

Program EVALPLOT
(Version 2015-2)

by

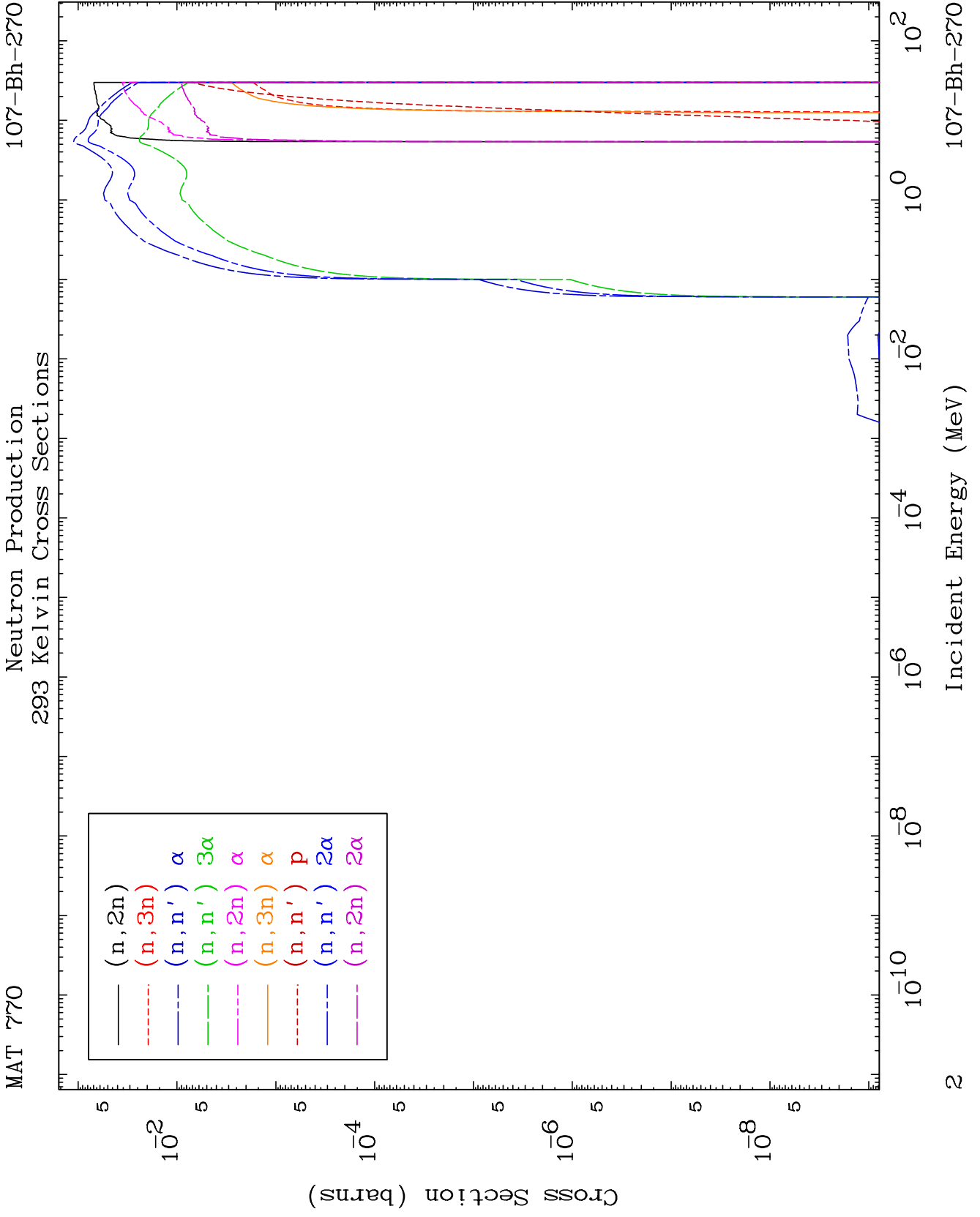
Dermott E. Cullen
(Present Contact Information)

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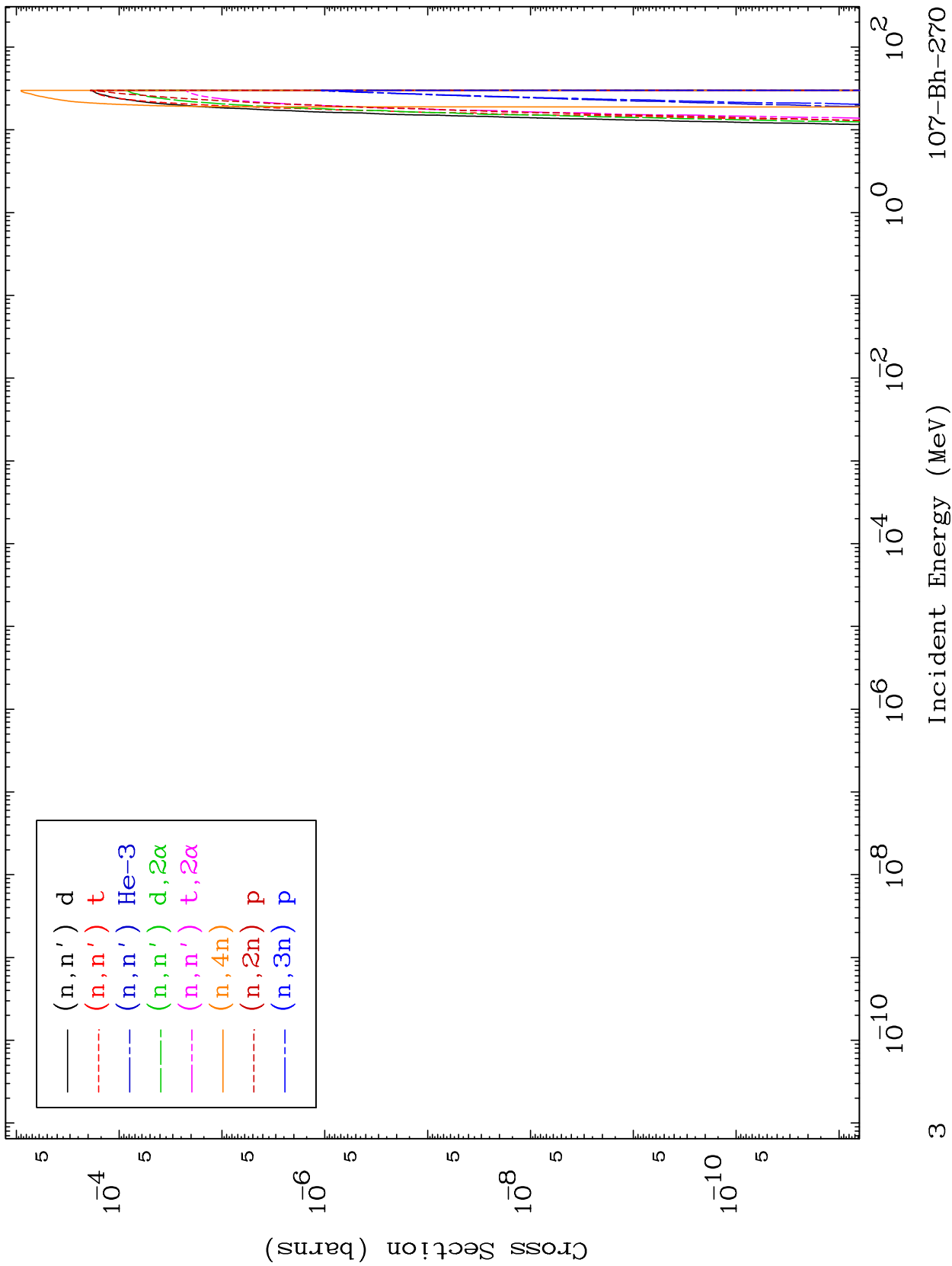
Press Mouse Button to Start

