

Program EVALPLOT
(Version 2015-2)

by

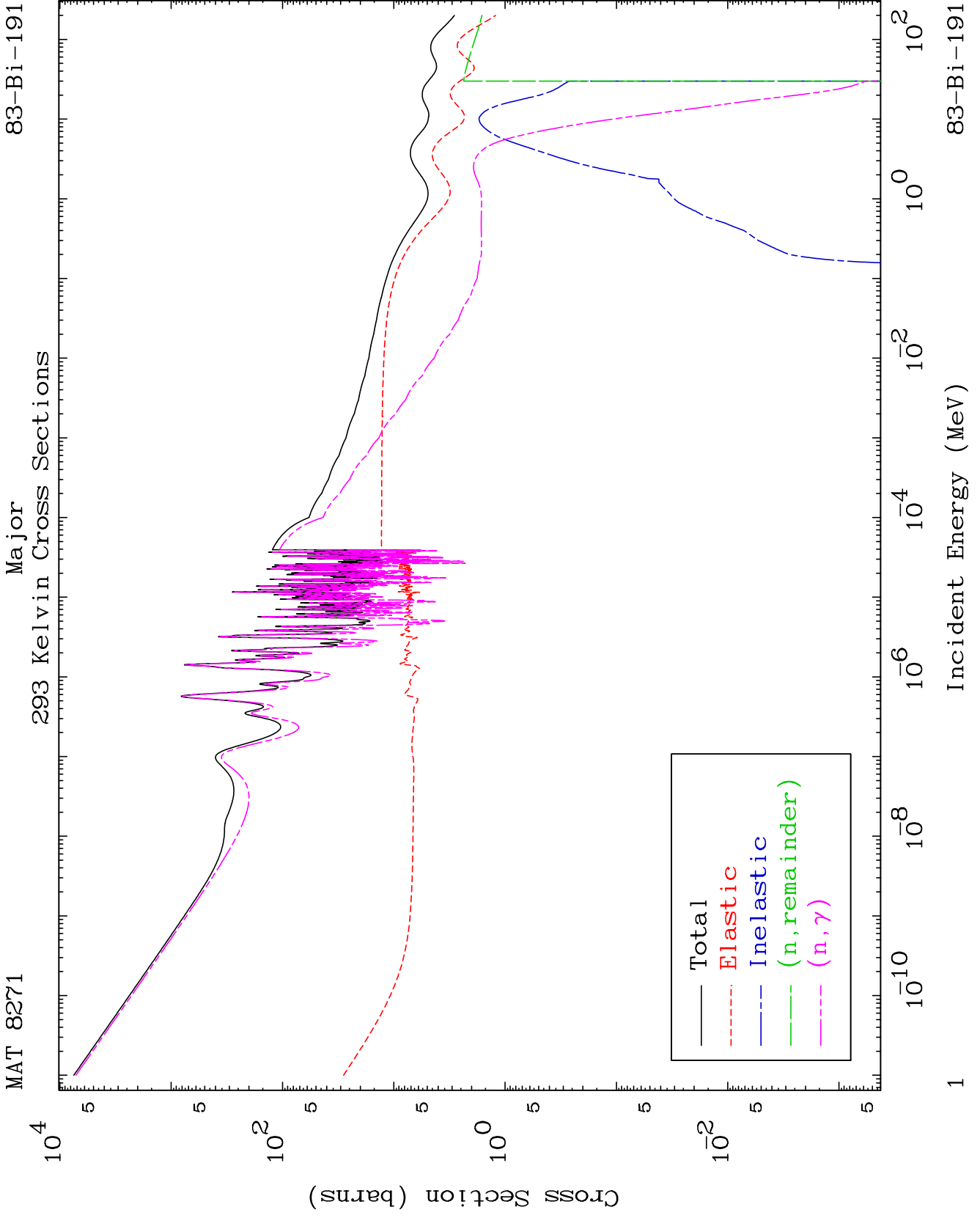
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(Present Contact Information)

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E.Mail:redcullen1@comcast.net
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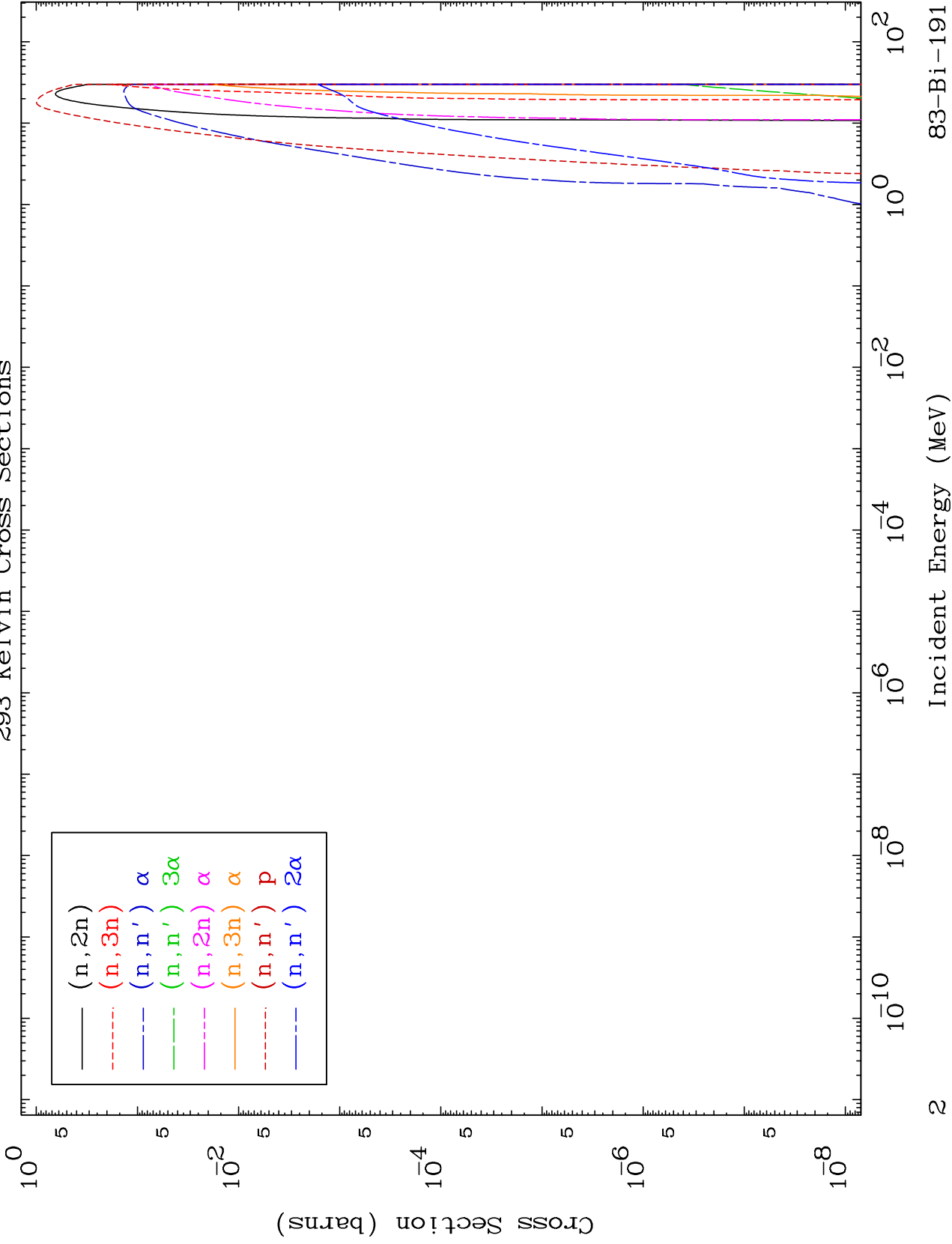
Press Mouse Button to Start



