

Program EVALPLOT
(Version 2015-2)

by

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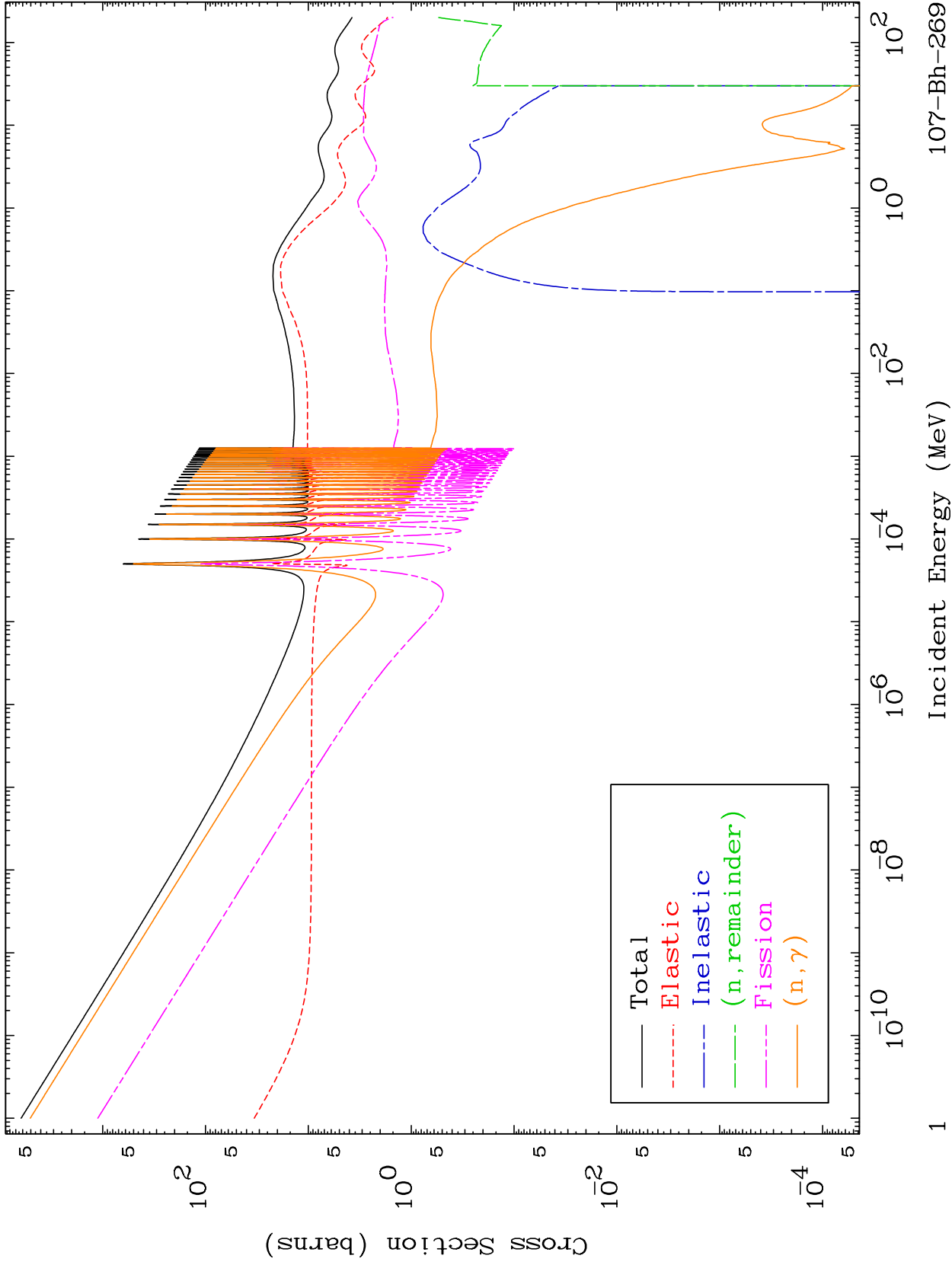
Press Mouse Button to Start

