

Program EVALPLOT  
(Version 2015-2)

by

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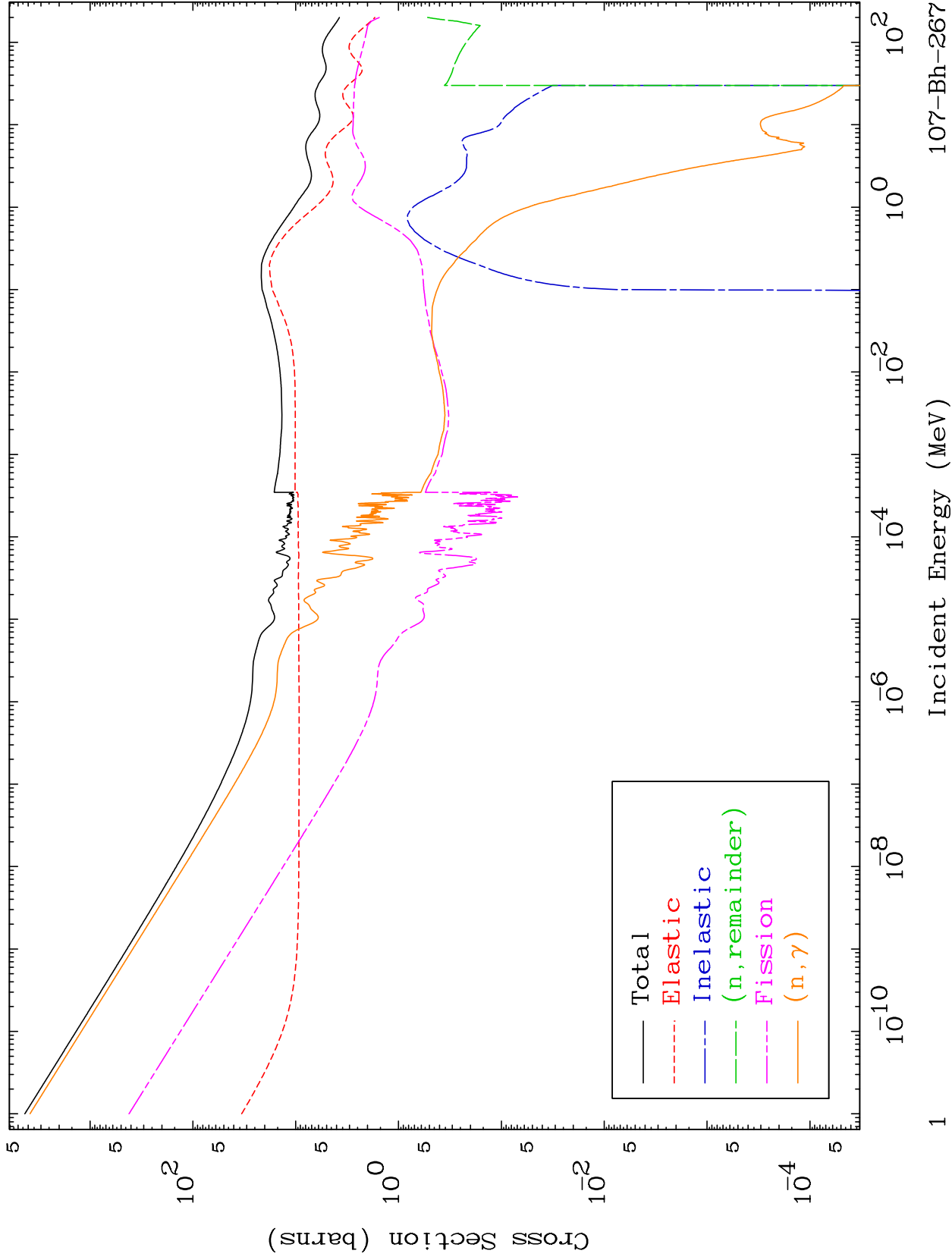
Press Mouse Button to Start

