

Program EVALPLOT
(Version 2015-2)

by

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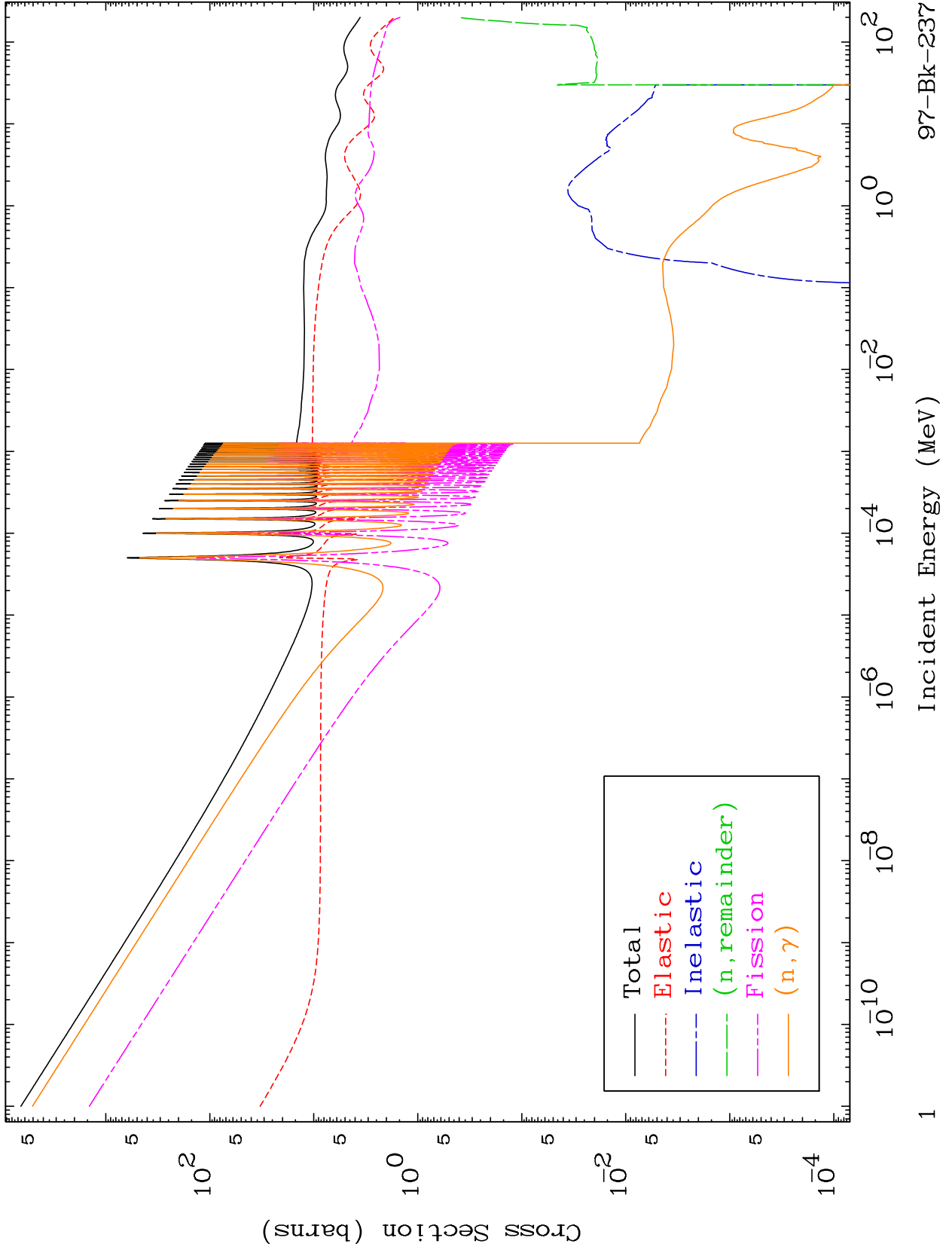
Press Mouse Button to Start