MAT 769

Major

293 Kelvin Cross Sections

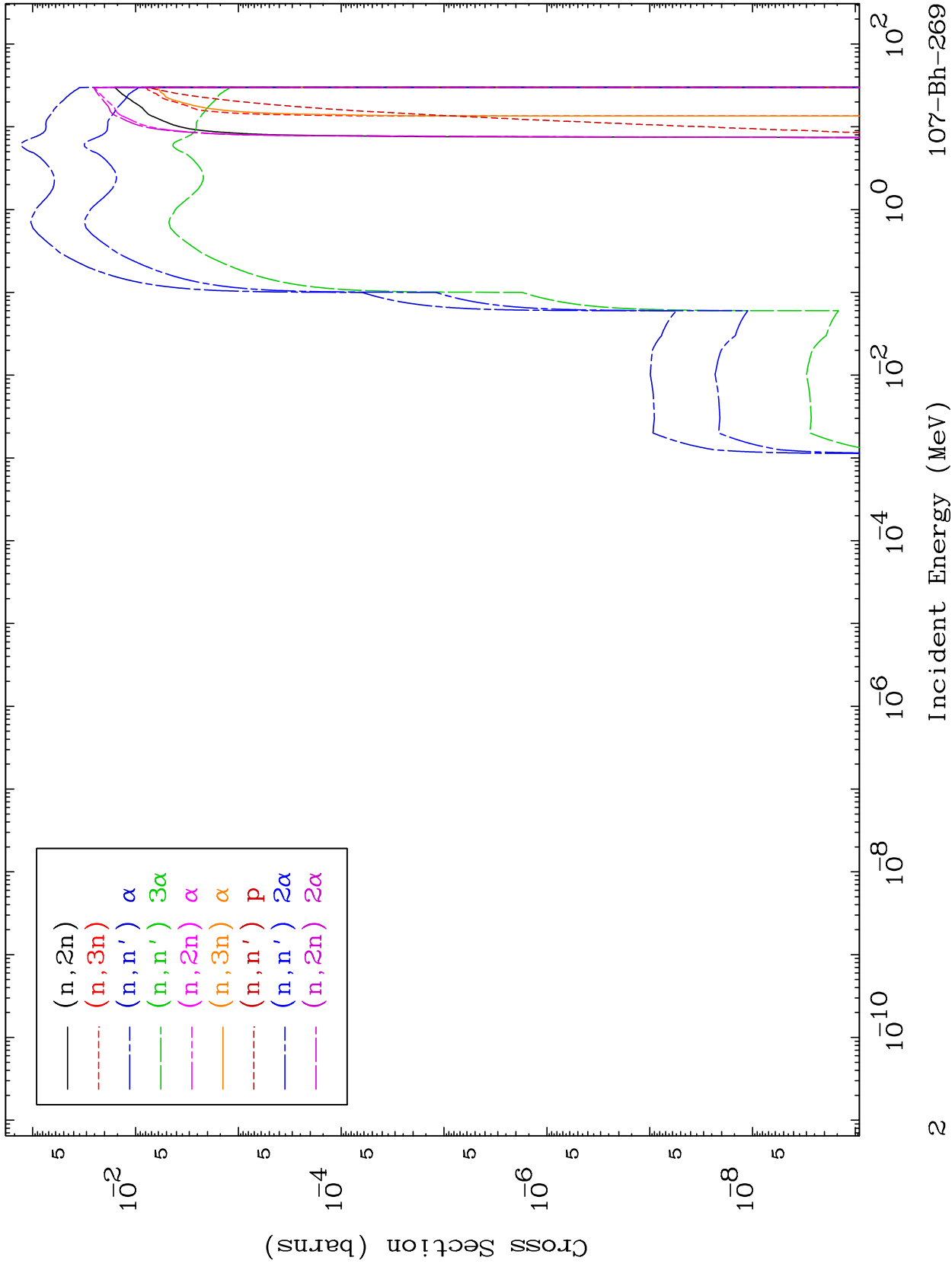
107-Bh-269



MAT 769

Neutron Production
293 Kelvin Cross Sections

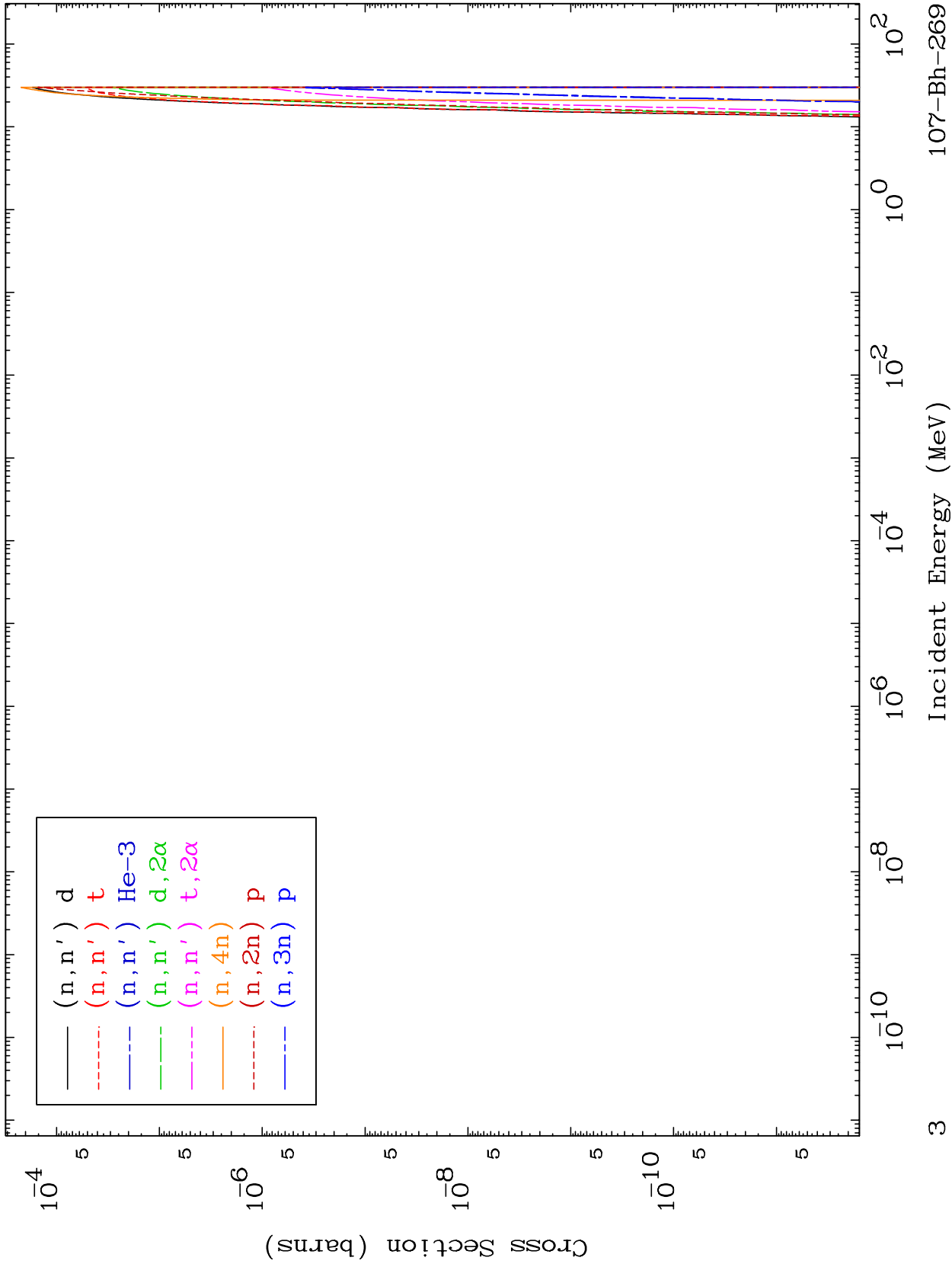
107-Bh-269



MAT 769

Neutron Production
293 Kelvin Cross Sections

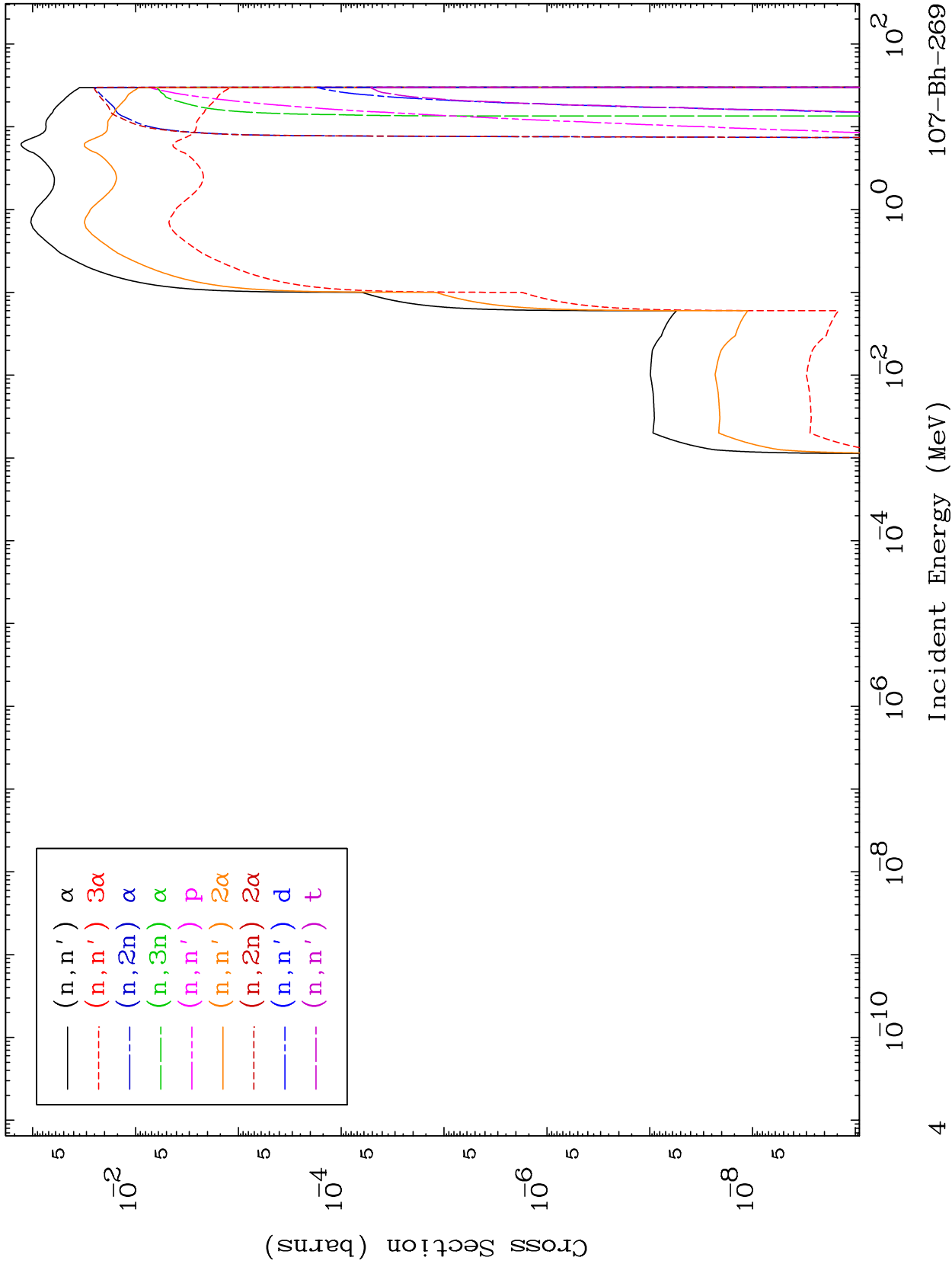
107-Bh-269



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Charged Particle
293 Kelvin Cross Sections

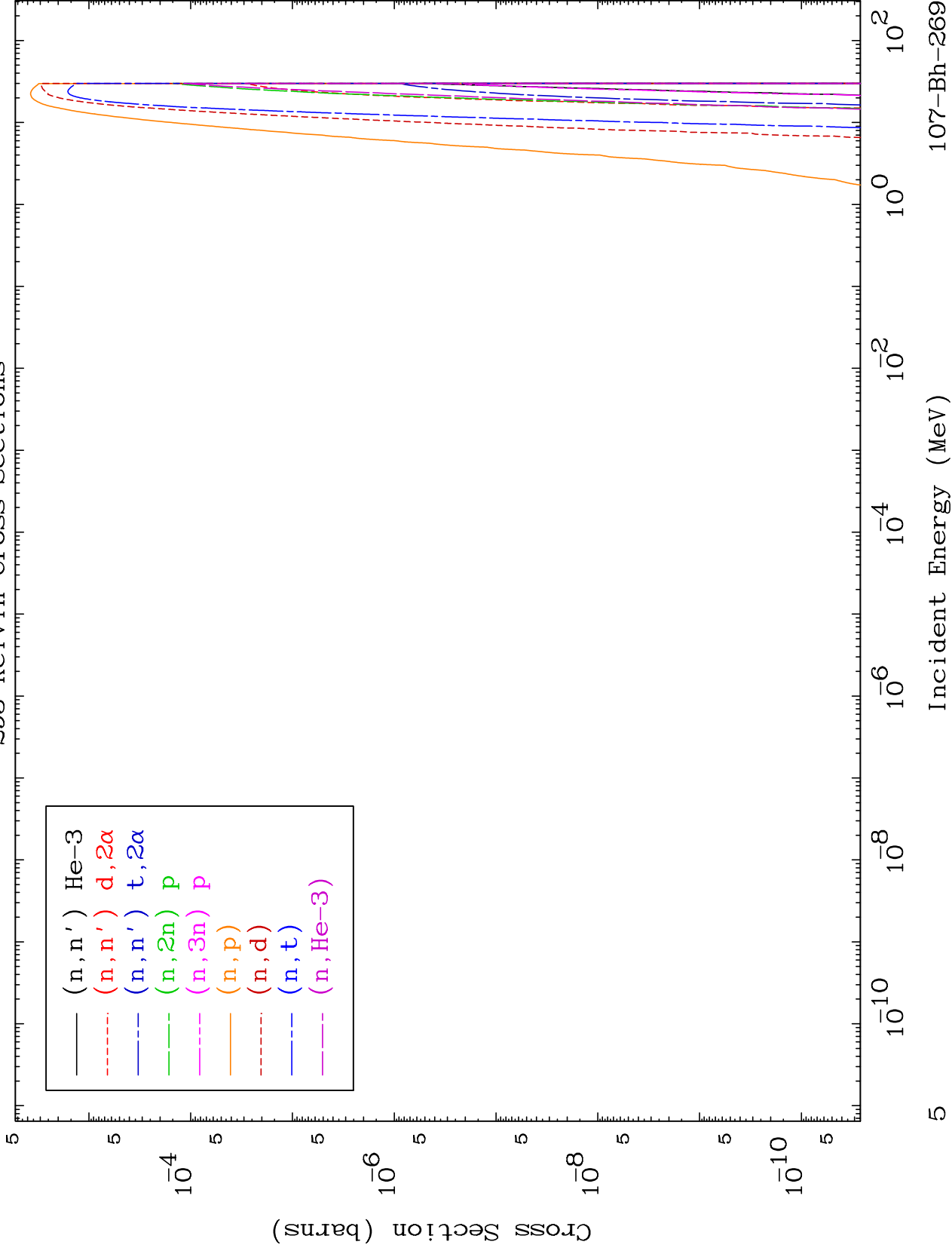
107-Bh-269

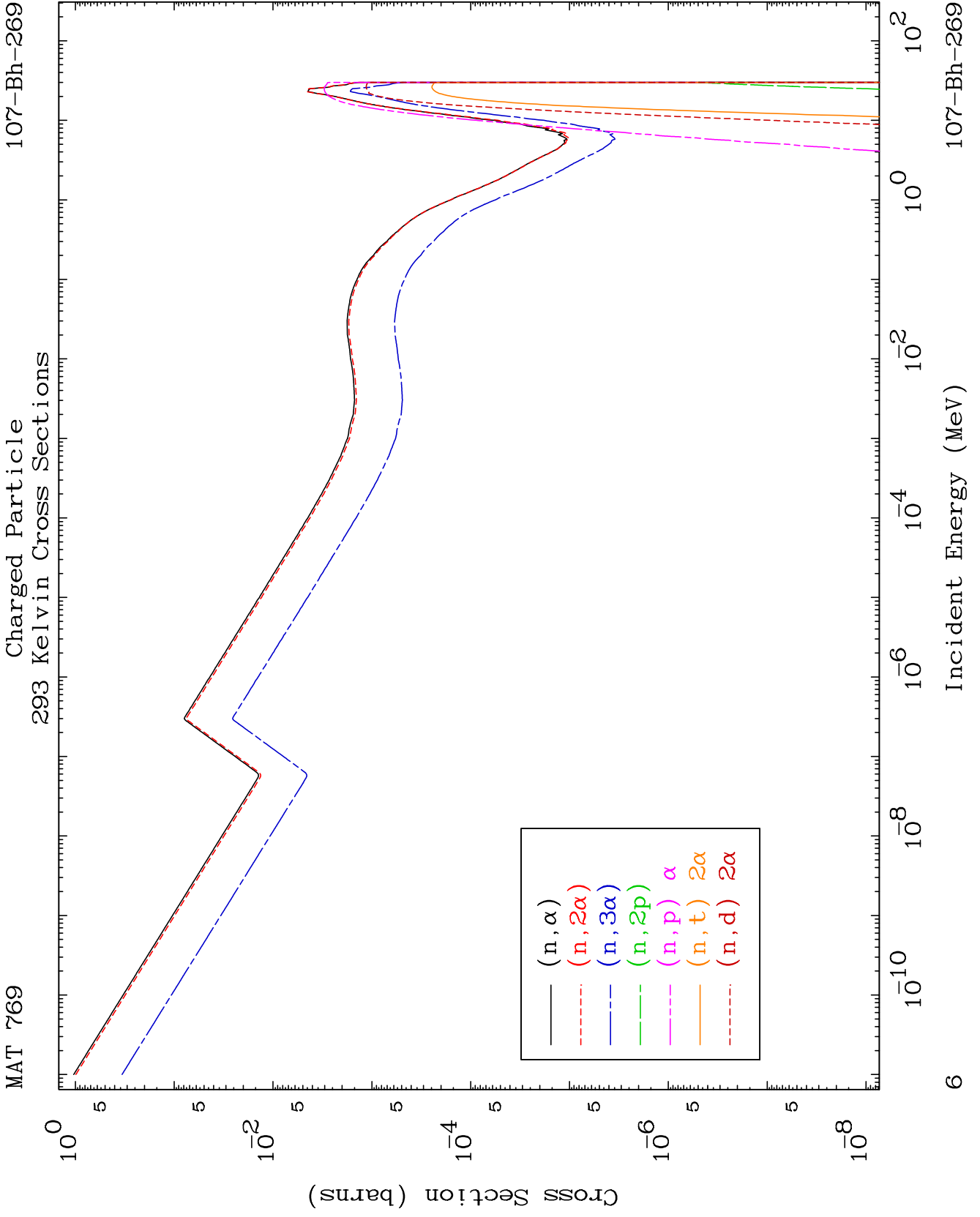


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Charged Particle
293 Kelvin Cross Sections