MAT 8271

Neutron Production
293 Kelvin Cross Sections

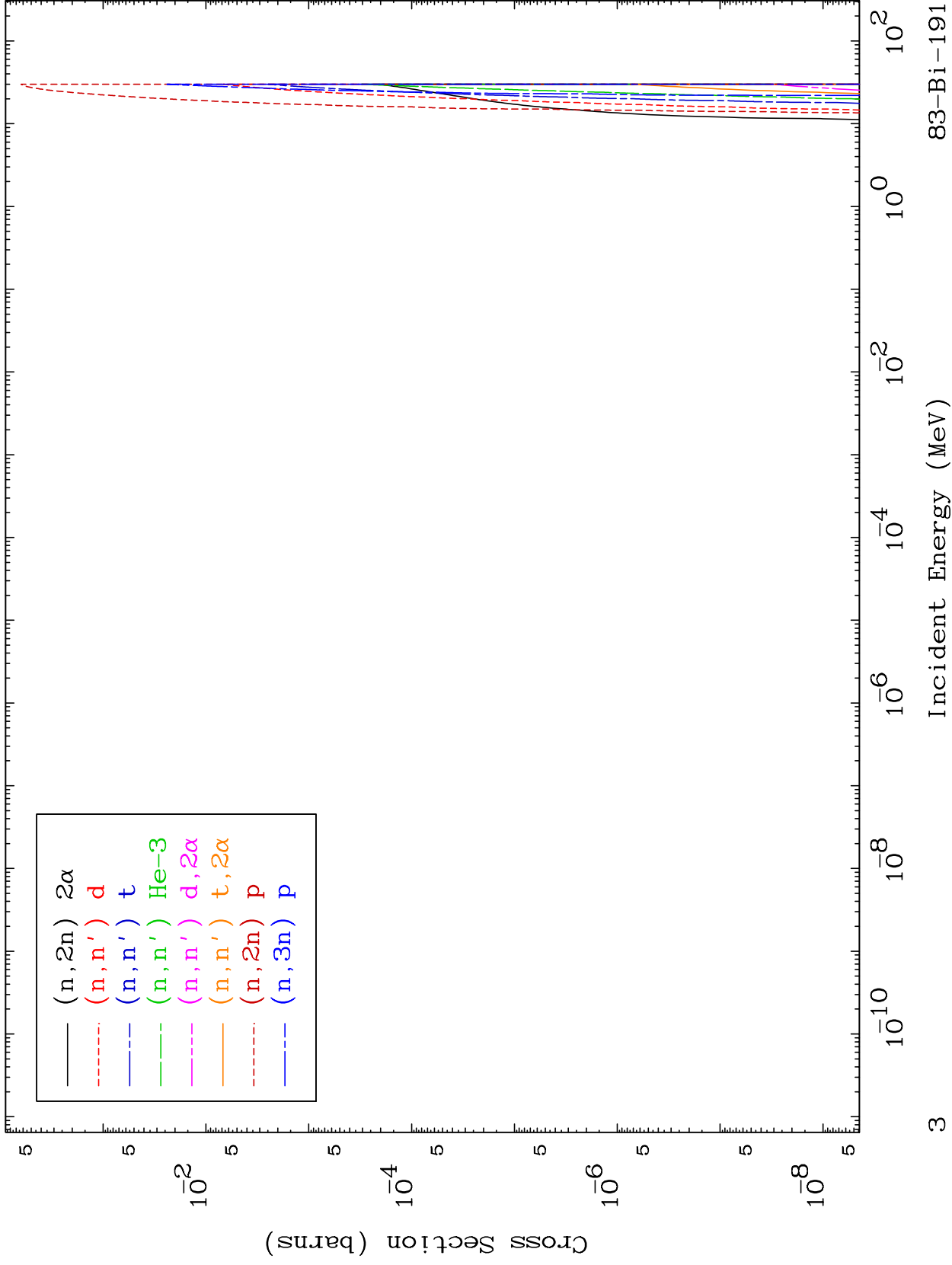
83-Bi-191



MAT 8271

Neutron Production
293 Kelvin Cross Sections

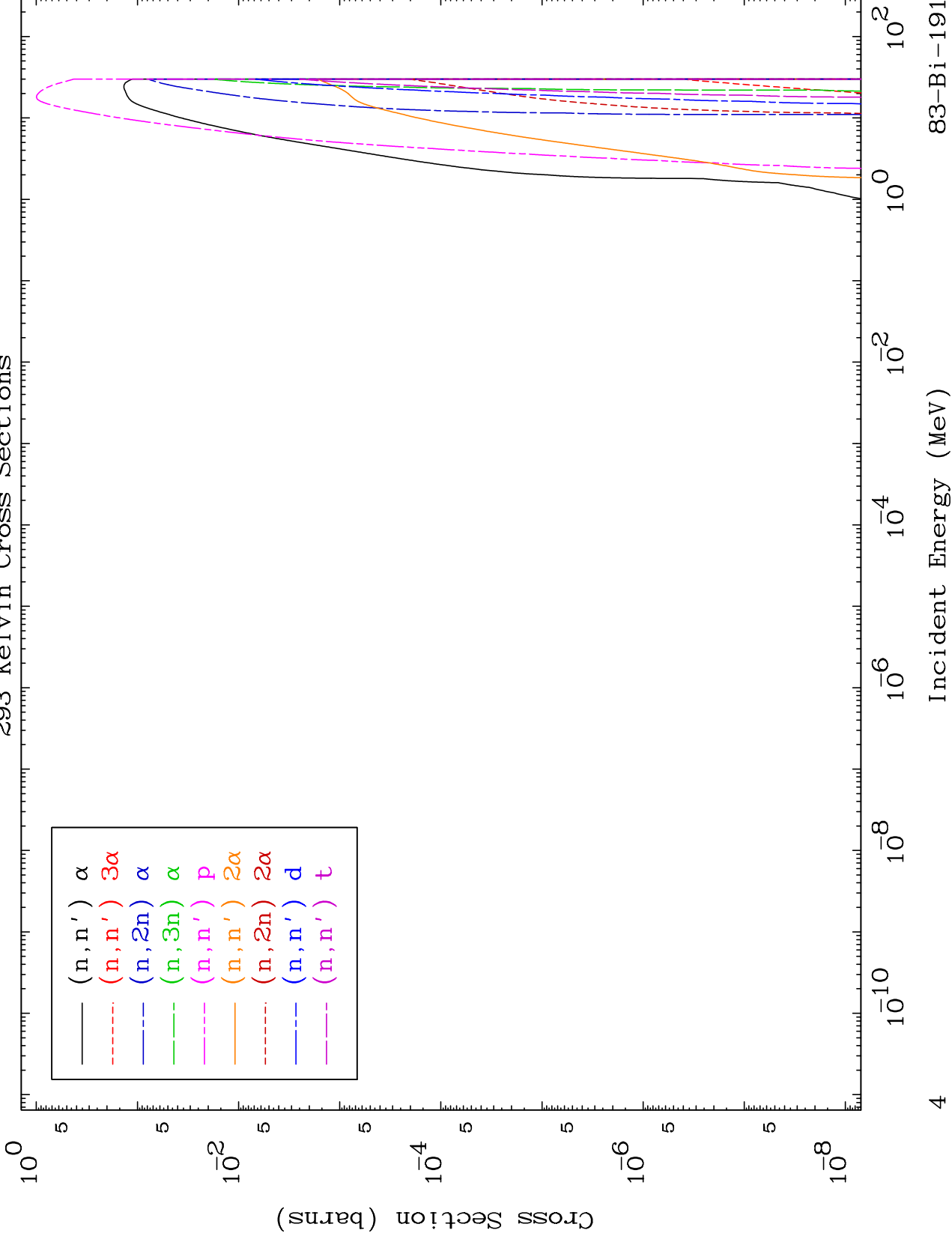
83-Bi-191



MAT 8271

Charged Particle
293 Kelvin Cross Sections

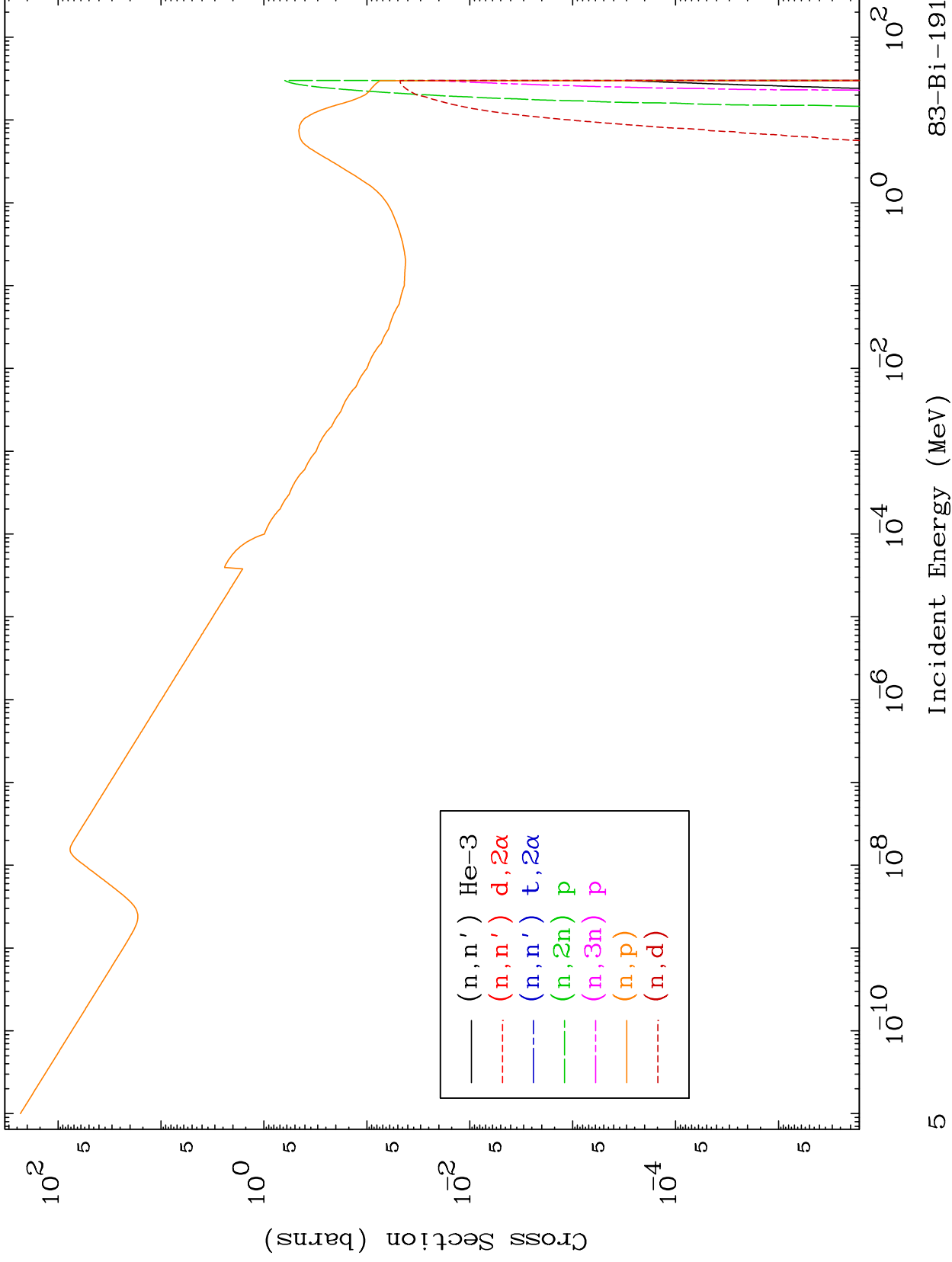
83-Bi-191



MAT 8271

Charged Particle
293 Kelvin Cross Sections

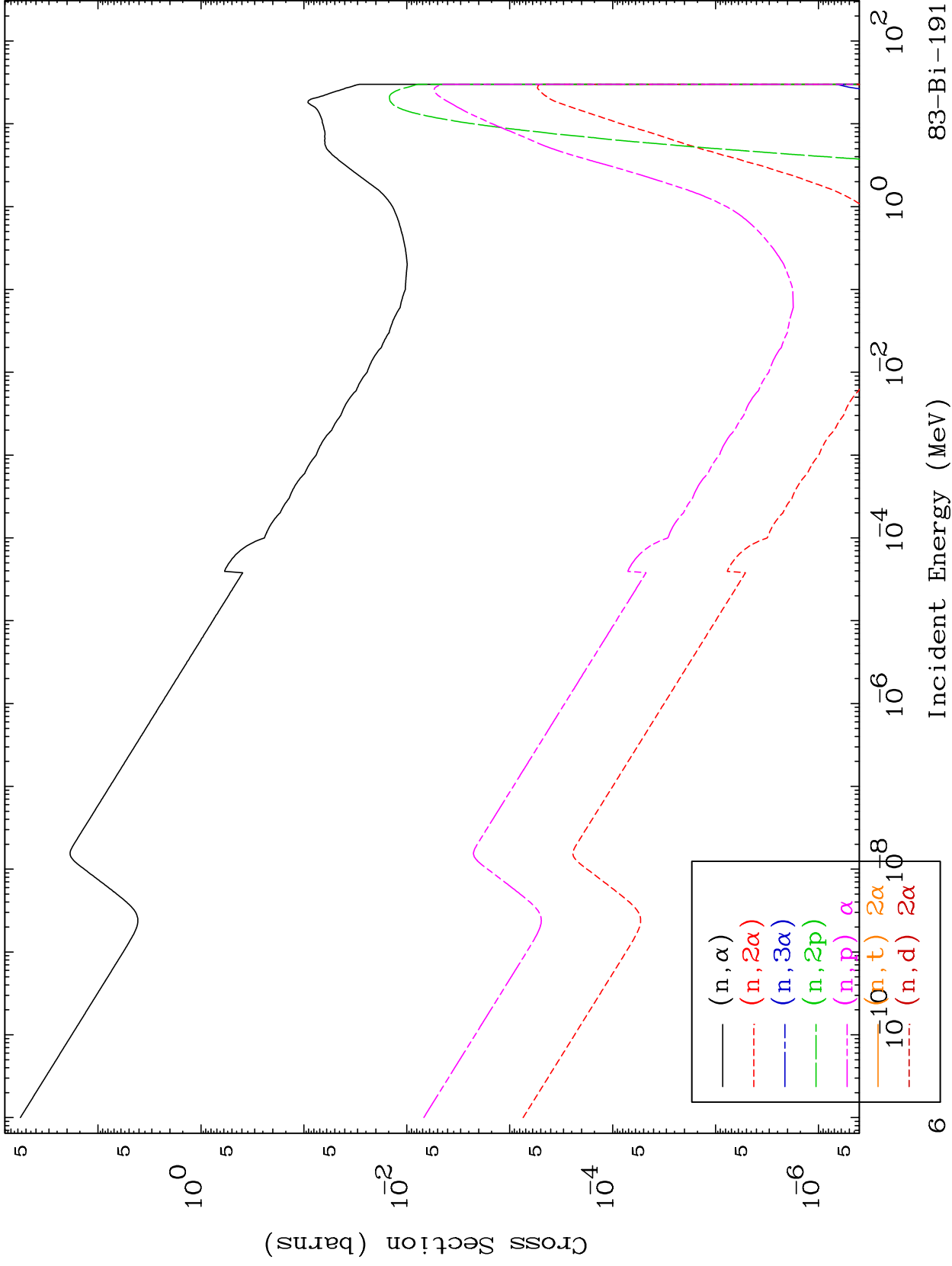
83-Bi-191



MAT 8271

Charged Particle
293 Kelvin Cross Sections

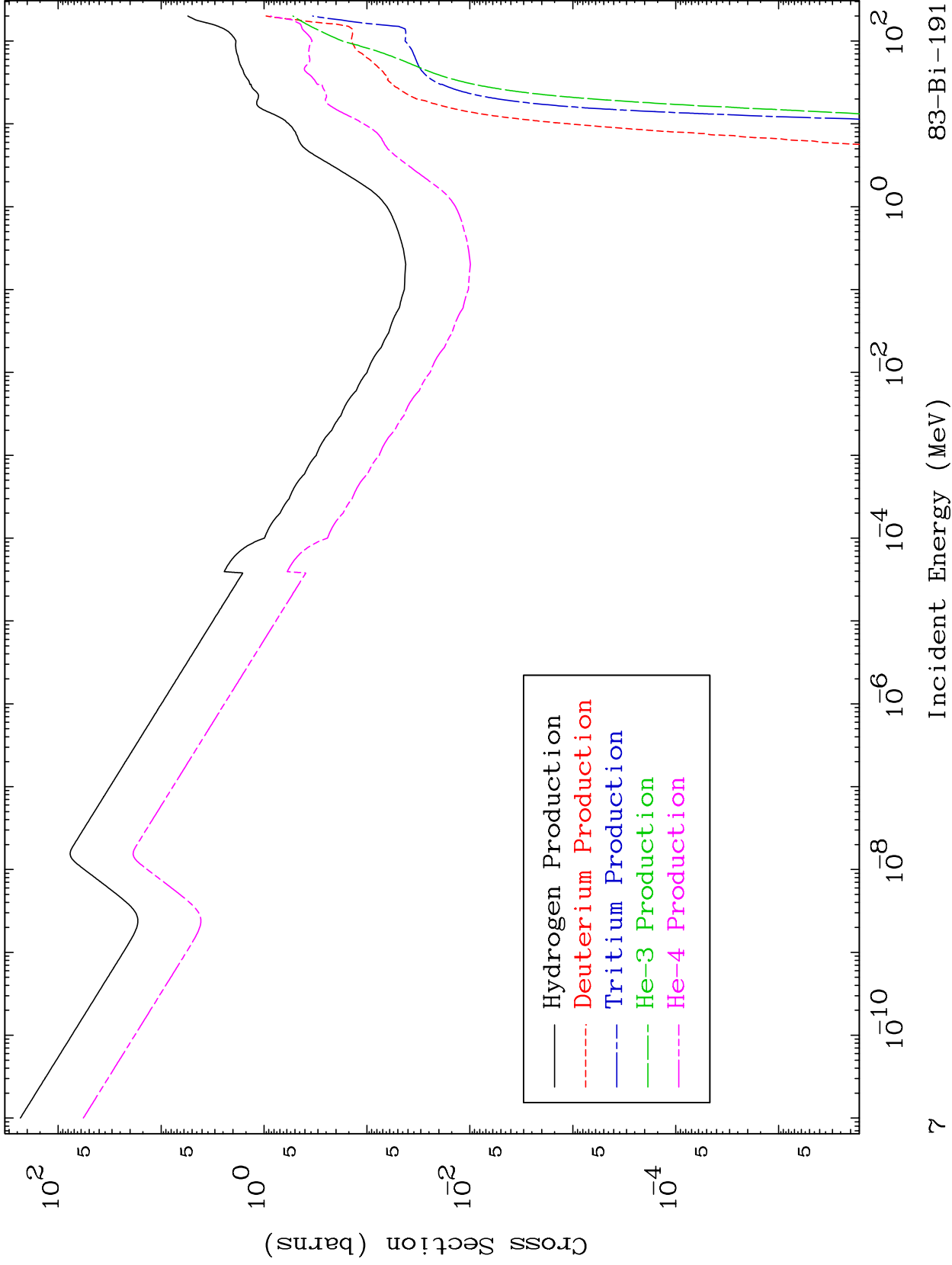
83-Bi-191



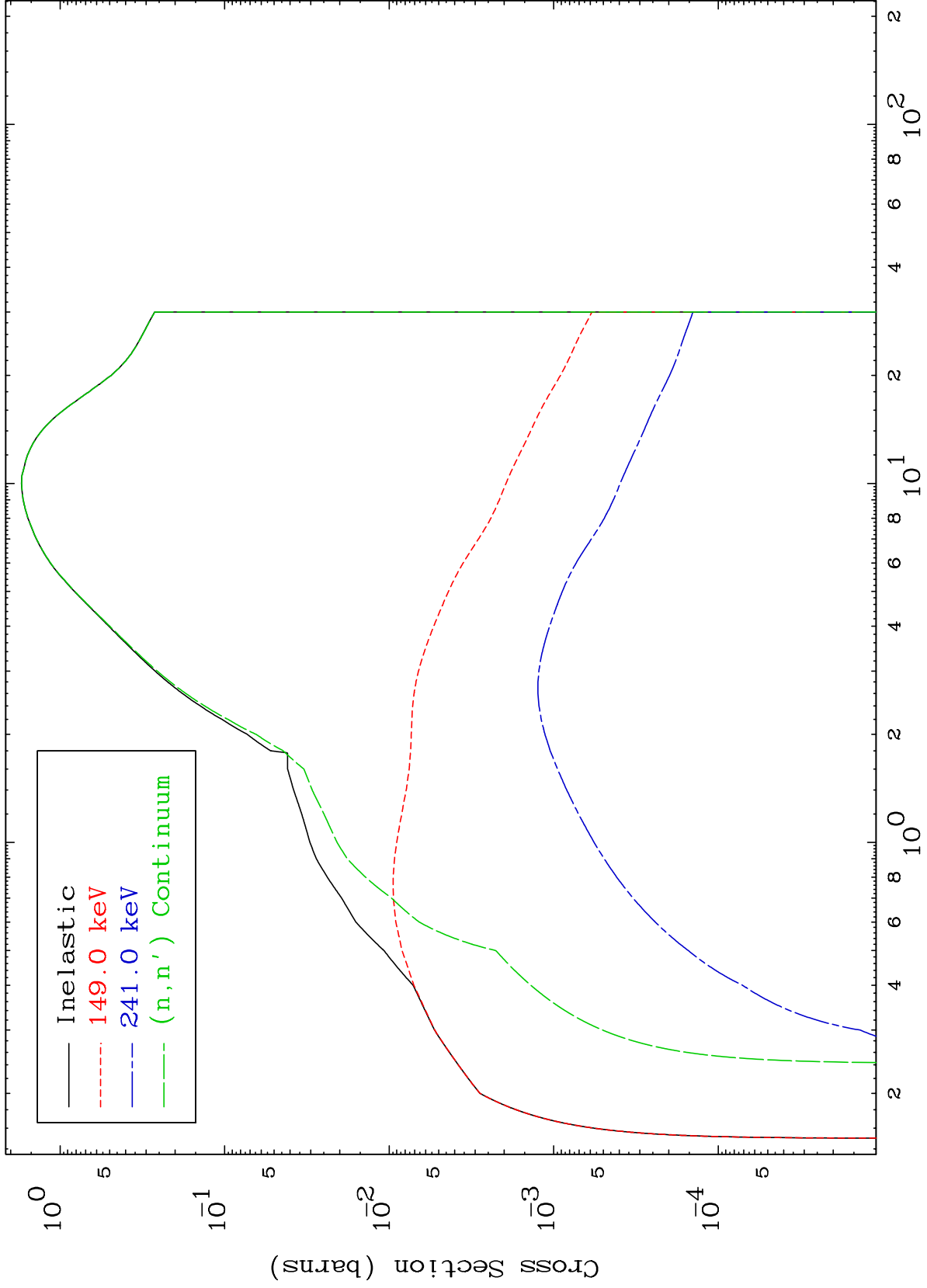
MAT 8271

Particle Production
293 Kelvin Cross Sections

83-Bi-191



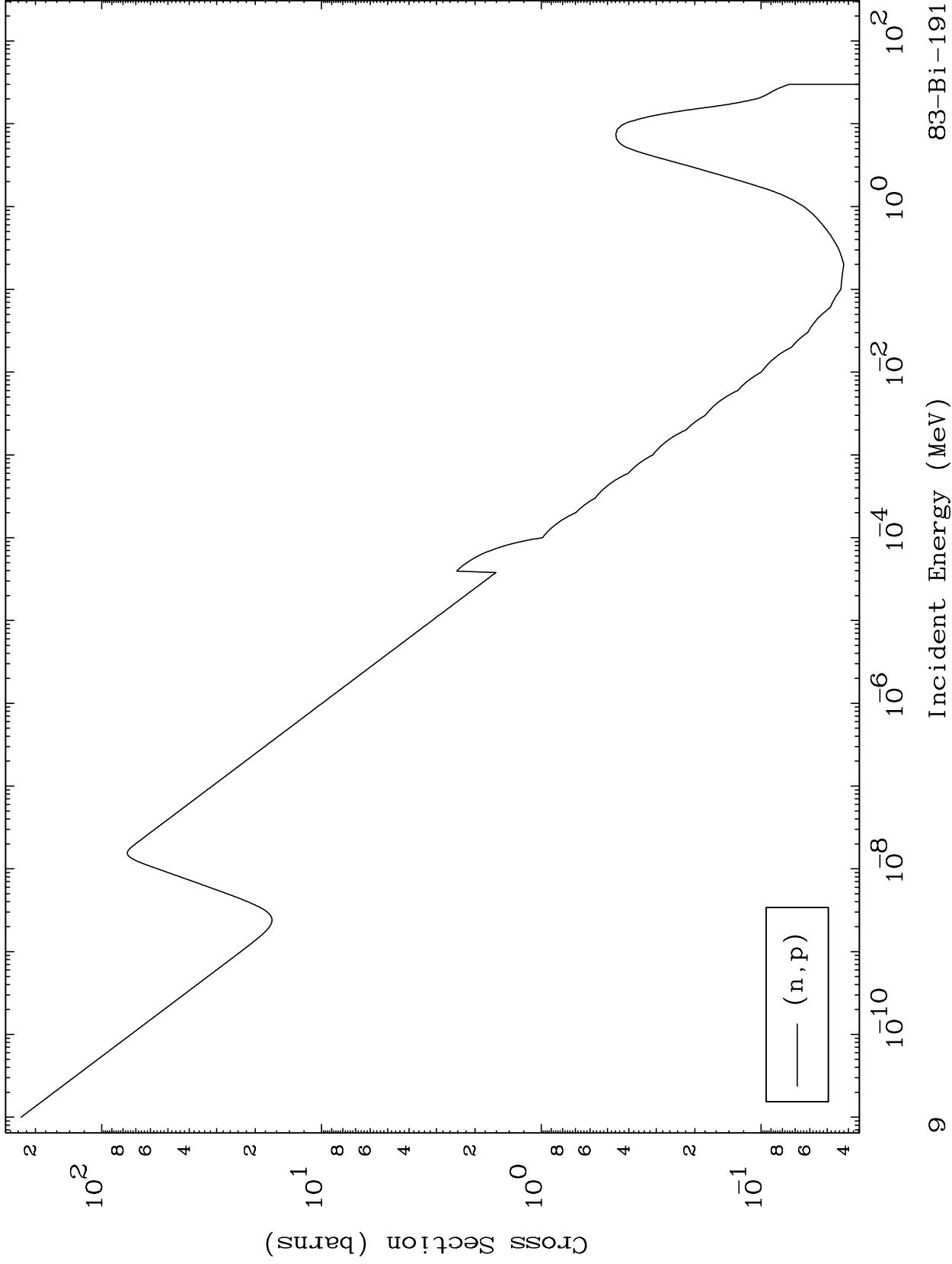
(n,n') Level
293 Kelvin Cross Sections



MAT 8271

(n,p) Levels
293 Kelvin Cross Sections

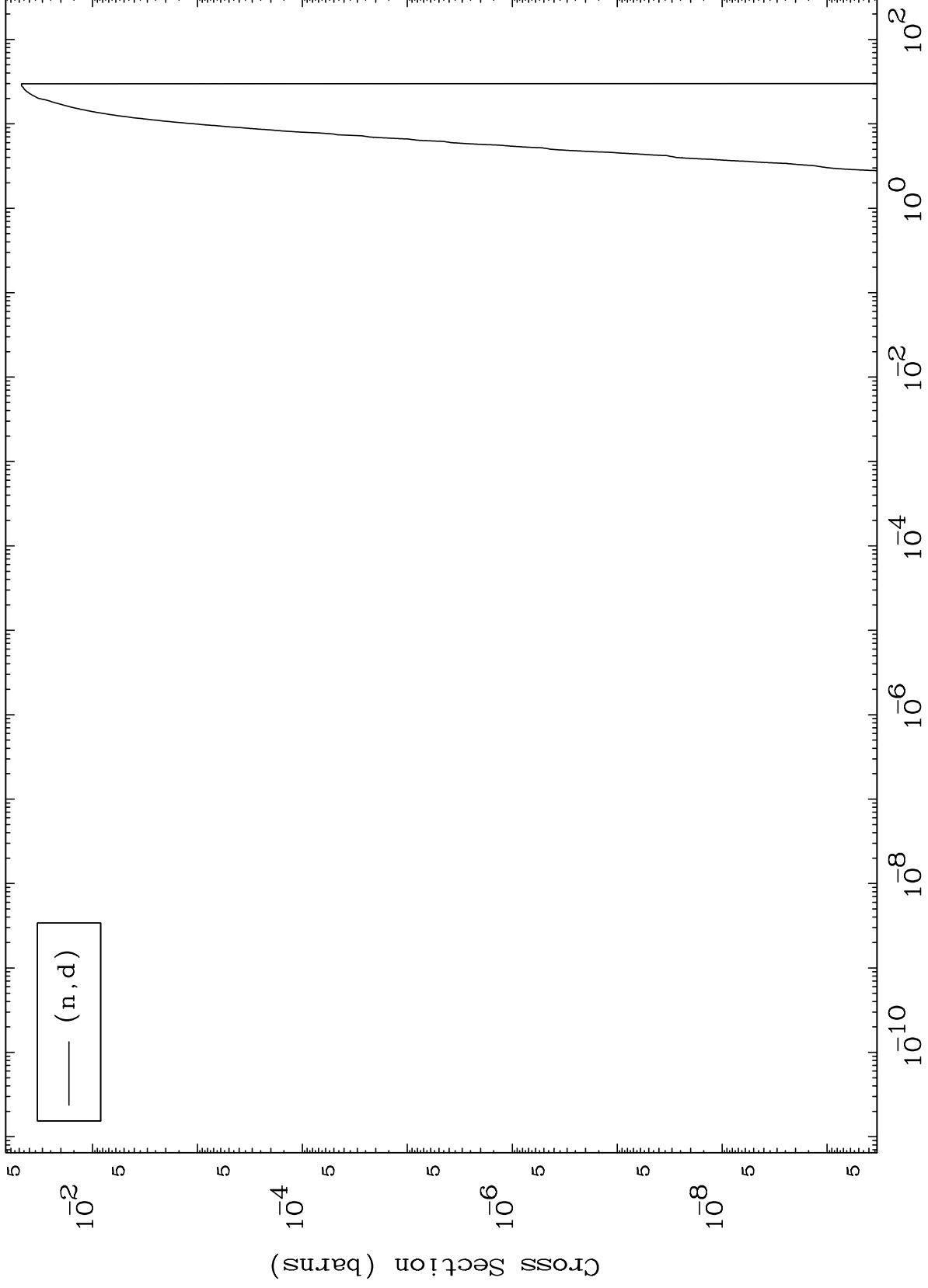
83-Bi-191



MAT 8271

(n,d) Levels
293 Kelvin Cross Sections

83-Bi-191



10

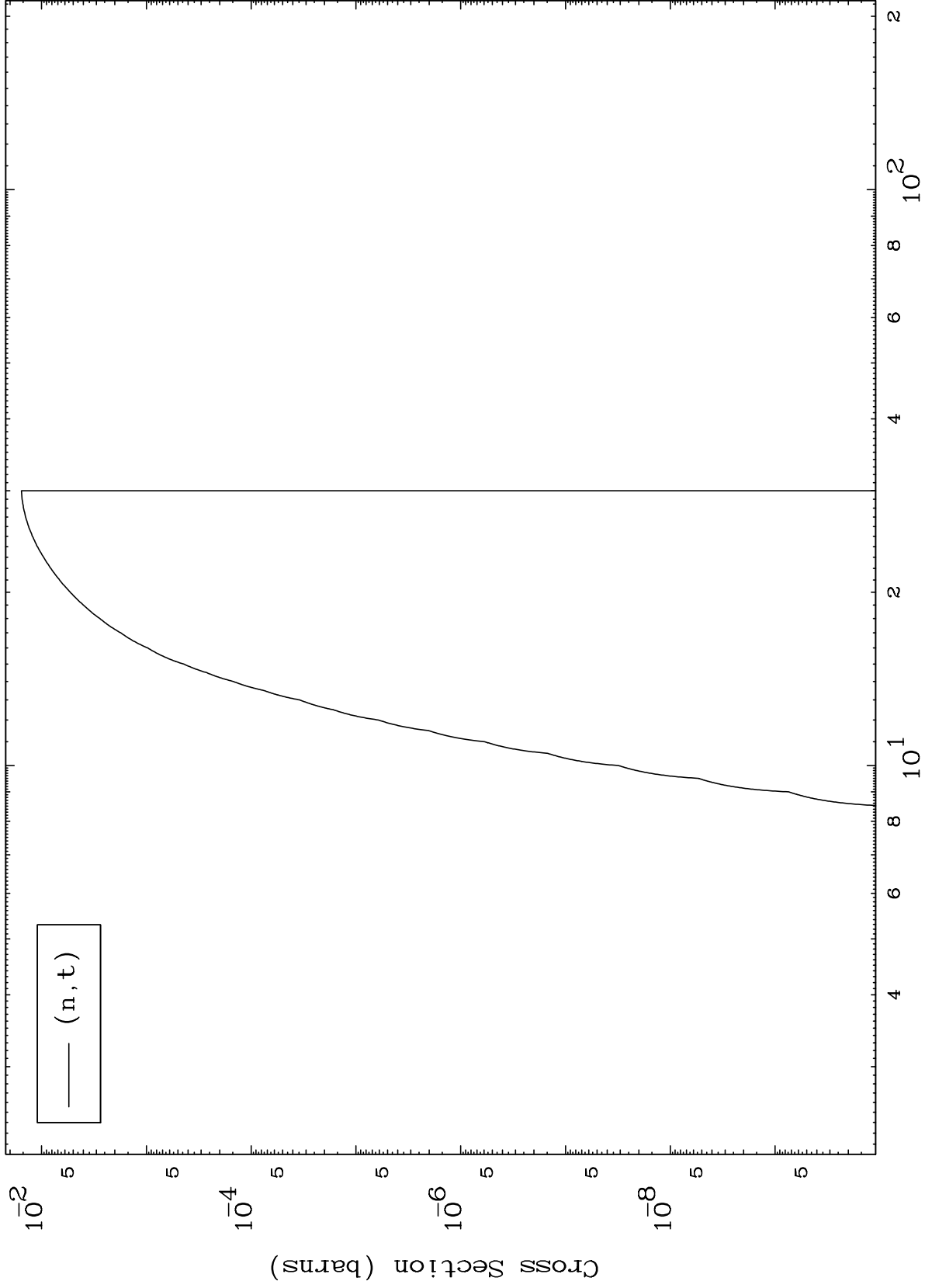
Incident Energy (MeV)