MAT 9716

Major

293 Kelvin Cross Sections

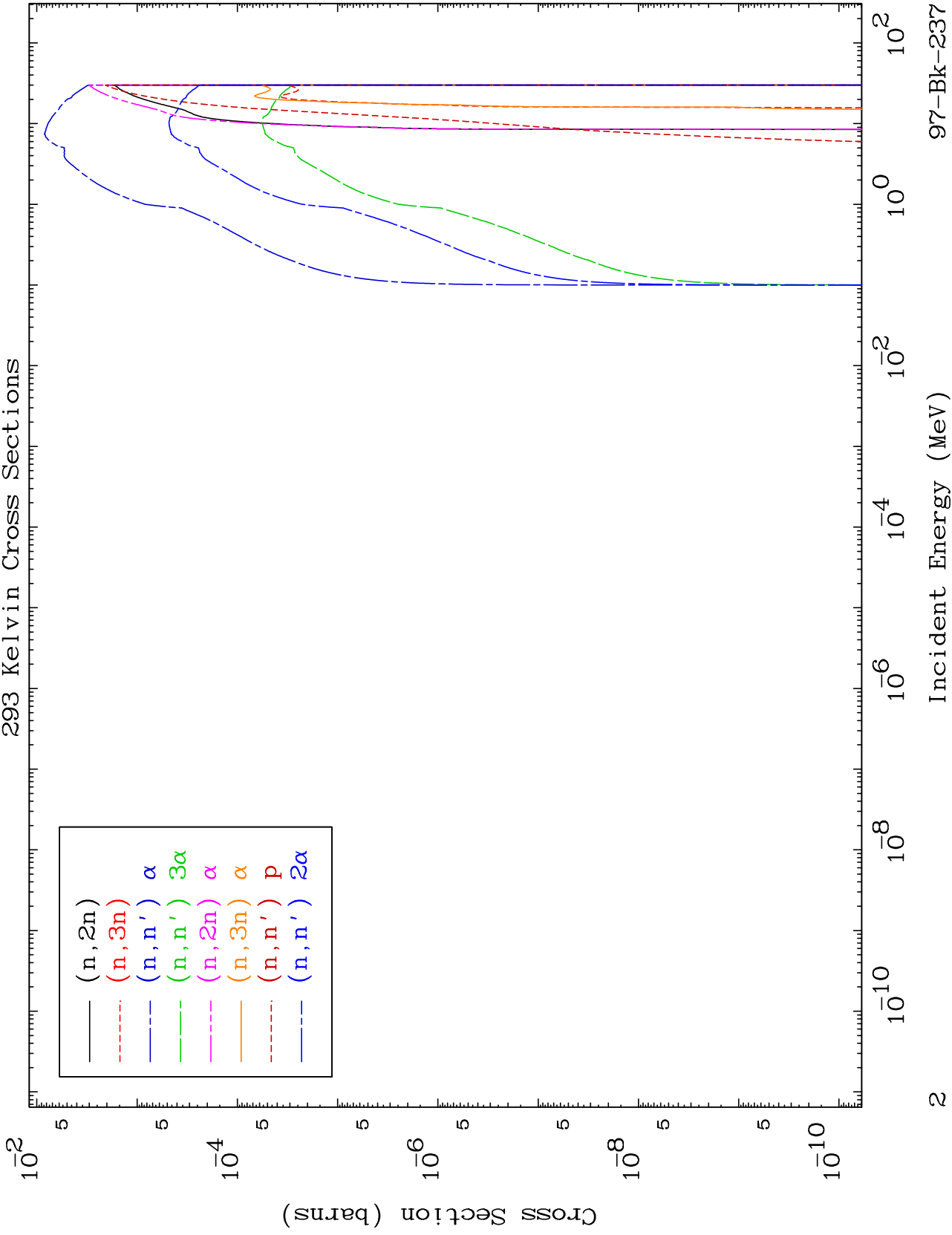
97-Bk-237



MAT 9716

Neutron Production
293 Kelvin Cross Sections

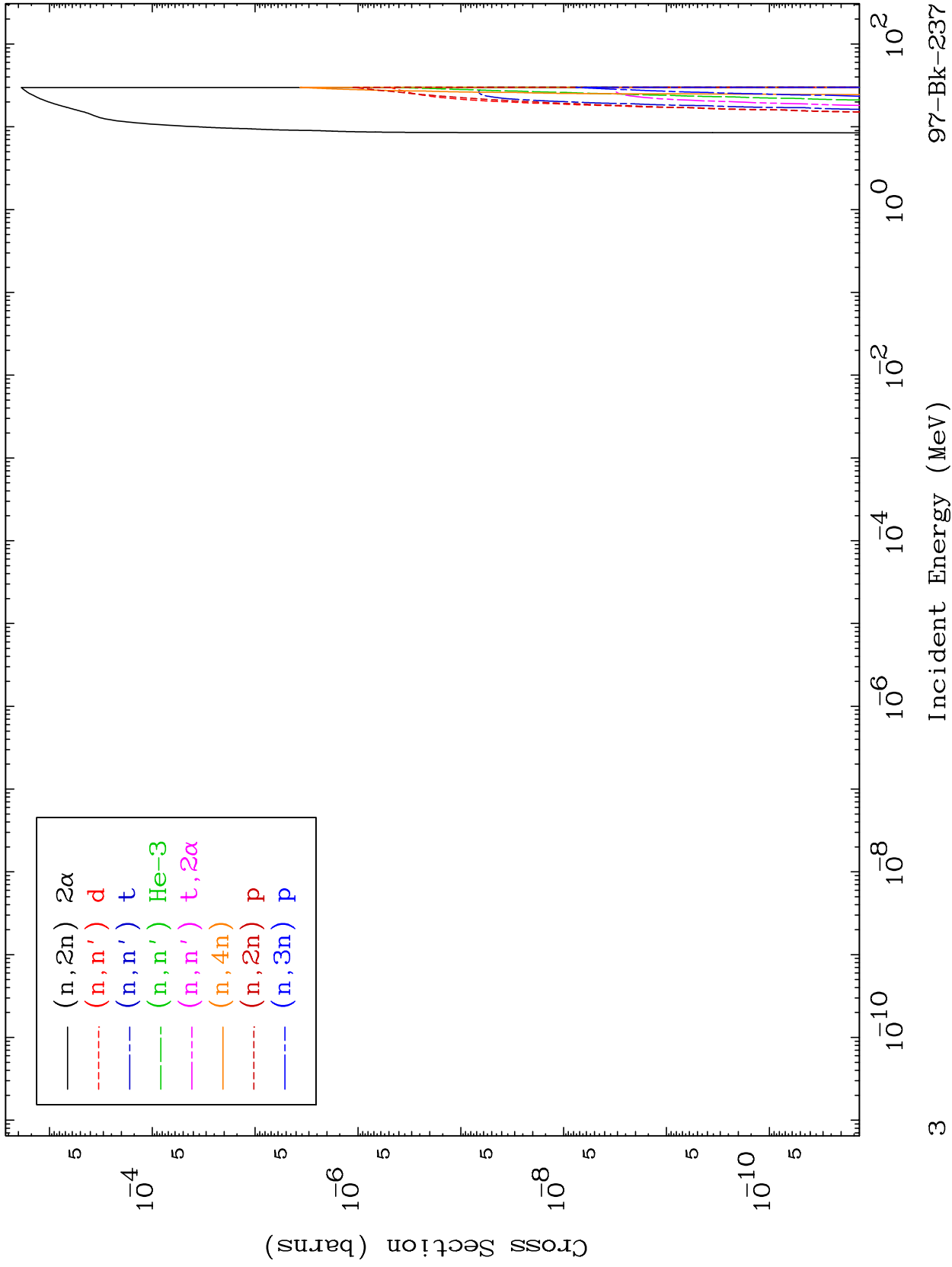
97-Bk-237



MAT 9716

Neutron Production
293 Kelvin Cross Sections

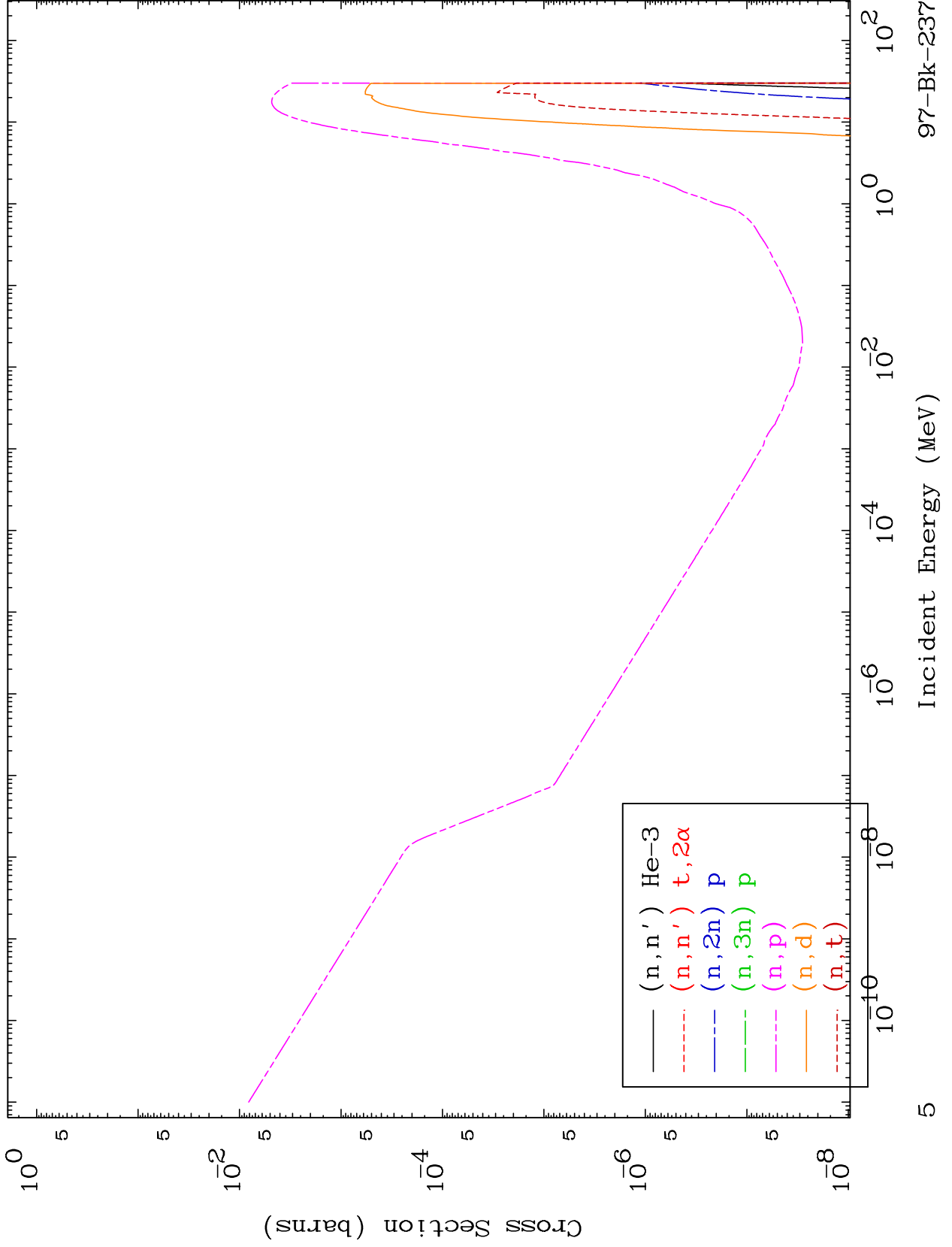
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Charged Particle
293 Kelvin Cross Sections

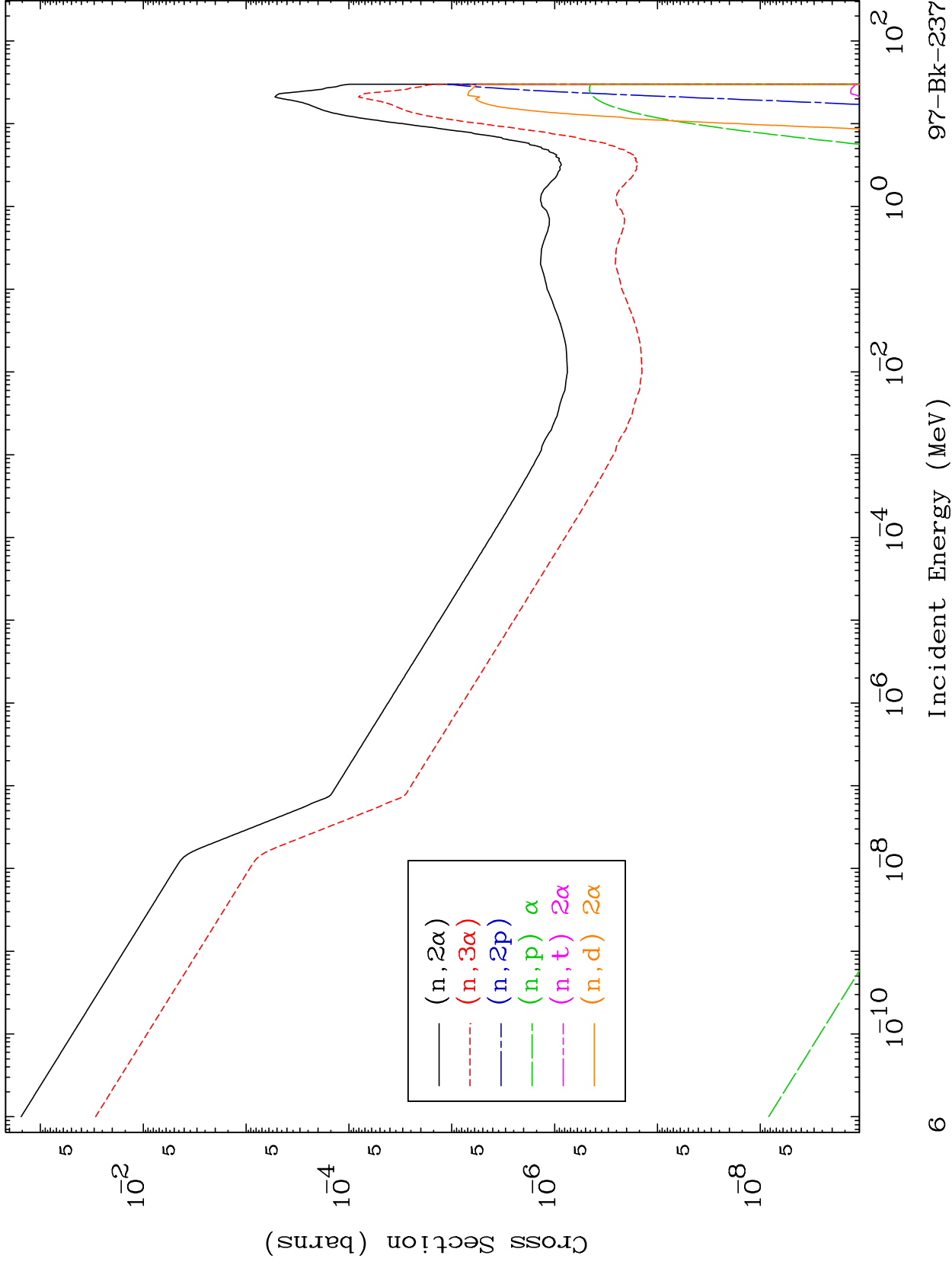
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Charged Particle
293 Kelvin Cross Sections

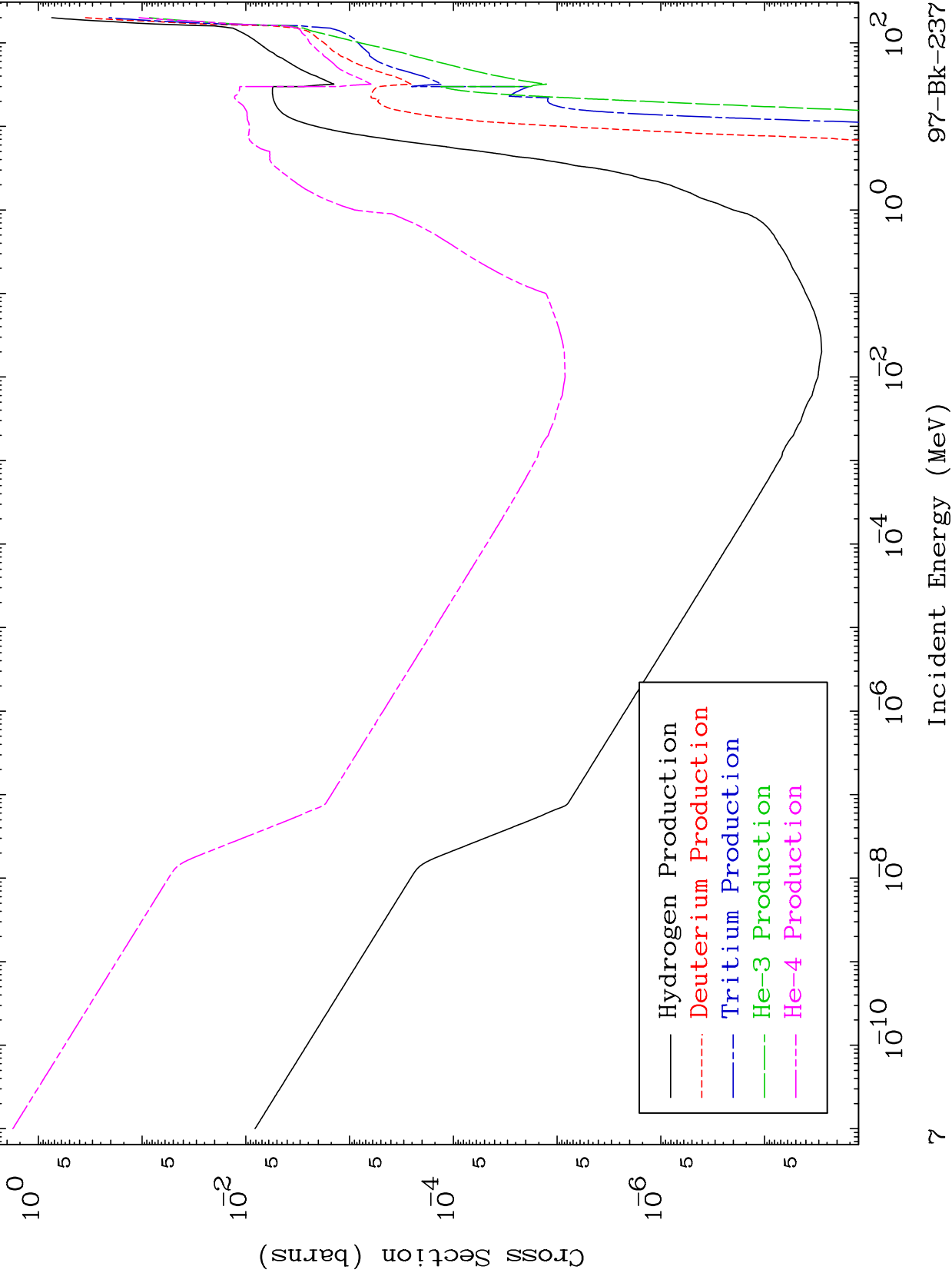
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Particle Production
293 Kelvin Cross Sections