MAT 767

Major

293 Kelvin Cross Sections

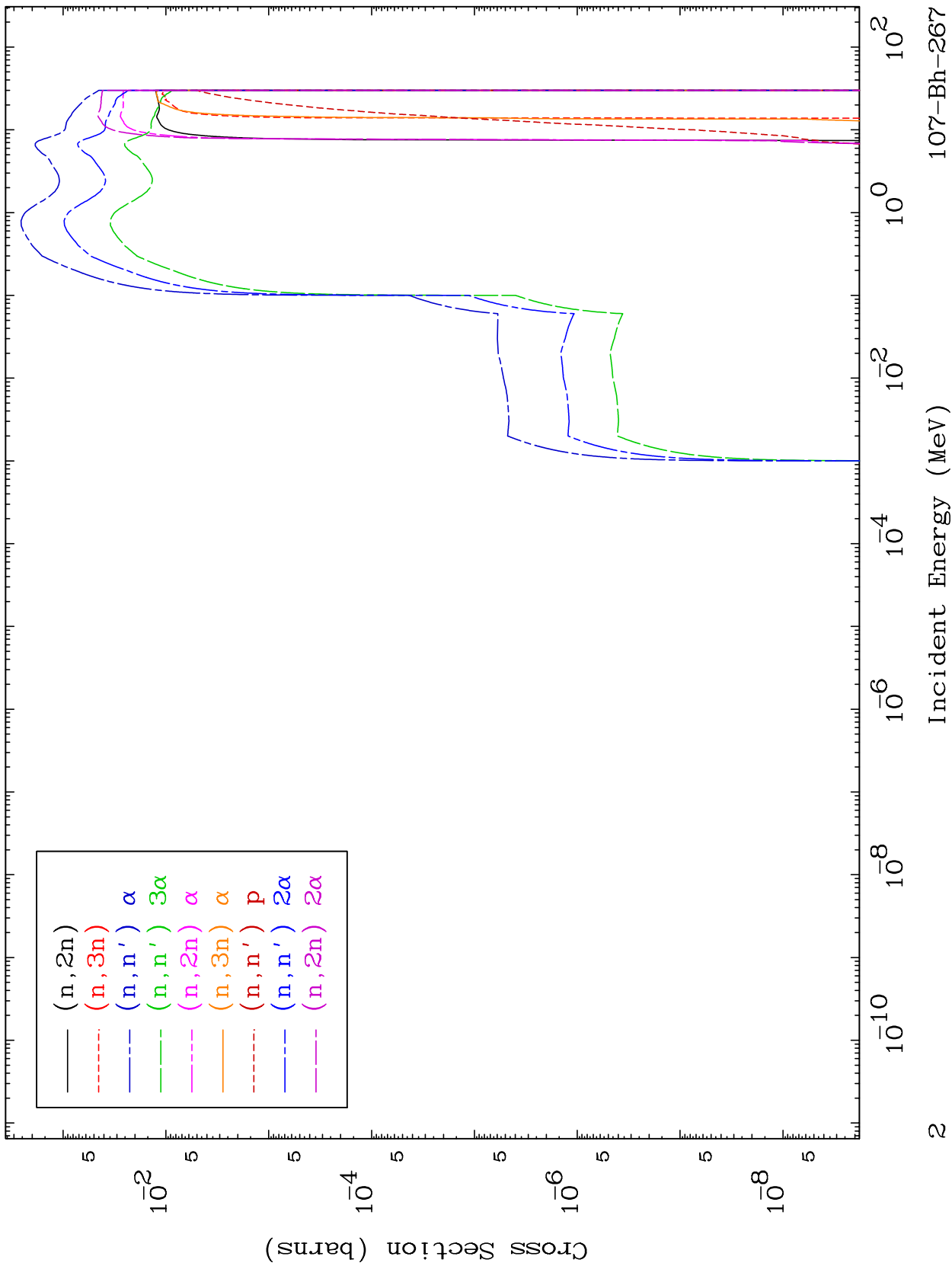
107-Bh-267



MAT 767

Neutron Production  
293 Kelvin Cross Sections

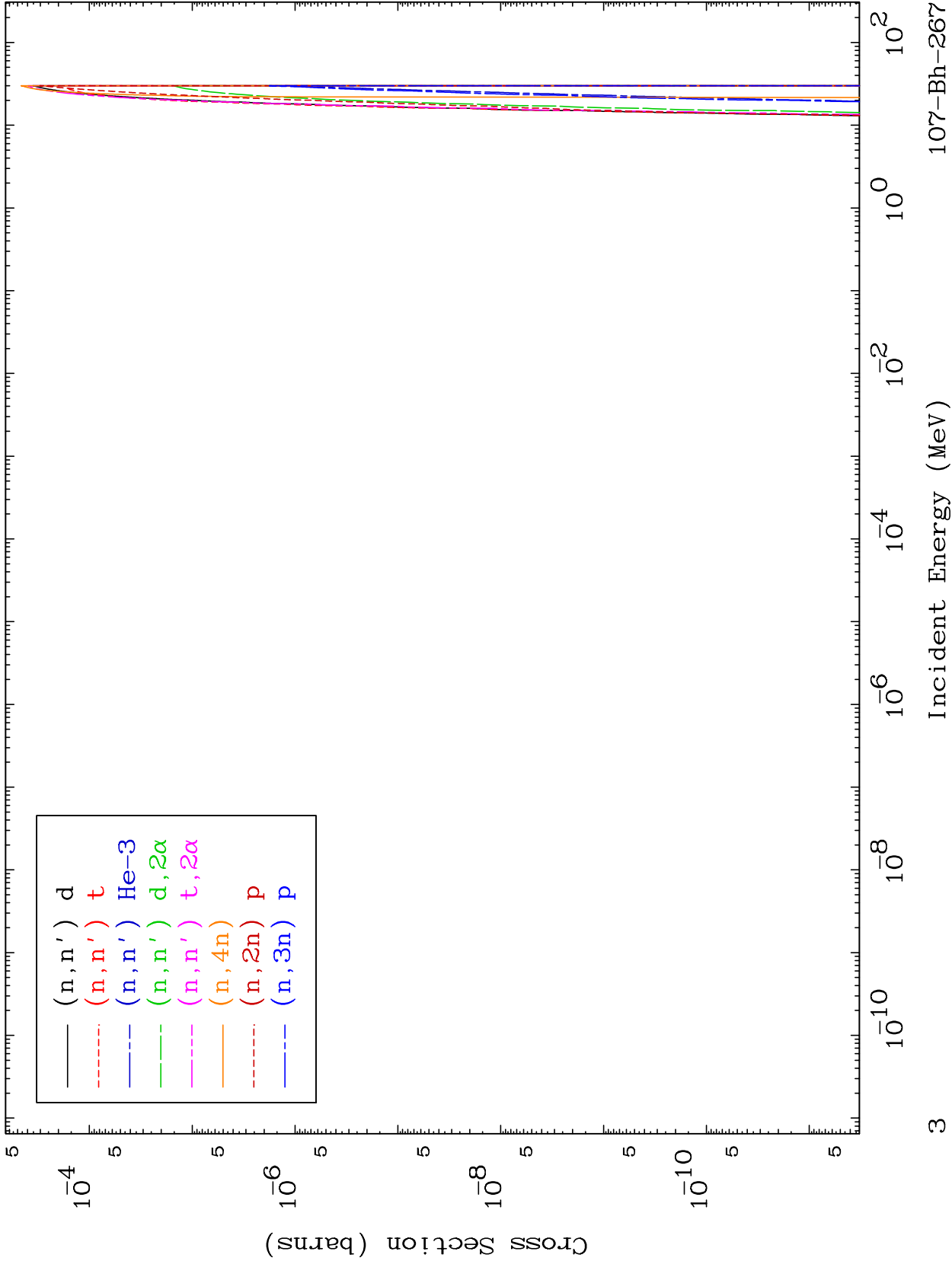
107-Bh-267



MAT 767

Neutron Production  
293 Kelvin Cross Sections

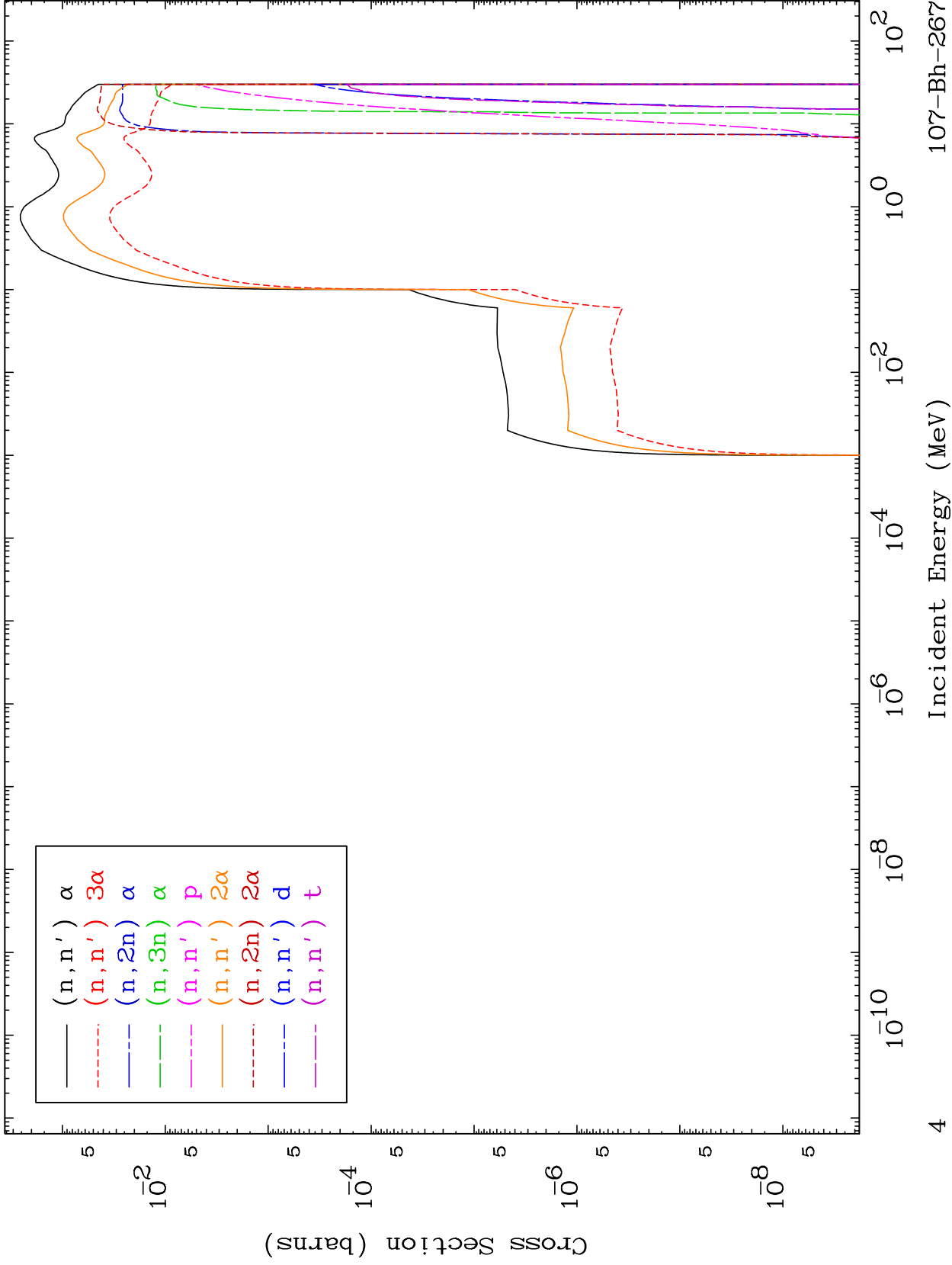
107-Bh-267



MAT 767

Charged Particle  
293 Kelvin Cross Sections

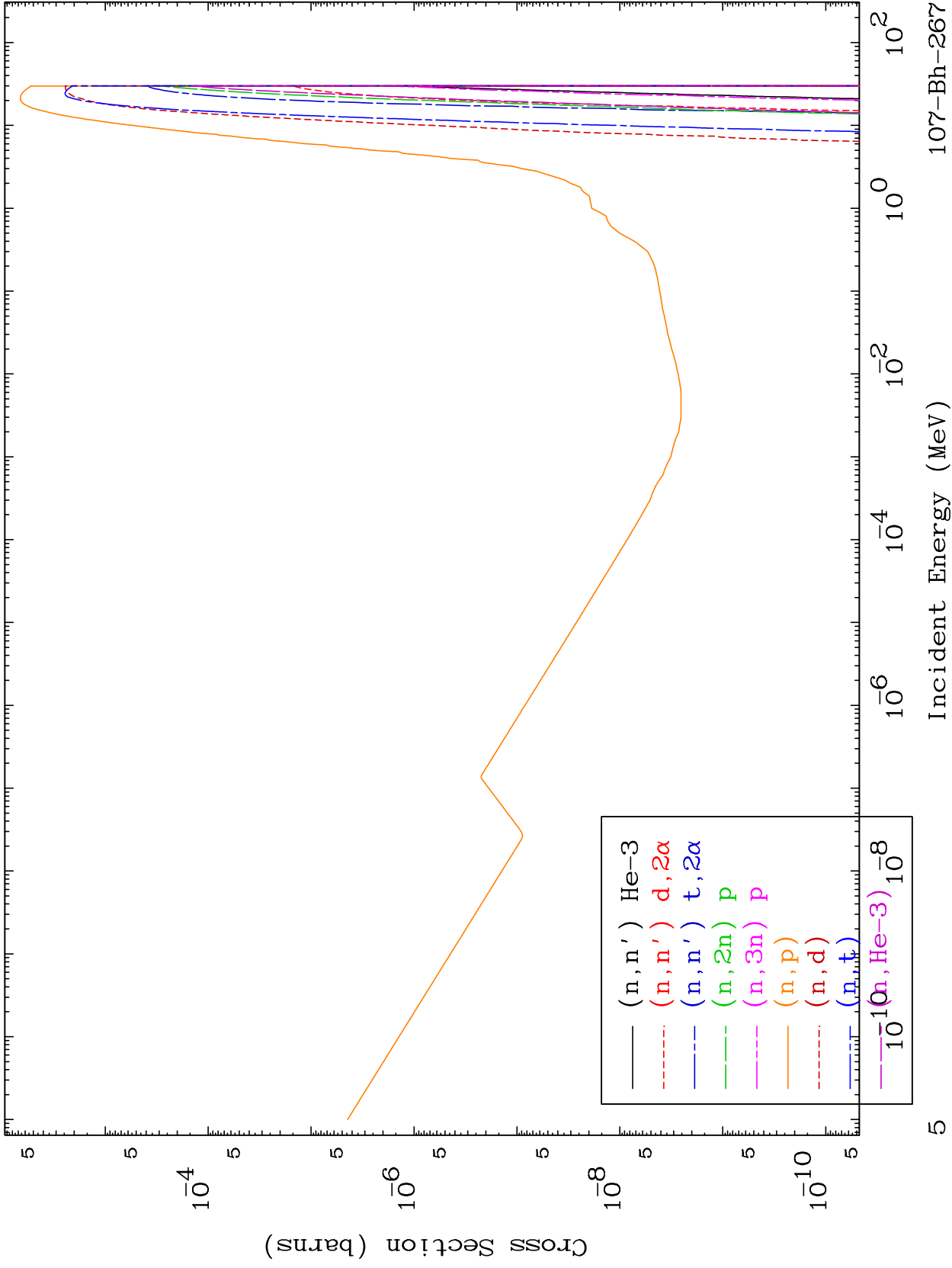
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Charged Particle  
293 Kelvin Cross Sections

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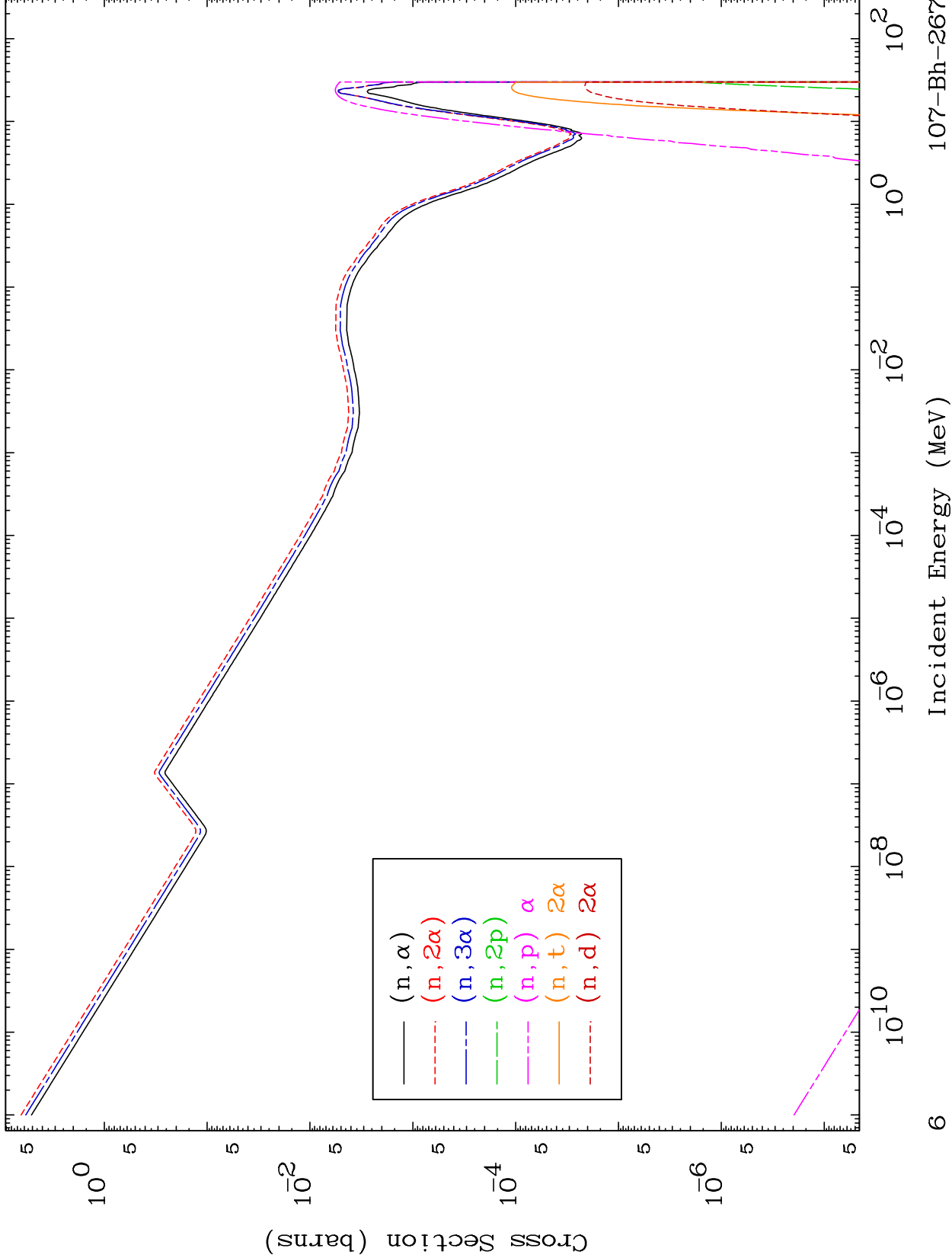


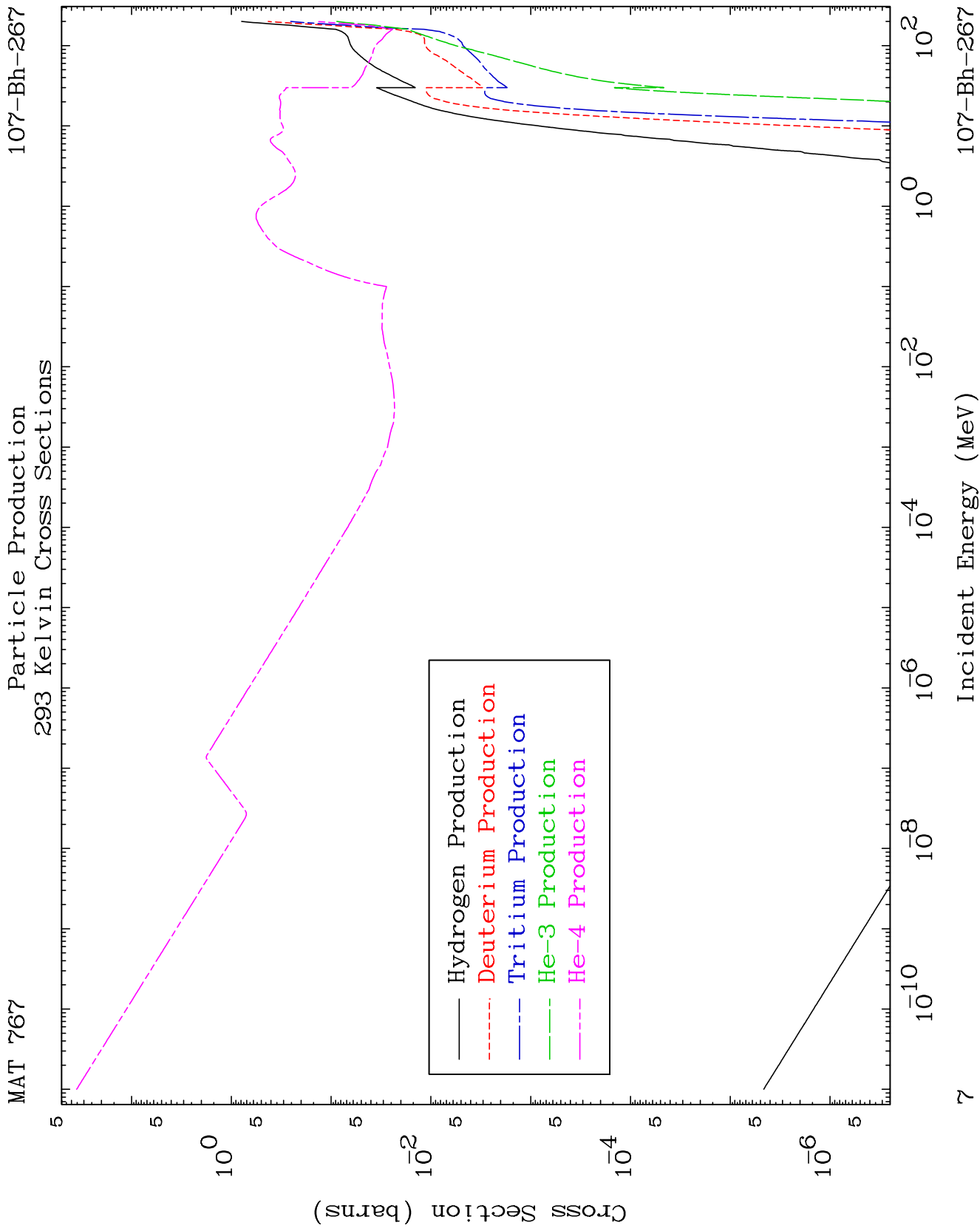
5

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Charged Particle  
293 Kelvin Cross Sections