107-Bh-269

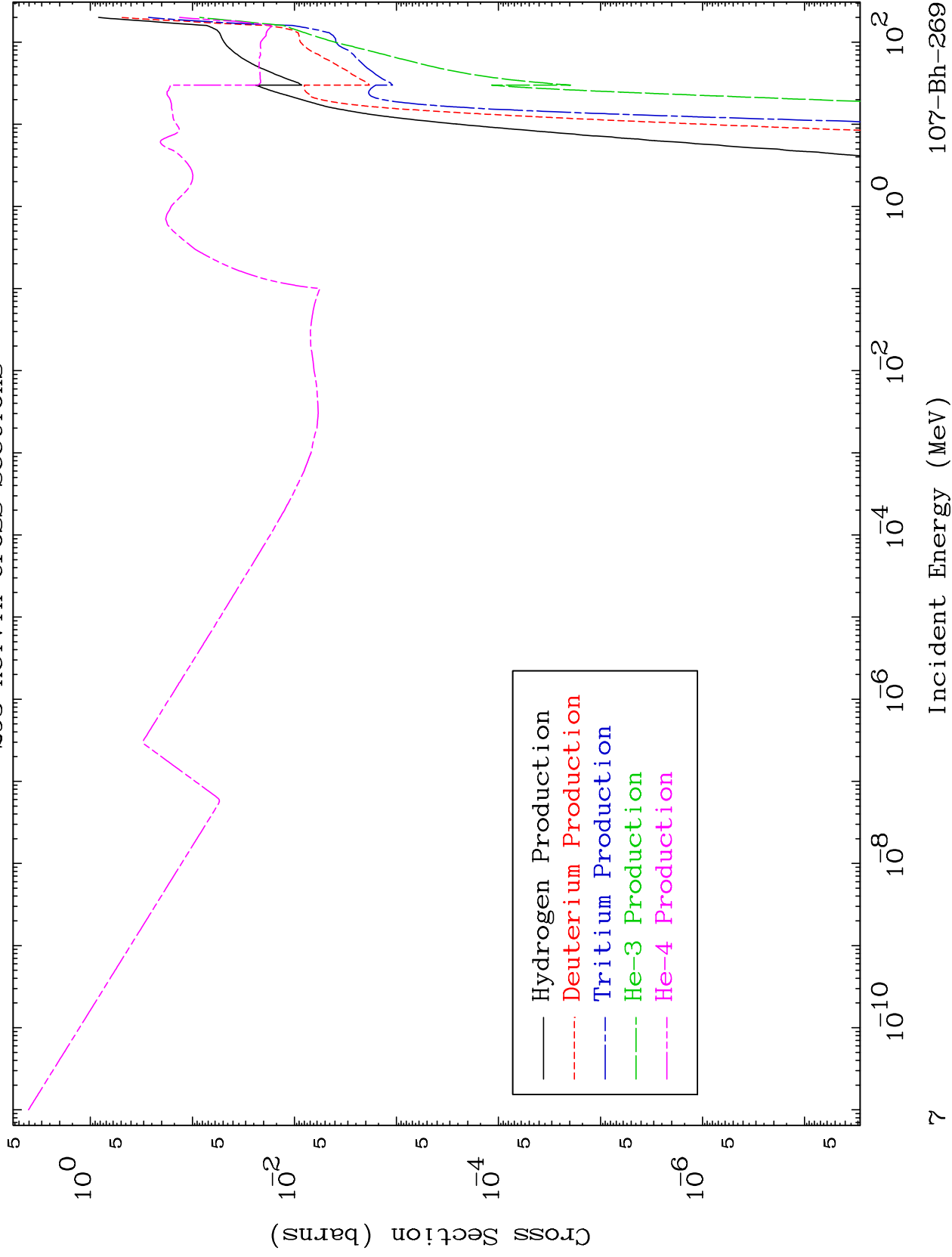




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Particle Production
293 Kelvin Cross Sections

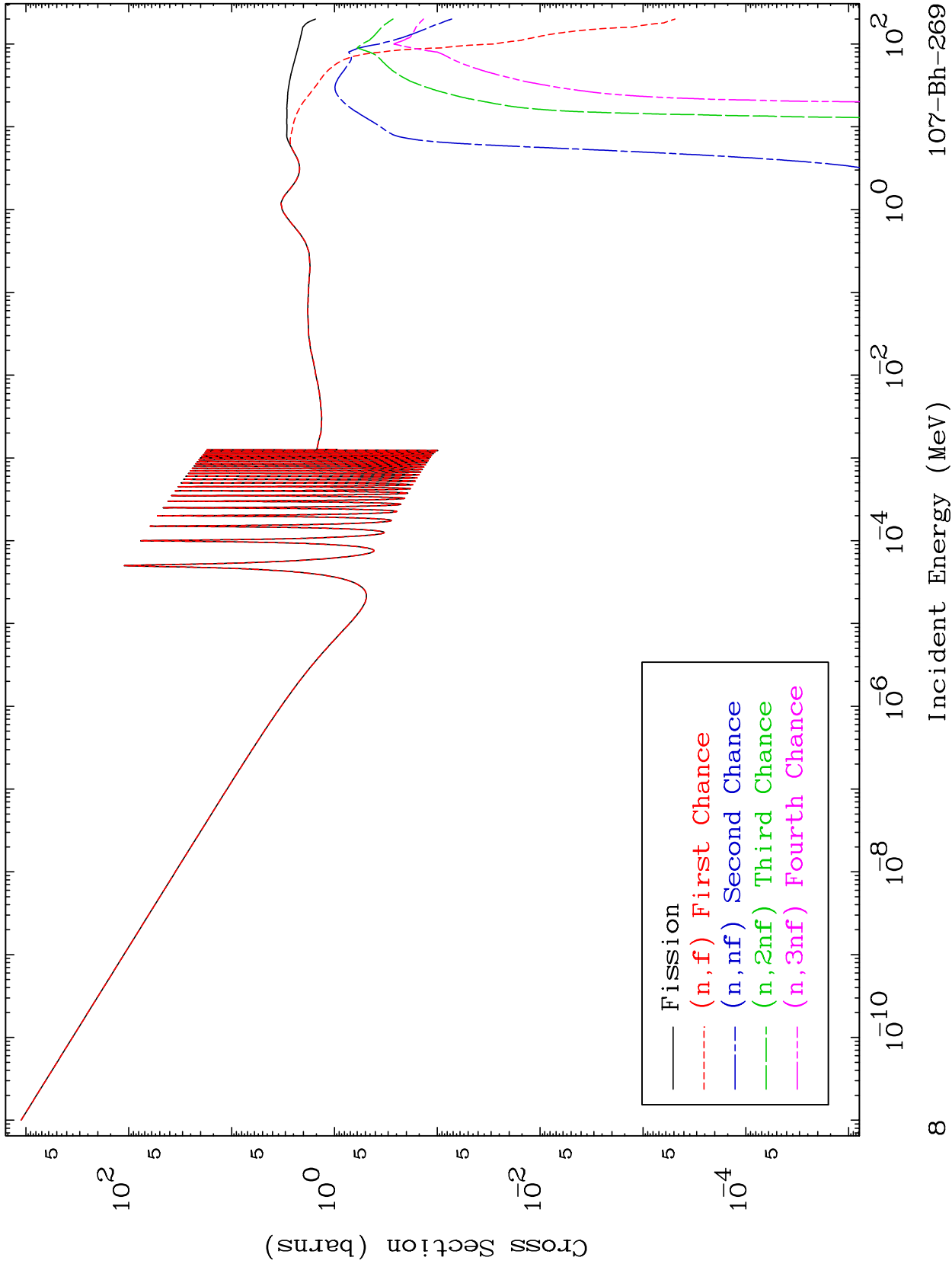
107-Bh-269



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Fission
293 Kelvin Cross Sections

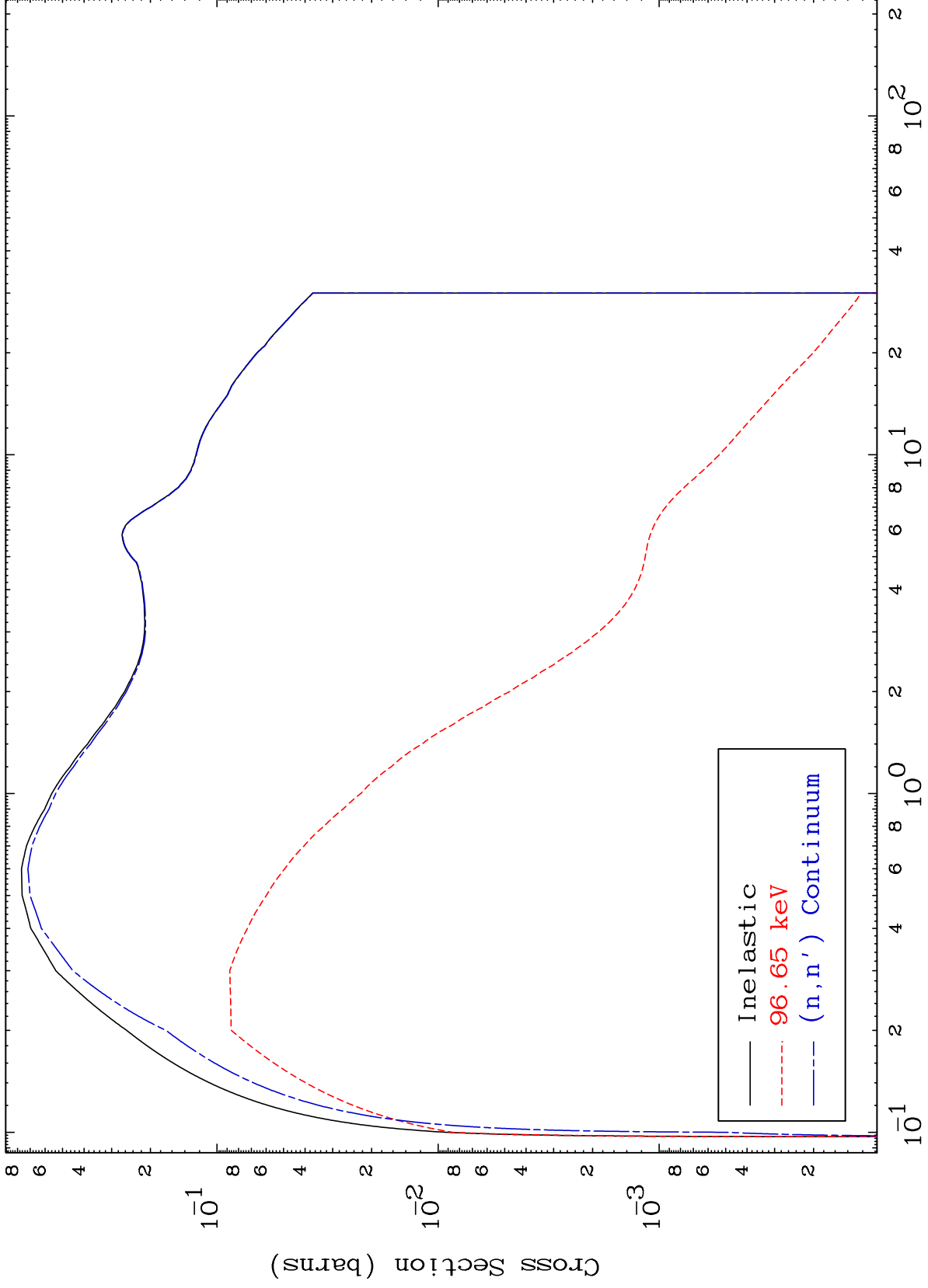
107-Bh-269



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(n,n') Level
293 Kelvin Cross Sections

107-Bh-269



Incident Energy (MeV)