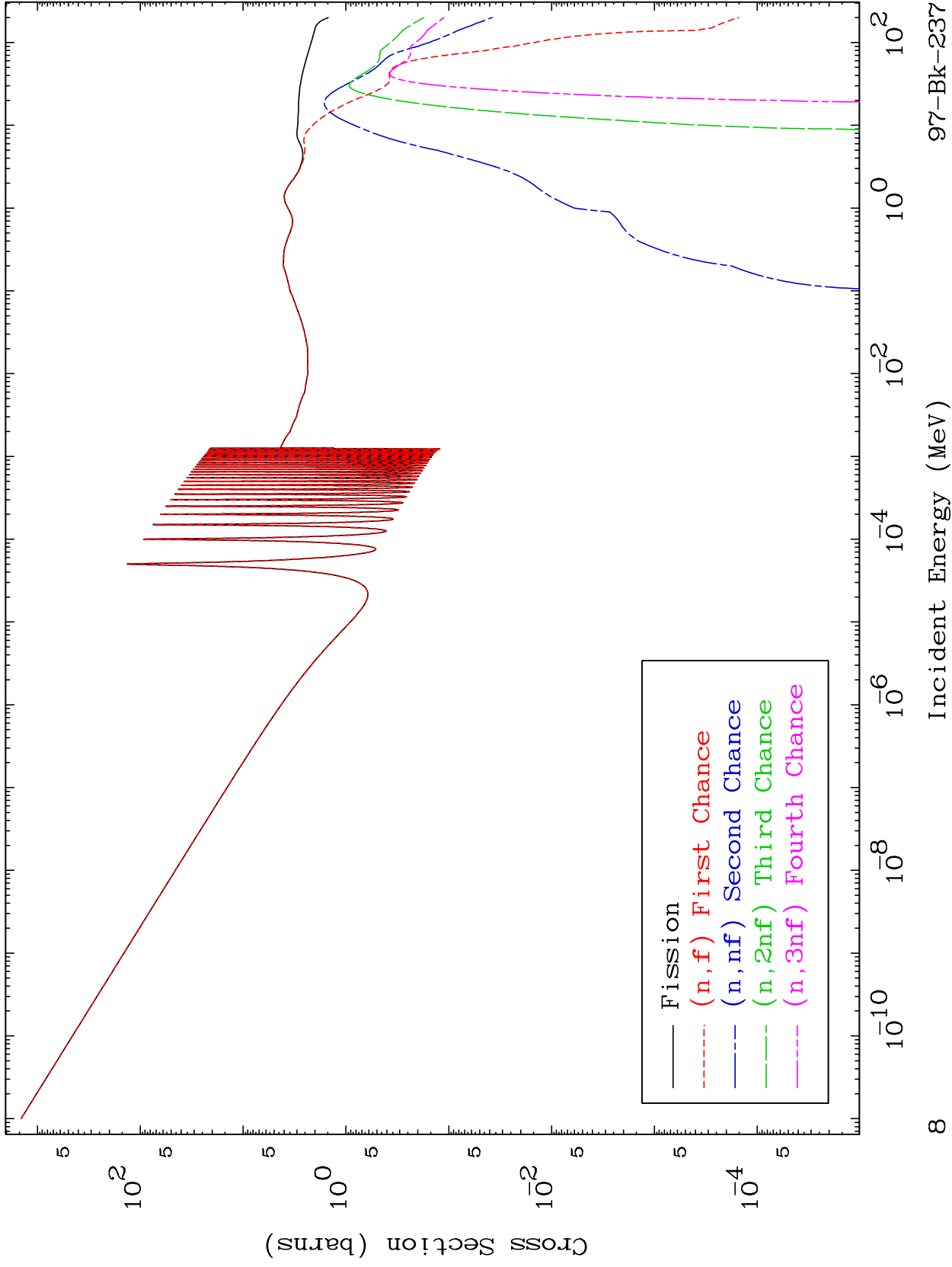
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Fission
293 Kelvin Cross Sections

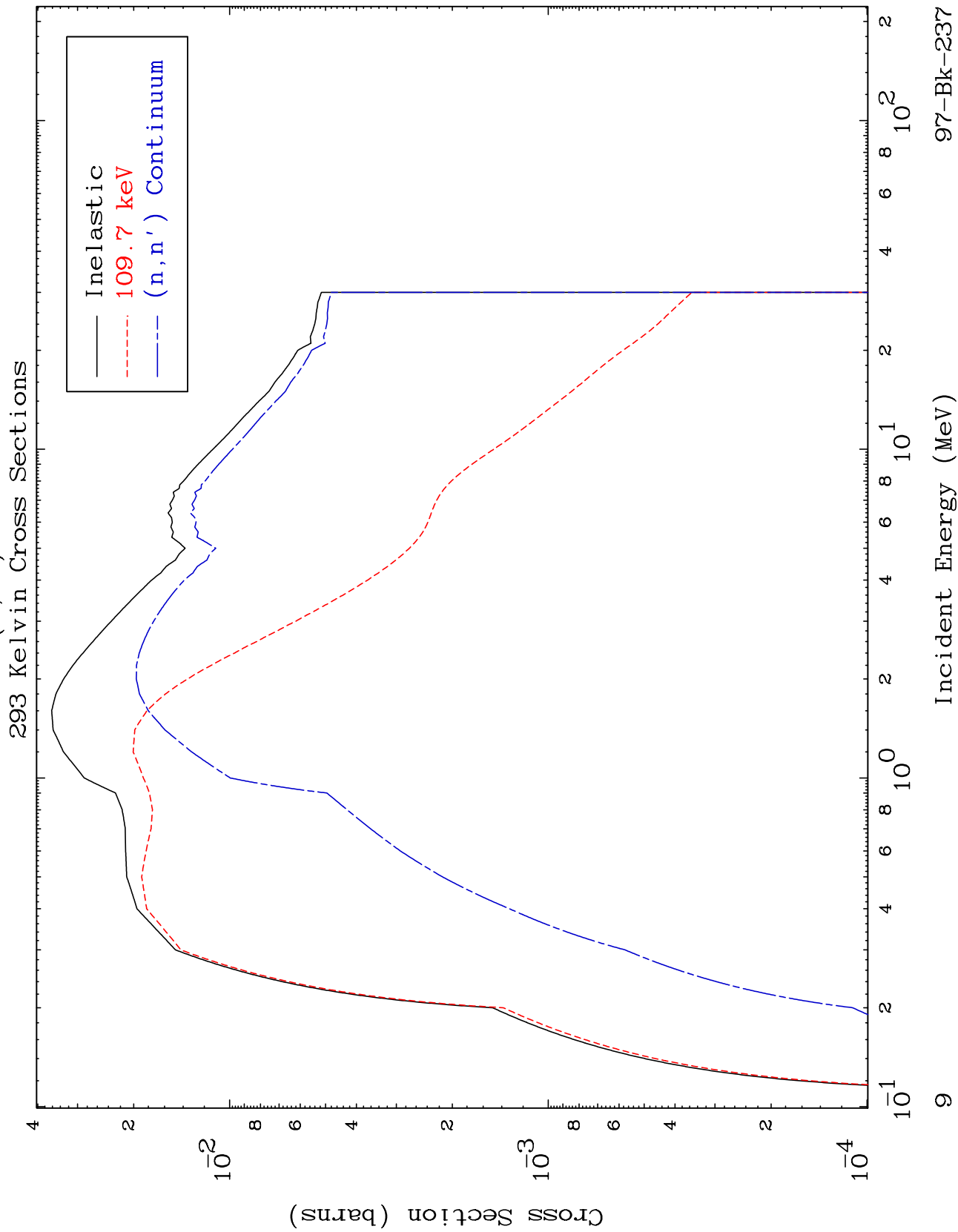
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(n,n') Level

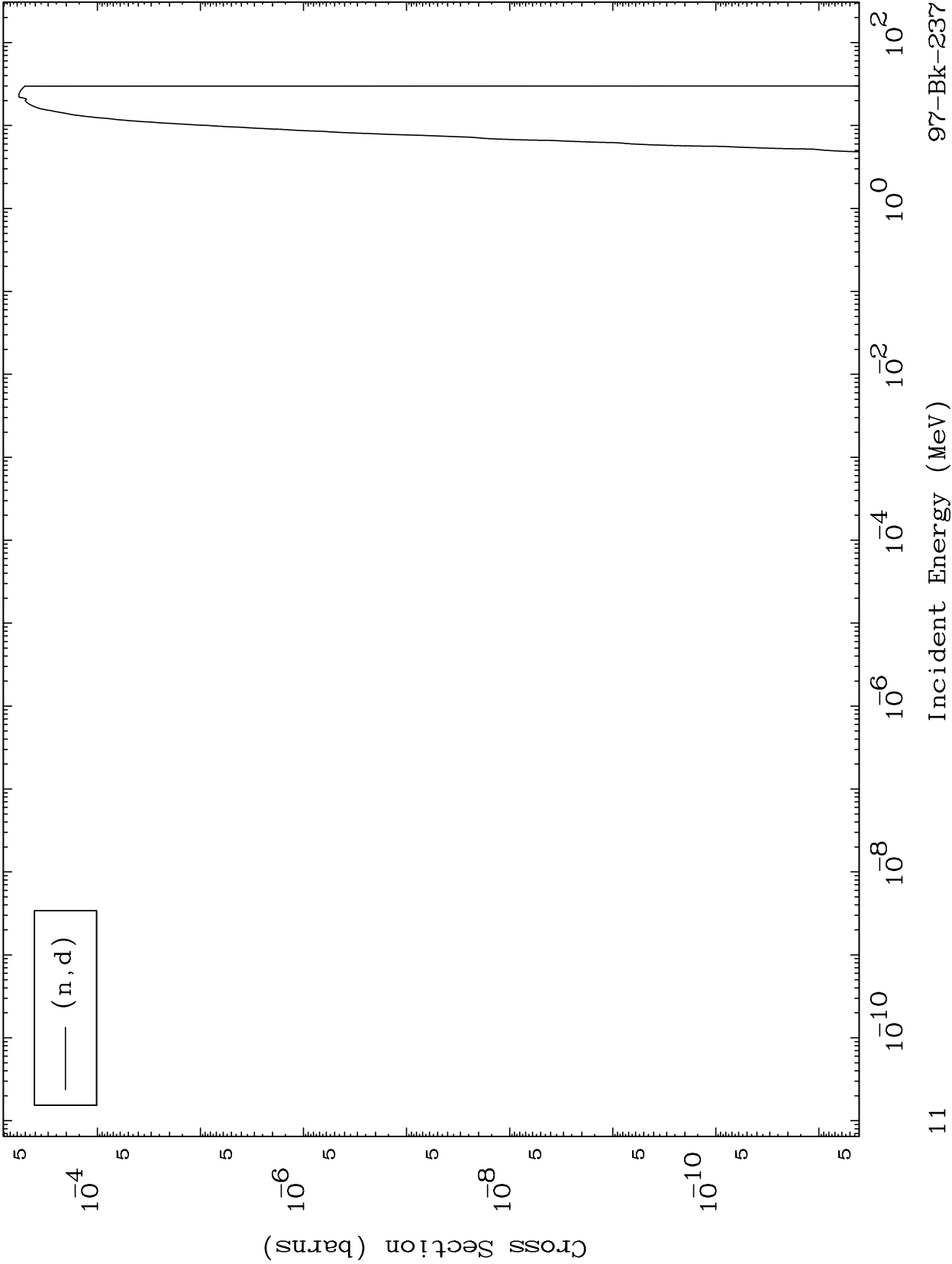
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(n,d) Levels
293 Kelvin Cross Sections

