

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

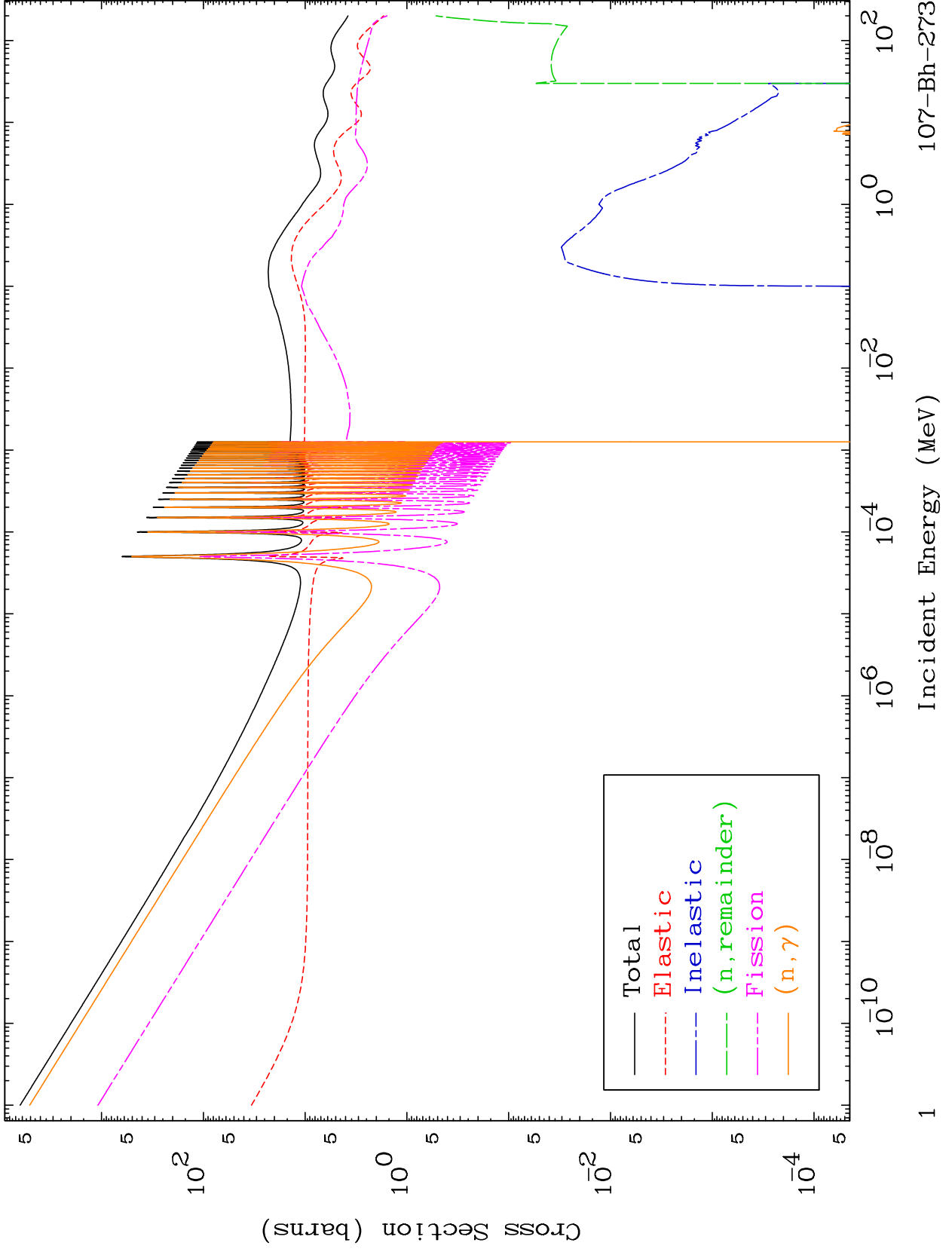
Press Mouse Button to Start

MAT 773

Major

293 Kelvin Cross Sections

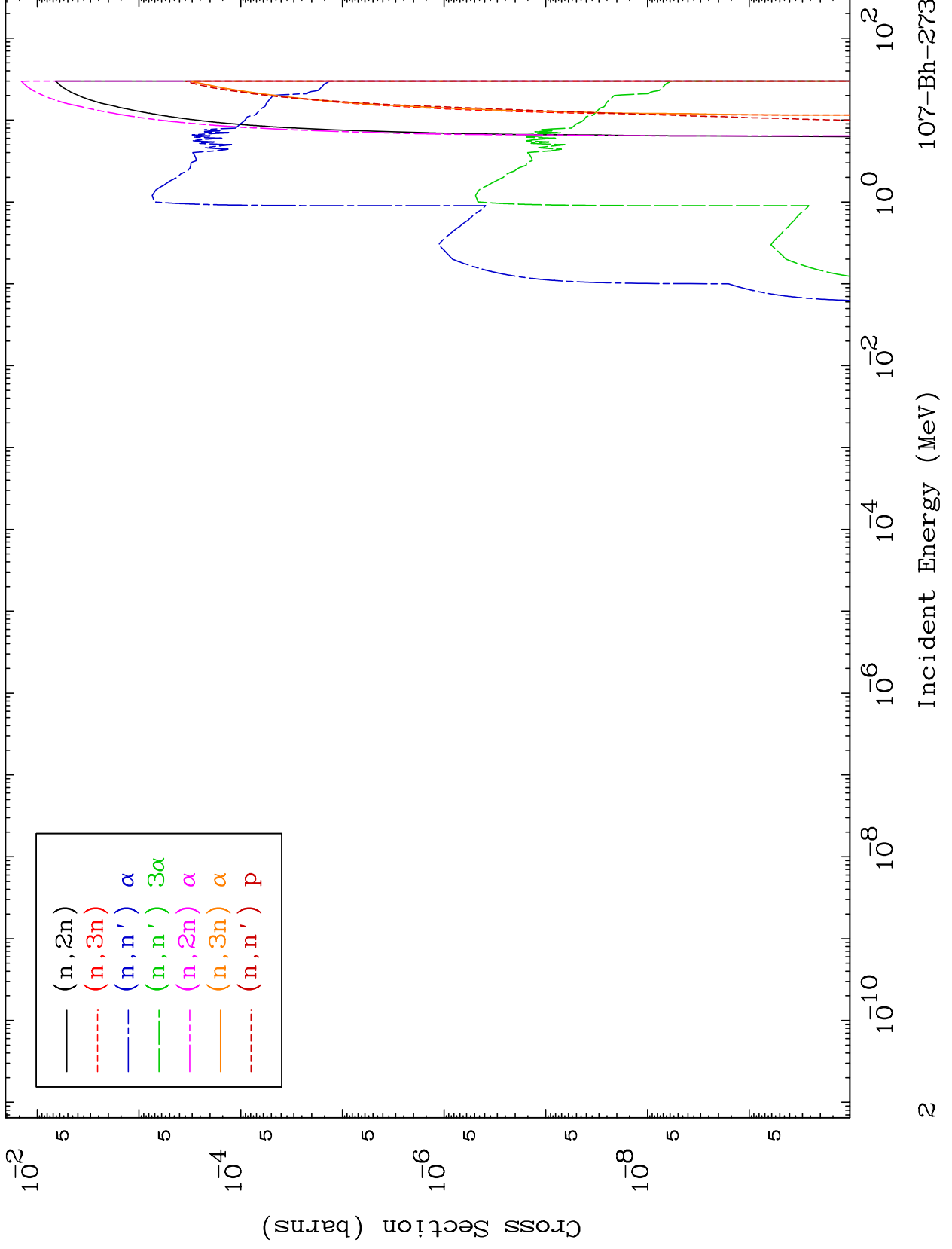
107-Bh-273

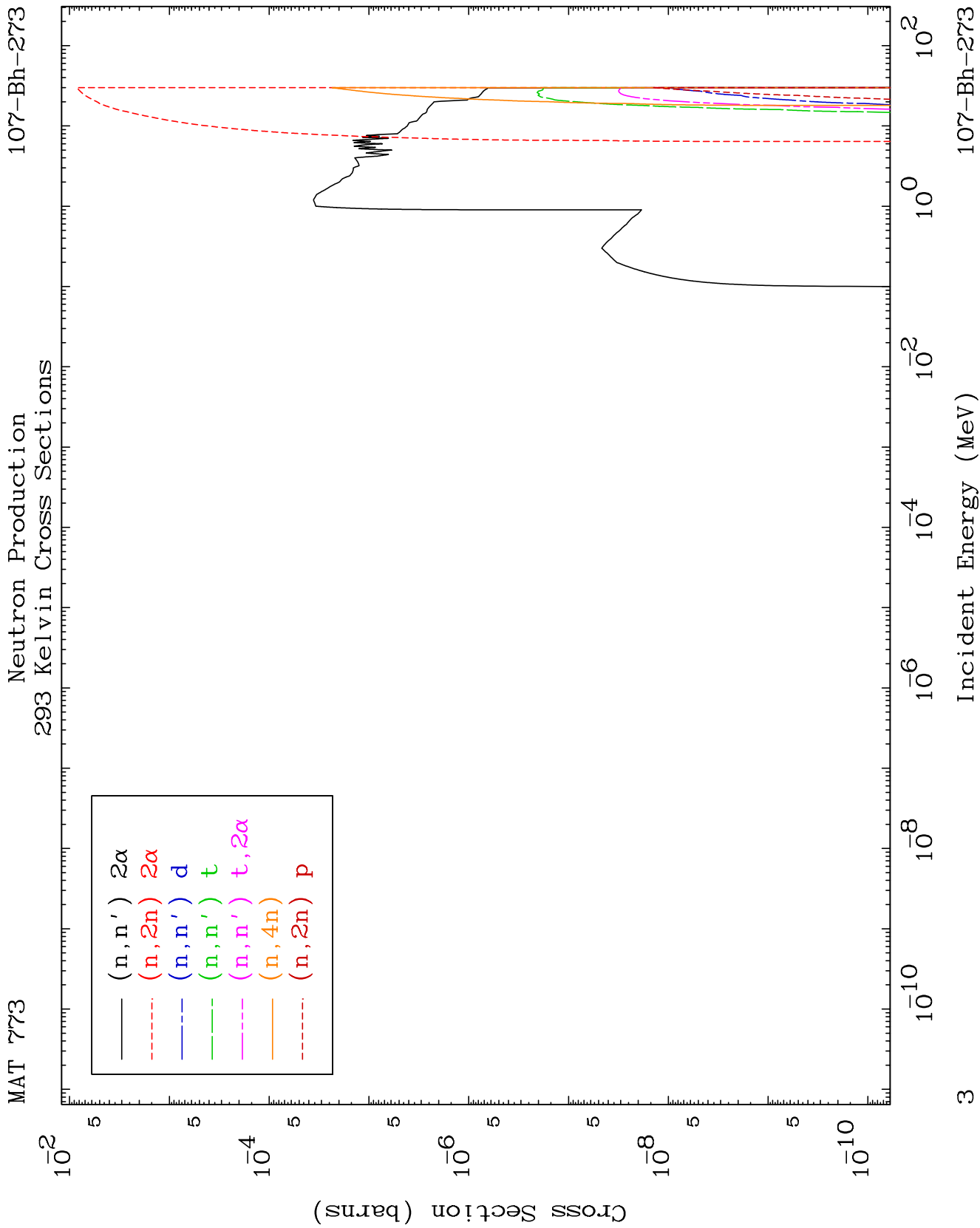


