

Program EVALPLOT  
(Version 2015-2)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

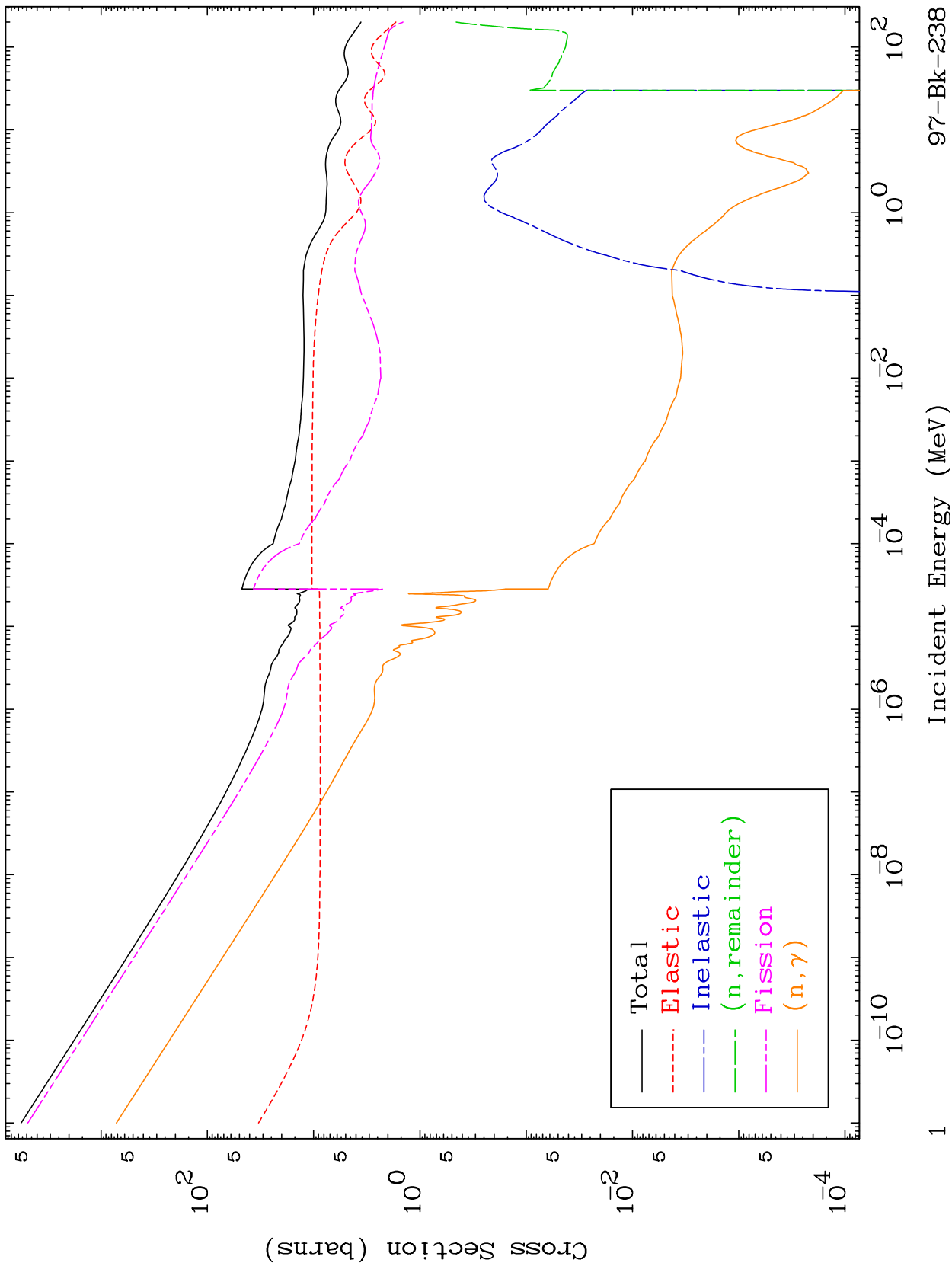
Press Mouse Button to Start

MAT 9719

Major

293 Kelvin Cross Sections

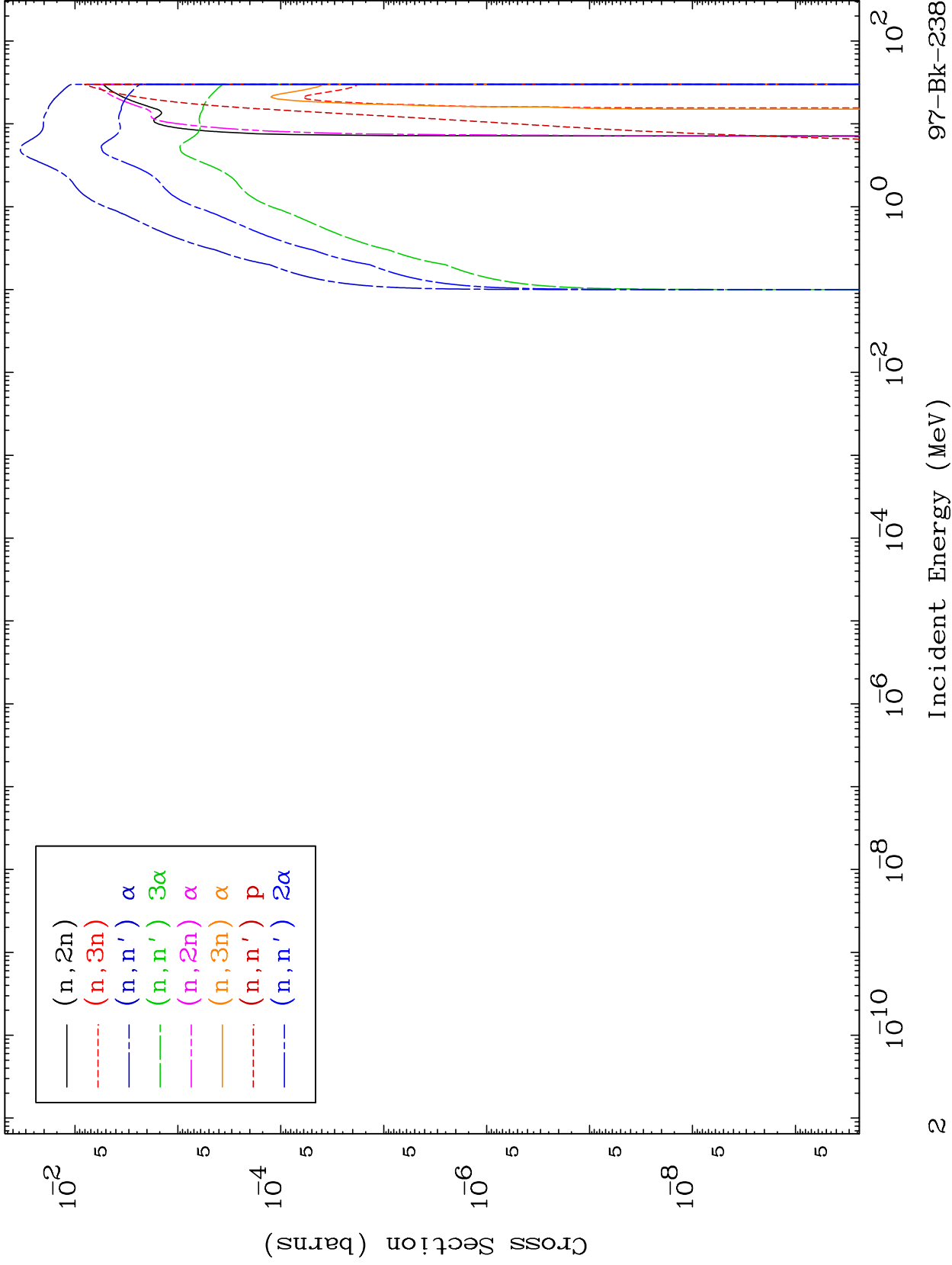
97-Bk-238



MAT 9719

Neutron Production  
293 Kelvin Cross Sections

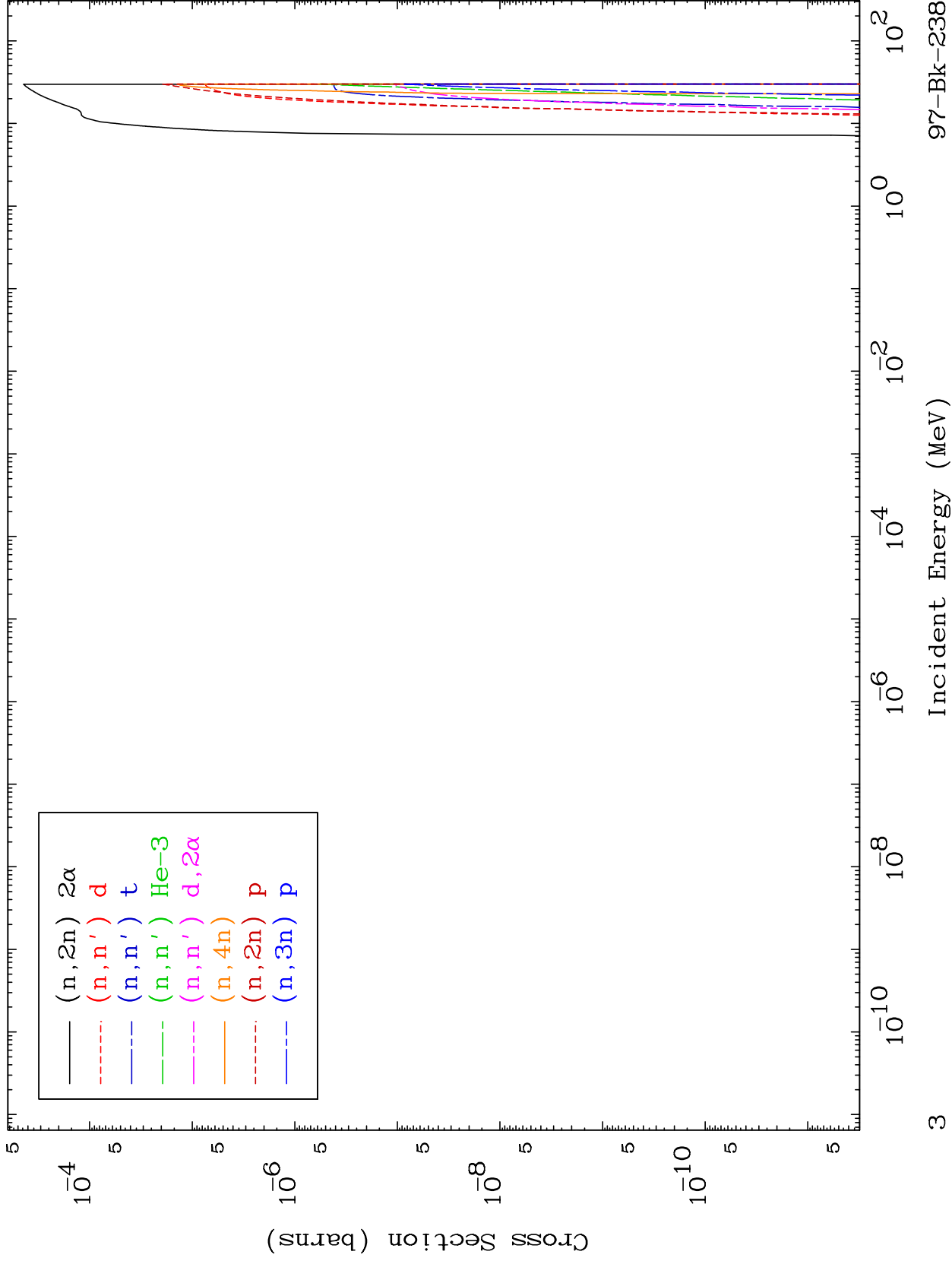