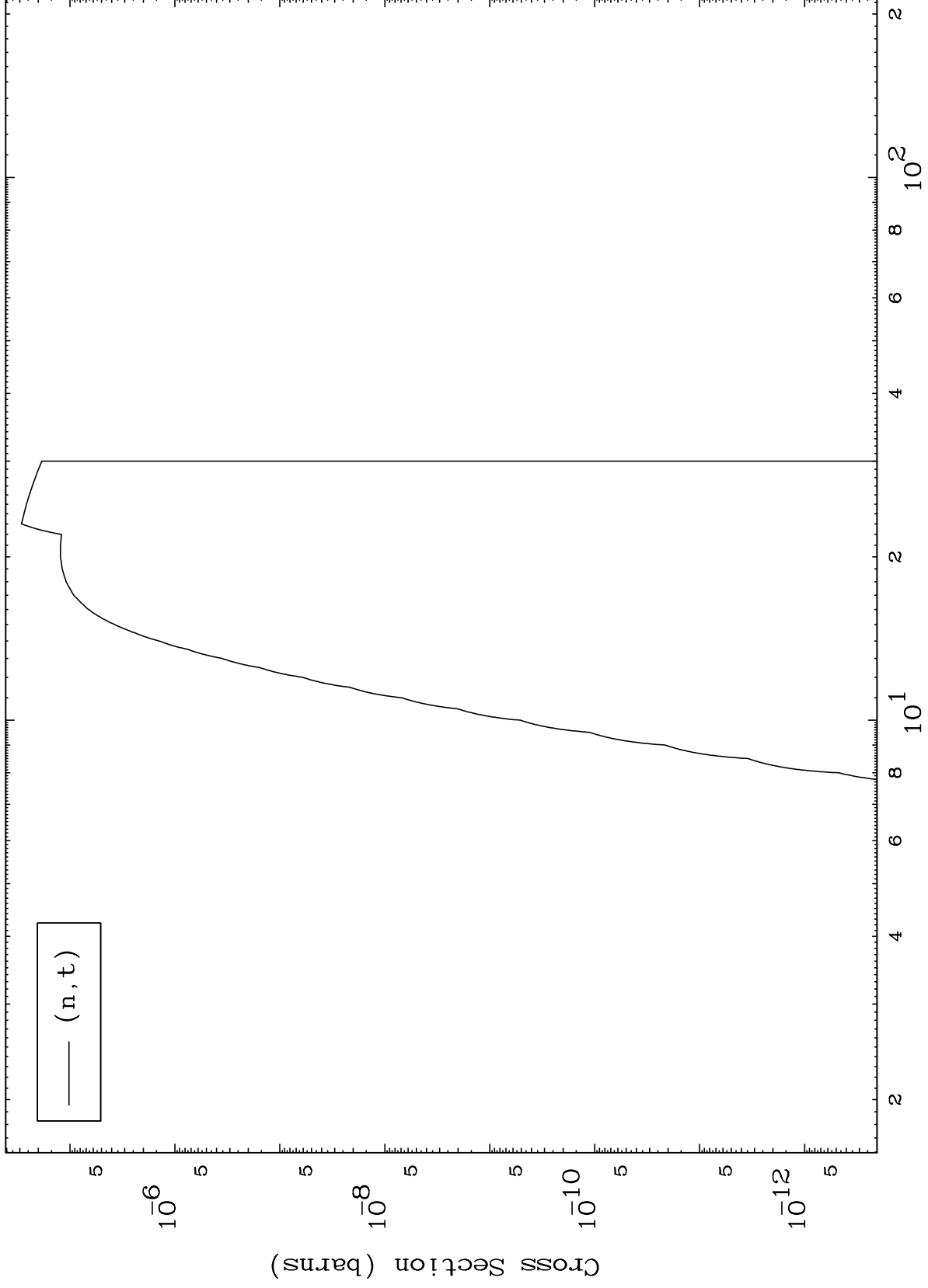
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(n,t) Levels
293 Kelvin Cross Sections

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12

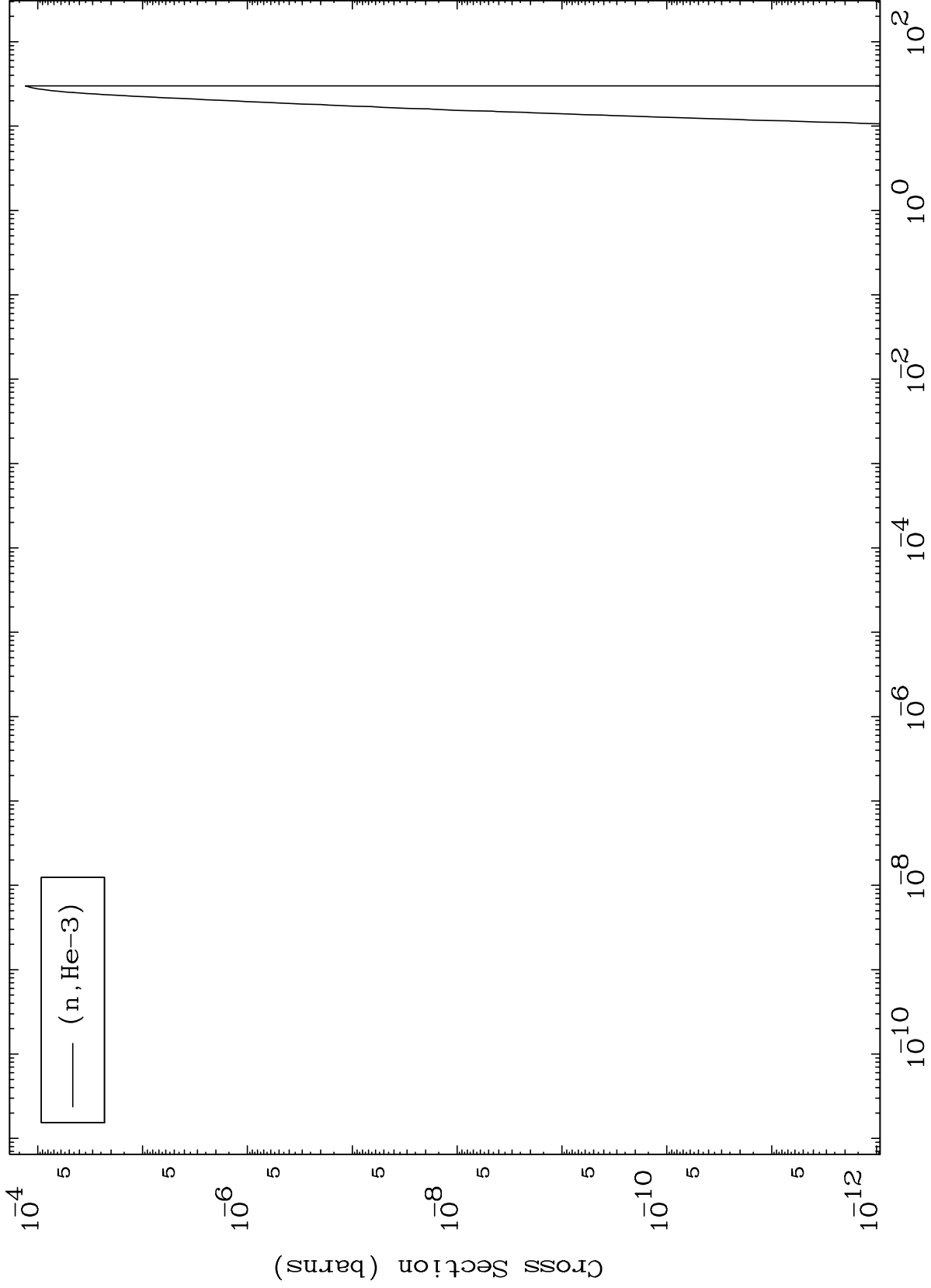
Incident Energy (MeV)

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(n,He3) Levels
293 Kelvin Cross Sections

