

Program EVALPLOT
(Version 2015-2)

by

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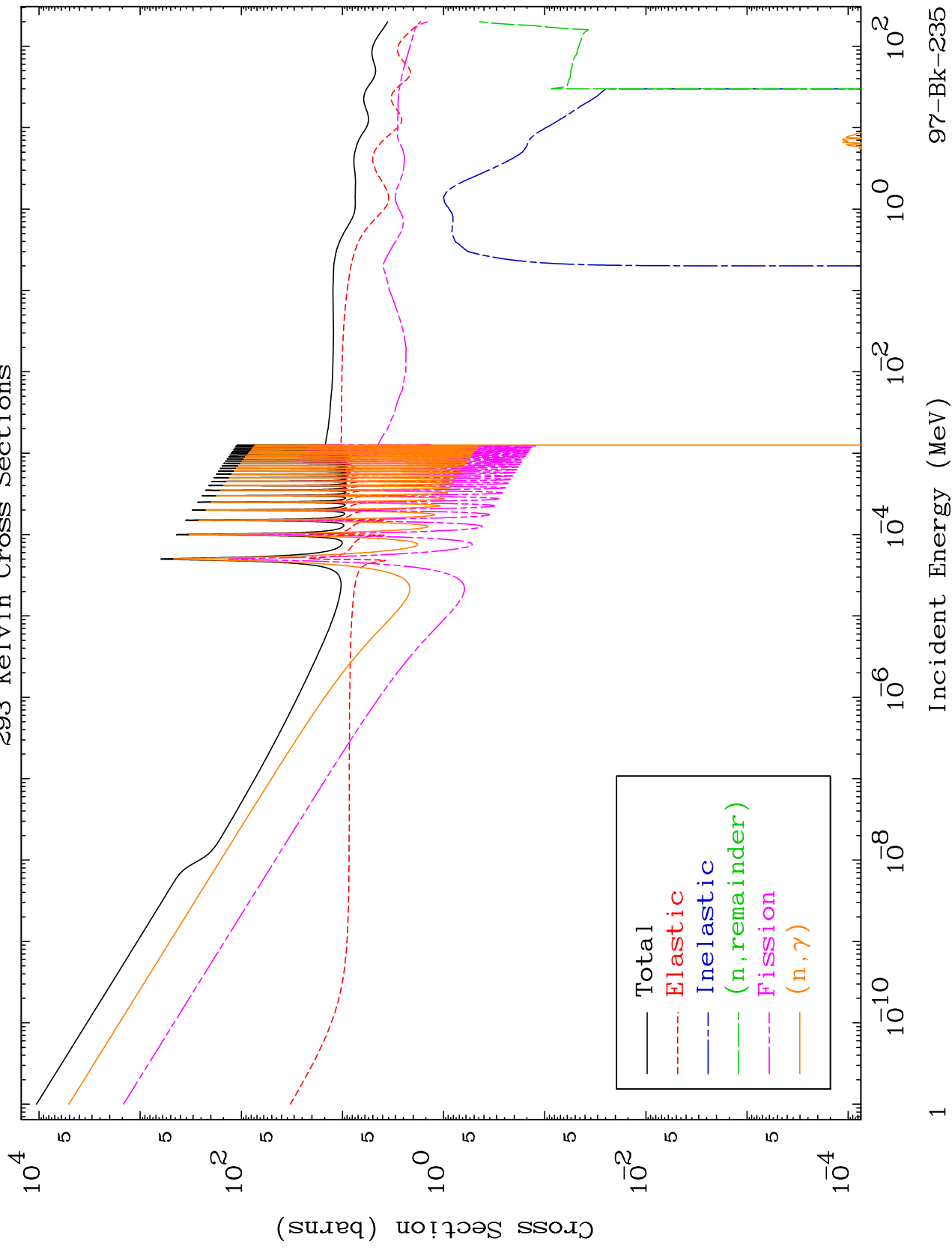
Press Mouse Button to Start