97-Bk-238



MAT 9719

Neutron Production  
293 Kelvin Cross Sections

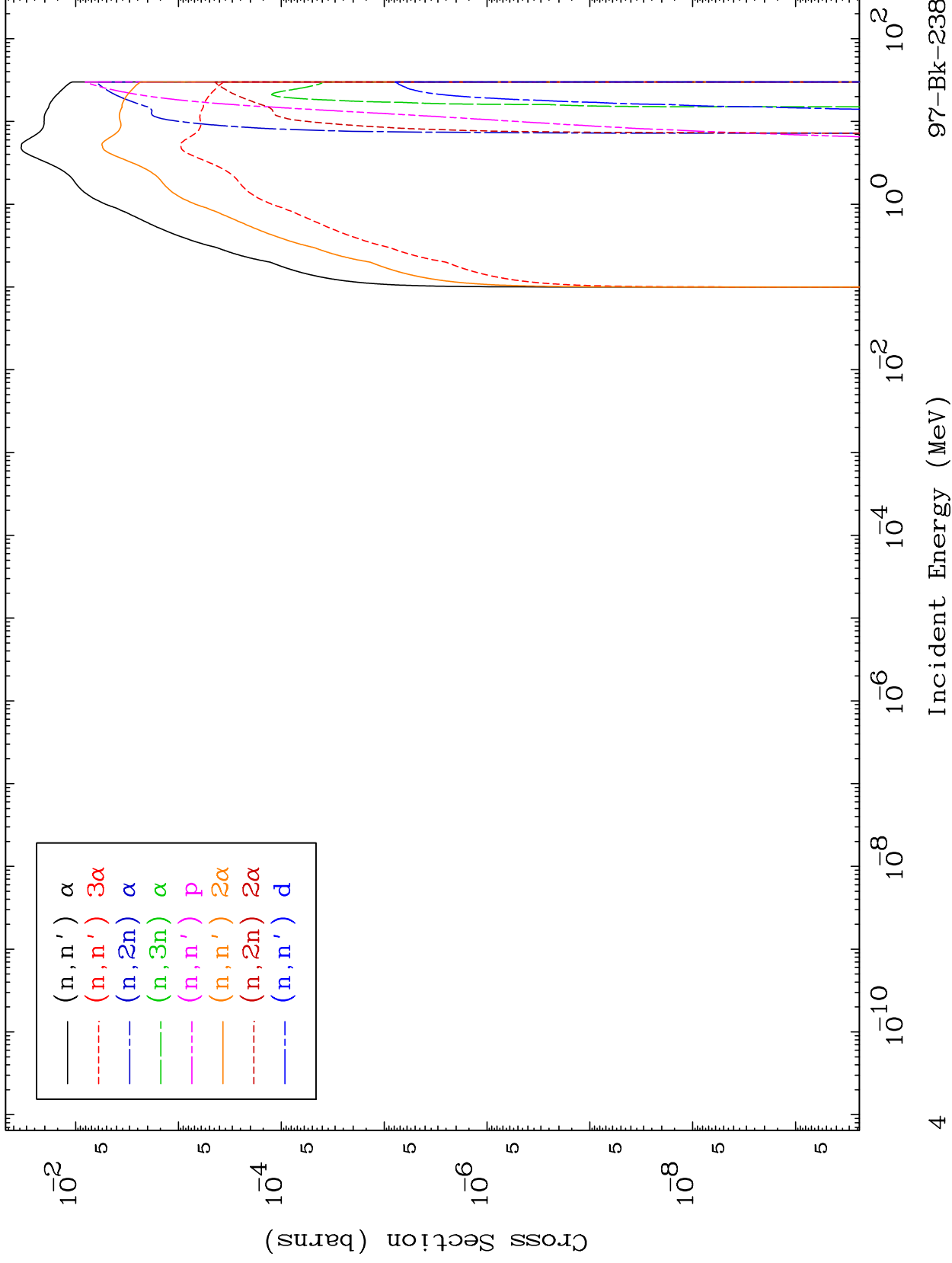
97-Bk-238



MAT 9719

Charged Particle  
293 Kelvin Cross Sections

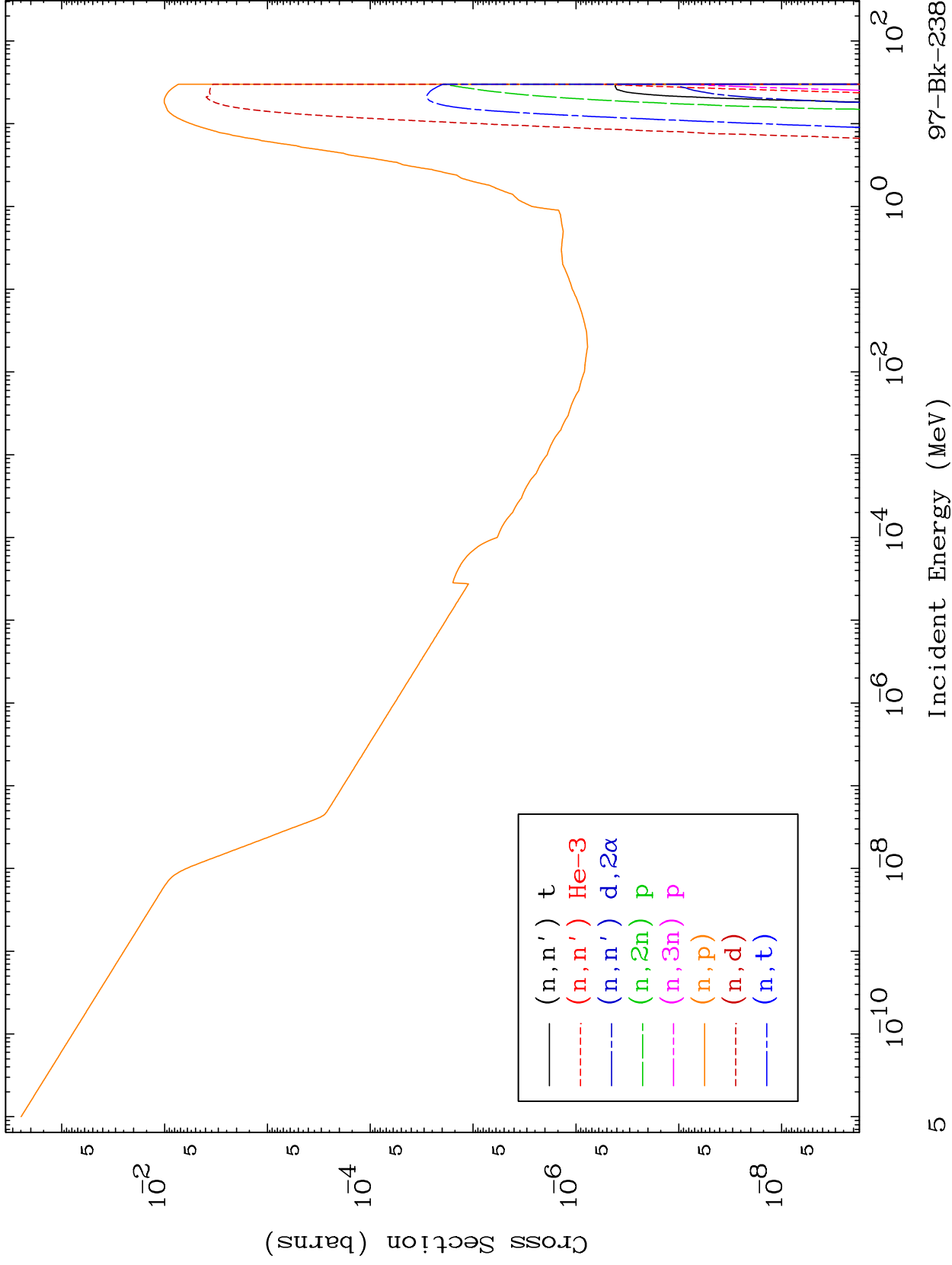
97-Bk-238



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Charged Particle  
293 Kelvin Cross Sections