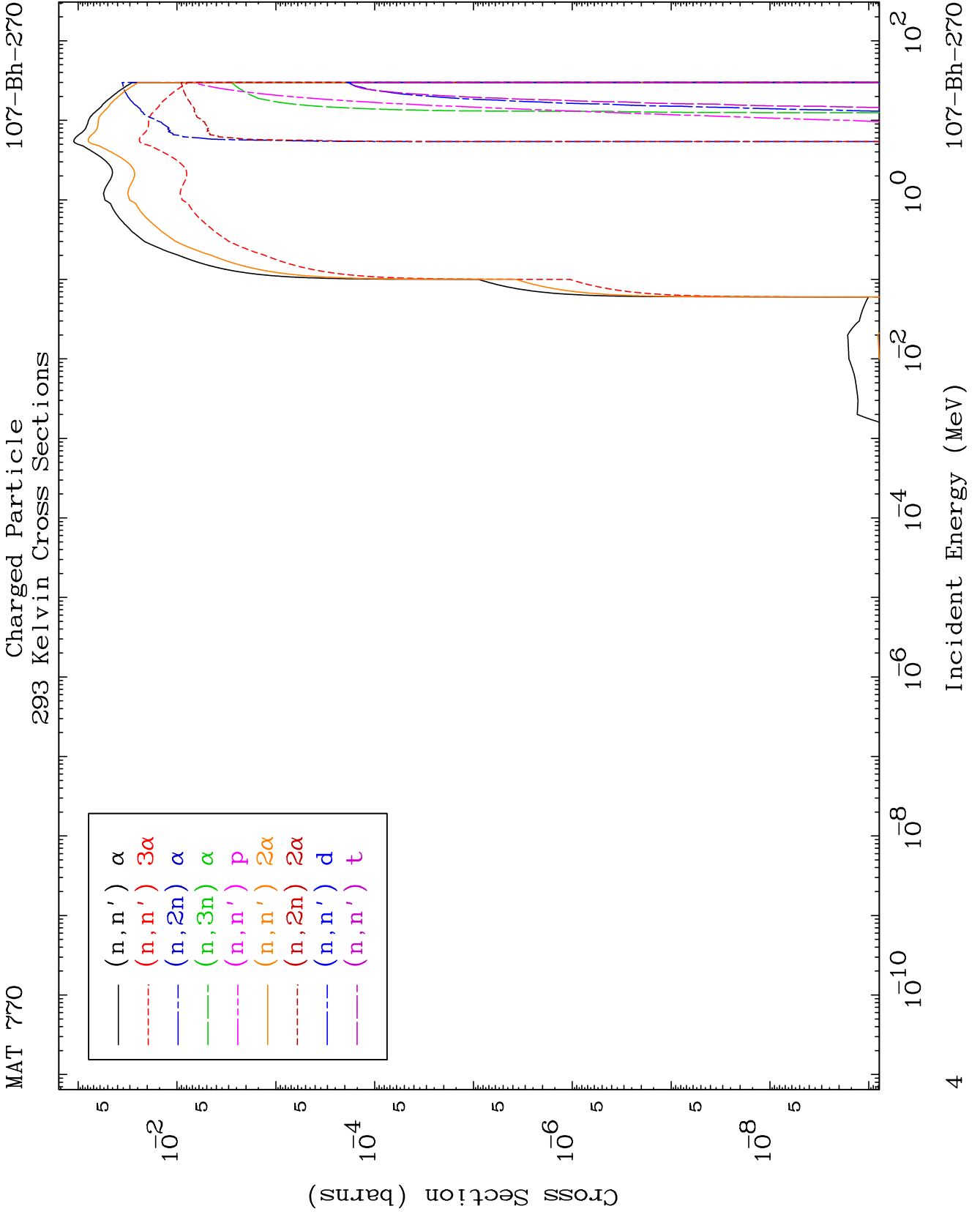


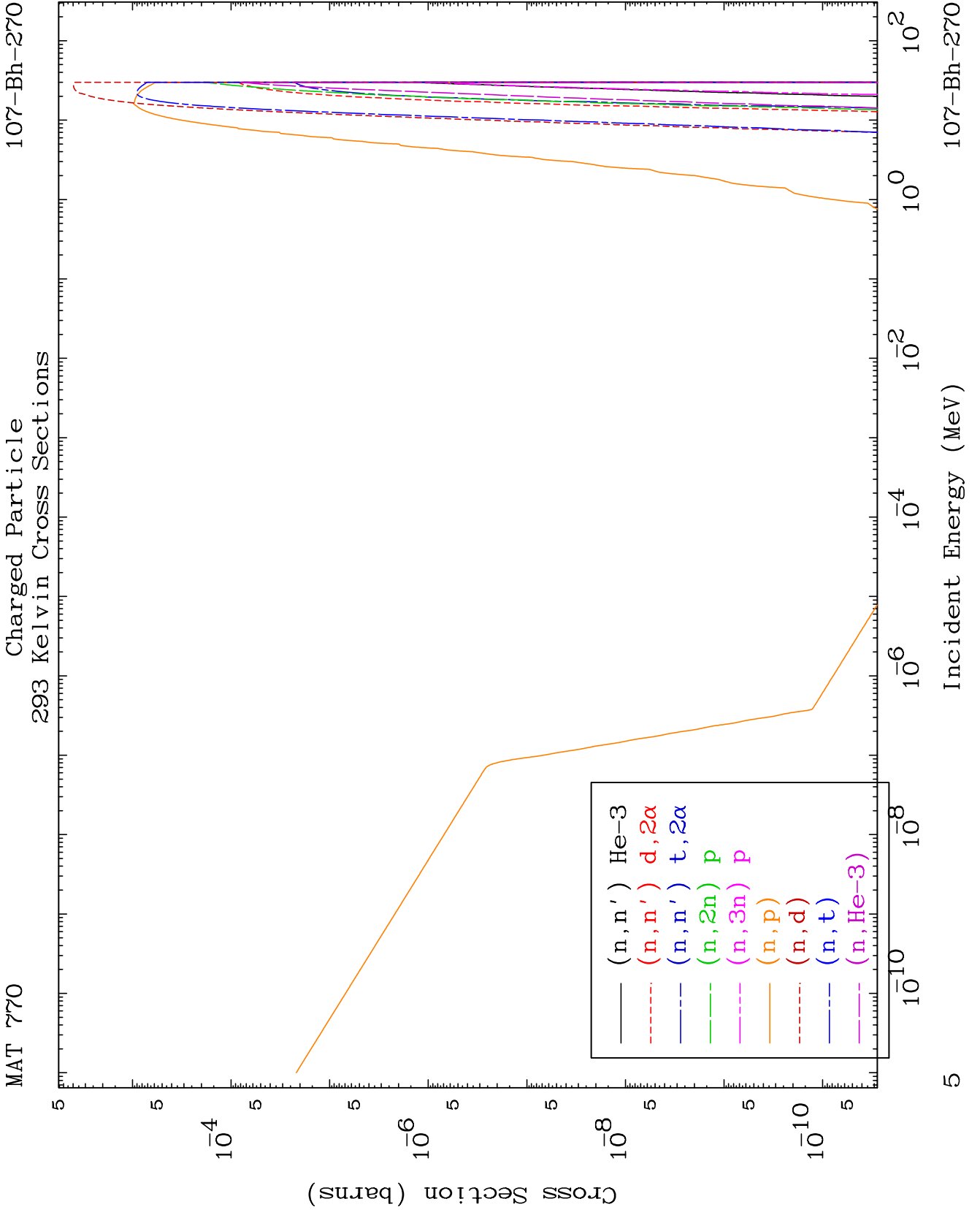
MAT 770

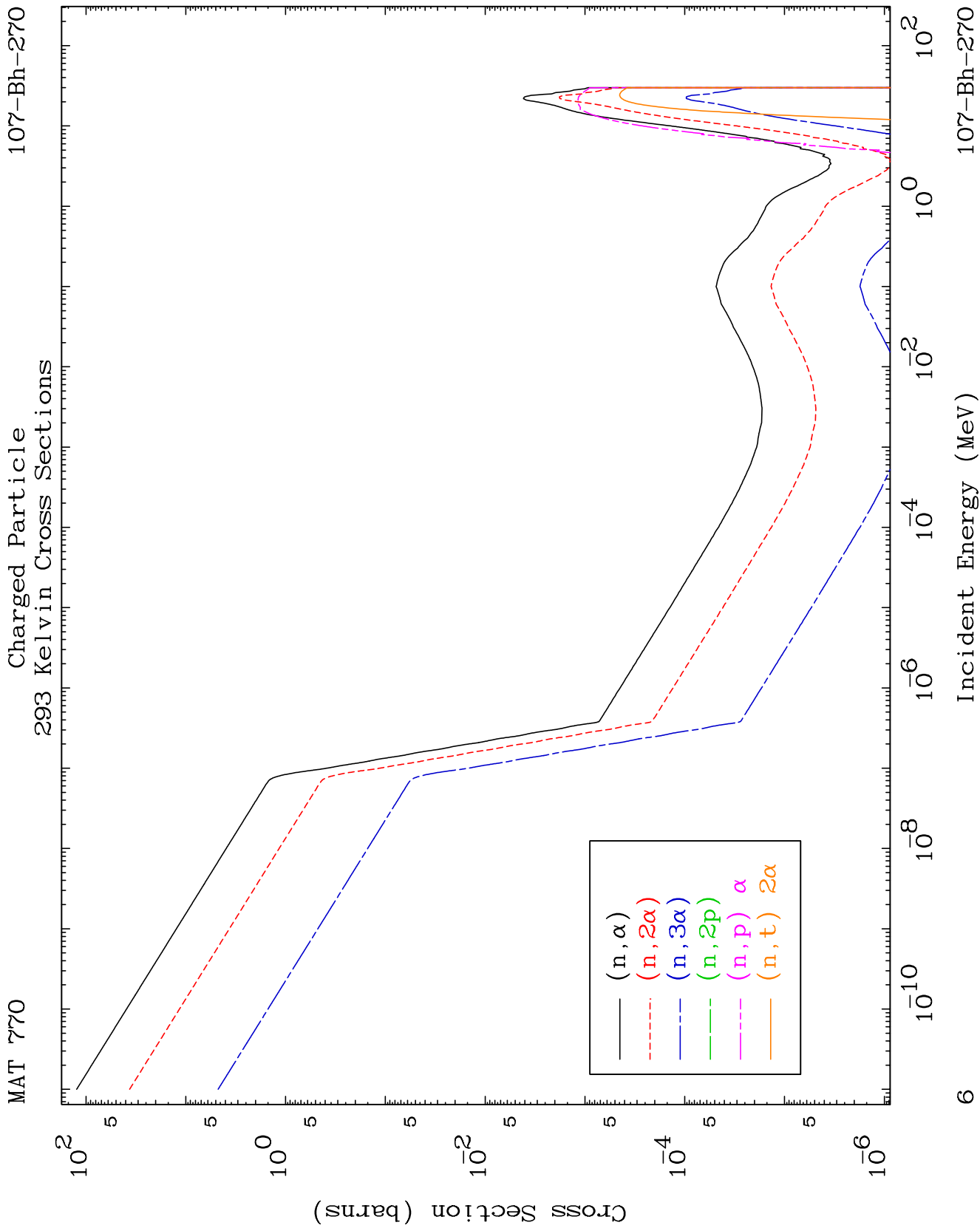
Neutron Production
293 Kelvin Cross Sections