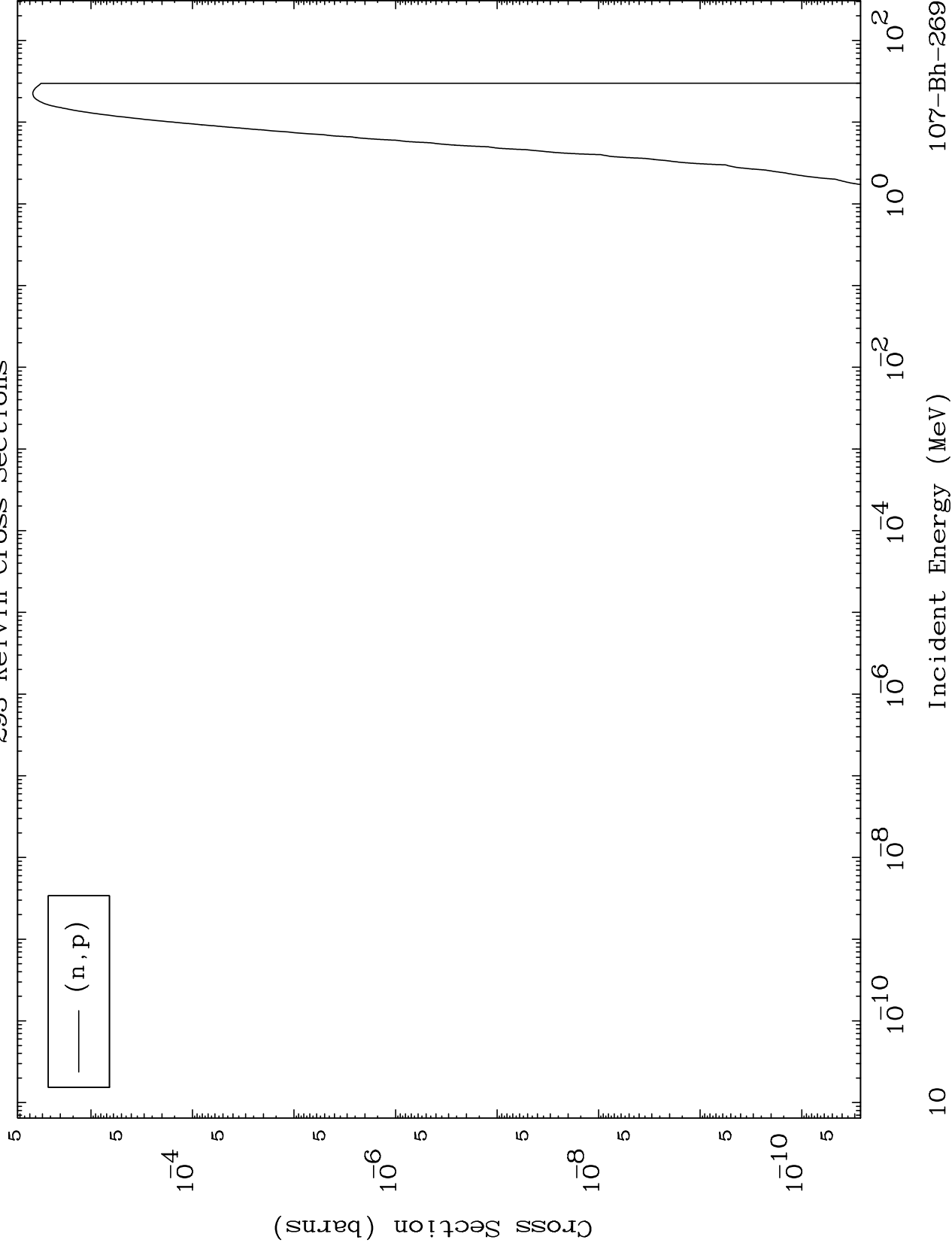
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9

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(n,p) Levels
293 Kelvin Cross Sections

107-Bh-269



(n,p)

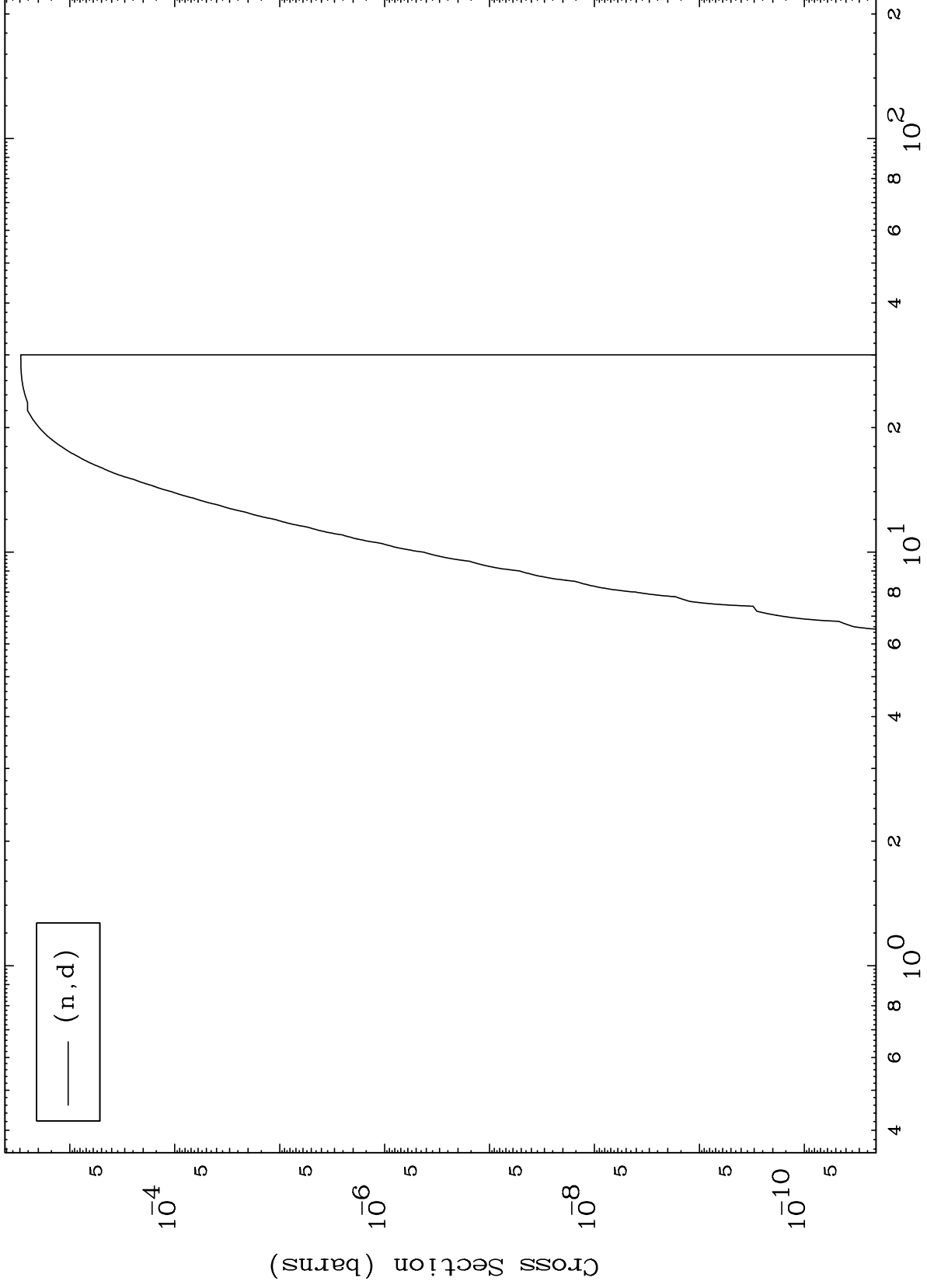
10

107-Bh-269

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(n,d) Levels
293 Kelvin Cross Sections

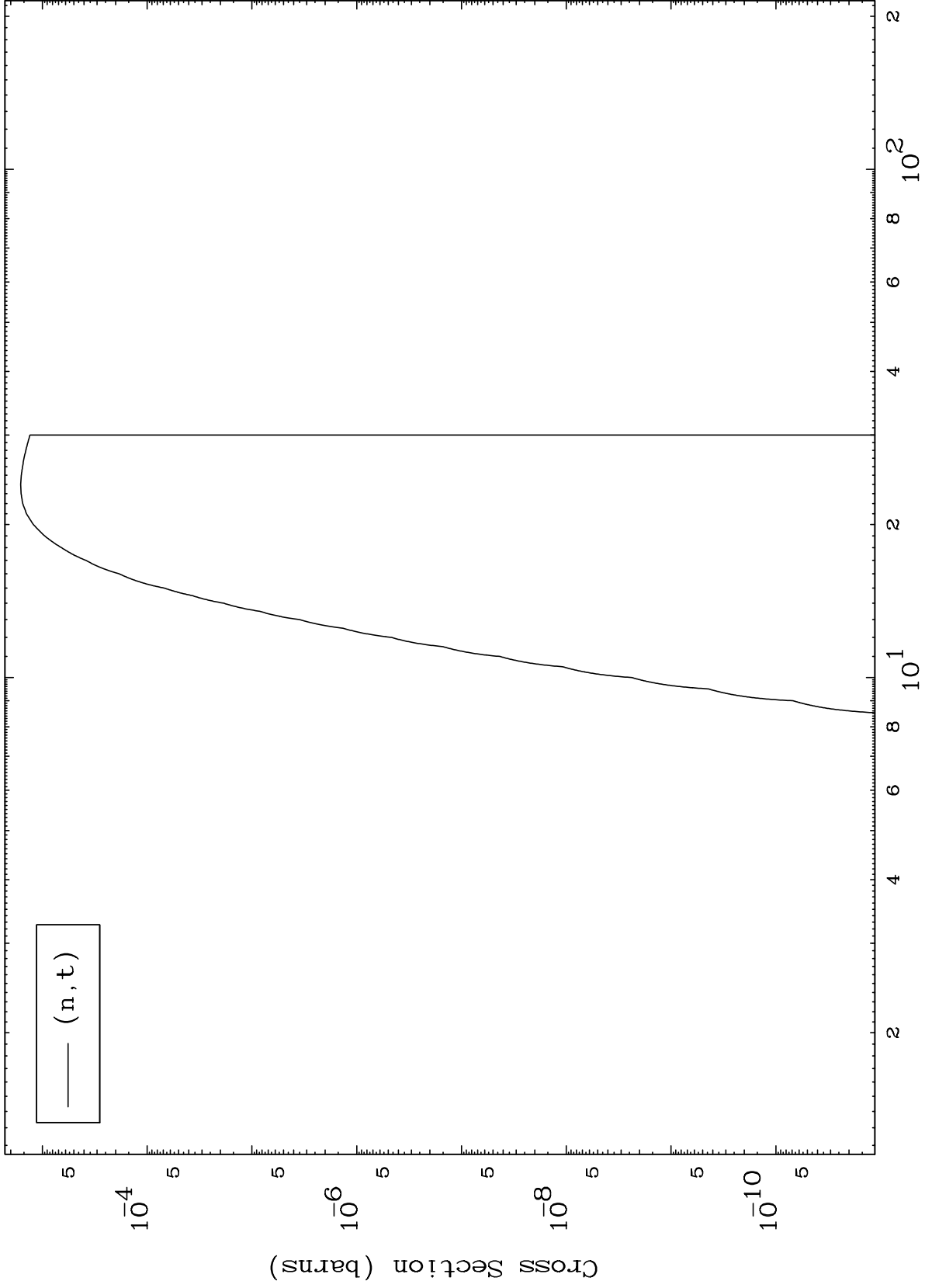
107-Bh-269



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(n,t) Levels
293 Kelvin Cross Sections