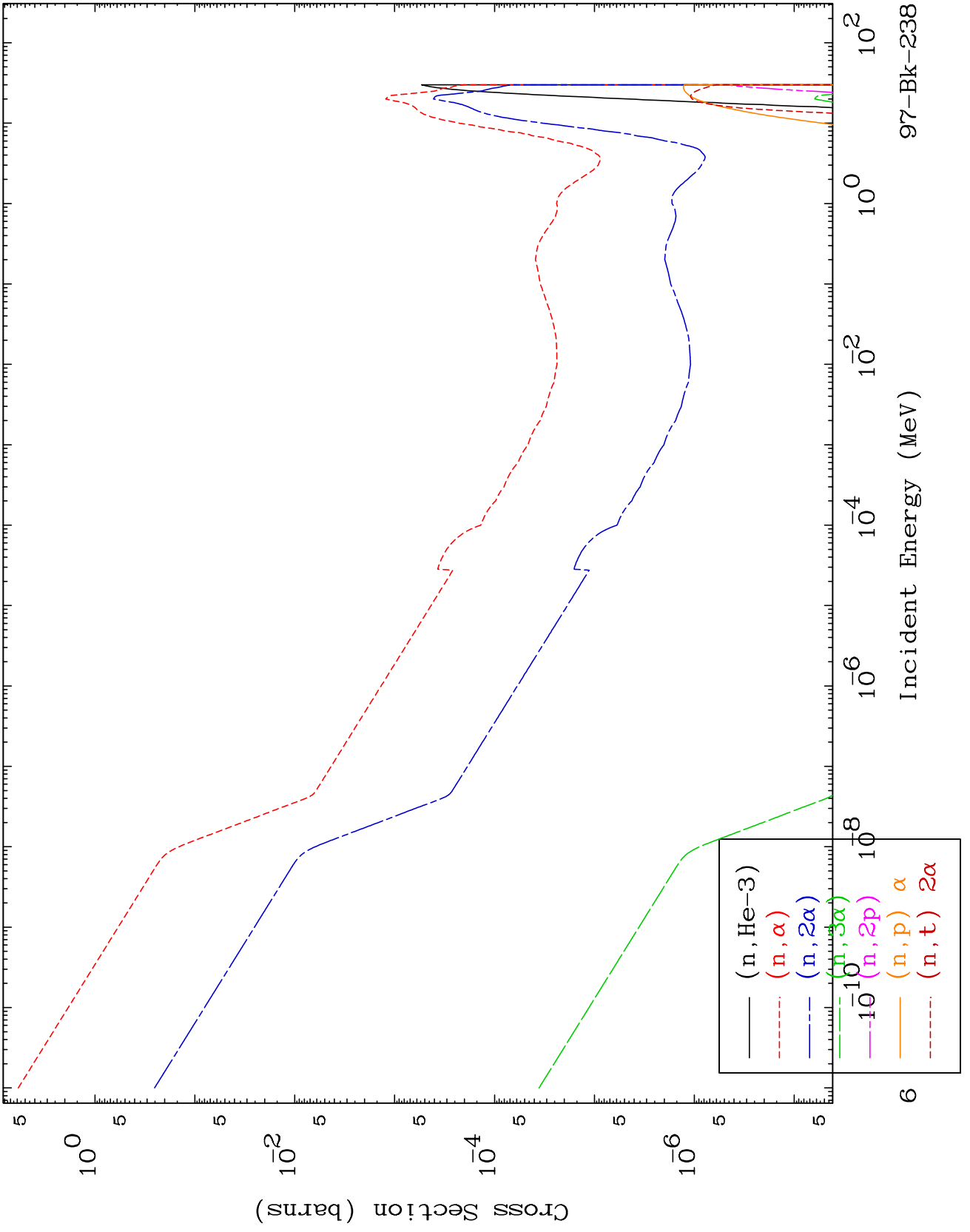
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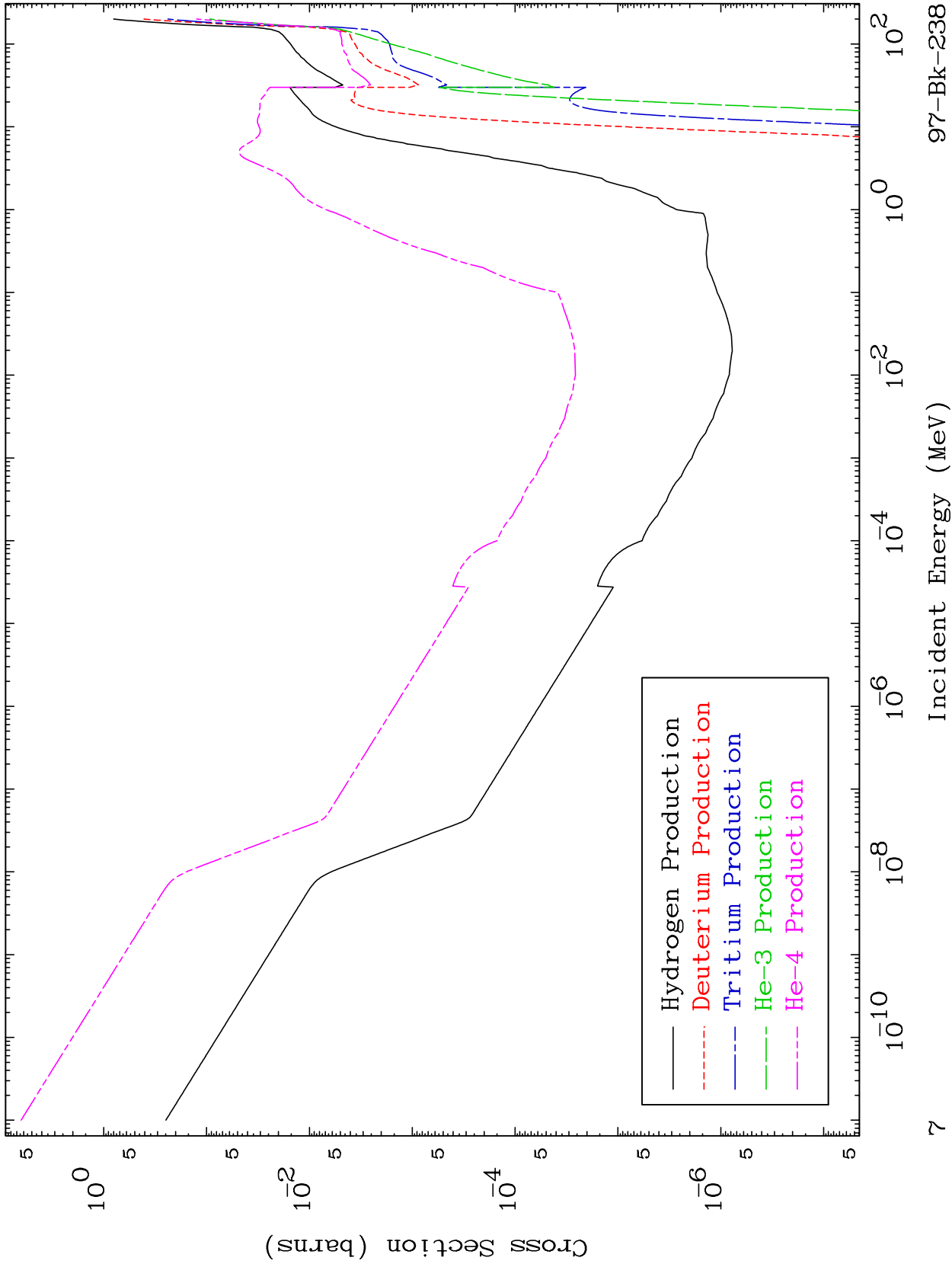
MAT 9719

Charged Particle  
293 Kelvin Cross Sections

97-Bk-238



97-Bk-238

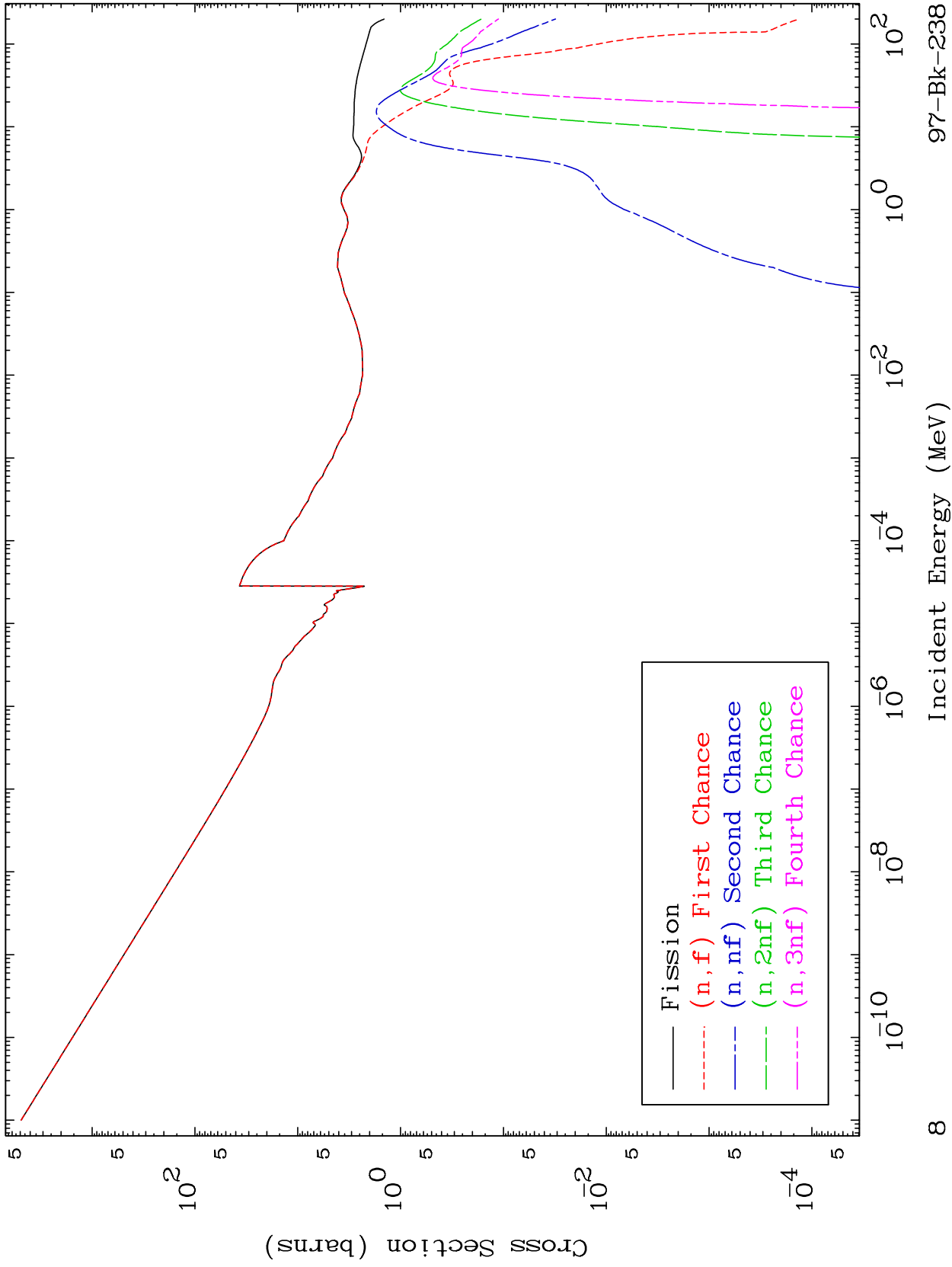




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Fission  
293 Kelvin Cross Sections

