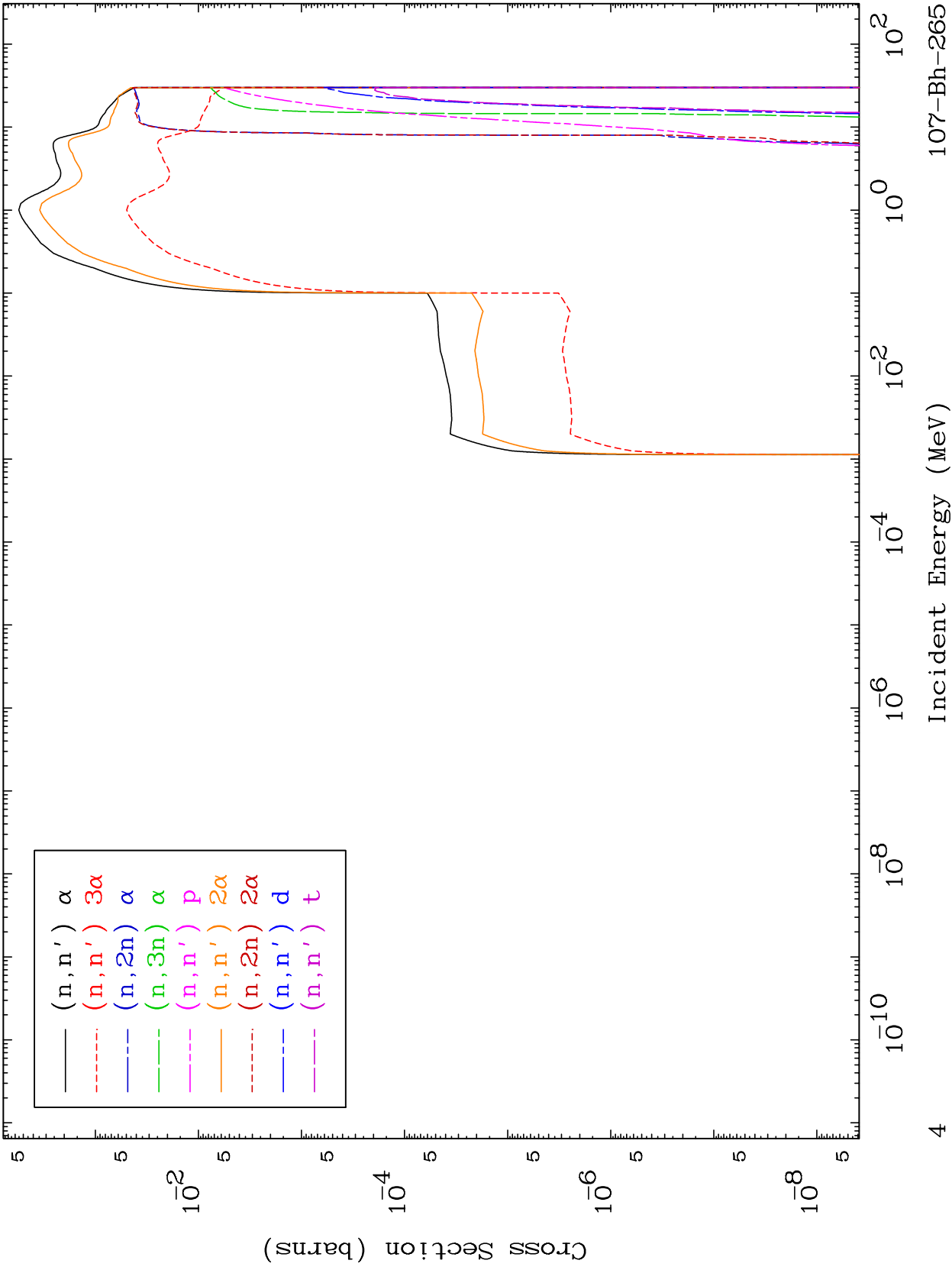
107-Bh-265



MAT 765

Charged Particle
293 Kelvin Cross Sections

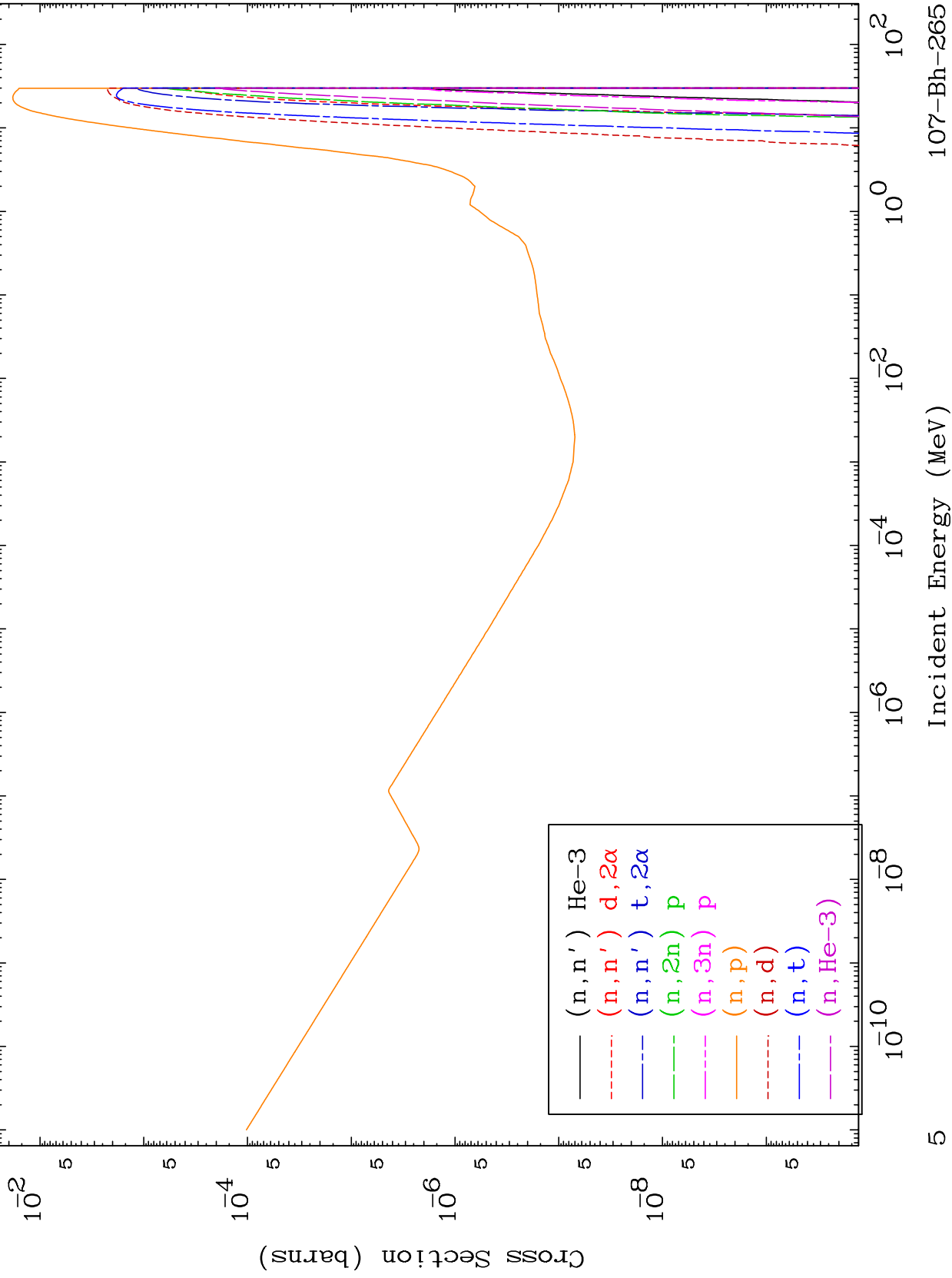
107-Bh-265

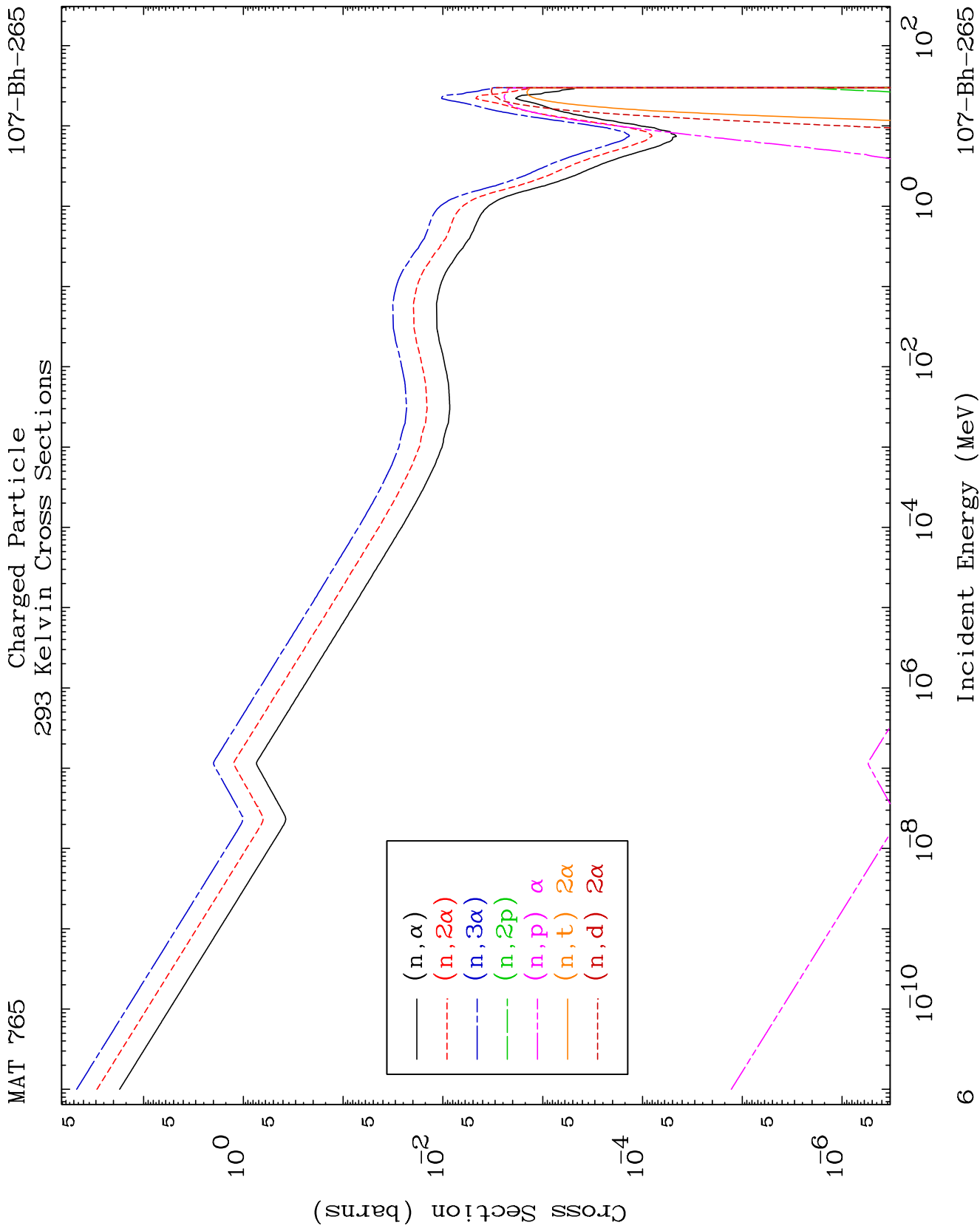


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Charged Particle
293 Kelvin Cross Sections