107-Bh-270





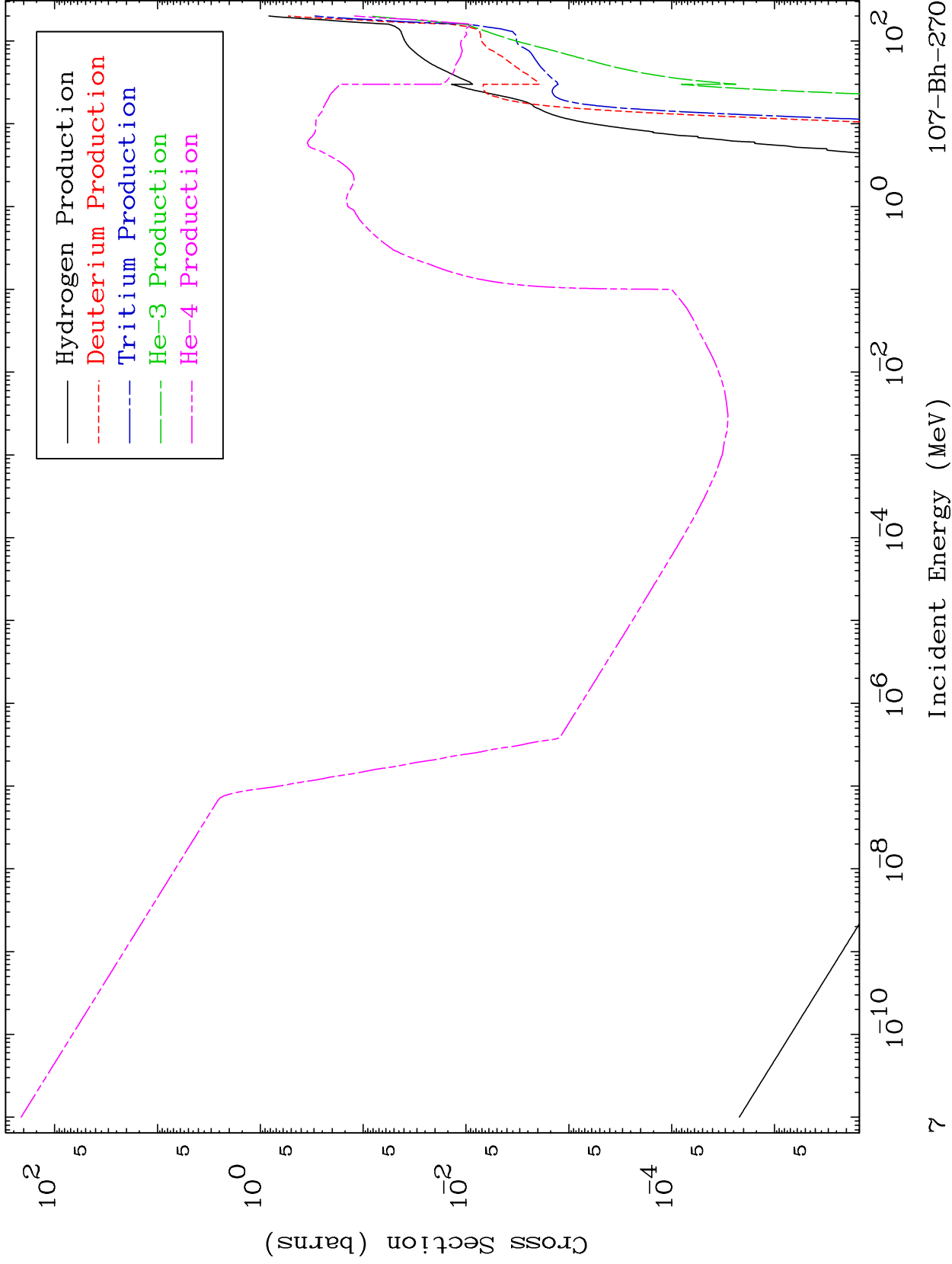


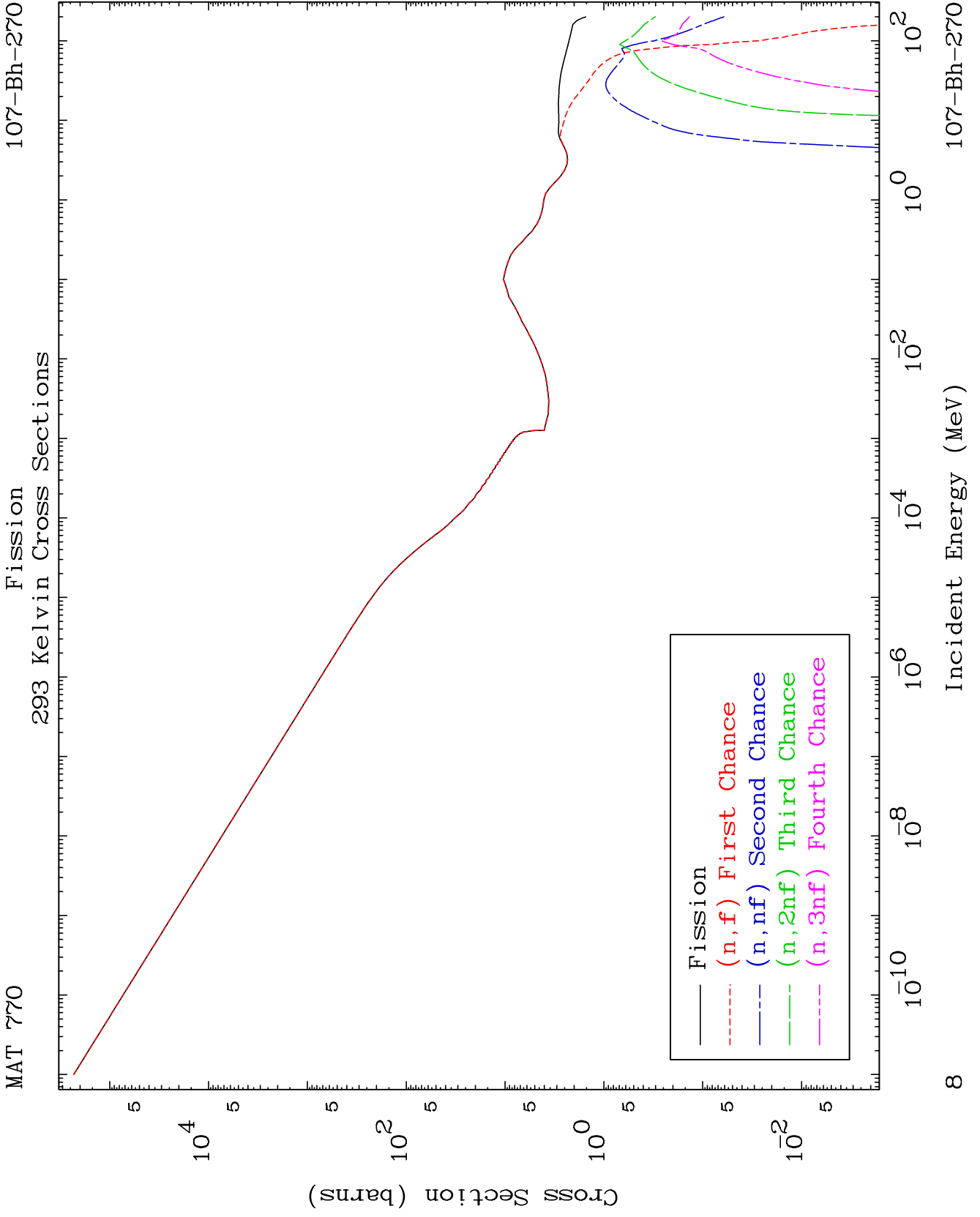


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Particle Production
293 Kelvin Cross Sections

107-Bh-270

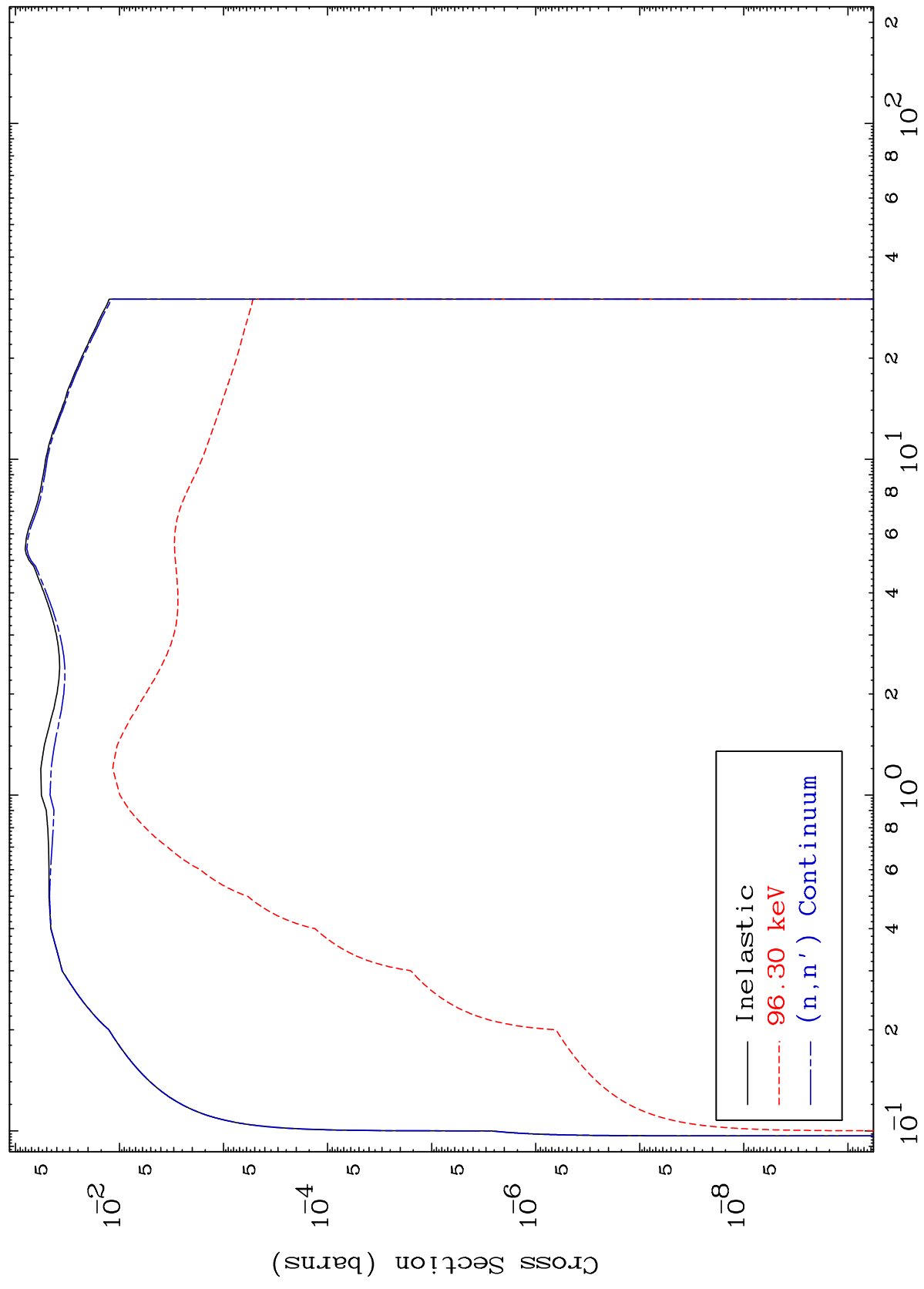




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(n,n') Level
293 Kelvin Cross Sections

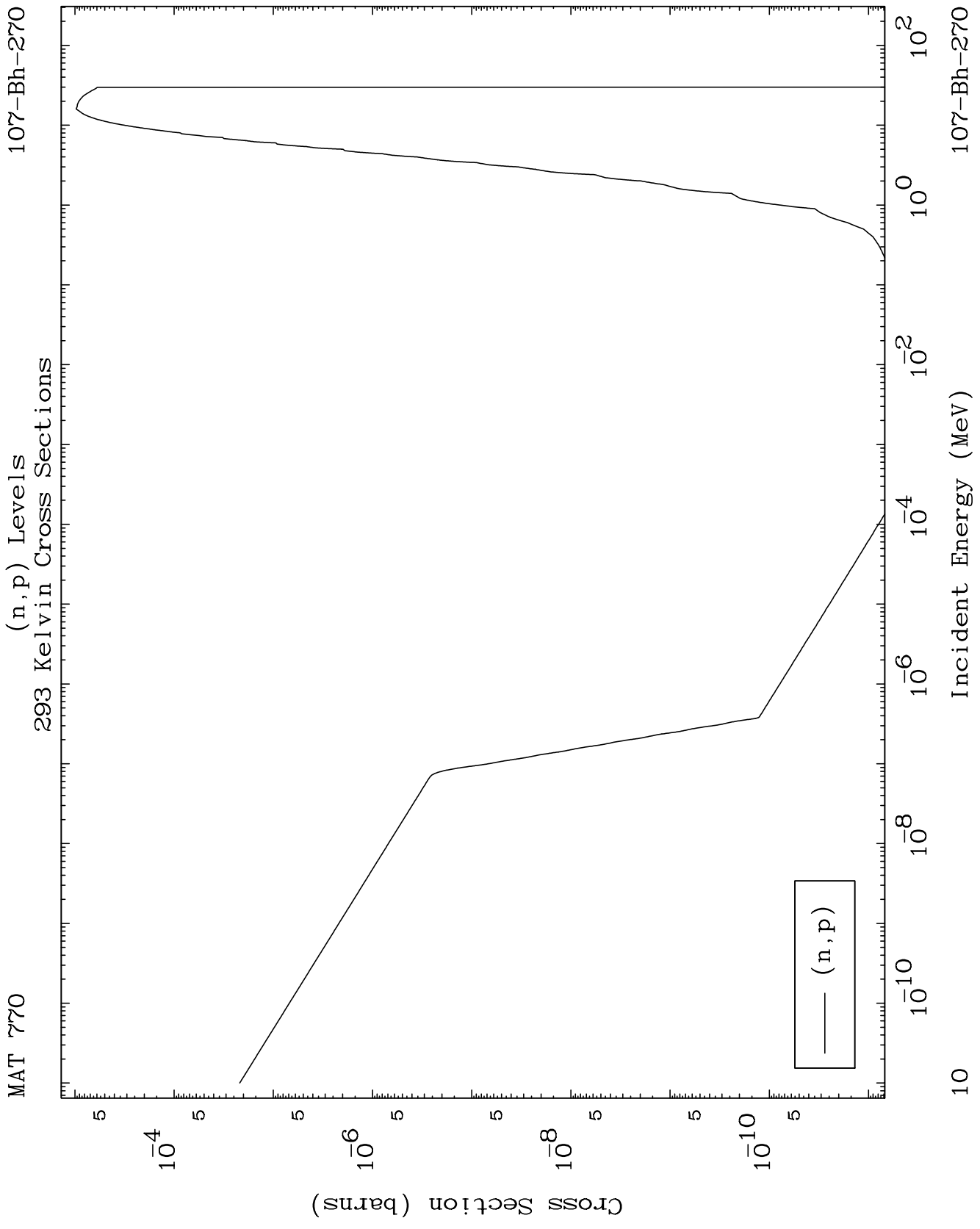
107-Bh-270



Incident Energy (MeV)