MAT 773

Neutron Production
293 Kelvin Cross Sections

107-Bh-273

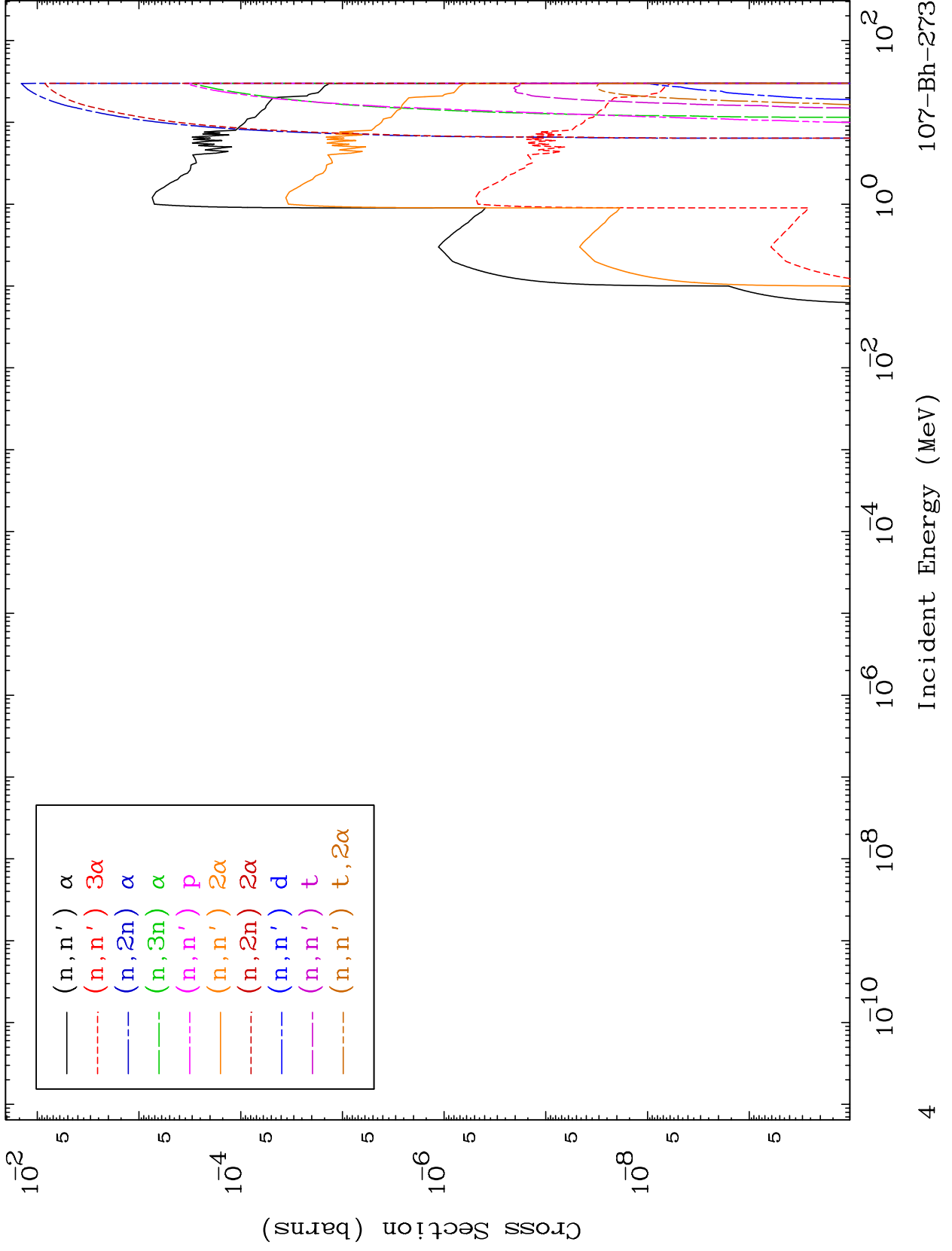


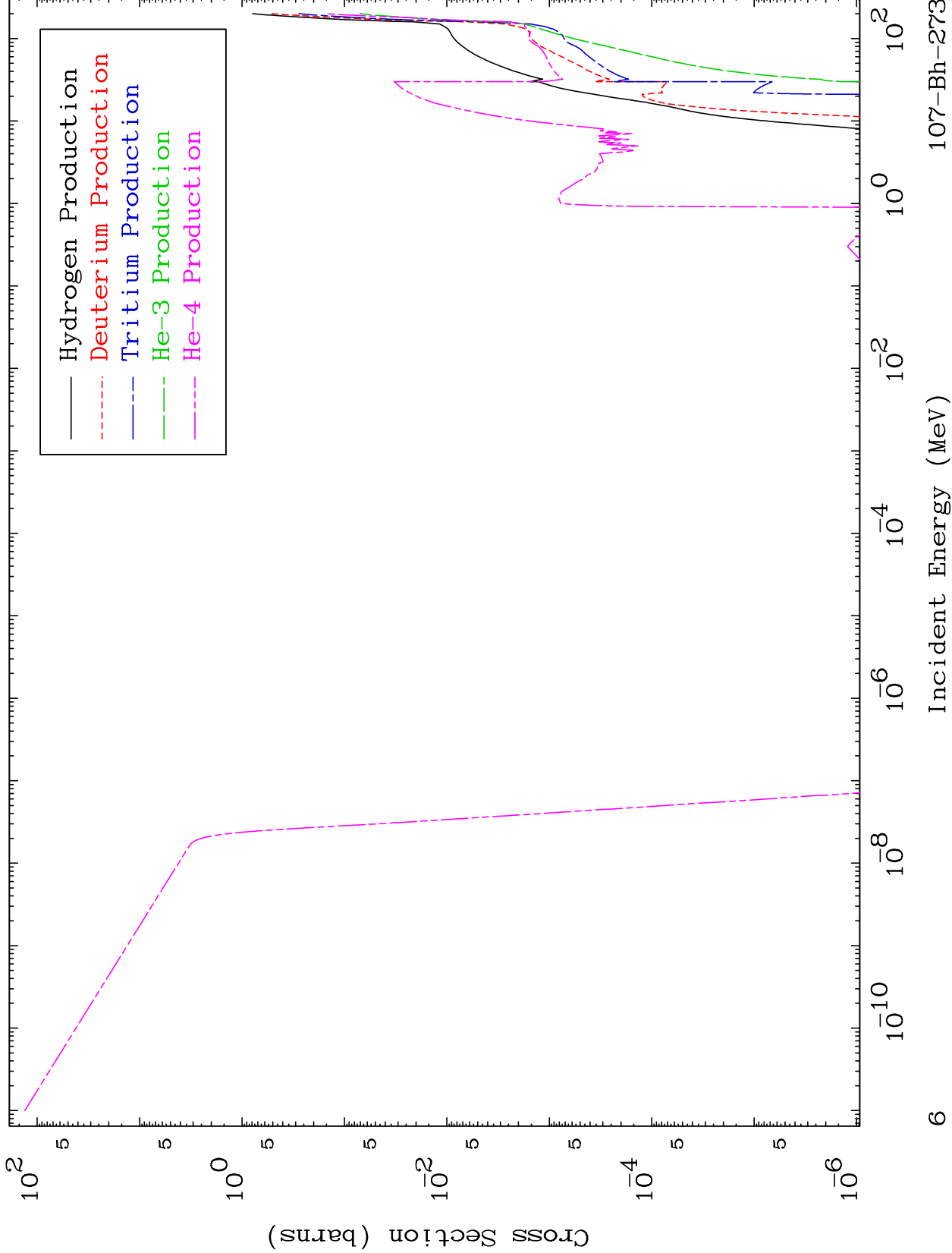


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Charged Particle
293 Kelvin Cross Sections