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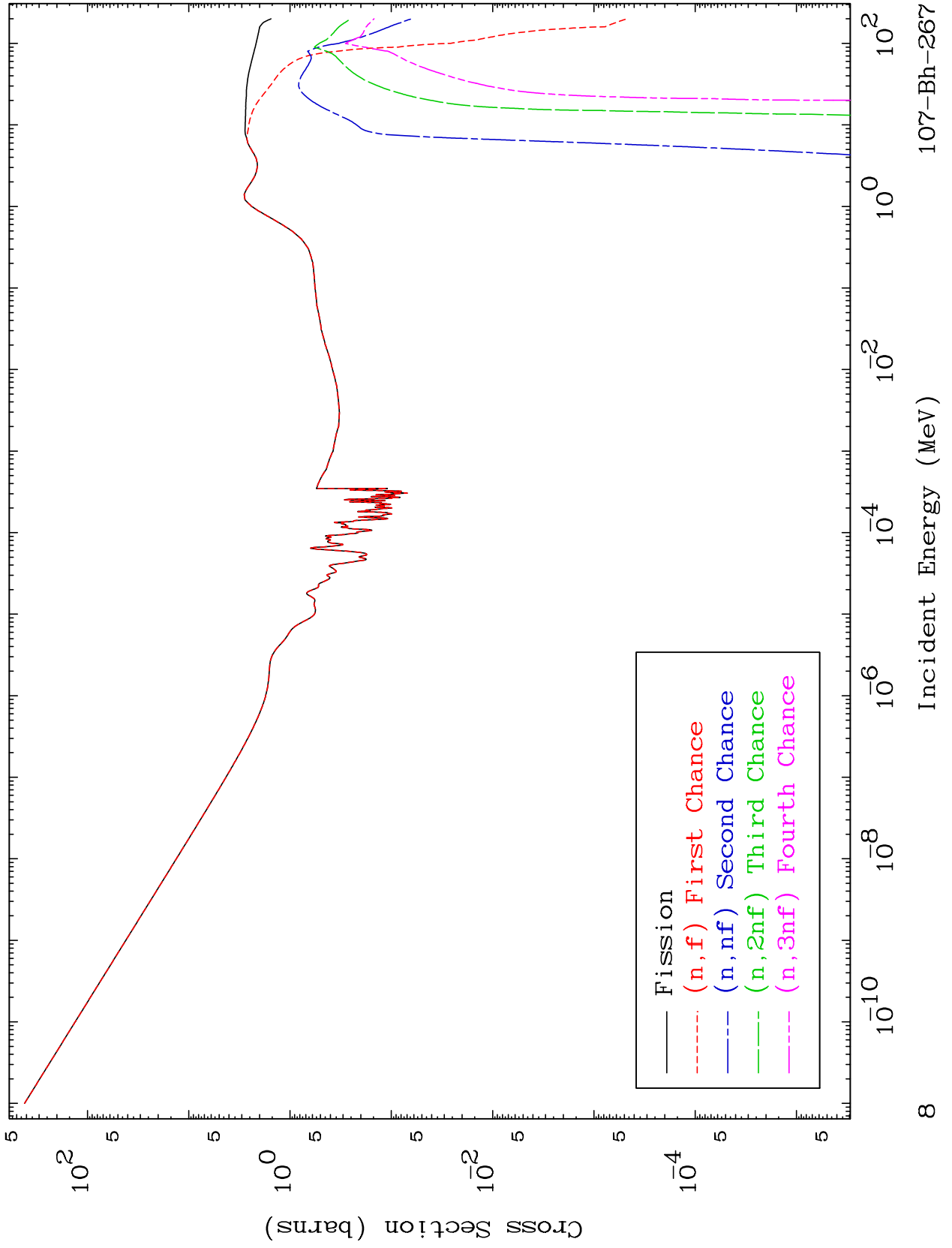




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Fission  
293 Kelvin Cross Sections

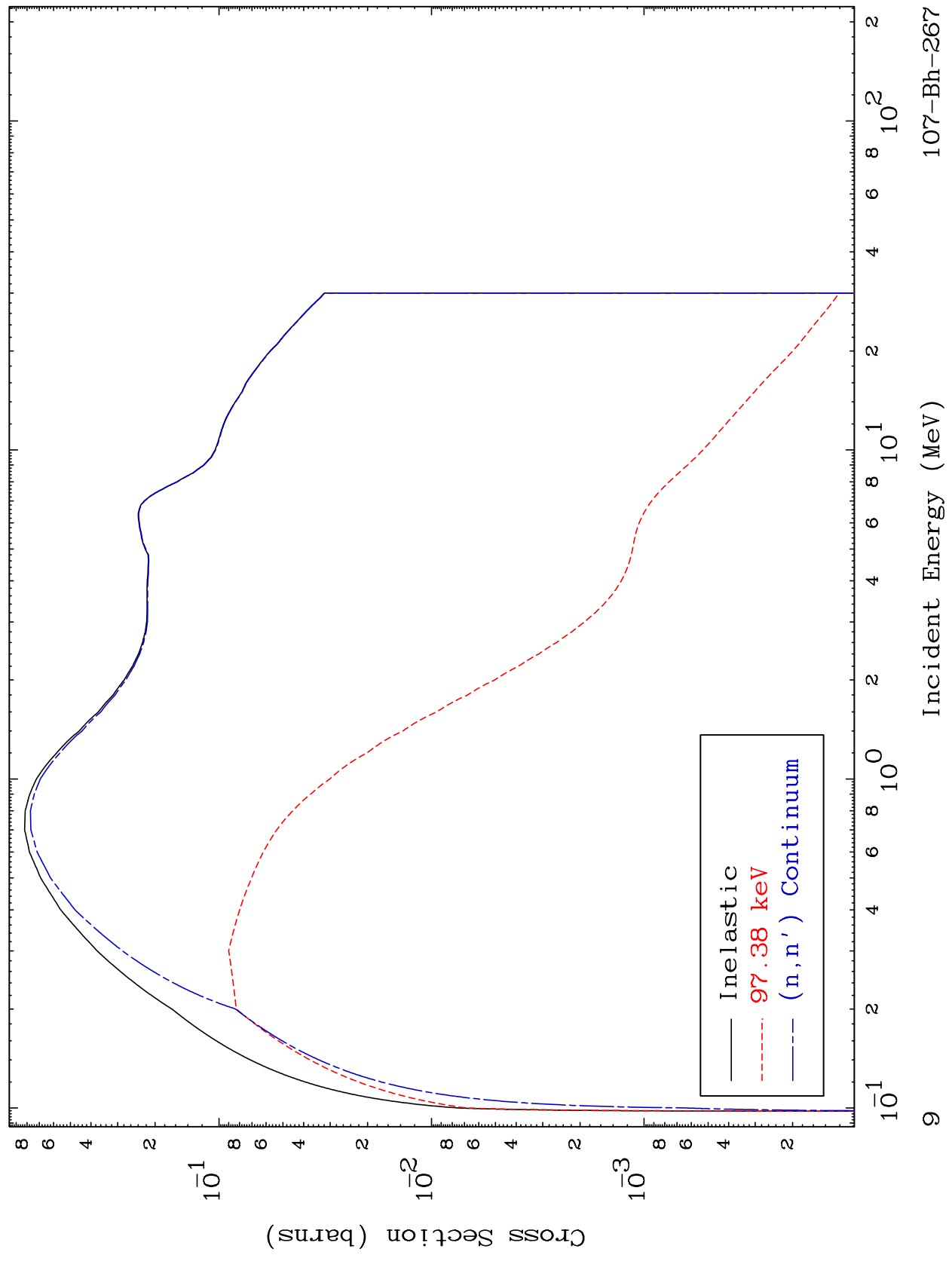
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MAT 767

(n,n') Level  
293 Kelvin Cross Sections

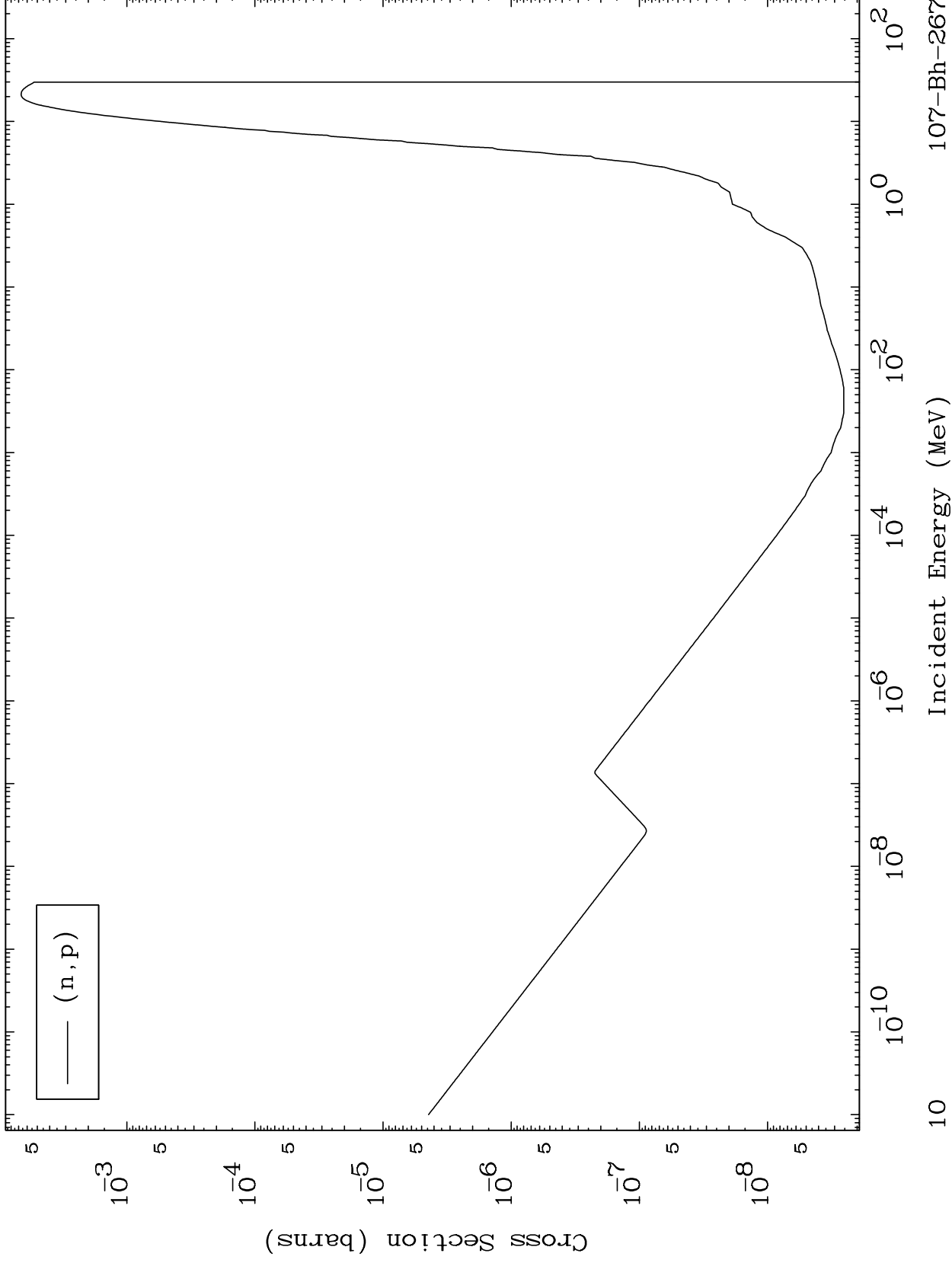
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(n,p) Levels  
293 Kelvin Cross Sections