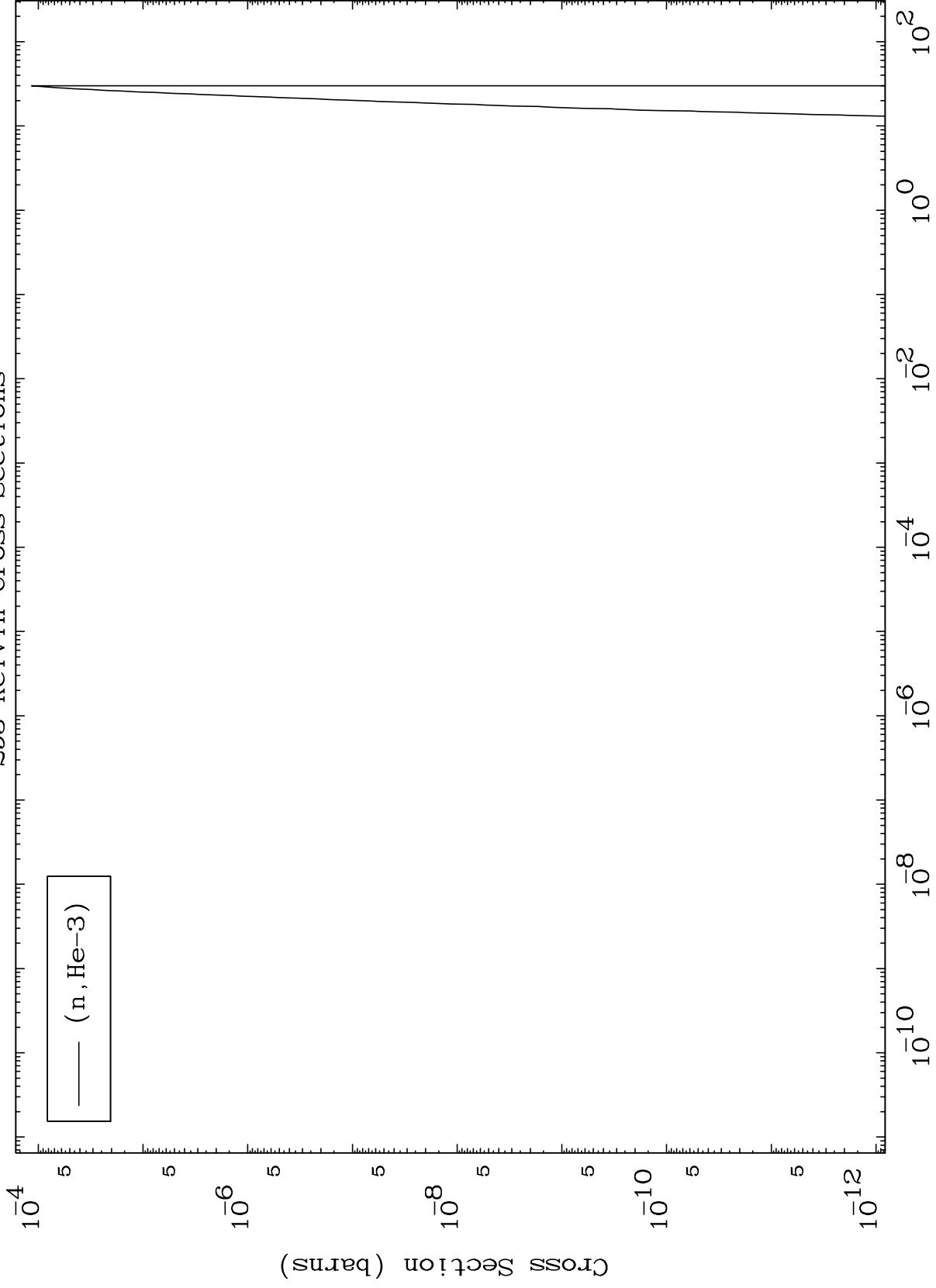
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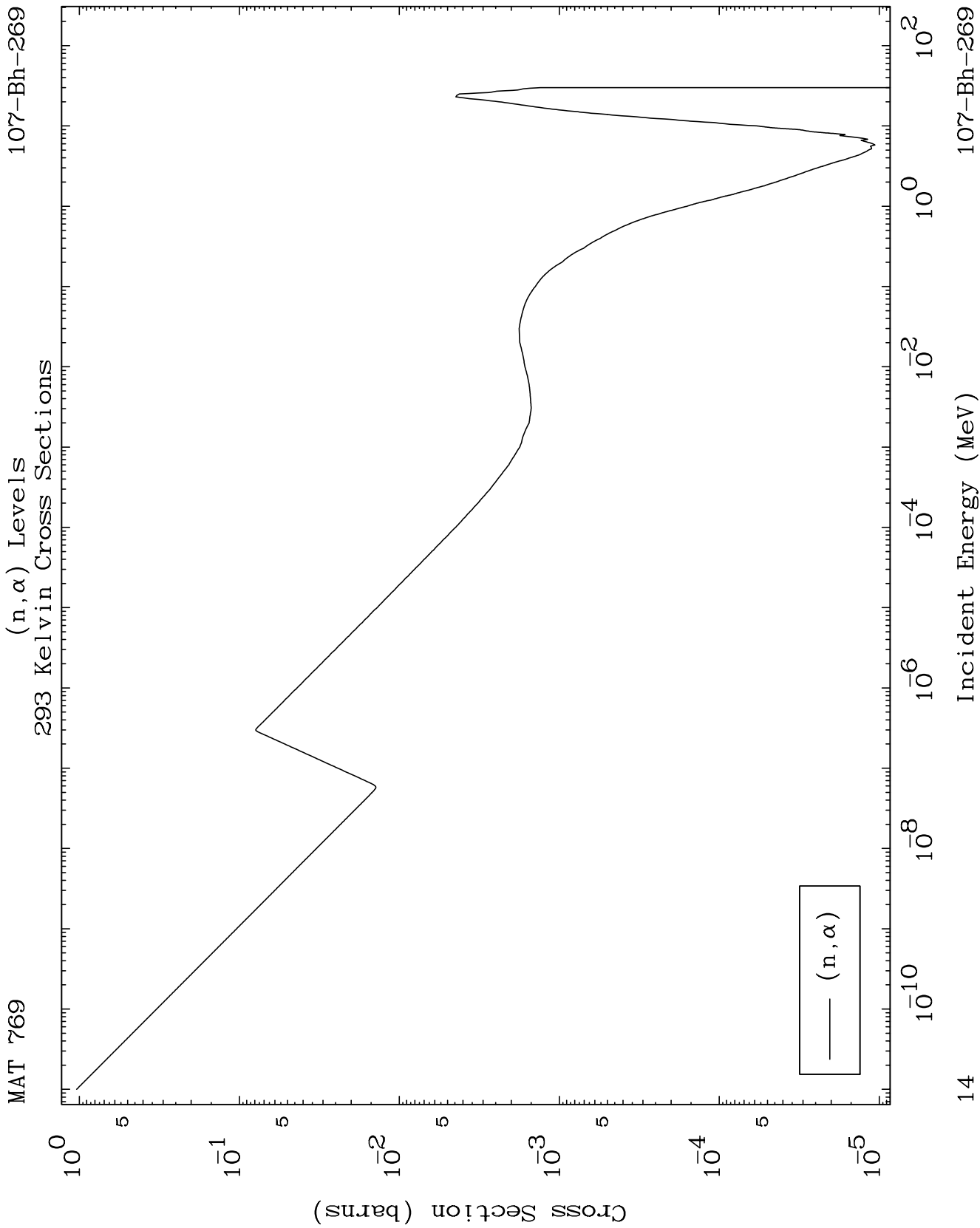


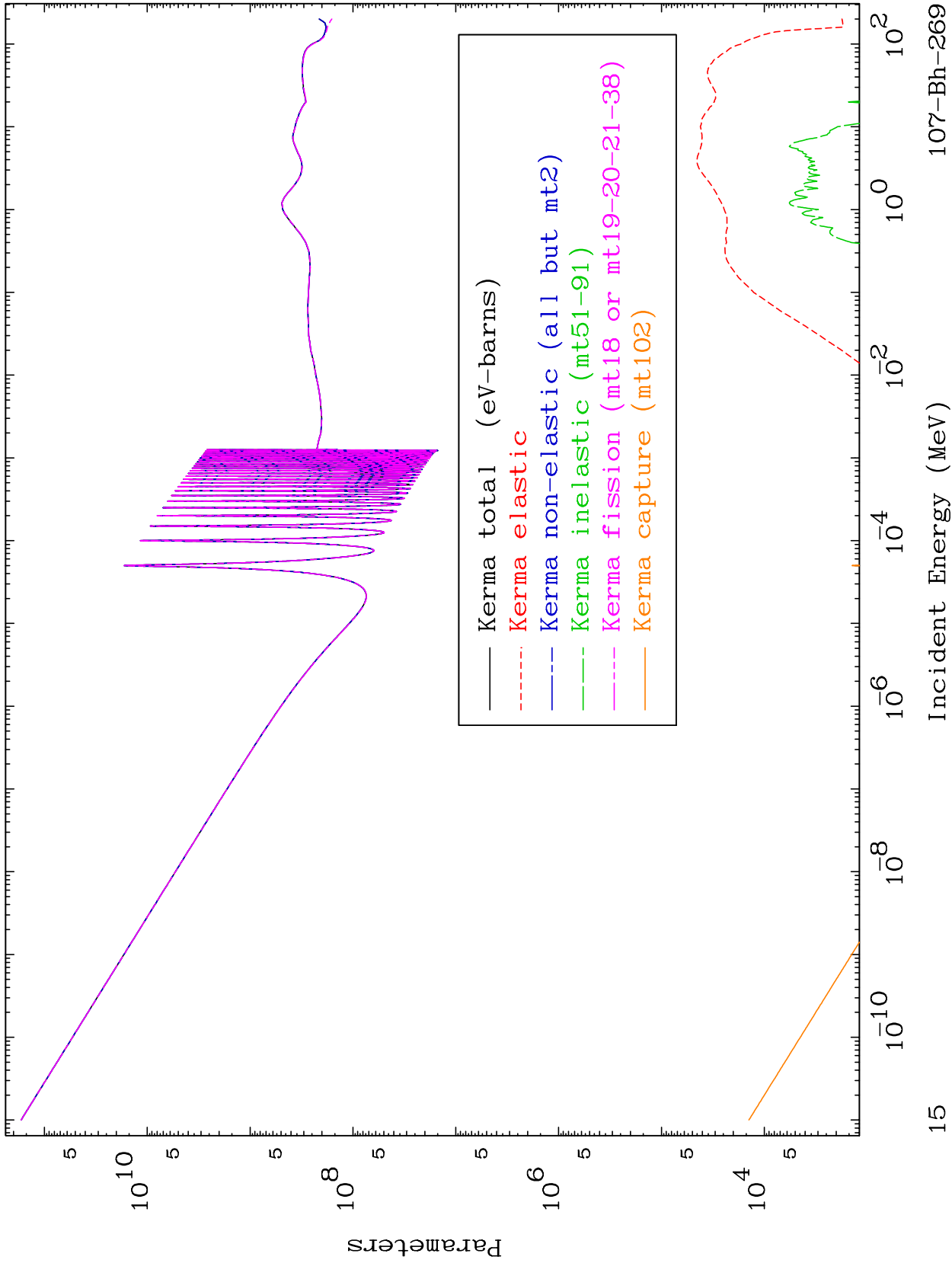
MAT 769

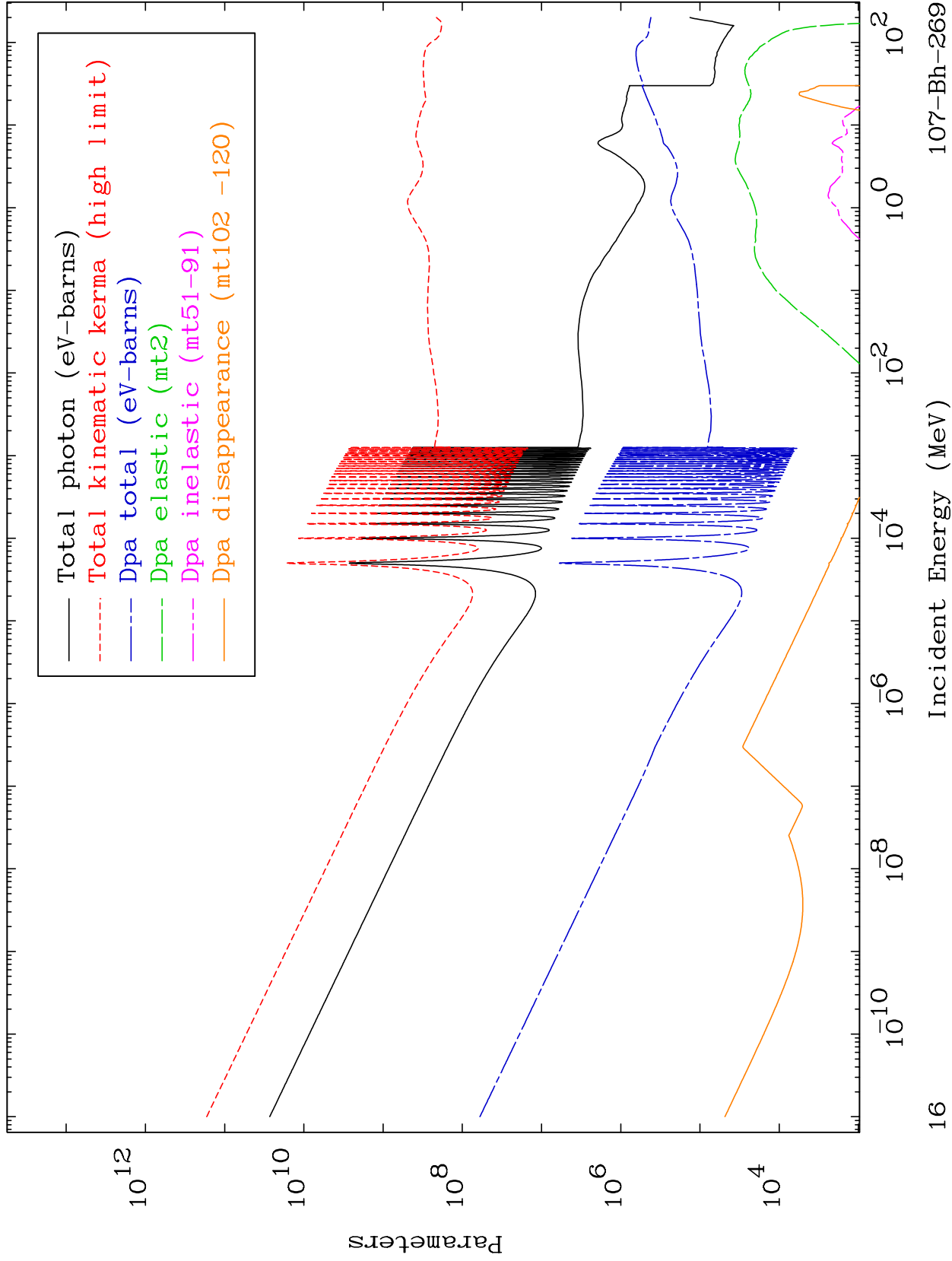
(n,He3) Levels
293 Kelvin Cross Sections