MAT 9710

Major

293 Kelvin Cross Sections

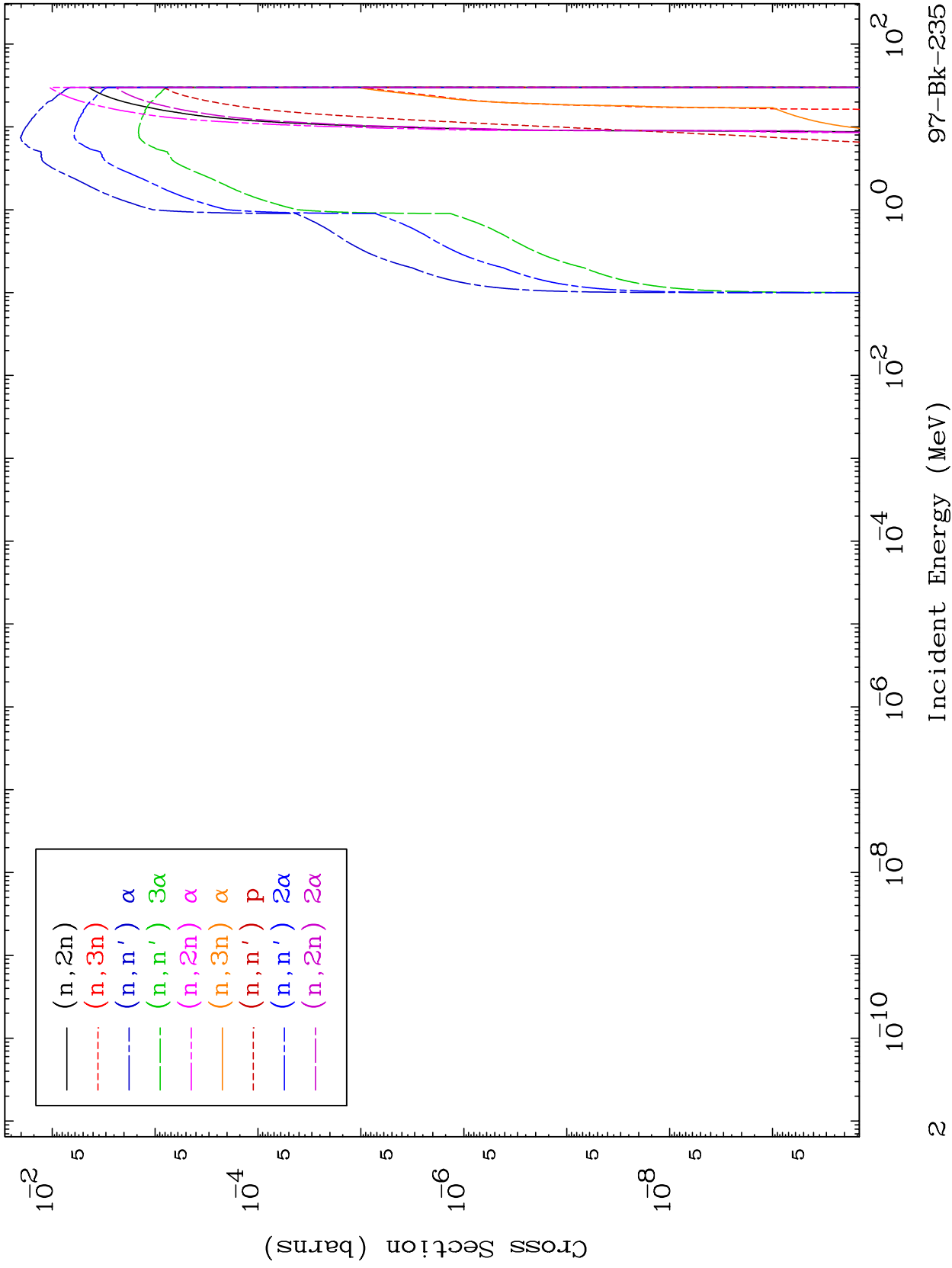
97-Bk-235



MAT 9710

Neutron Production
293 Kelvin Cross Sections

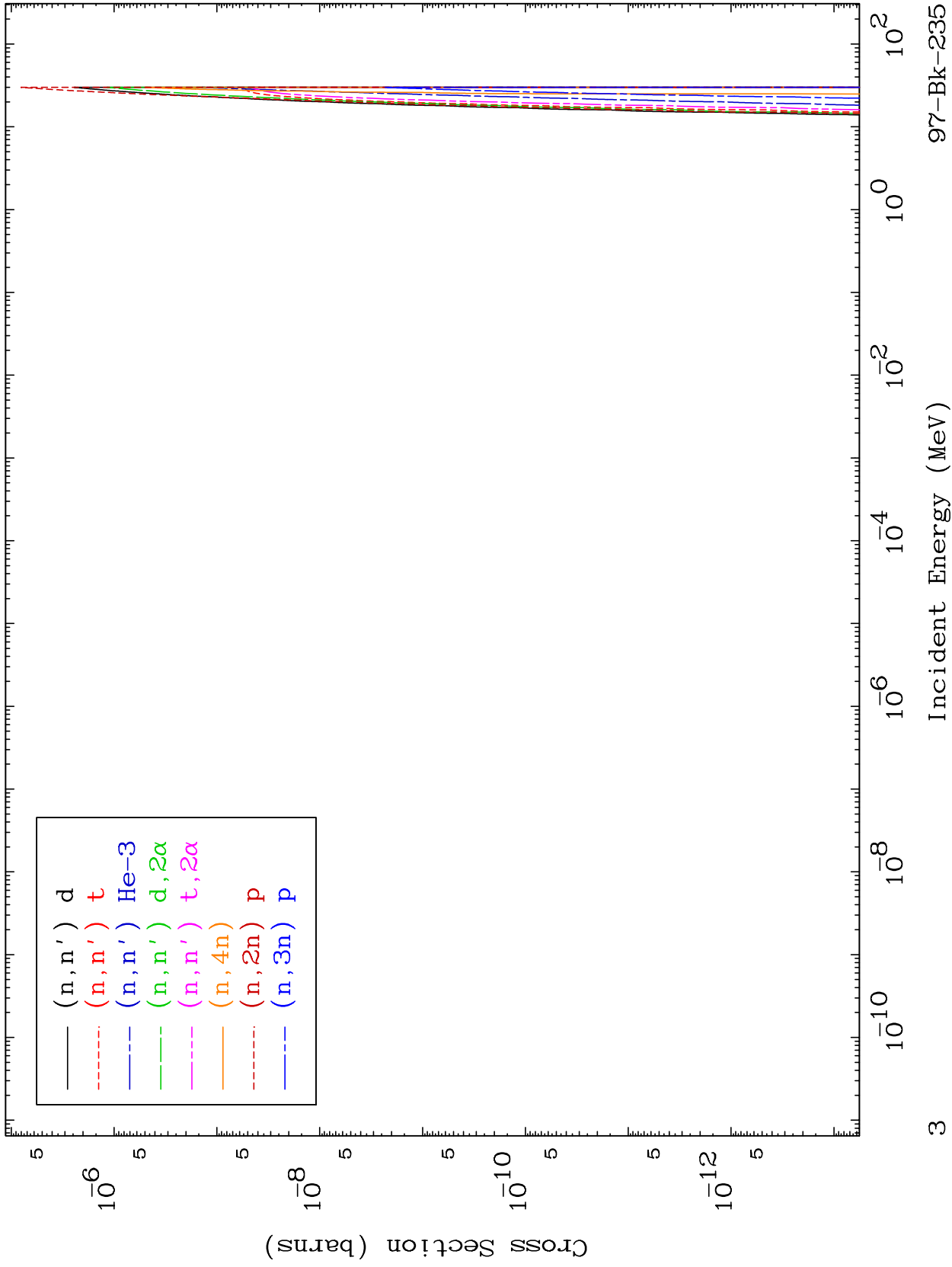
97-Bk-235



MAT 9710

Neutron Production
293 Kelvin Cross Sections

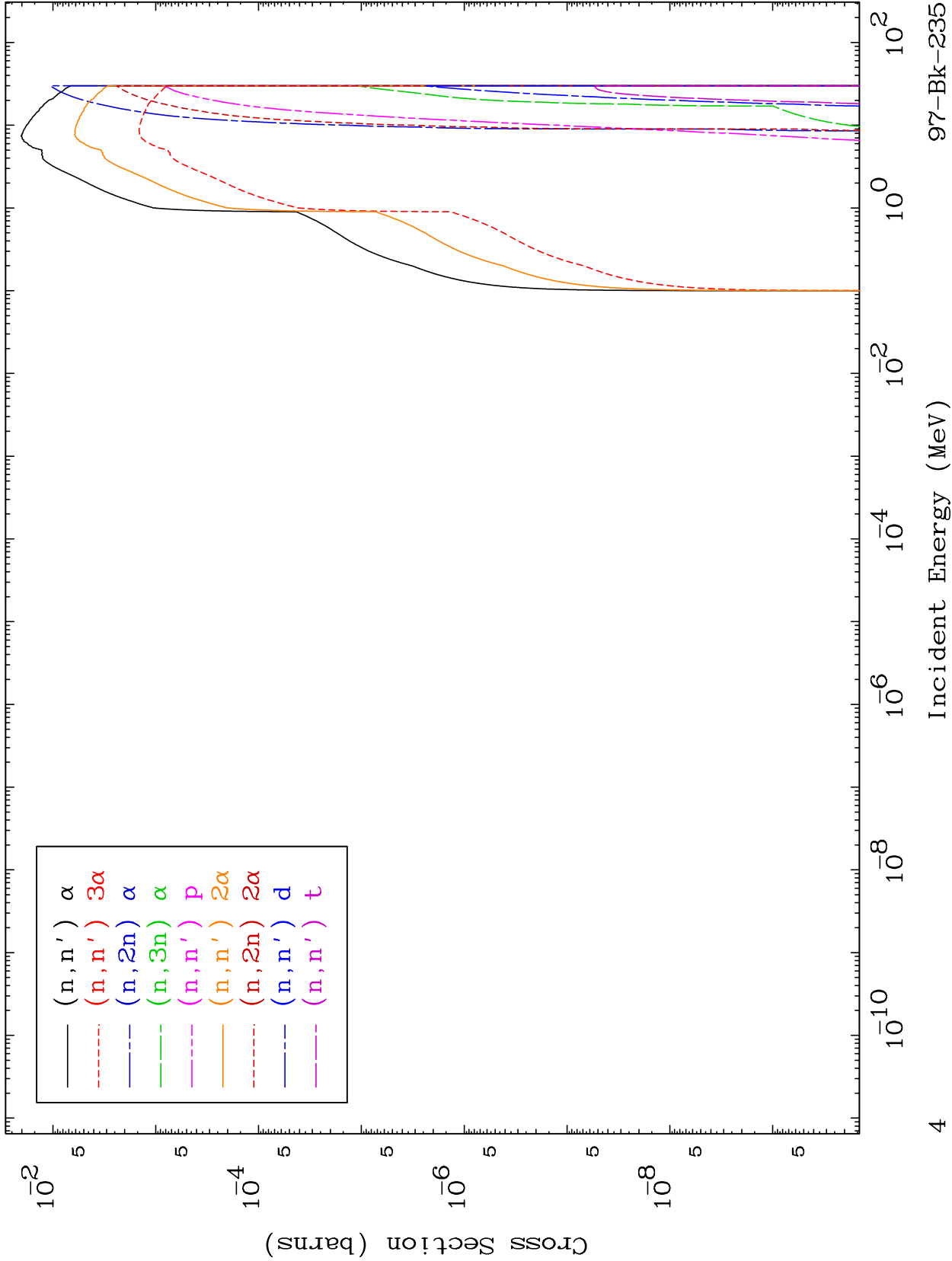
97-Bk-235



MAT 9710