107-Bh-273

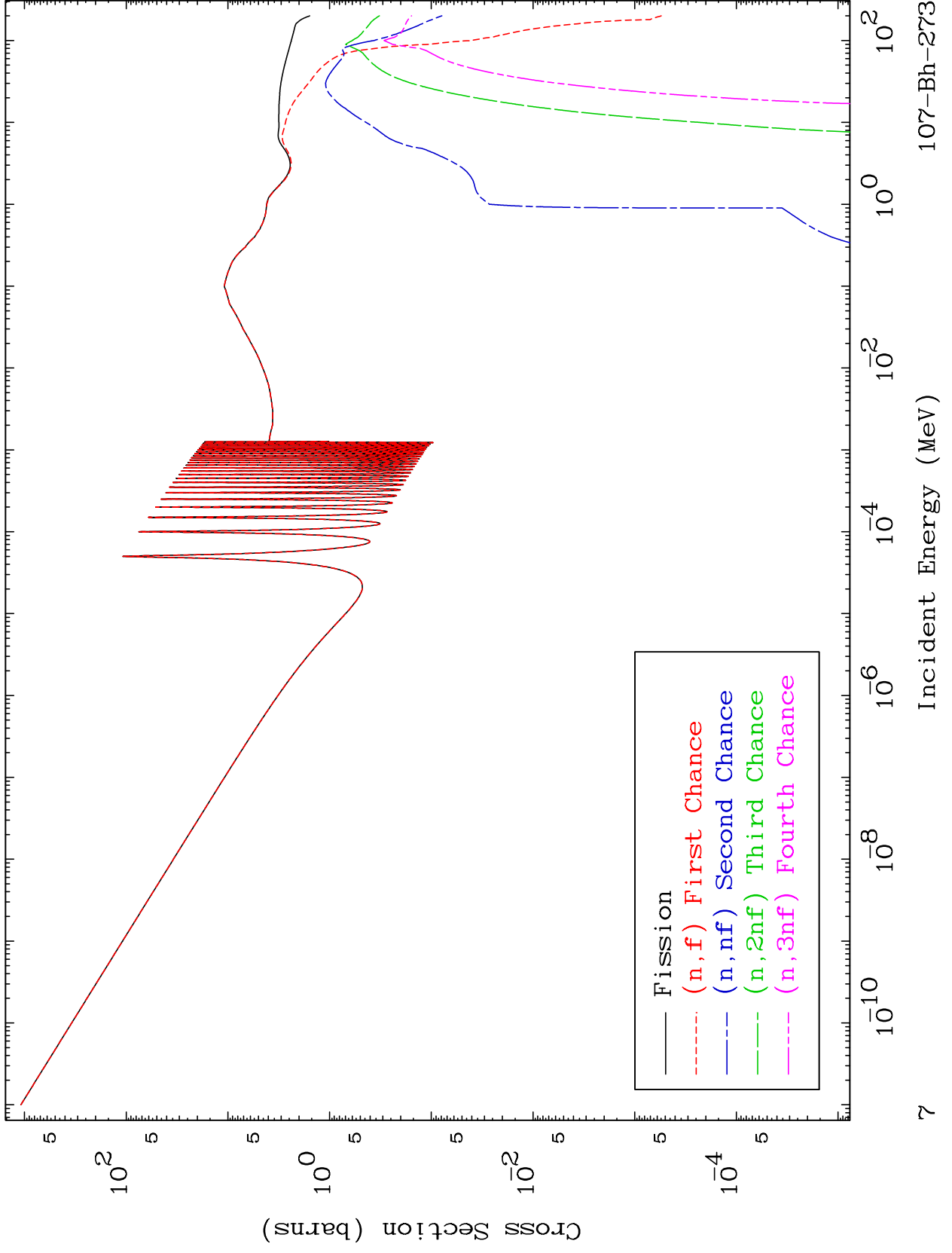




MAT 773

Fission
293 Kelvin Cross Sections

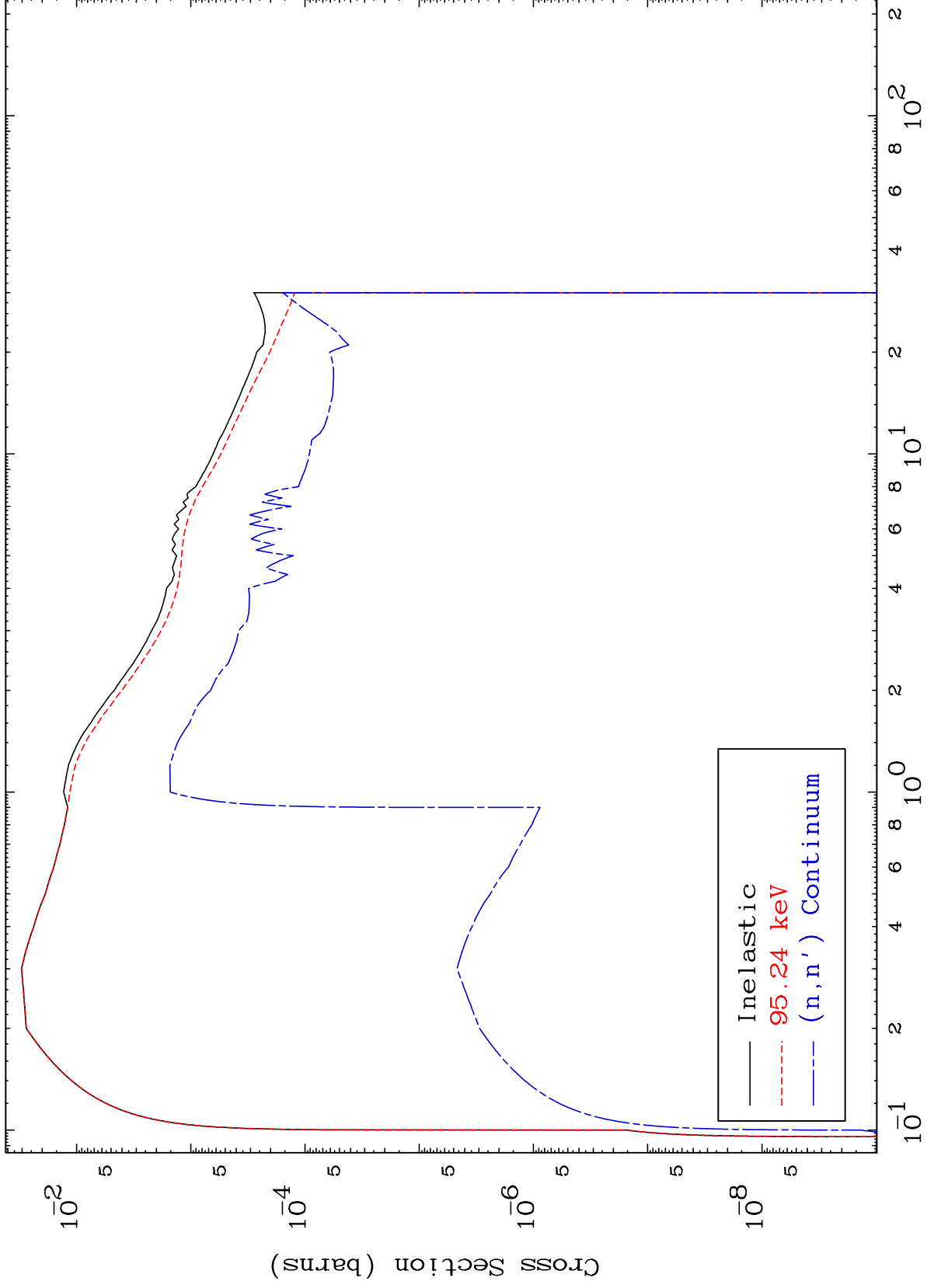
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MAT 773

(n,n') Level
293 Kelvin Cross Sections

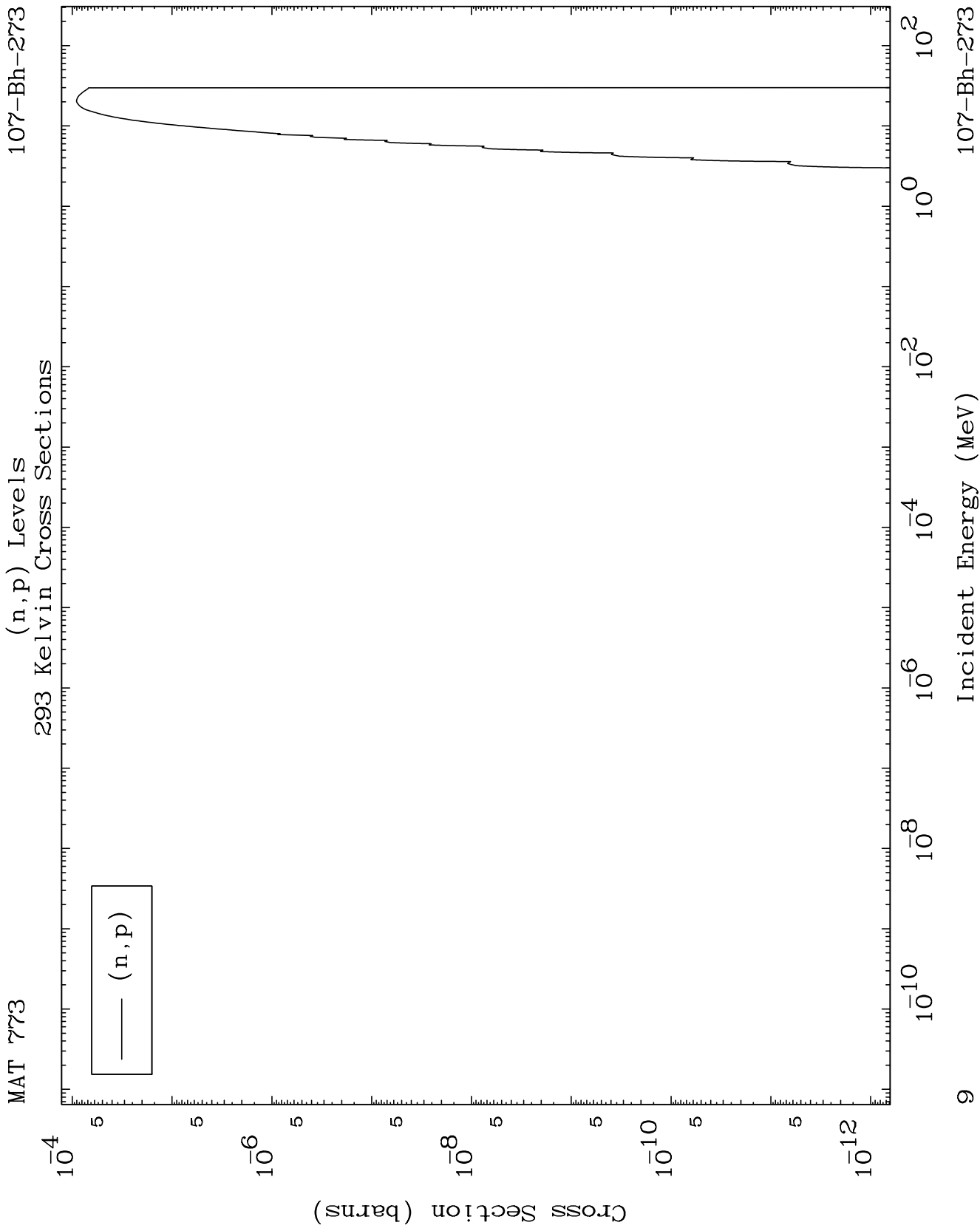
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107-Bh-273