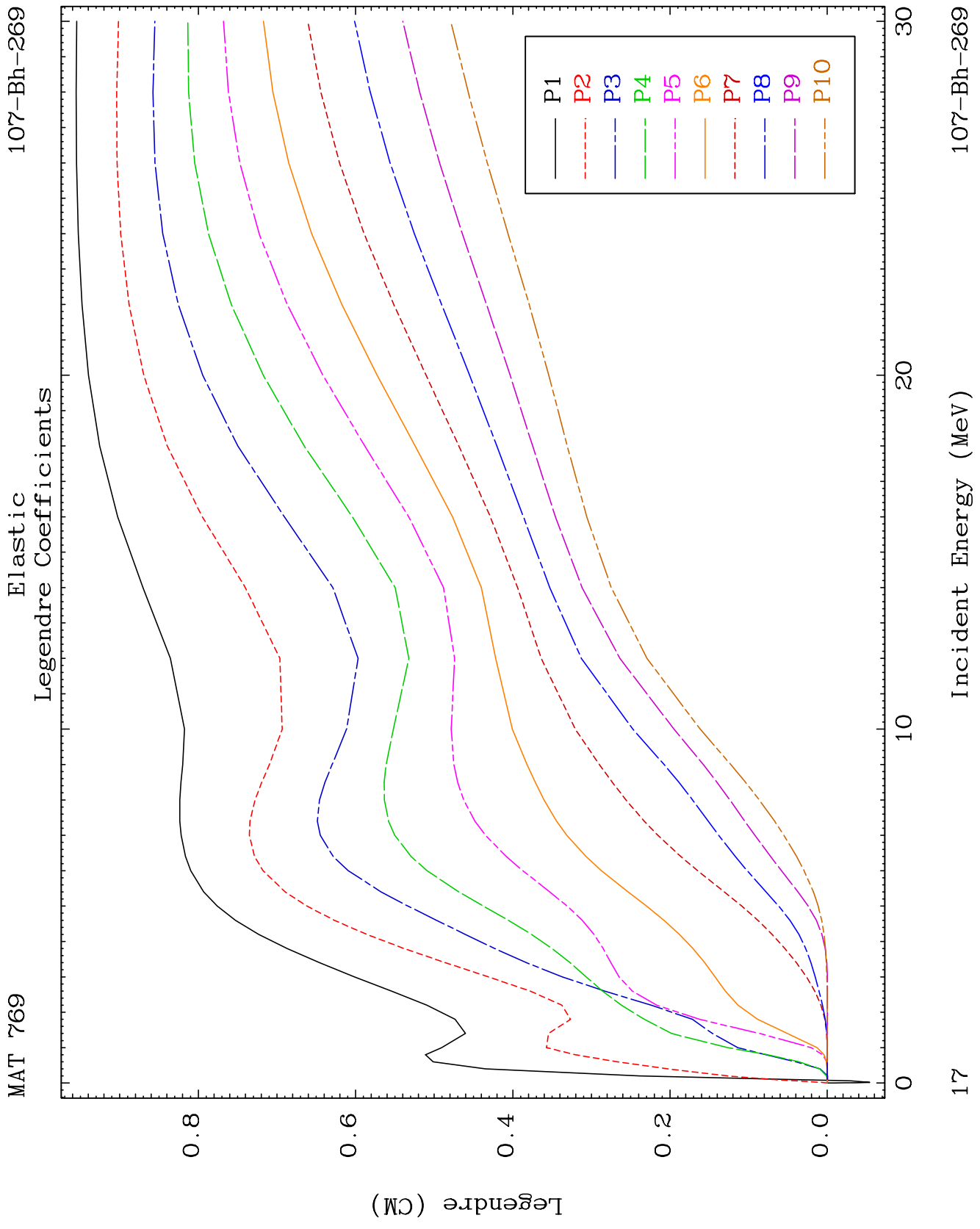
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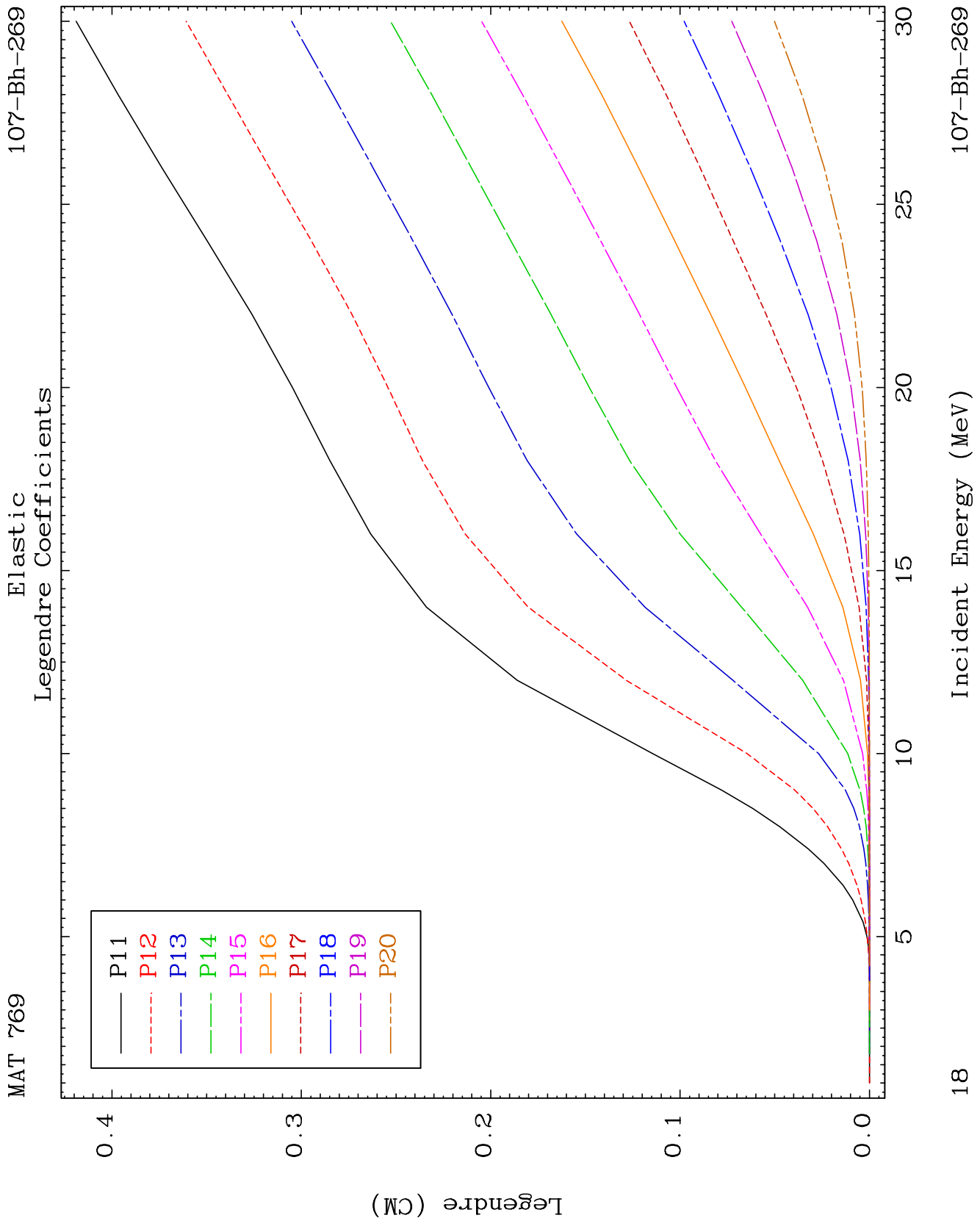




107-Bh-269

Incident Energy (MeV)

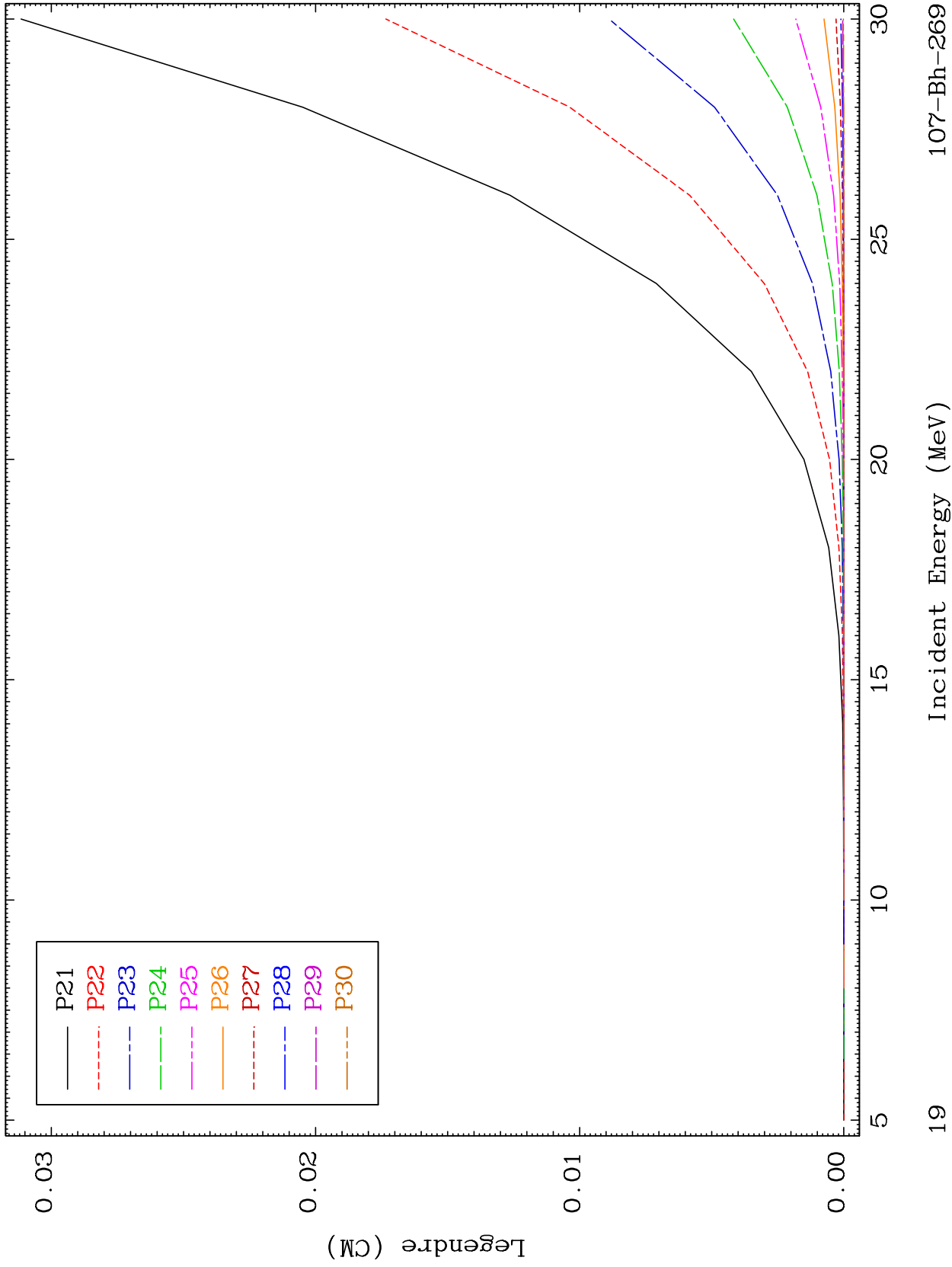
17



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Elastic Legendre Coefficients