Charged Particle
293 Kelvin Cross Sections

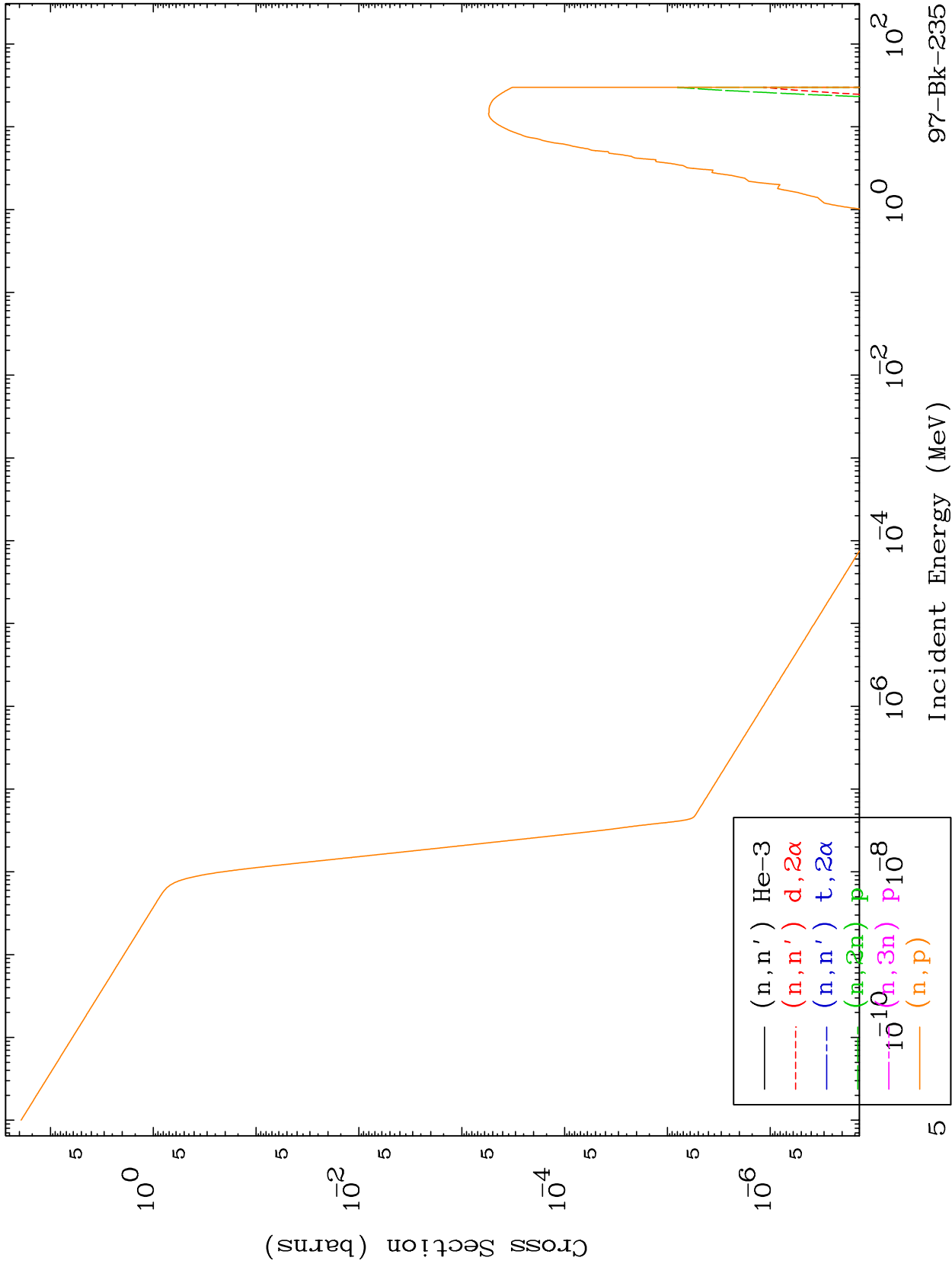
97-Bk-235



MAT 9710

Charged Particle
293 Kelvin Cross Sections

97-Bk-235



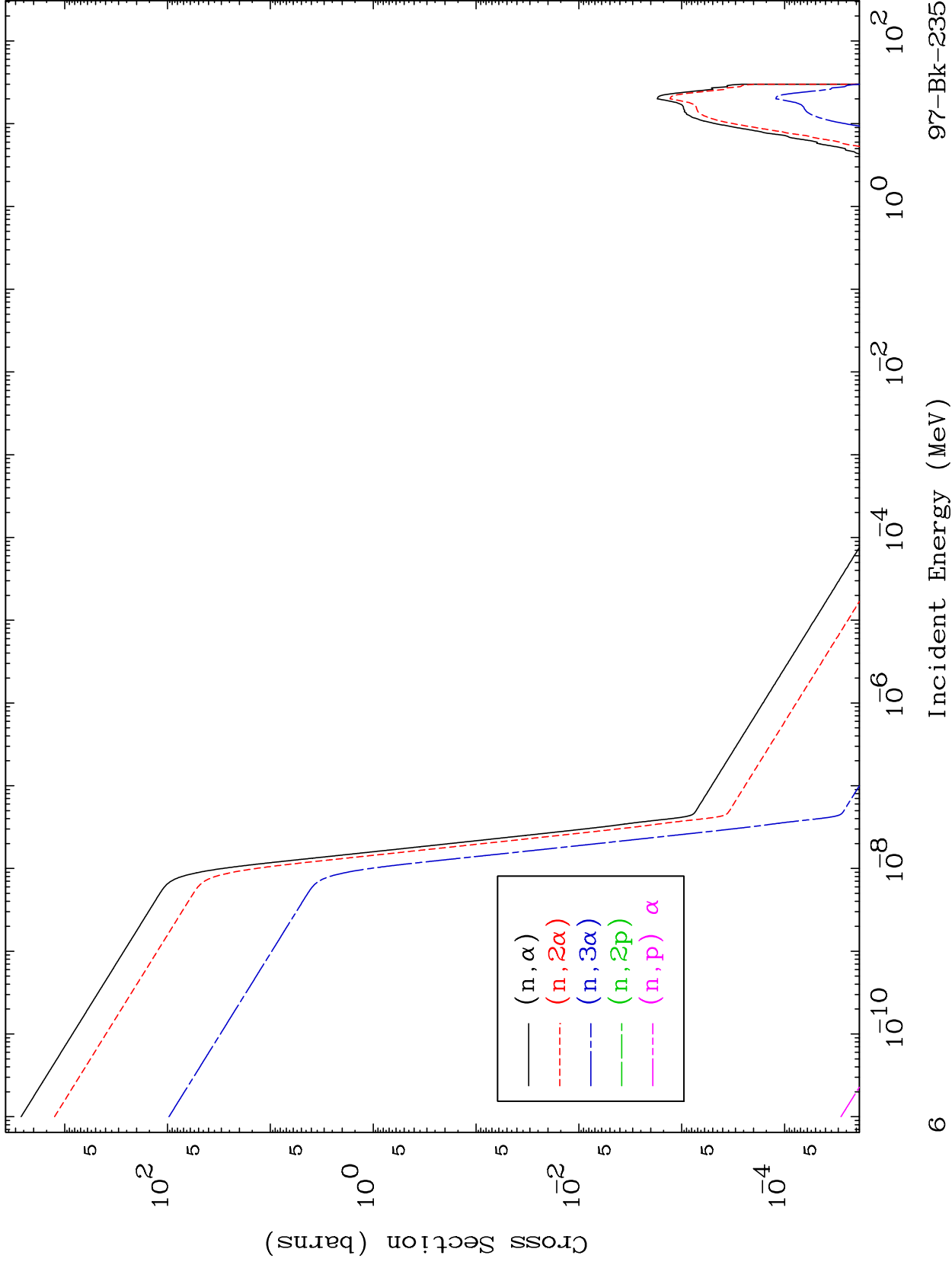
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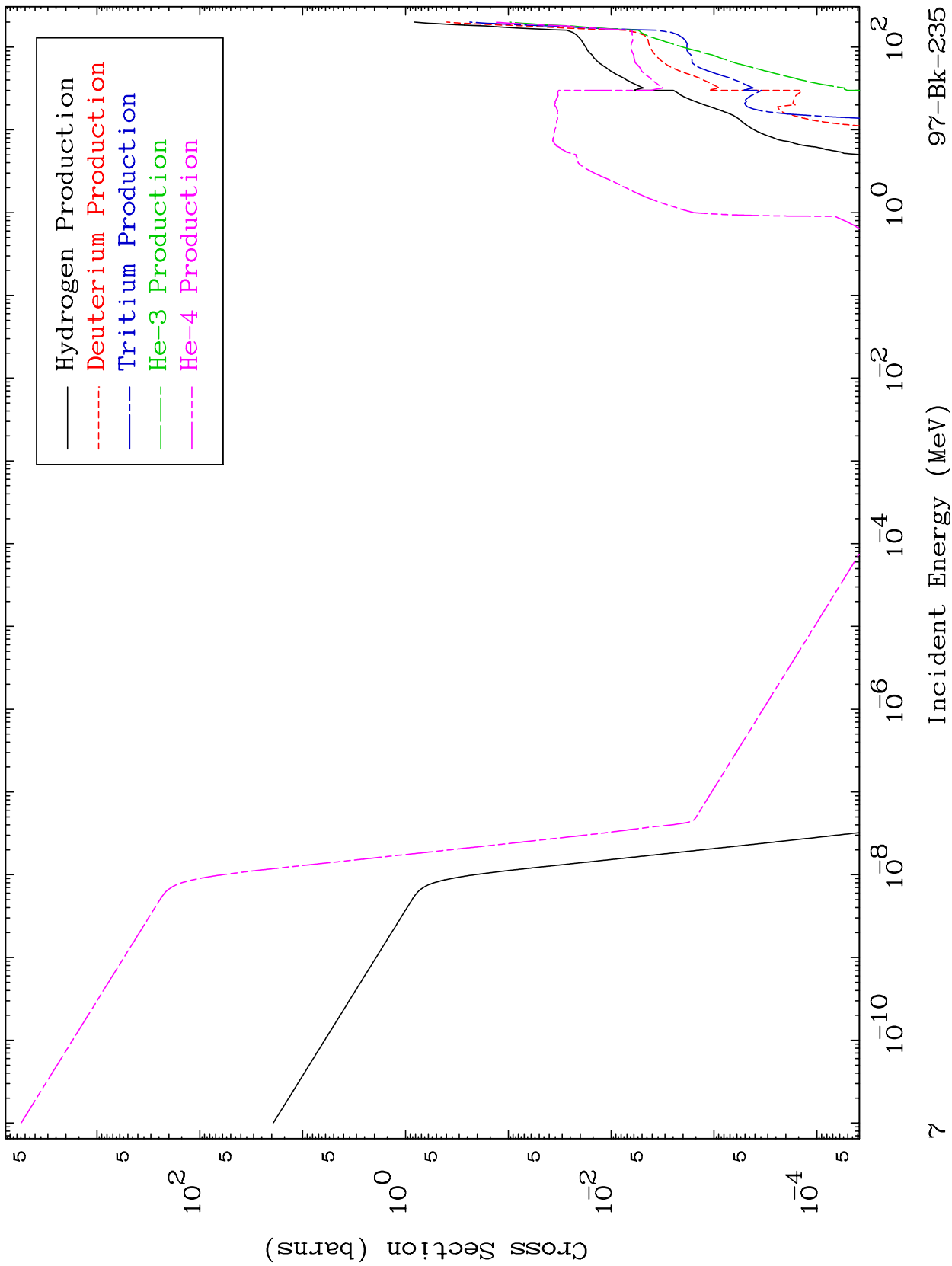
Incident Energy (MeV)