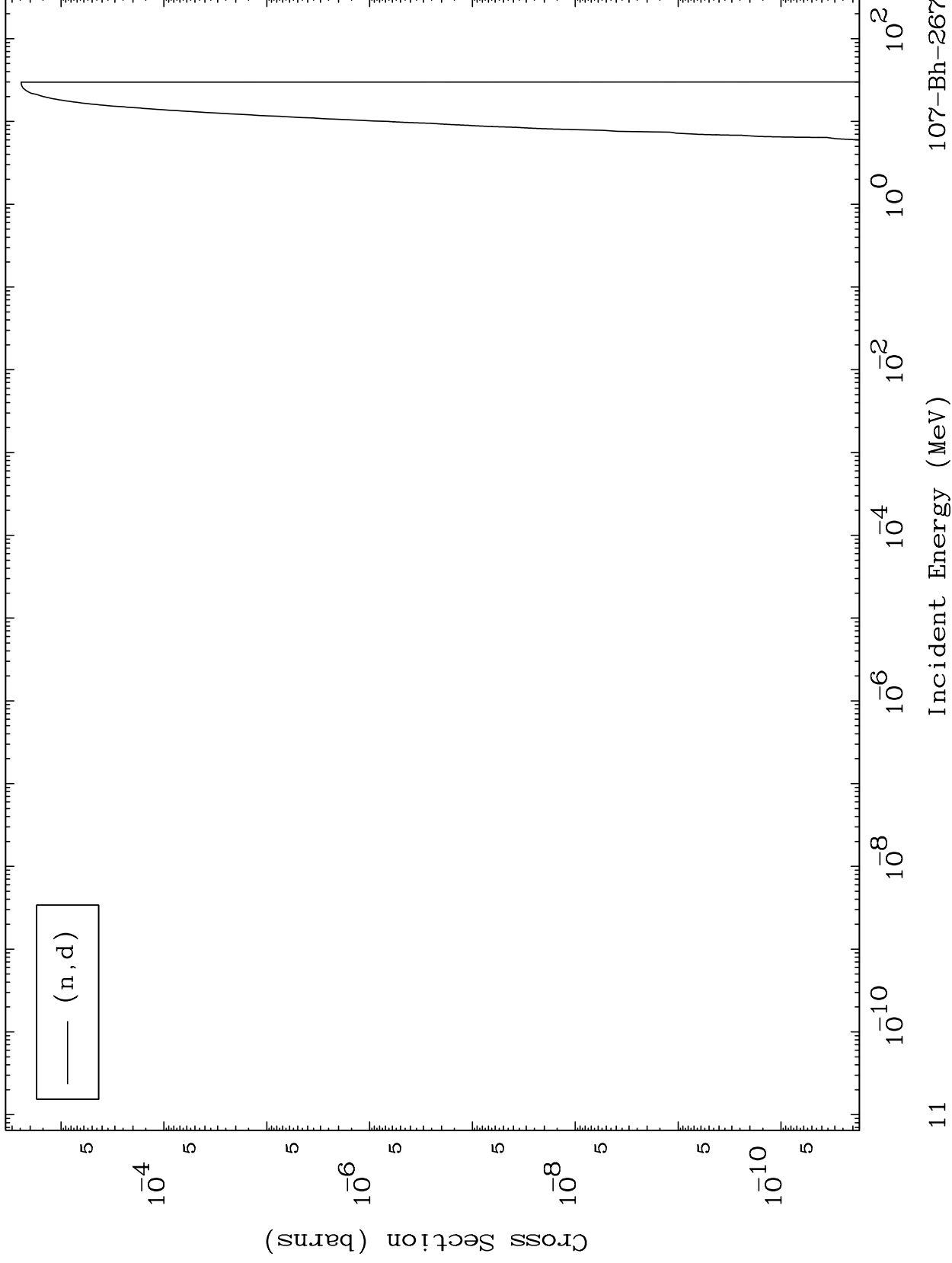
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MAT 767

(n,d) Levels  
293 Kelvin Cross Sections

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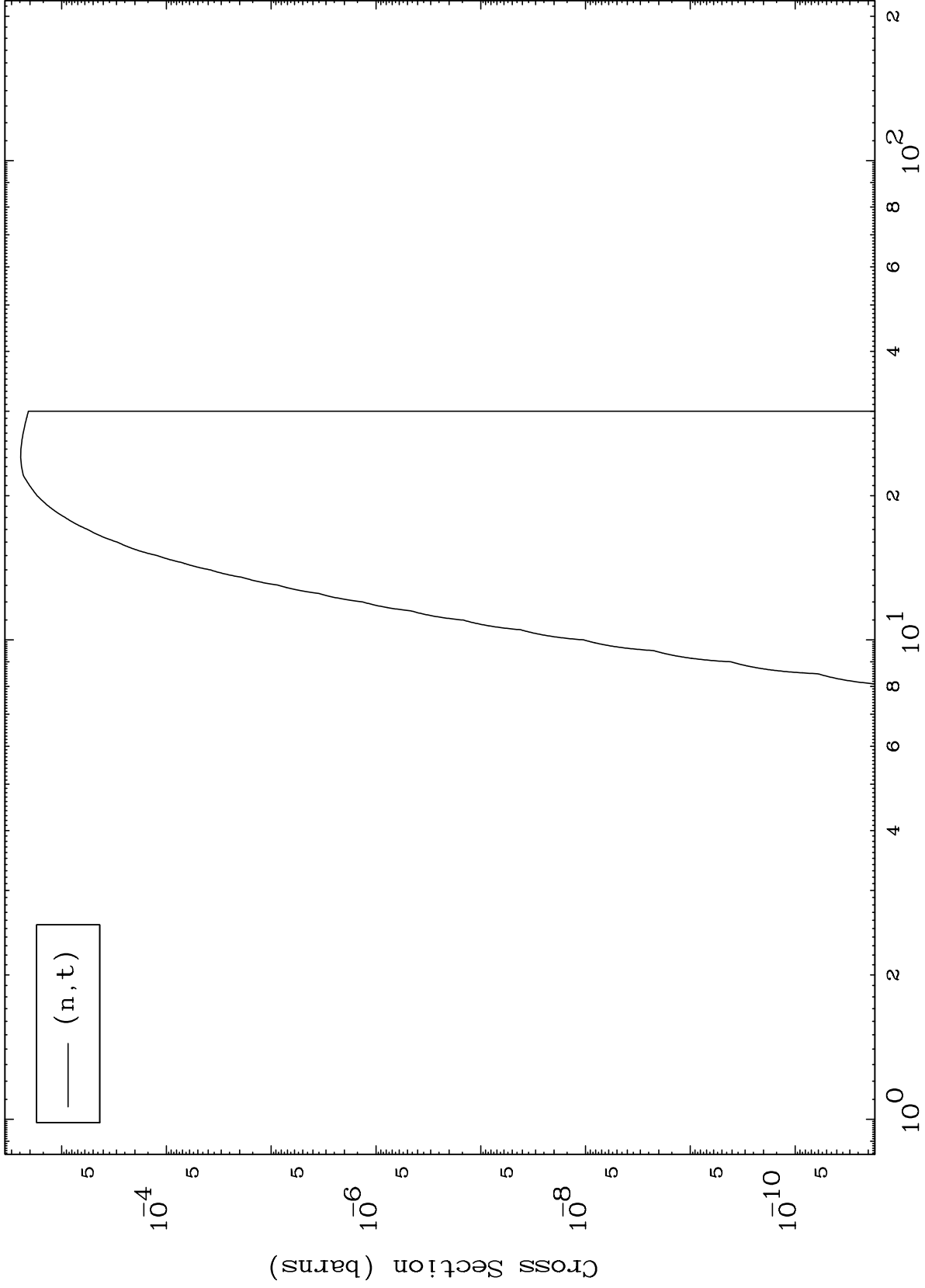


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MAT 767

(n,t) Levels  
293 Kelvin Cross Sections

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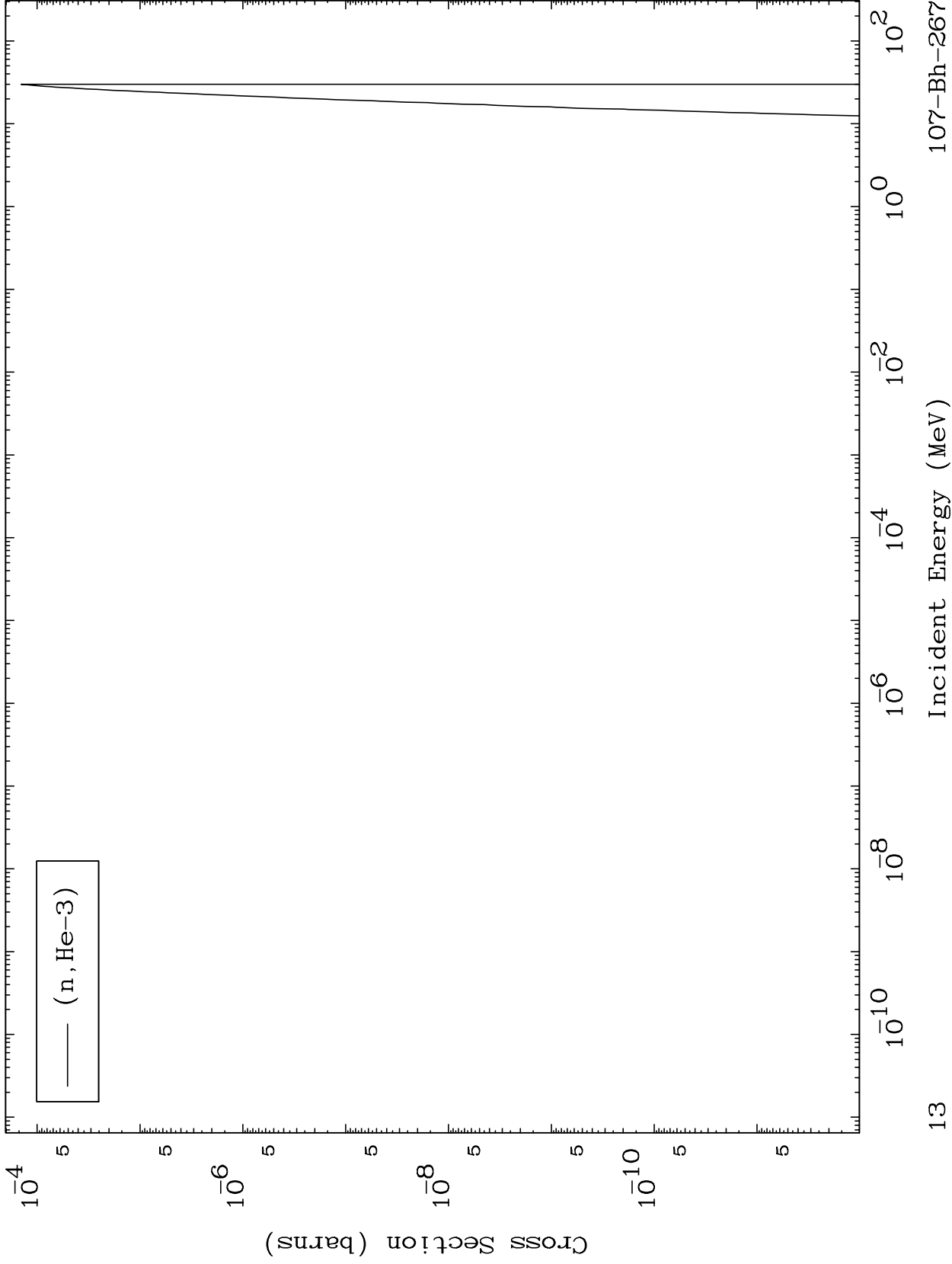
Incident Energy (MeV)