97-Bk-238

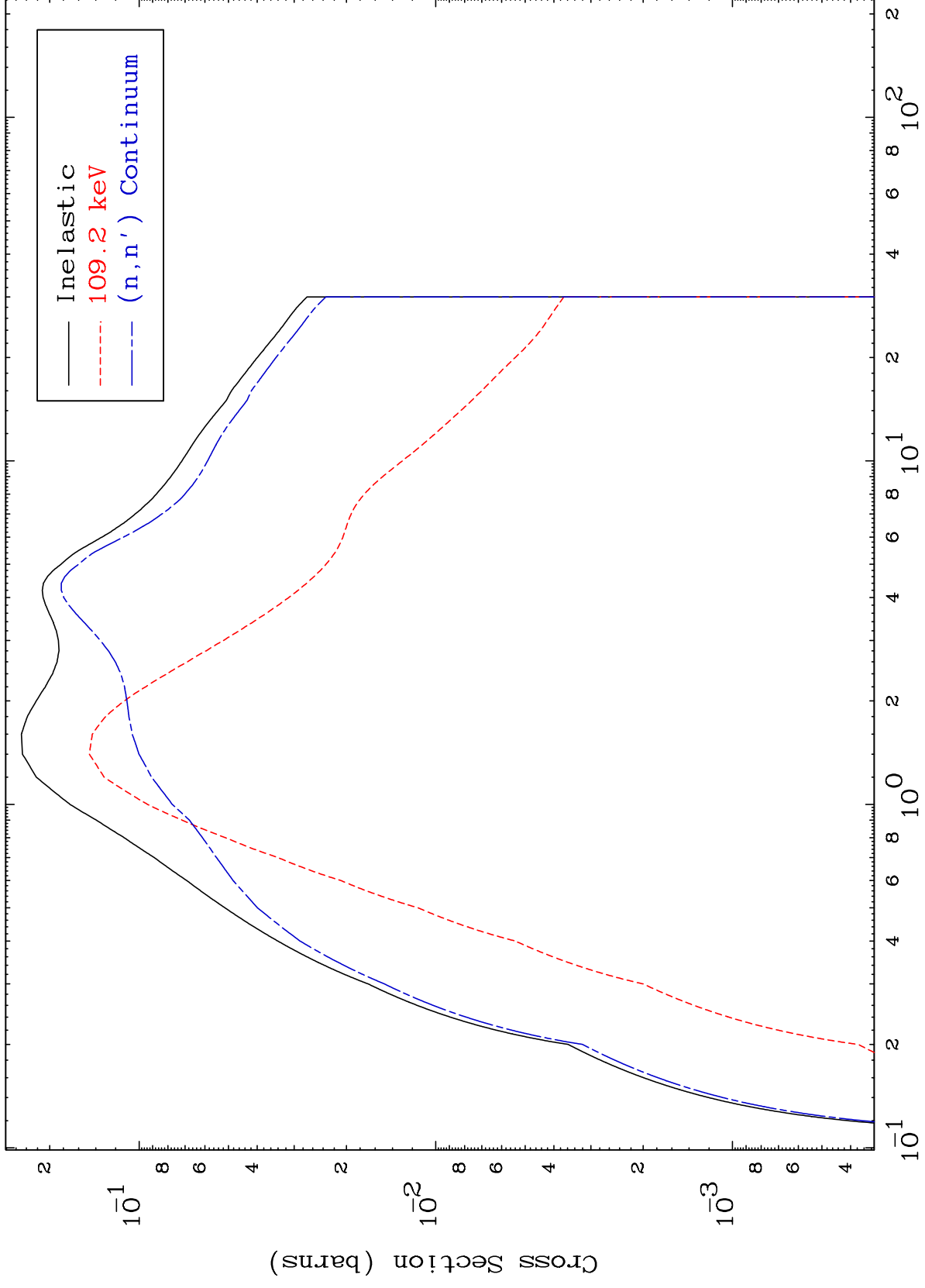


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(n,n') Level

97-Bk-238

293 Kelvin Cross Sections



Incident Energy (MeV)

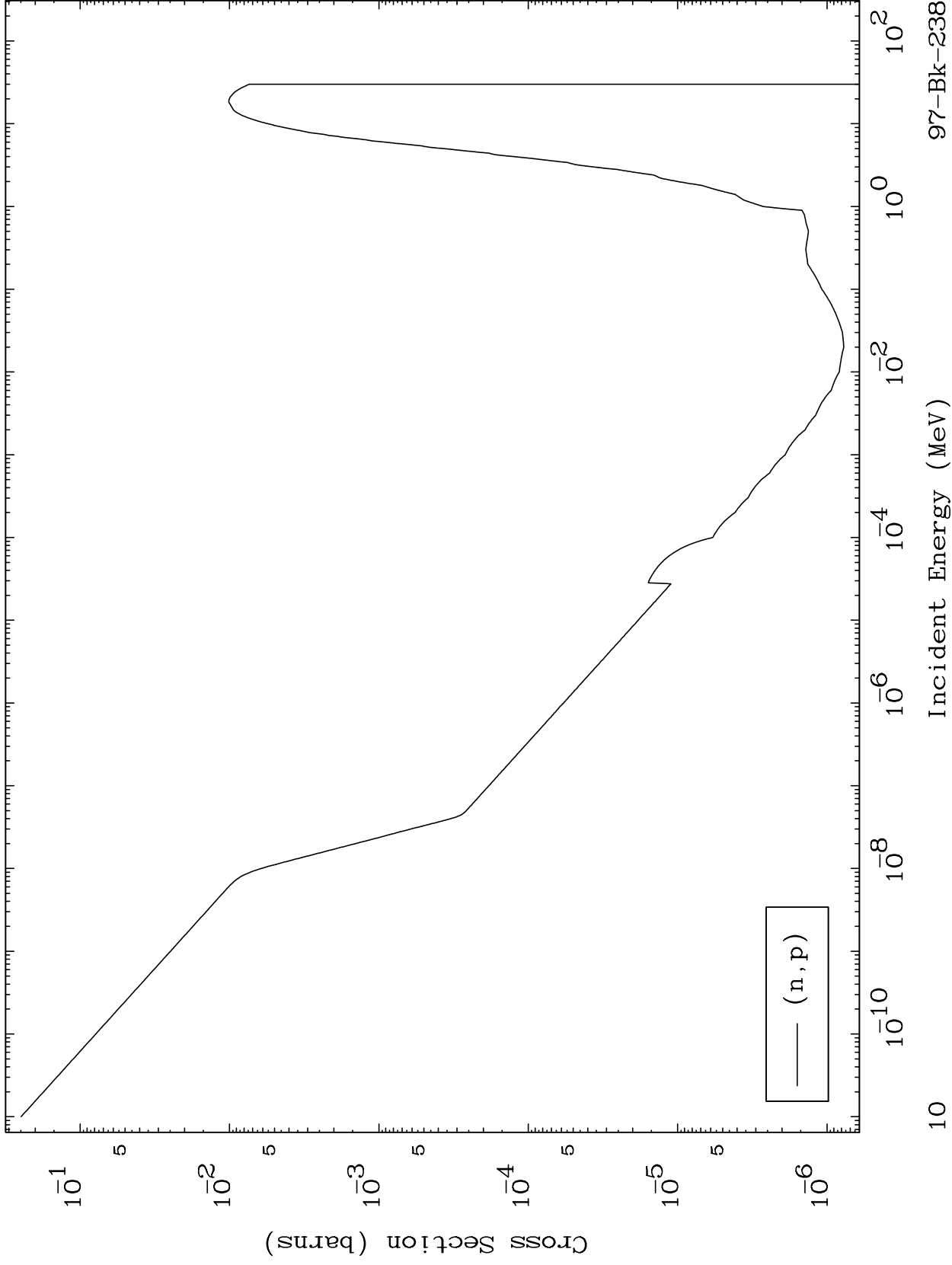
97-Bk-238

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(n,p) Levels  
293 Kelvin Cross Sections

97-Bk-238



10

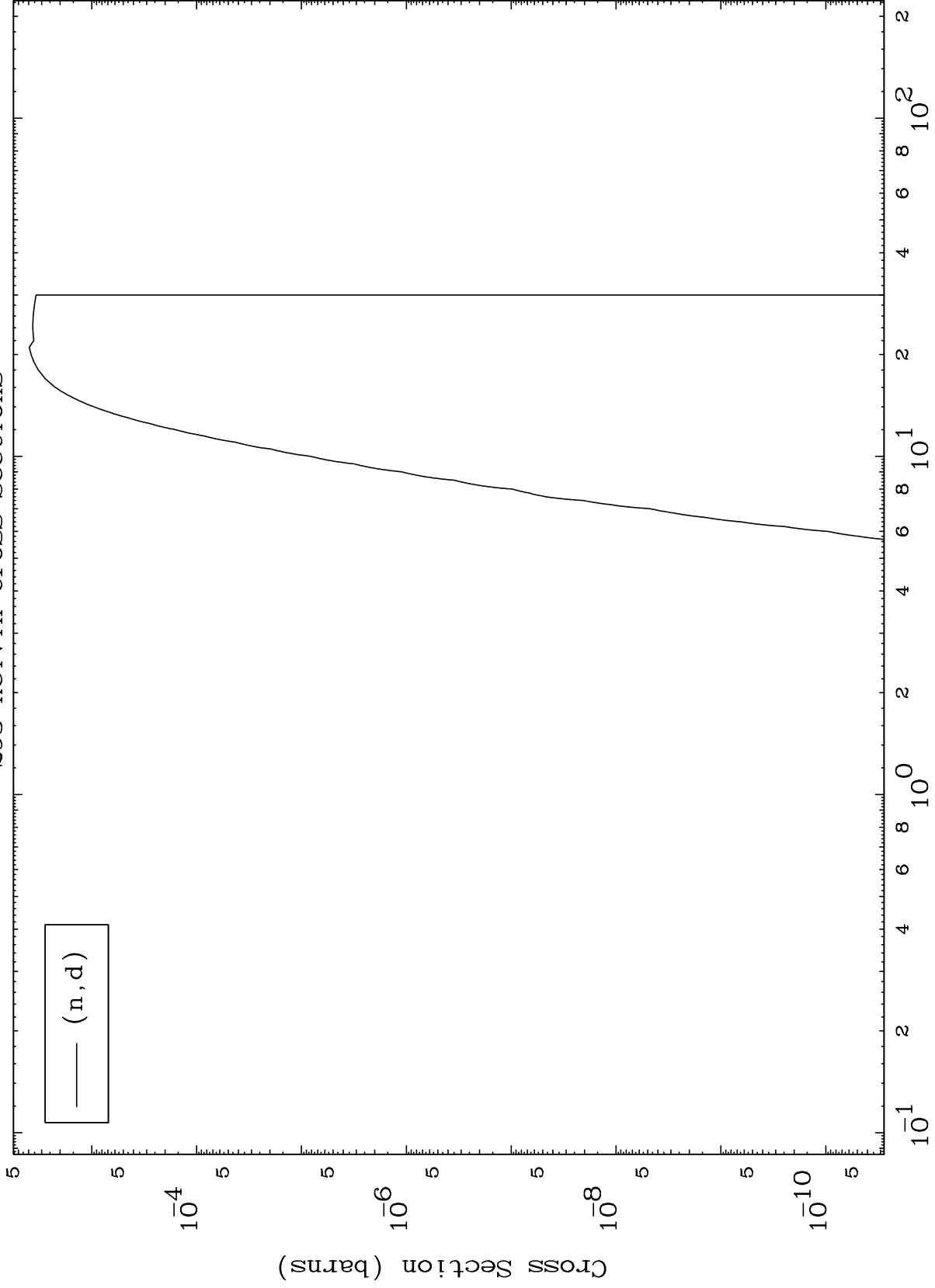
Incident Energy (MeV)