107-Bh-270

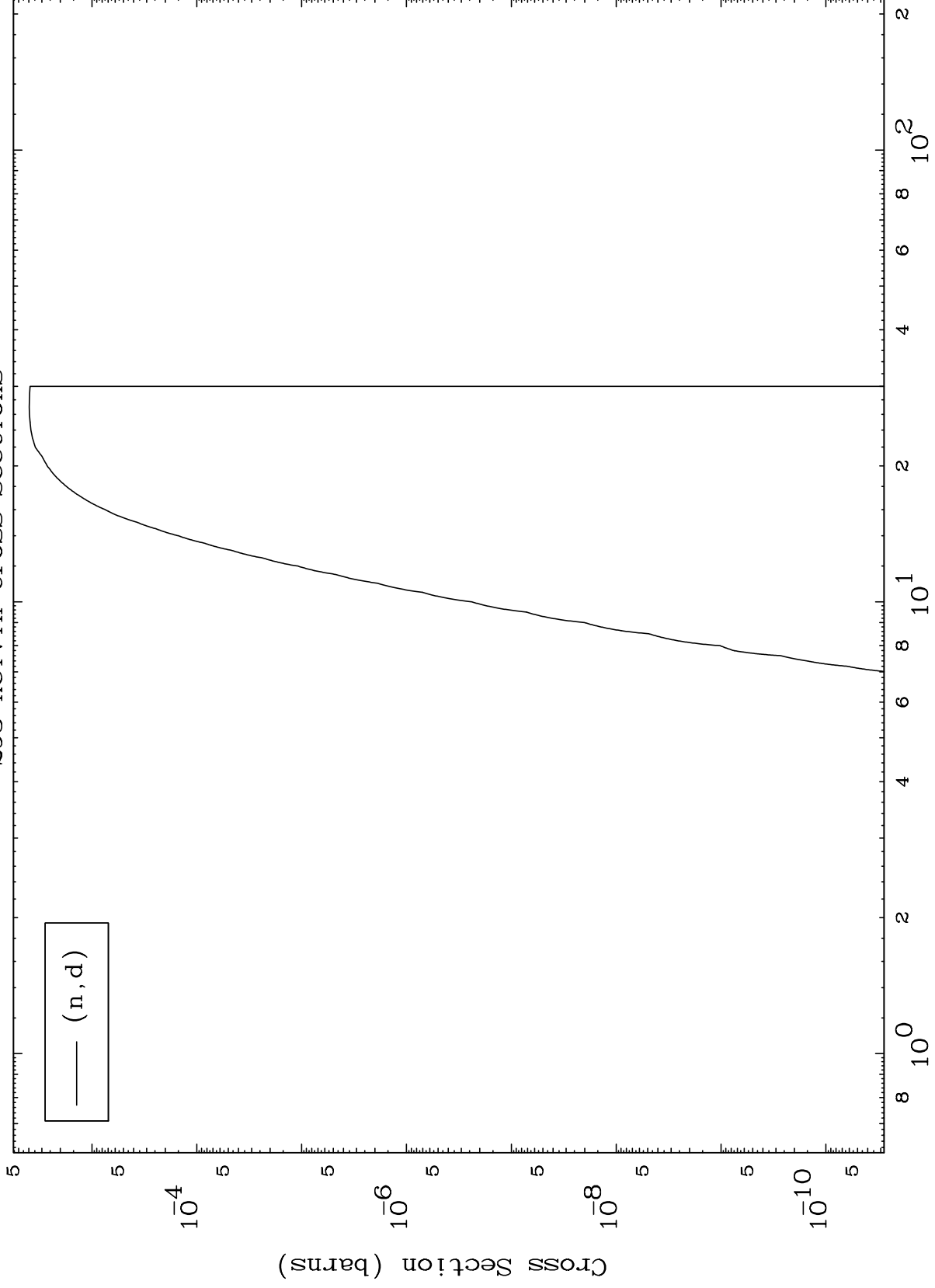
9



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(n,d) Levels
293 Kelvin Cross Sections

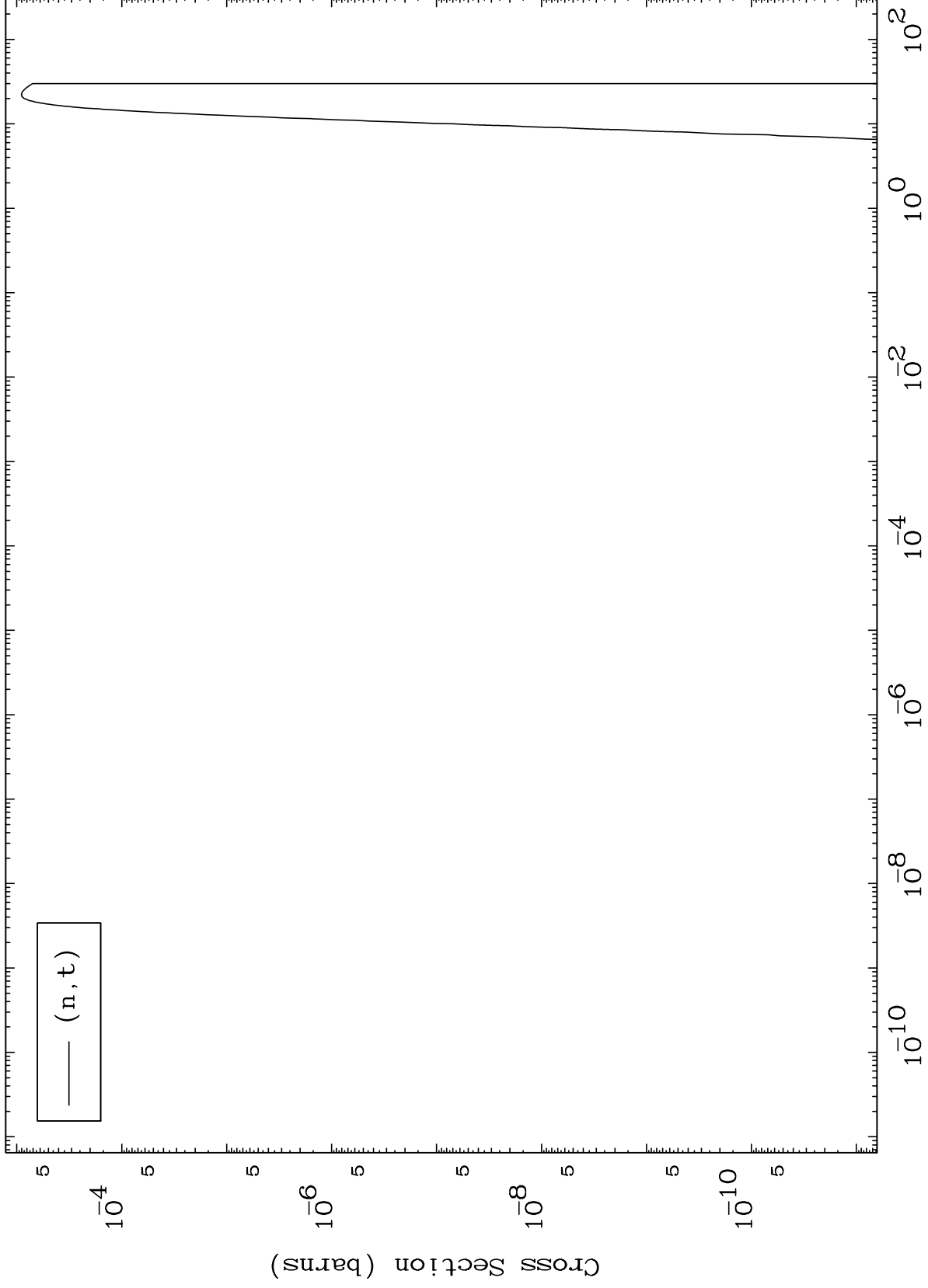
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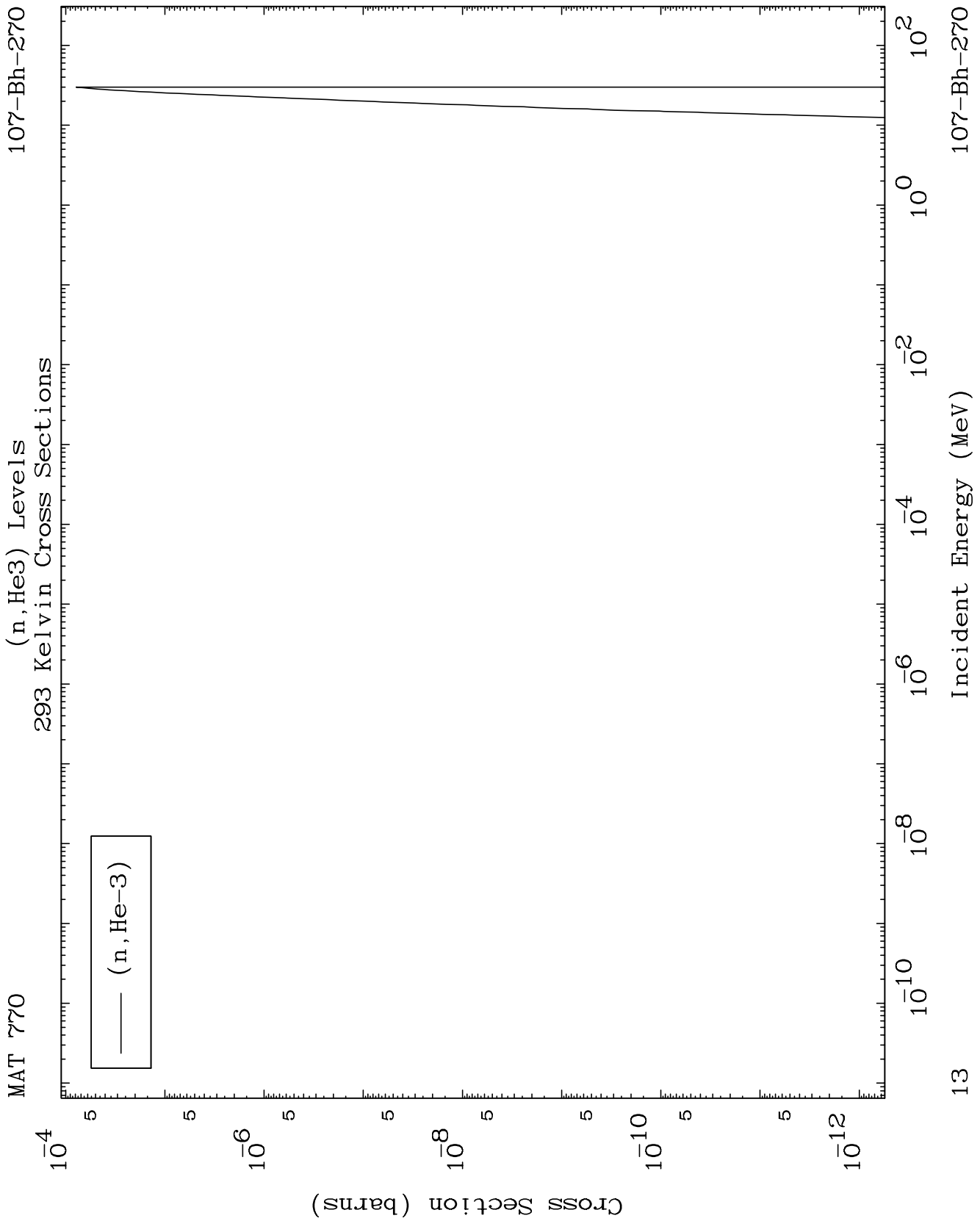