107-Bh-265

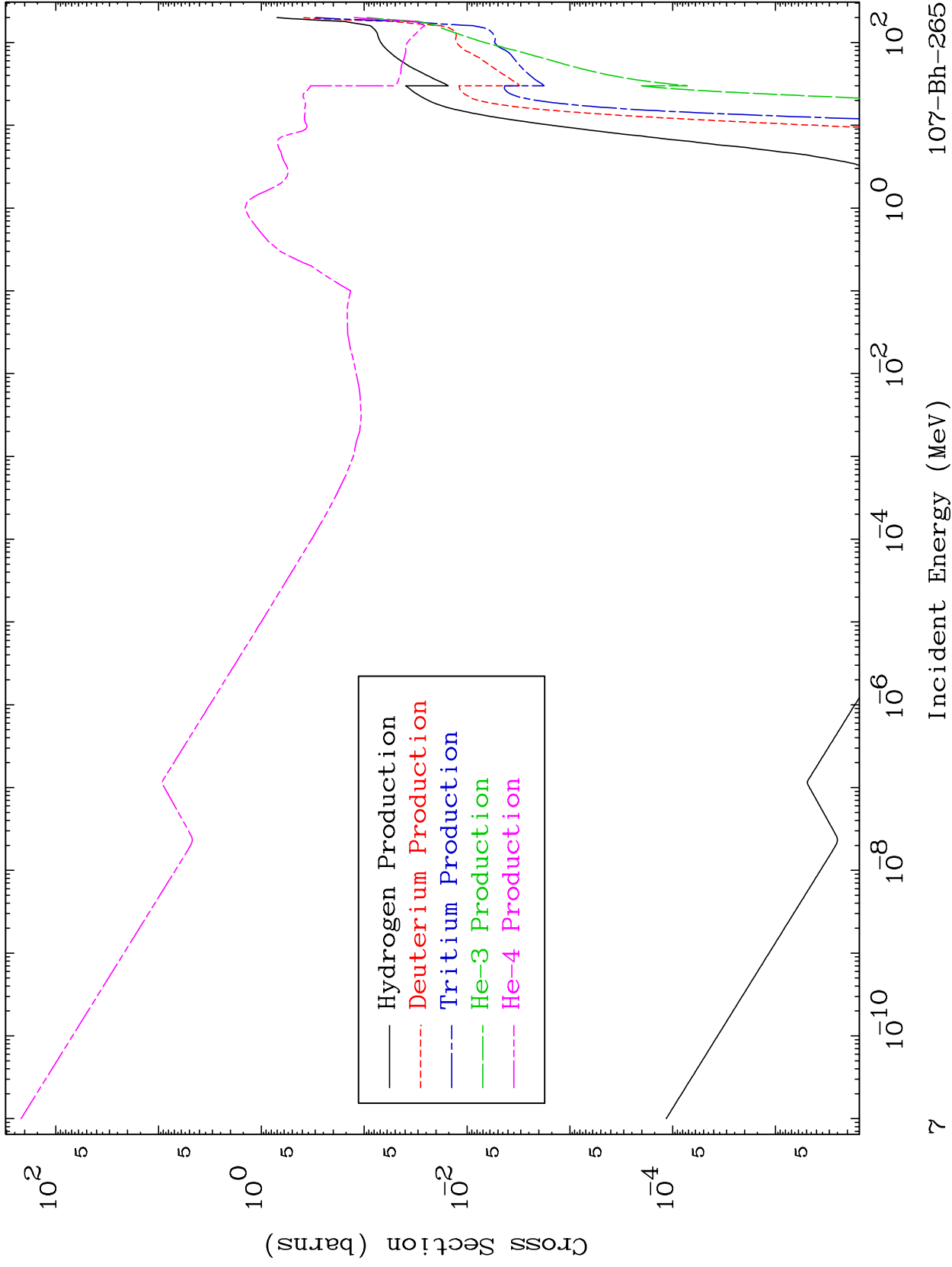




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Particle Production
293 Kelvin Cross Sections

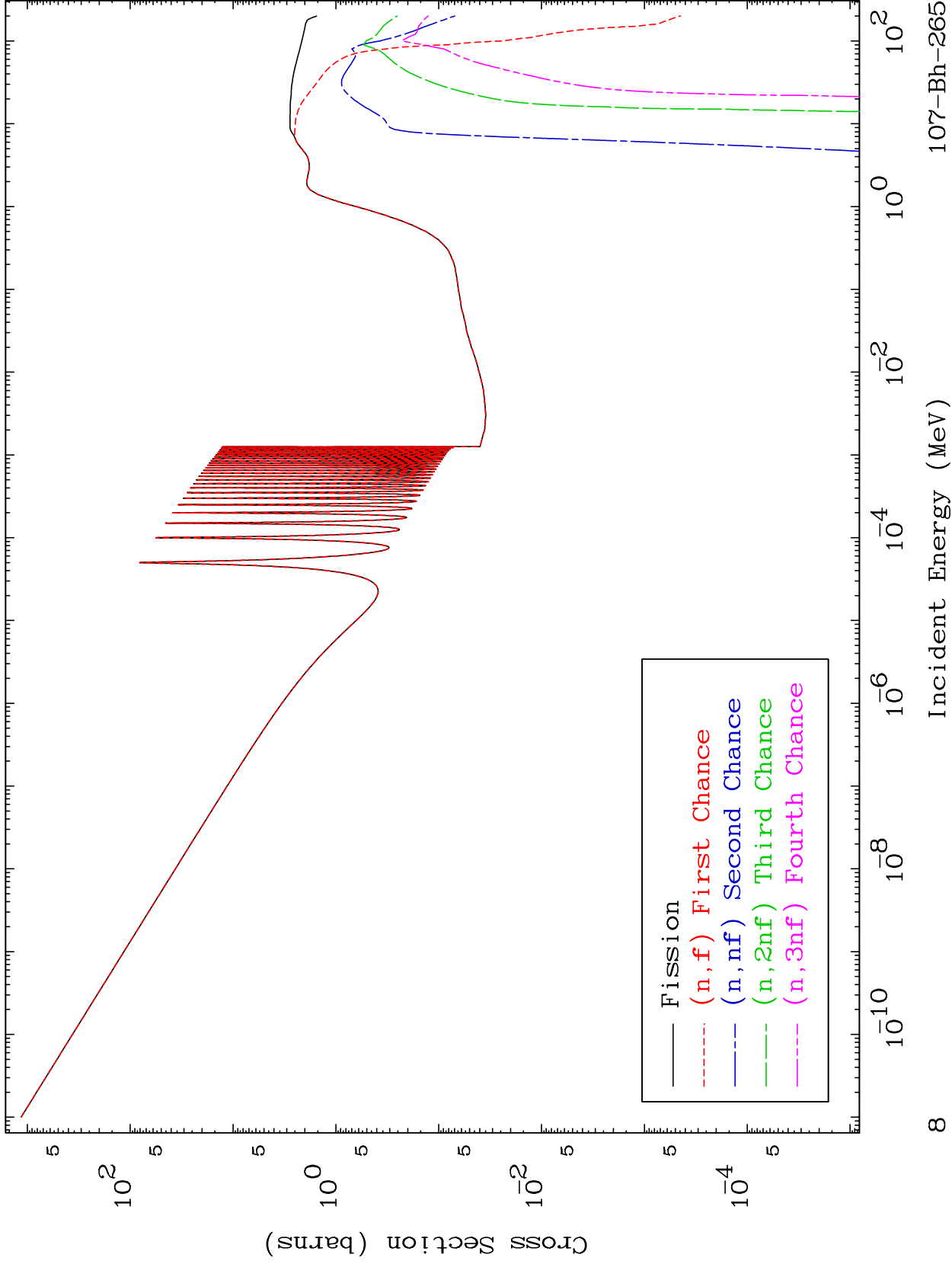
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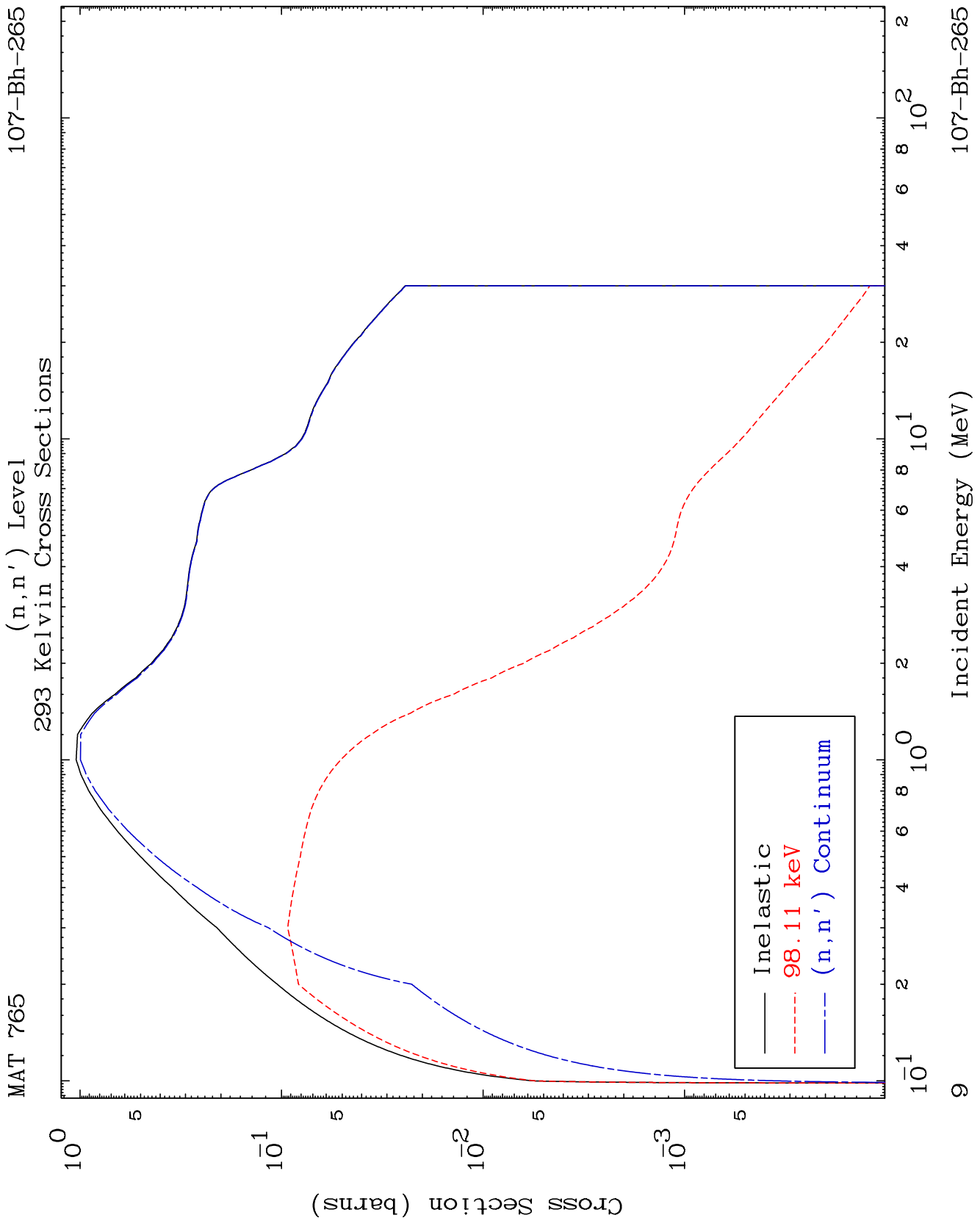