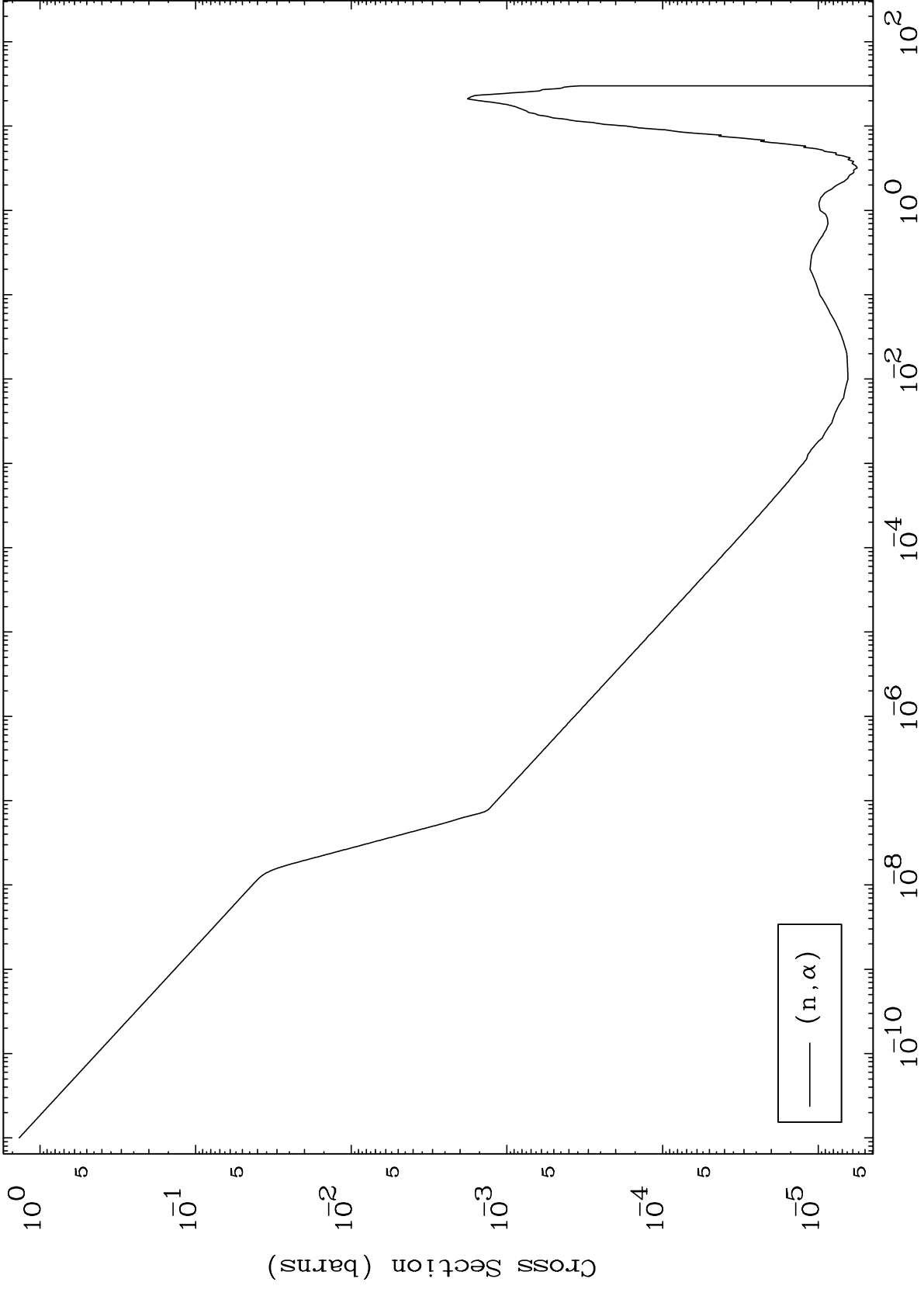
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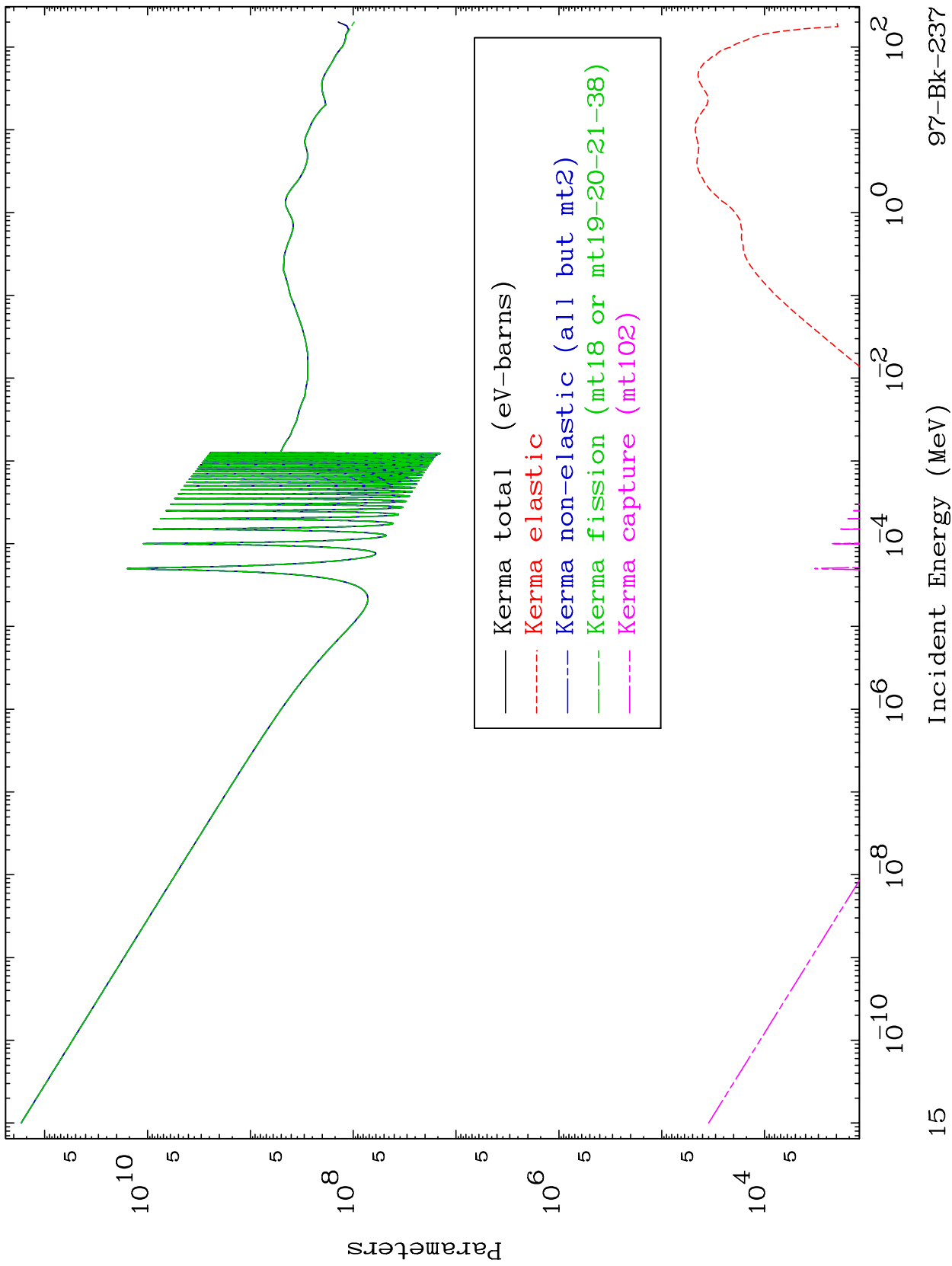


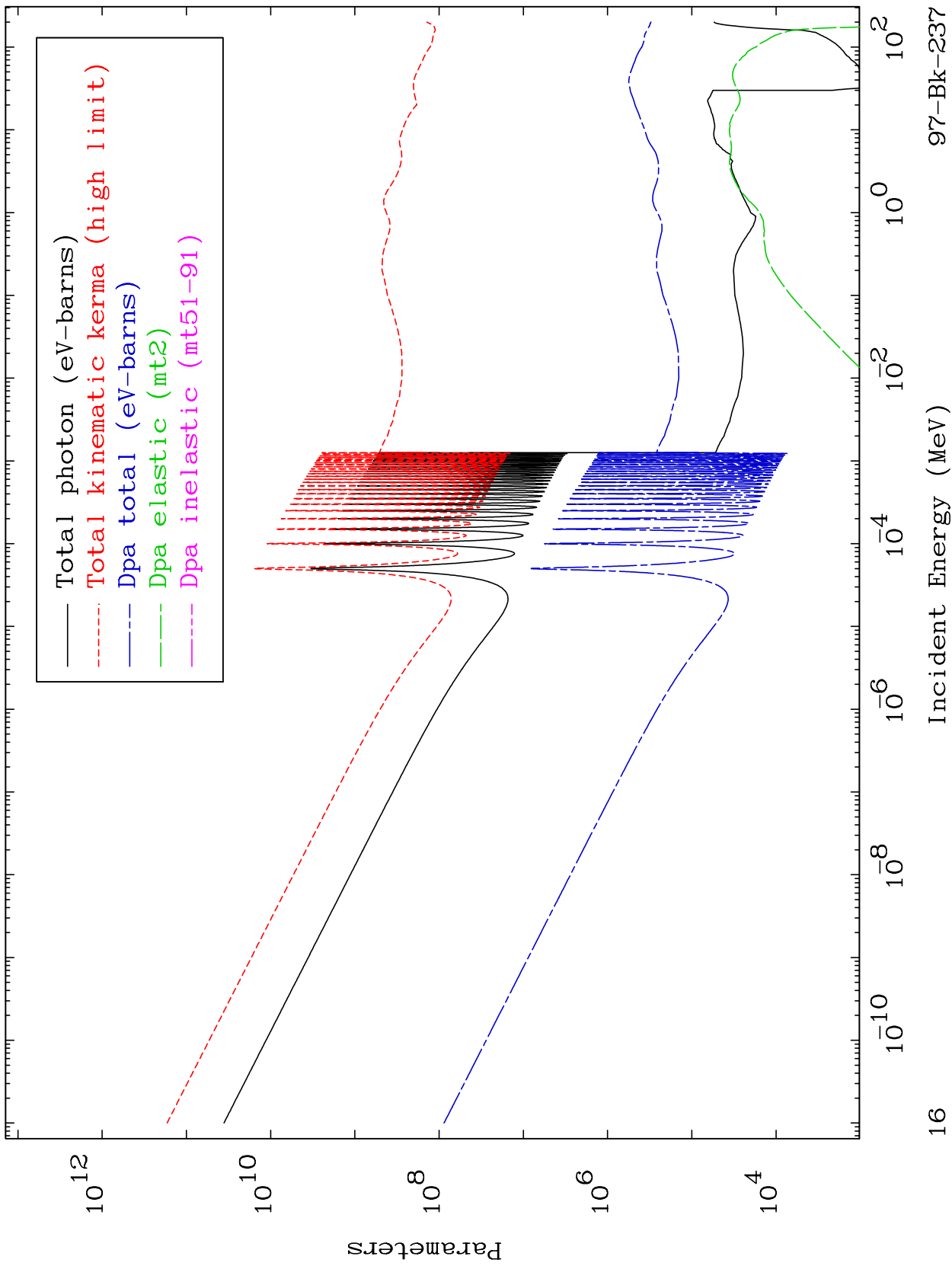
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(n, α) Levels
293 Kelvin Cross Sections