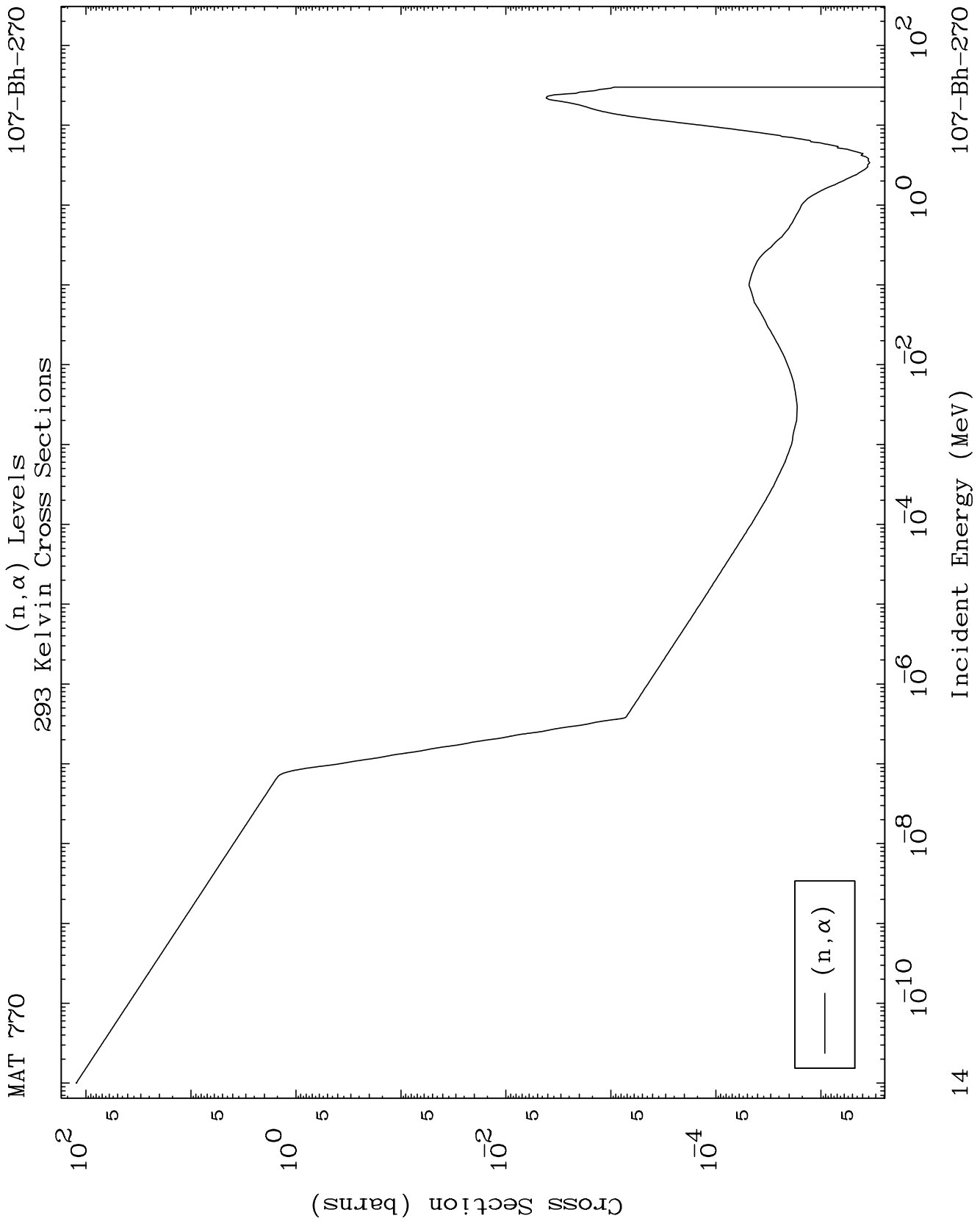
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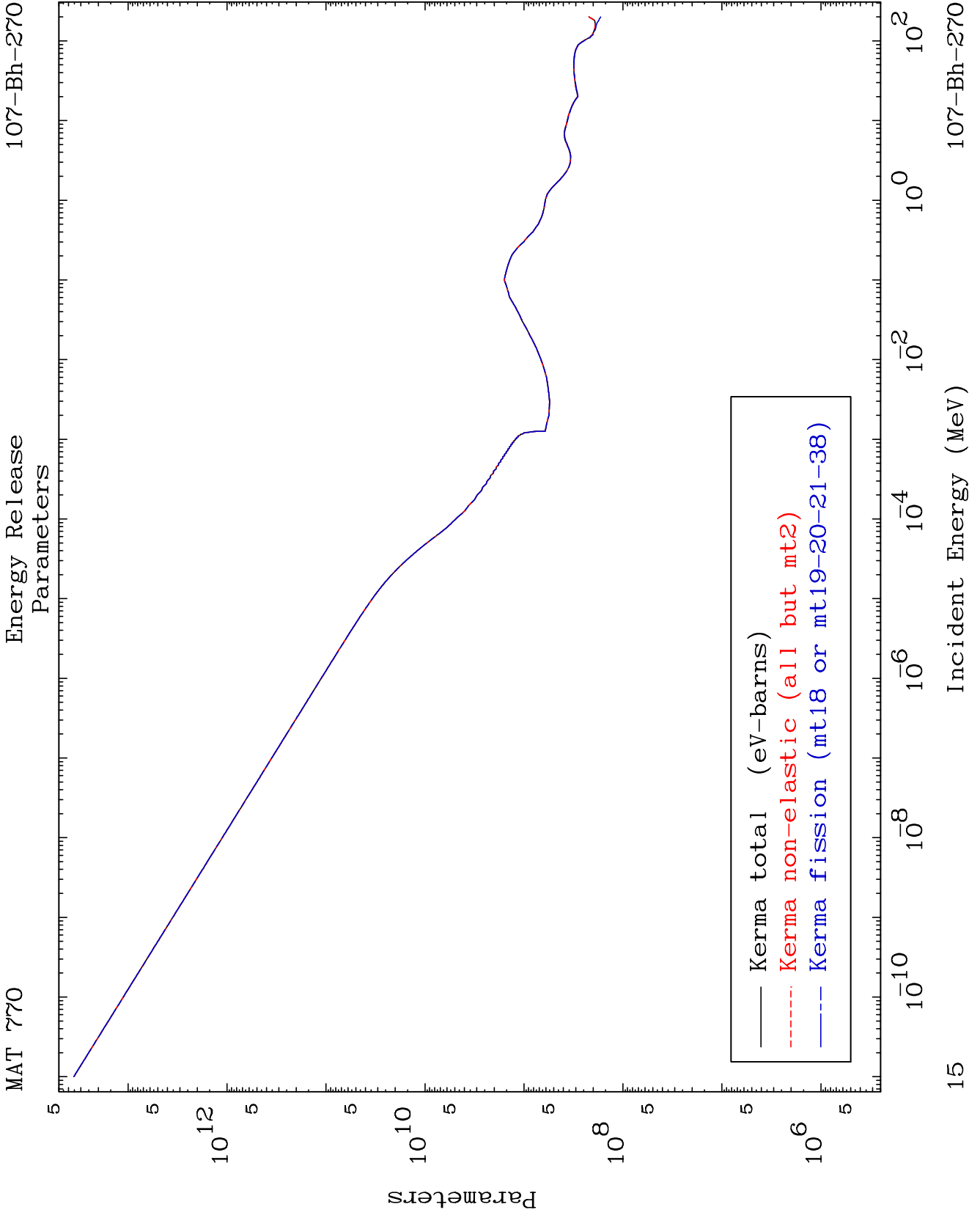
(n,t) Levels
293 Kelvin Cross Sections