83-Bi-191

MAT 8271

(n,t) Levels
293 Kelvin Cross Sections

83-Bi-191



11

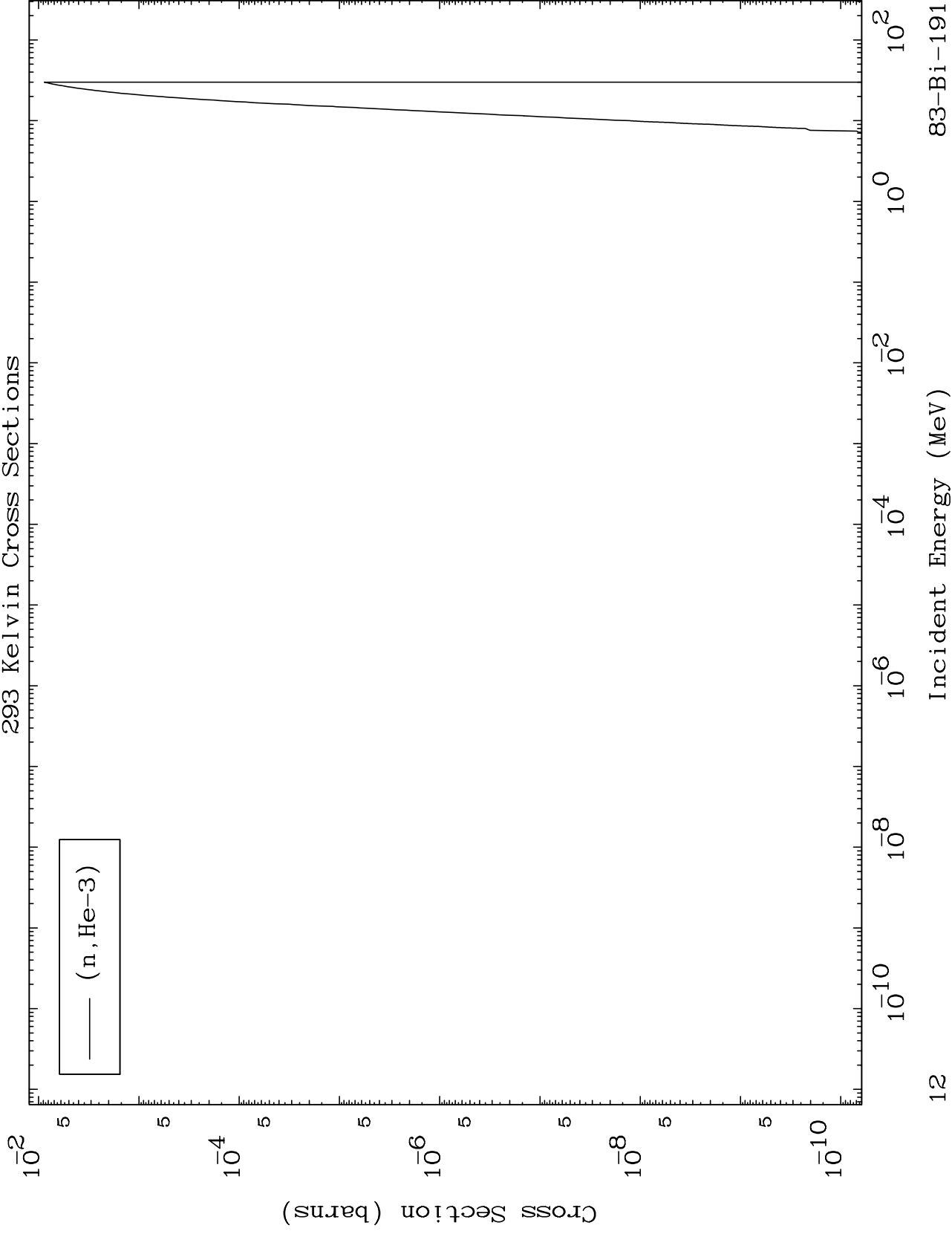
Incident Energy (MeV)

83-Bi-191

MAT 8271

(n,He3) Levels
293 Kelvin Cross Sections

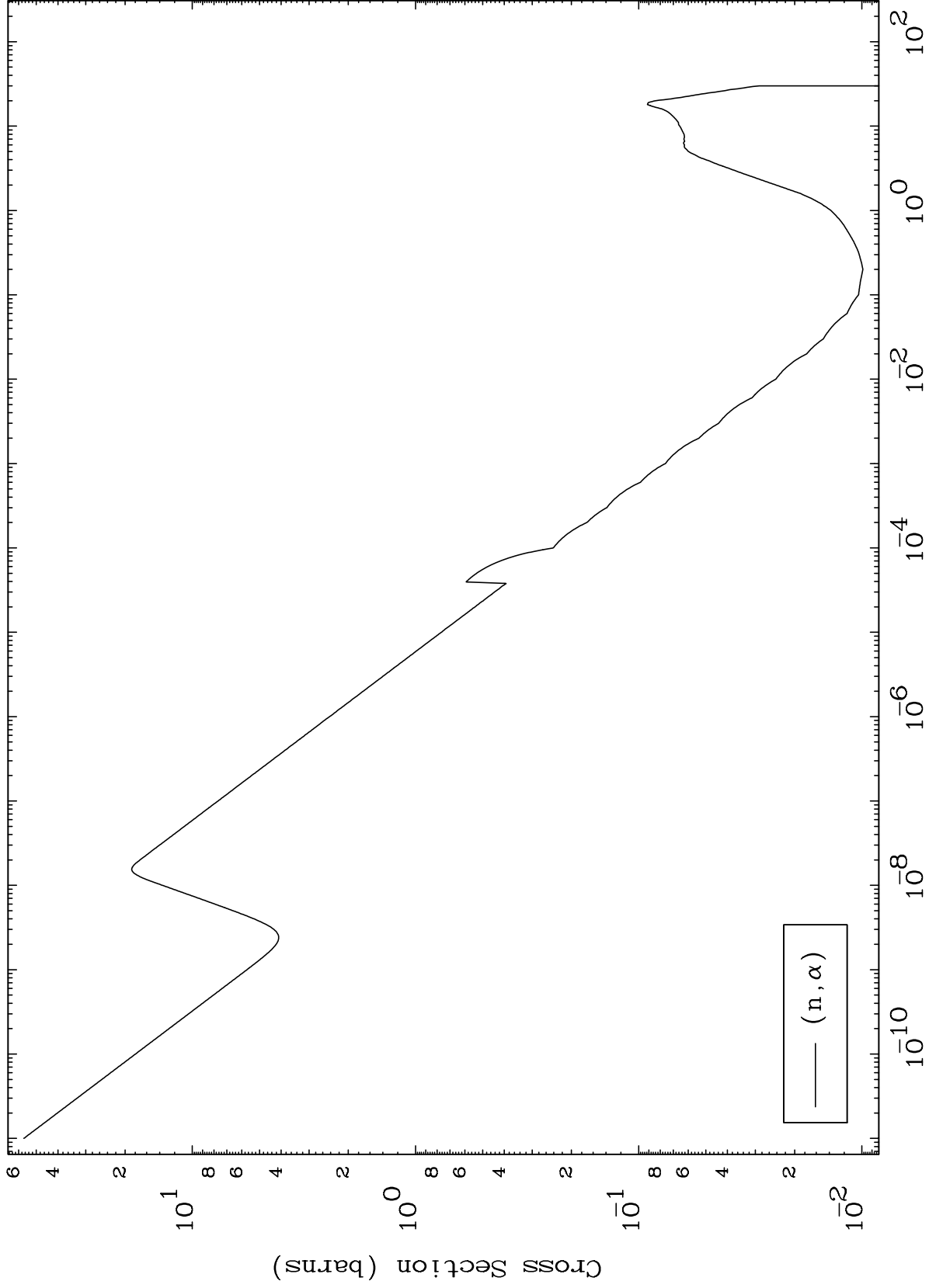
83-Bi-191



MAT 8271

(n, α) Levels
293 Kelvin Cross Sections

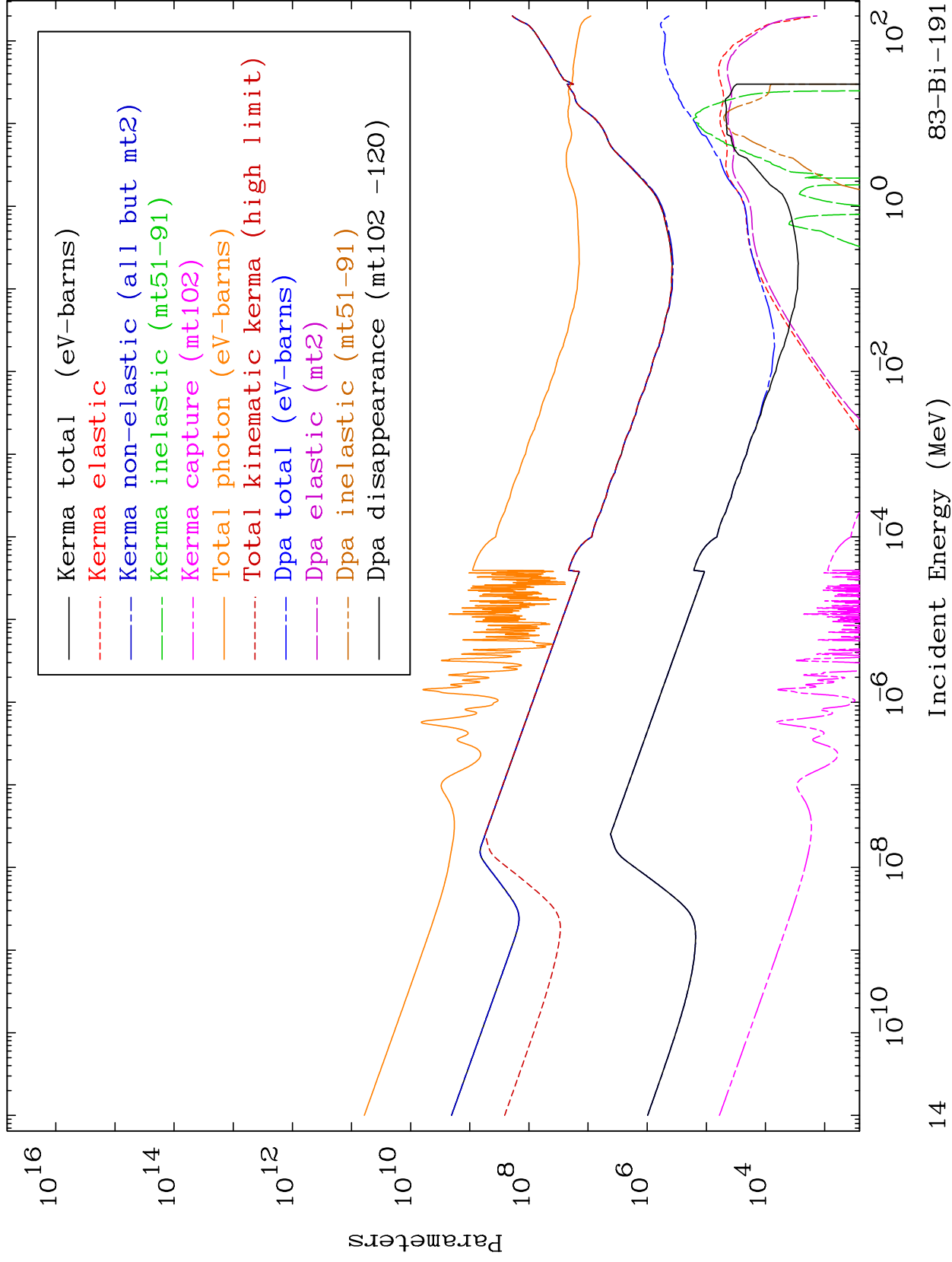
83-Bi-191

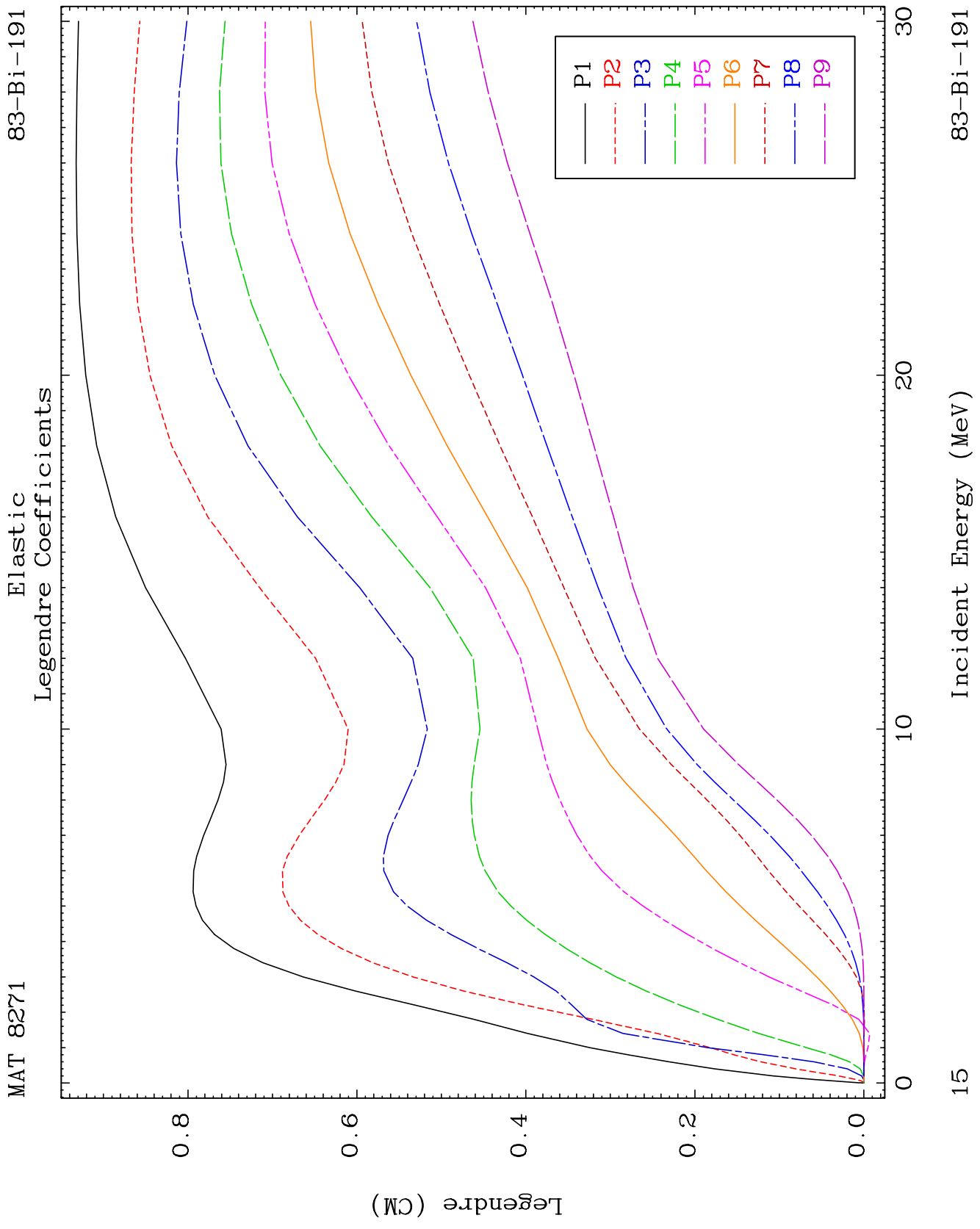


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Incident Energy (MeV)

83-Bi-191

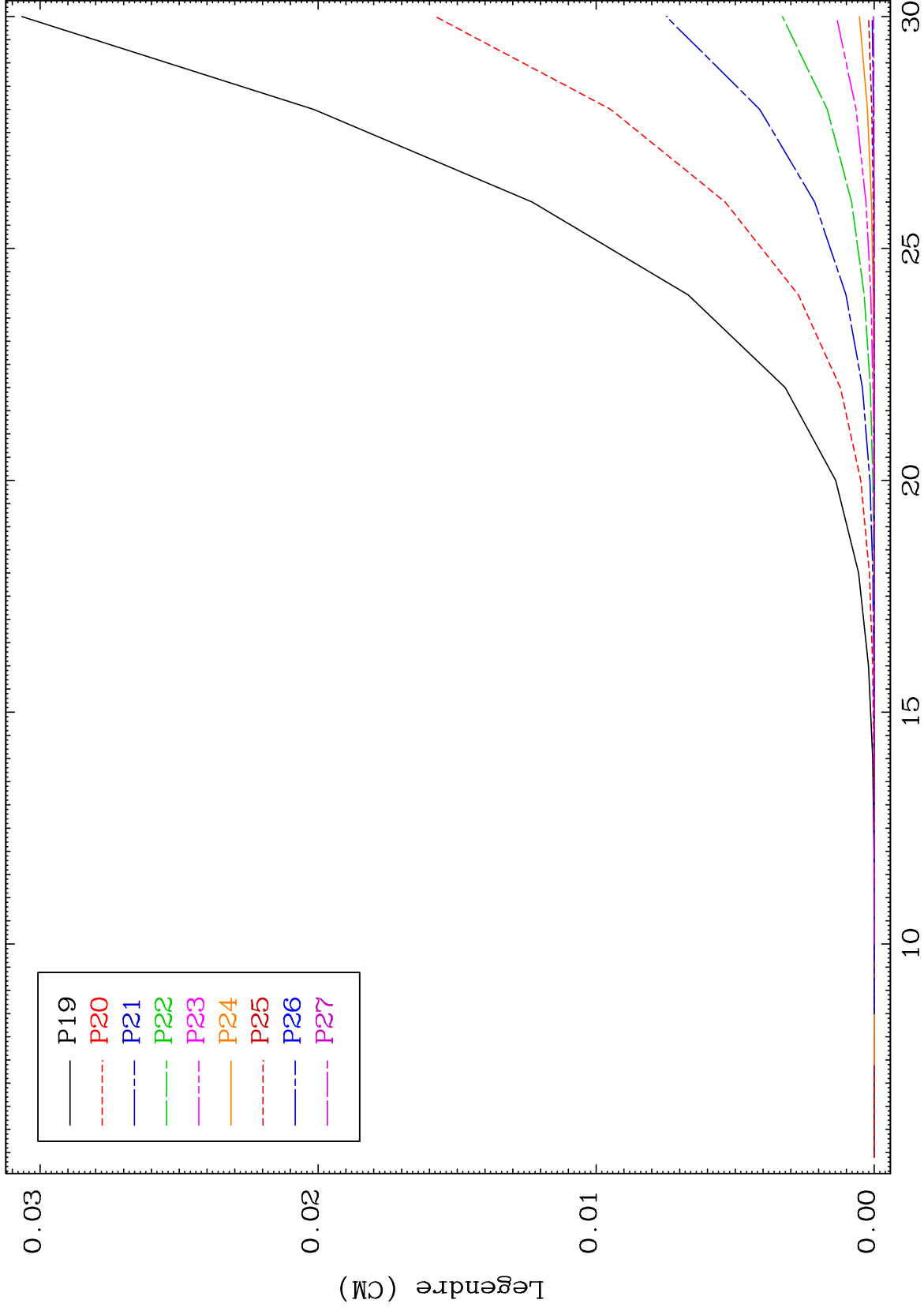




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Elastic Legendre Coefficients

83-Bi-191



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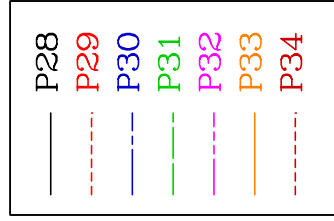
Incident Energy (MeV)

83-Bi-191

MAT 8271

Elastic
Legendre Coefficients

83-Bi-191



$\times 10^{-6}$

Legendre (CM)

8

6

4

2

0

15

20

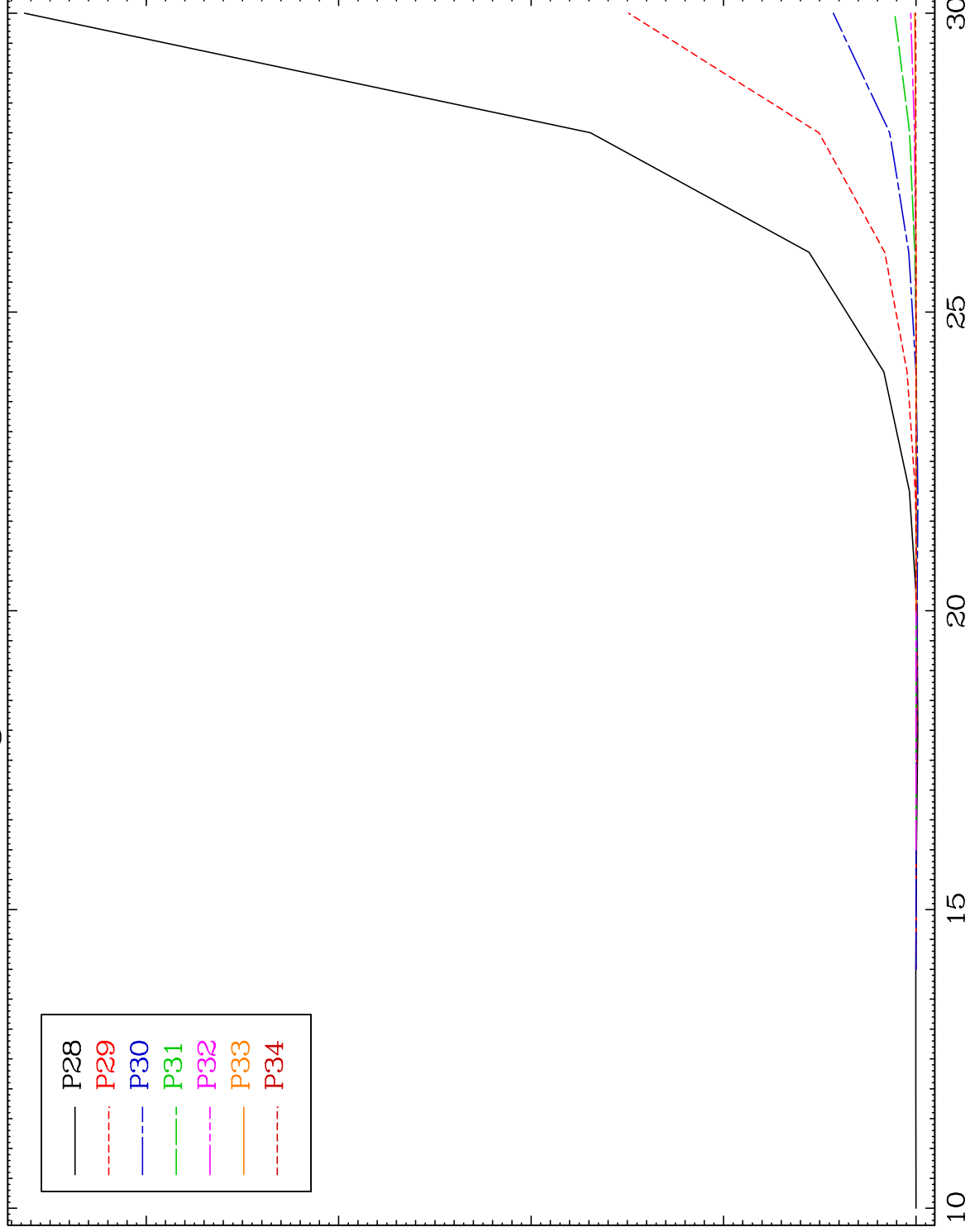
25

30

18

Incident Energy (MeV)