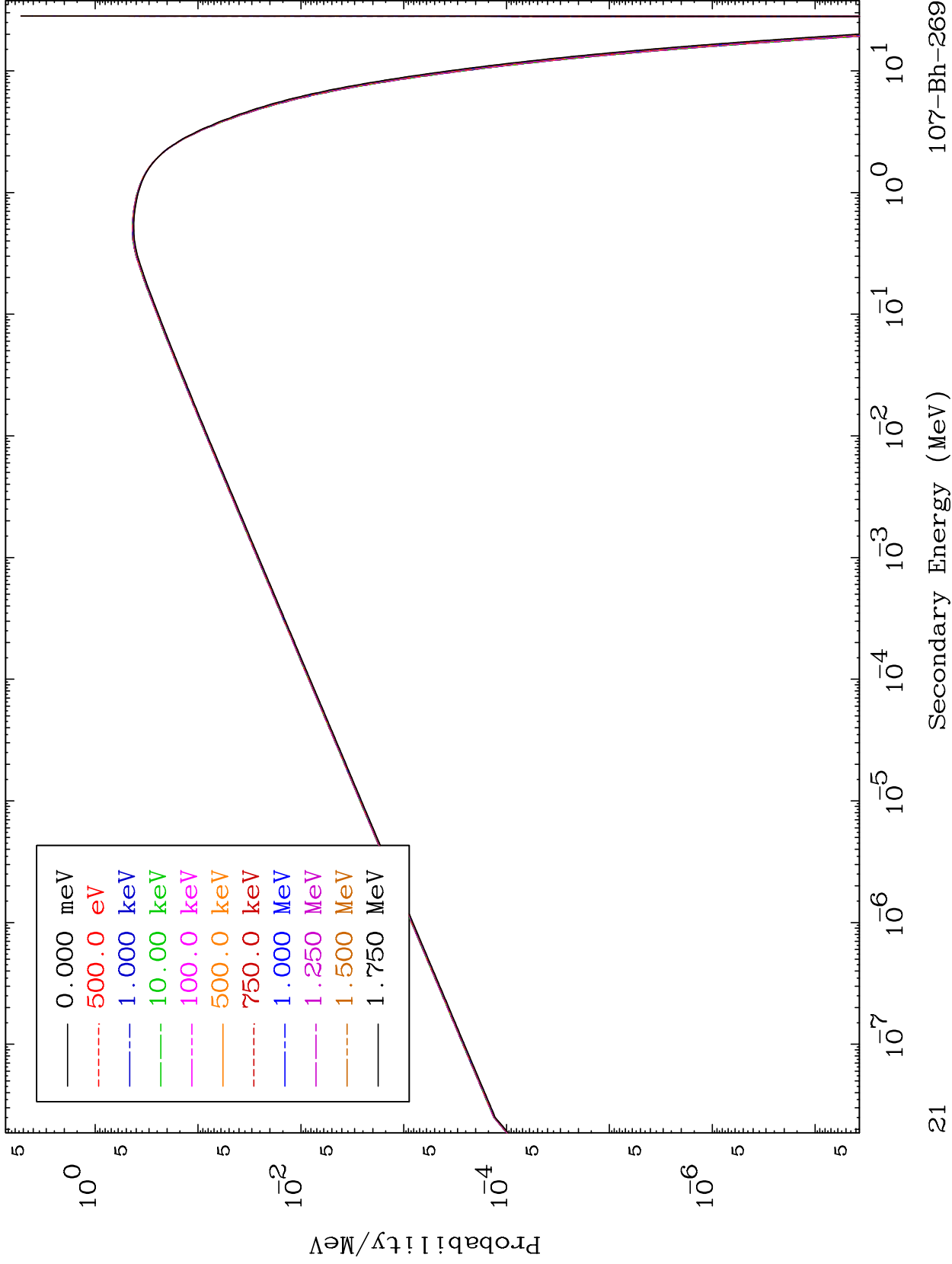
107-Bh-269



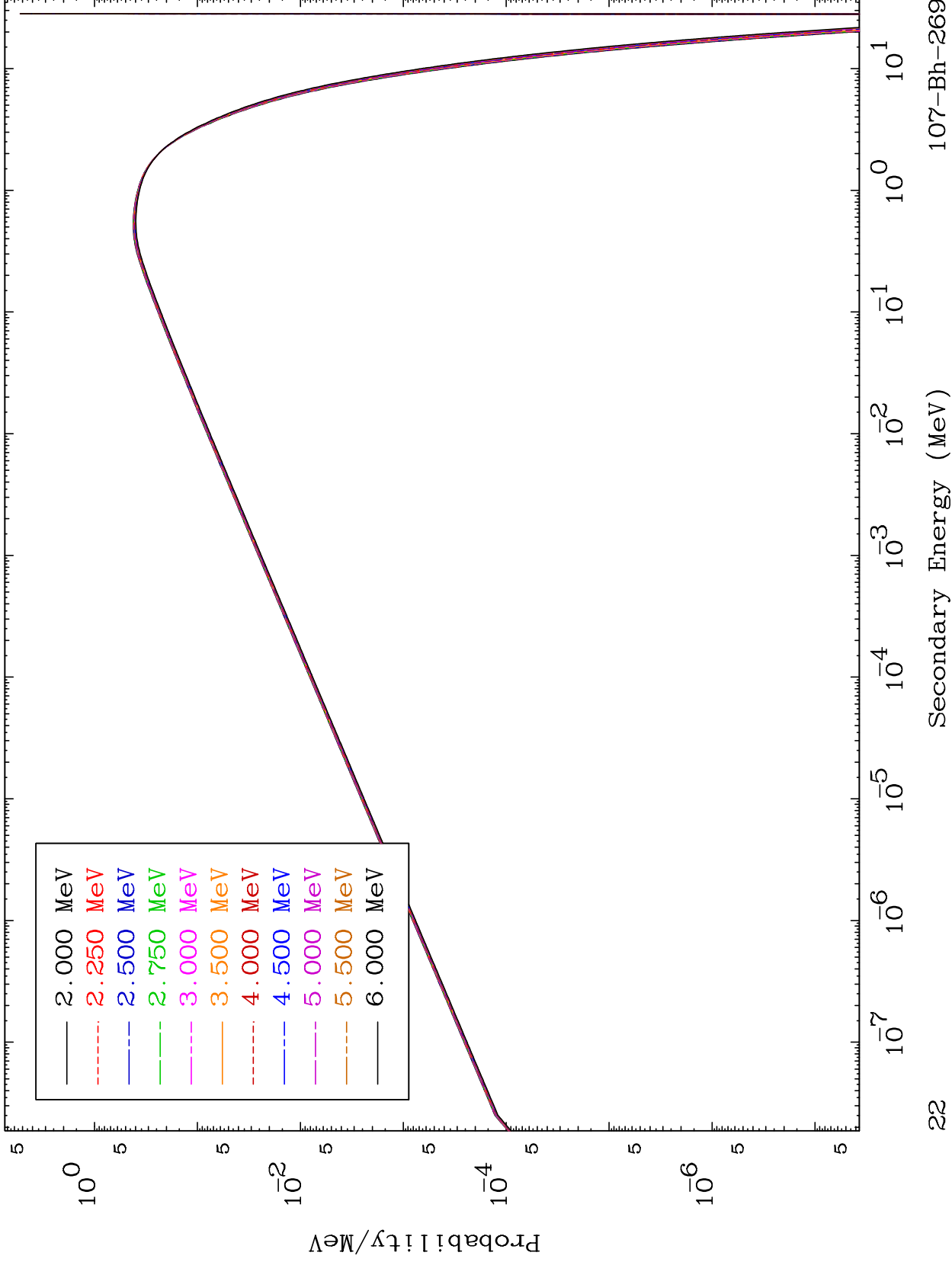
19

Incident Energy (MeV)