107-Bh-273

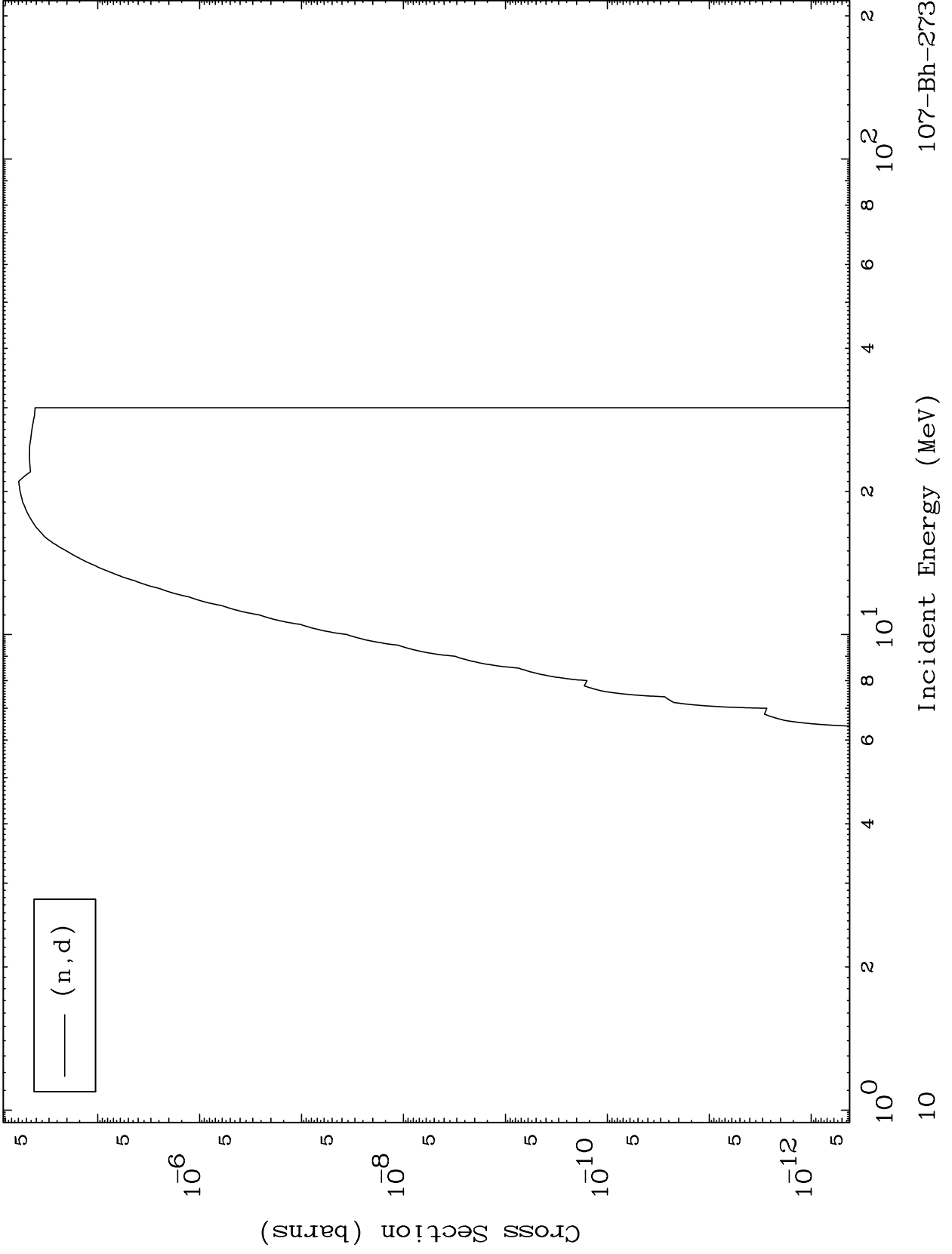
8



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(n,d) Levels
293 Kelvin Cross Sections

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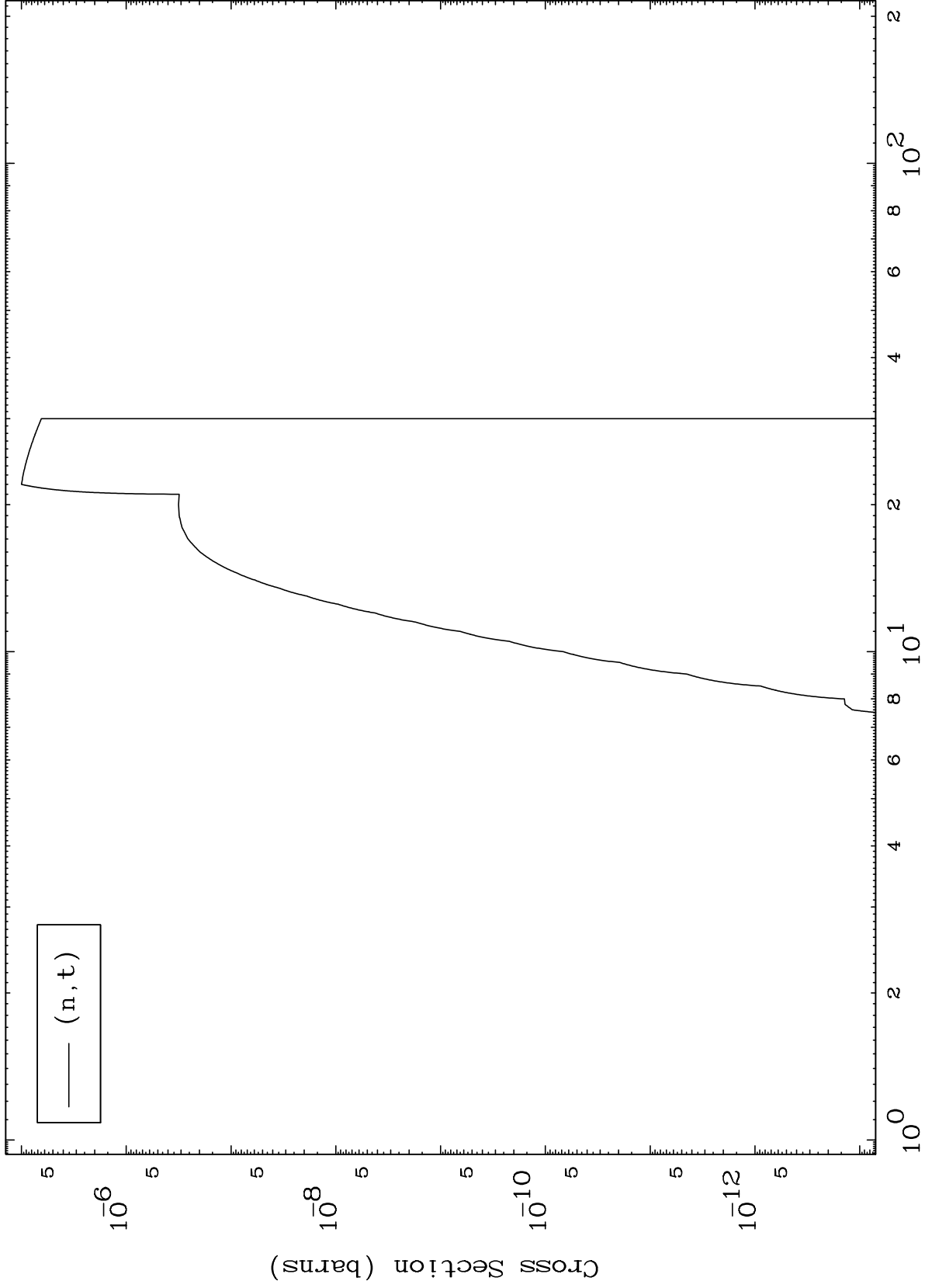


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(n,t) Levels
293 Kelvin Cross Sections

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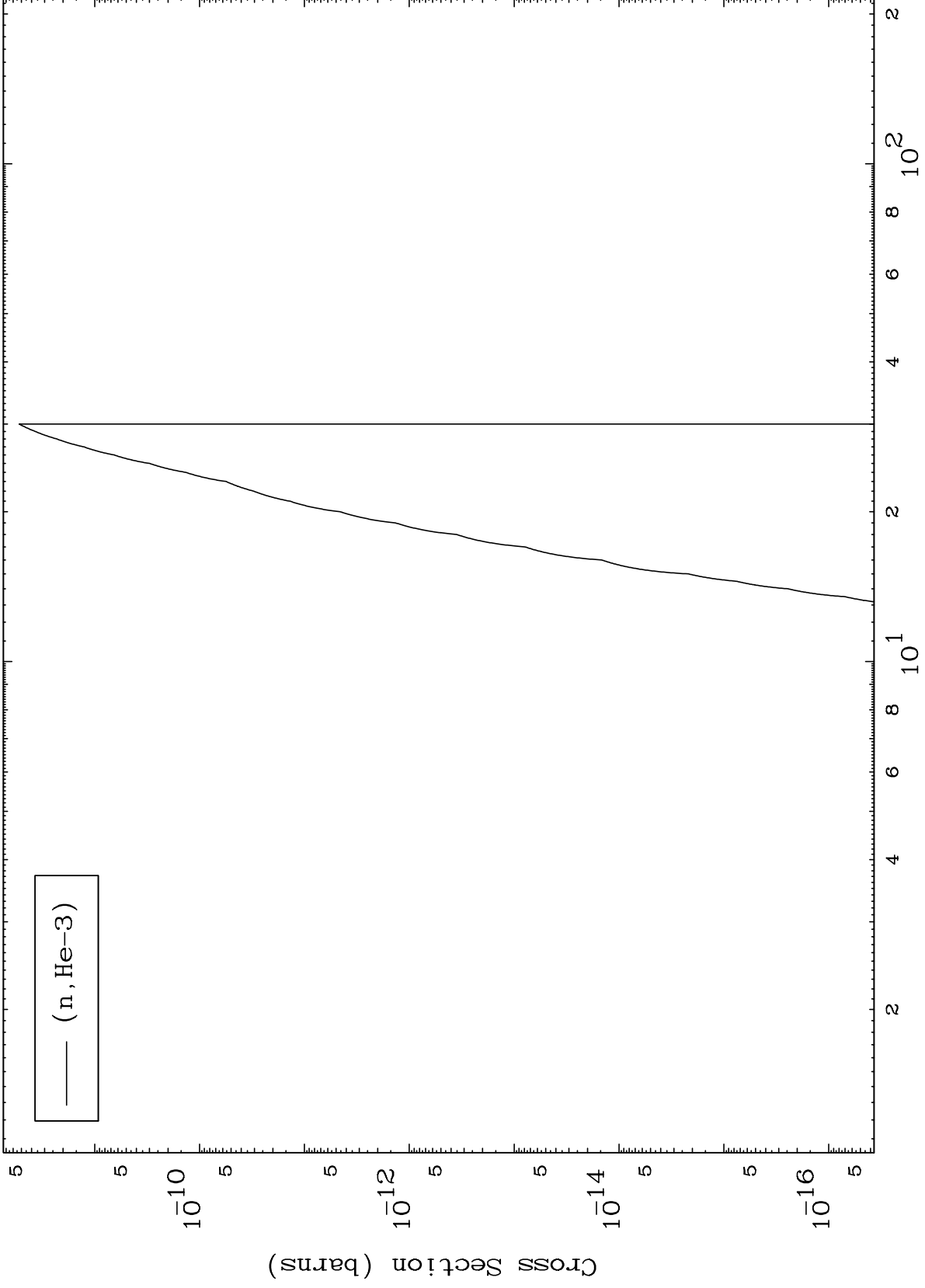
Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