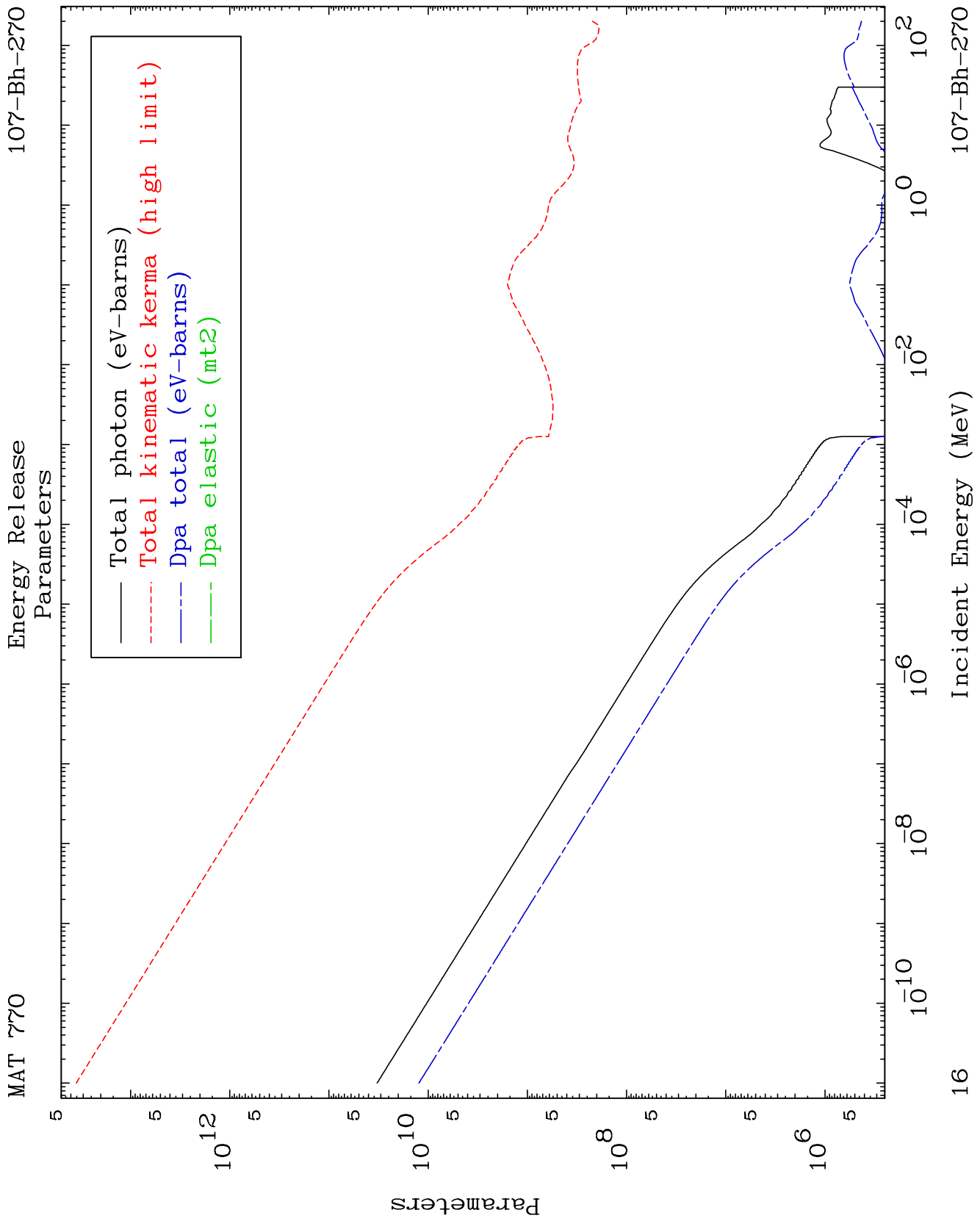
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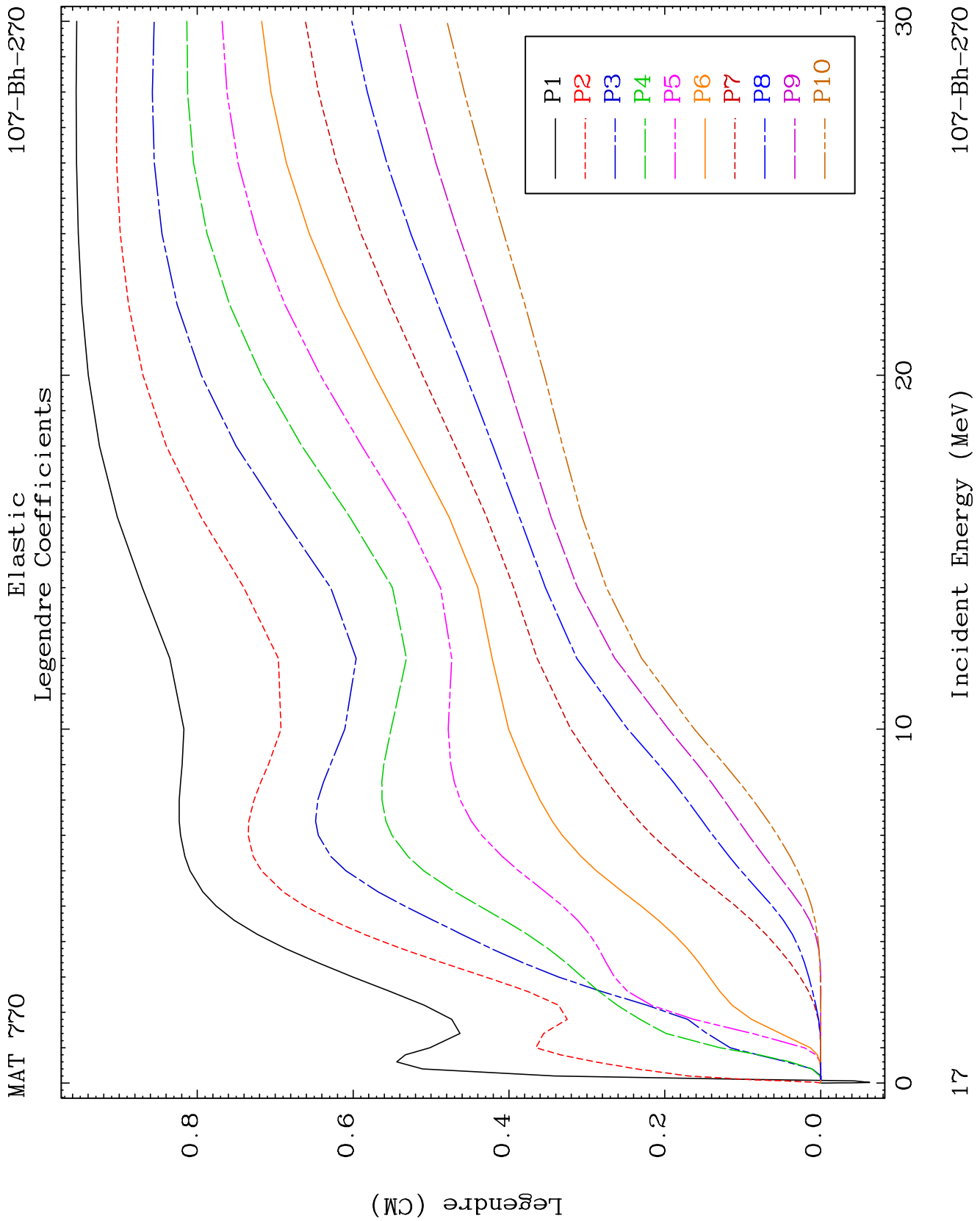