MAT 9710

Charged Particle
293 Kelvin Cross Sections

97-Bk-235

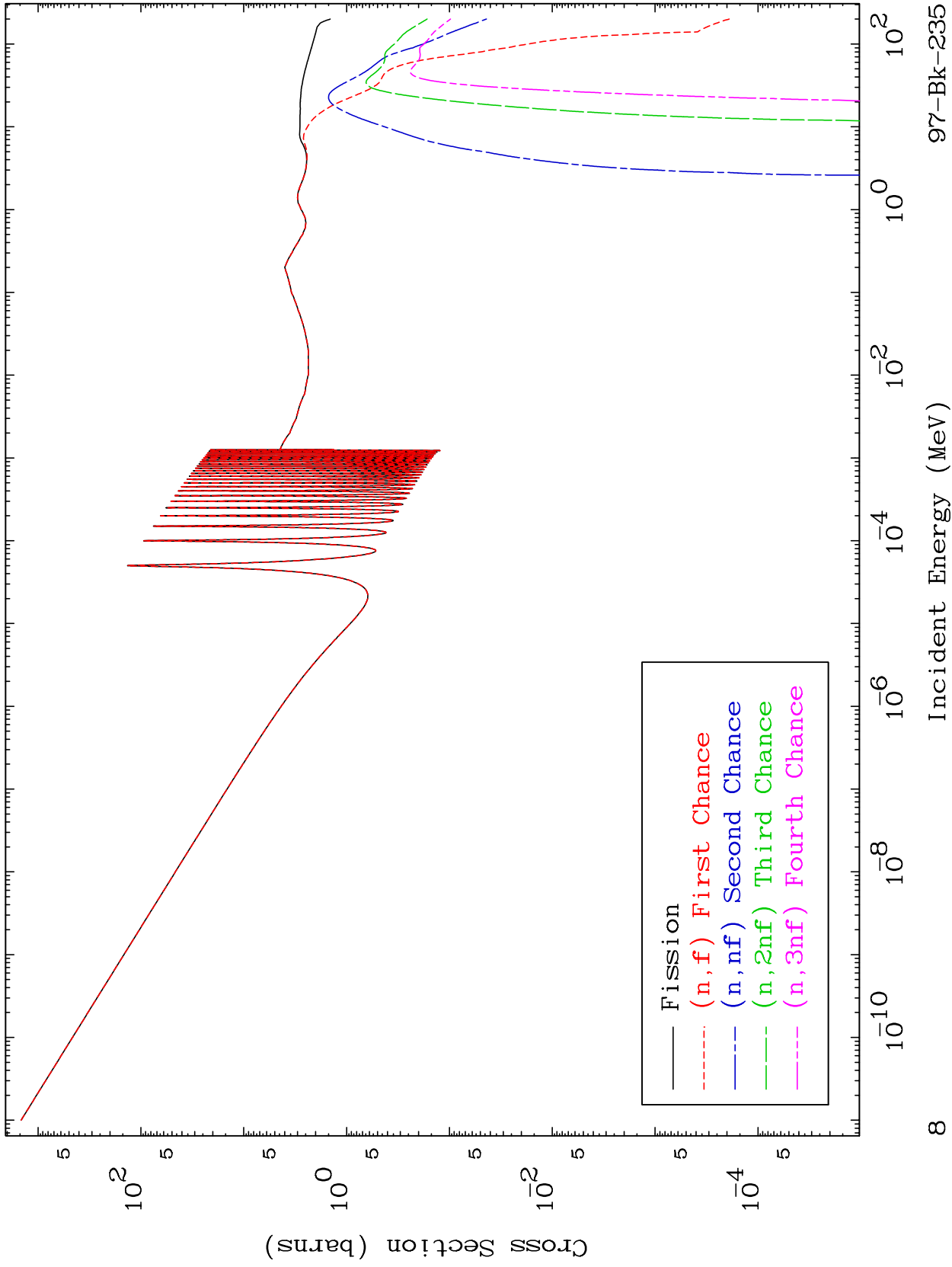




MAT 9710

Fission
293 Kelvin Cross Sections

97-Bk-235

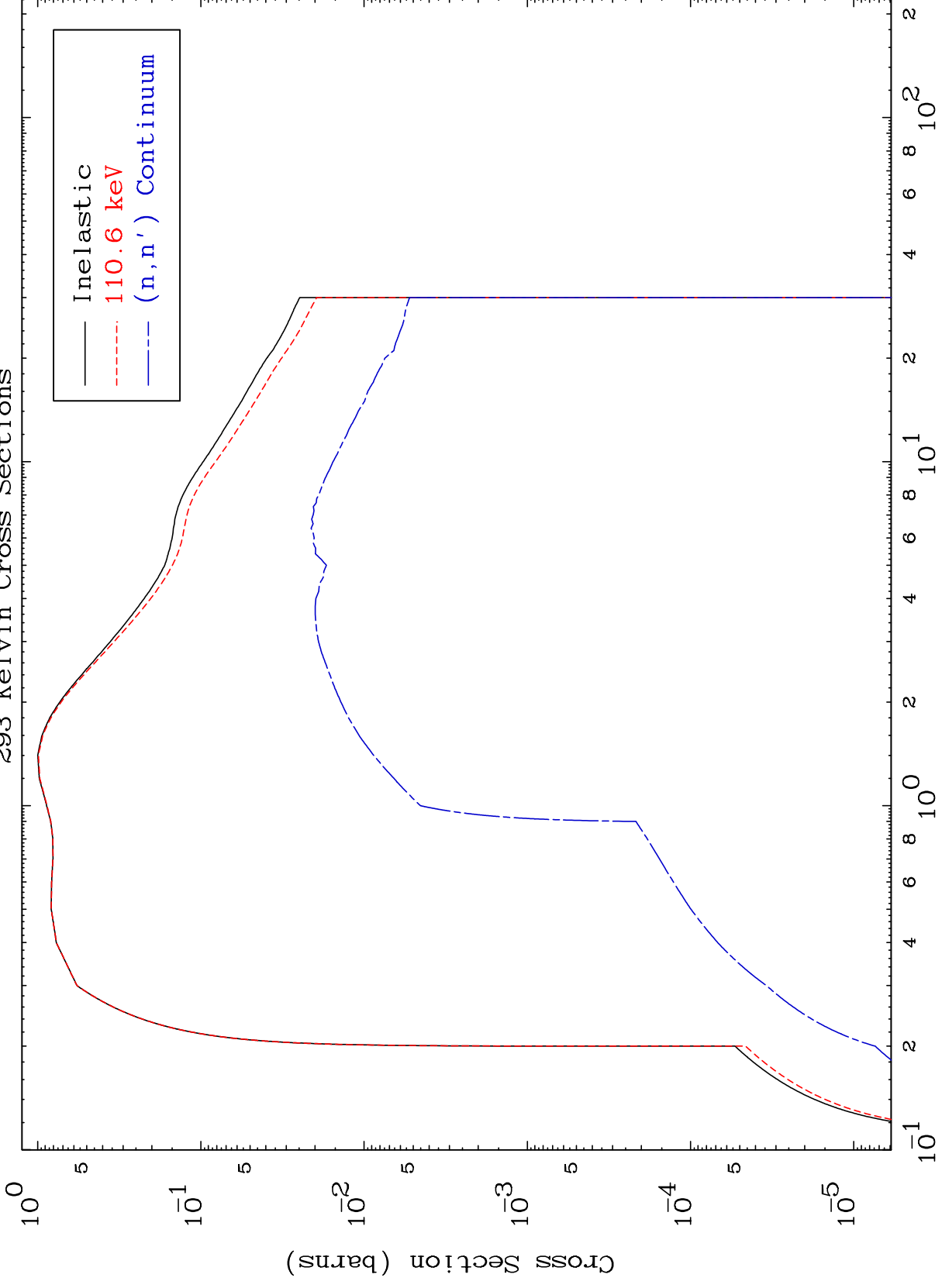


MAT 9710

(n,n') Level

97-Bk-235

293 Kelvin Cross Sections



Incident Energy (MeV)

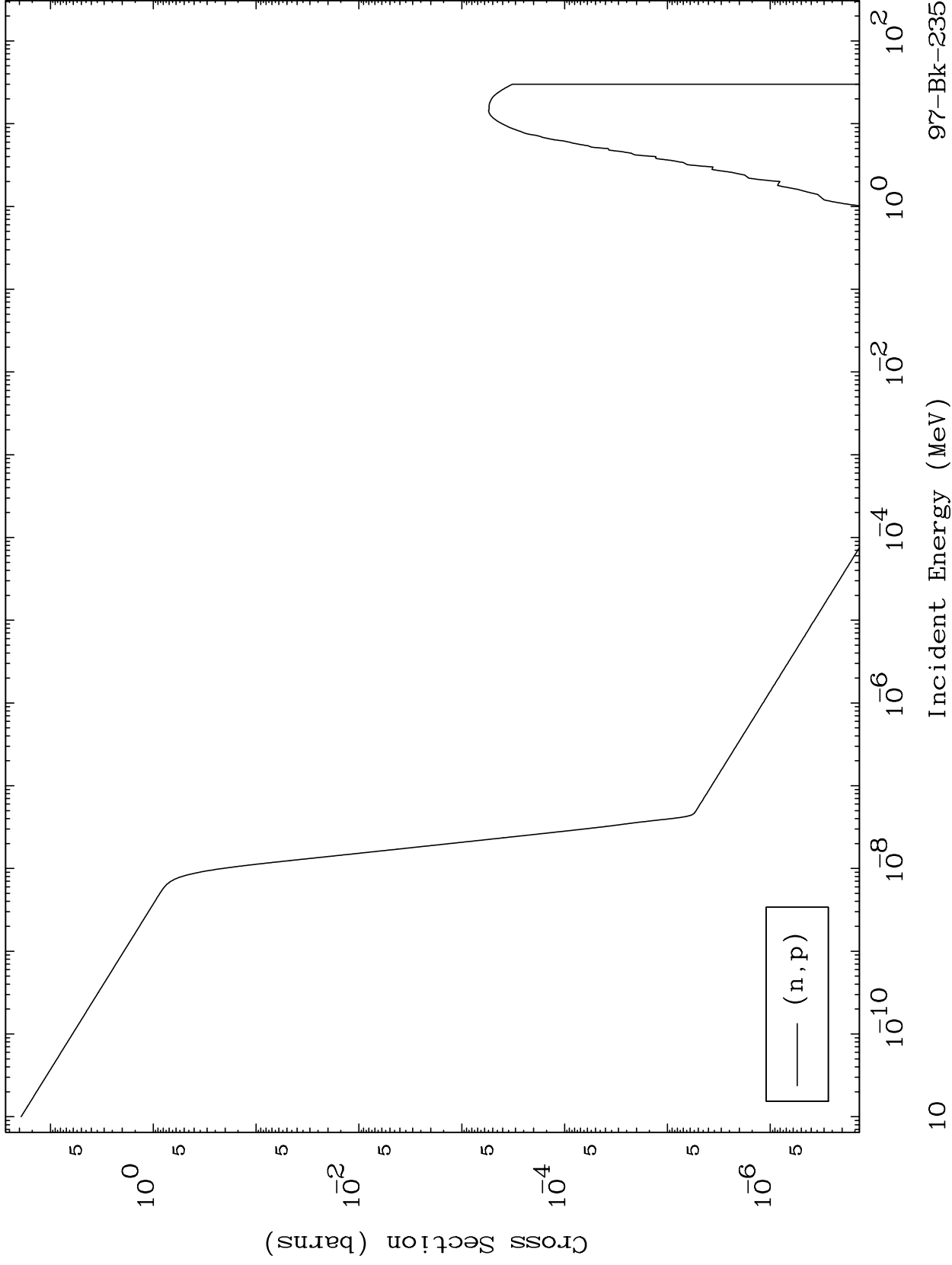
97-Bk-235

9

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(n,p) Levels
293 Kelvin Cross Sections