97-Bk-238

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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-238



11

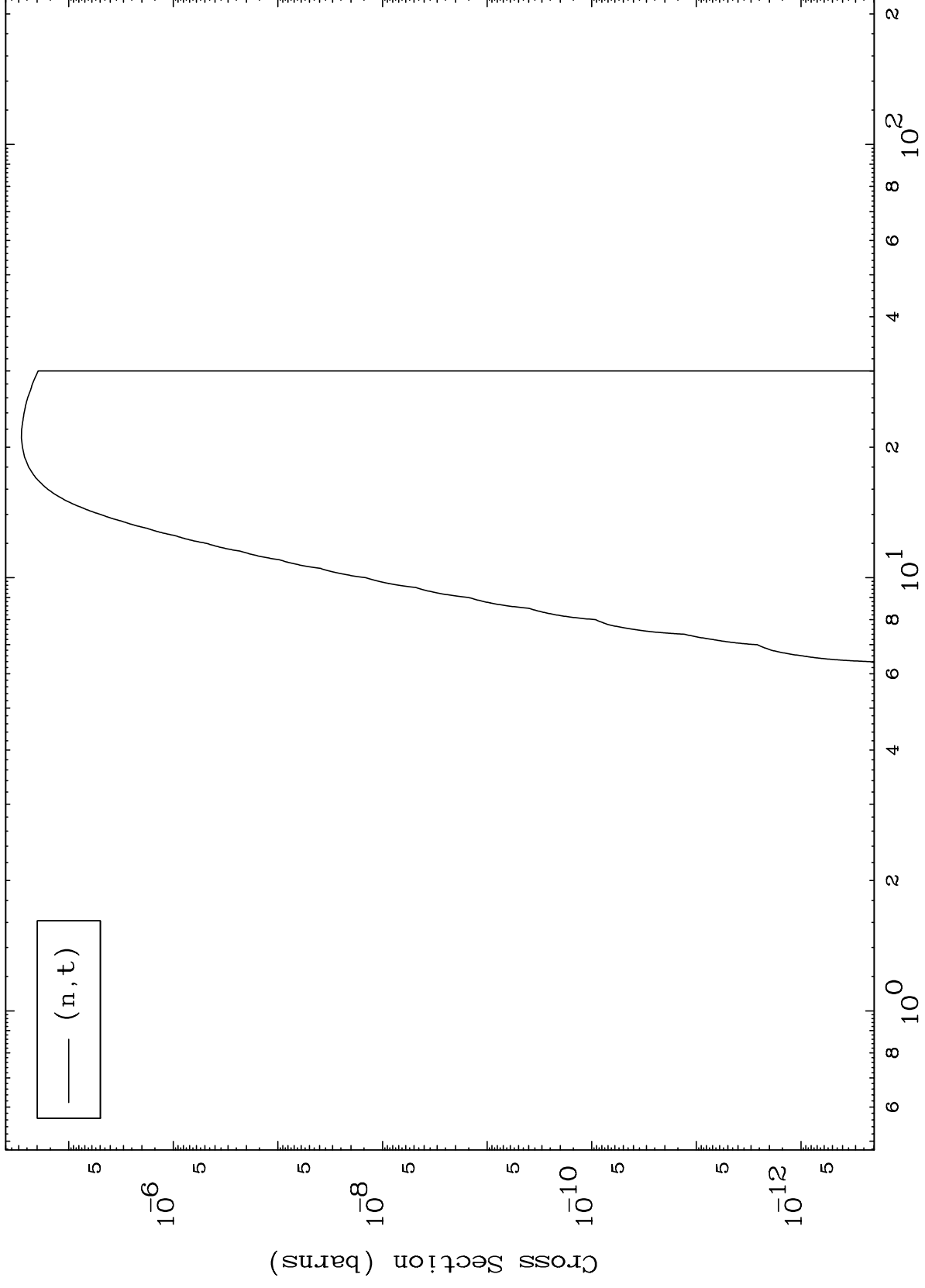
Incident Energy (MeV)

97-Bk-238

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(n,t) Levels  
293 Kelvin Cross Sections

97-Bk-238



12

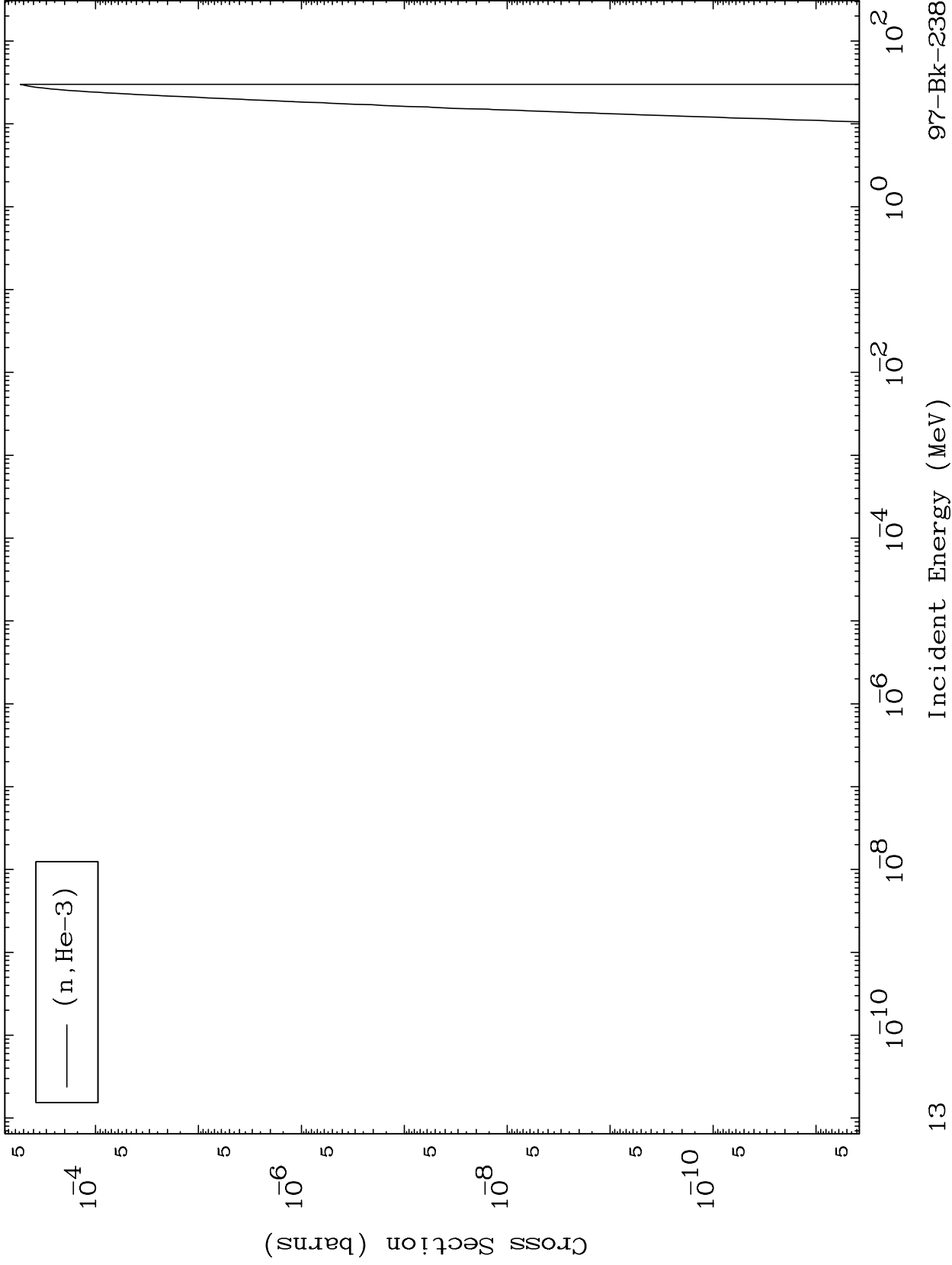
Incident Energy (MeV)

97-Bk-238

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(n,He3) Levels  
293 Kelvin Cross Sections