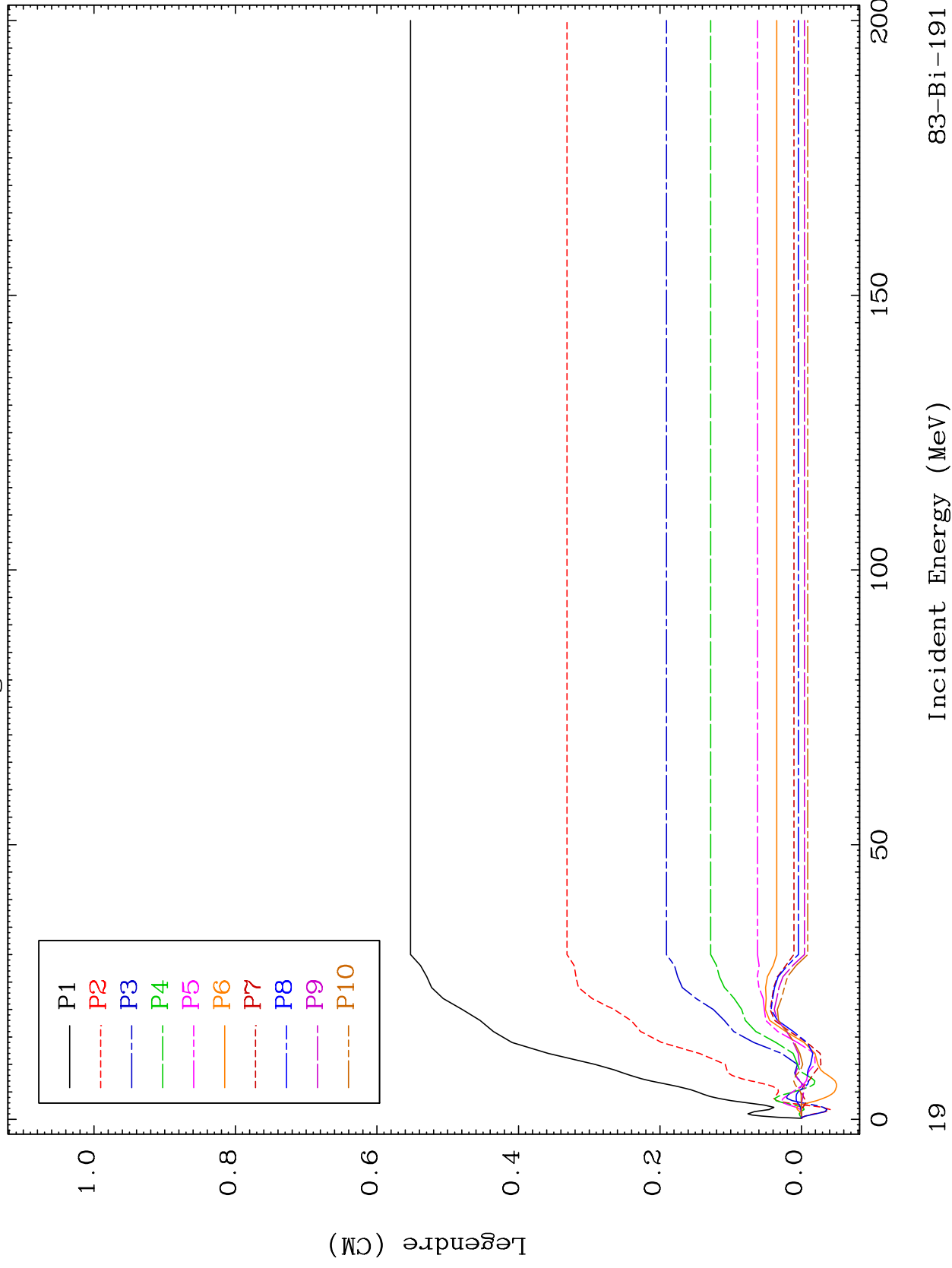
83-Bi-191



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149.0 keV (n,n') Level
Legendre Coefficients

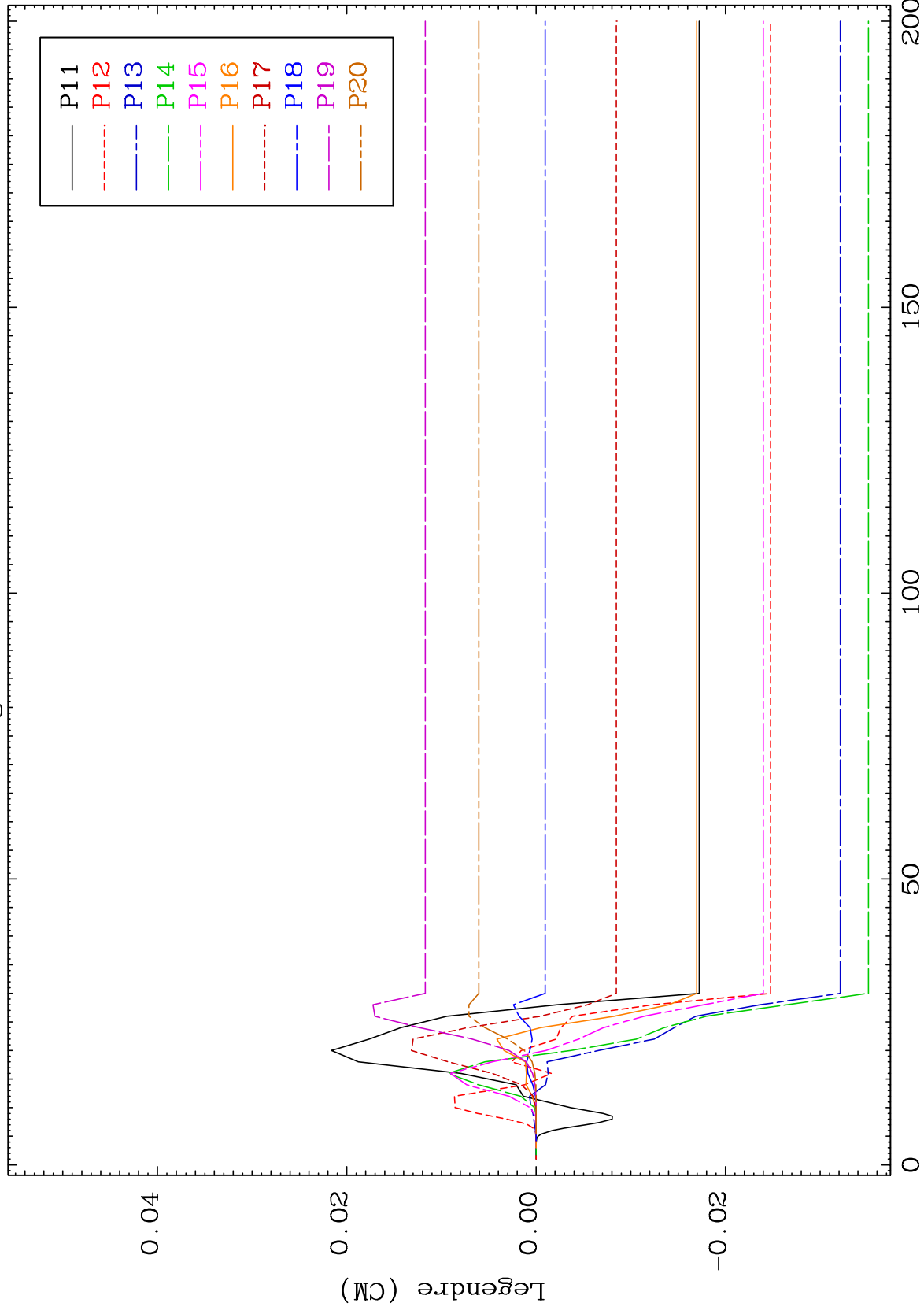
83-Bi-191



MAT 8271

149.0 keV (n,n') Level
Legendre Coefficients

83-Bi-191



0

50

100

150

200

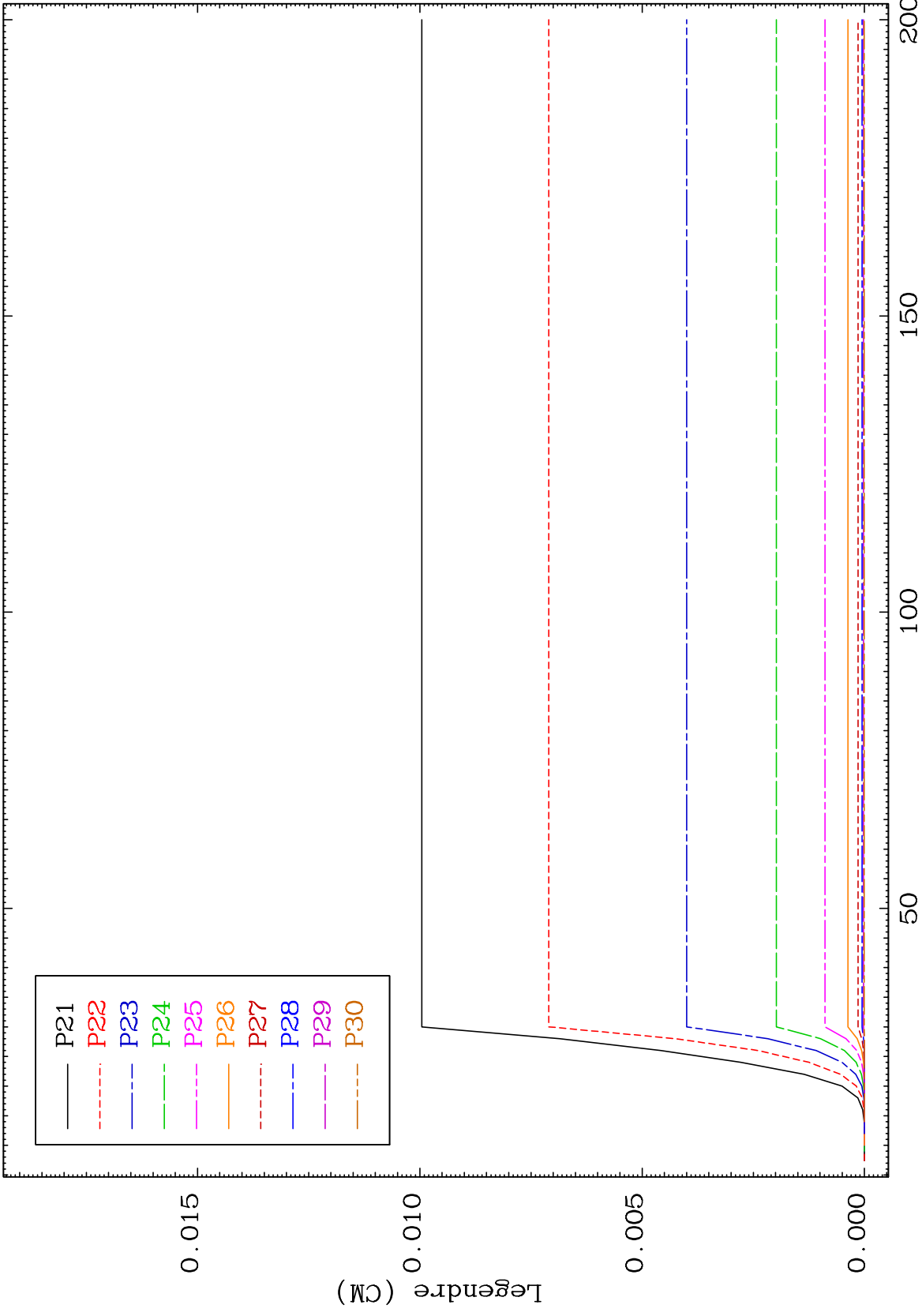
Incident Energy (MeV)

83-Bi-191

MAT 8271

149.0 keV (n,n') Level
Legendre Coefficients

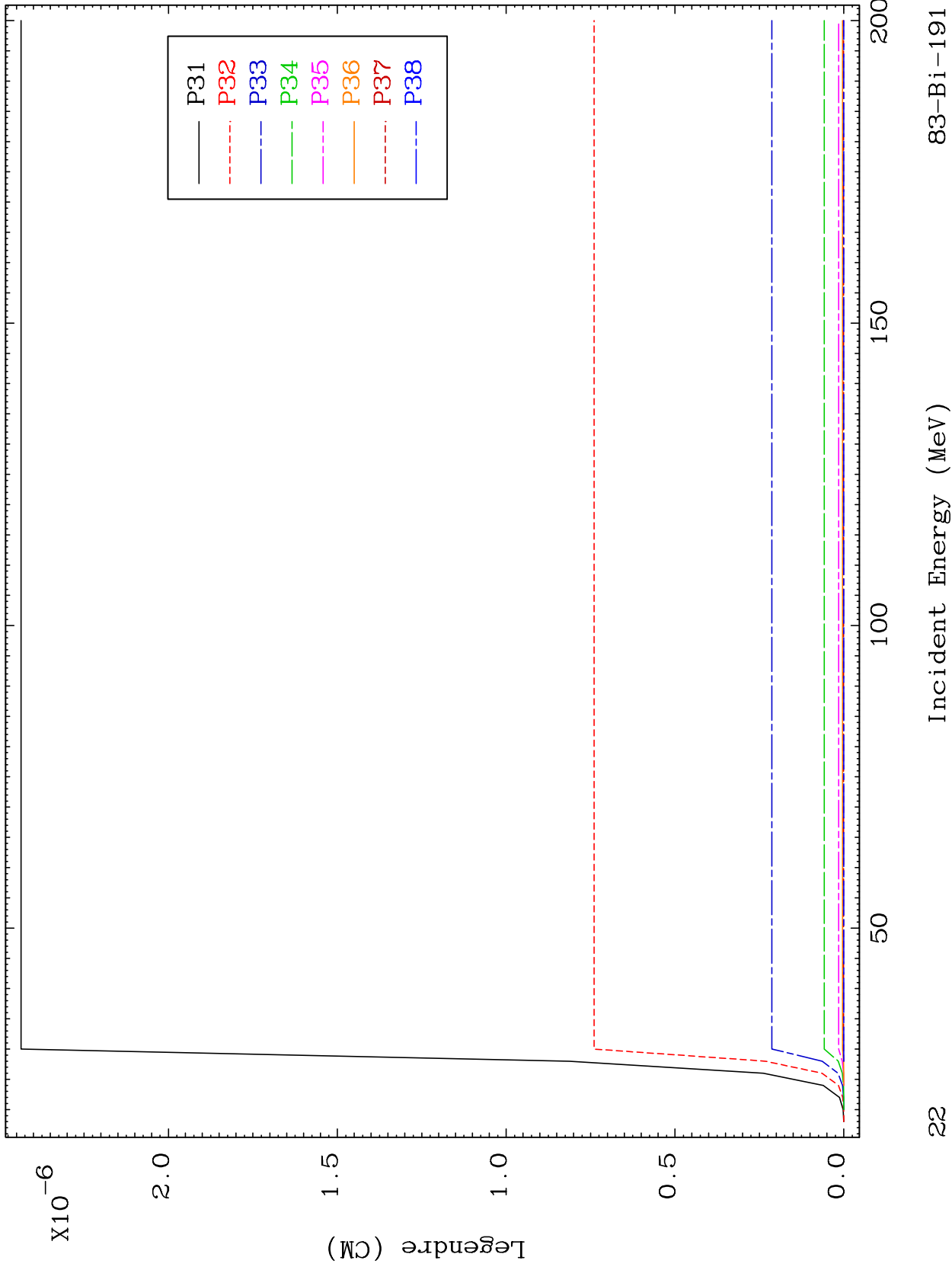
83-Bi-191



21

Incident Energy (MeV)