97-Bk-235

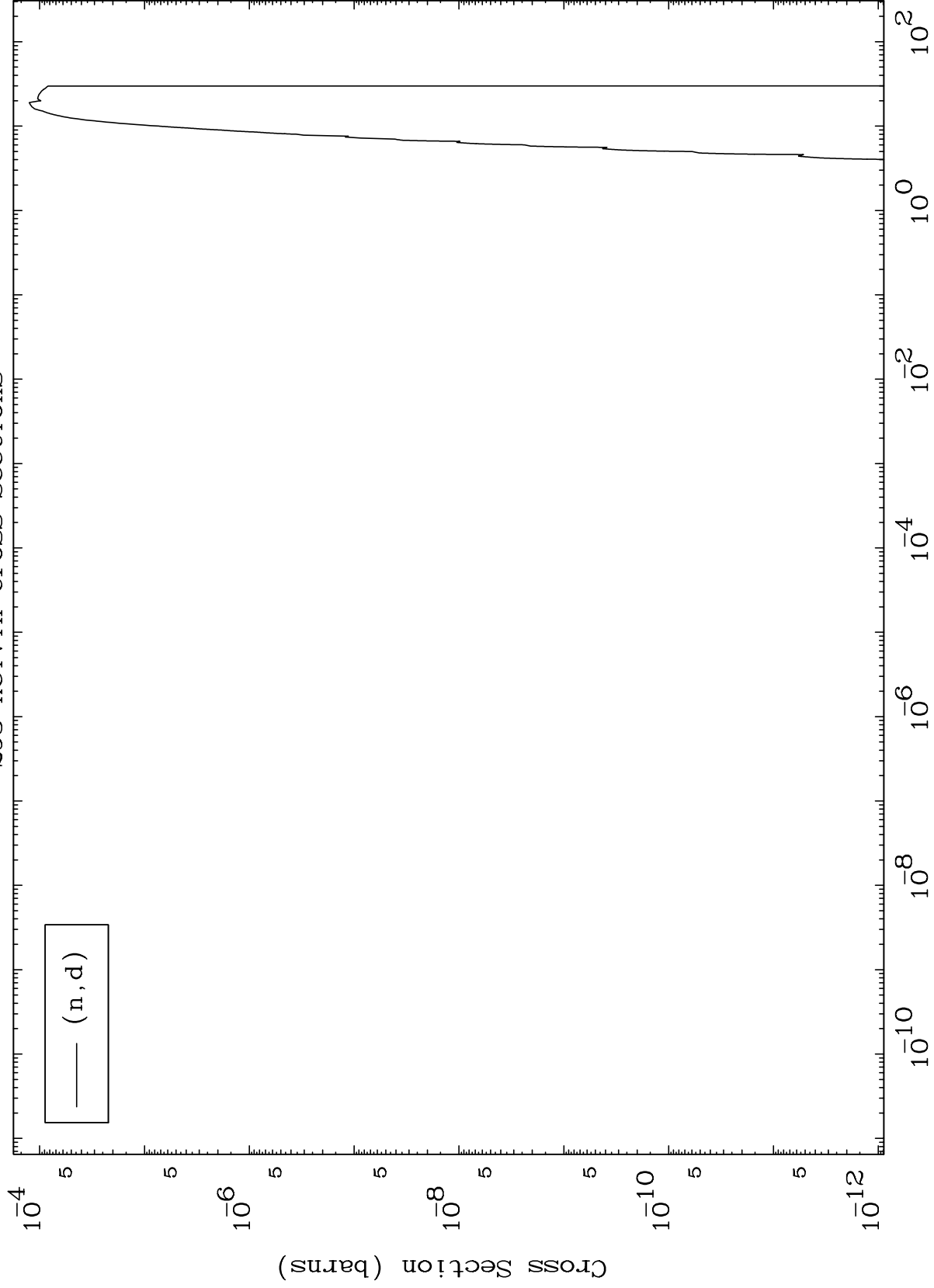


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MAT 9710

(n,d) Levels
293 Kelvin Cross Sections

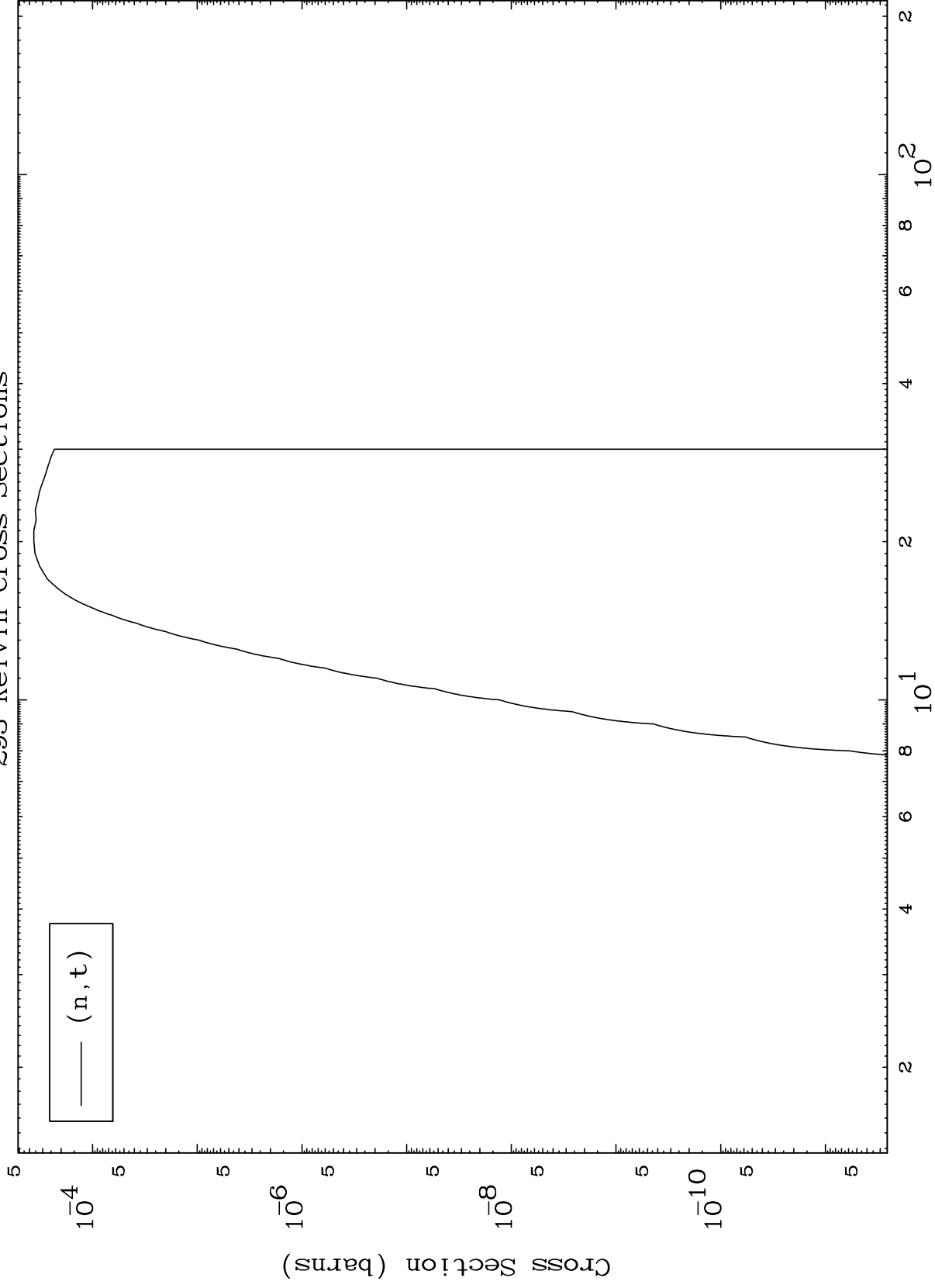
97-Bk-235



MAT 9710

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-235



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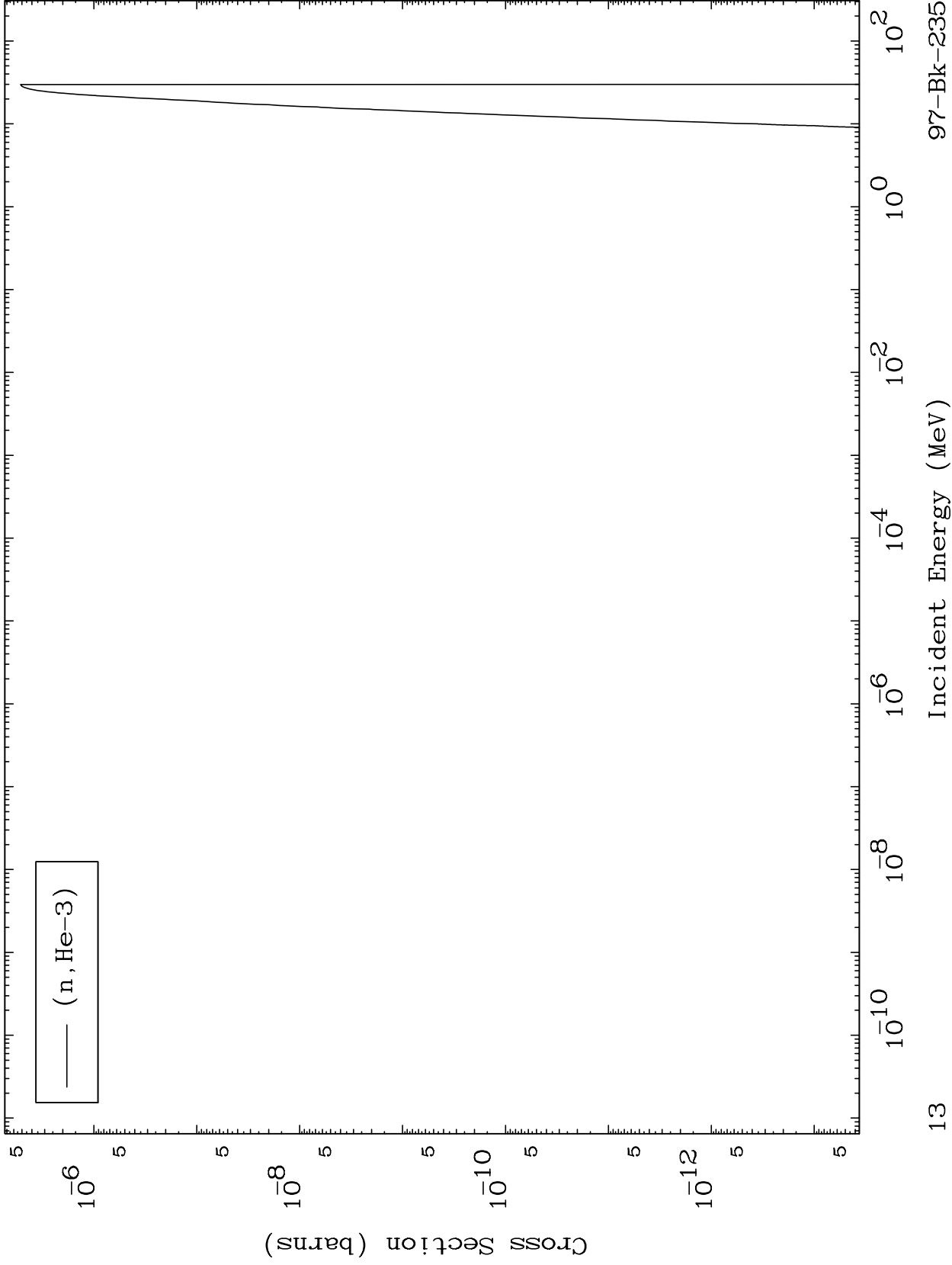
Incident Energy (MeV)

97-Bk-235

MAT 9710

(n,He3) Levels
293 Kelvin Cross Sections

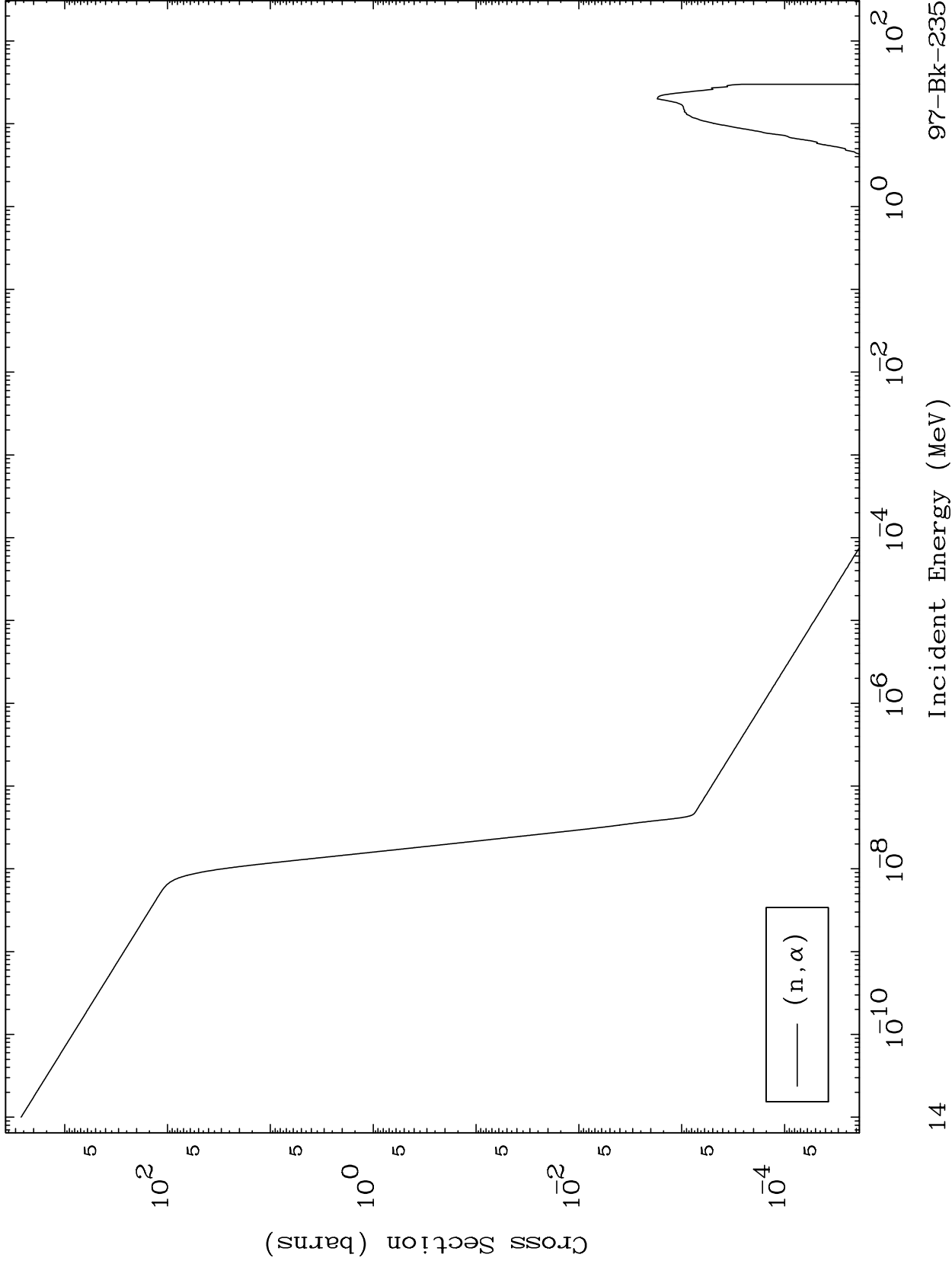
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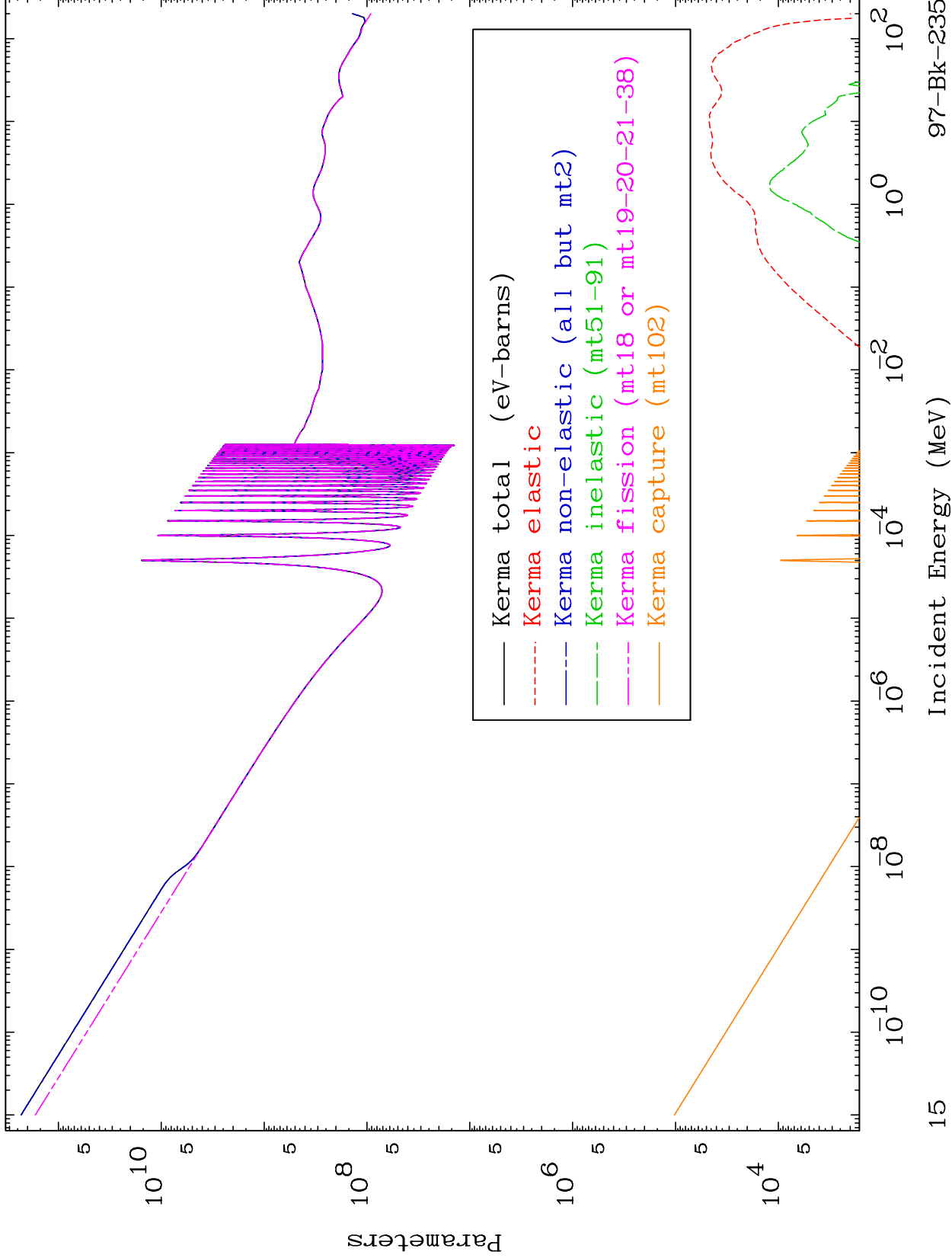