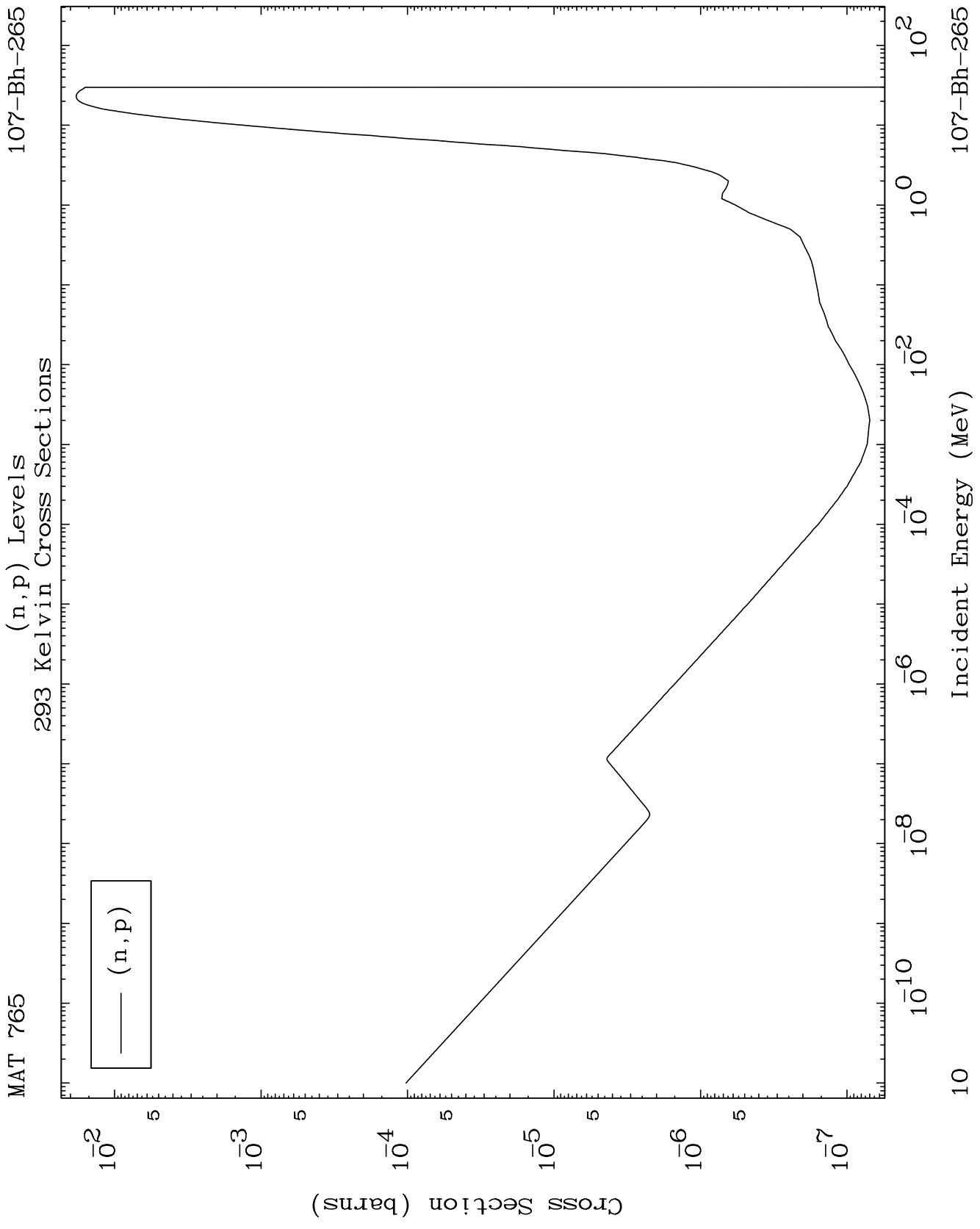
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Fission
293 Kelvin Cross Sections

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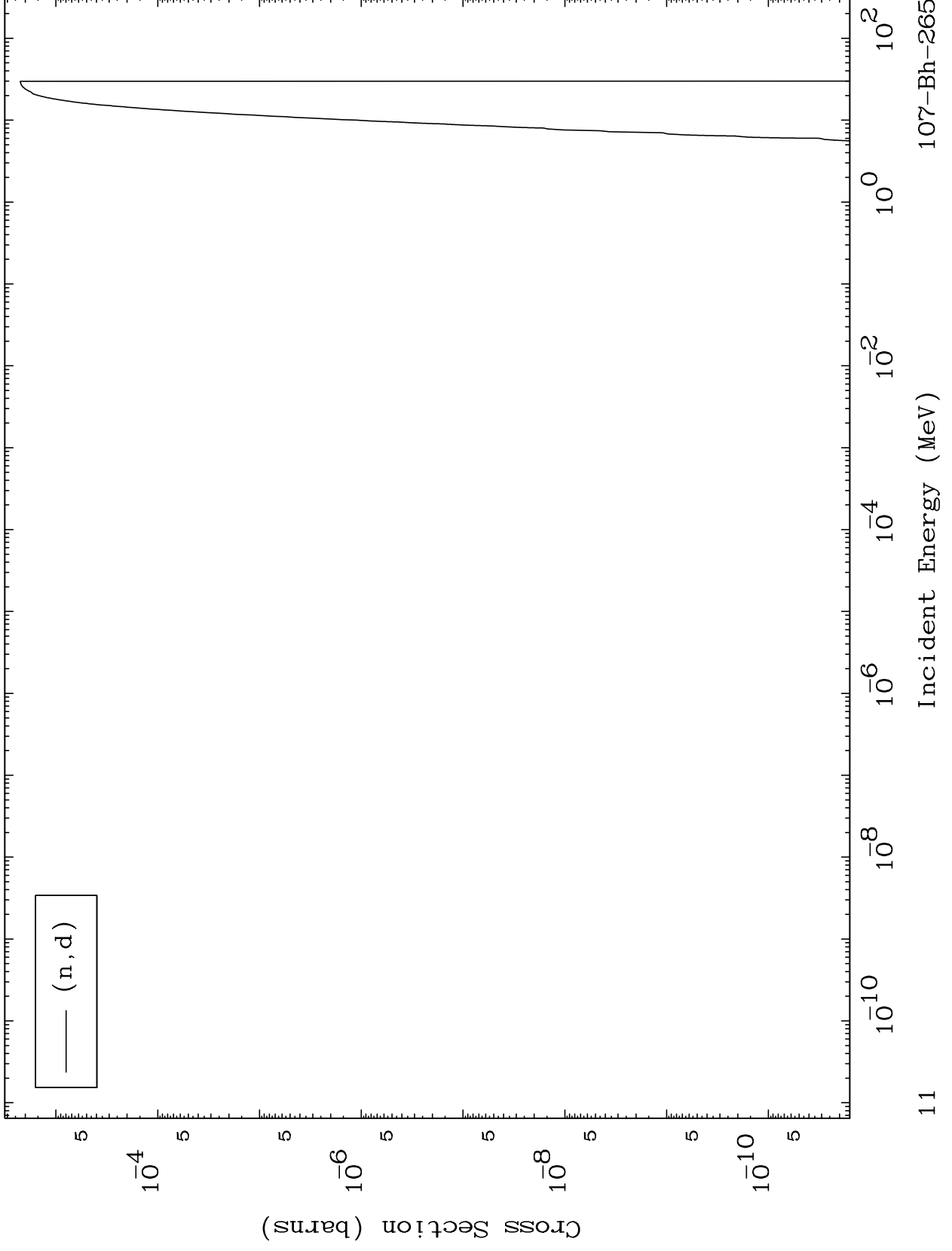




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(n,d) Levels
293 Kelvin Cross Sections