97-Bk-237



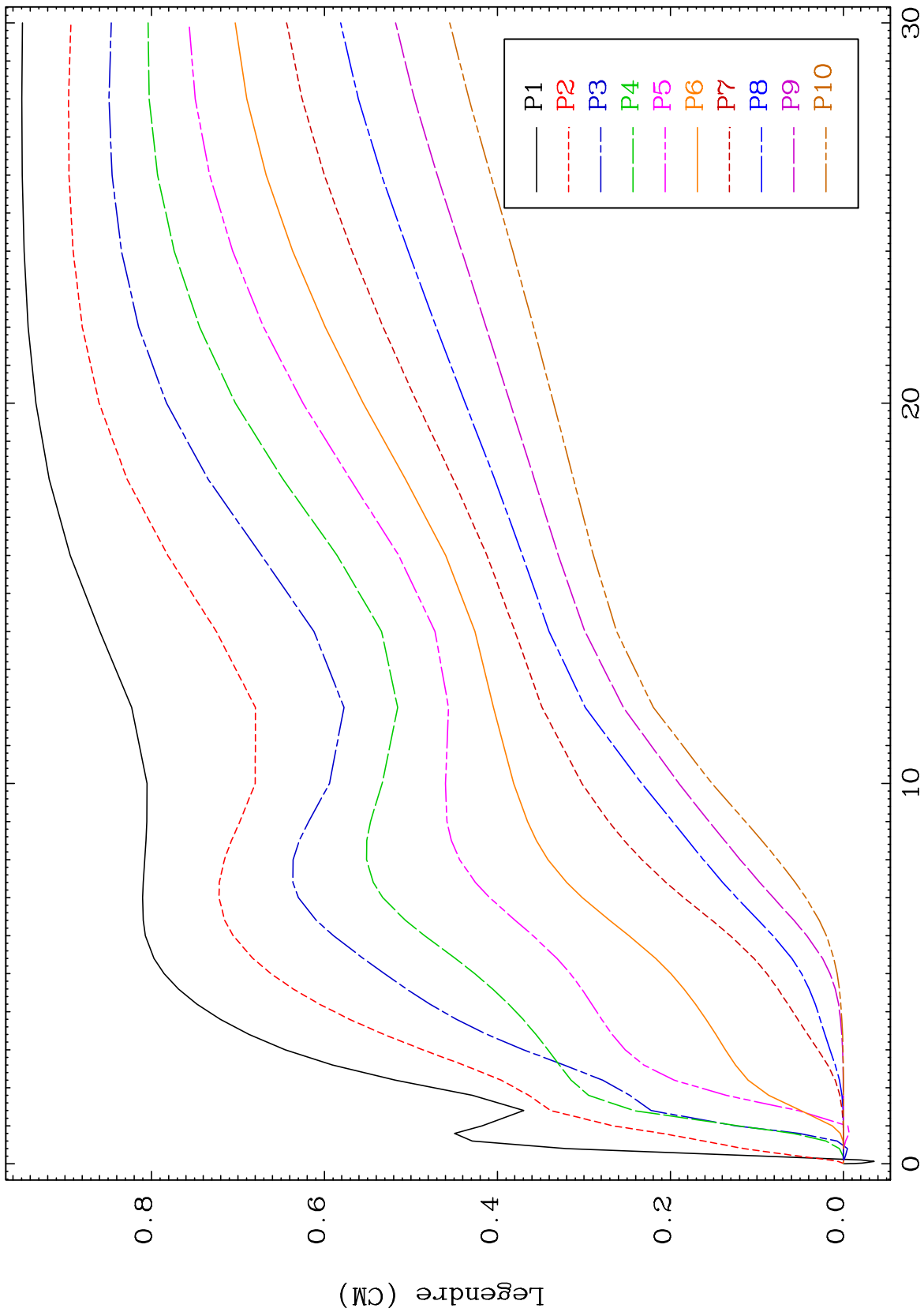




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Elastic Legendre Coefficients

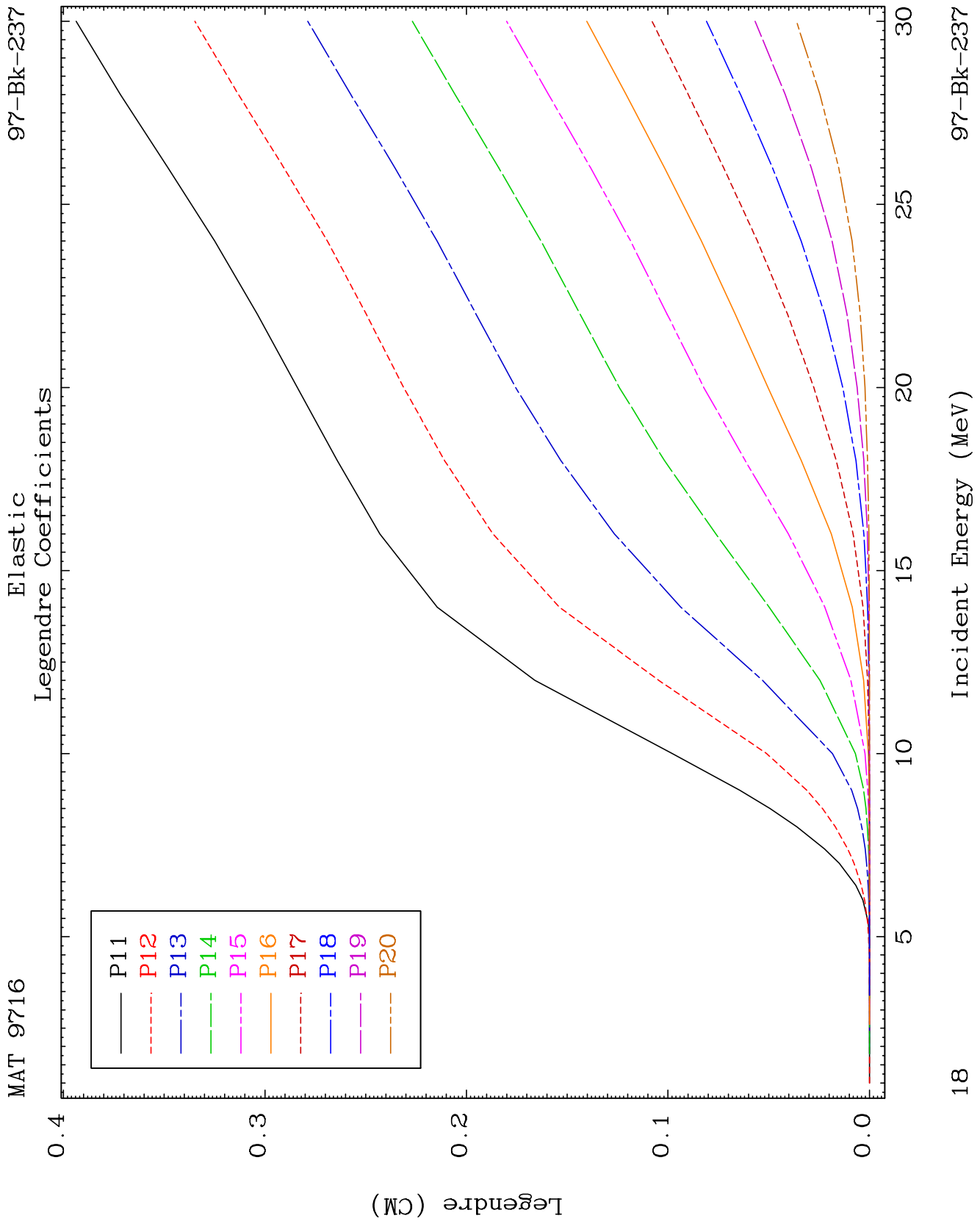
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17

Incident Energy (MeV)

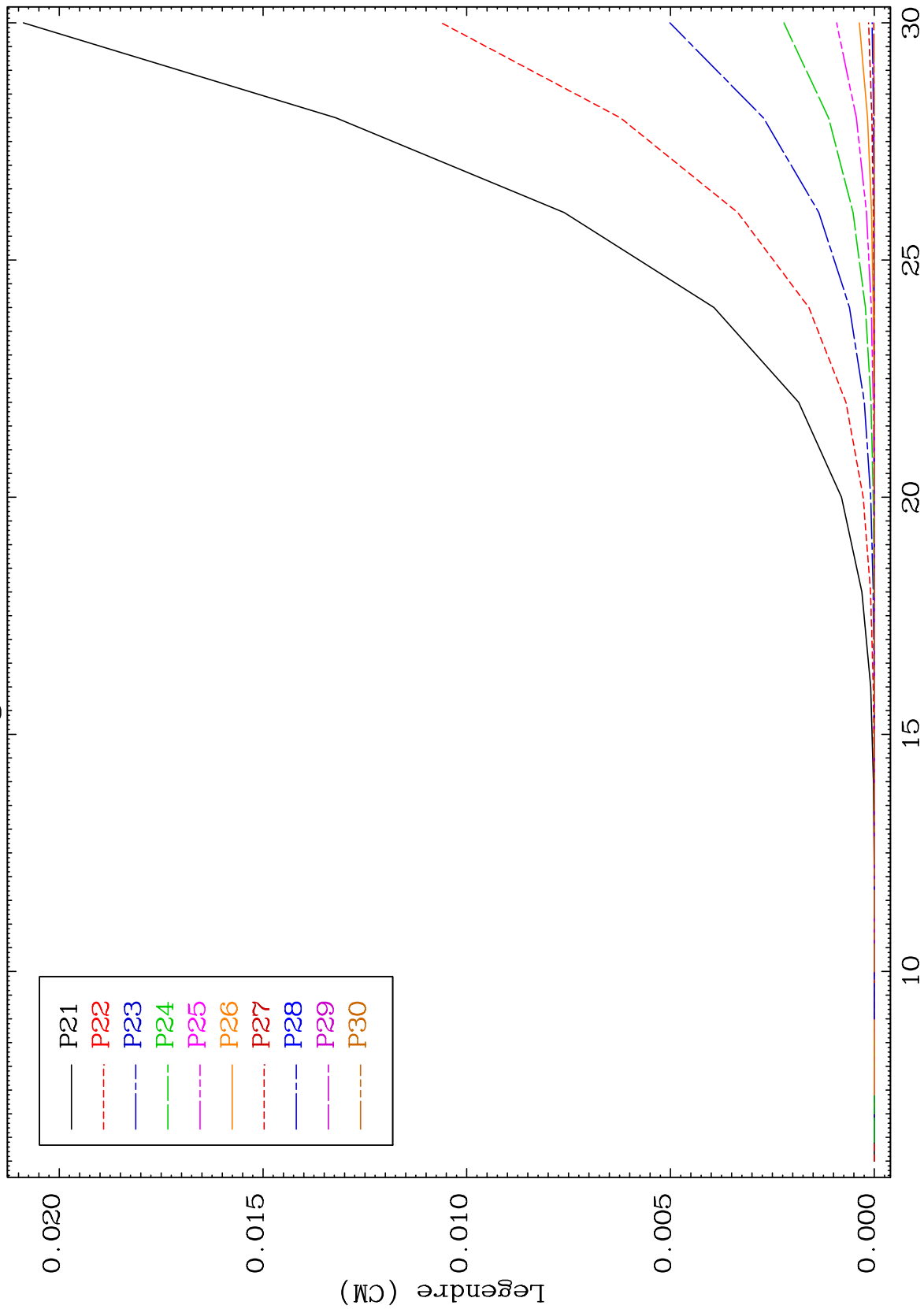
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Elastic Legendre Coefficients

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19

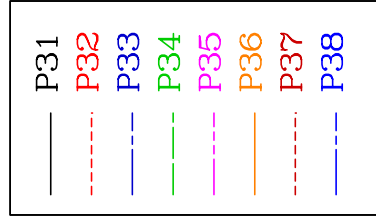
Incident Energy (MeV)

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Elastic
Legendre Coefficients

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$\times 10^{-6}$

Legendre (CM)