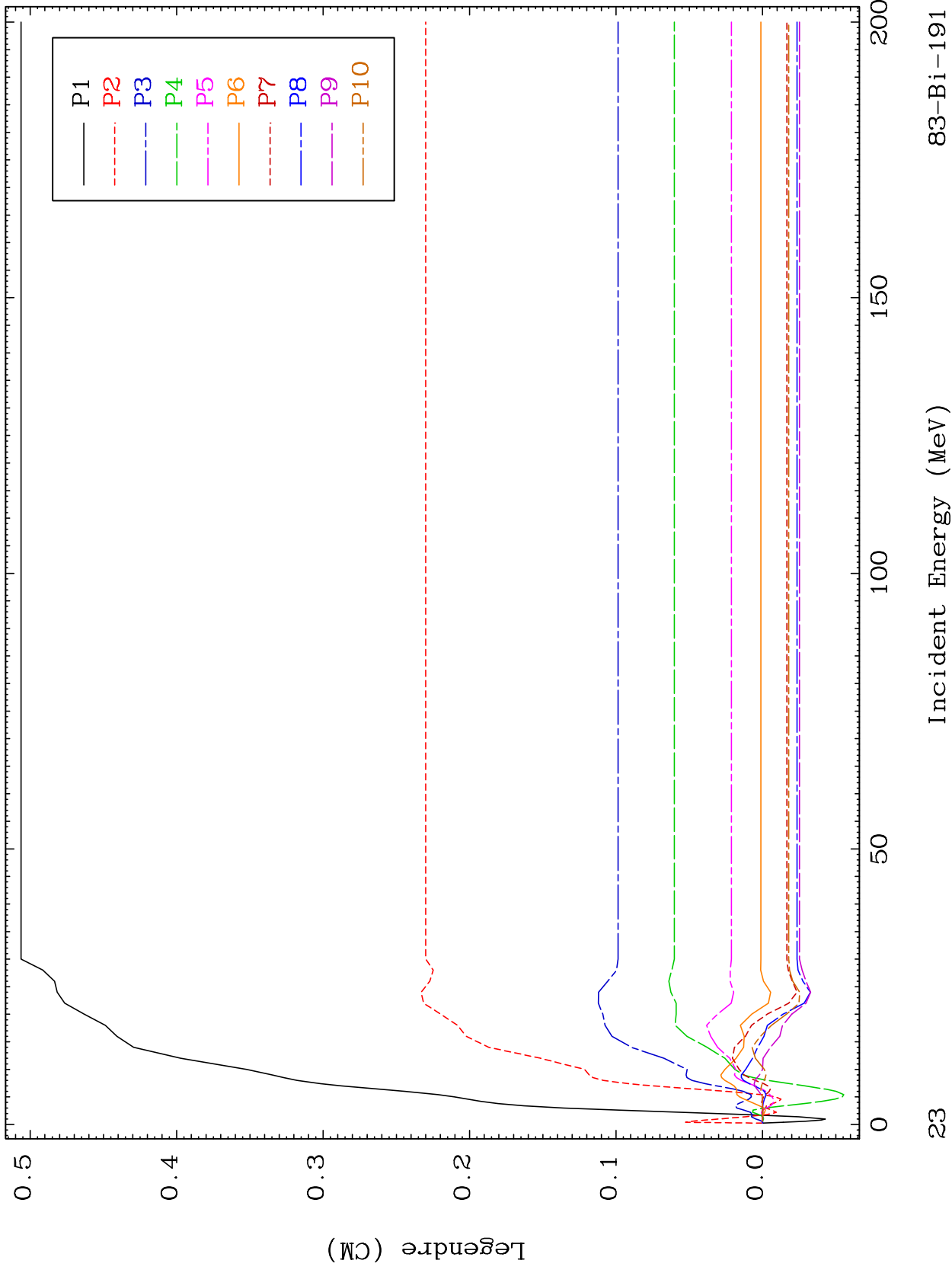
83-Bi-191



MAT 8271

241.0 keV (n,n') Level
Legendre Coefficients

83-Bi-191

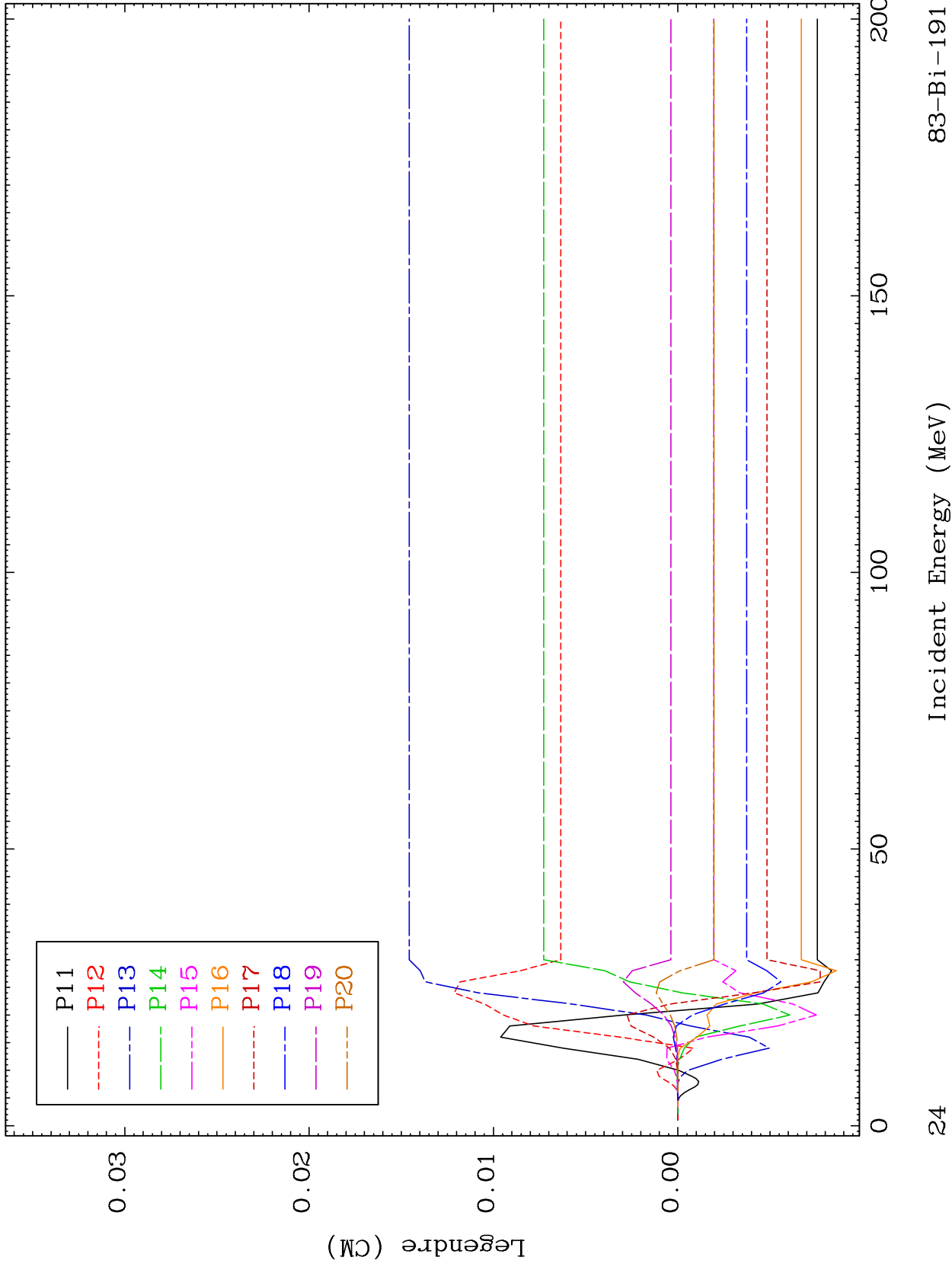


23

MAT 8271

241.0 keV (n,n') Level
Legendre Coefficients

83-Bi-191



24

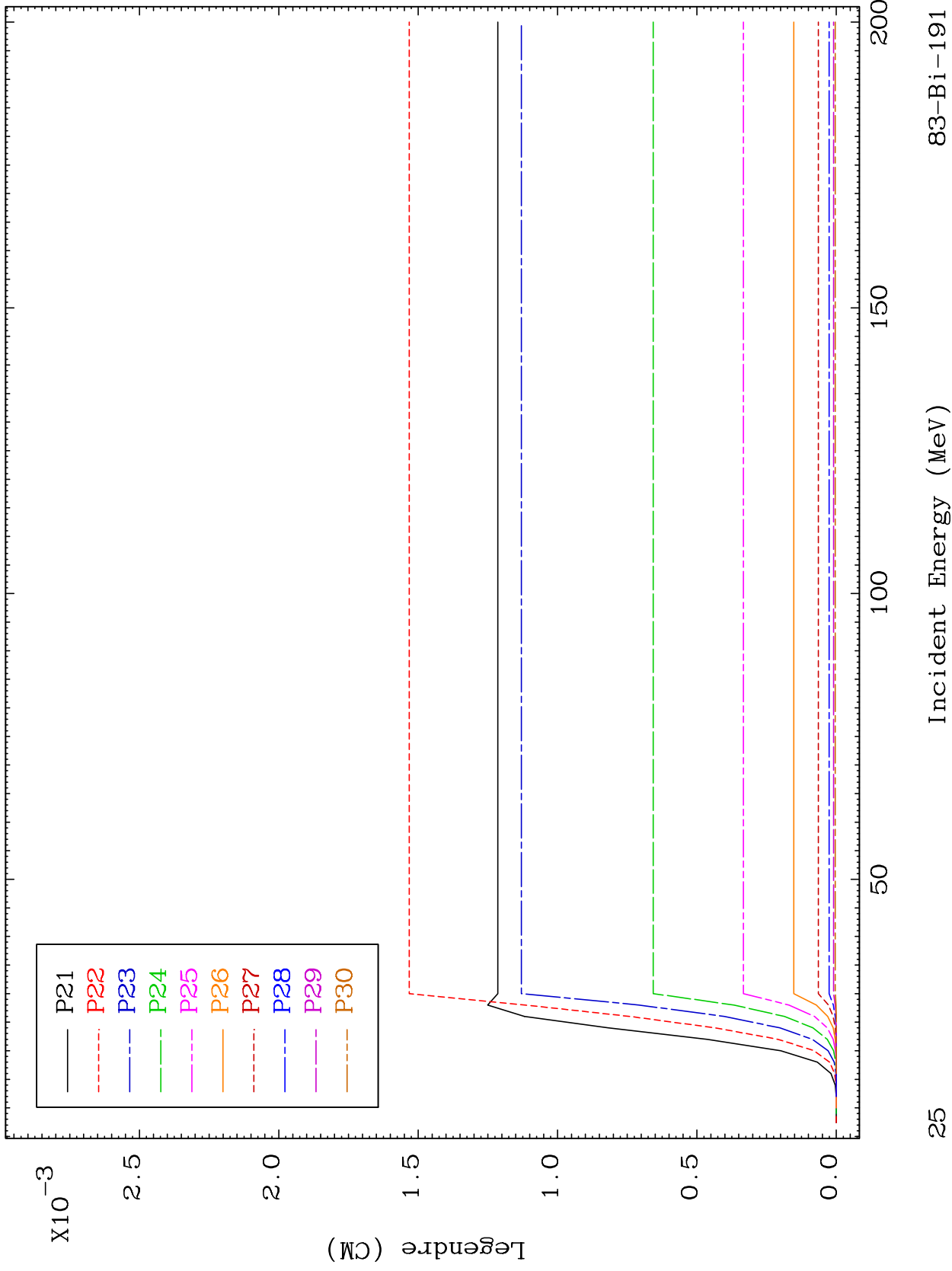
Incident Energy (MeV)

83-Bi-191

MAT 8271

241.0 keV (n,n') Level
Legendre Coefficients

83-Bi-191



25

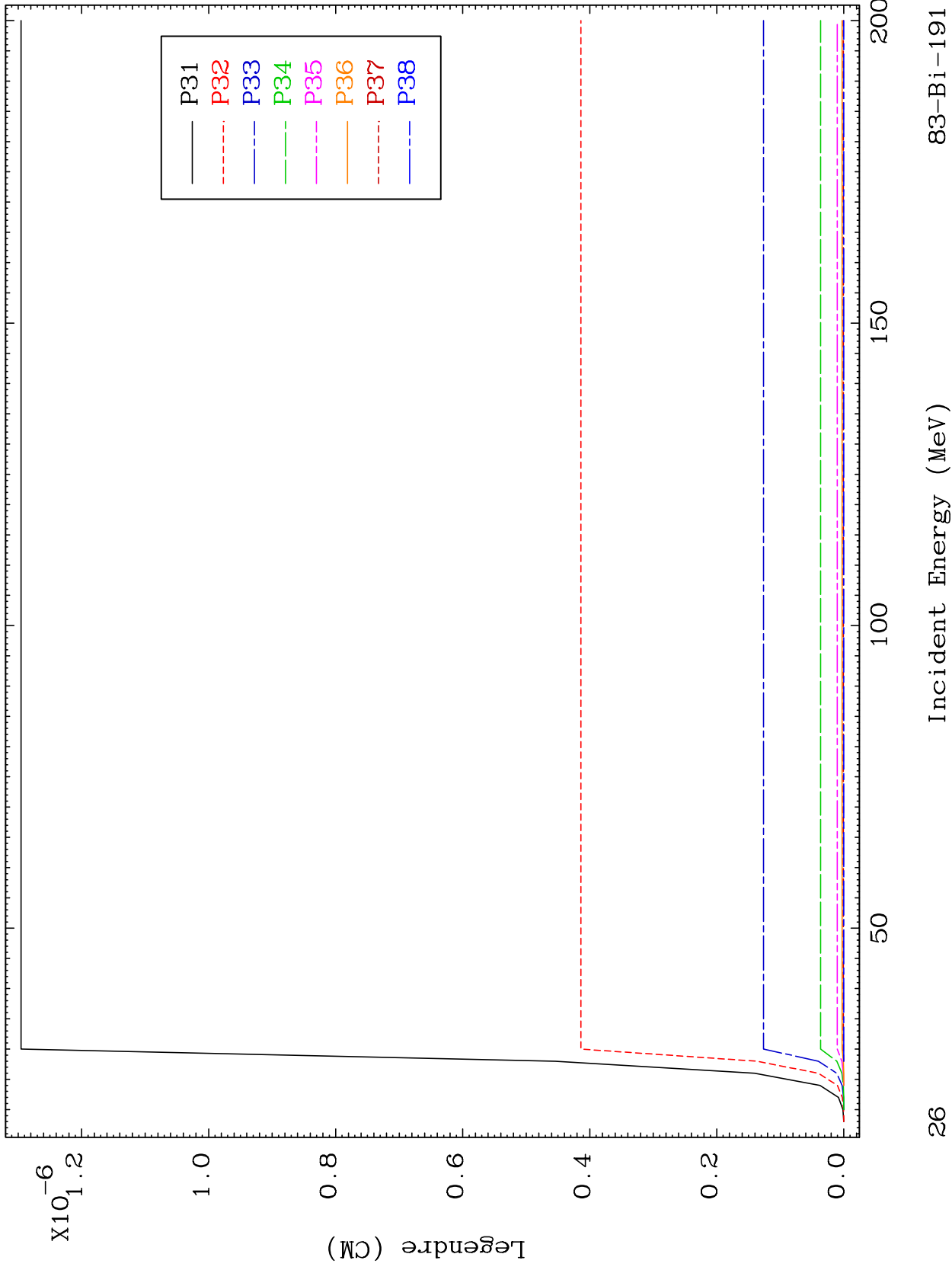
Incident Energy (MeV)

83-Bi-191

MAT 8271

241.0 keV (n,n') Level
Legendre Coefficients

83-Bi-191



26

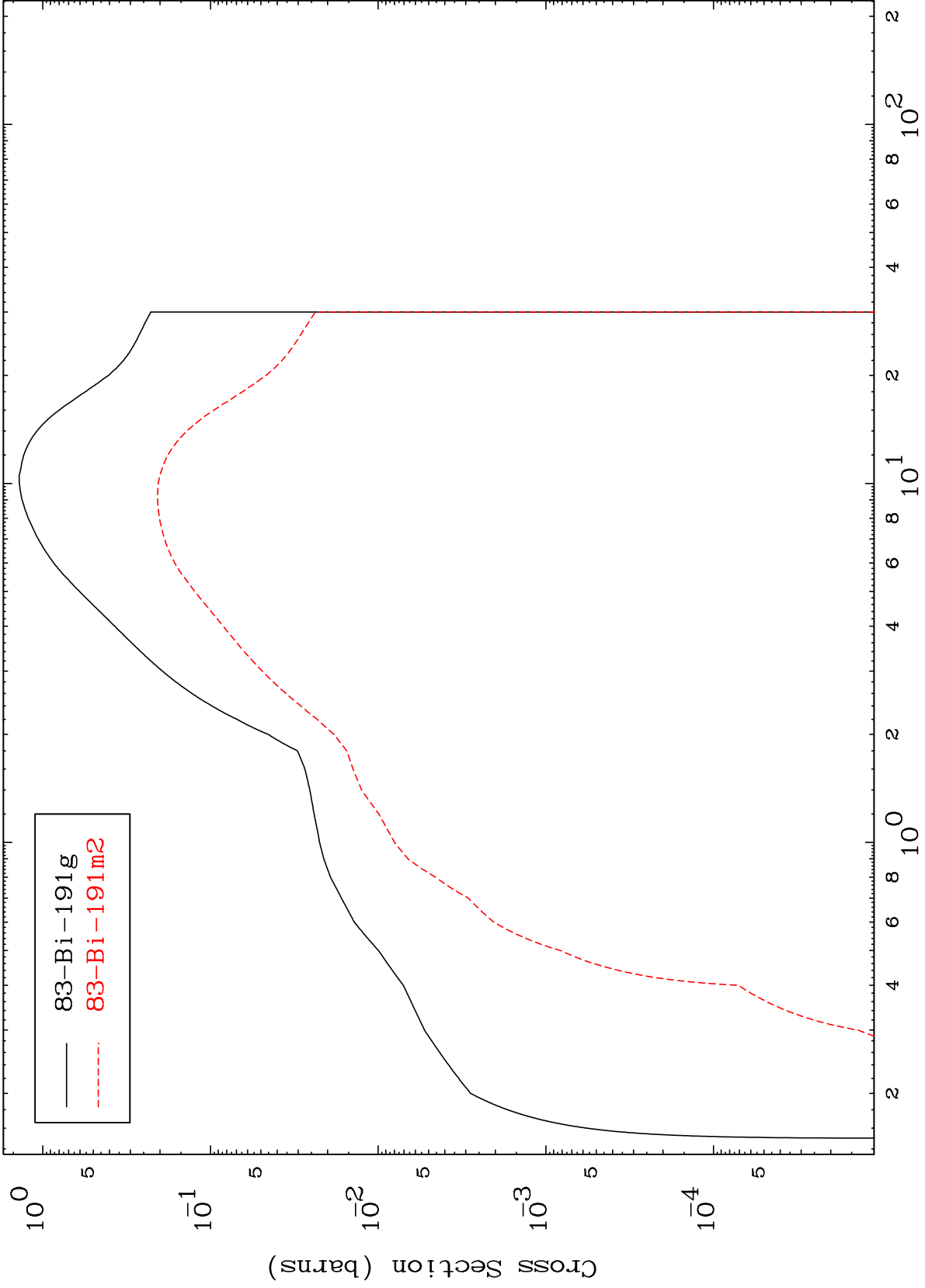
Incident Energy (MeV)

83-Bi-191

MAT 8271

83-Bi-191

Inelastic
Radionuclide Production Cross Section



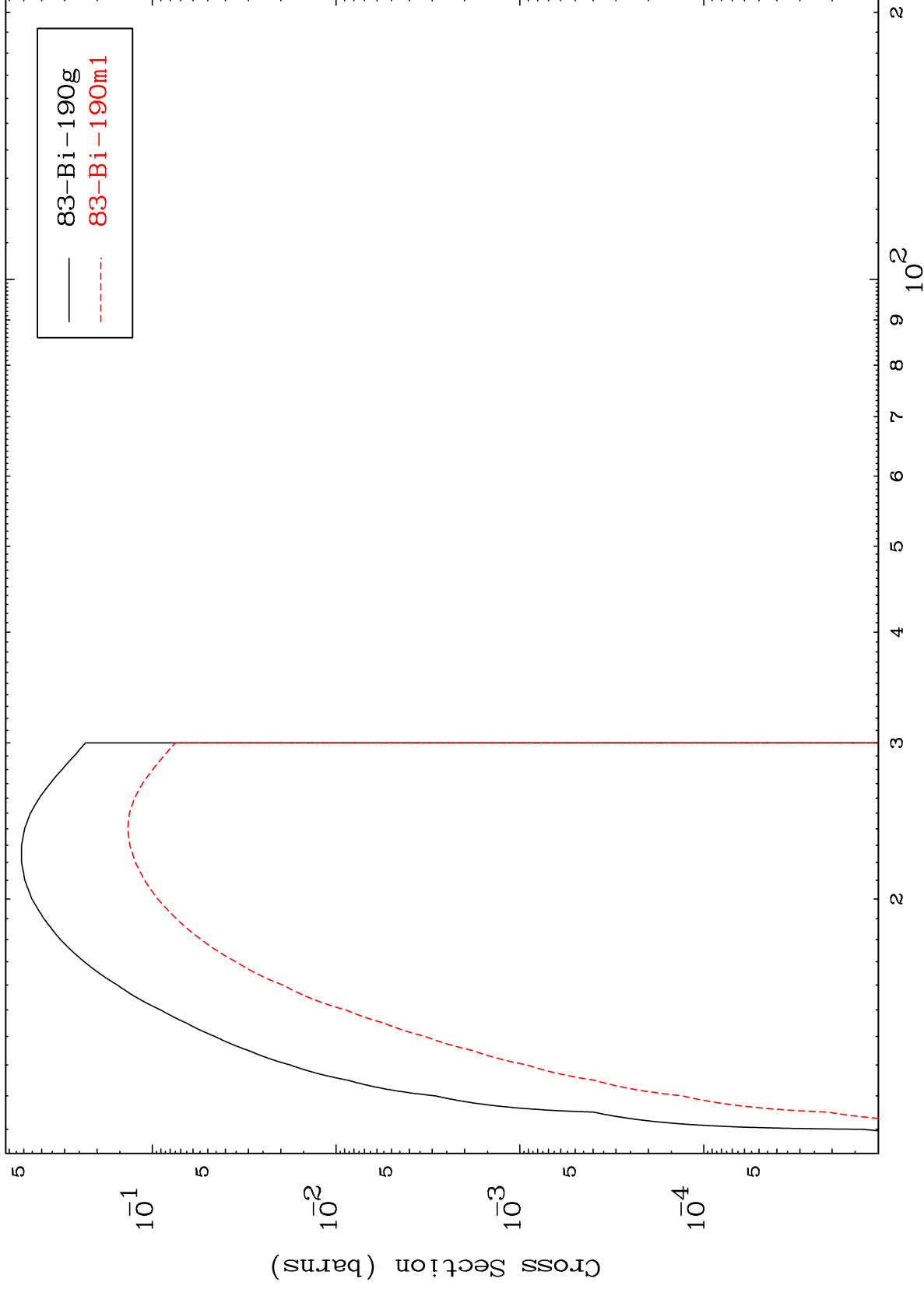
83-Bi-191g
83-Bi-191m2

27

83-Bi-191

Incident Energy (MeV)

Radionuclide Production Cross Section

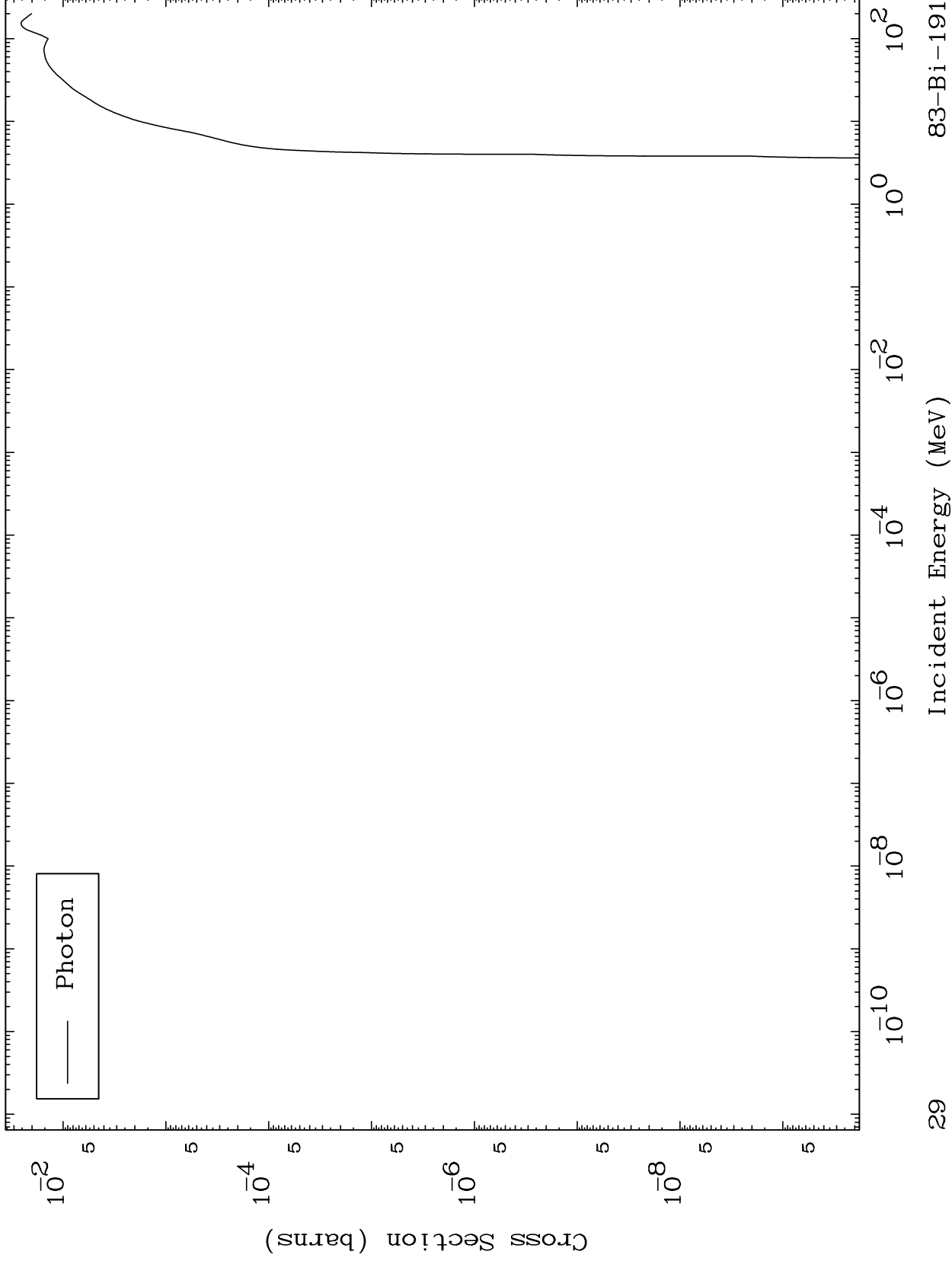


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Fission

83-Bi-191

Radionuclide Production Cross Section



29

Incident Energy (MeV)