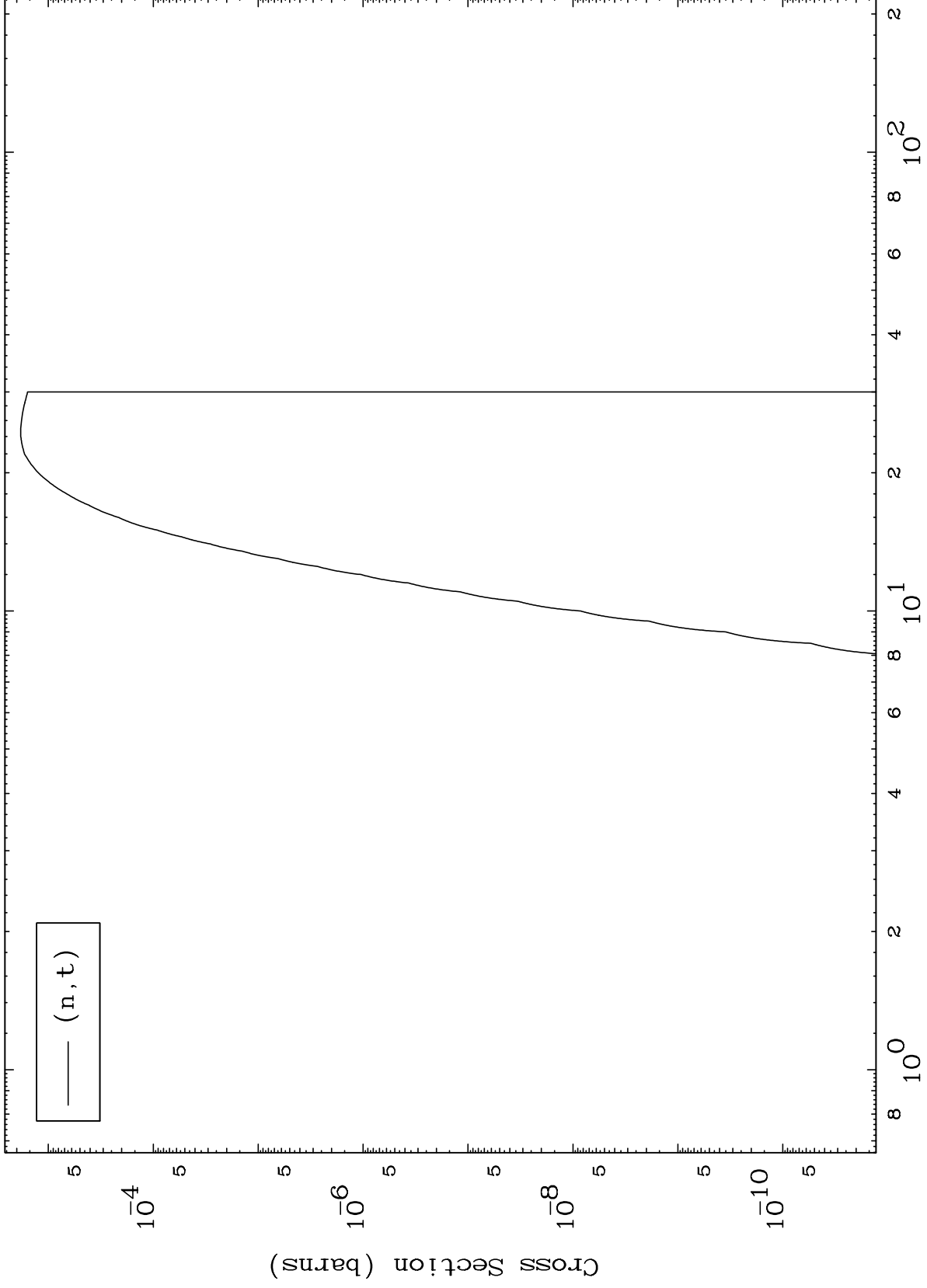
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(n,t) Levels
293 Kelvin Cross Sections

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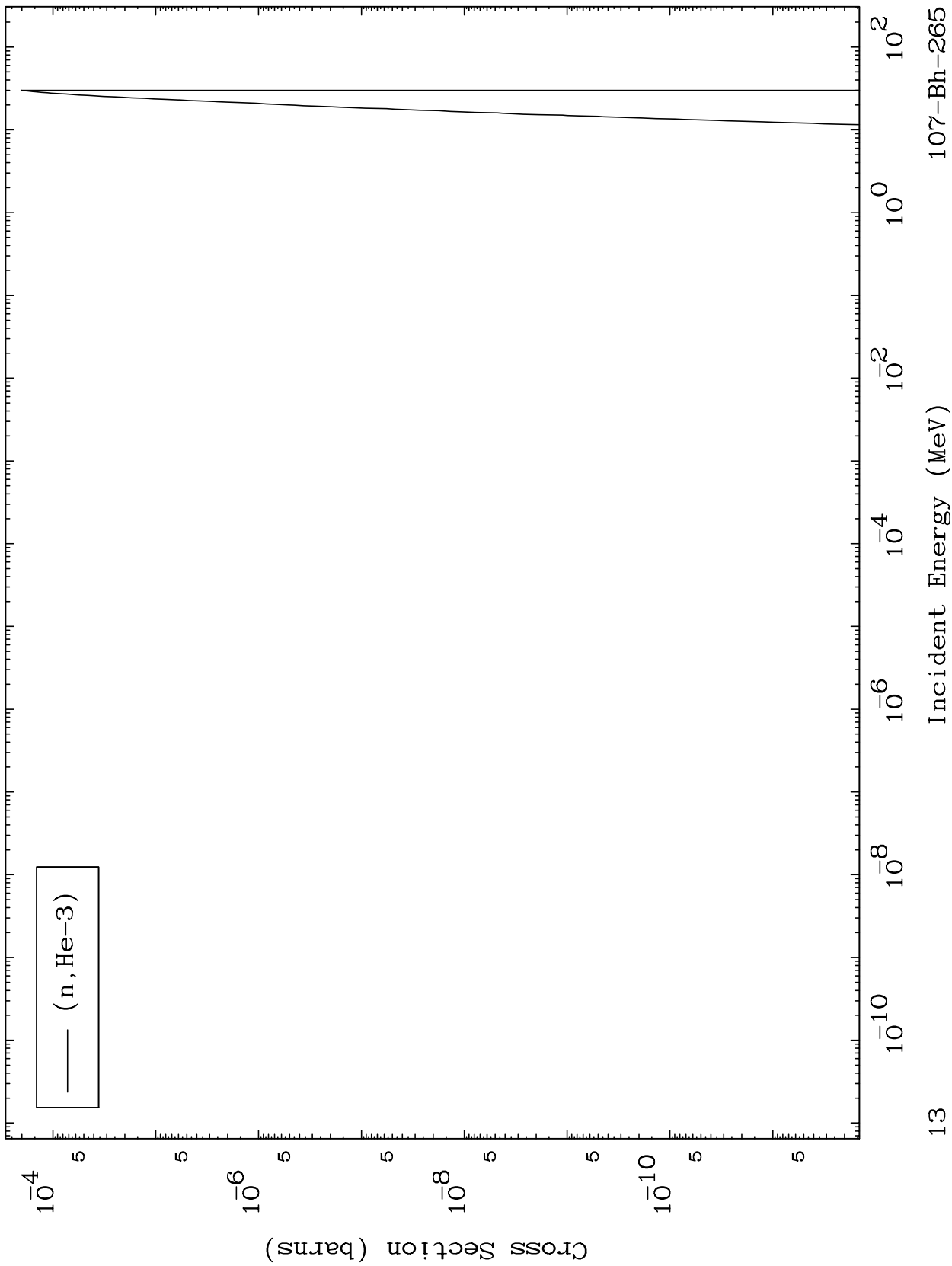
Incident Energy (MeV)