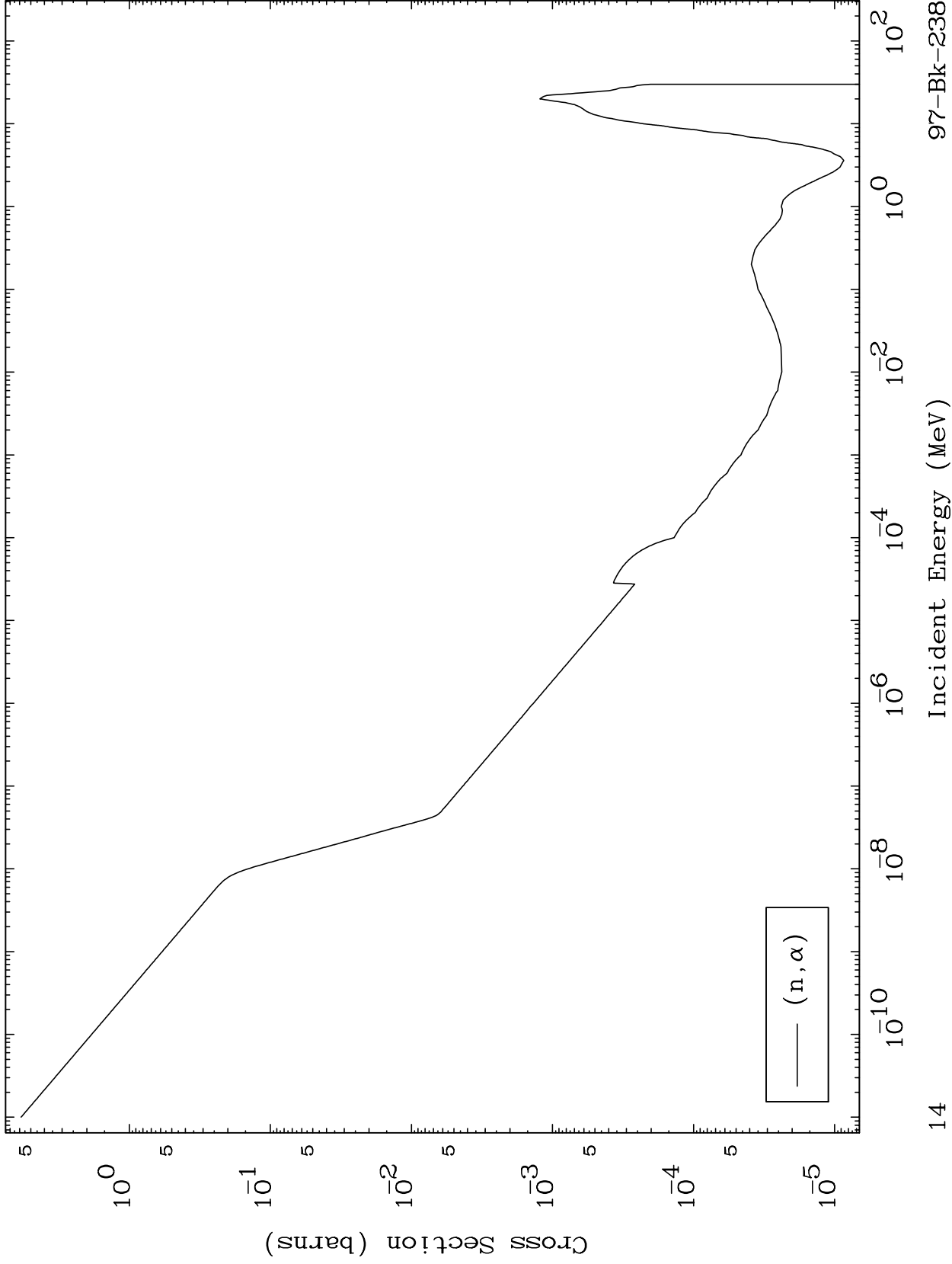
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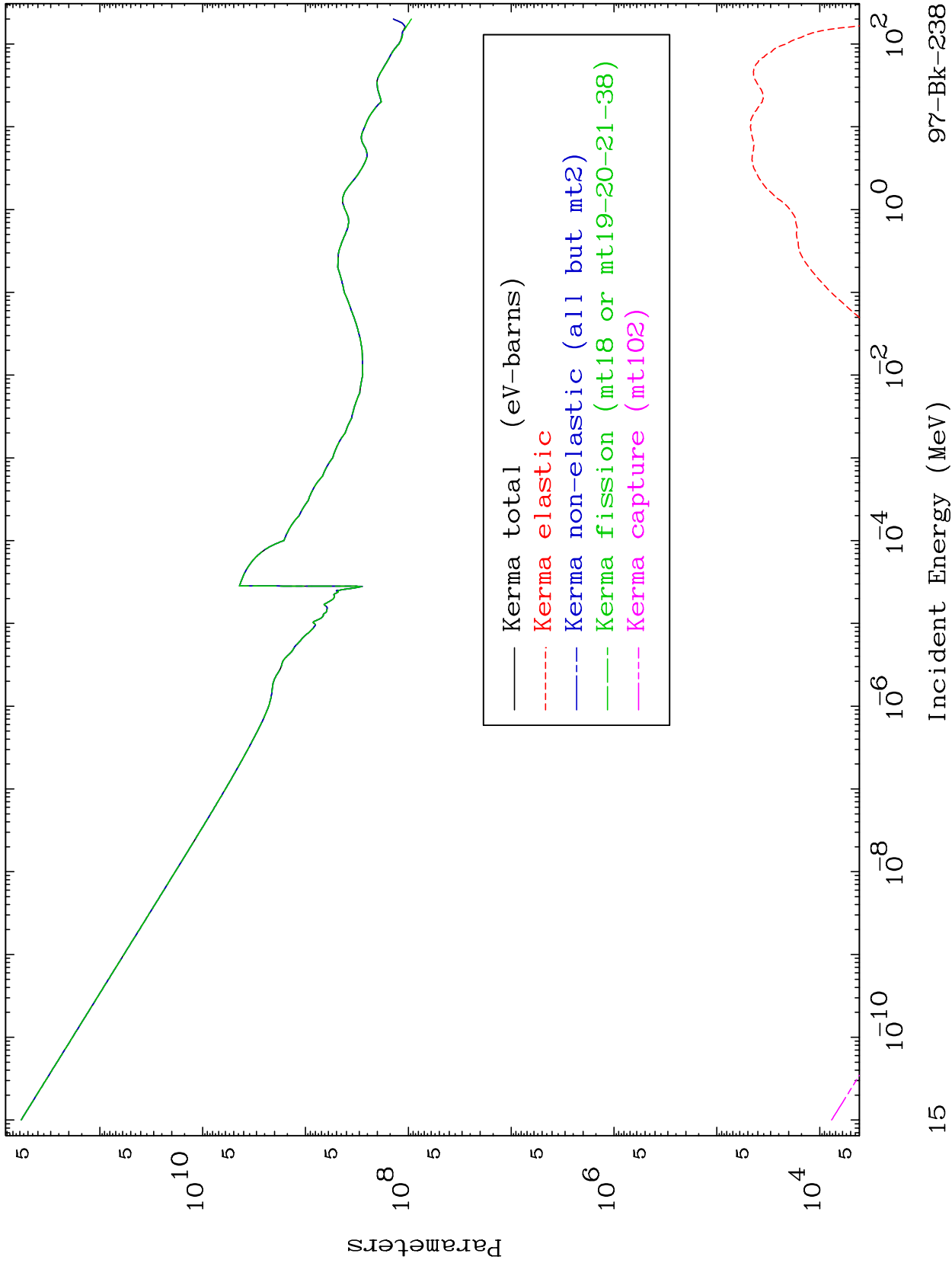


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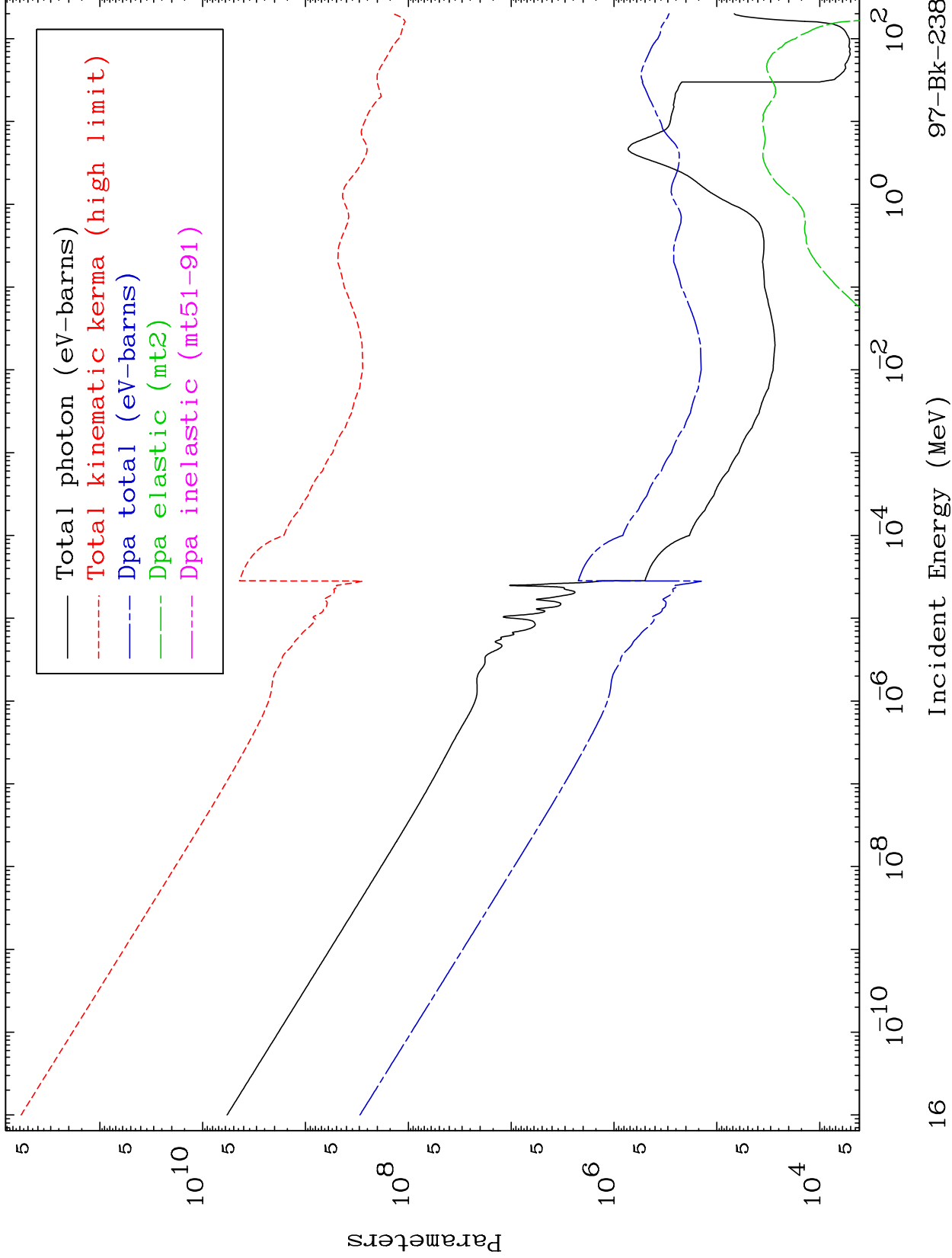
(n,α) Levels  
293 Kelvin Cross Sections