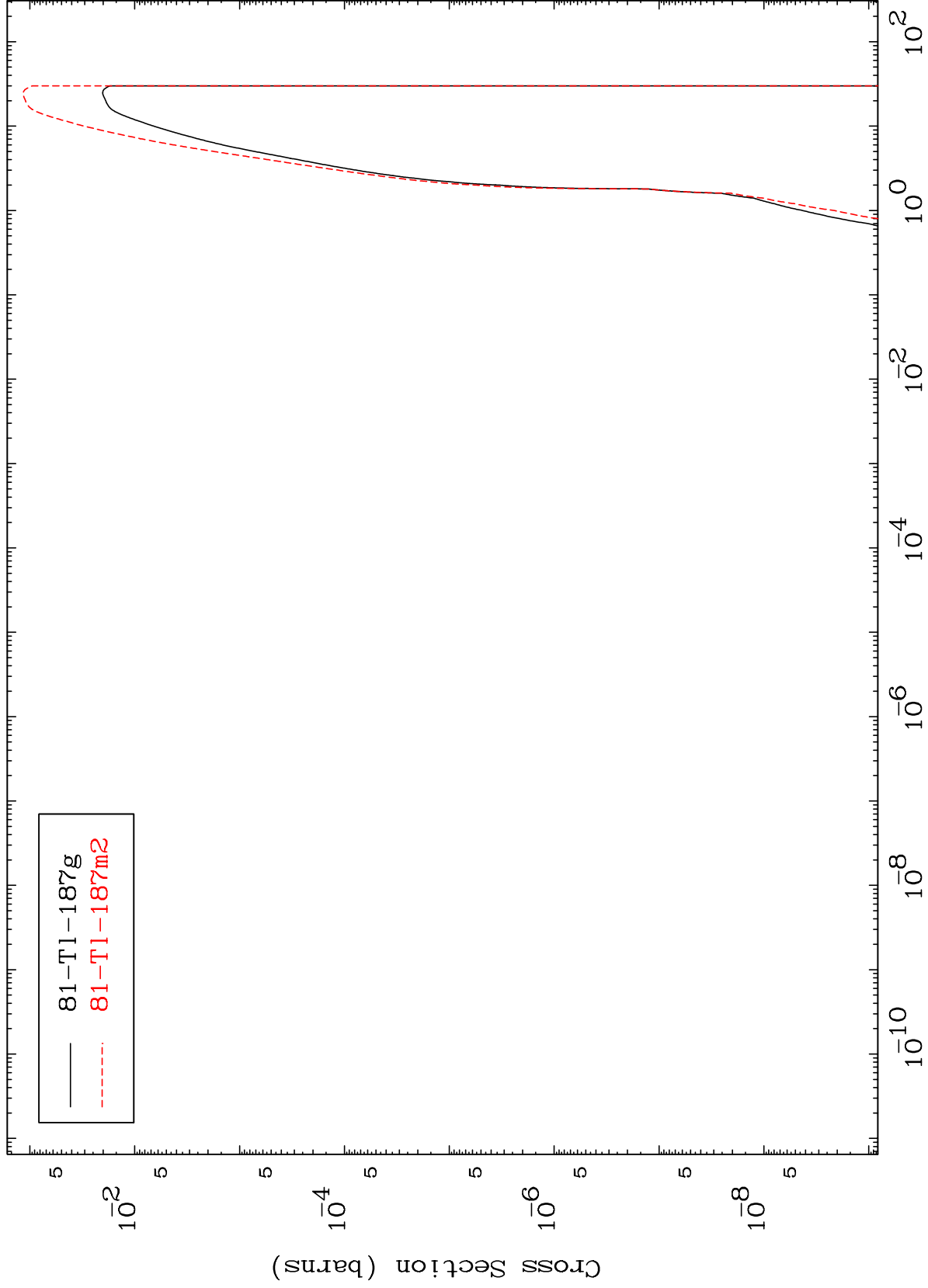
83-Bi-191

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$(n, n') \alpha$

83-Bi-191

Radionuclide Production Cross Section



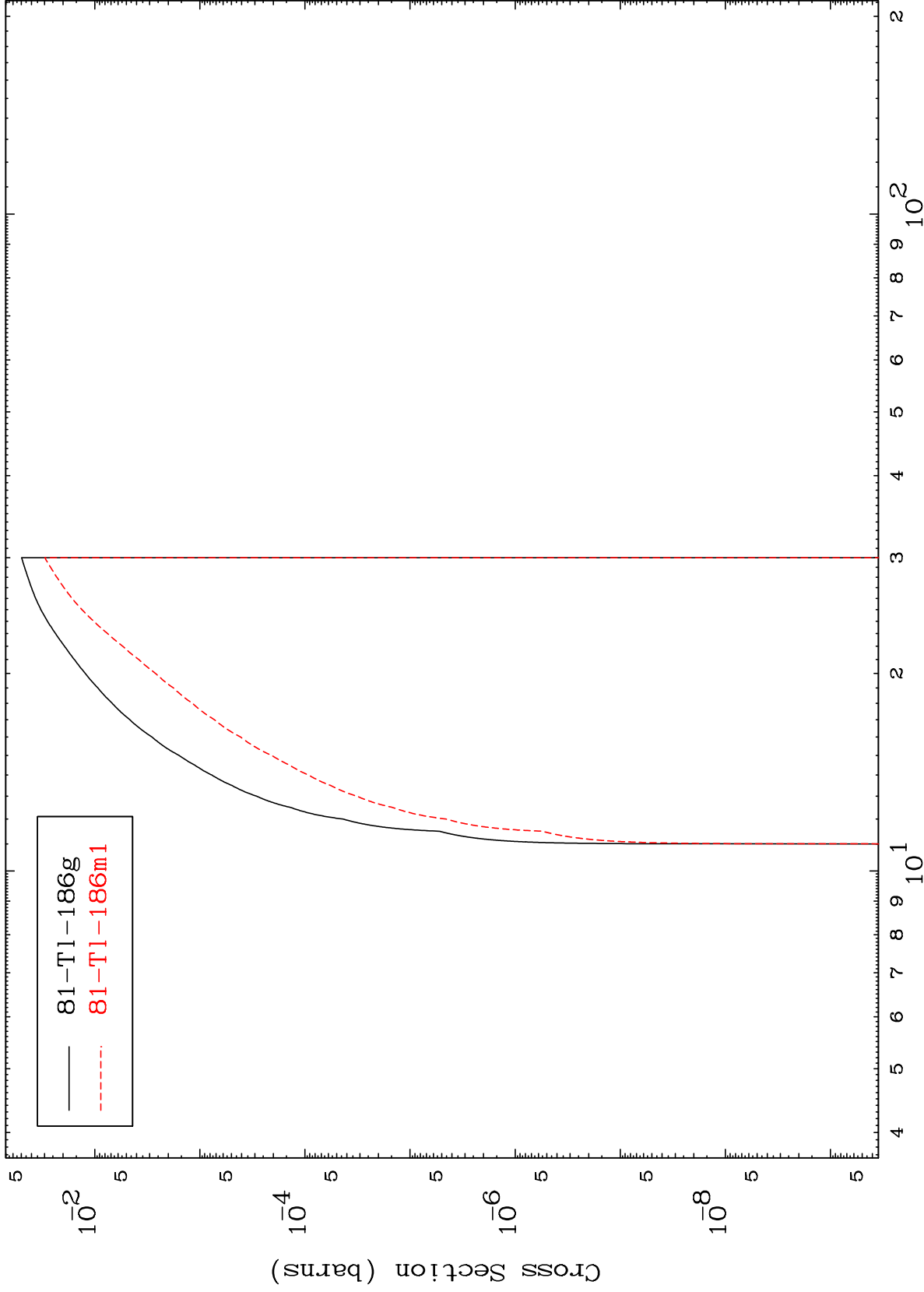
30

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$(n,2n)$ α

83-Bi-191

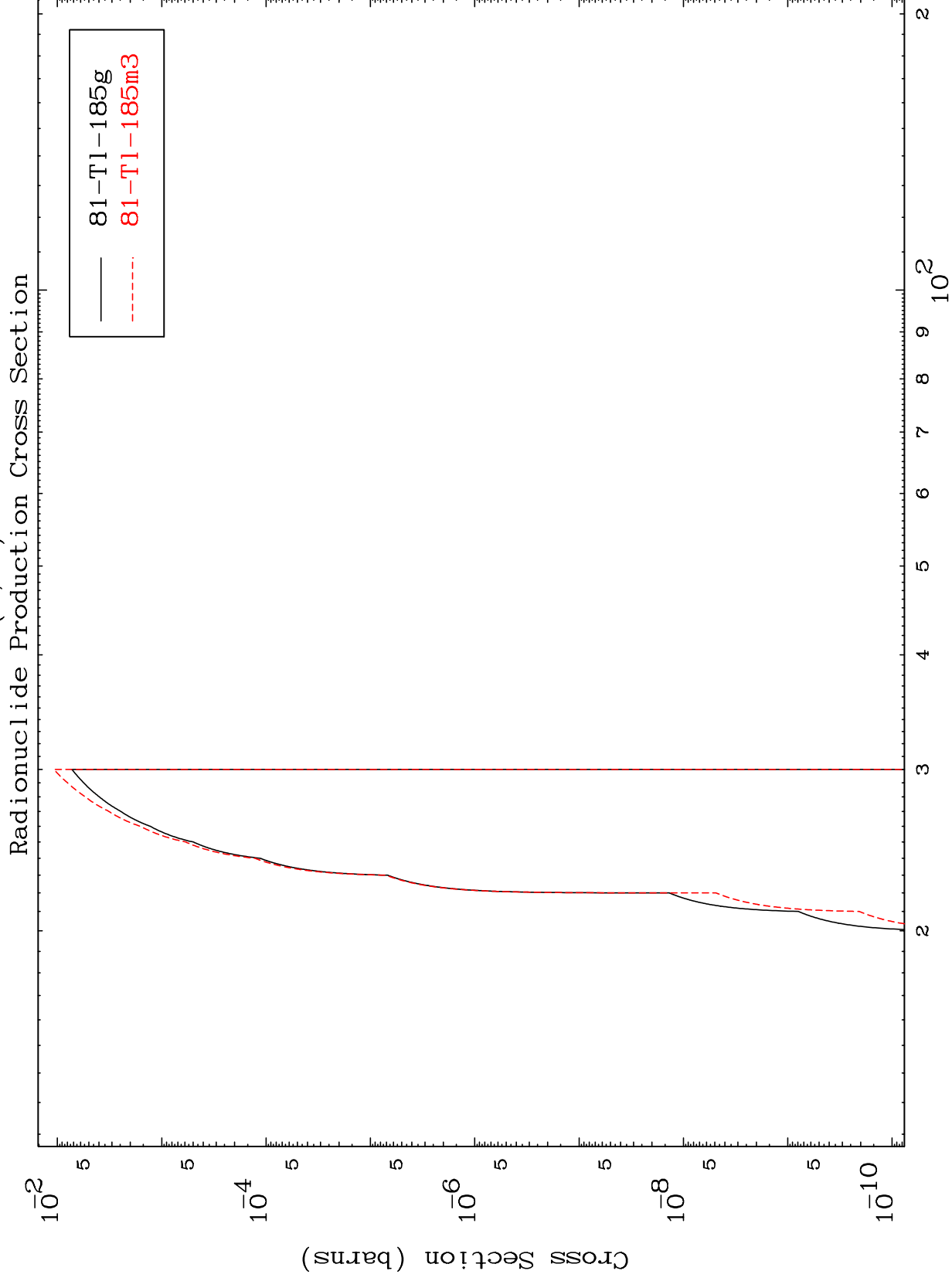
Radionuclide Production Cross Section



31

Incident Energy (MeV)

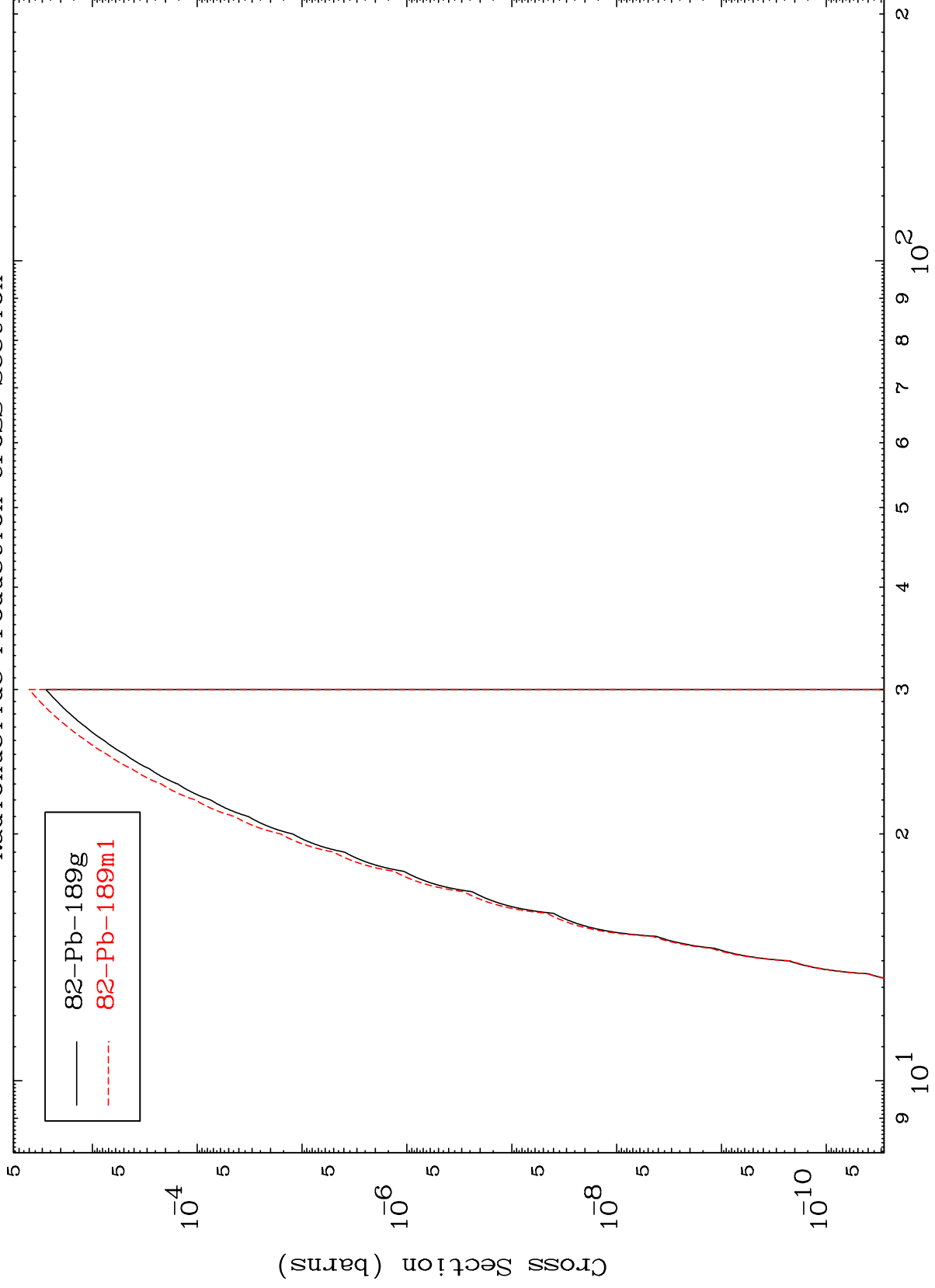
83-Bi-191



MAT 8271

83-Bi-191

(n,n') d
Radionuclide Production Cross Section



33

Incident Energy (MeV)