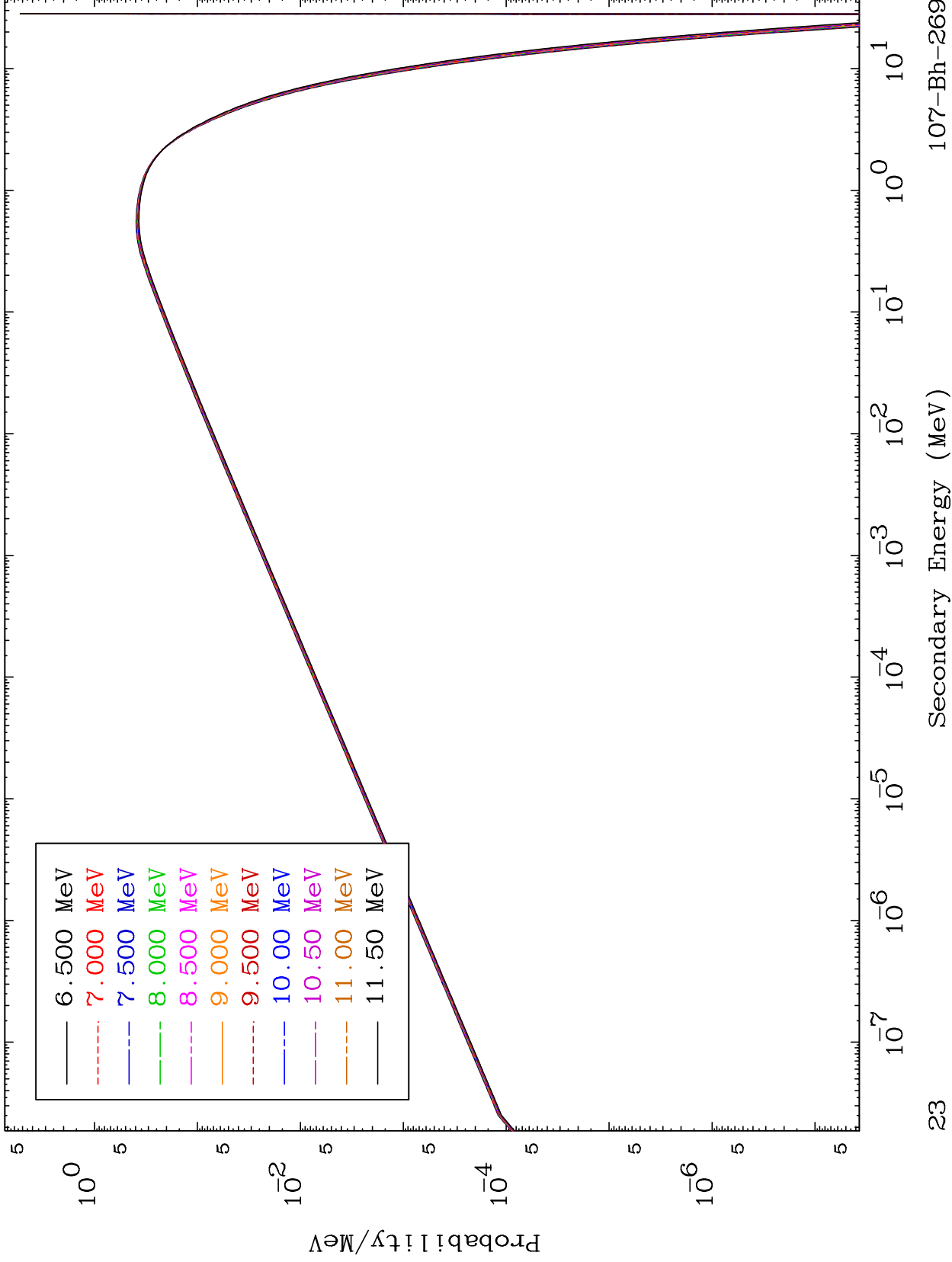
107-Bh-269



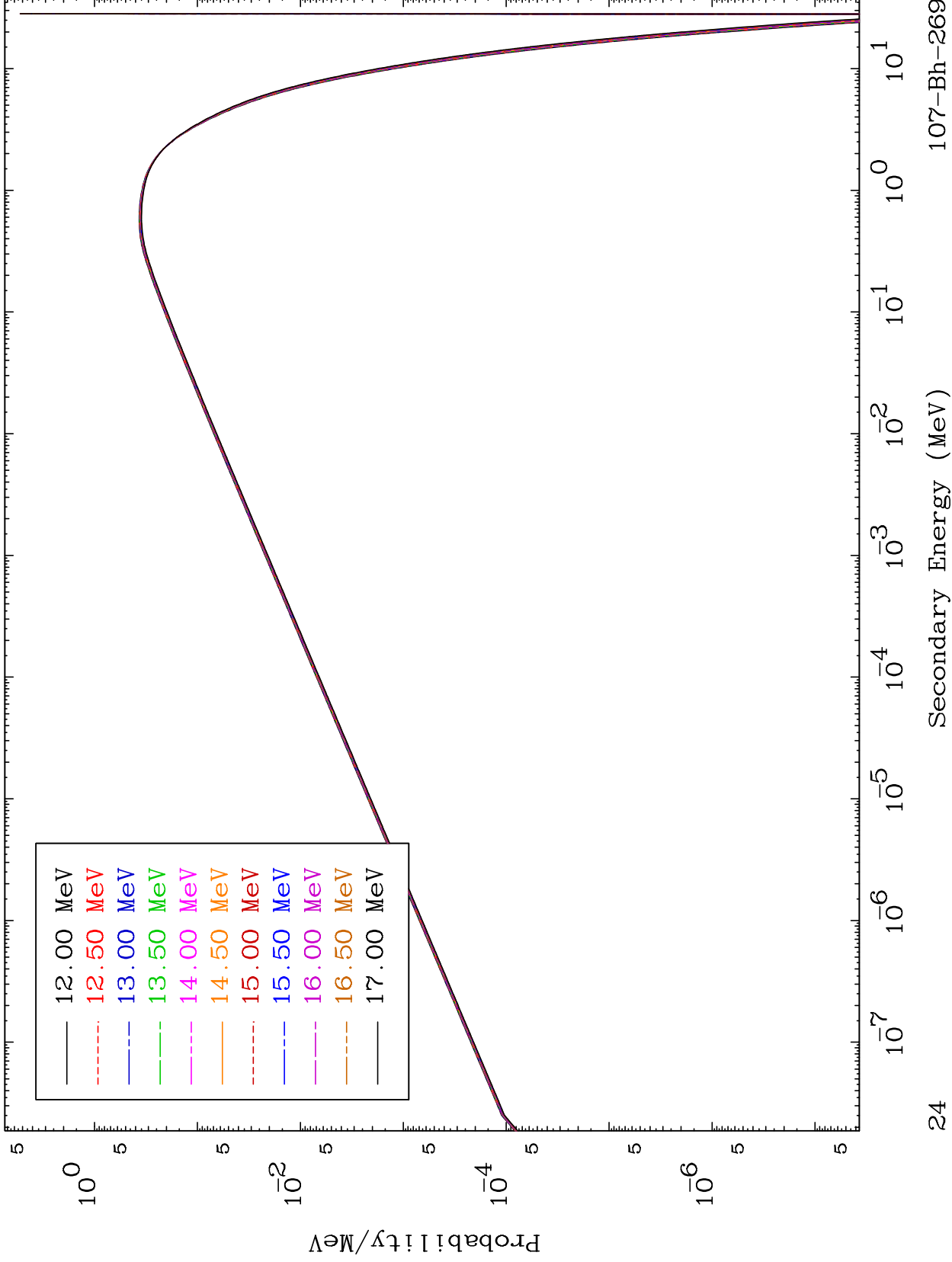
Fission
Energy Distribution



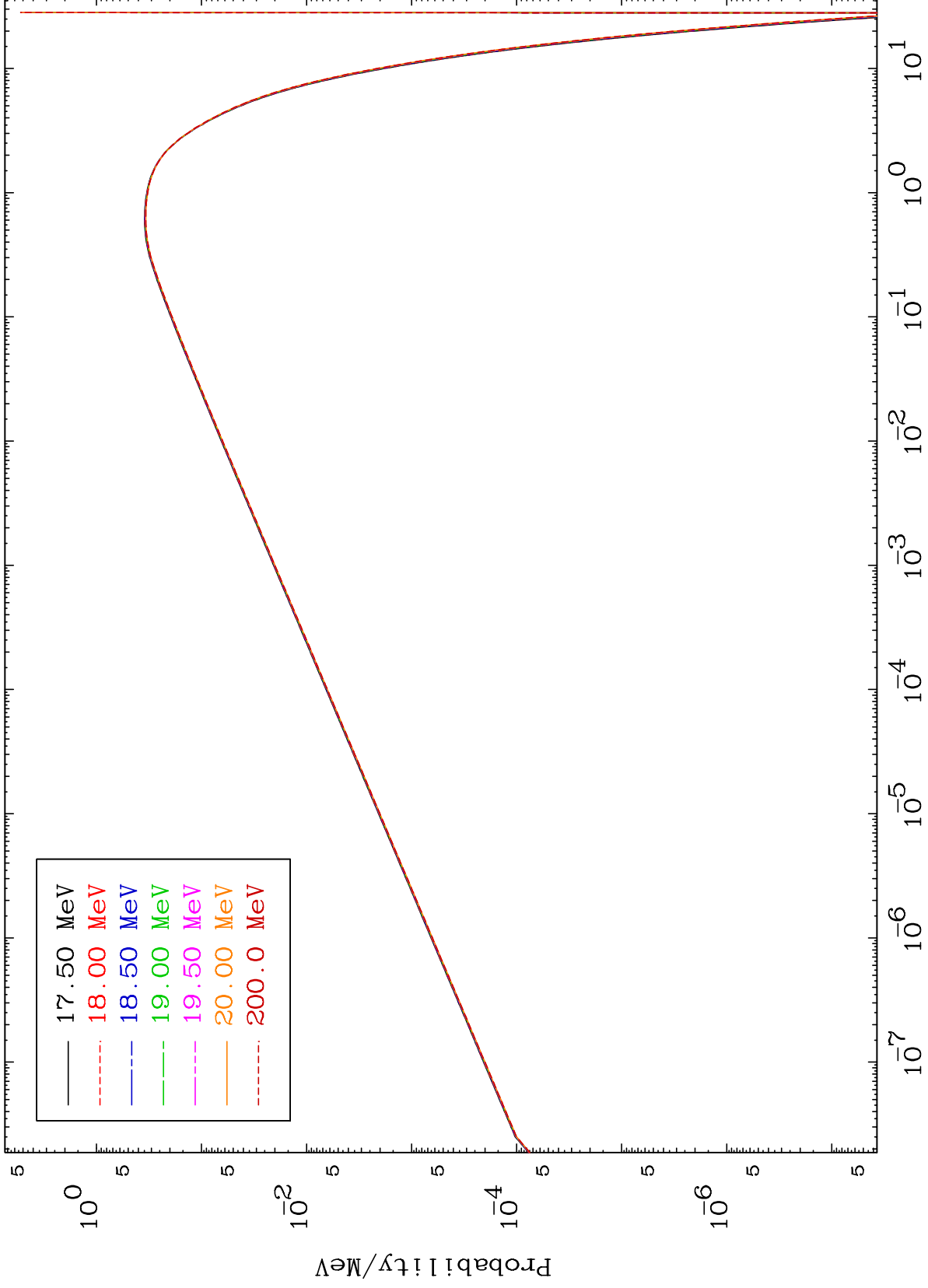
Fission
Energy Distribution



Fission
Energy Distribution



Fission
Energy Distribution

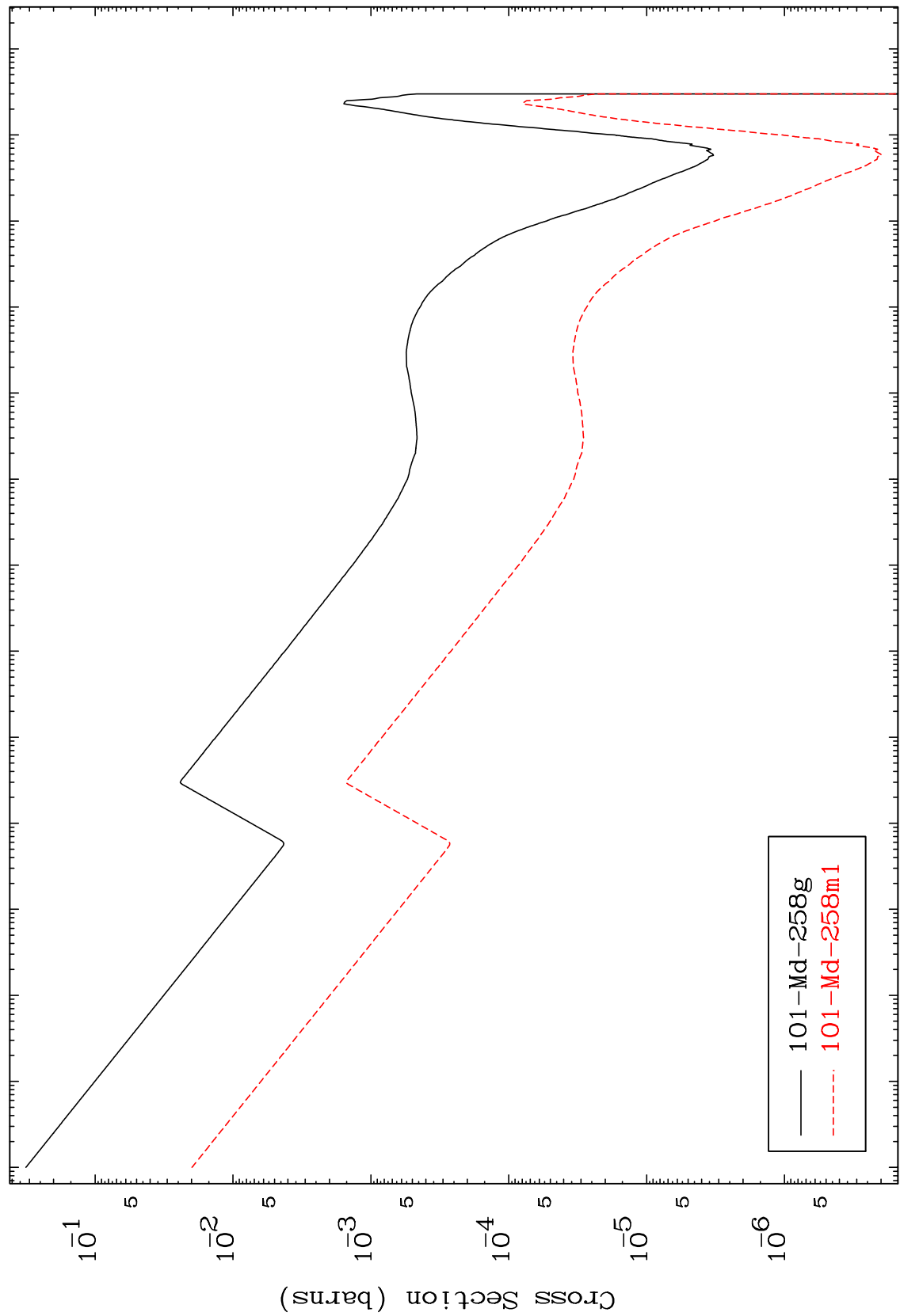


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(n, 3α)

107-Bh-269

Radionuclide Production Cross Section



— 101-Md-258g
- - - 101-Md-258m1