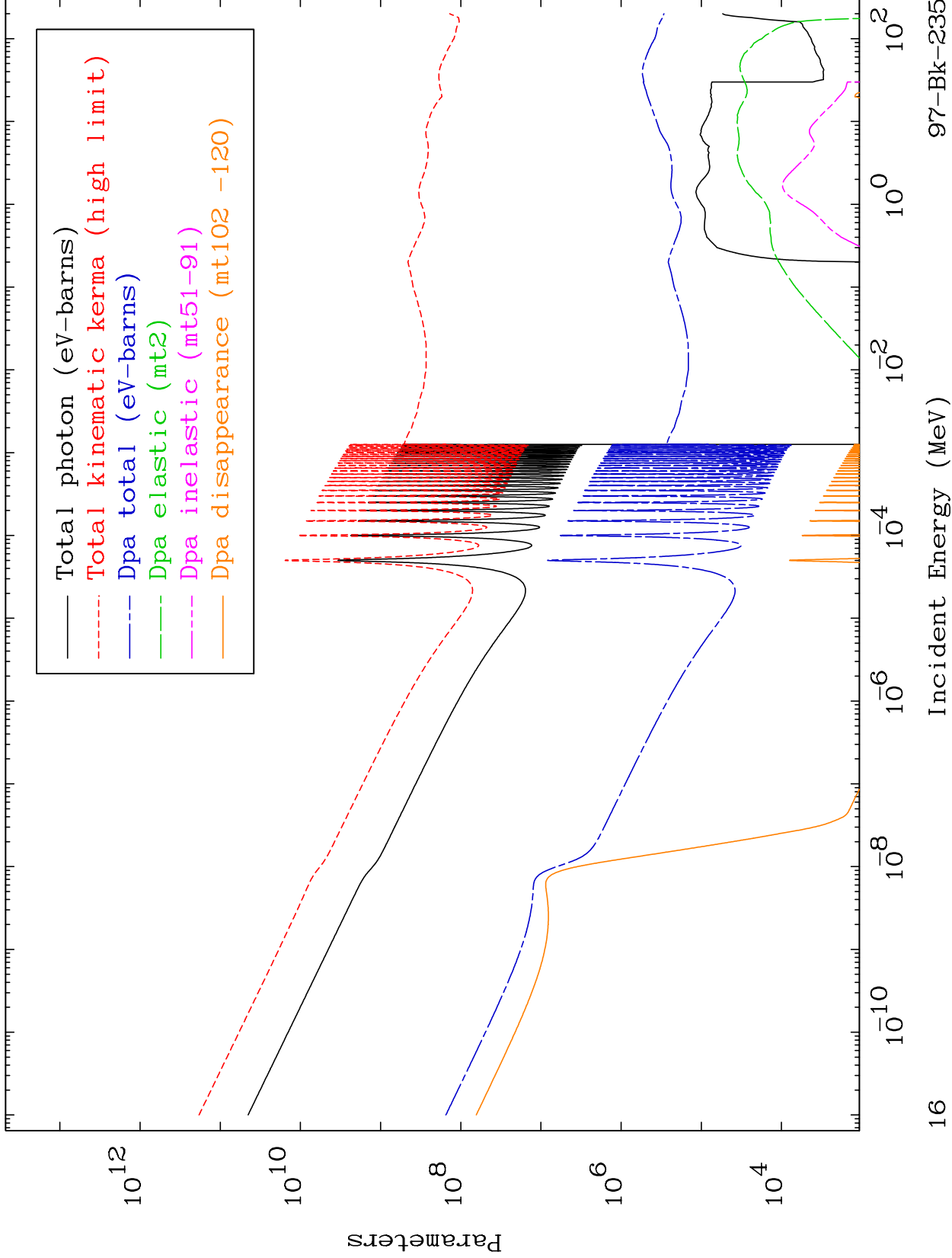
MAT 9710

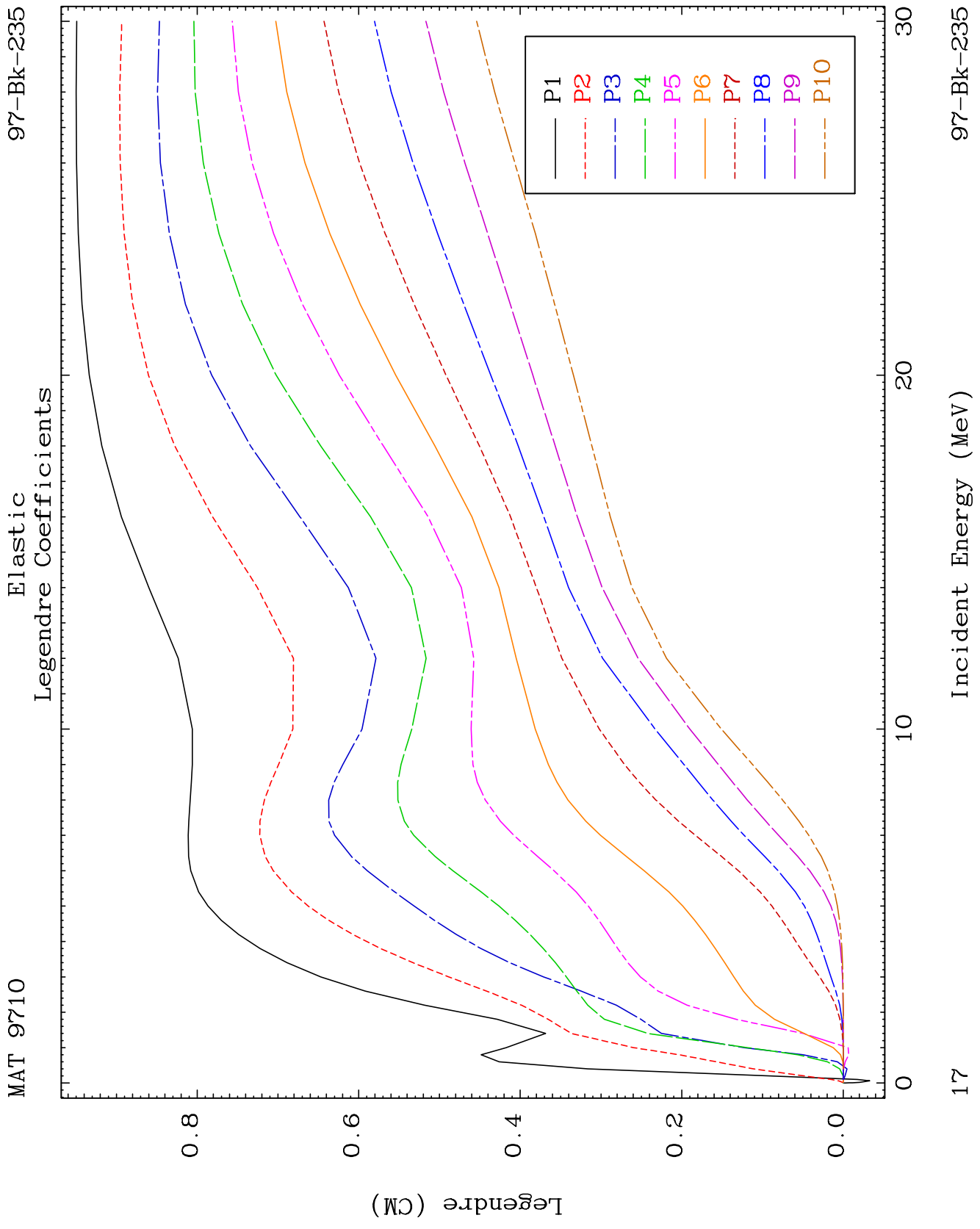
(n, α) Levels
293 Kelvin Cross Sections

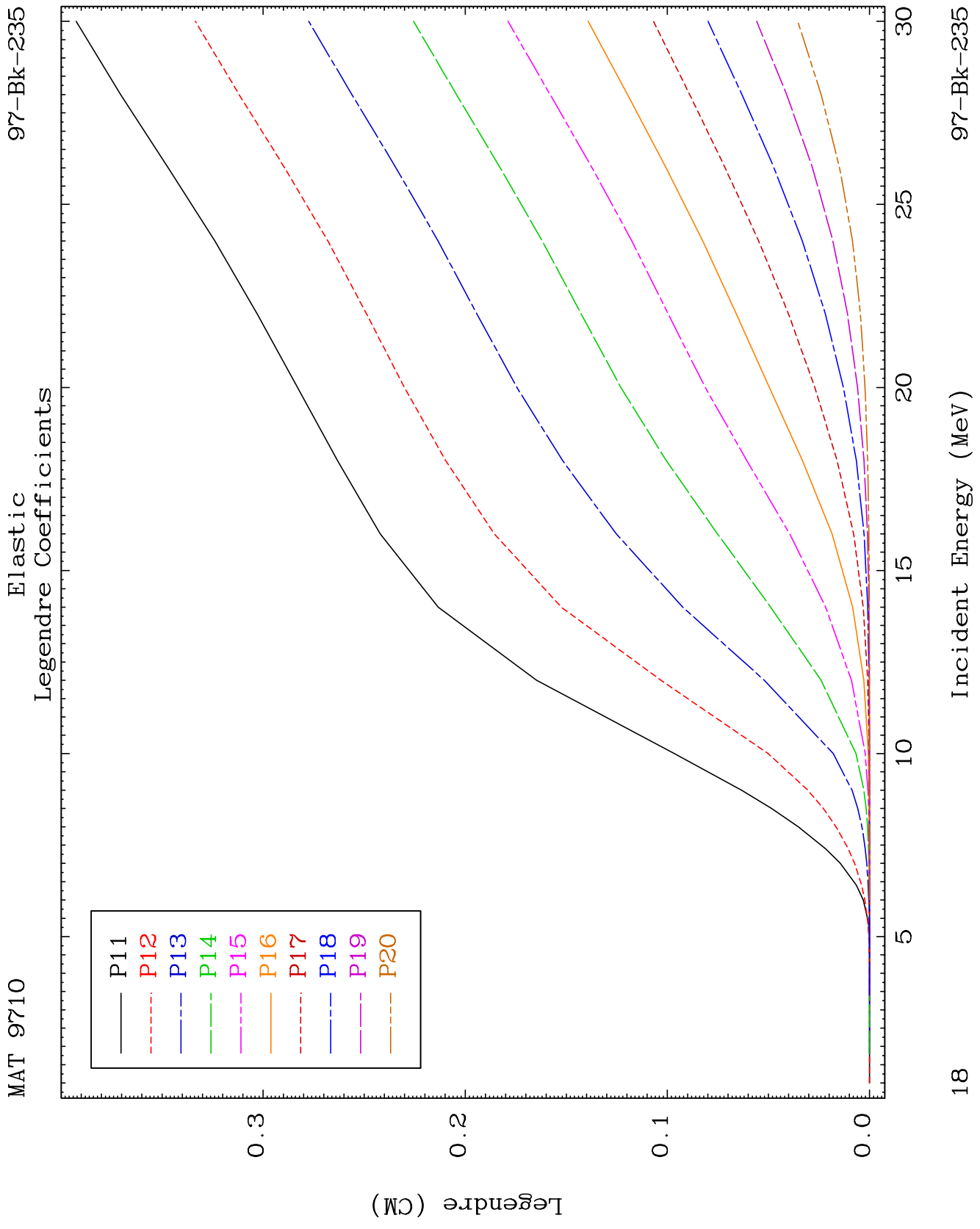
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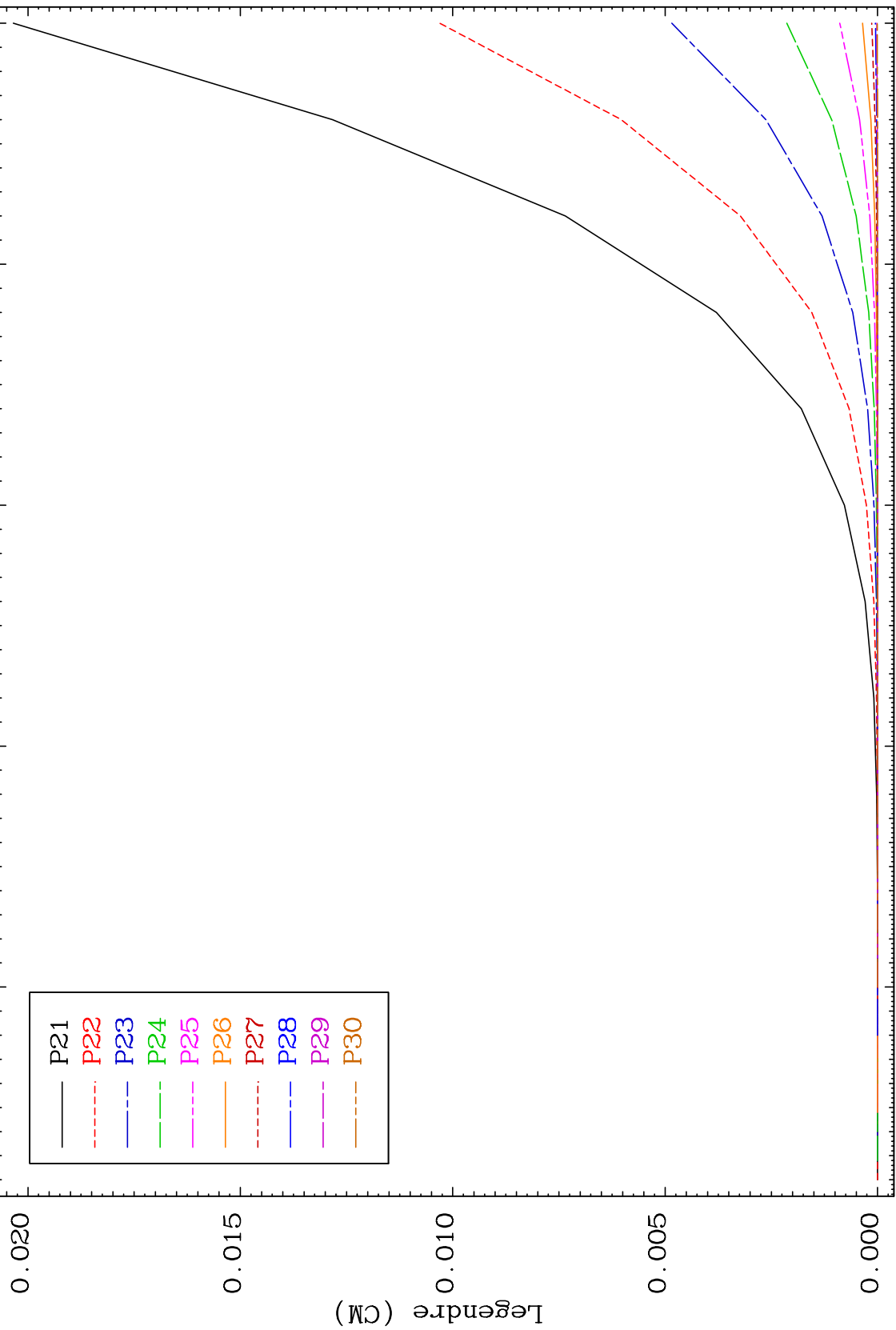