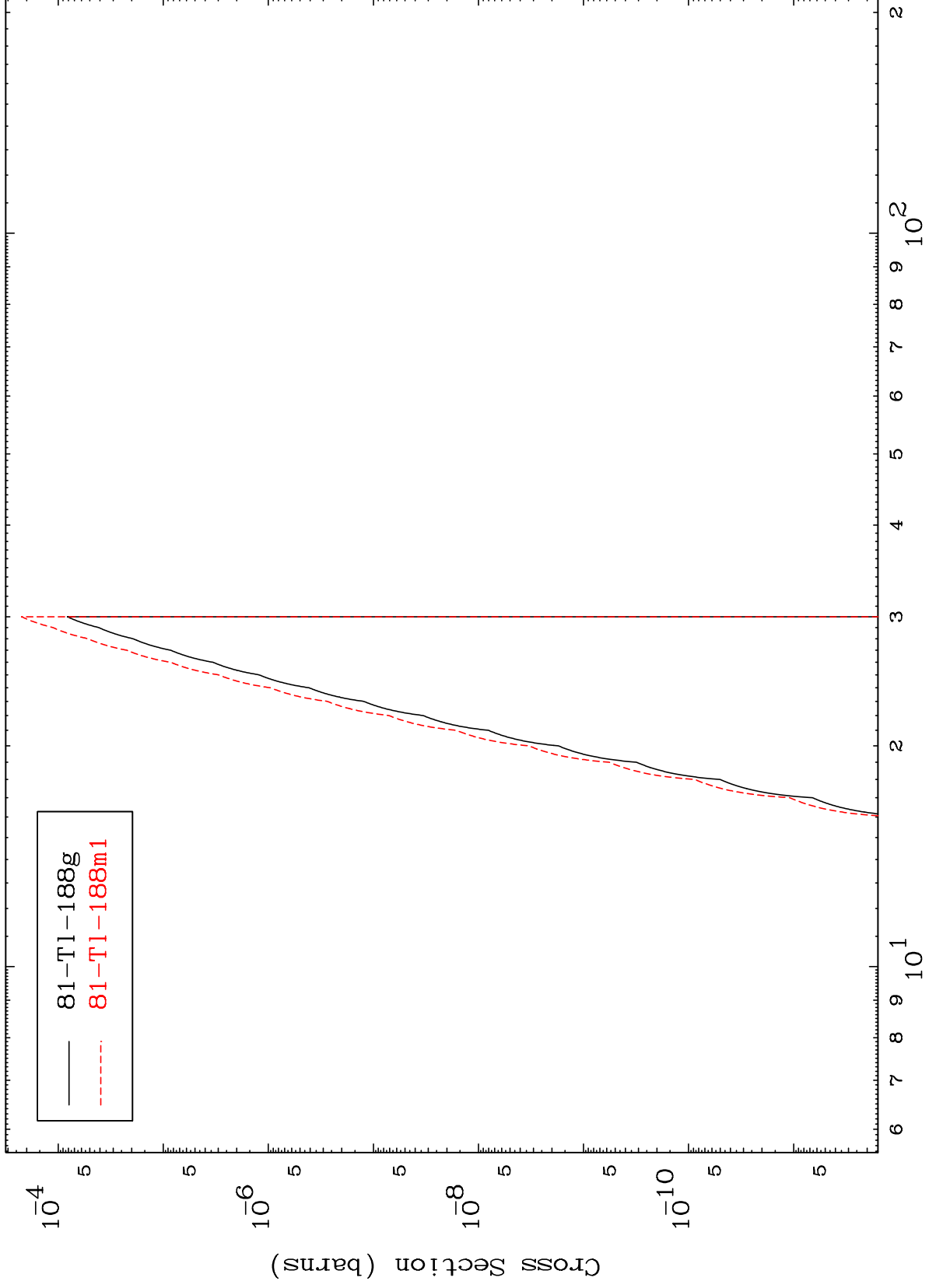
83-Bi-191

MAT 8271

(n,n') He-3

83-Bi-191

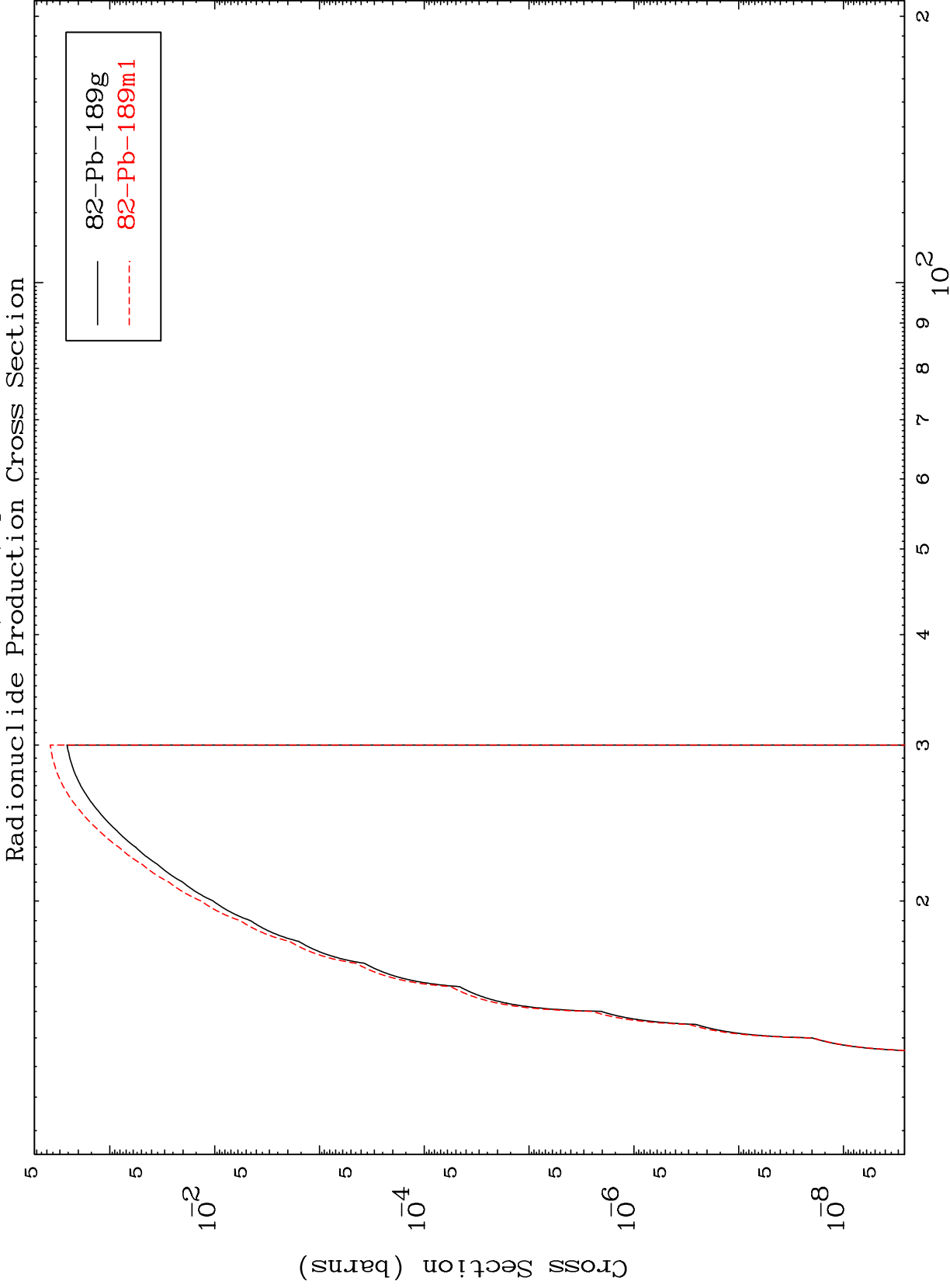
Radionuclide Production Cross Section



34

Incident Energy (MeV)

83-Bi-191

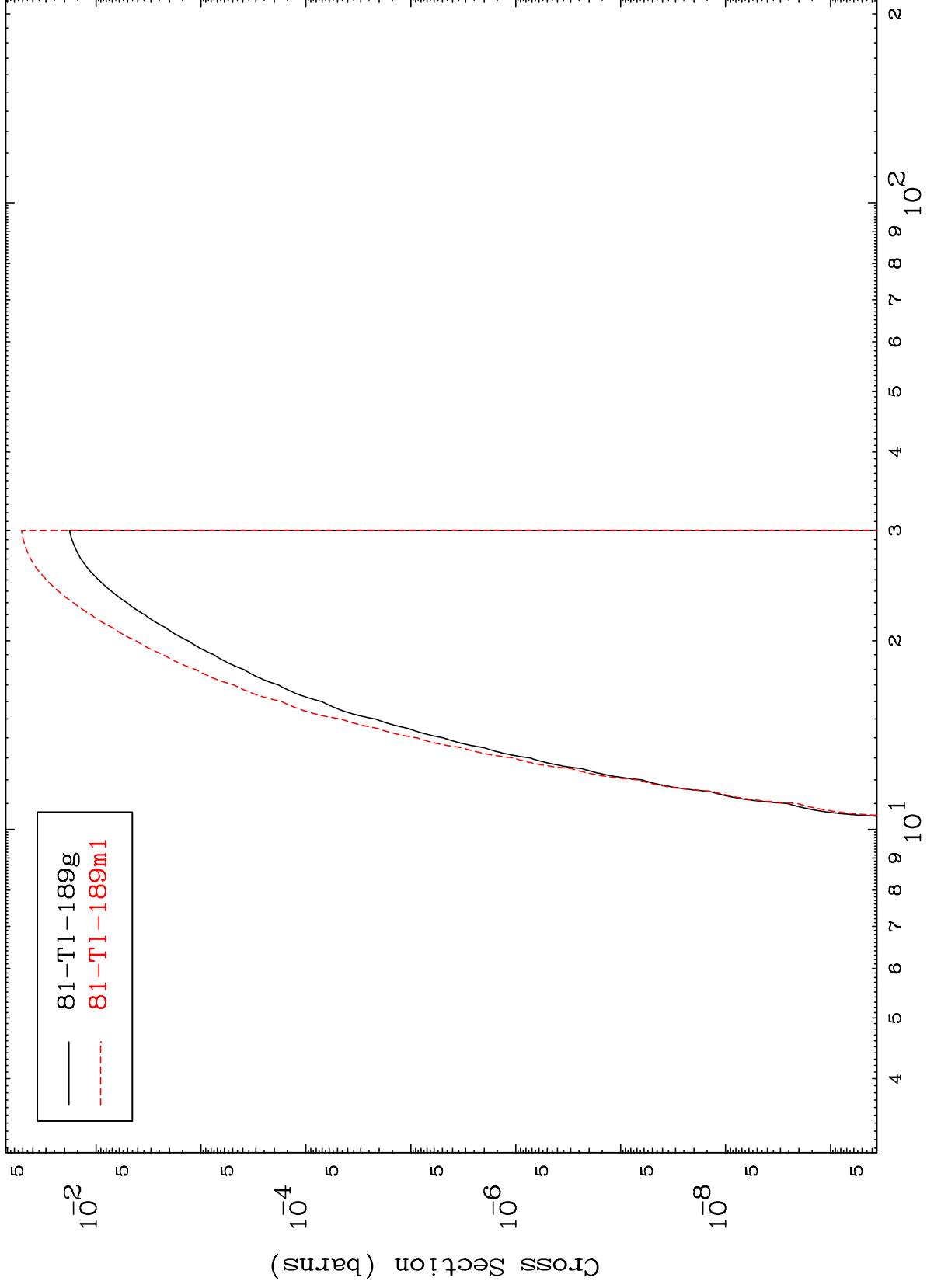


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(n,2n) p

83-Bi-191

Radionuclide Production Cross Section



36

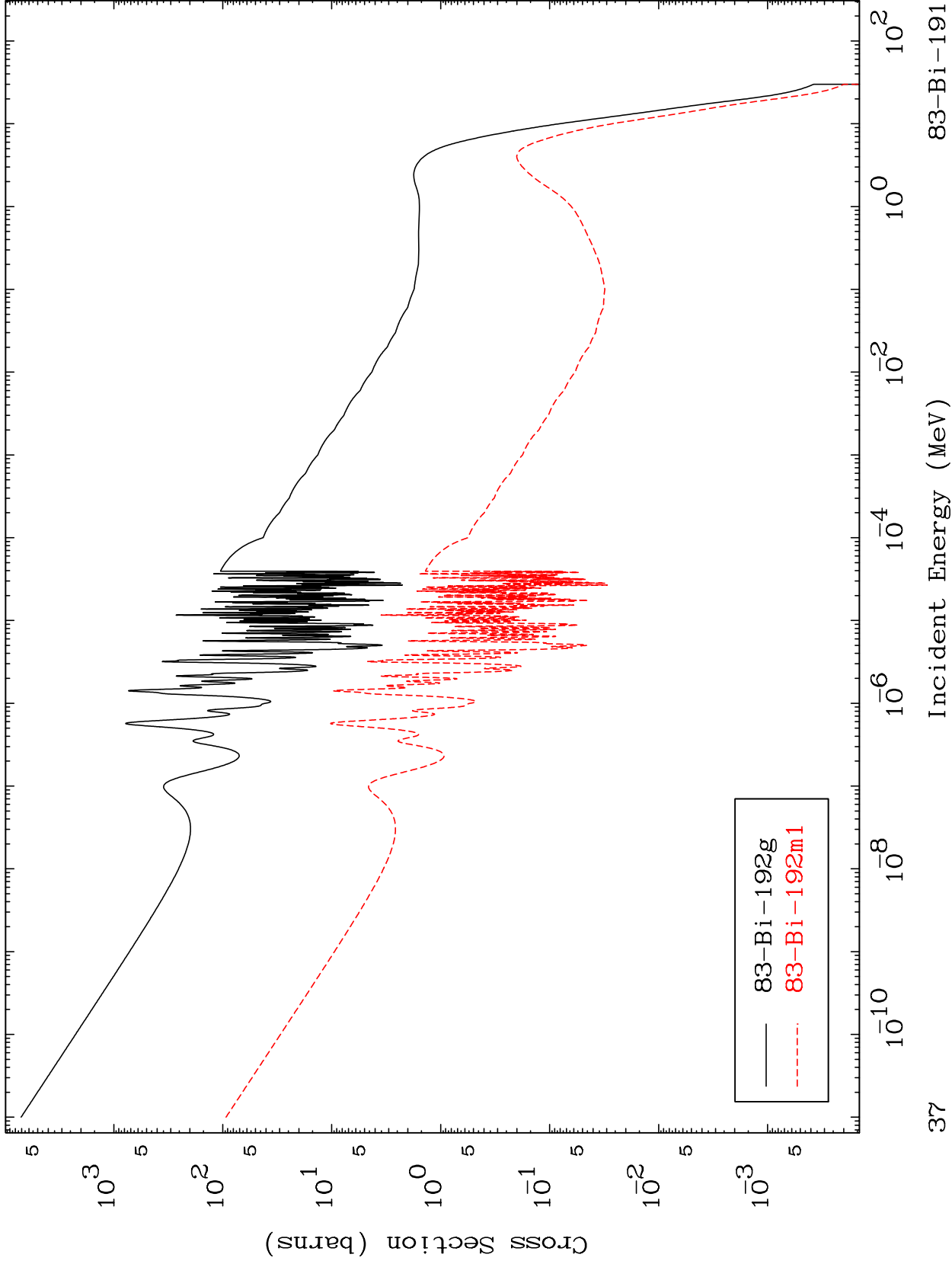
Incident Energy (MeV)

83-Bi-191

MAT 8271

⁸³Bi-191

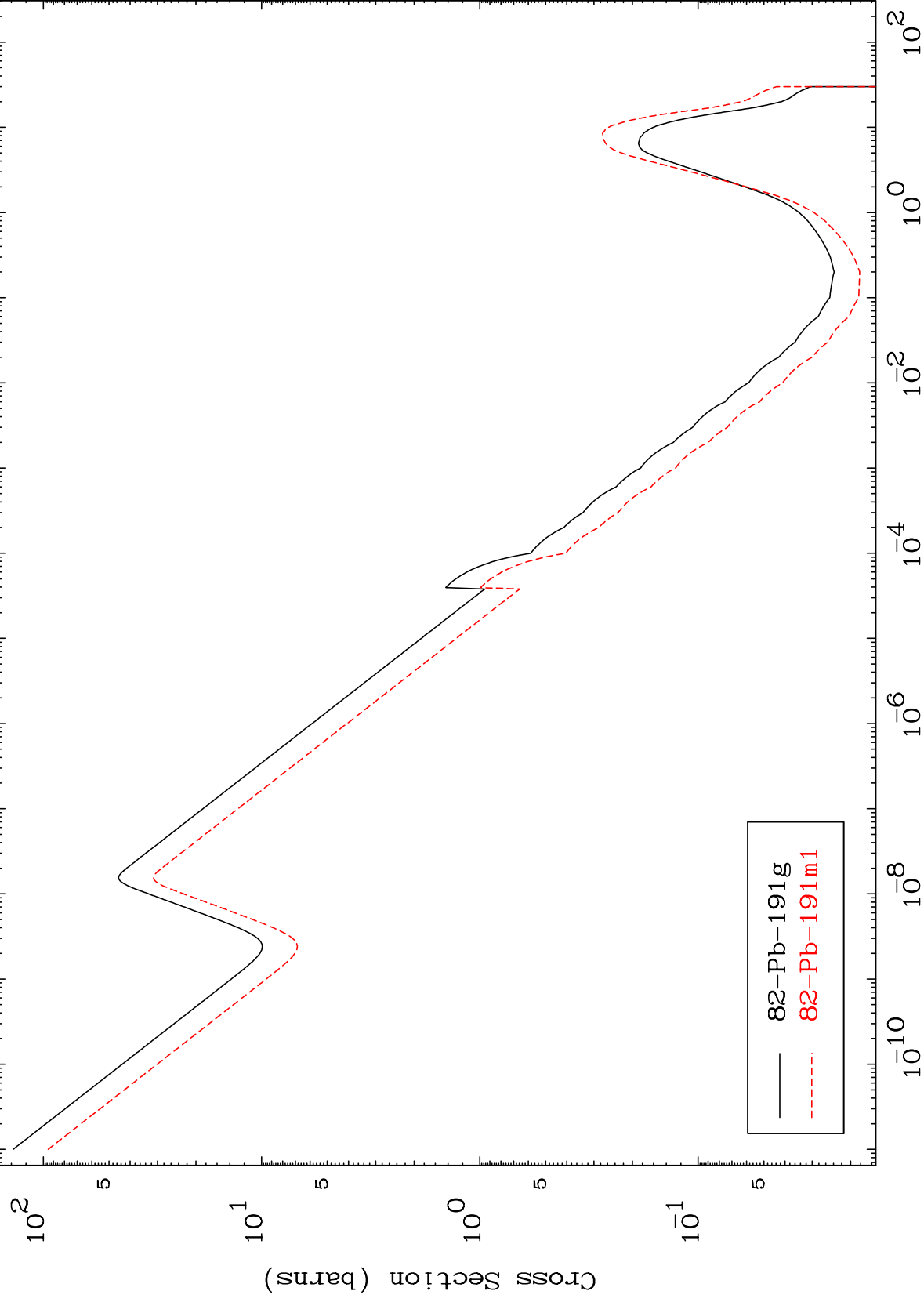
(n,γ)
Radionuclide Production Cross Section



MAT 8271

83-Bi-191

Radionuclide Production Cross Section



38

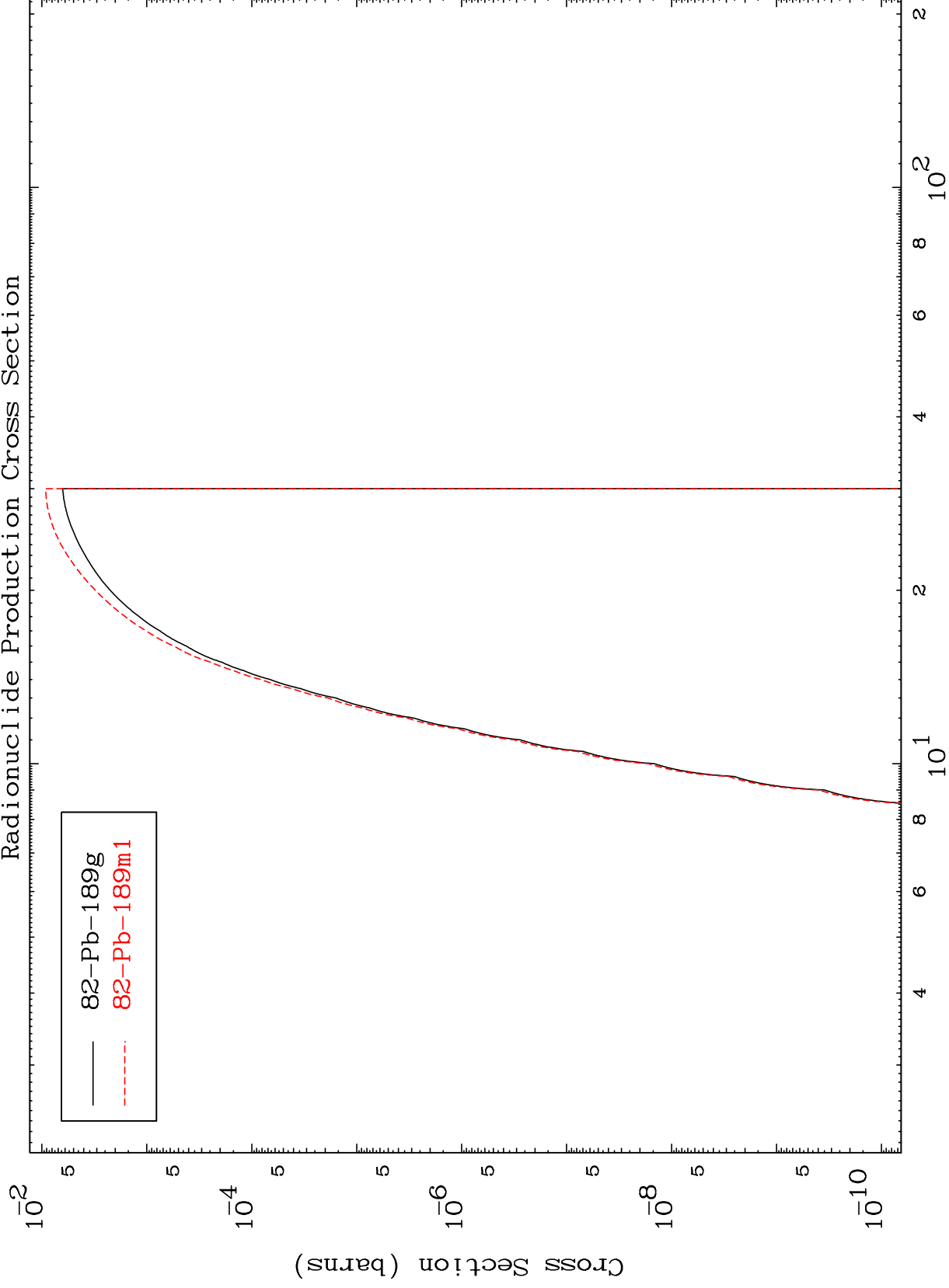
83-Bi-191

Incident Energy (MeV)

MAT 8271

83-Bi-191

Radionuclide Production Cross Section



39

83-Bi-191

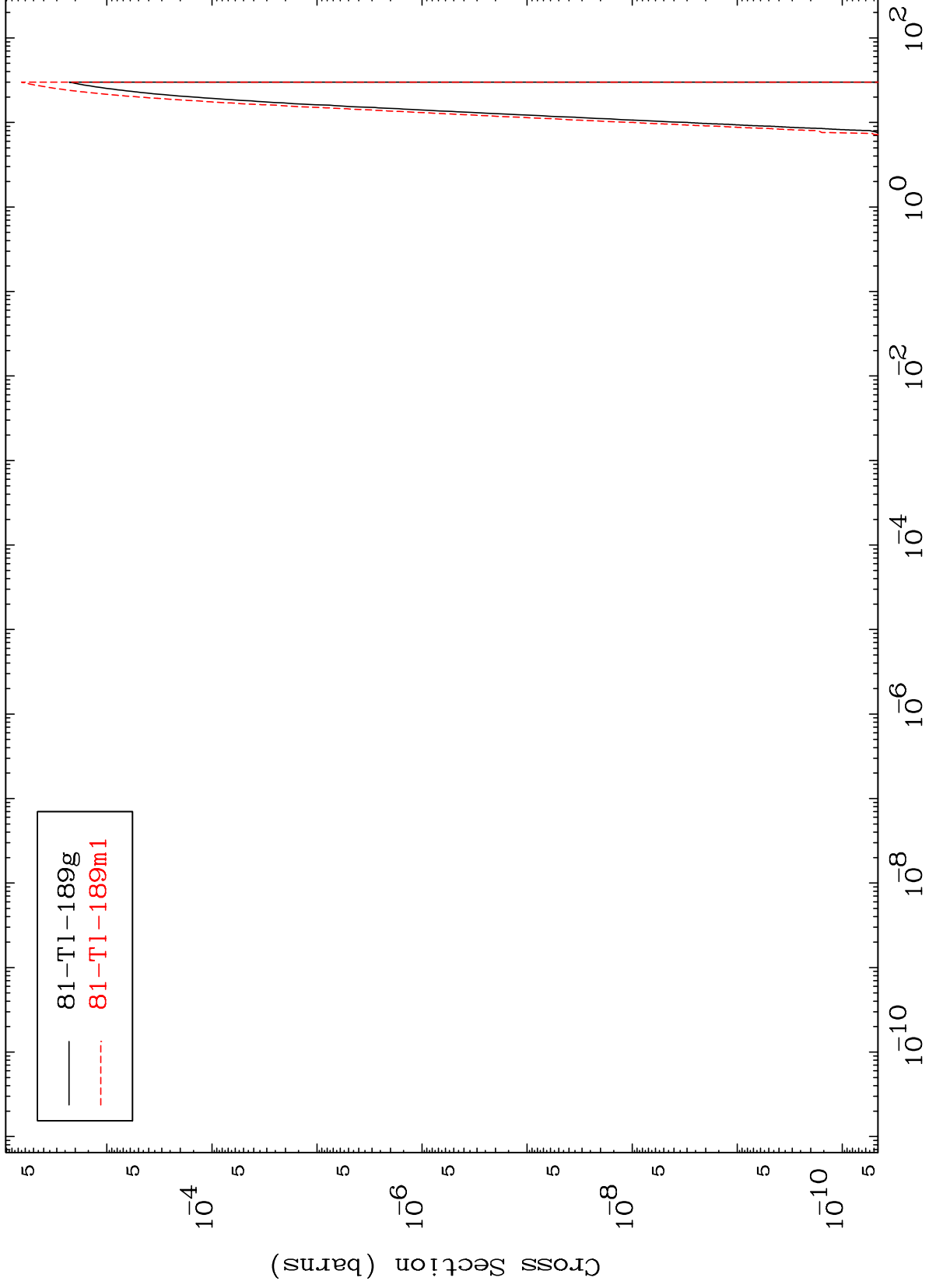
Incident Energy (MeV)