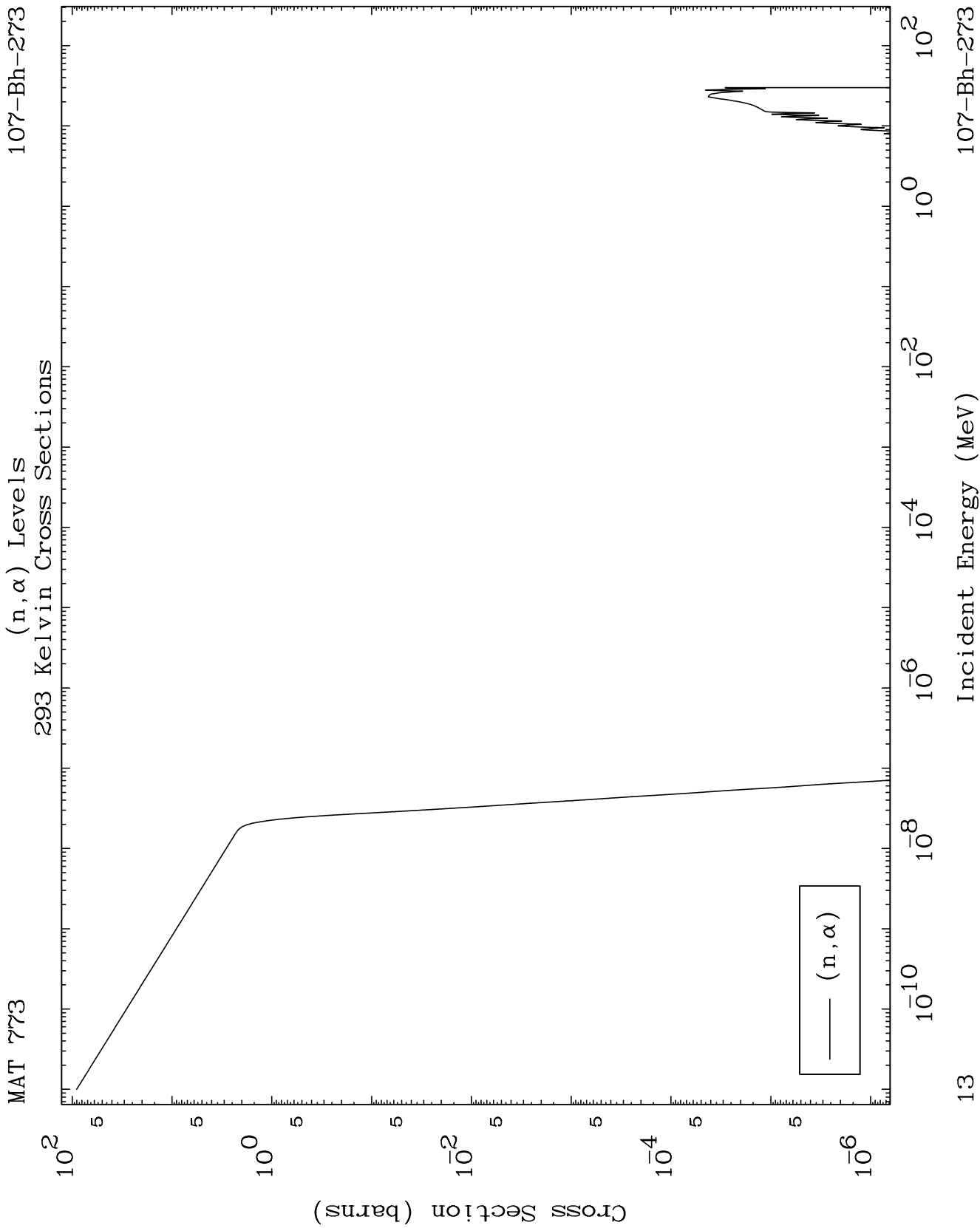
107-Bh-273

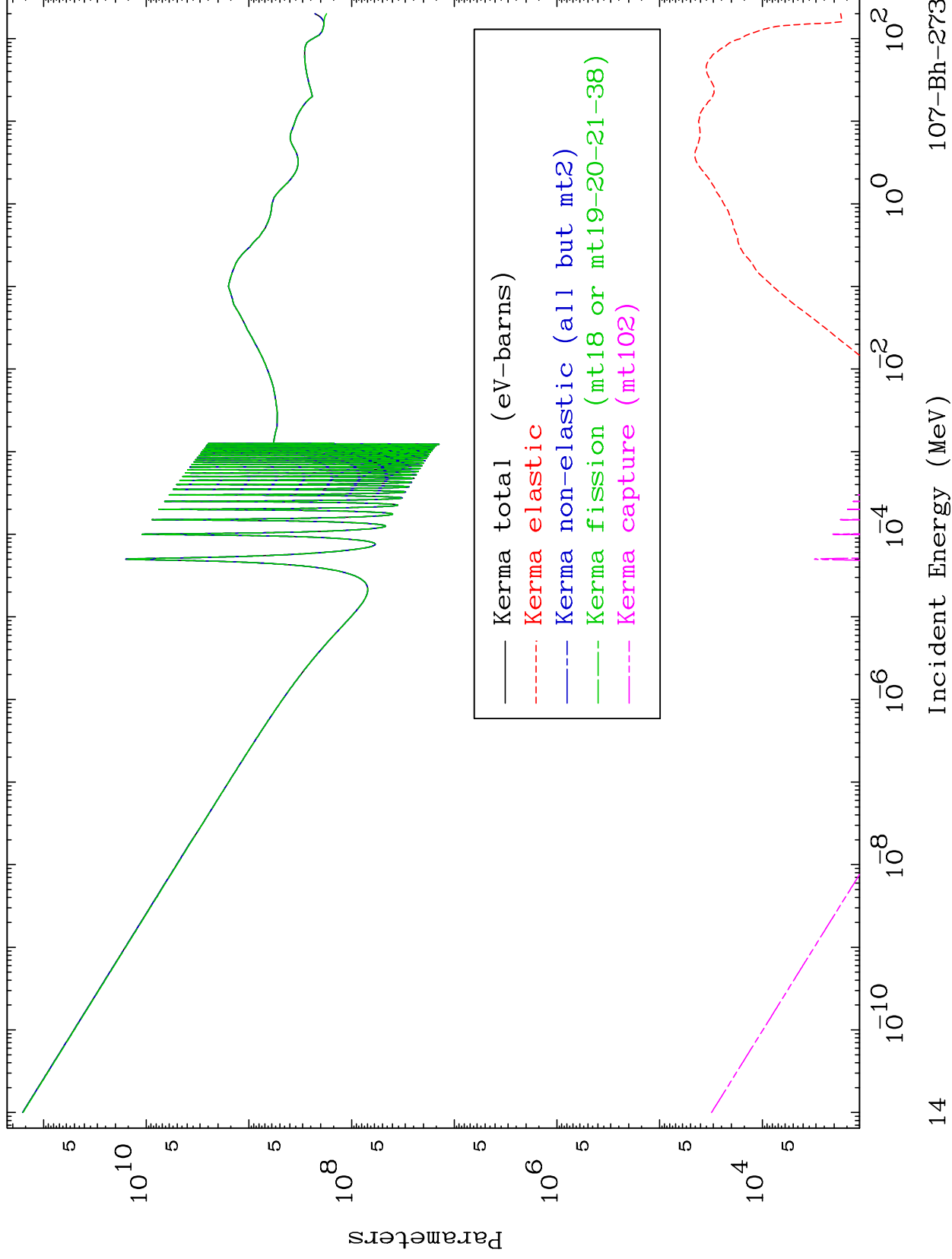


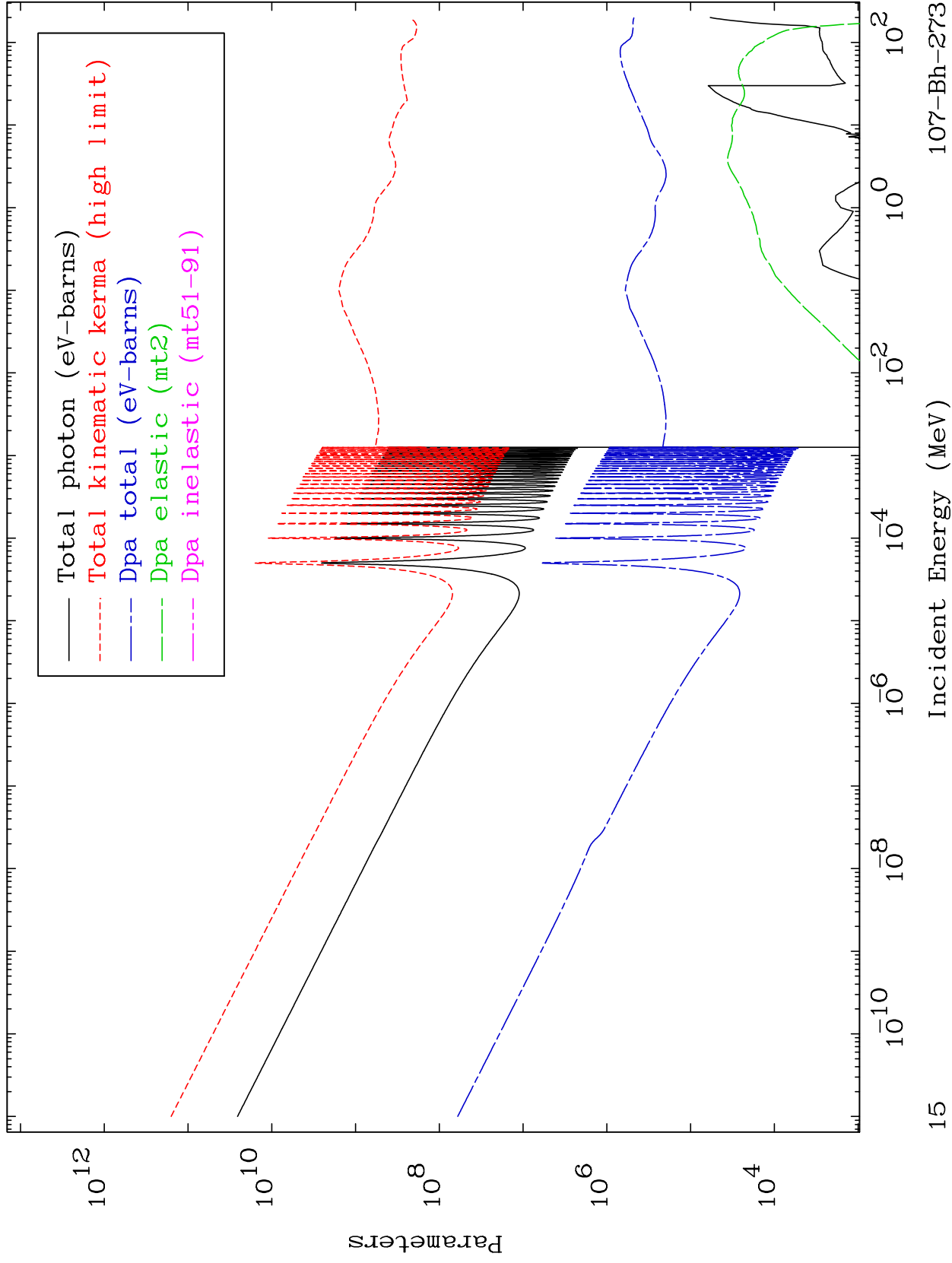
12

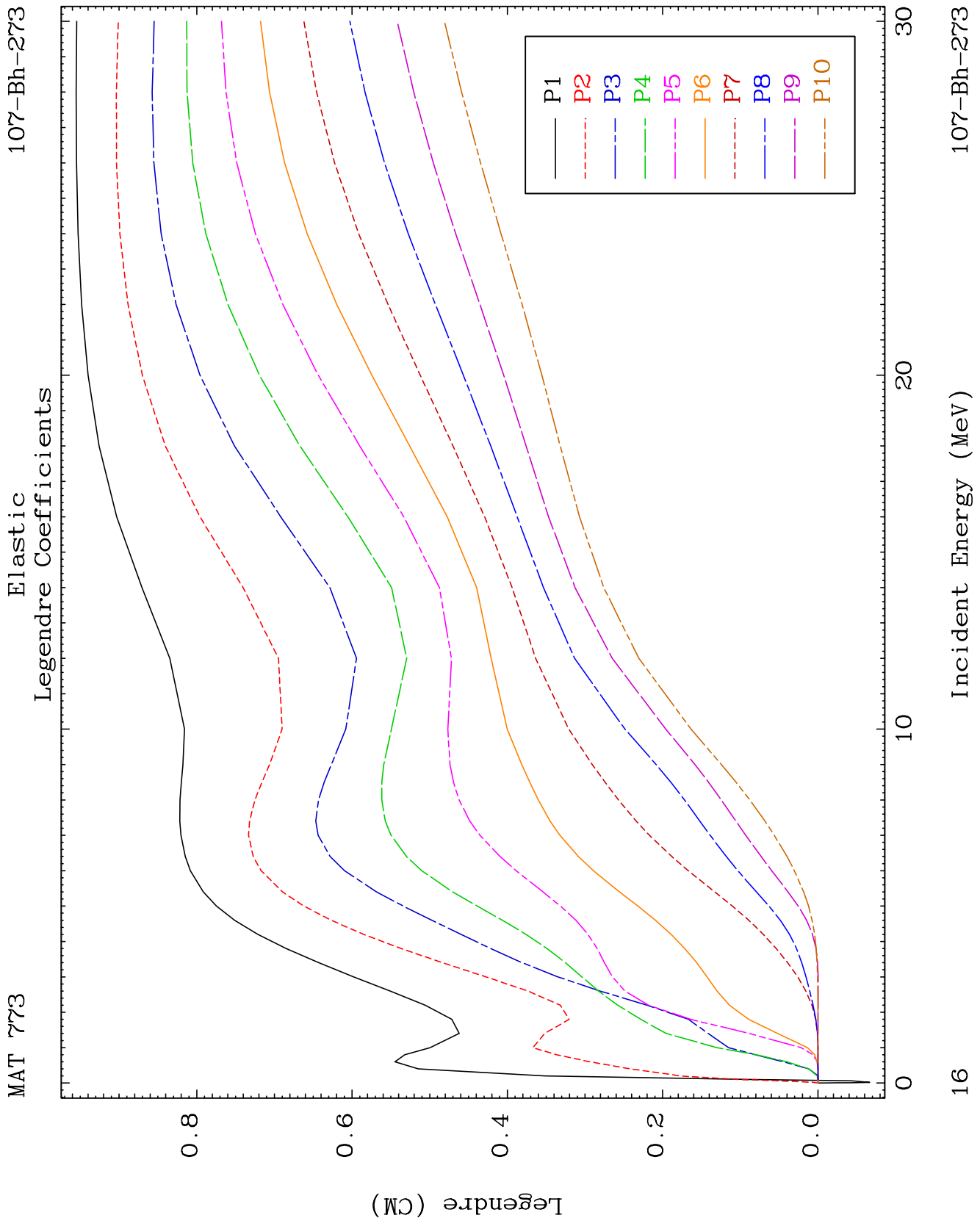
Incident Energy (MeV)