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Elastic
Legendre Coefficients

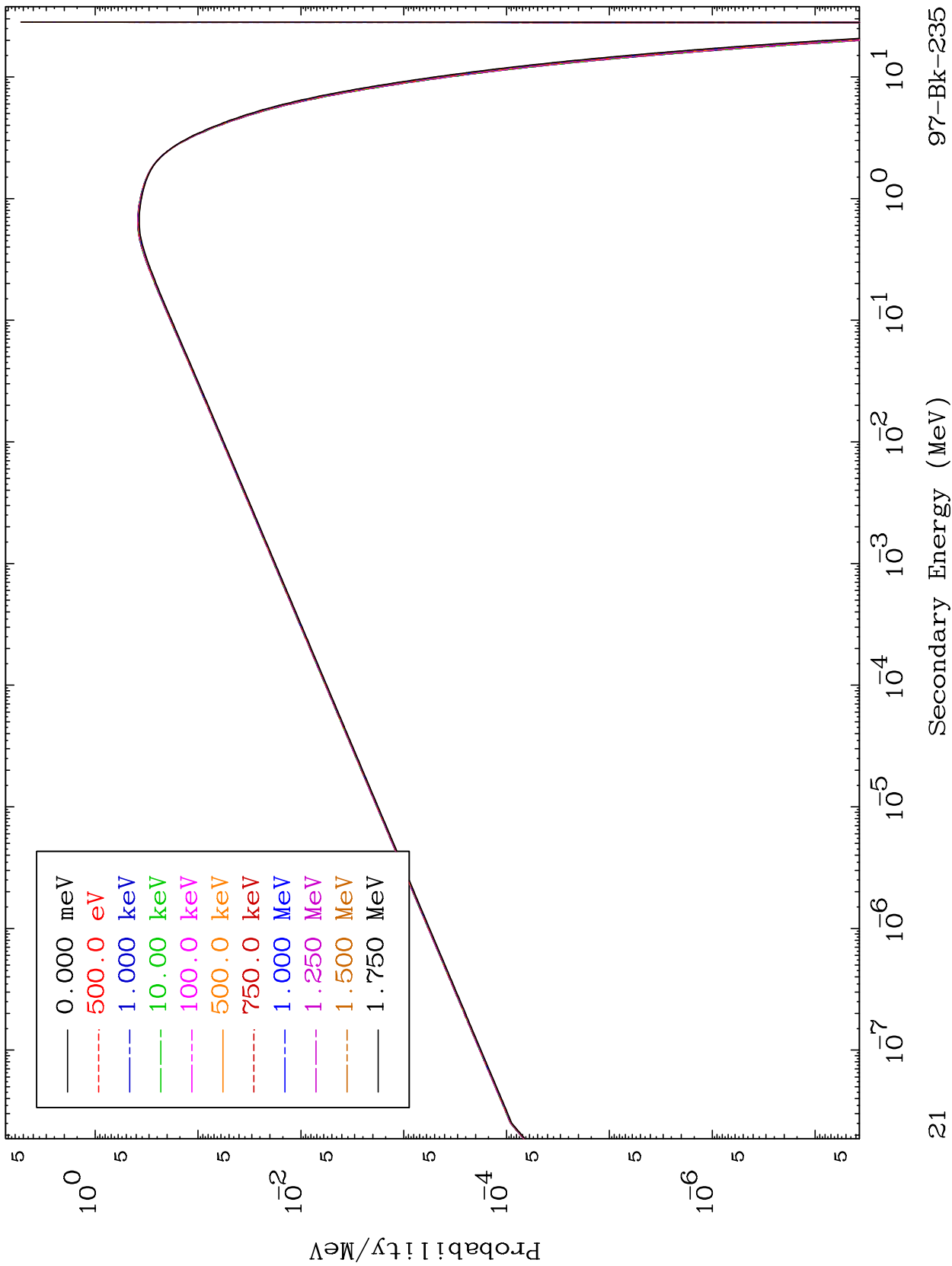
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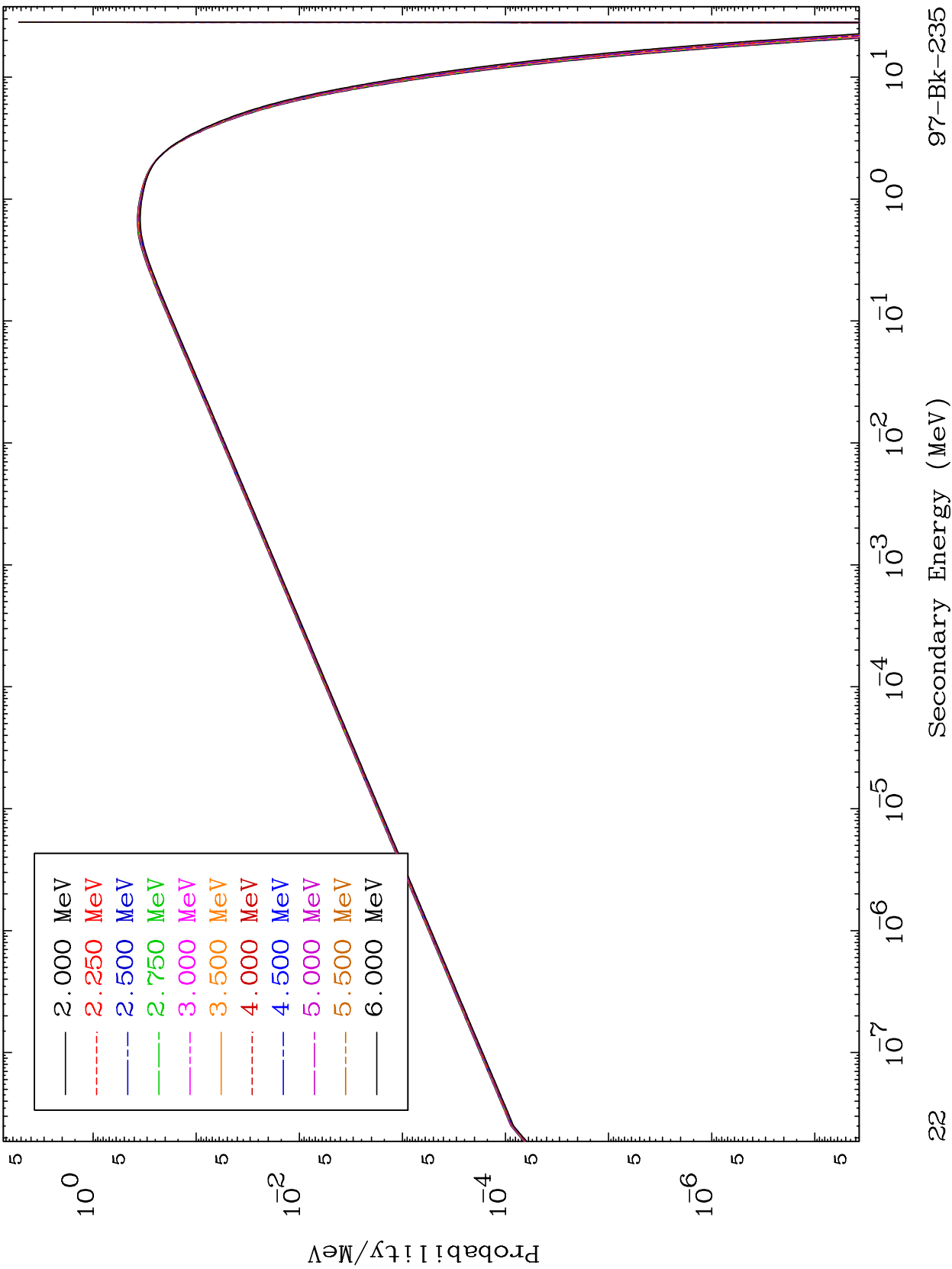
19