1.5

1.0

0.5

0.0

15

20

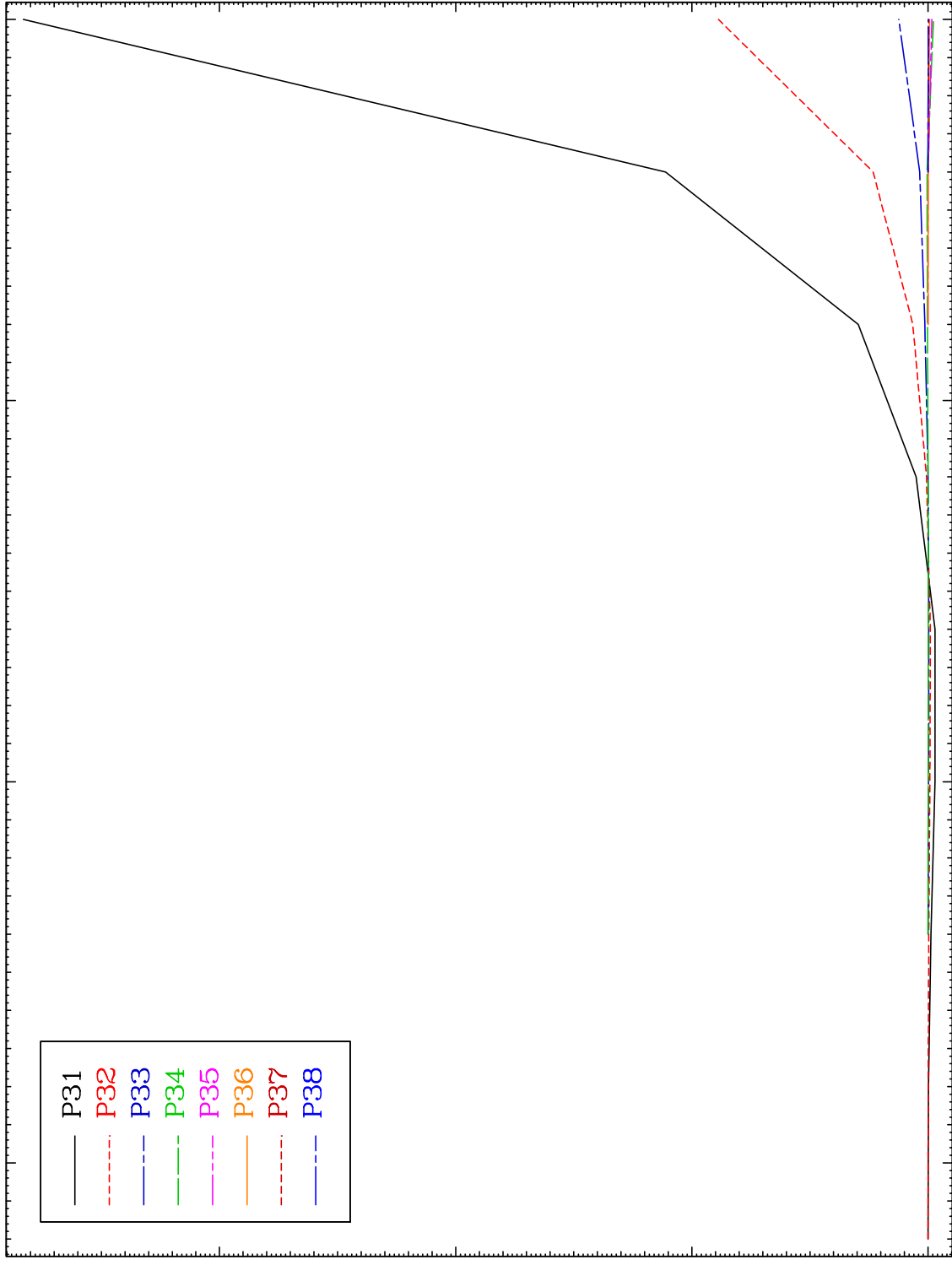
25

30

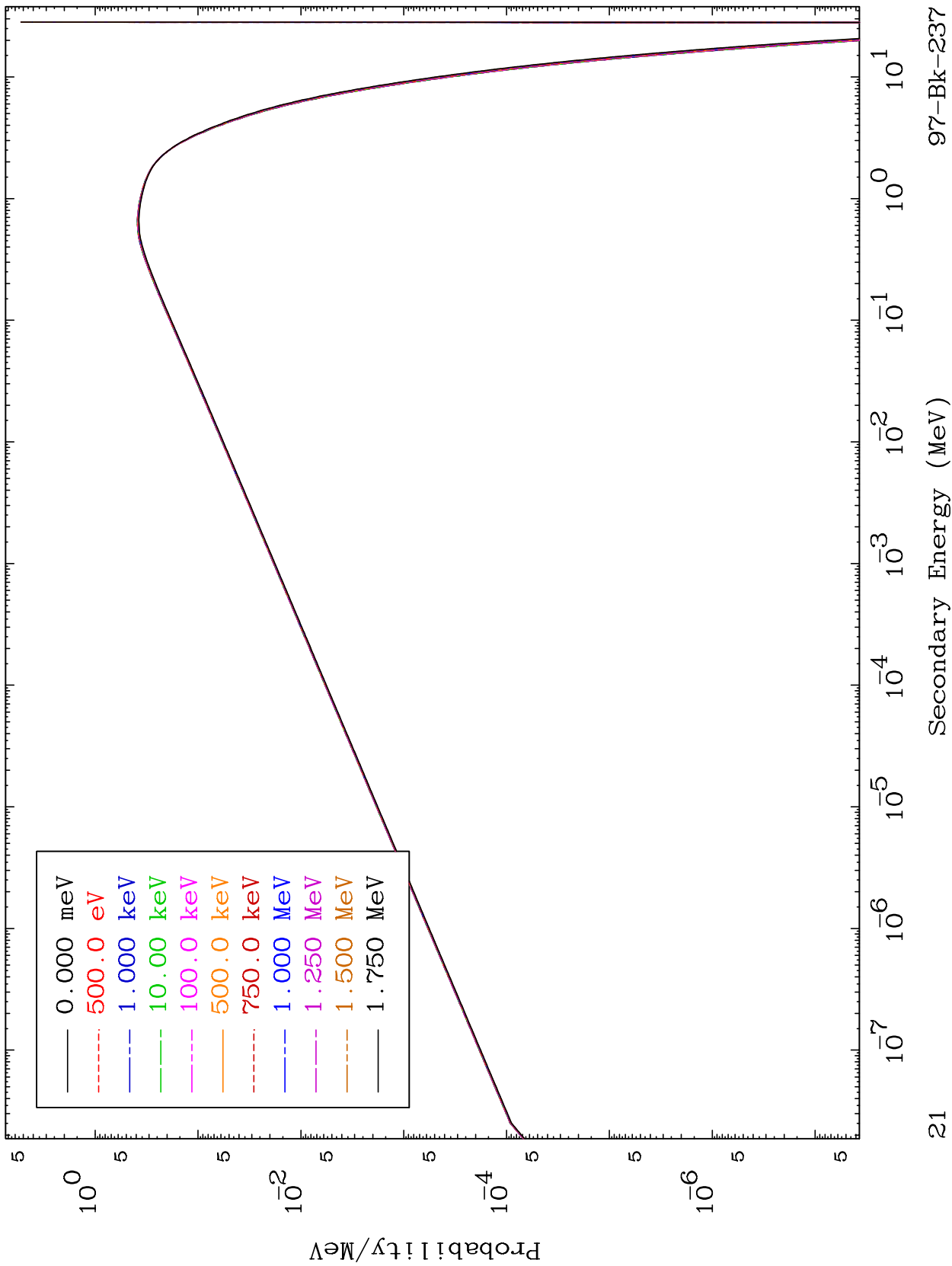
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Incident Energy (MeV)

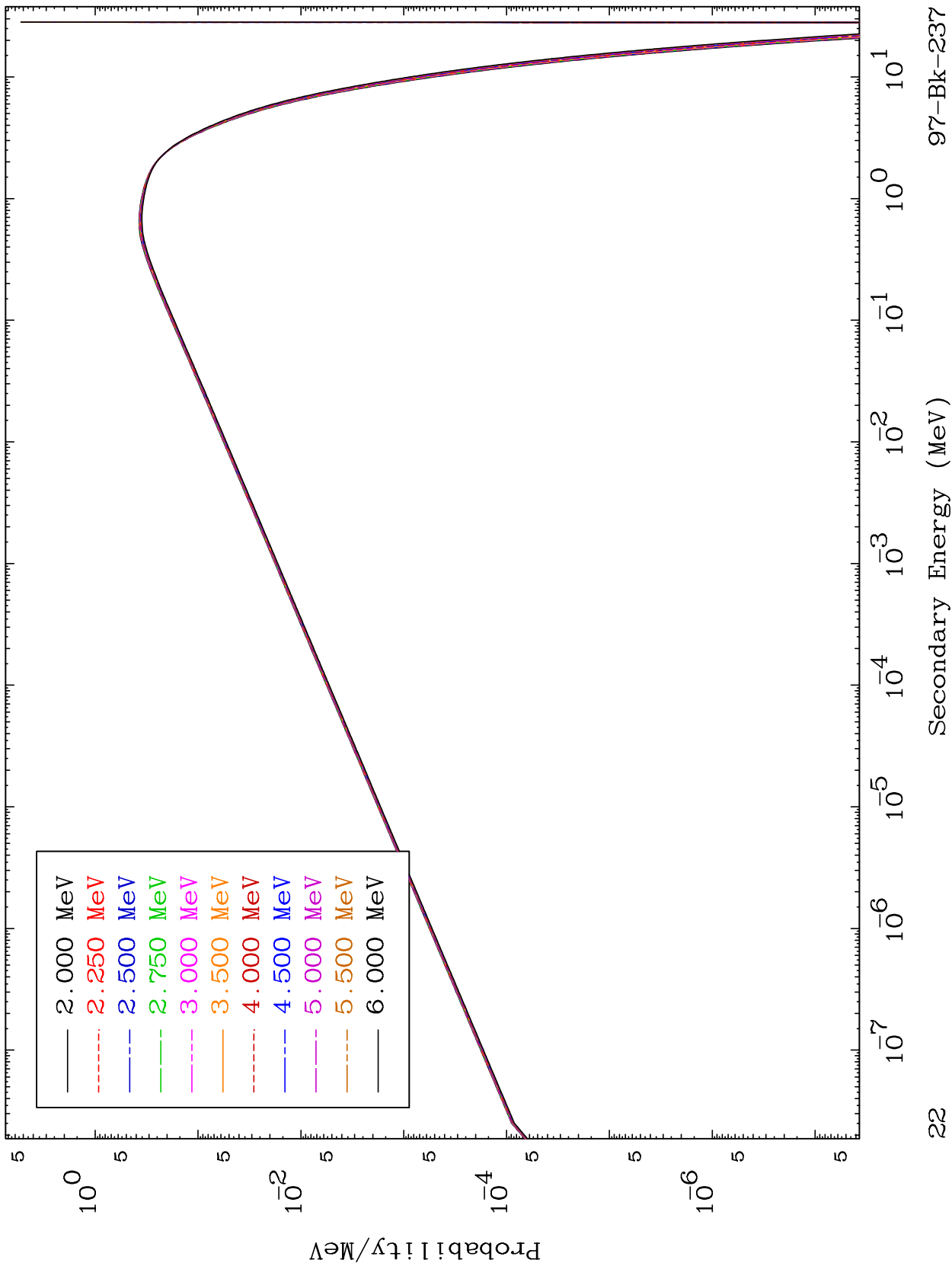
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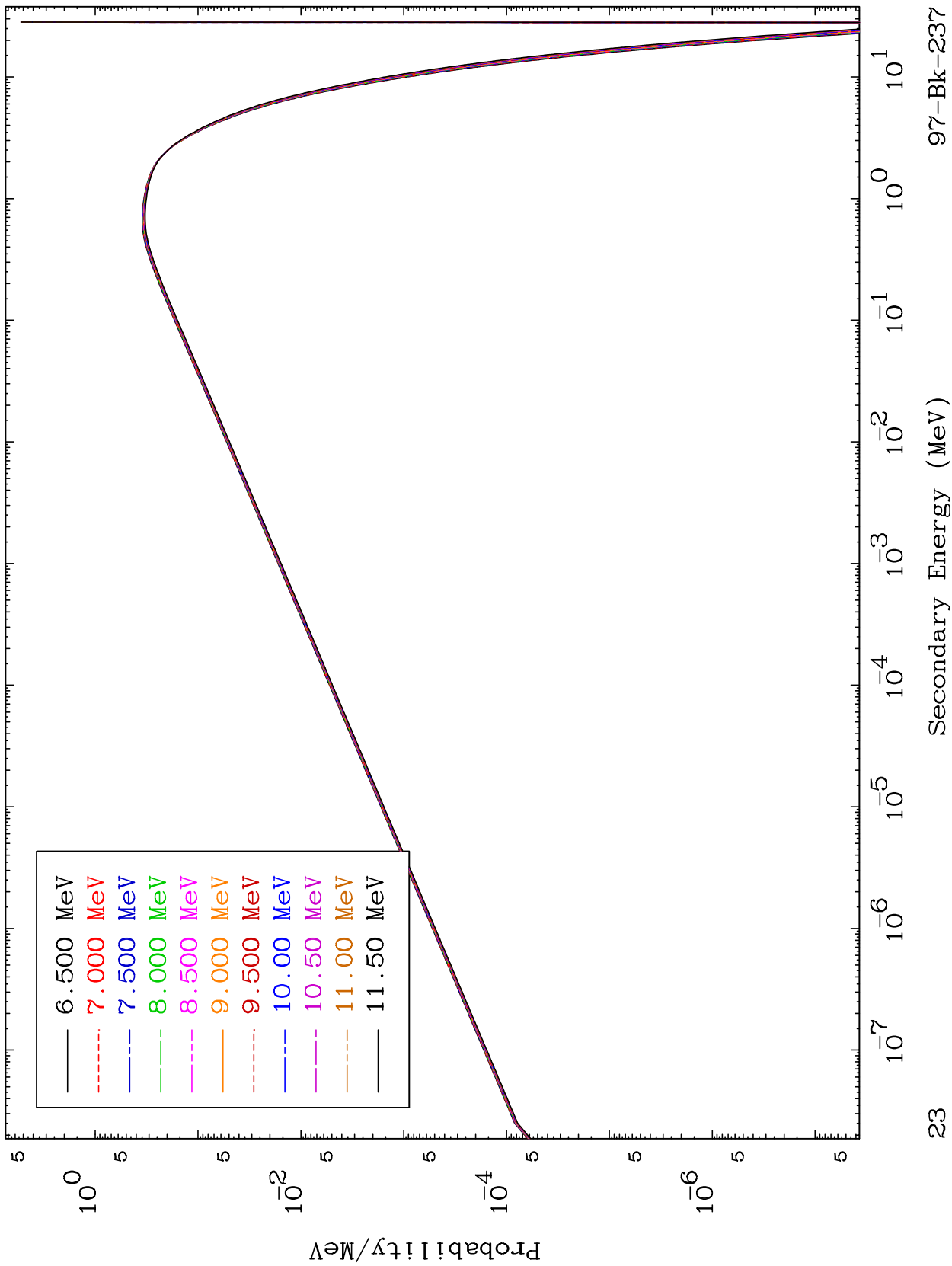
Fission Energy Distribution