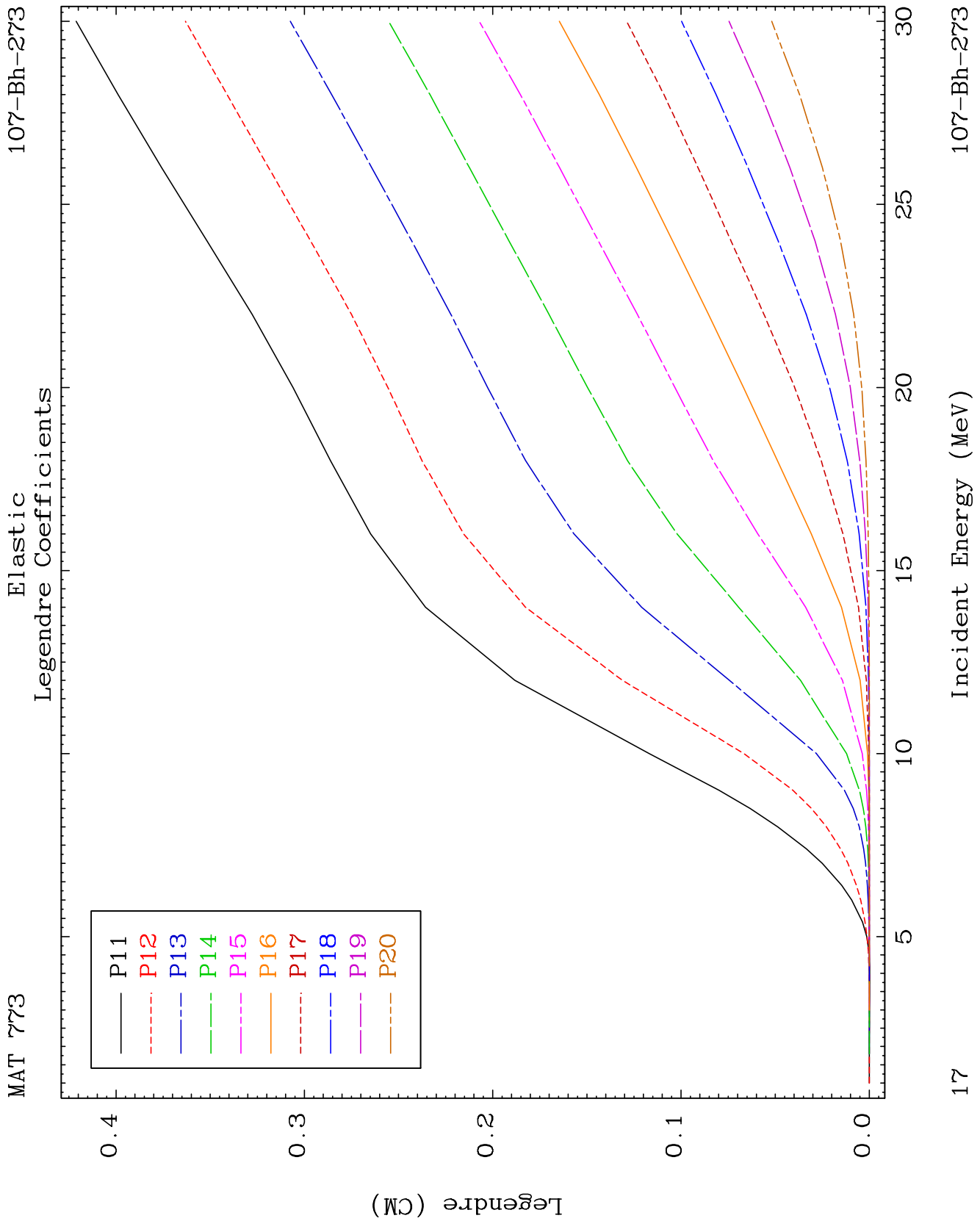
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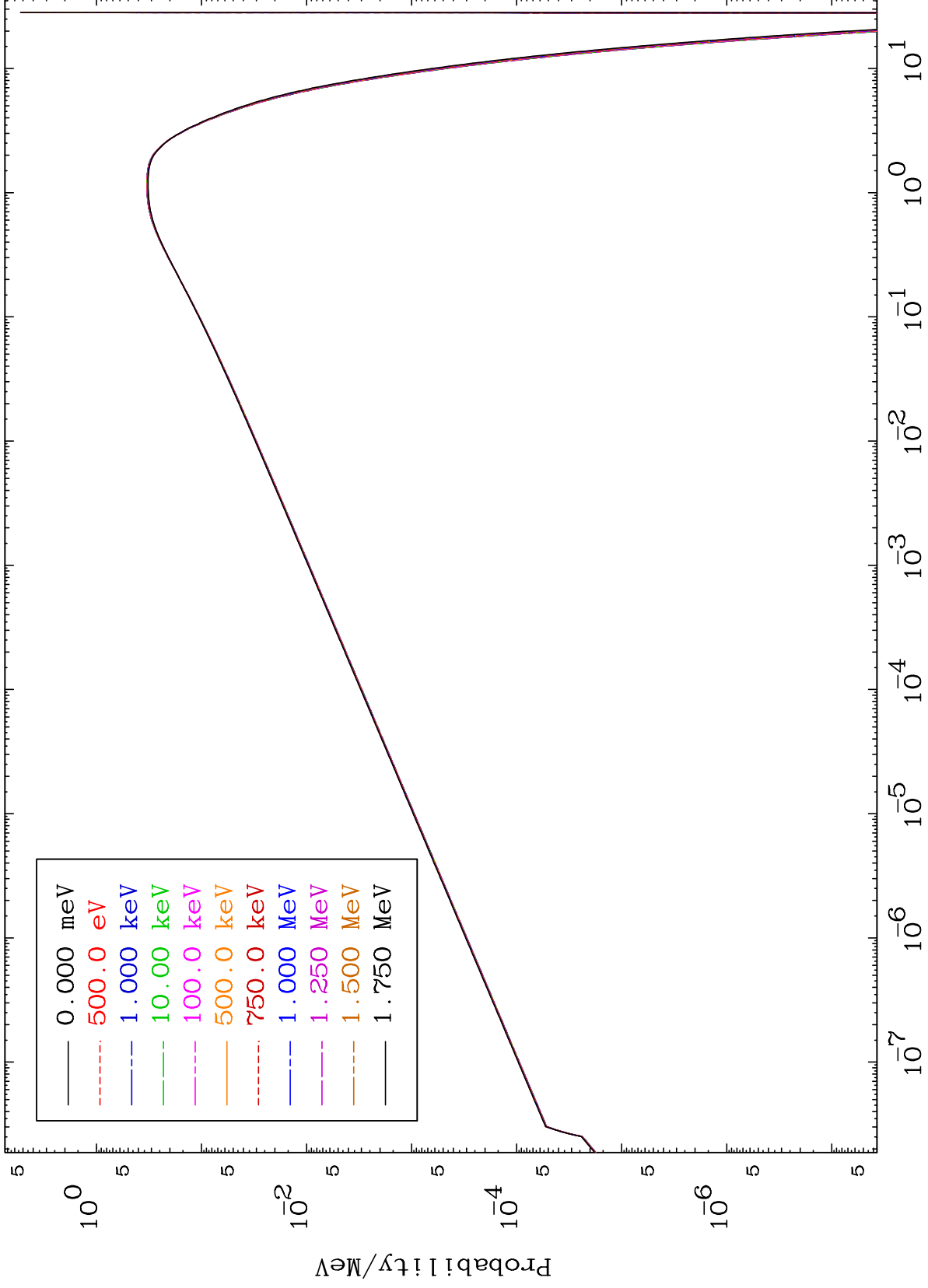