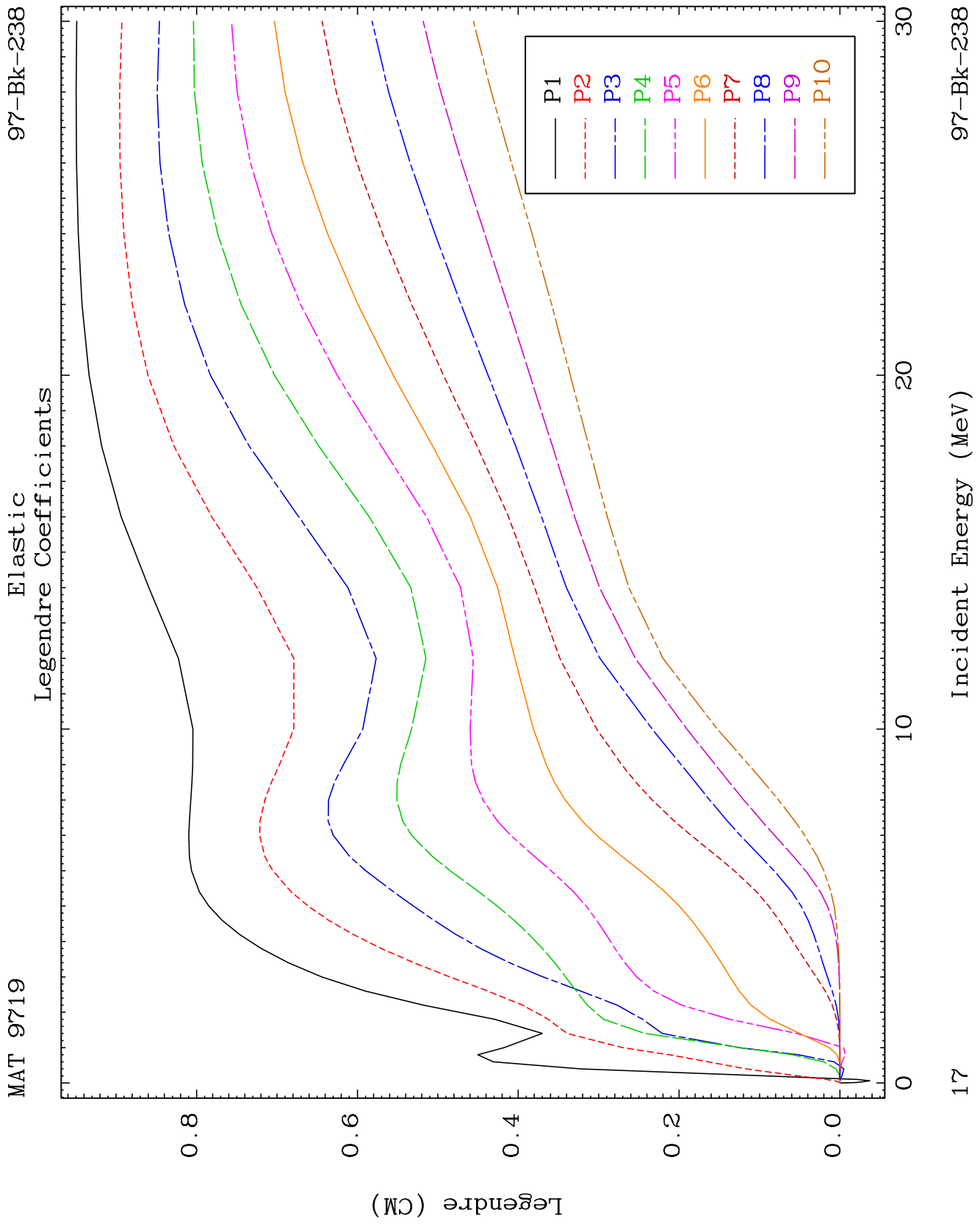
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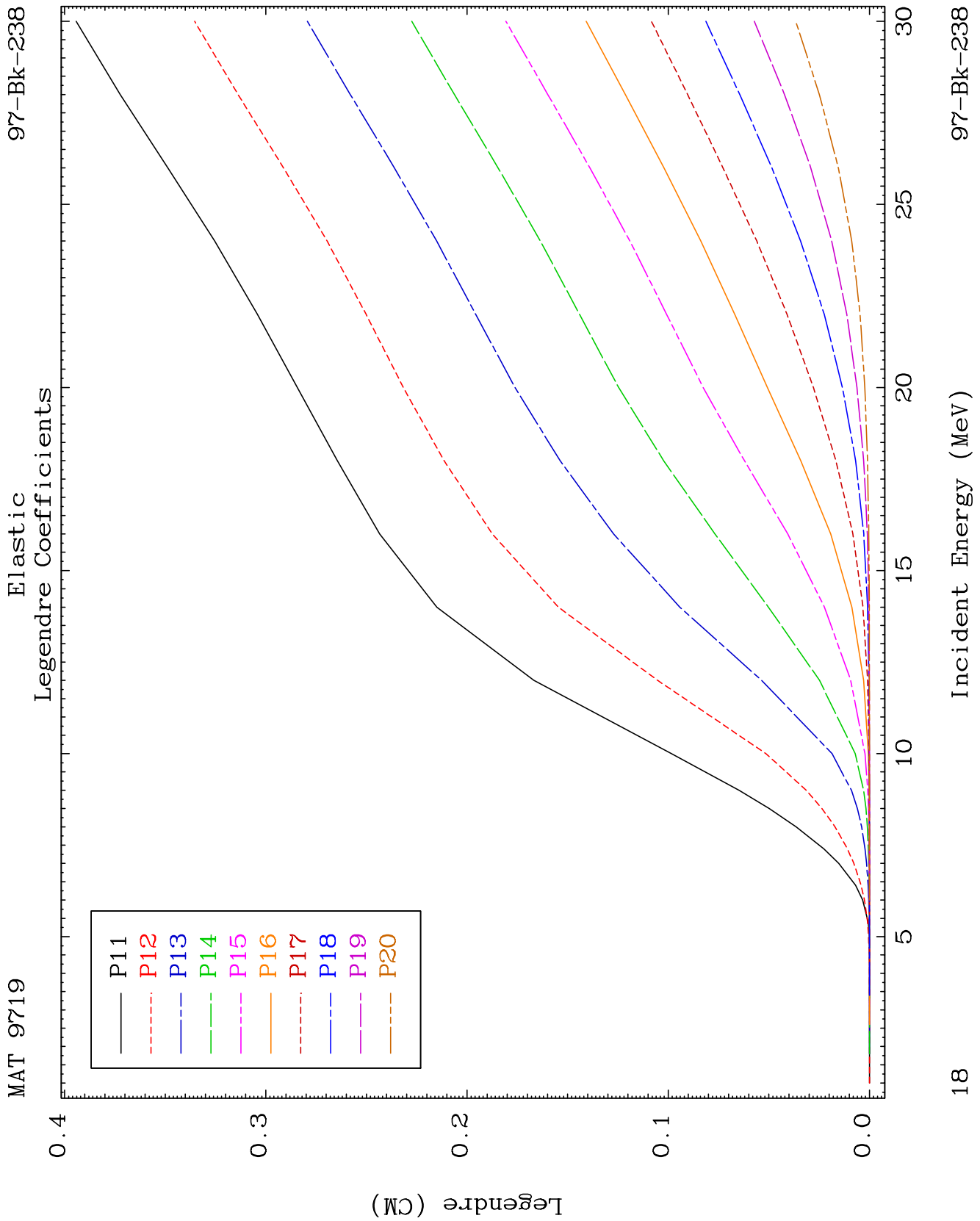




97-Bk-238

Incident Energy (MeV)

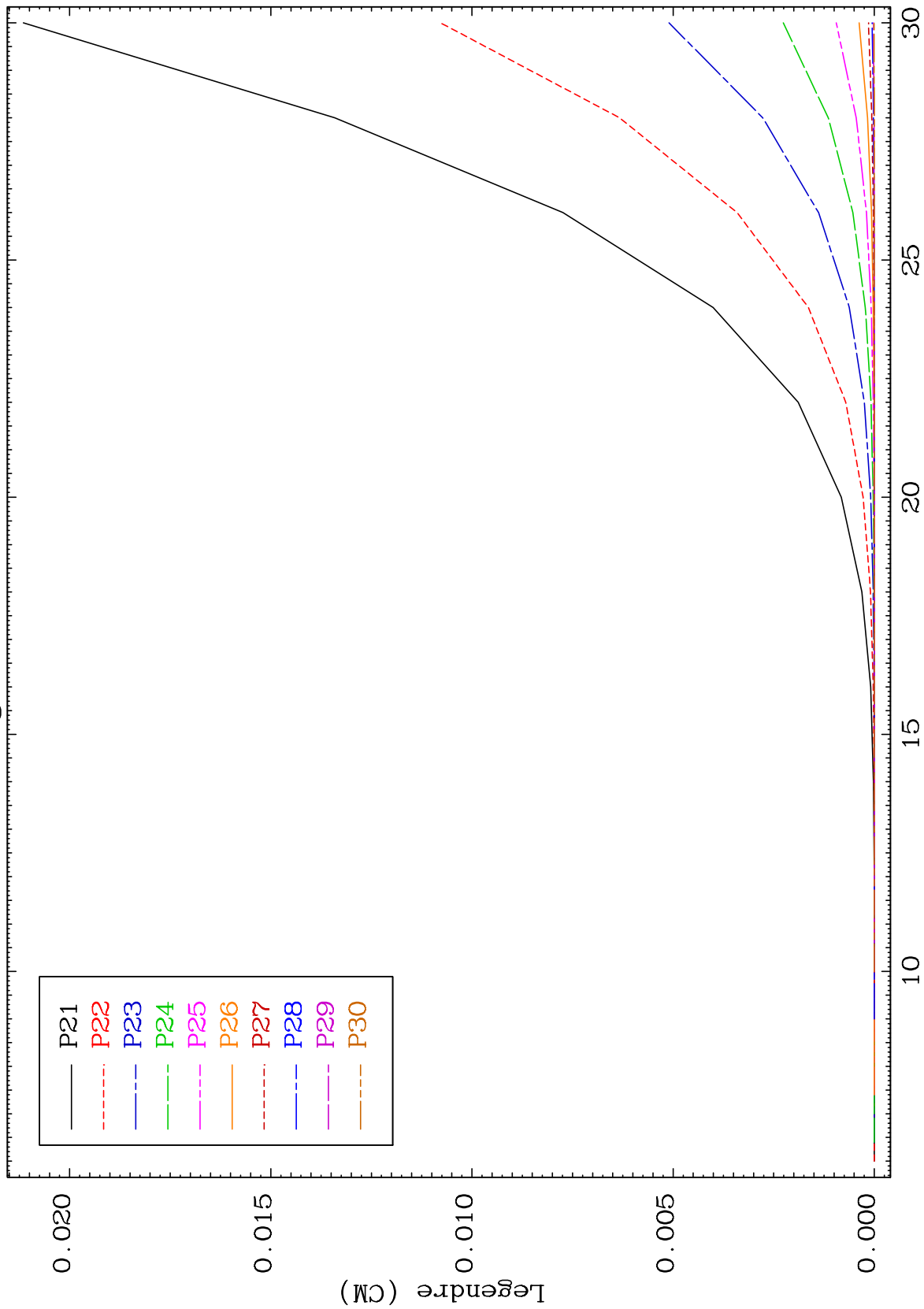
17



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### Elastic Legendre Coefficients