Incident Energy (MeV)

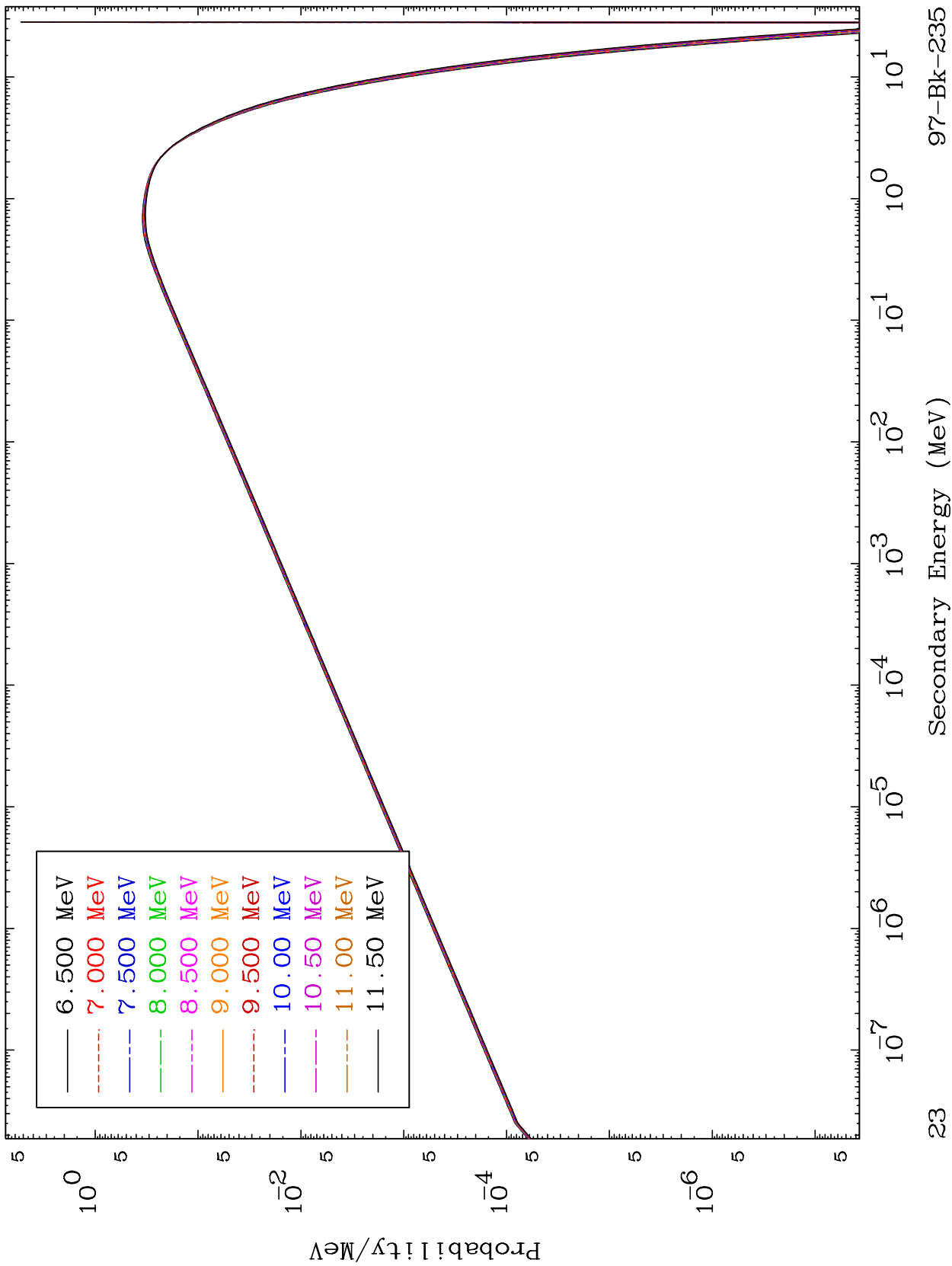
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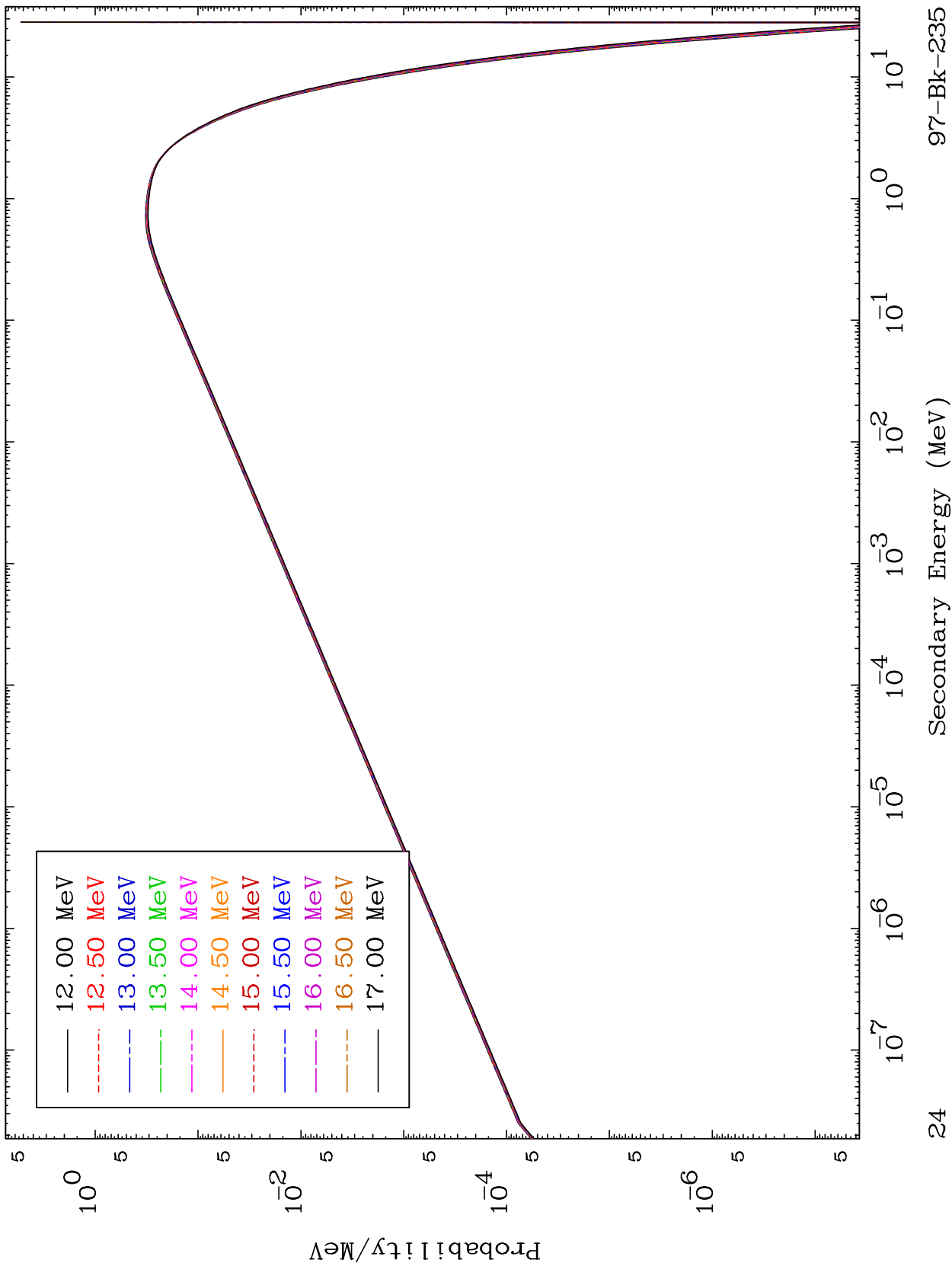
Fission Energy Distribution



Fission Energy Distribution



Fission
Energy Distribution



Fission
Energy Distribution