MAT 8271

(n,He-3)

83-Bi-191

Radionuclide Production Cross Section



81-Tl-189g
81-Tl-189m1

40

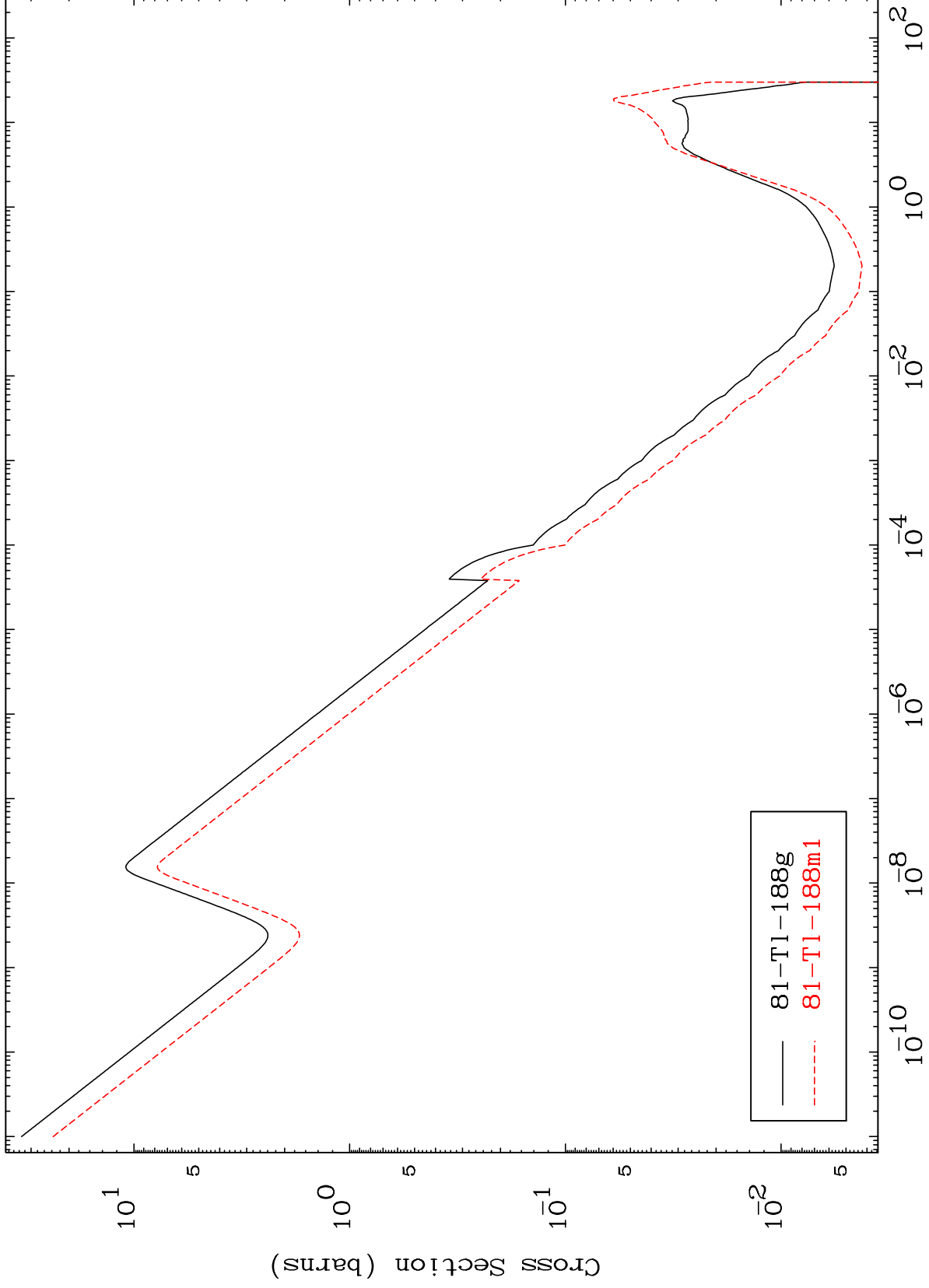
Incident Energy (MeV)

83-Bi-191

MAT 8271

83-Bi-191

(n, α)
Radionuclide Production Cross Section



41

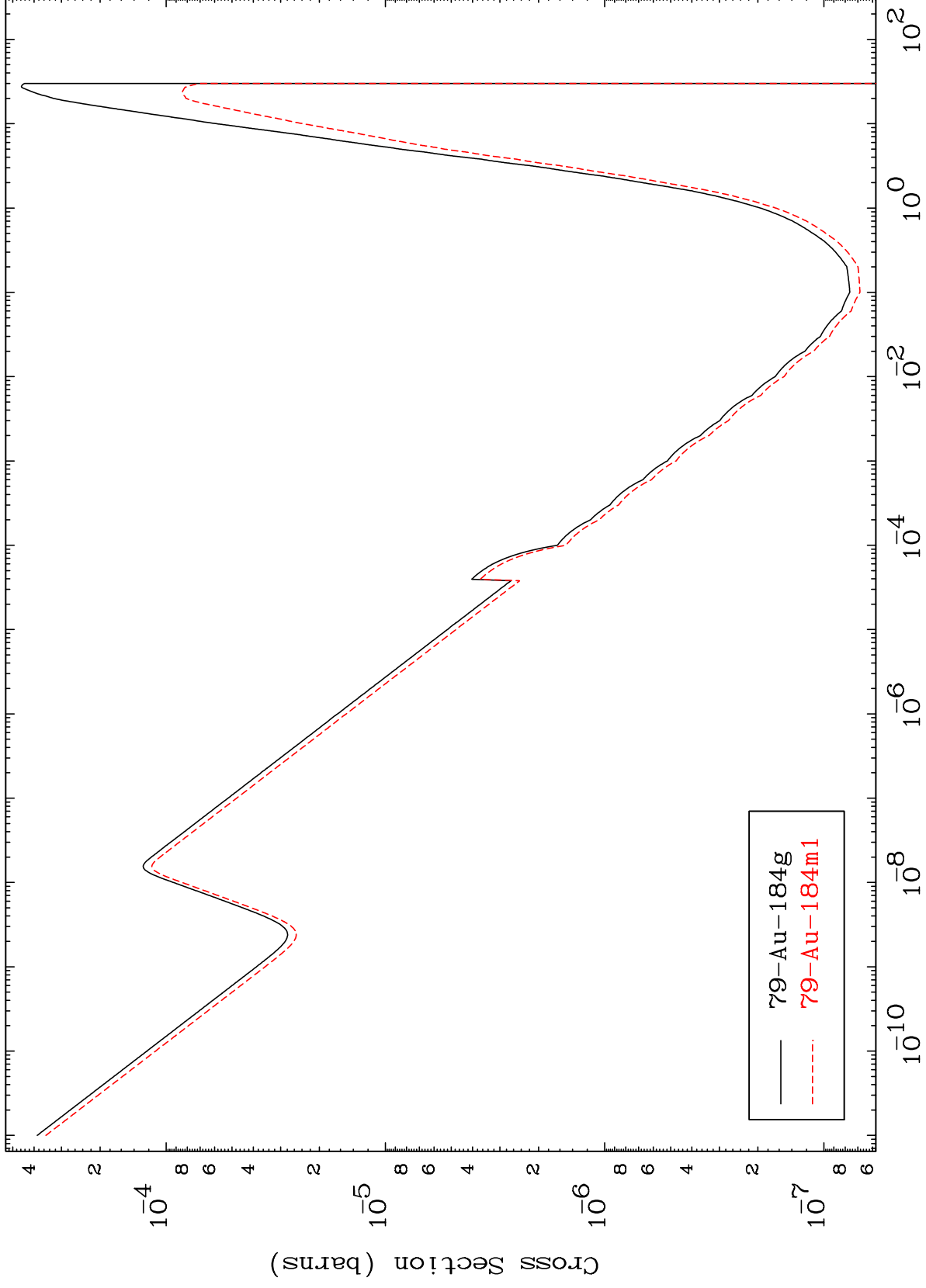
83-Bi-191

Incident Energy (MeV)

MAT 8271

83-Bi-191

(n,2α)
Radionuclide Production Cross Section



83-Bi-191

Incident Energy (MeV)

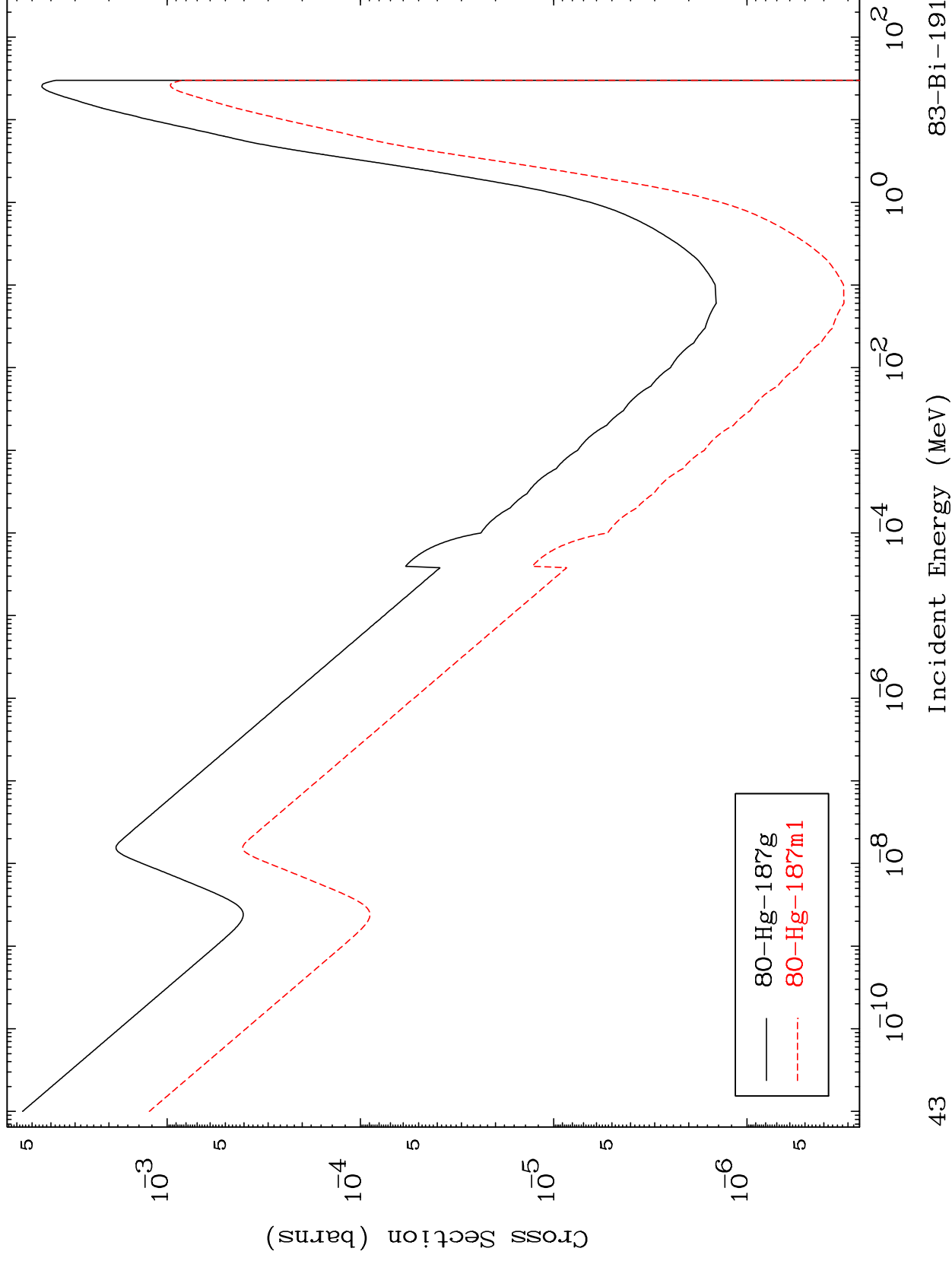
42

MAT 8271

(n,p) α

83-Bi-191

Radionuclide Production Cross Section



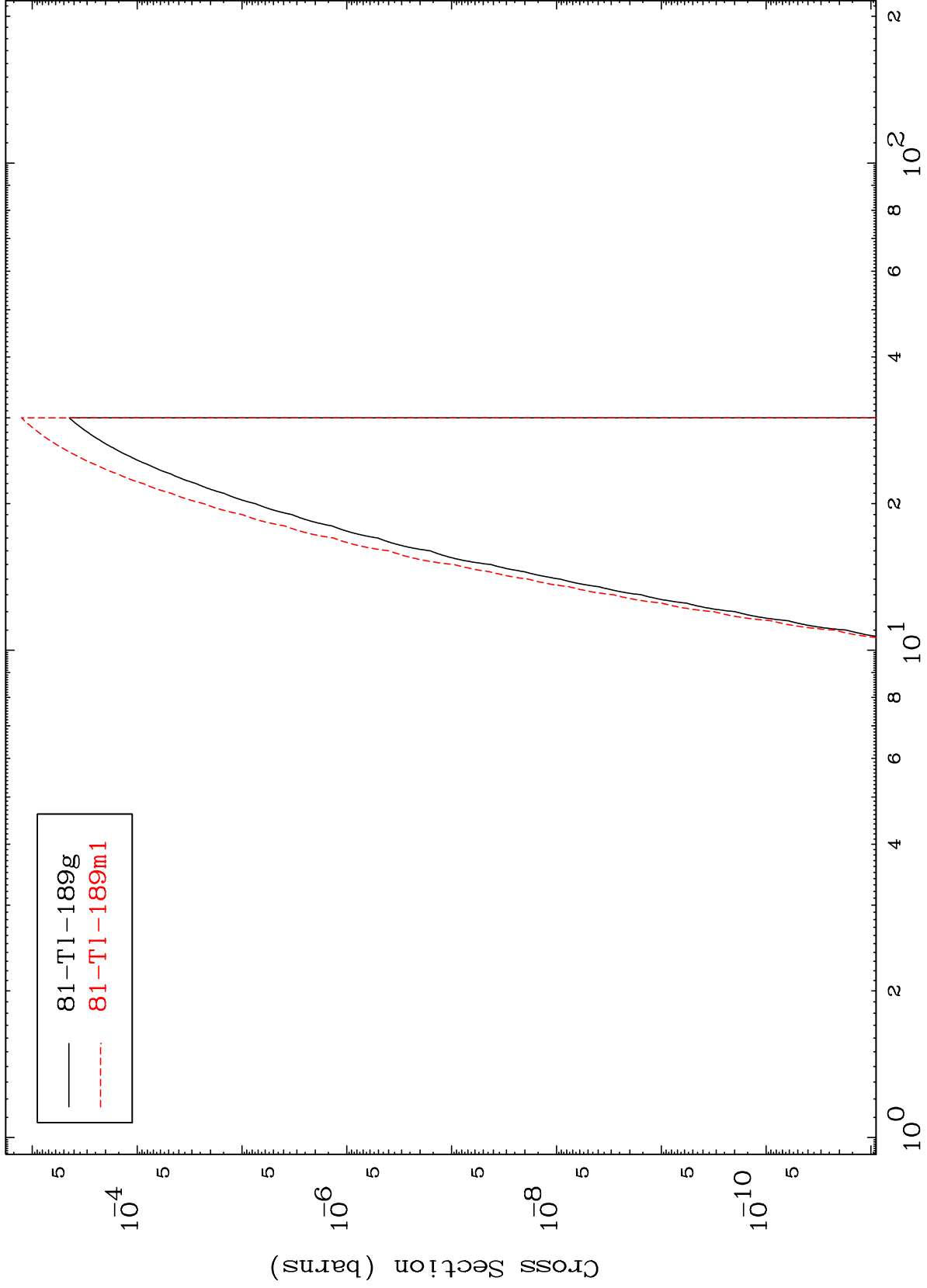
43

MAT 8271

(n,p) d

83-Bi-191

Radionuclide Production Cross Section



44

Incident Energy (MeV)

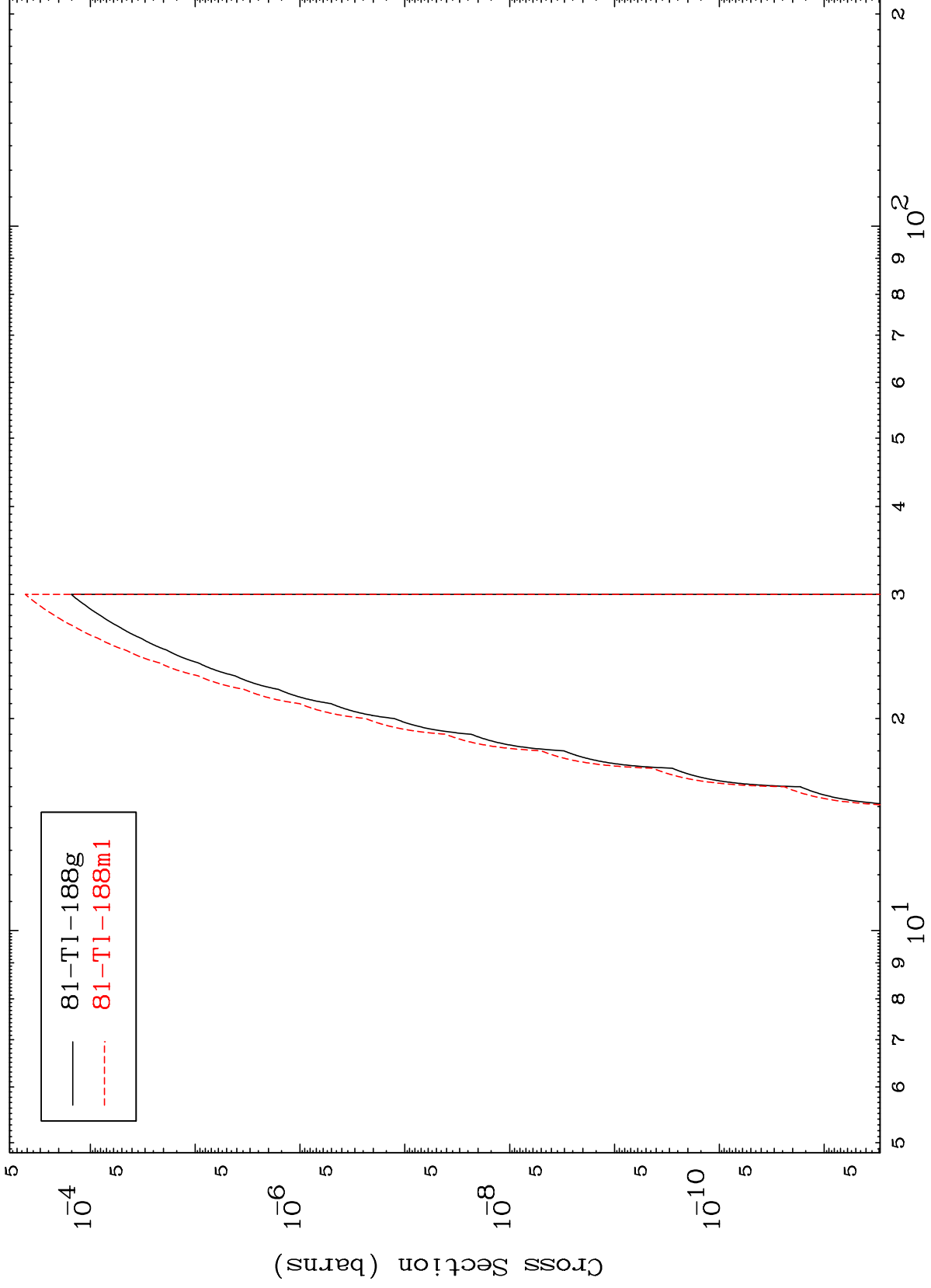
83-Bi-191

MAT 8271

(n,p) t

83-Bi-191

Radionuclide Production Cross Section



45

Incident Energy (MeV)

83-Bi-191