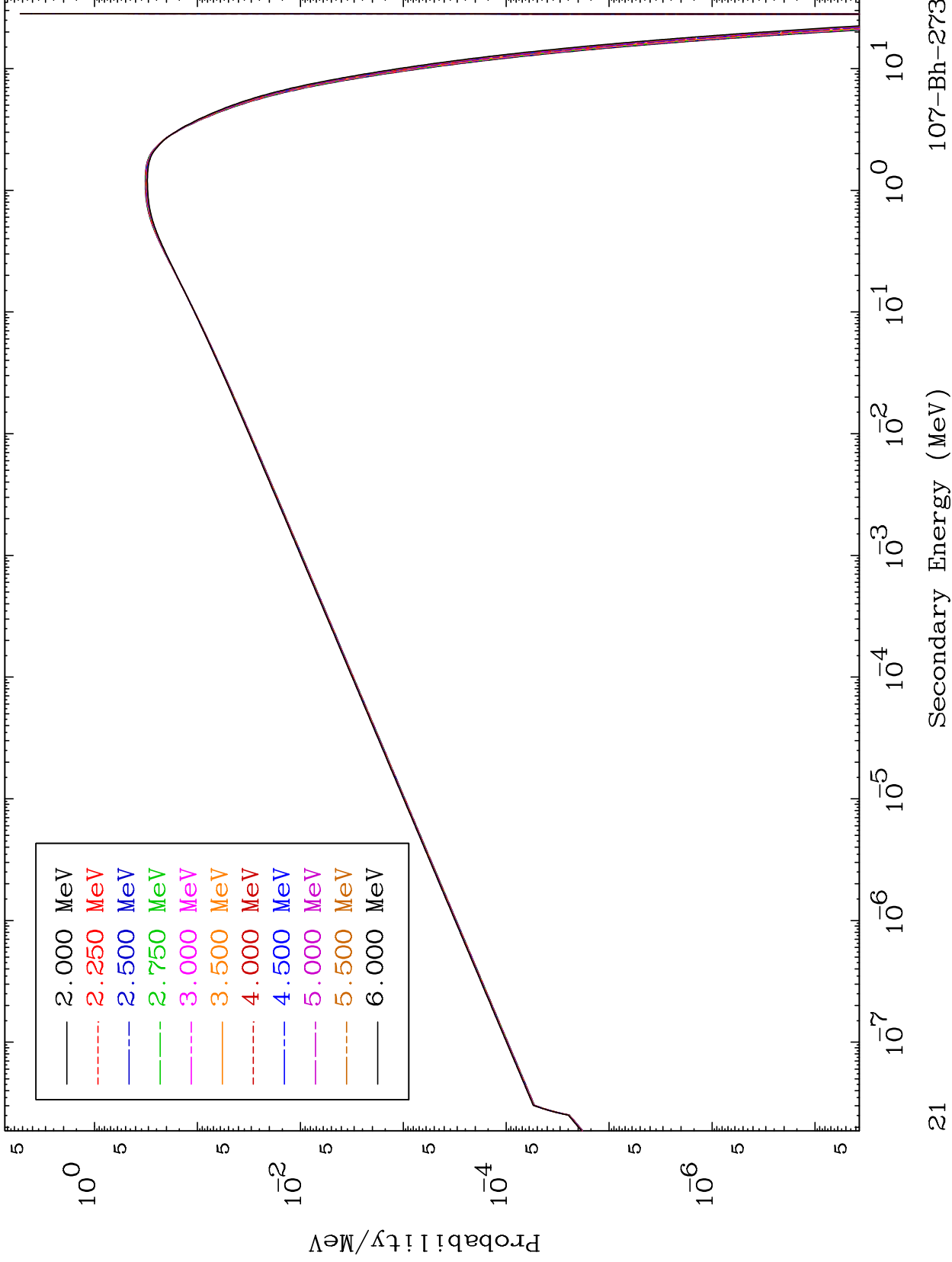




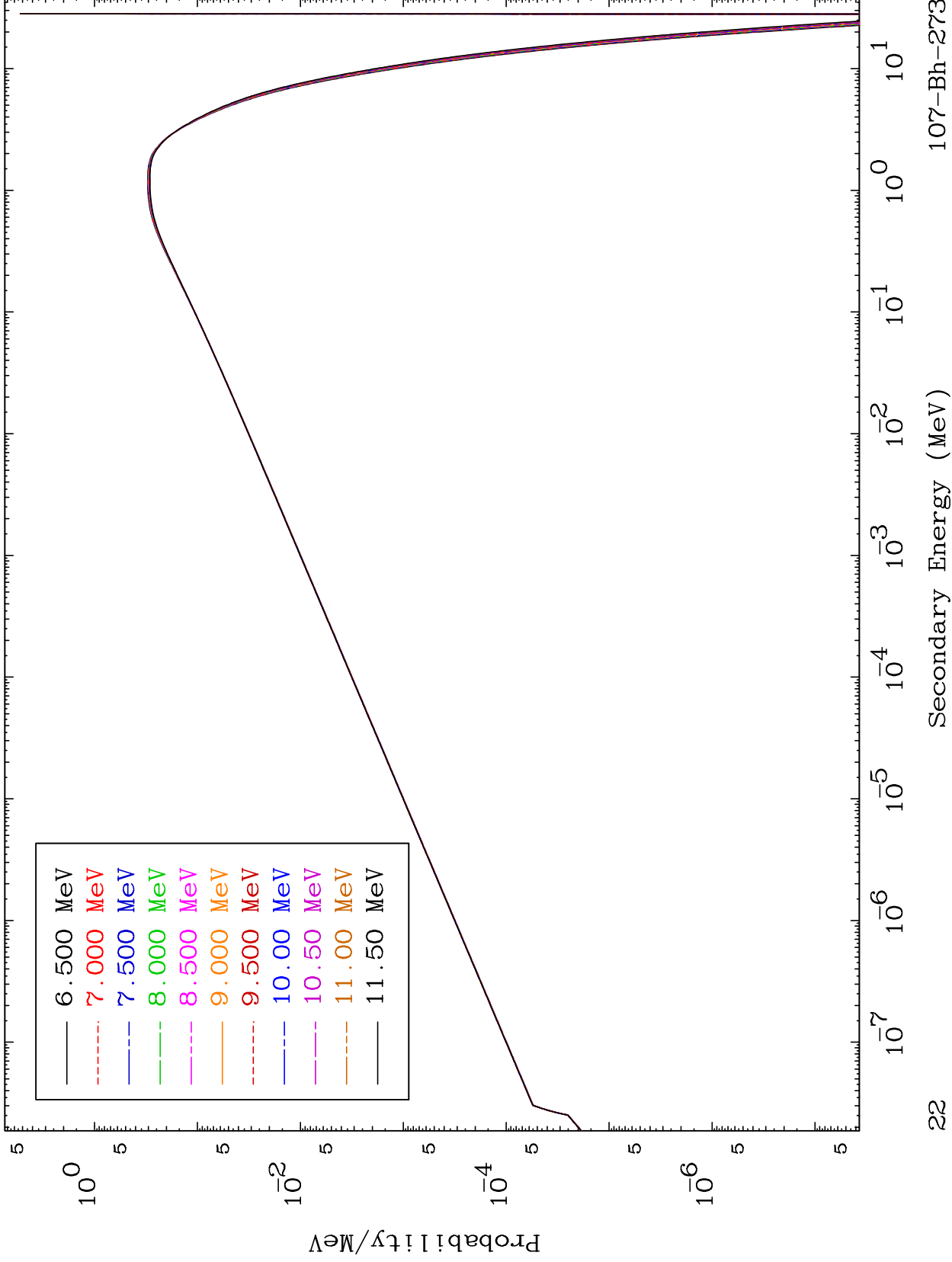
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Fission Energy Distribution

107-Bh-273



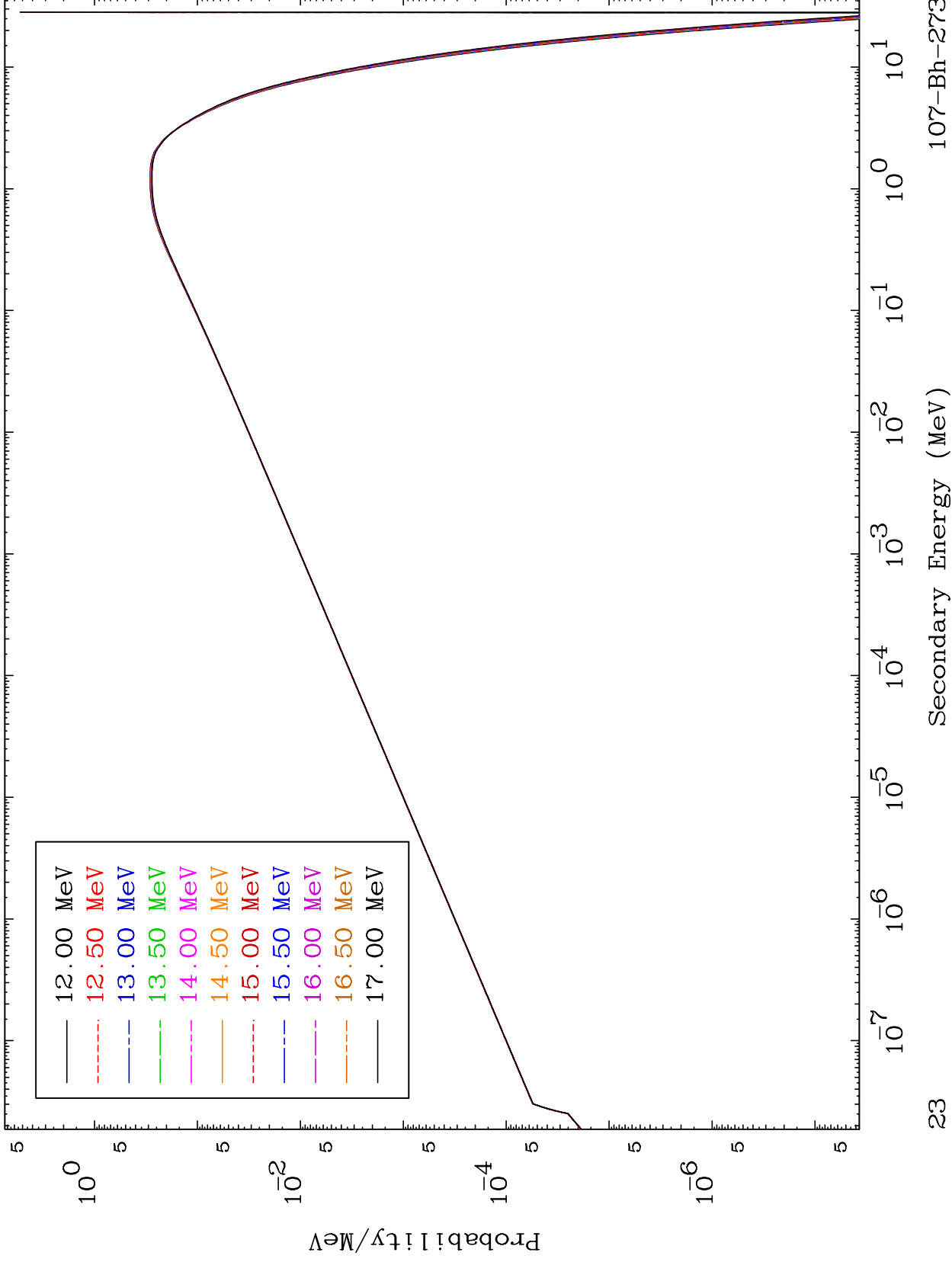
107-Bh-273