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MAT 767

(n,He3) Levels  
293 Kelvin Cross Sections

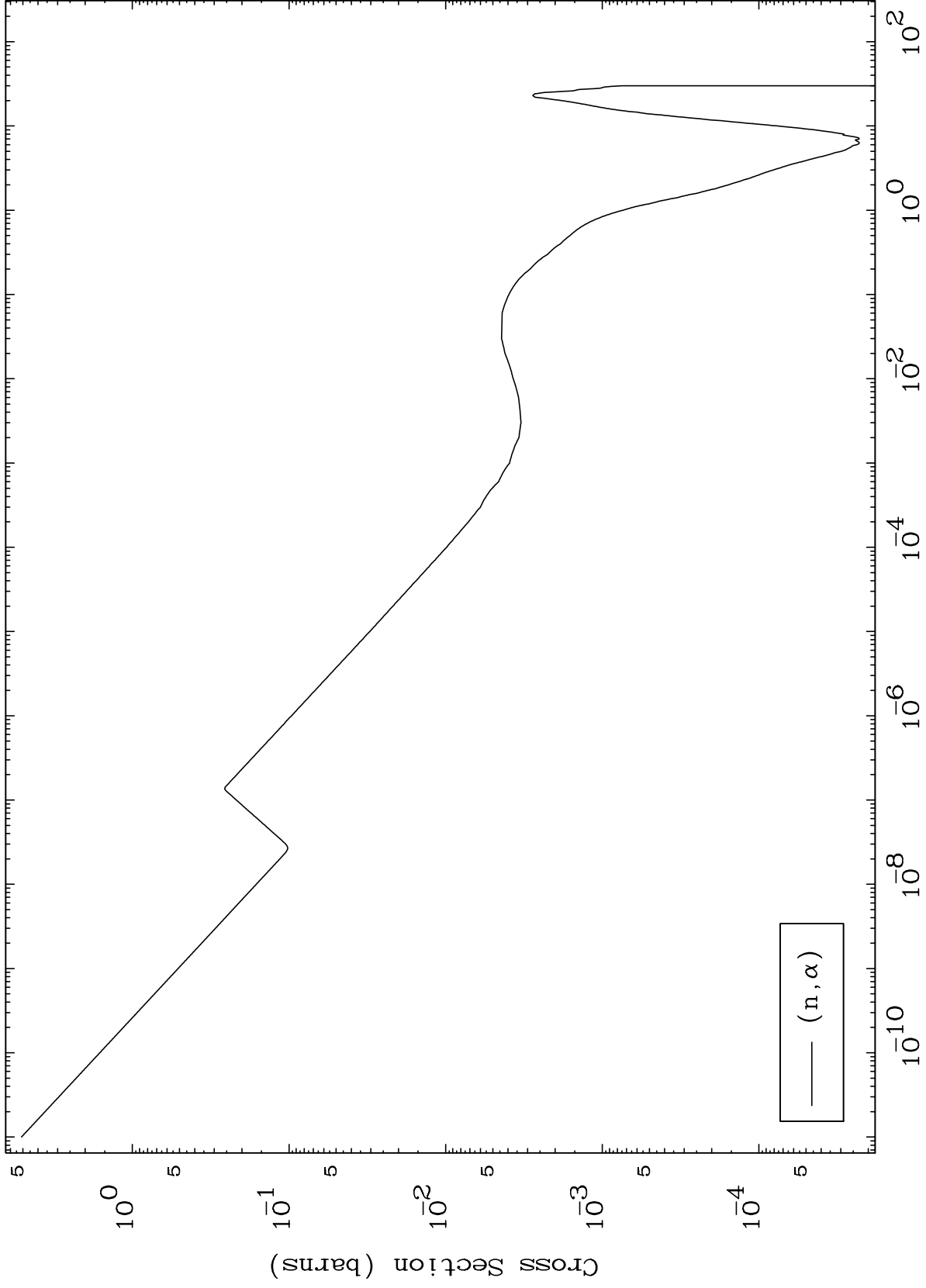
107-Bh-267



MAT 767

(n,α) Levels  
293 Kelvin Cross Sections

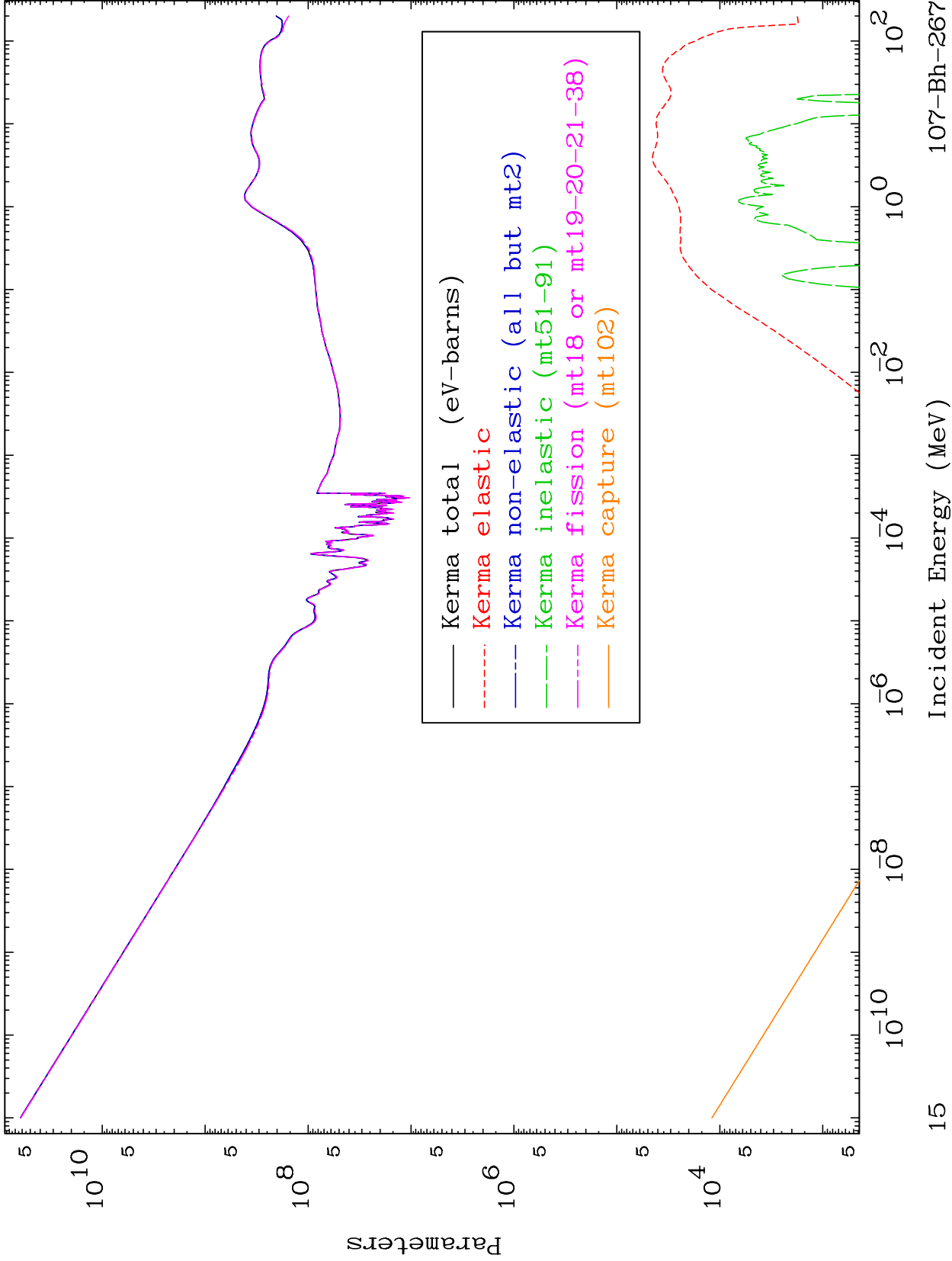
107-Bh-267



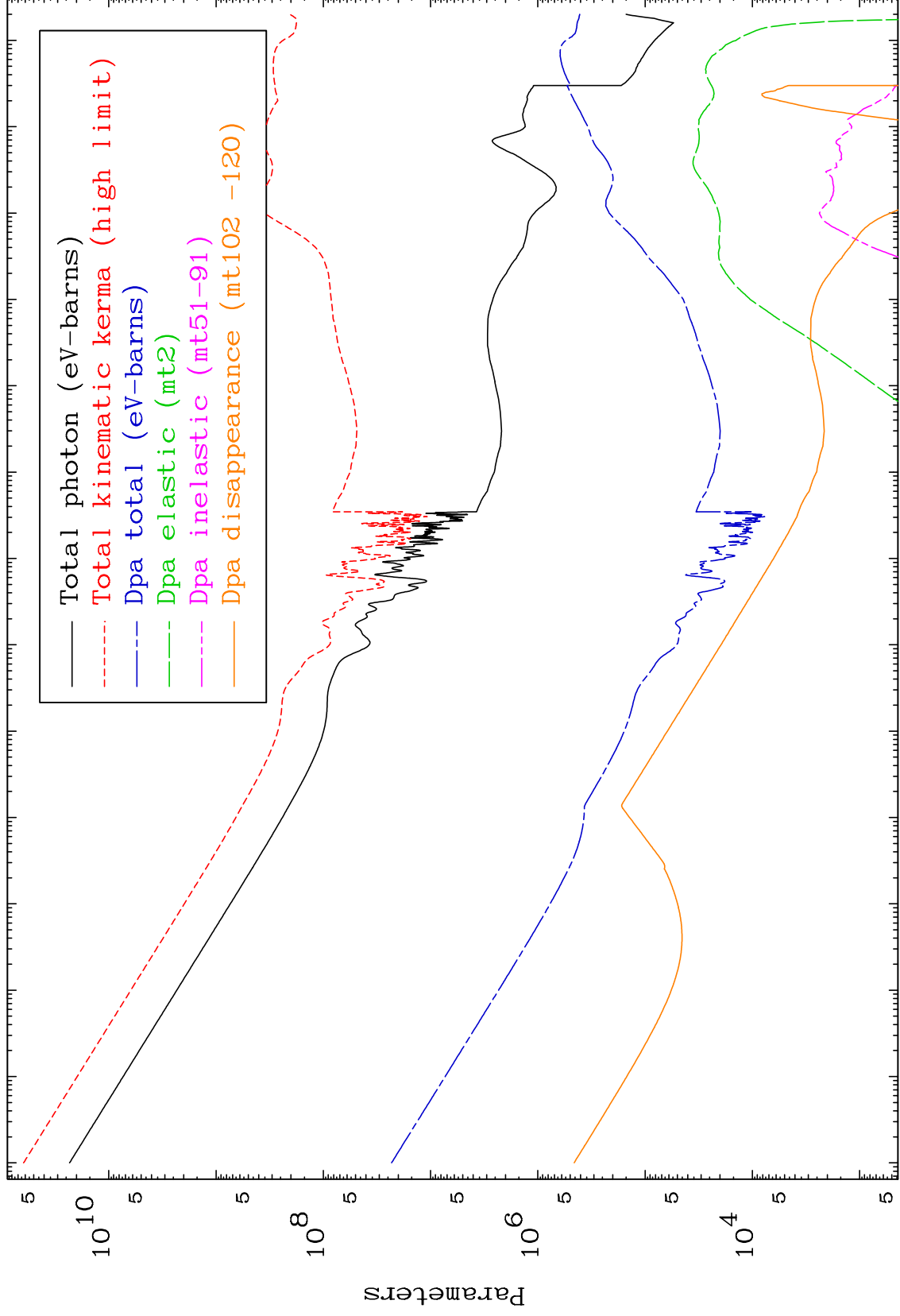
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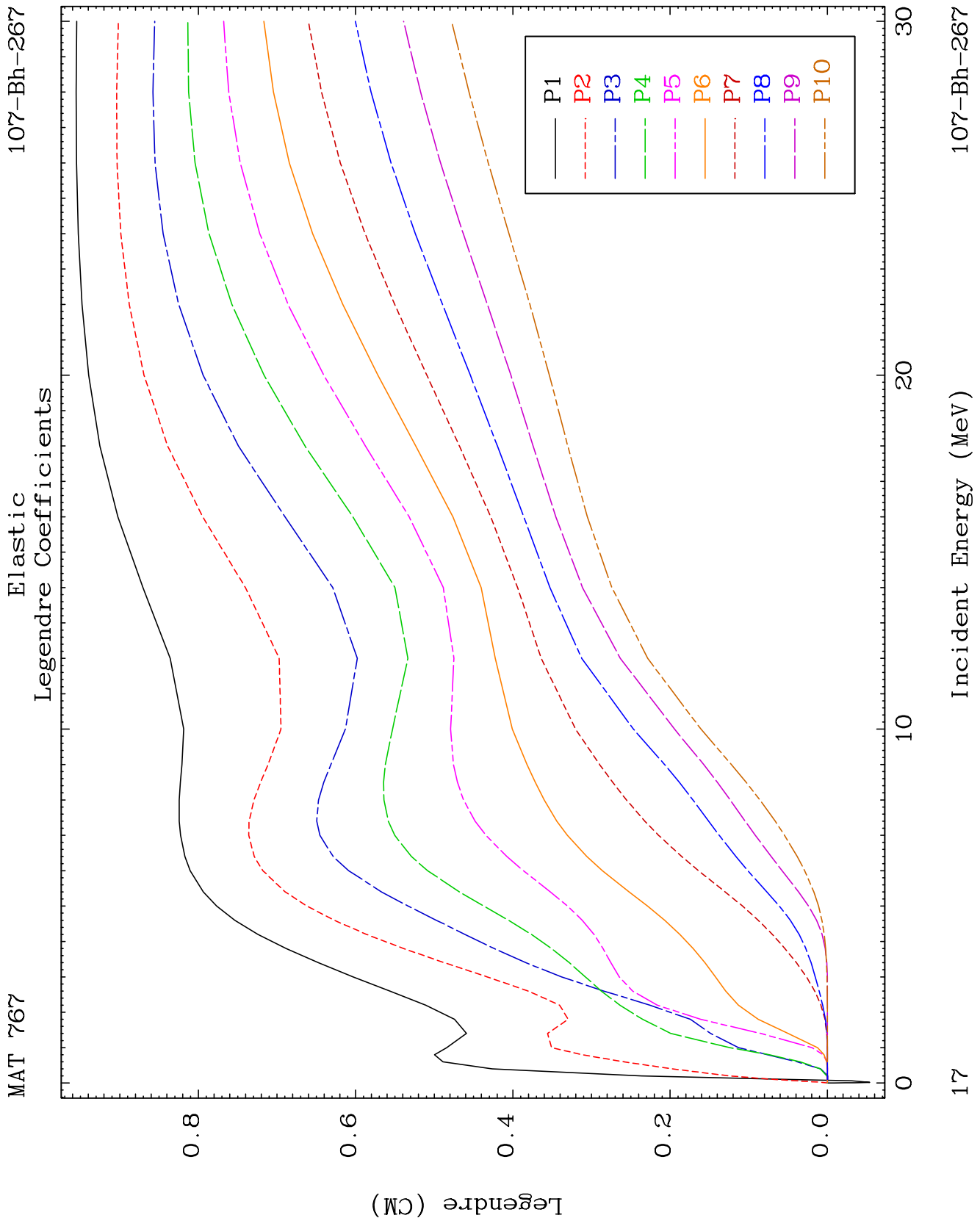
Incident Energy (MeV)