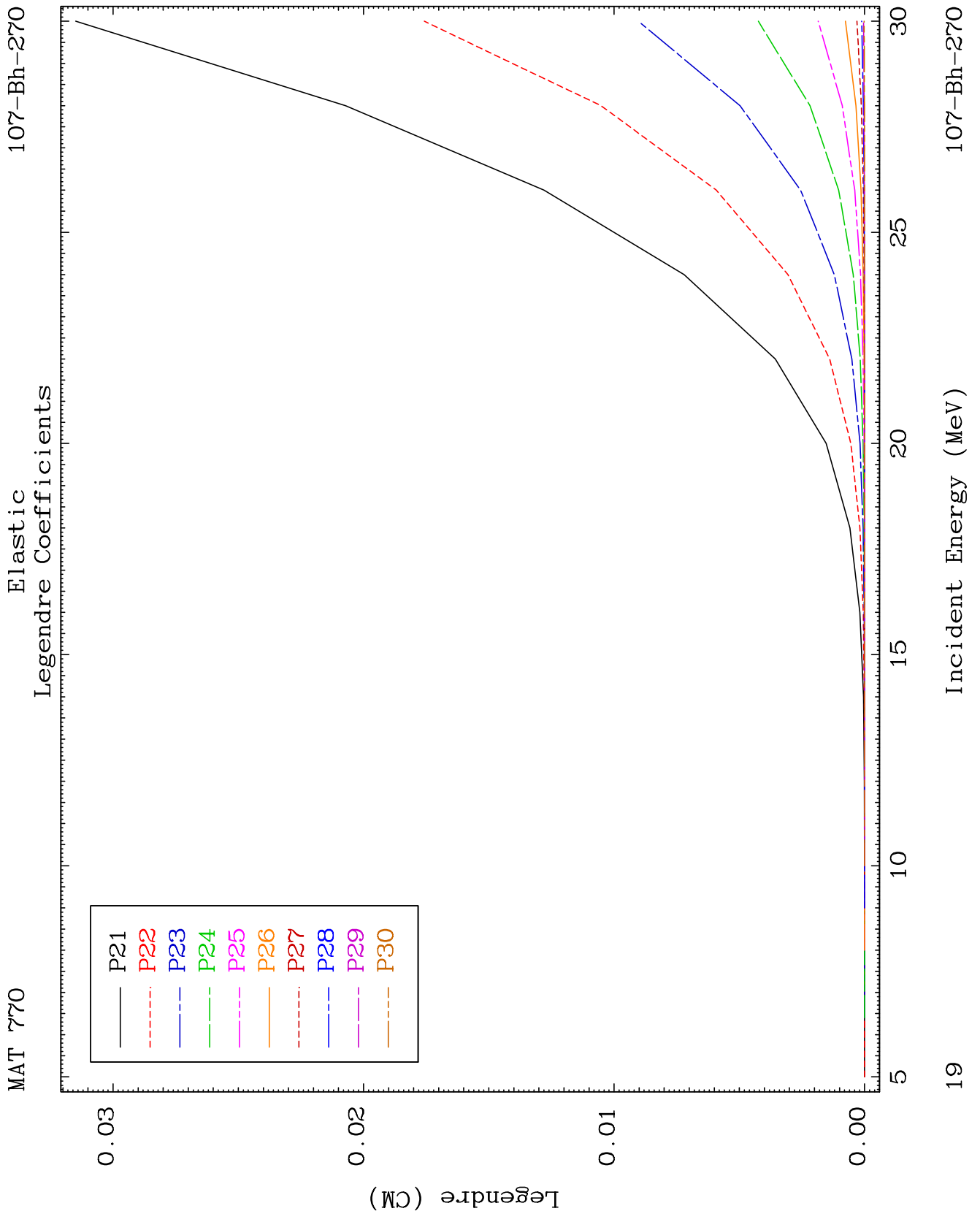








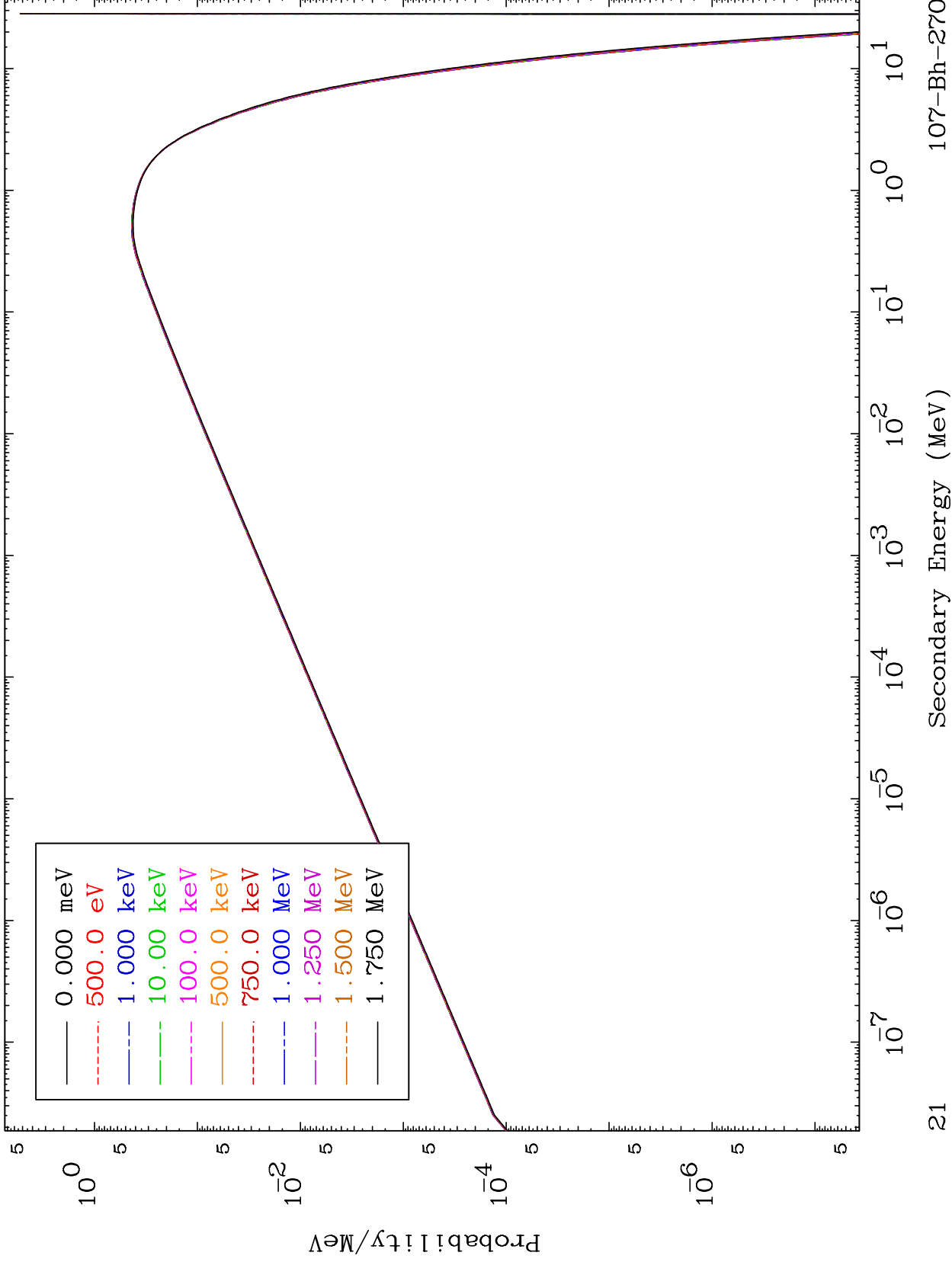




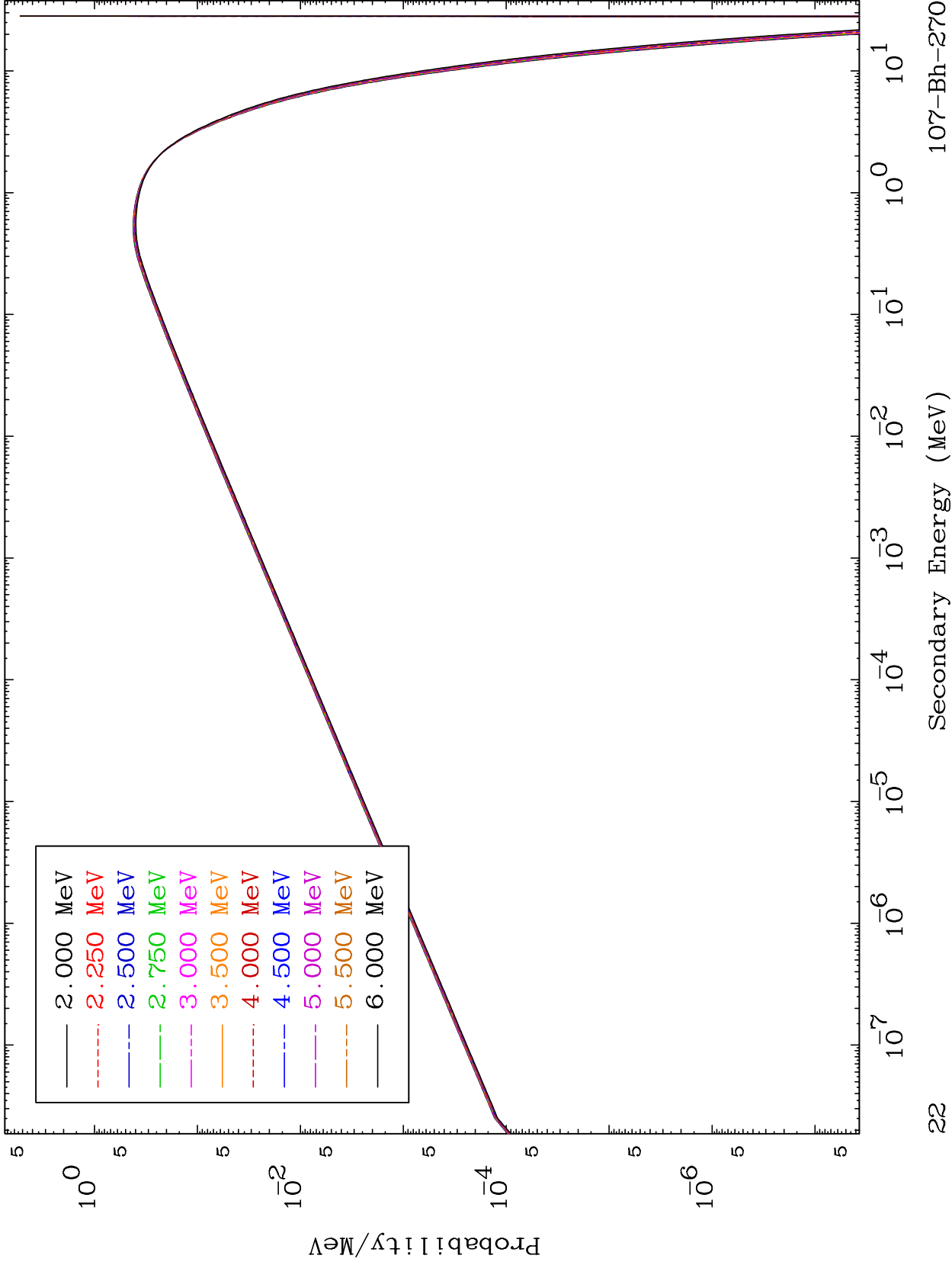
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Fission Energy Distribution