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Fission Energy Distribution

107-Bh-273

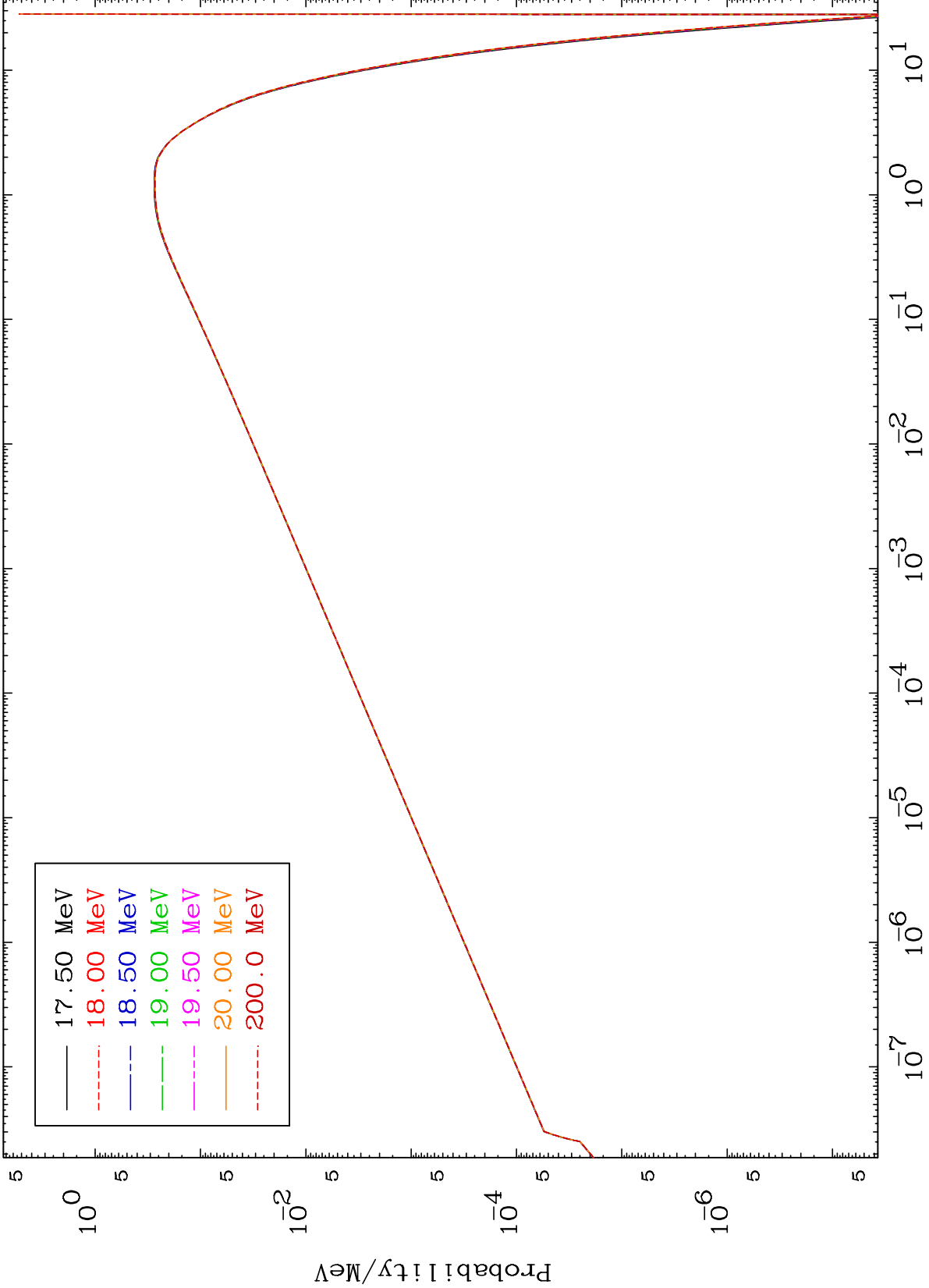


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Fission Energy Distribution

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Secondary Energy (MeV)