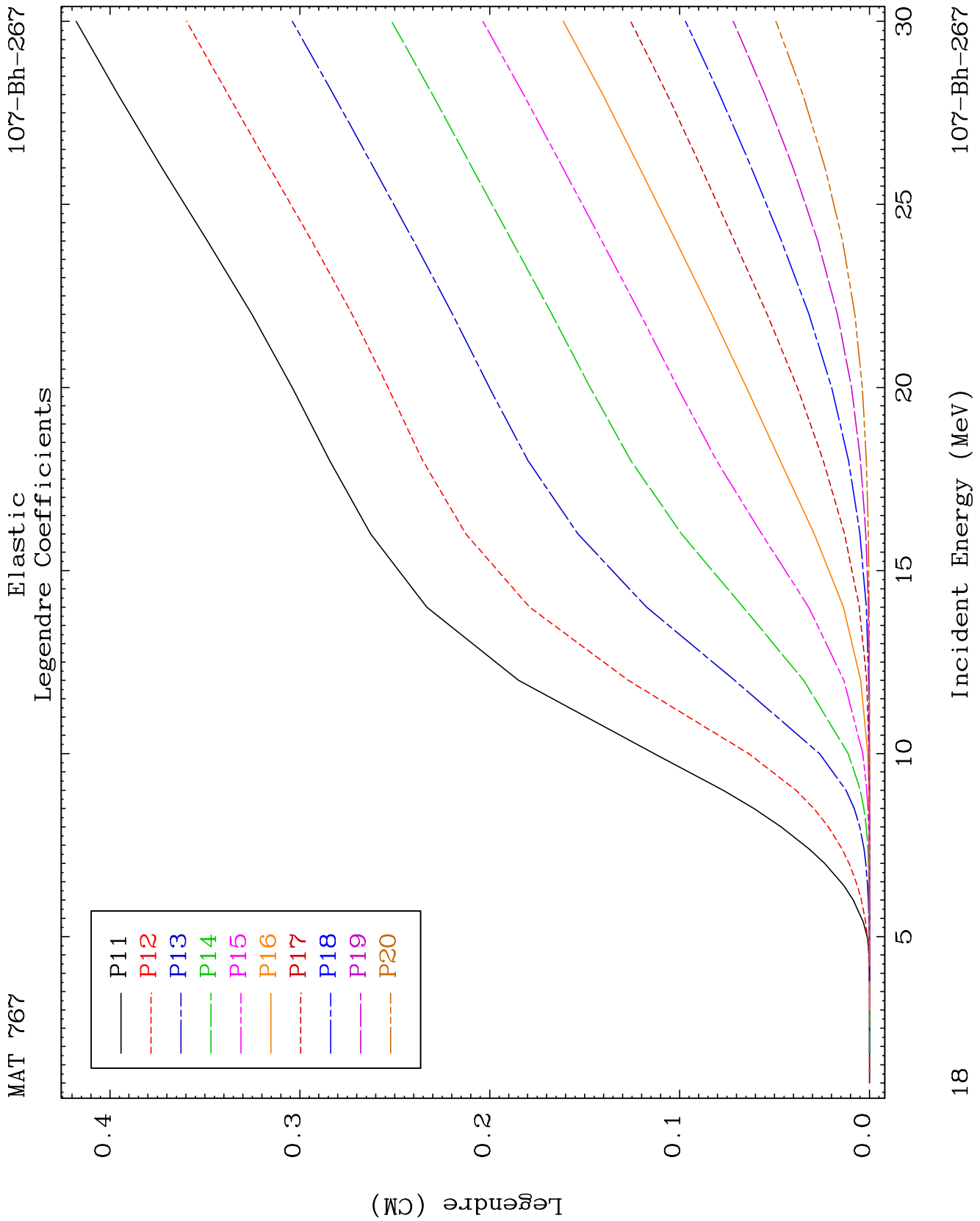
107-Bh-267







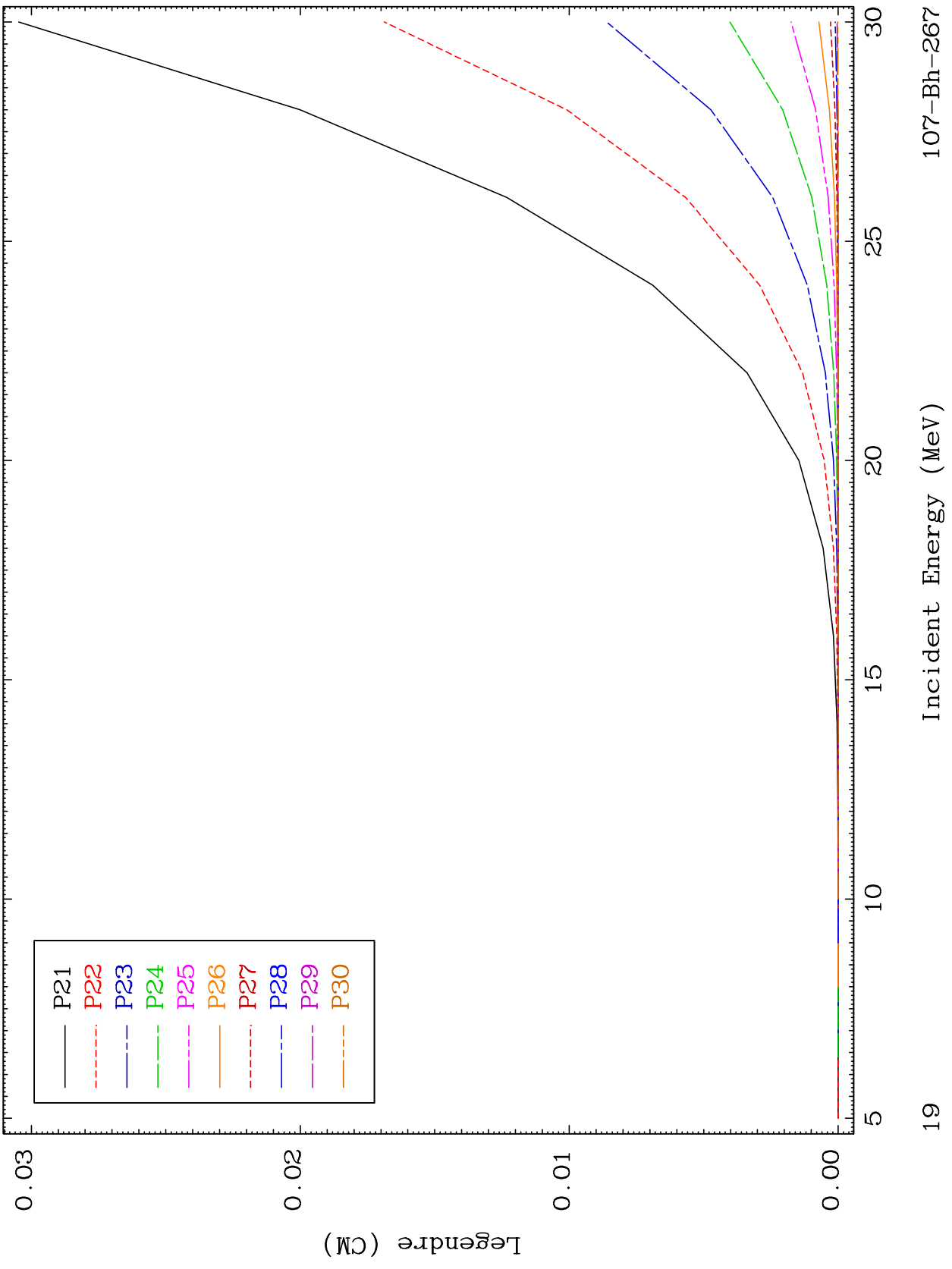




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### Elastic Legendre Coefficients

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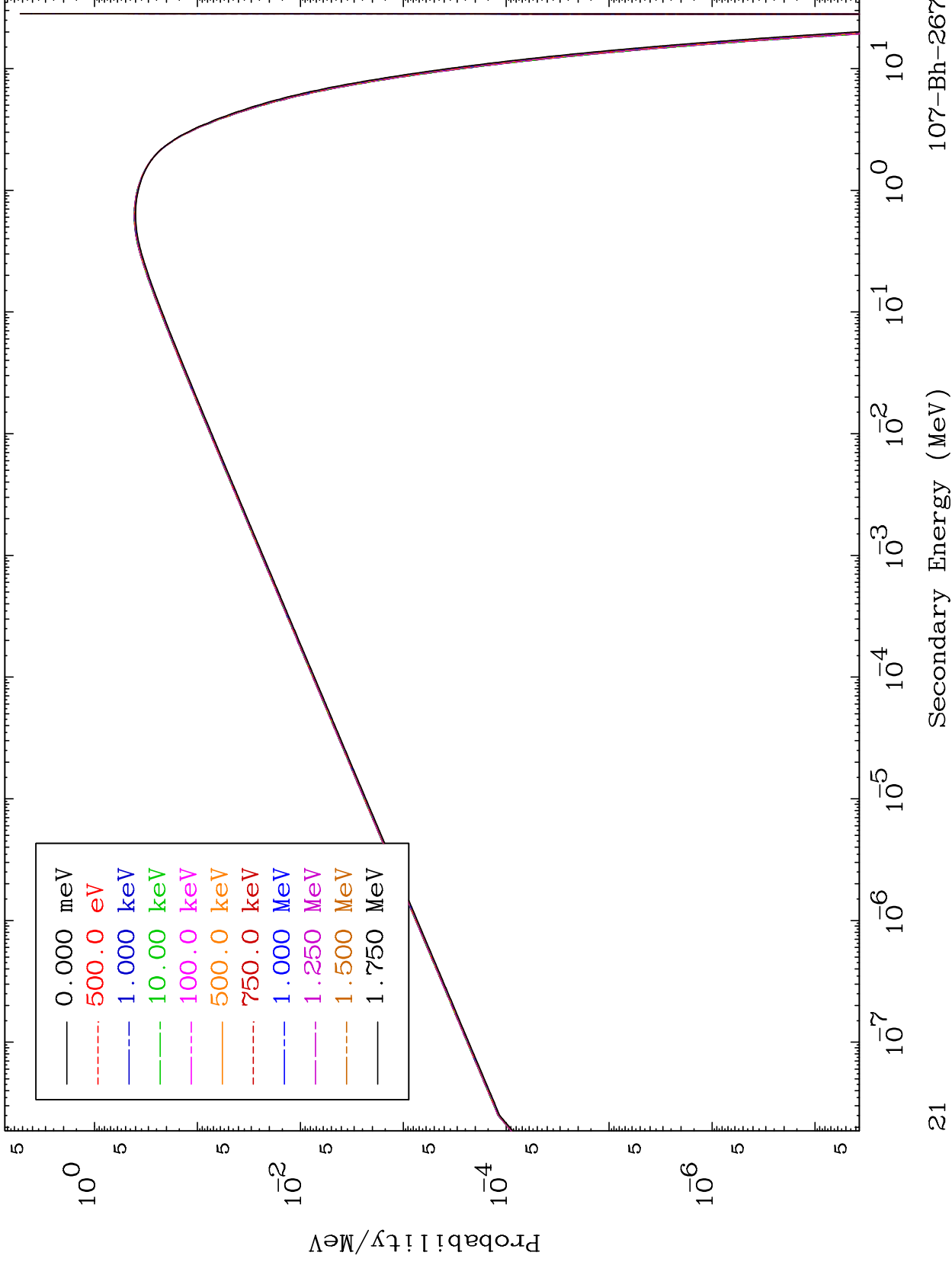
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Incident Energy (MeV)