107-Bh-270



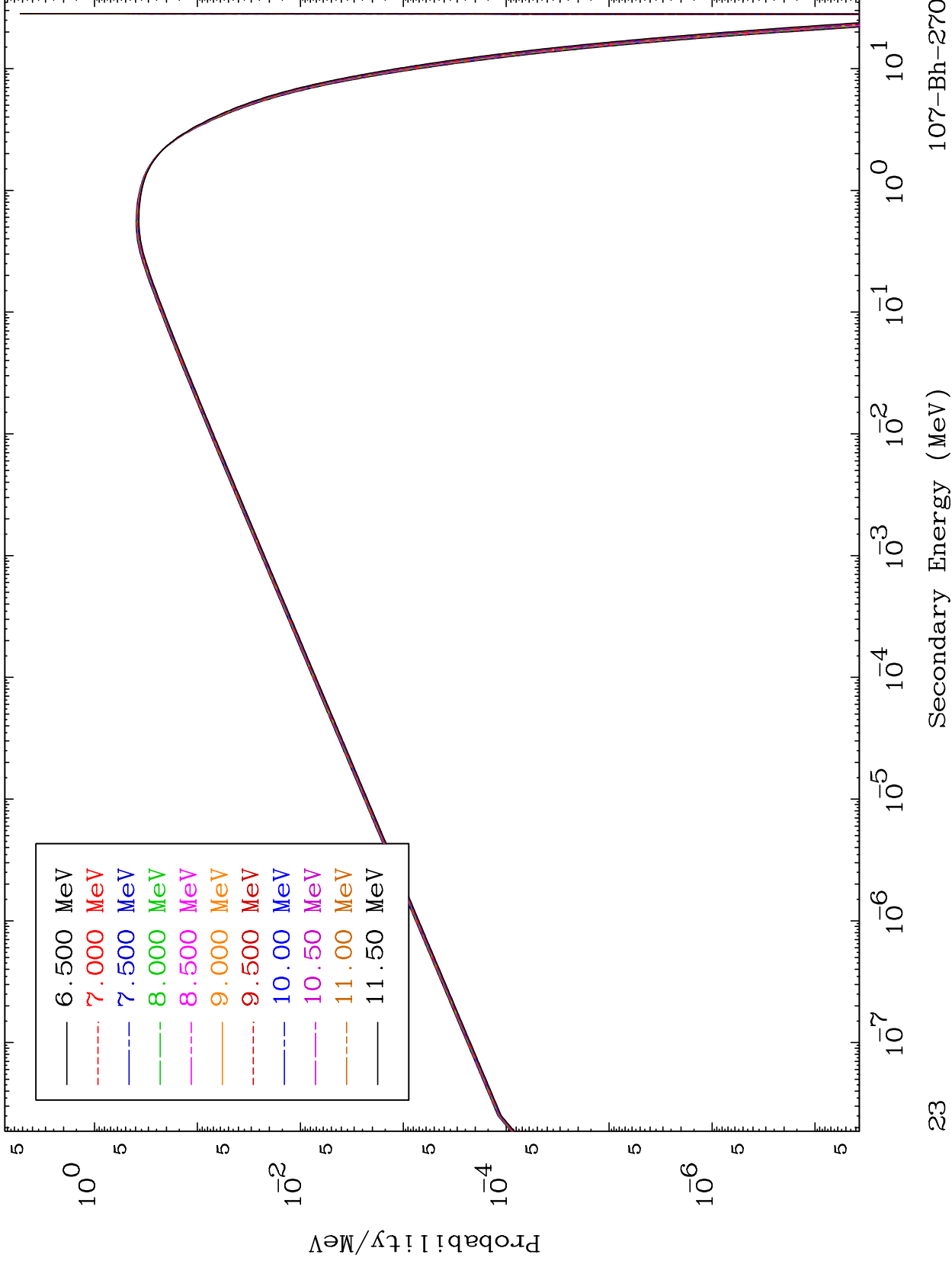
Fission Energy Distribution



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Fission Energy Distribution

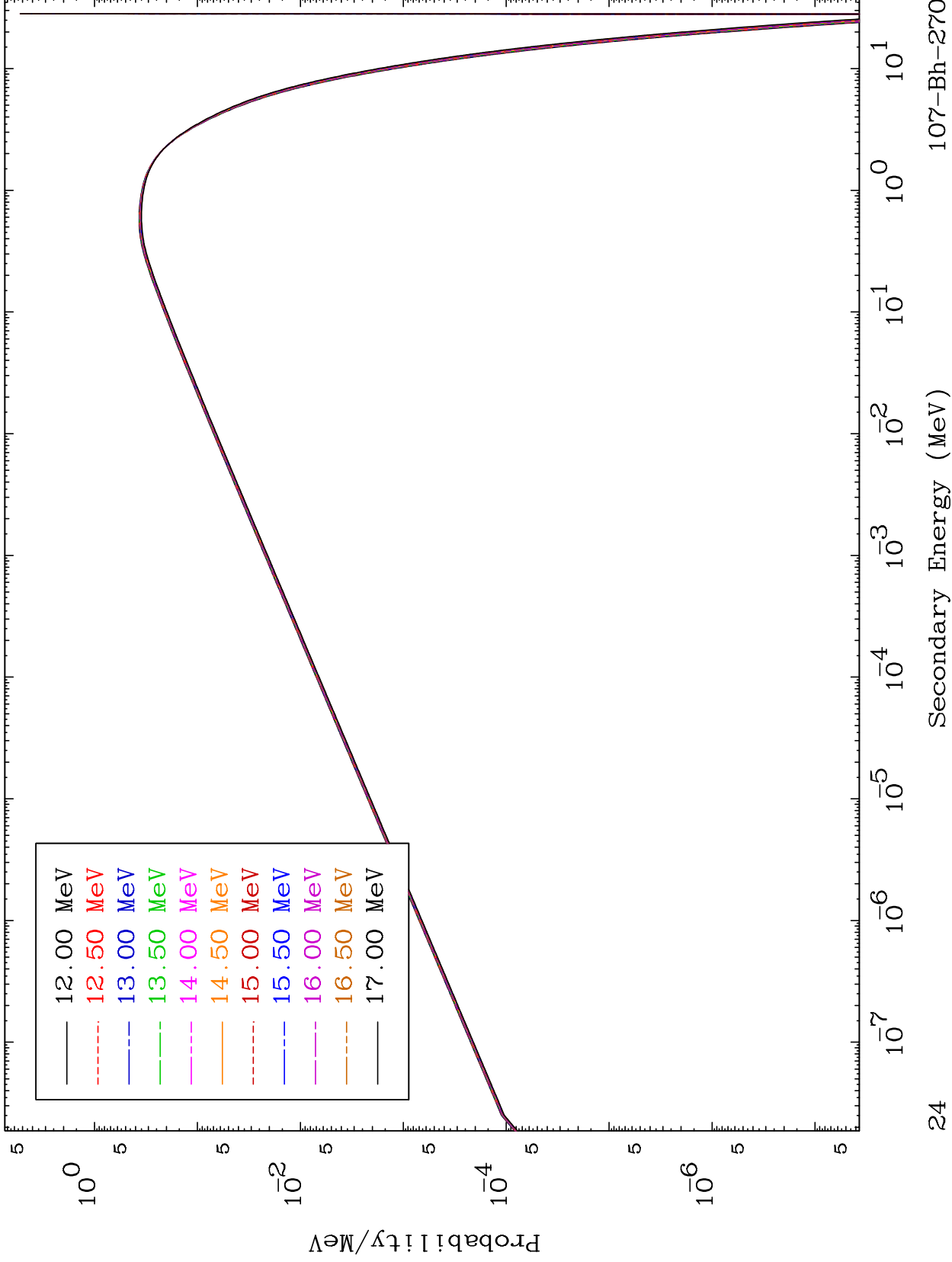
107-Bh-270



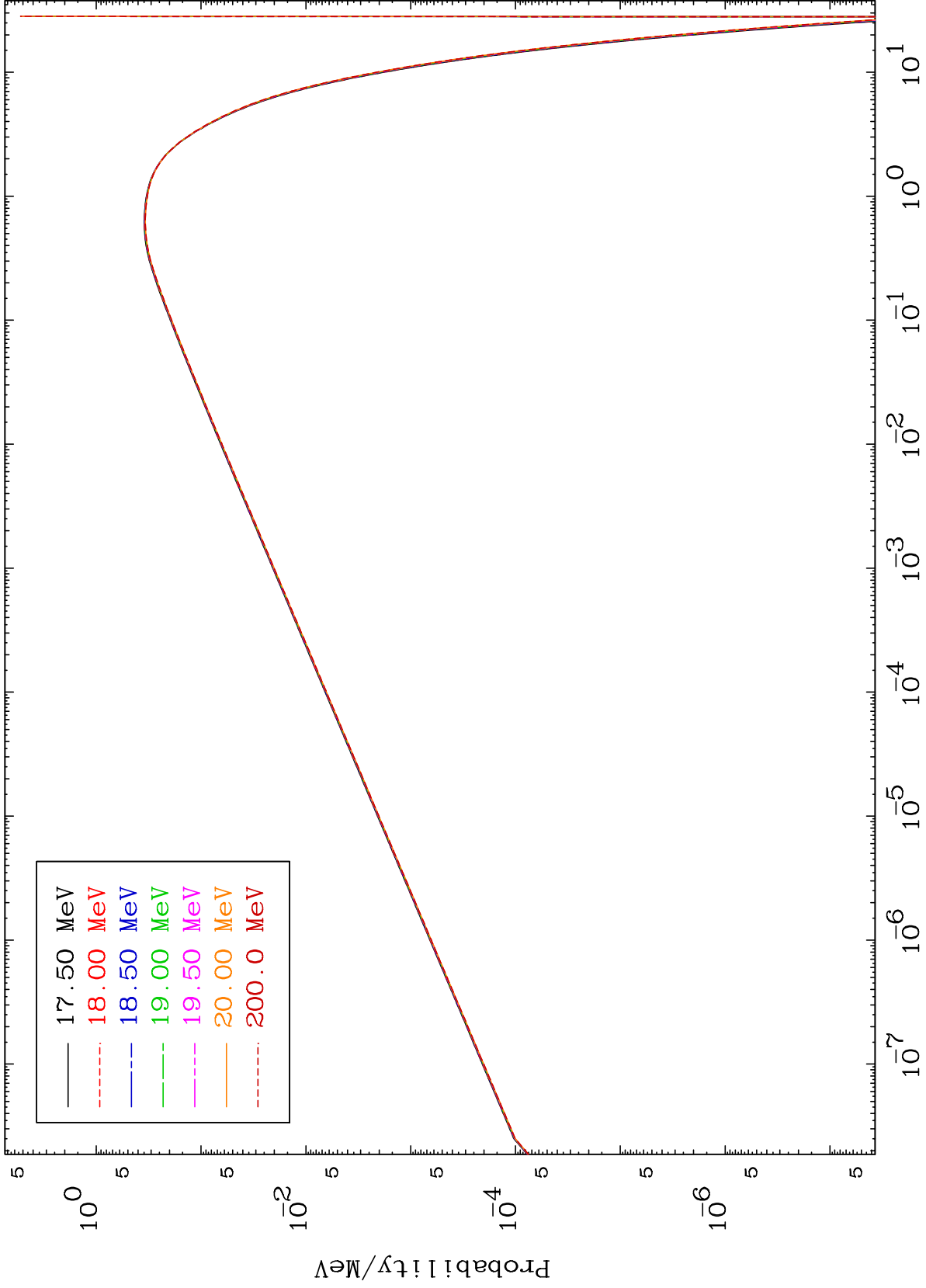
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Fission
Energy Distribution

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Fission
Energy Distribution

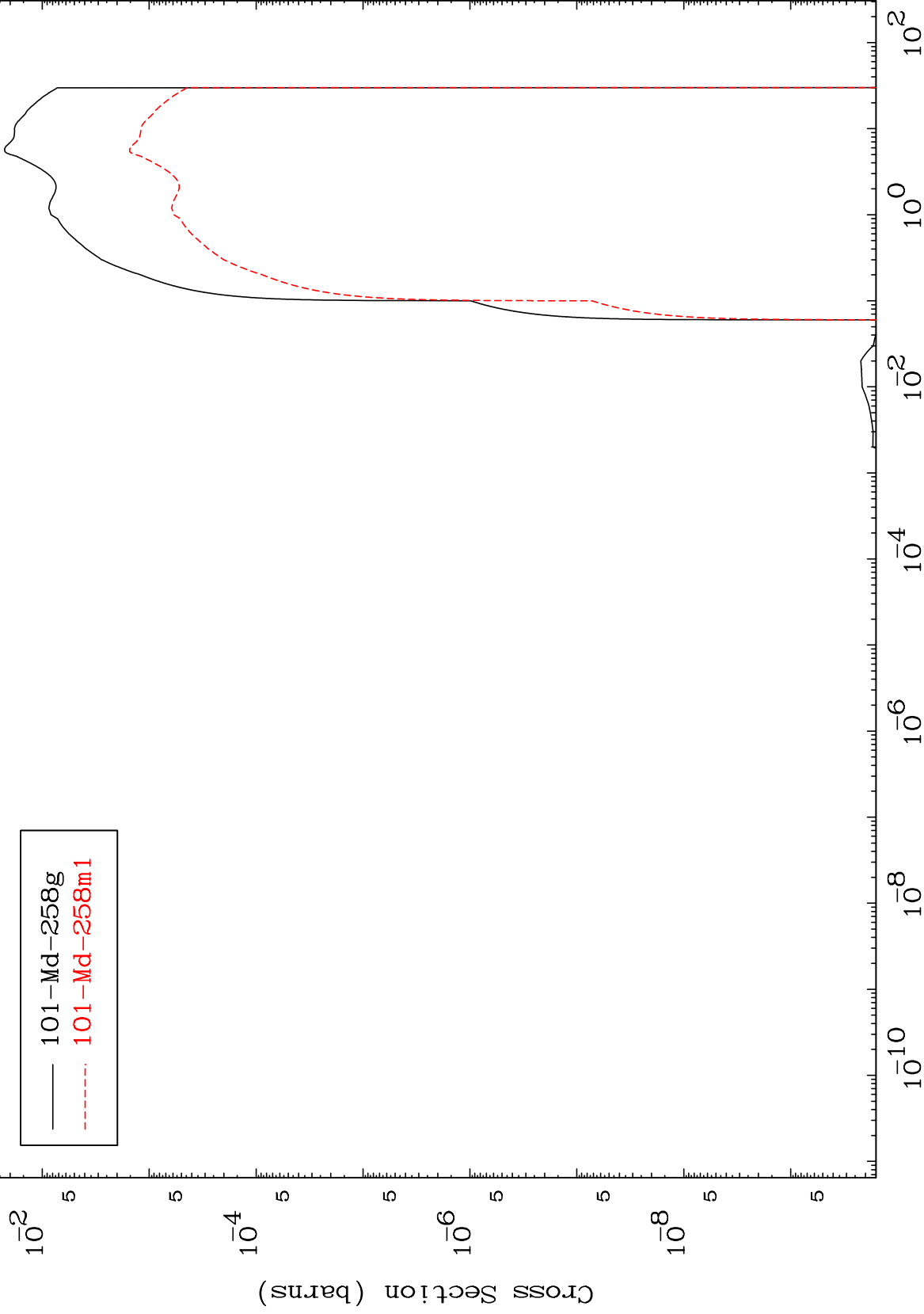


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(n,n') 3 α

107-Bh-270

Radionuclide Production Cross Section



26

Incident Energy (MeV)

107-Bh-270