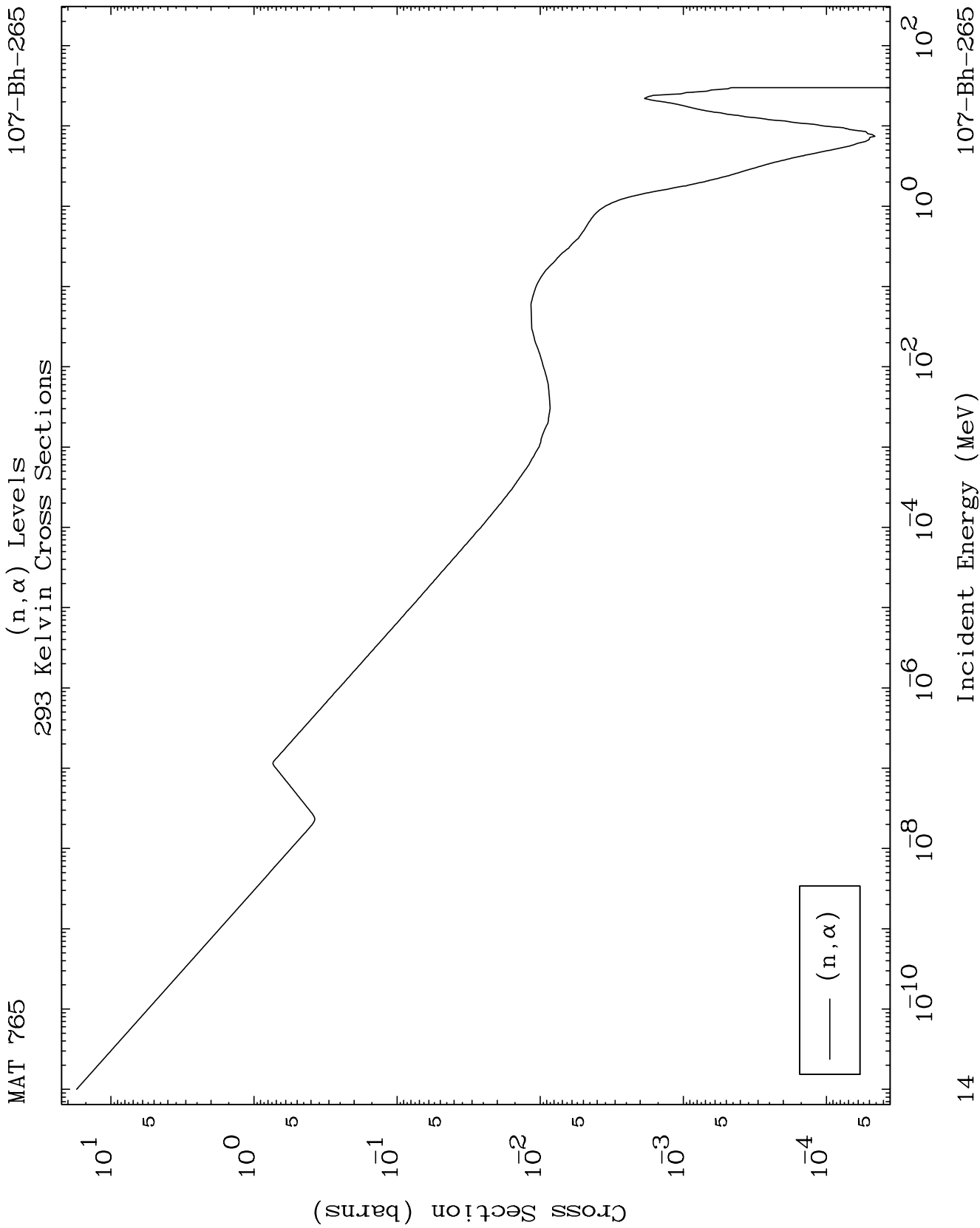
107-Bh-265

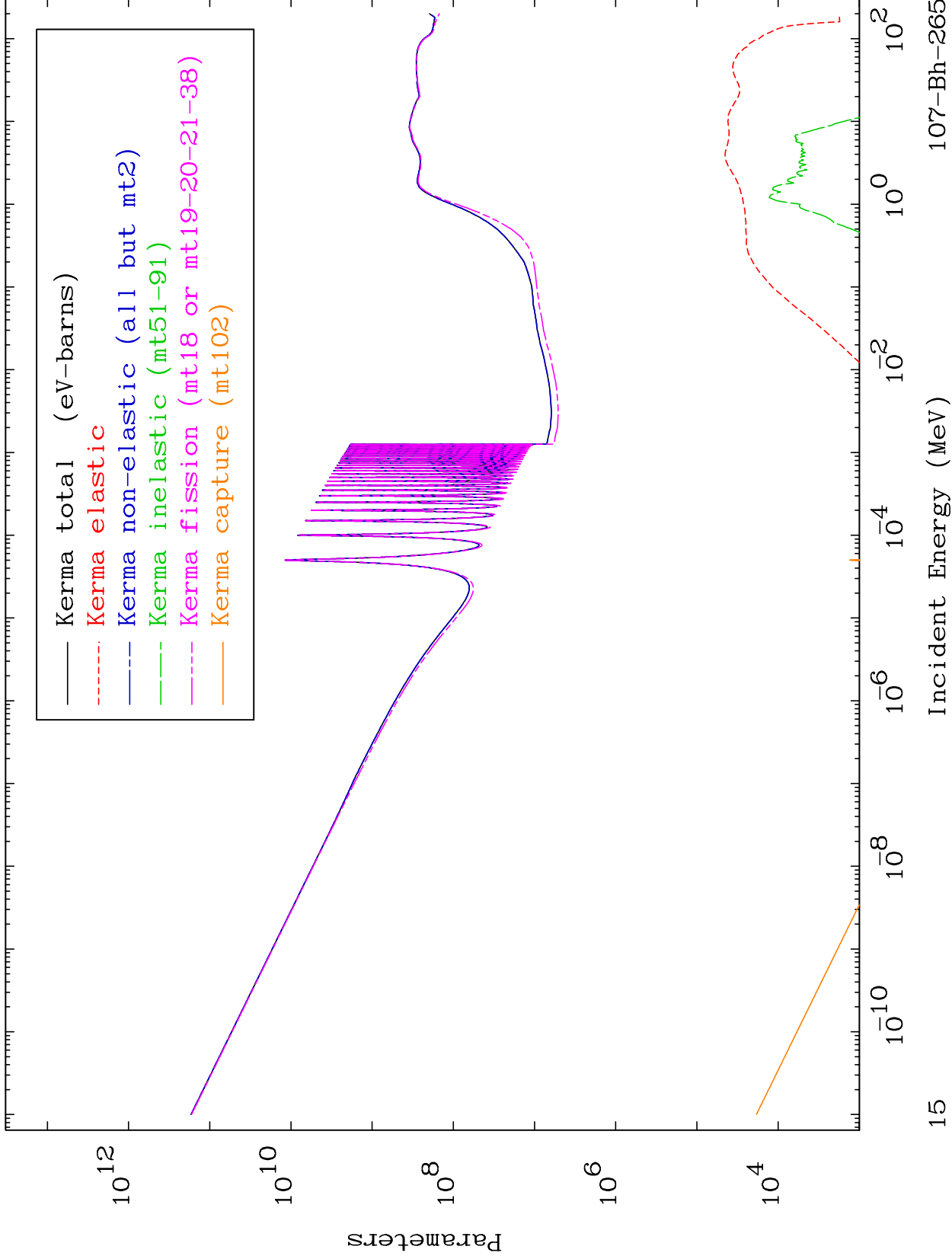
MAT 765

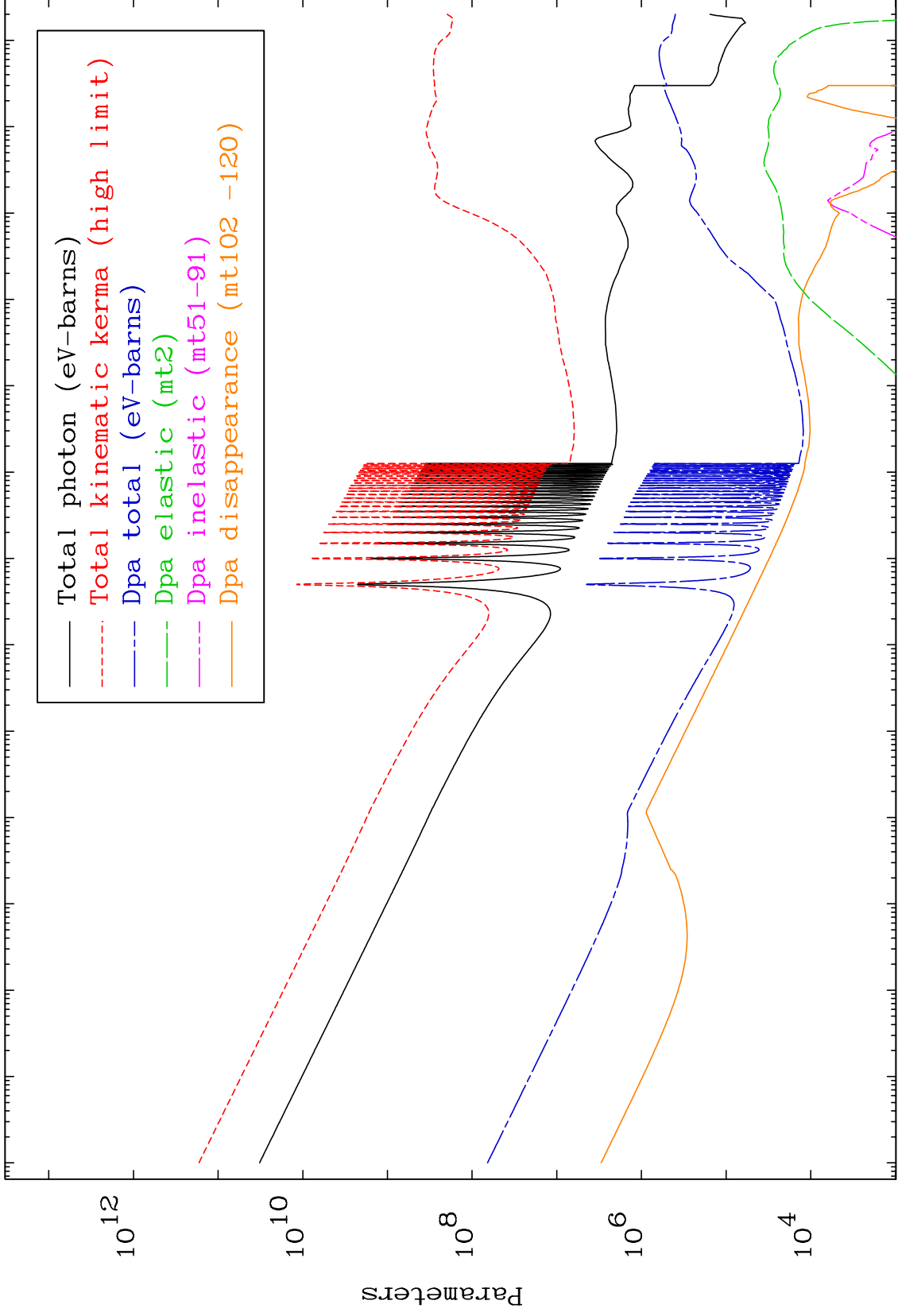
(n,He3) Levels
293 Kelvin Cross Sections