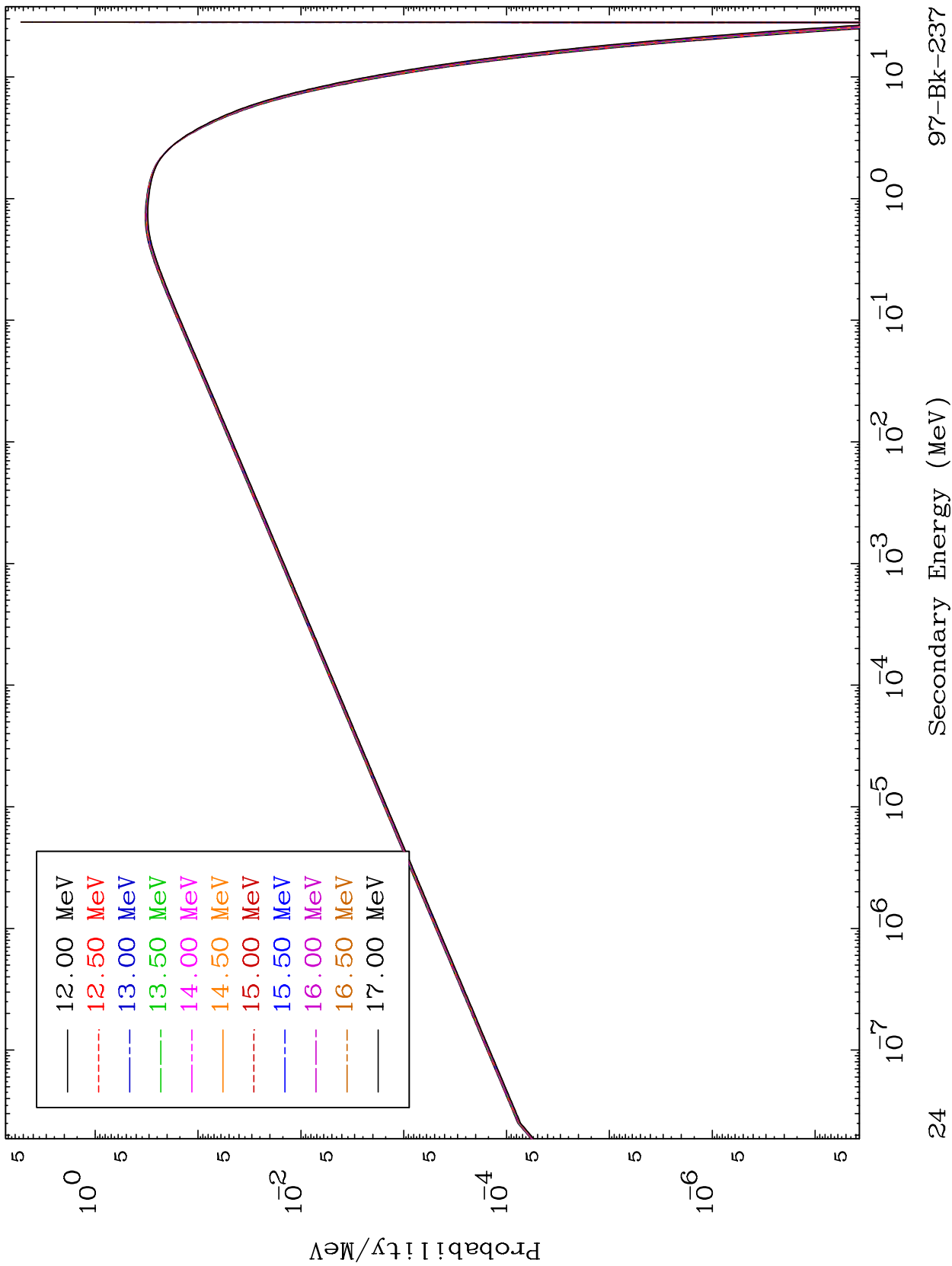
Fission Energy Distribution



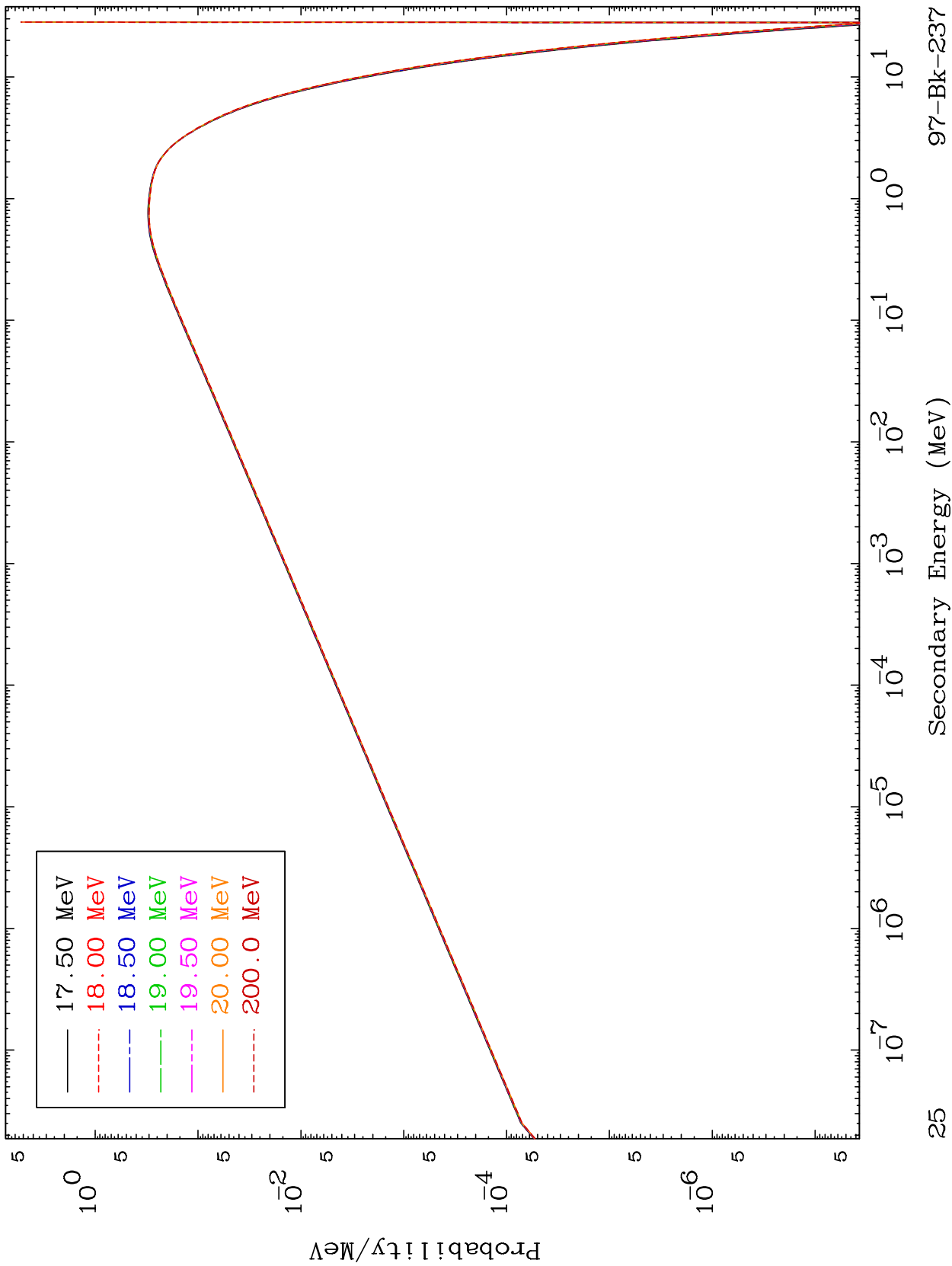
Fission Energy Distribution



Fission
Energy Distribution



Fission
Energy Distribution



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(n,2p)

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Radionuclide Production Cross Section