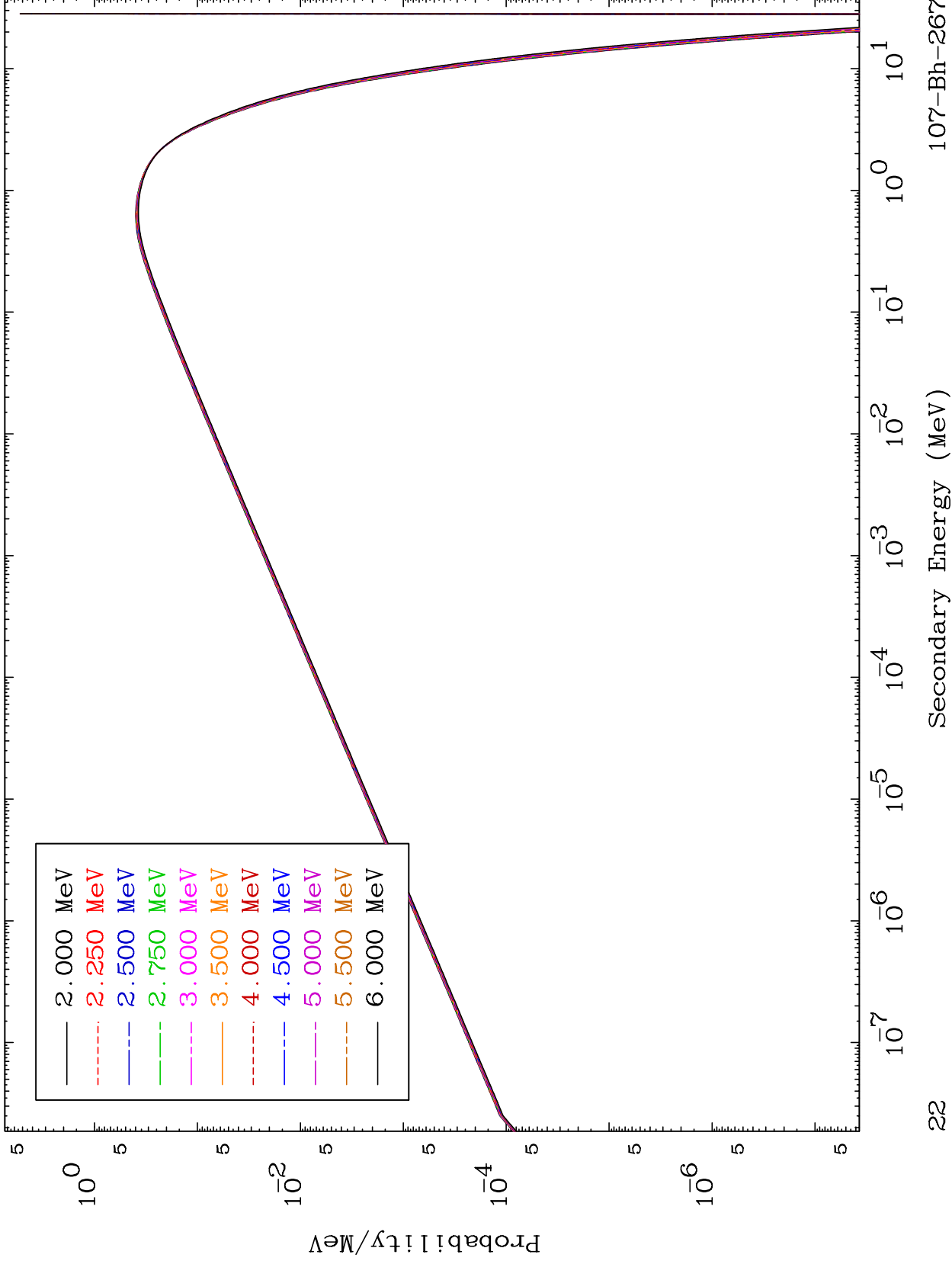
19



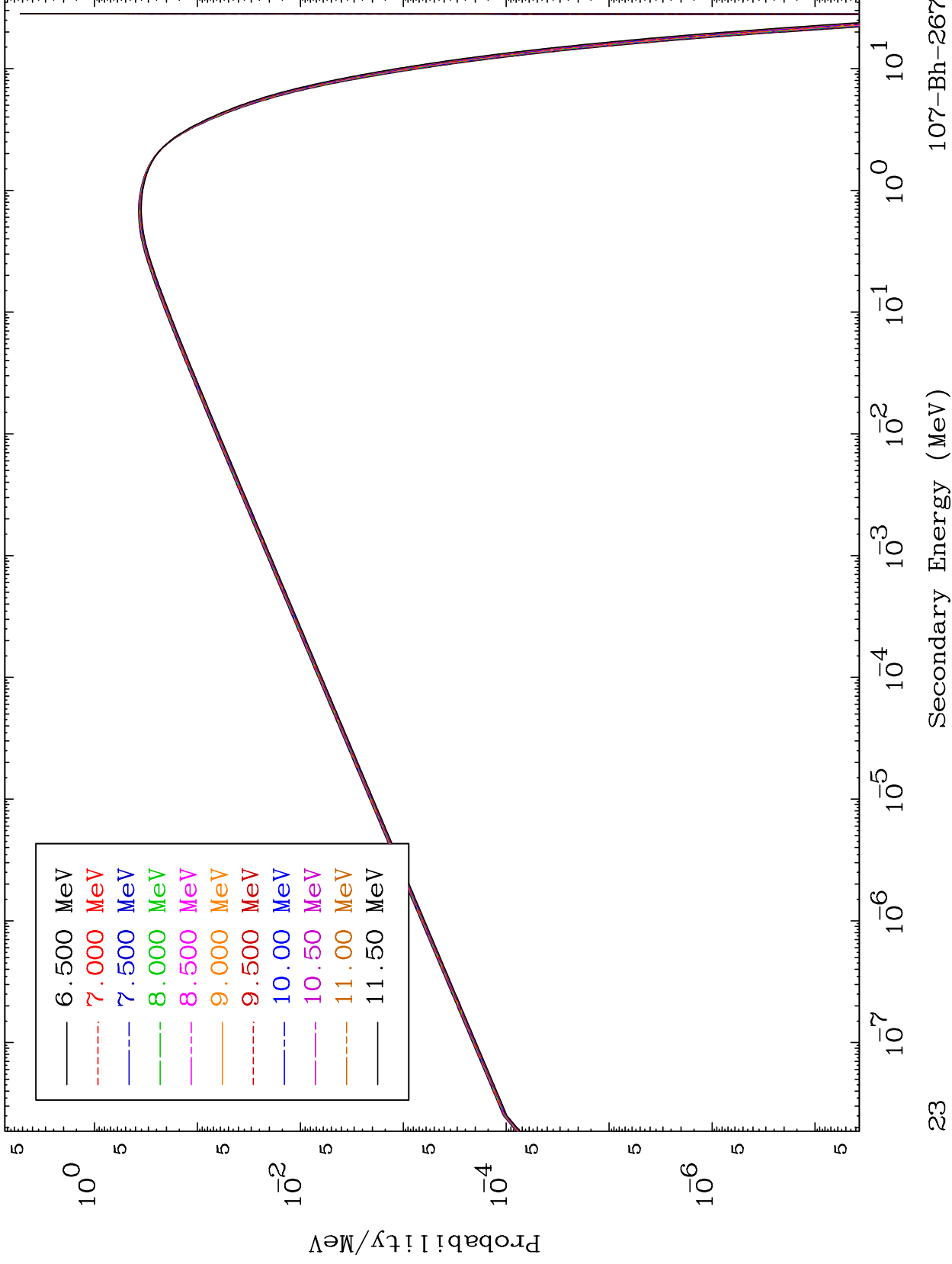
Fission Energy Distribution



Fission  
Energy Distribution

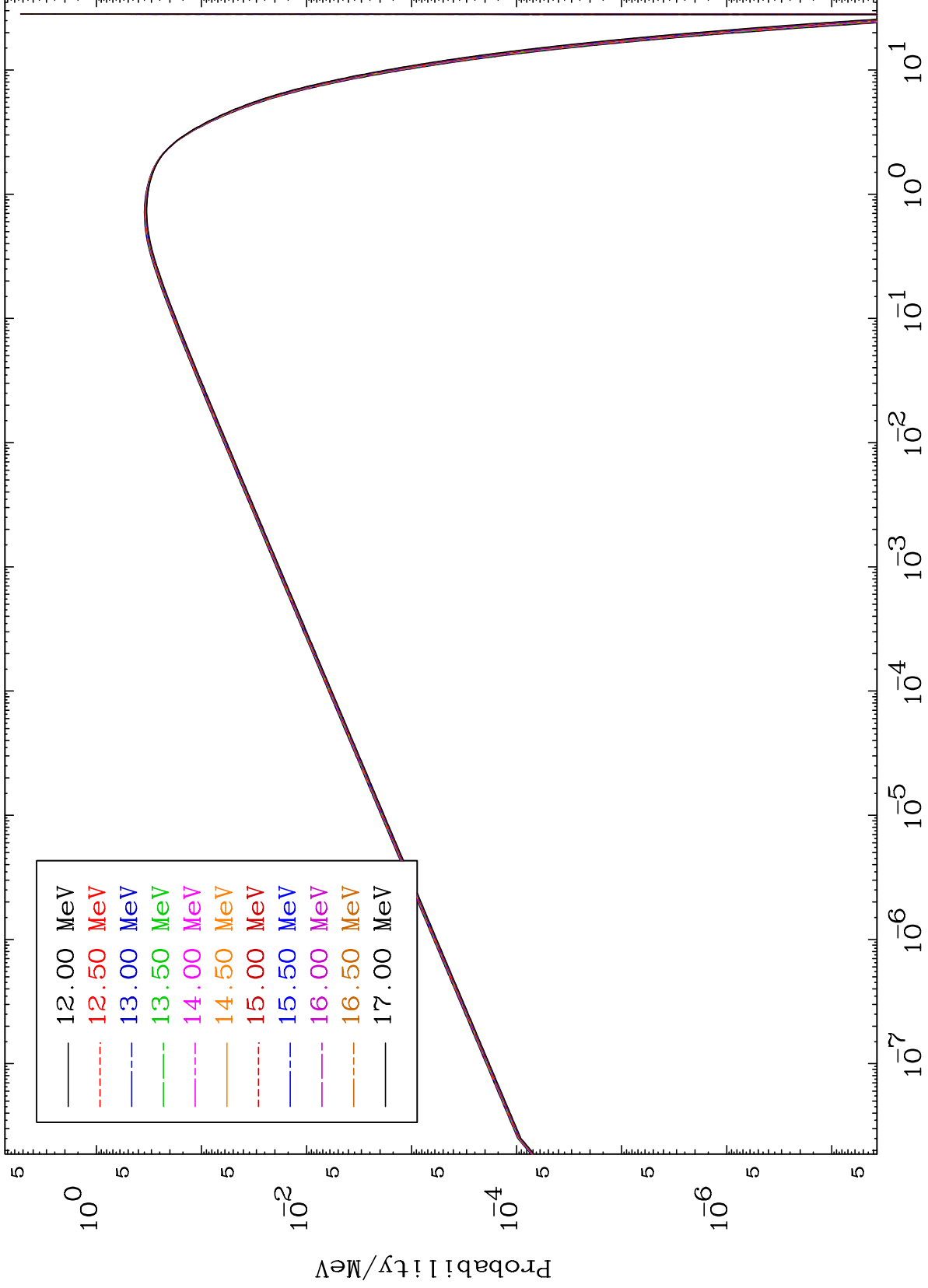


Fission  
Energy Distribution

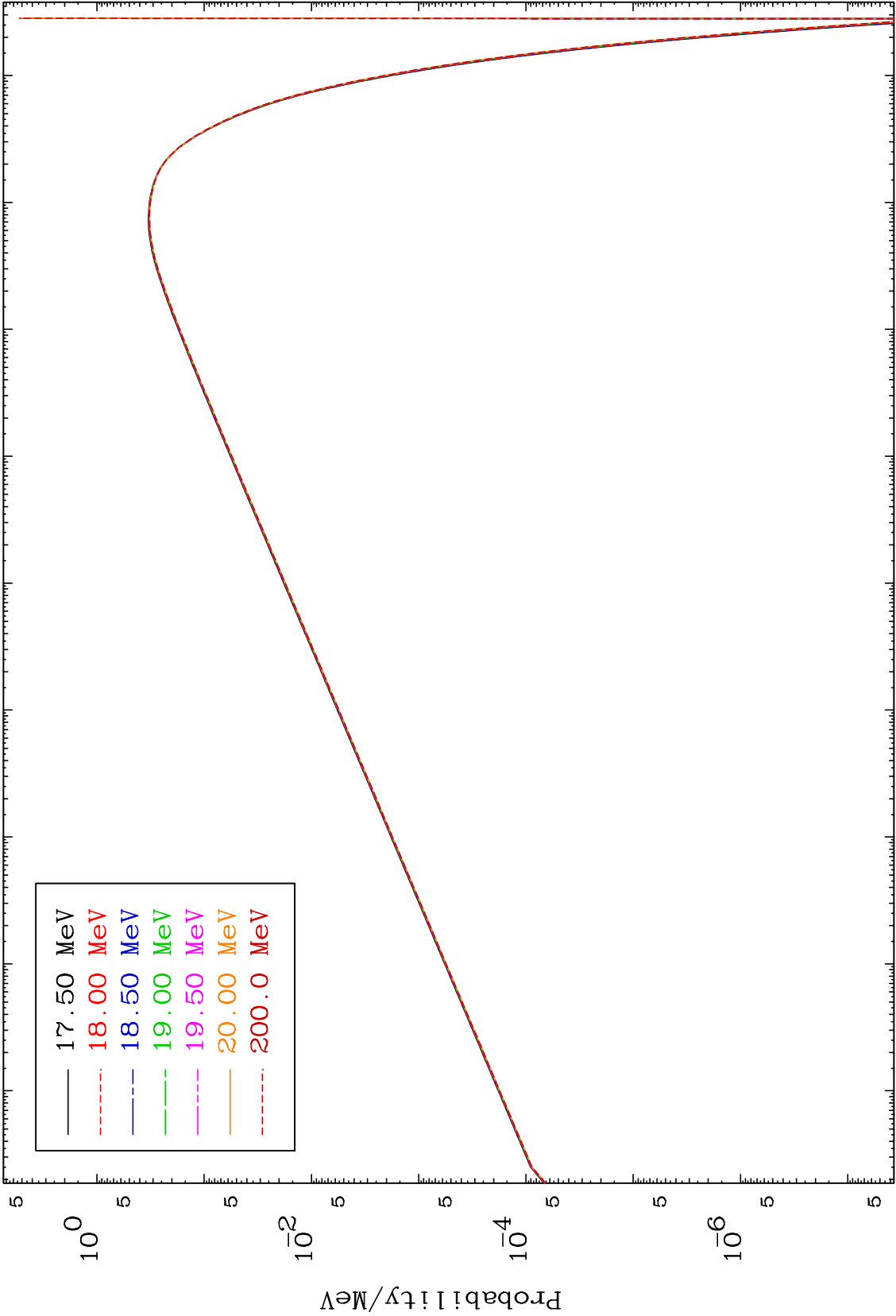




Fission  
Energy Distribution



Fission  
Energy Distribution

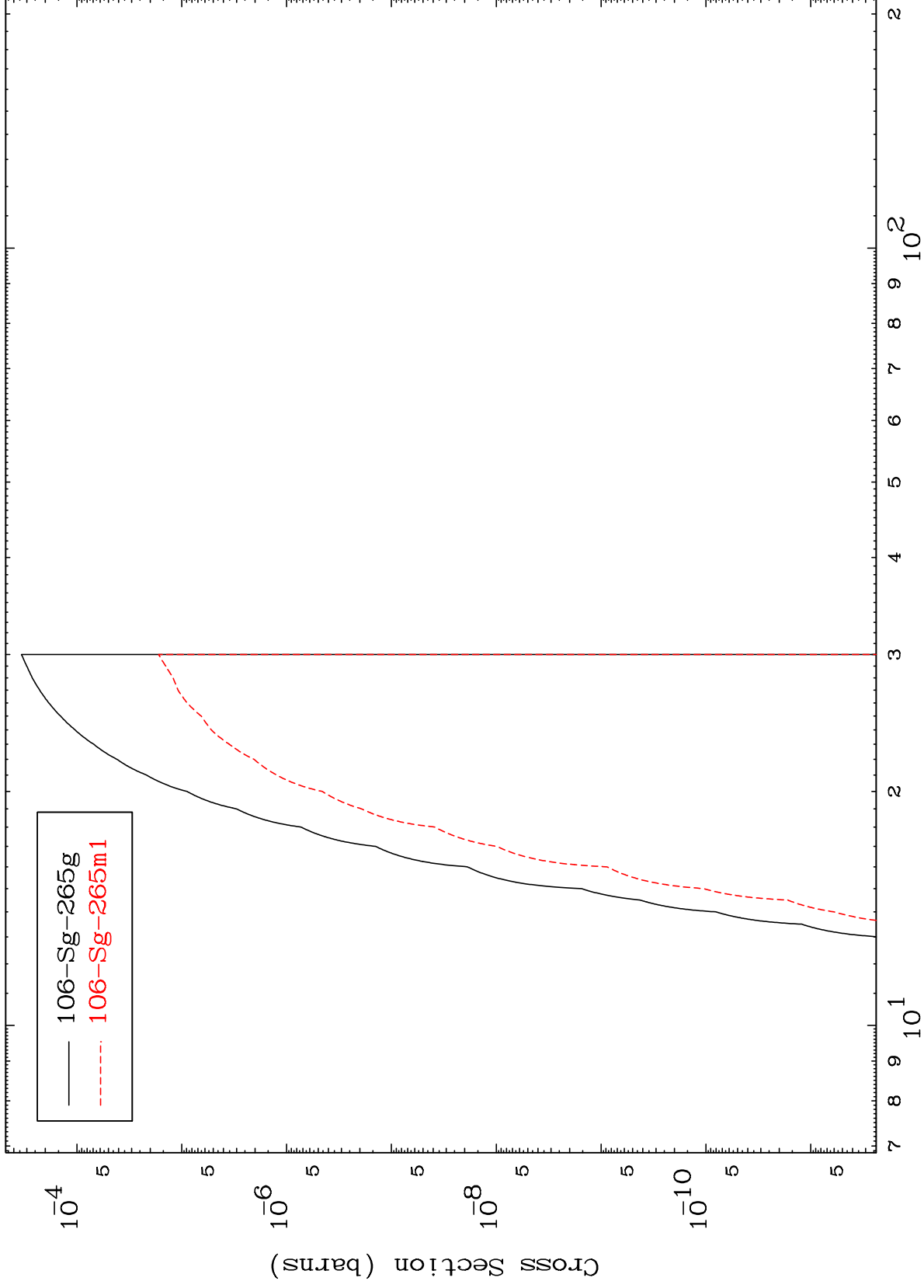


MAT 767

(n,n') d

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Radionuclide Production Cross Section