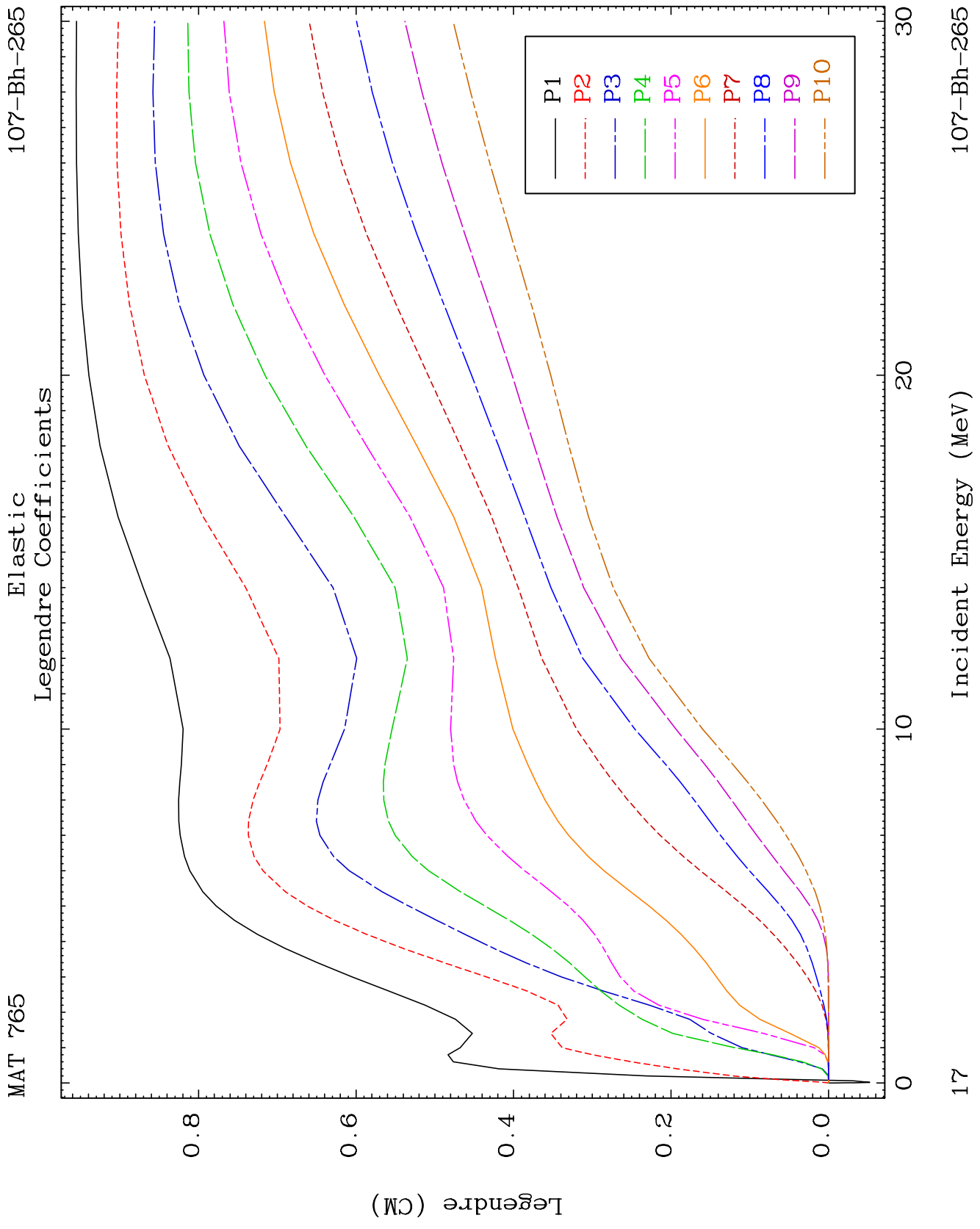
107-Bh-265

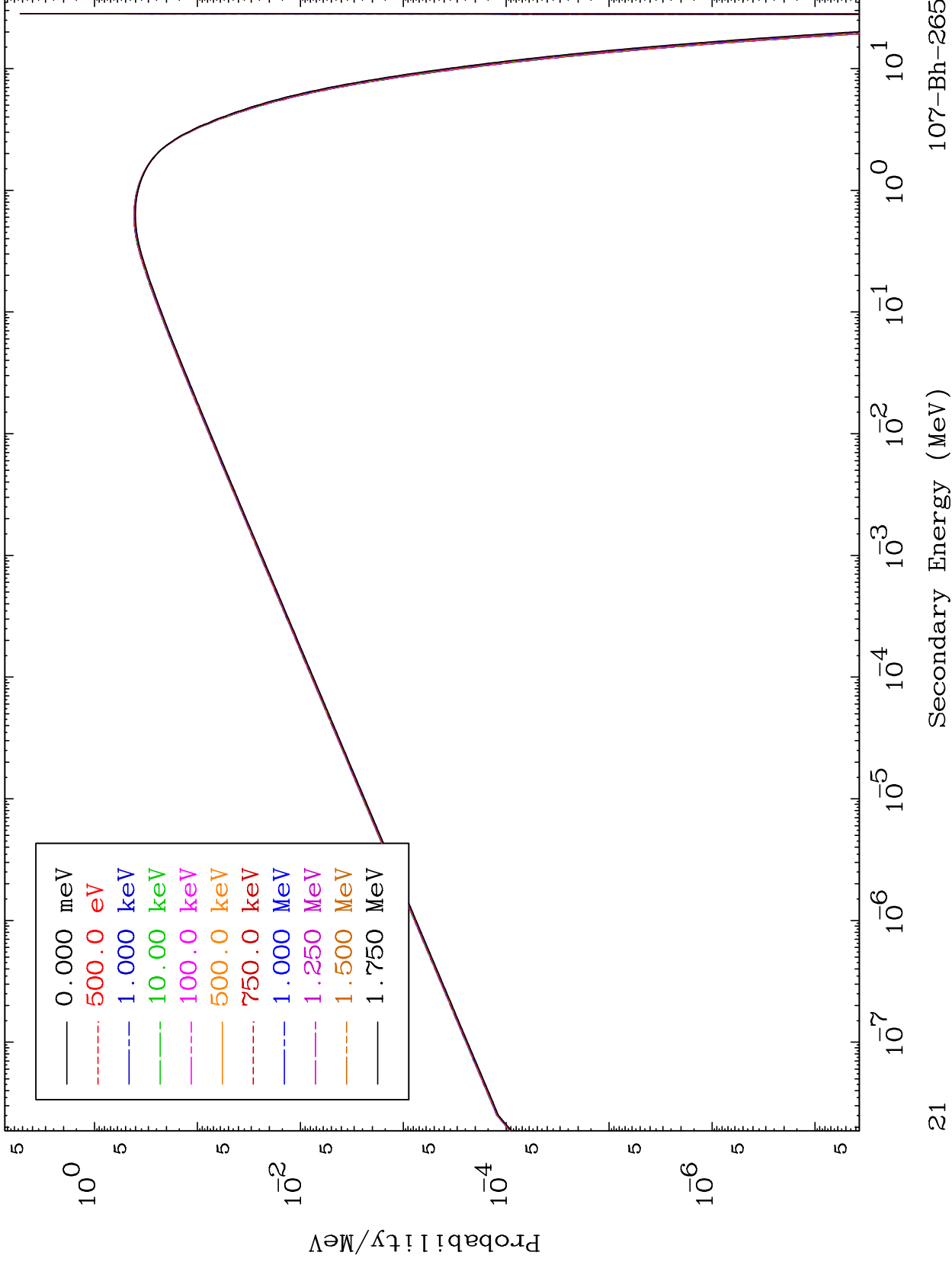




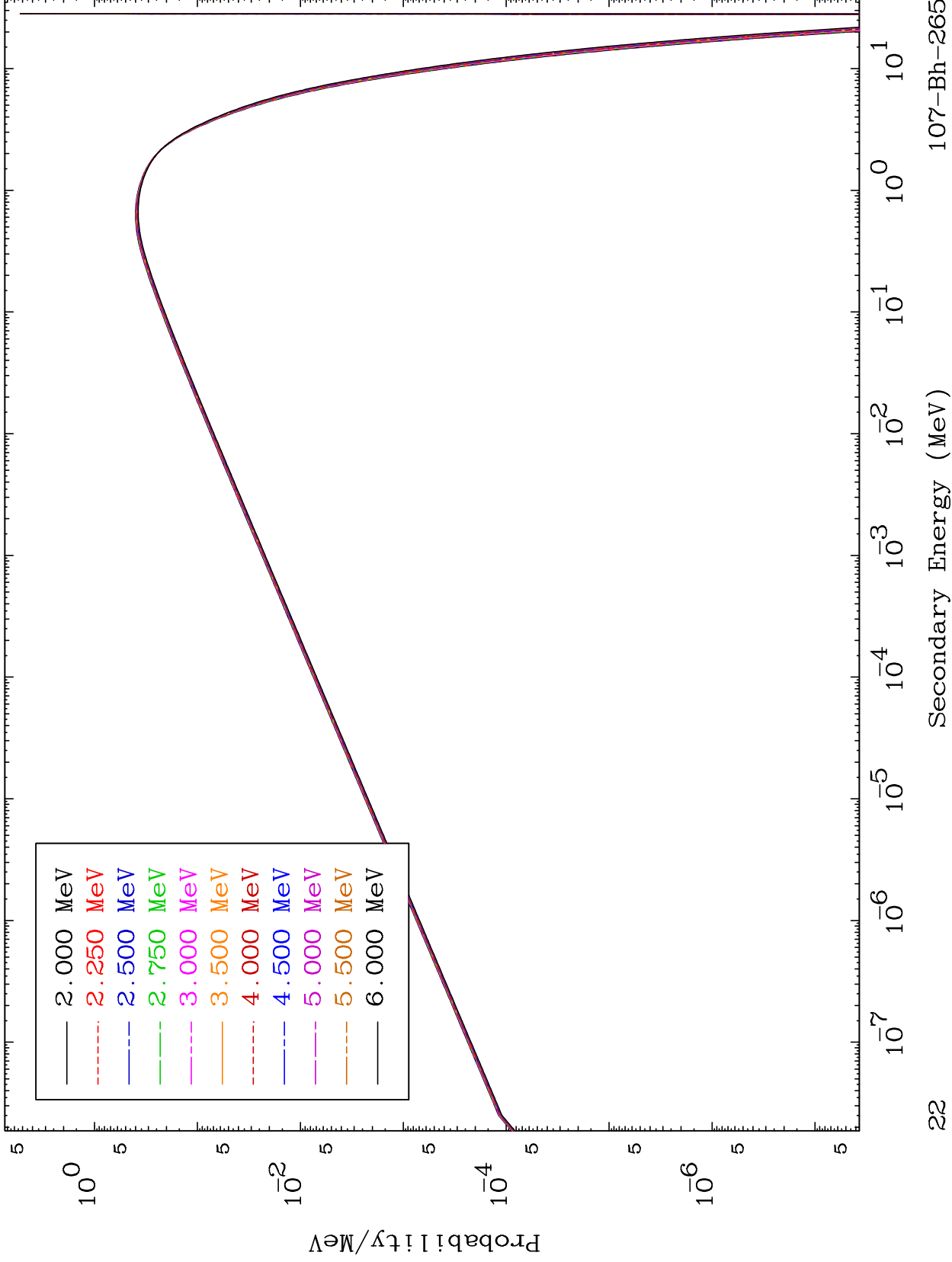




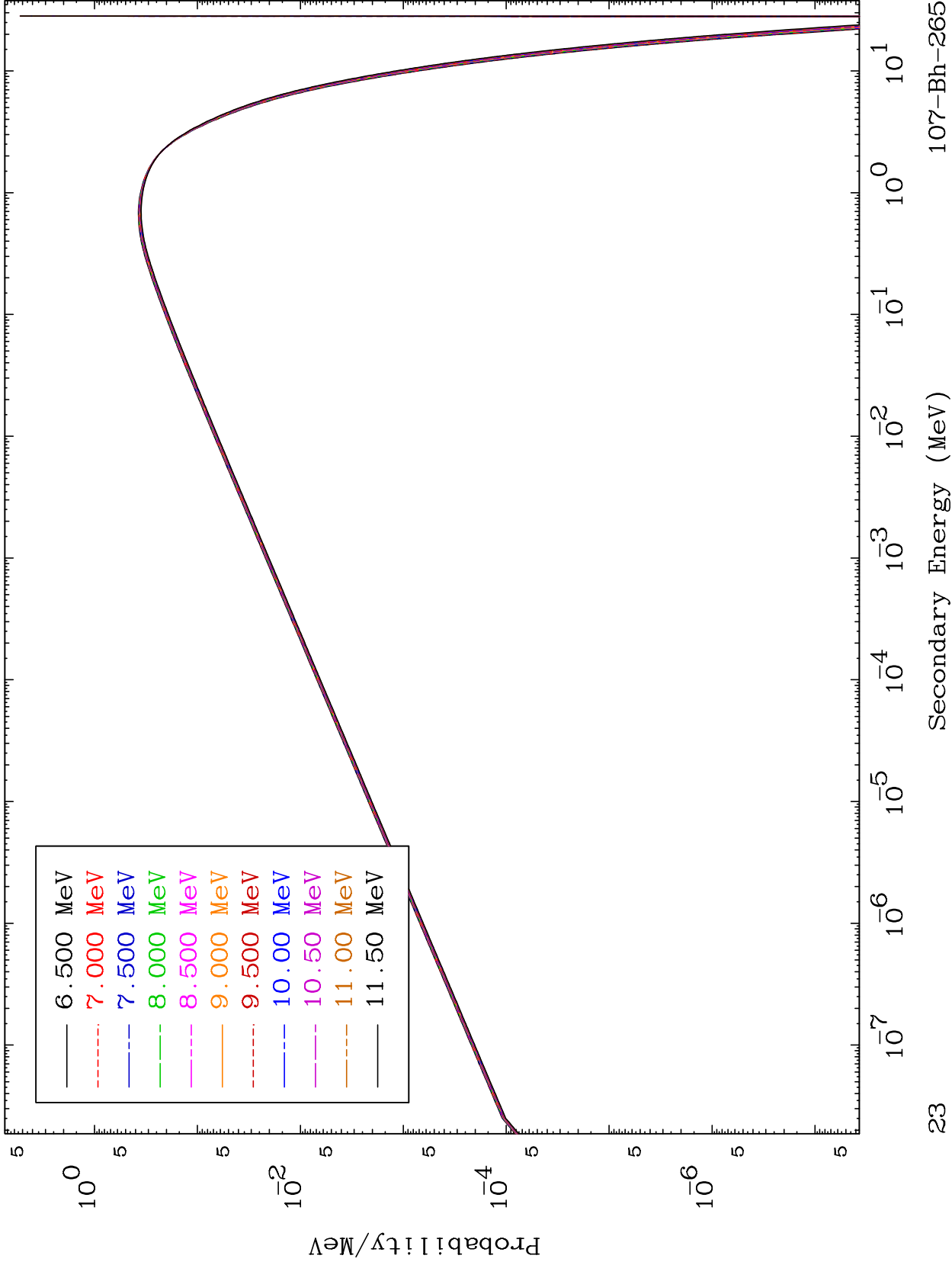




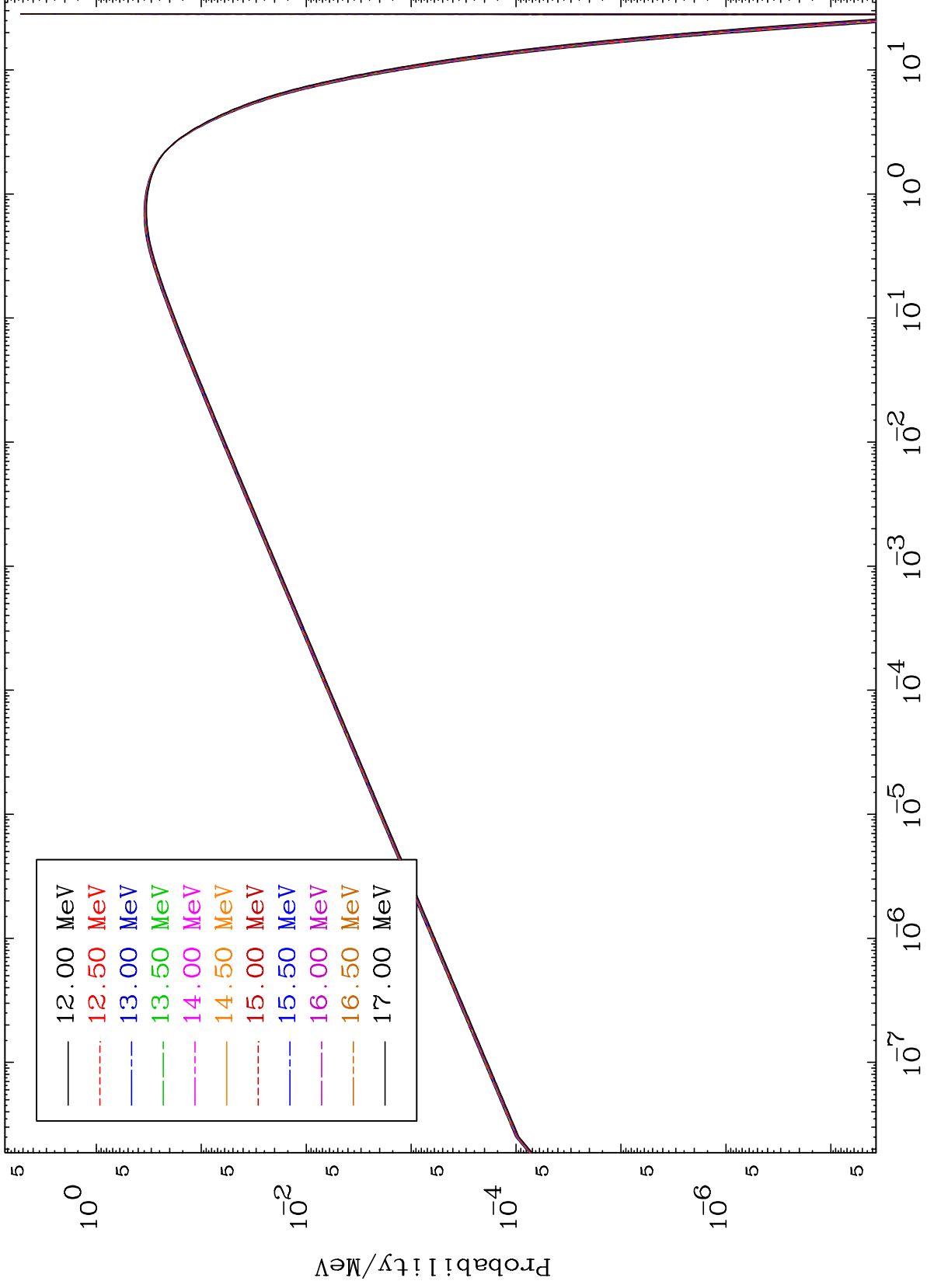
Fission
Energy Distribution



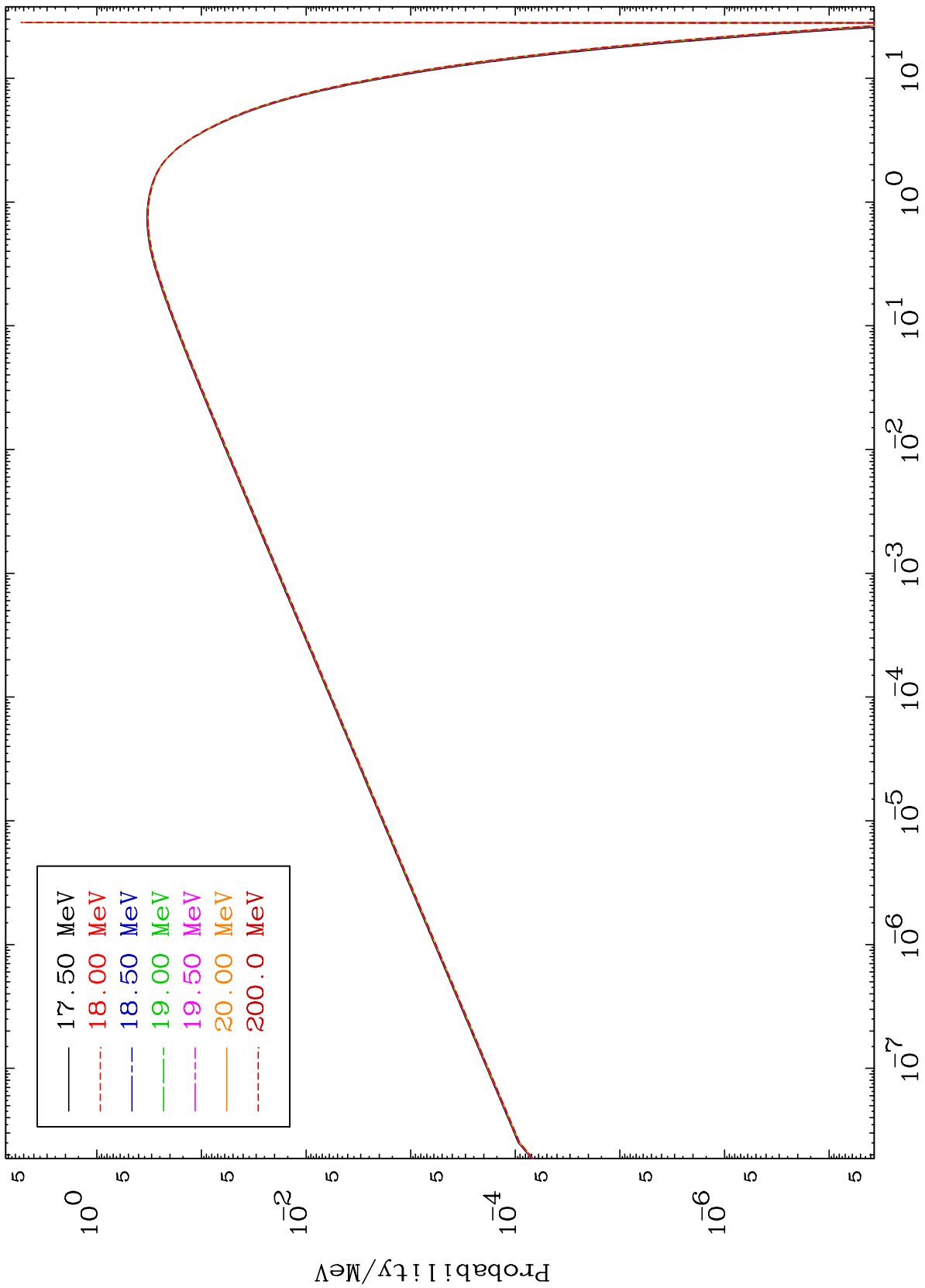
Fission
Energy Distribution



Fission