26

Incident Energy (MeV)

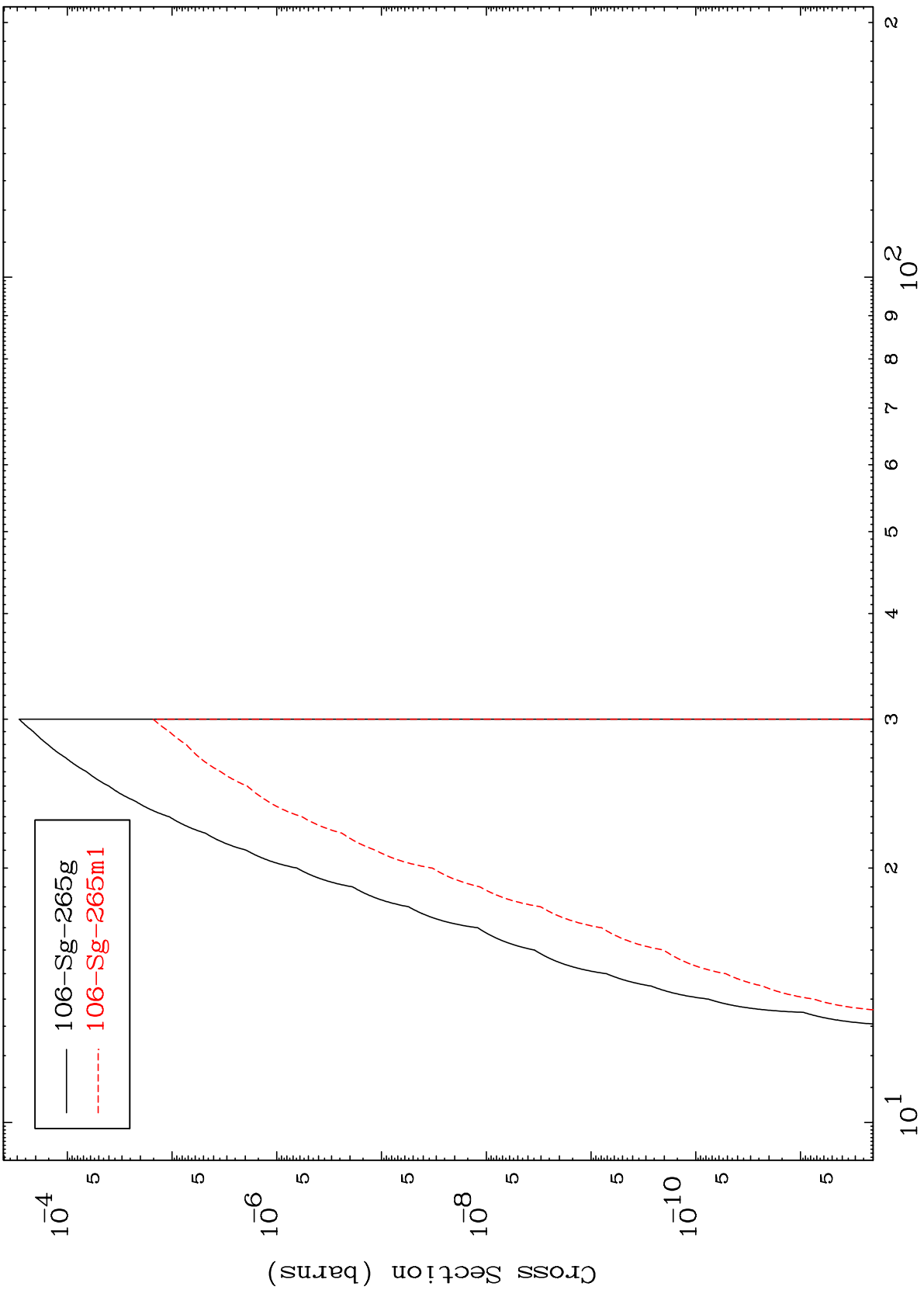
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(n,2n) p

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Radionuclide Production Cross Section



27

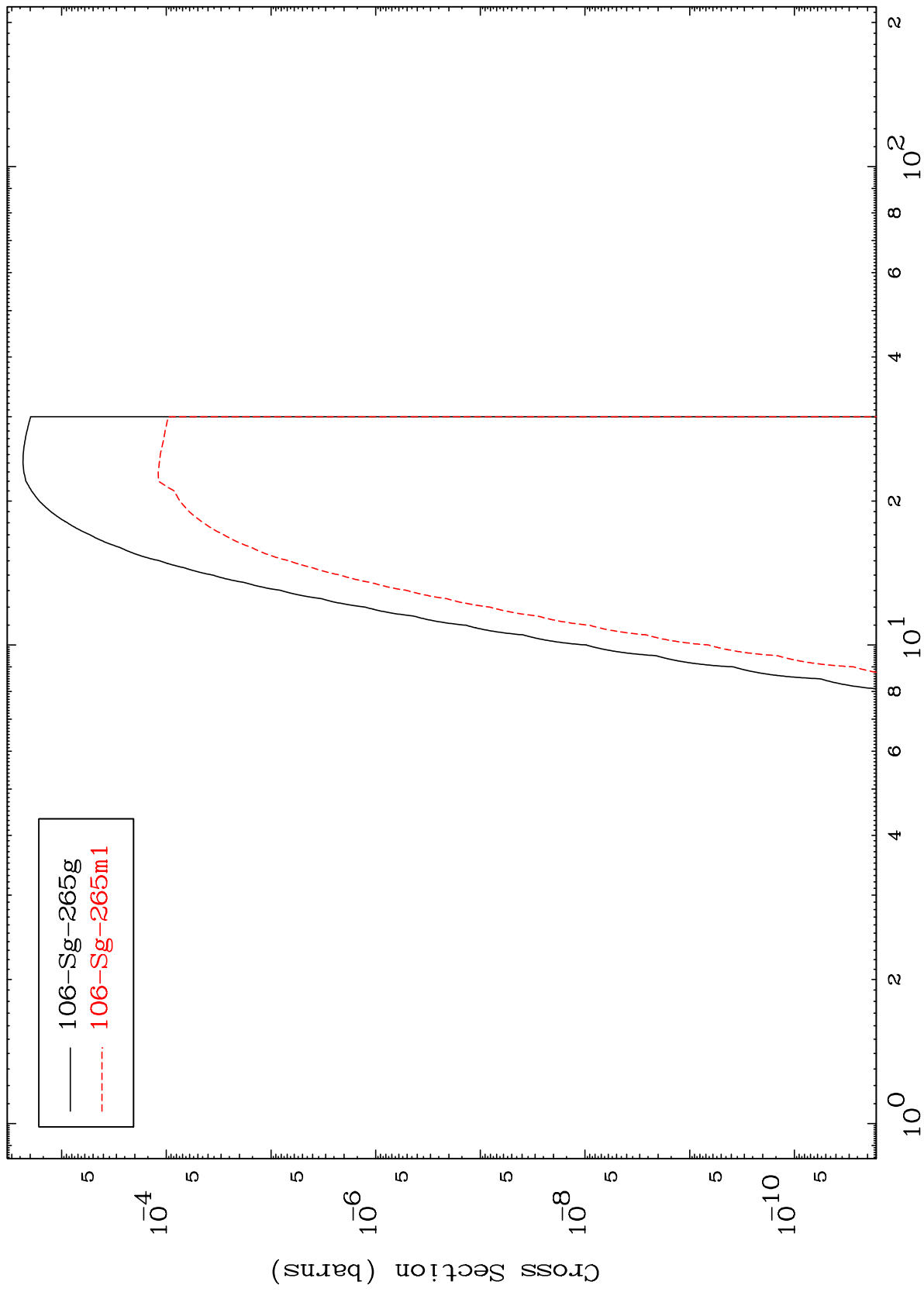
Incident Energy (MeV)

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107-Bh-267

Radionuclide Production Cross Section



Incident Energy (MeV)

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