Energy Distribution



Fission
Energy Distribution

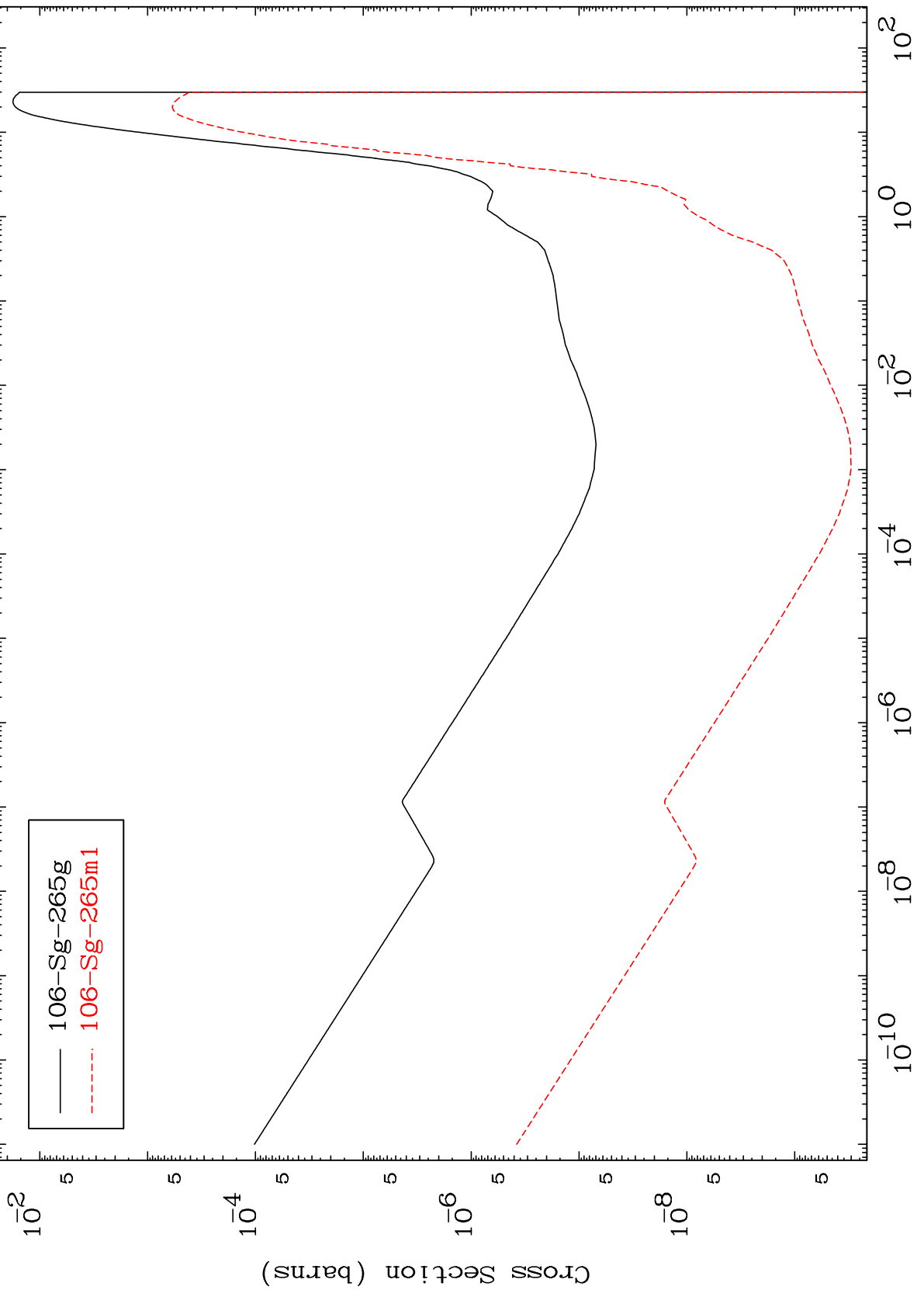


Secondary Energy (MeV)

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107-Bh-265

(n,p)
Radionuclide Production Cross Section



— 106-Sg-265g
- - - 106-Sg-265m1

107-Bh-265

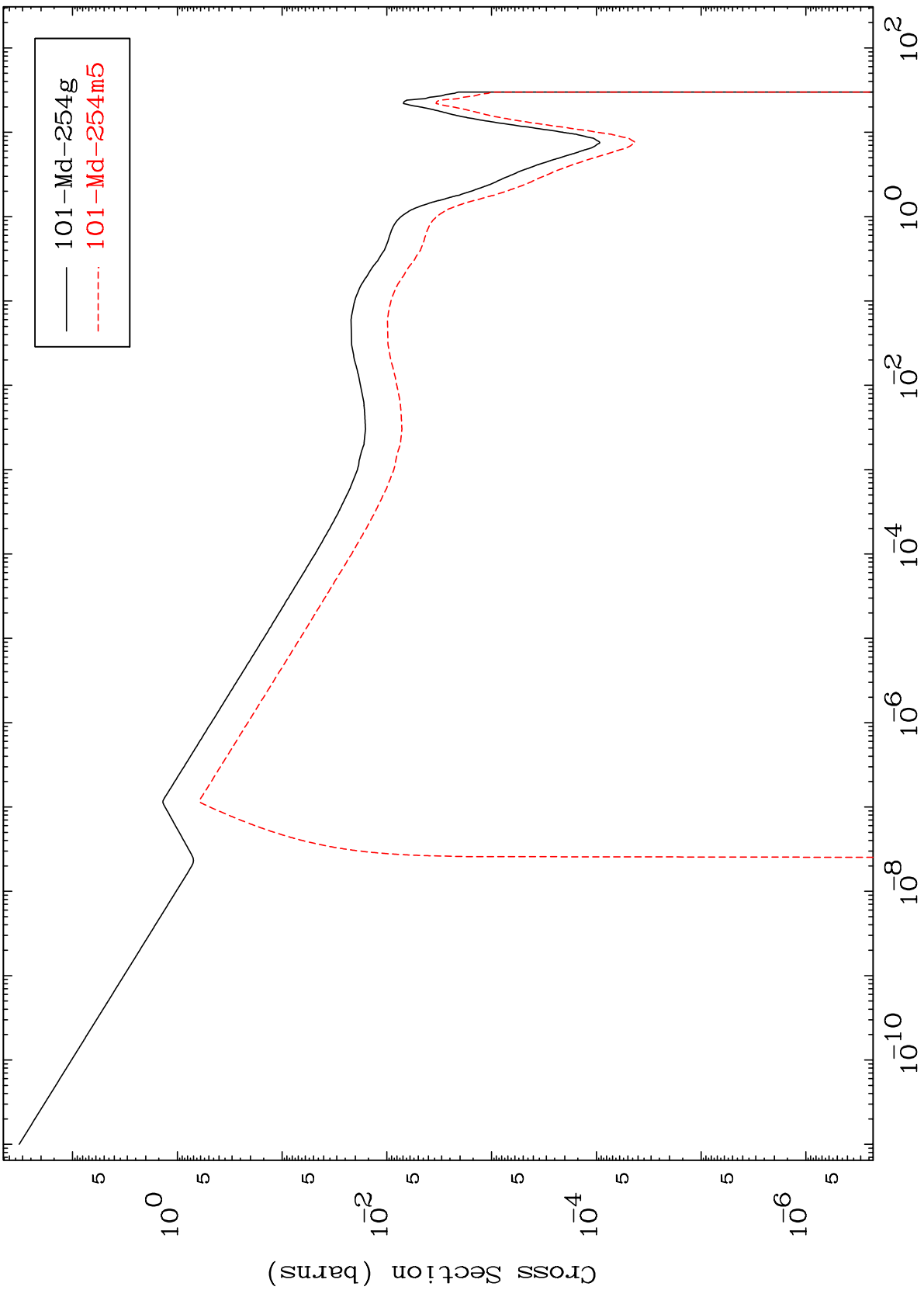
Incident Energy (MeV)

26

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107-Bh-265

$(n, 3\alpha)$
Radionuclide Production Cross Section

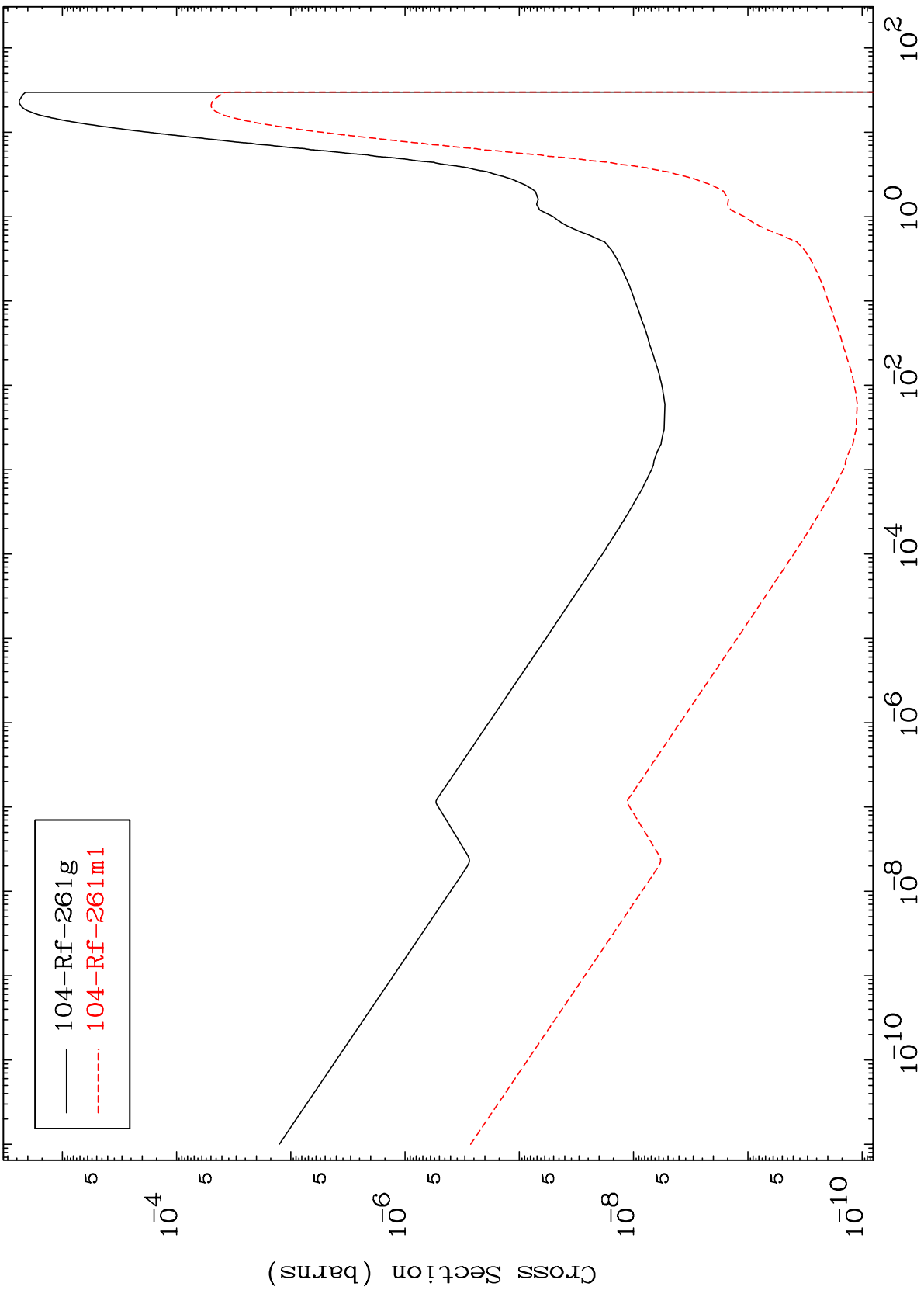
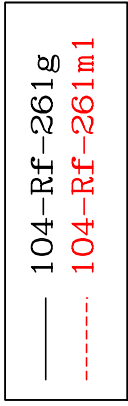


MAT 765

(n,p) α

107-Bh-265

Radionuclide Production Cross Section



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Incident Energy (MeV)

107-Bh-265