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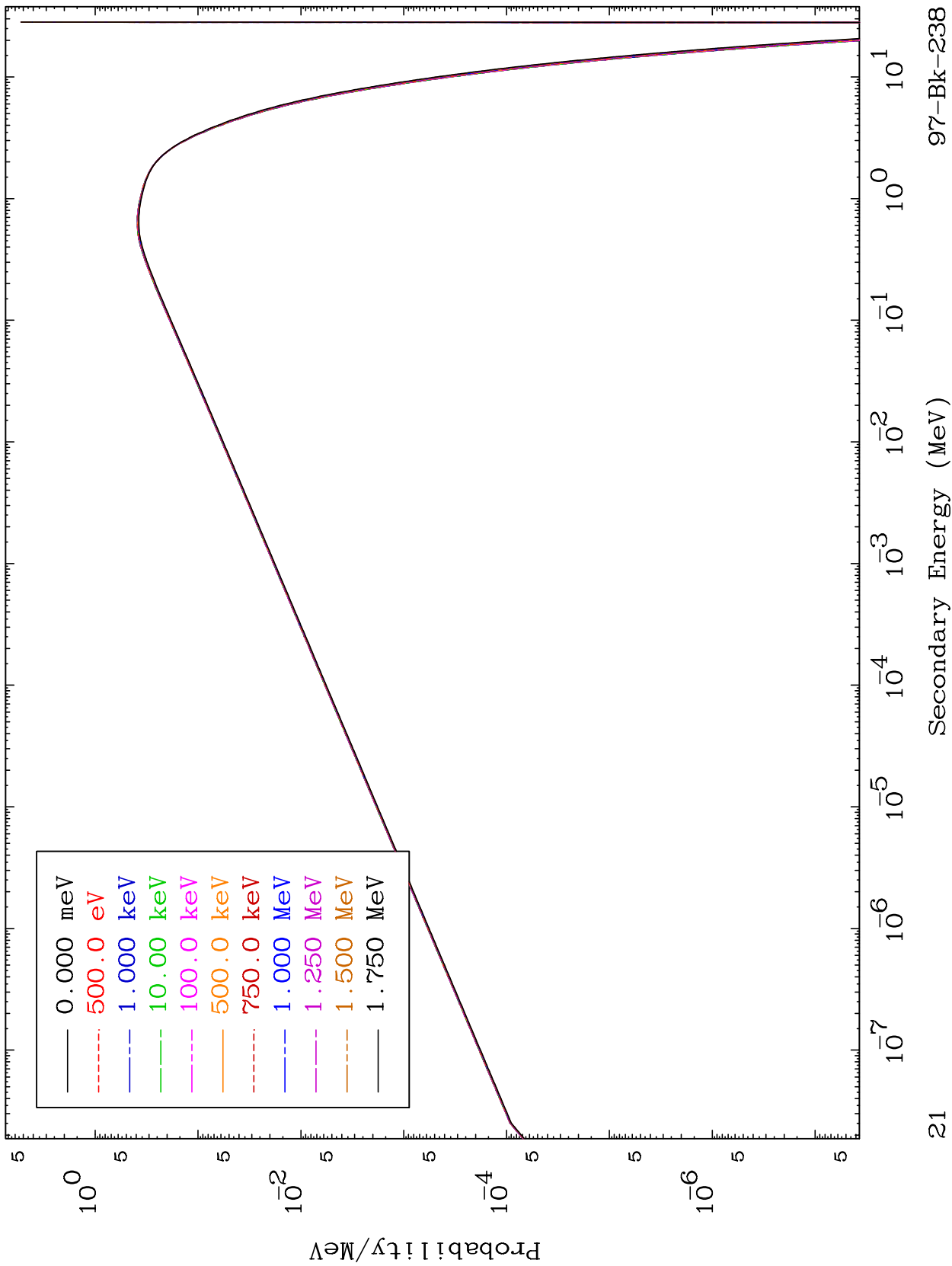
19

Incident Energy (MeV)

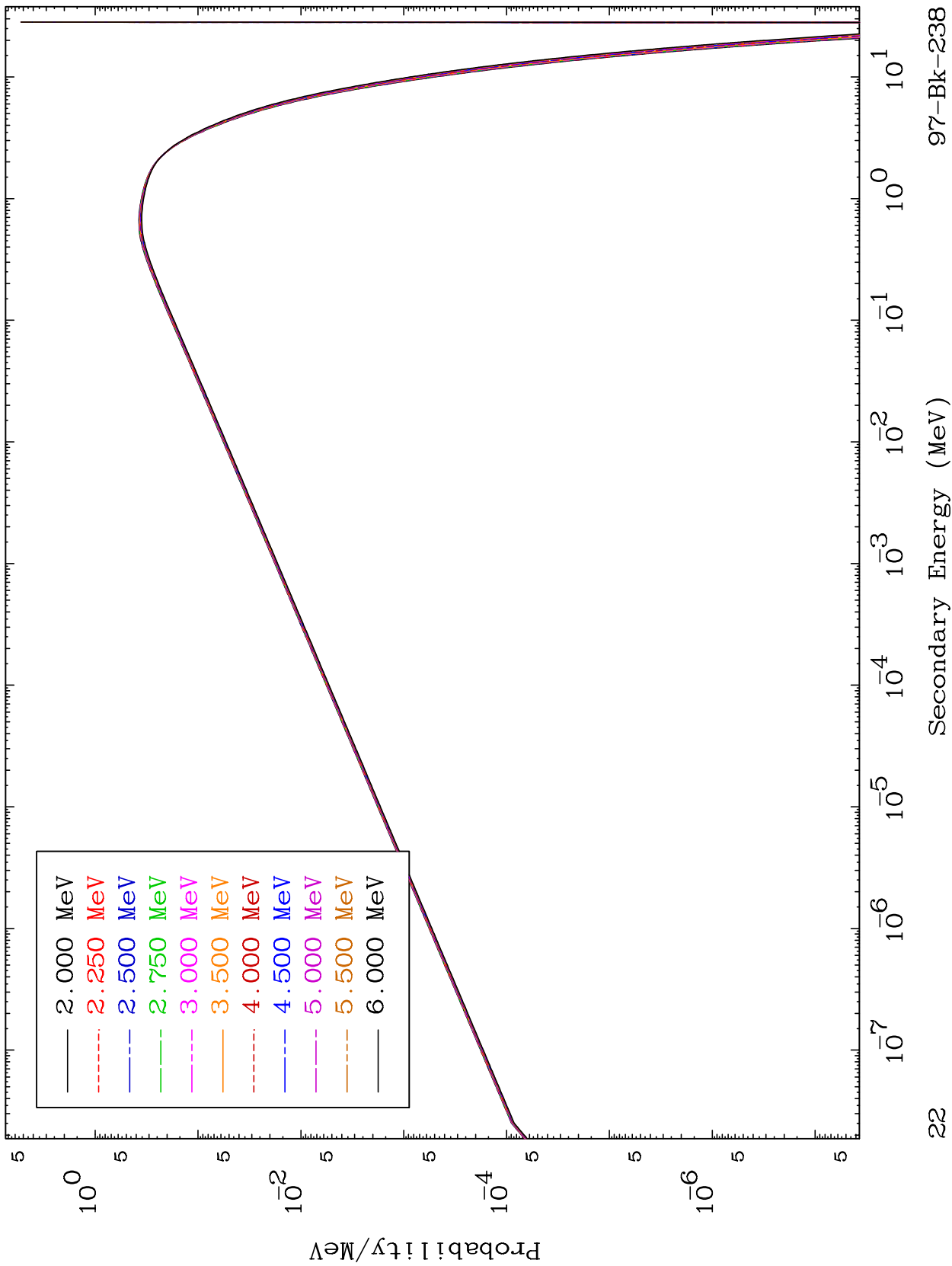
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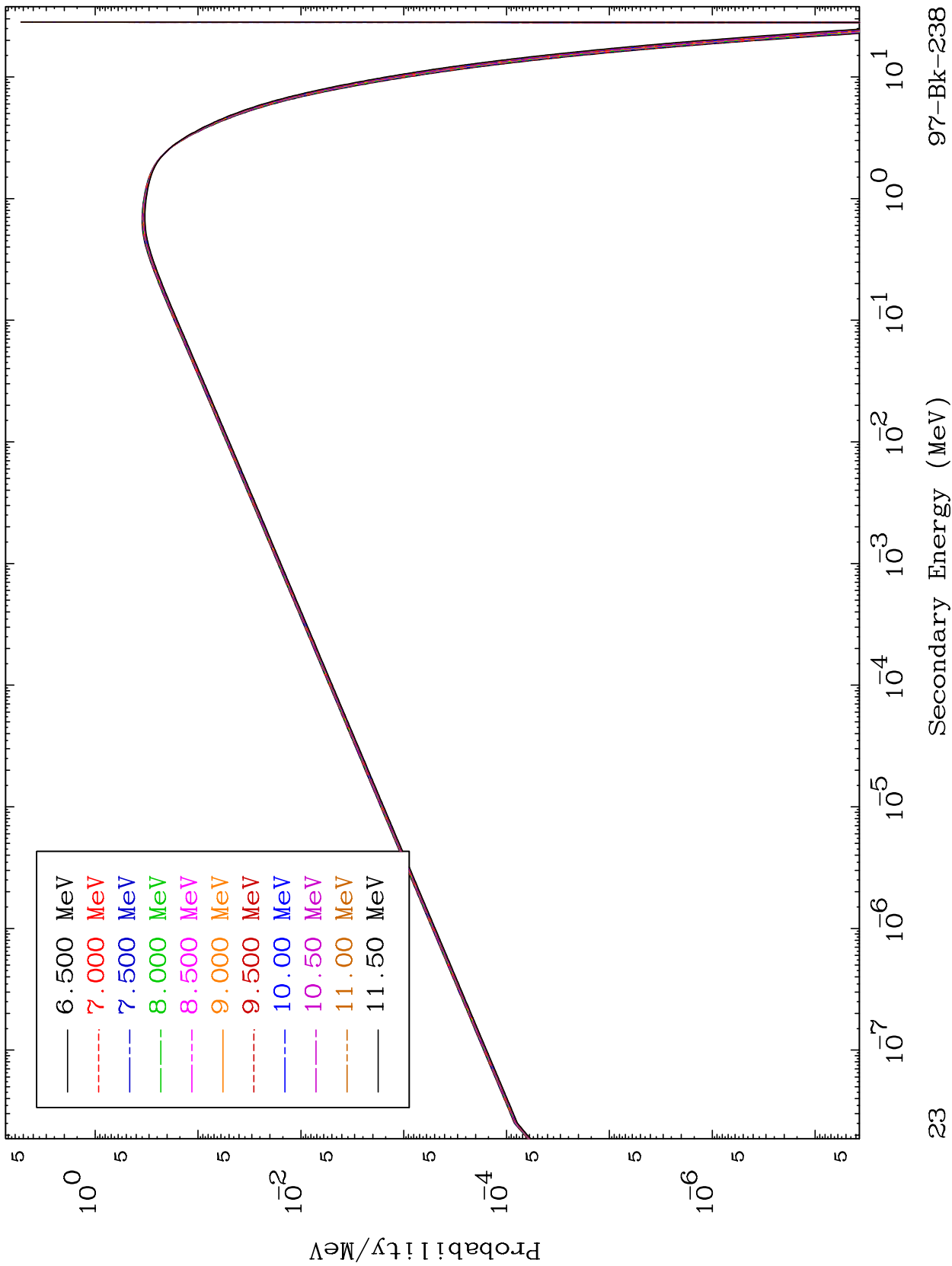
Fission Energy Distribution



Fission Energy Distribution

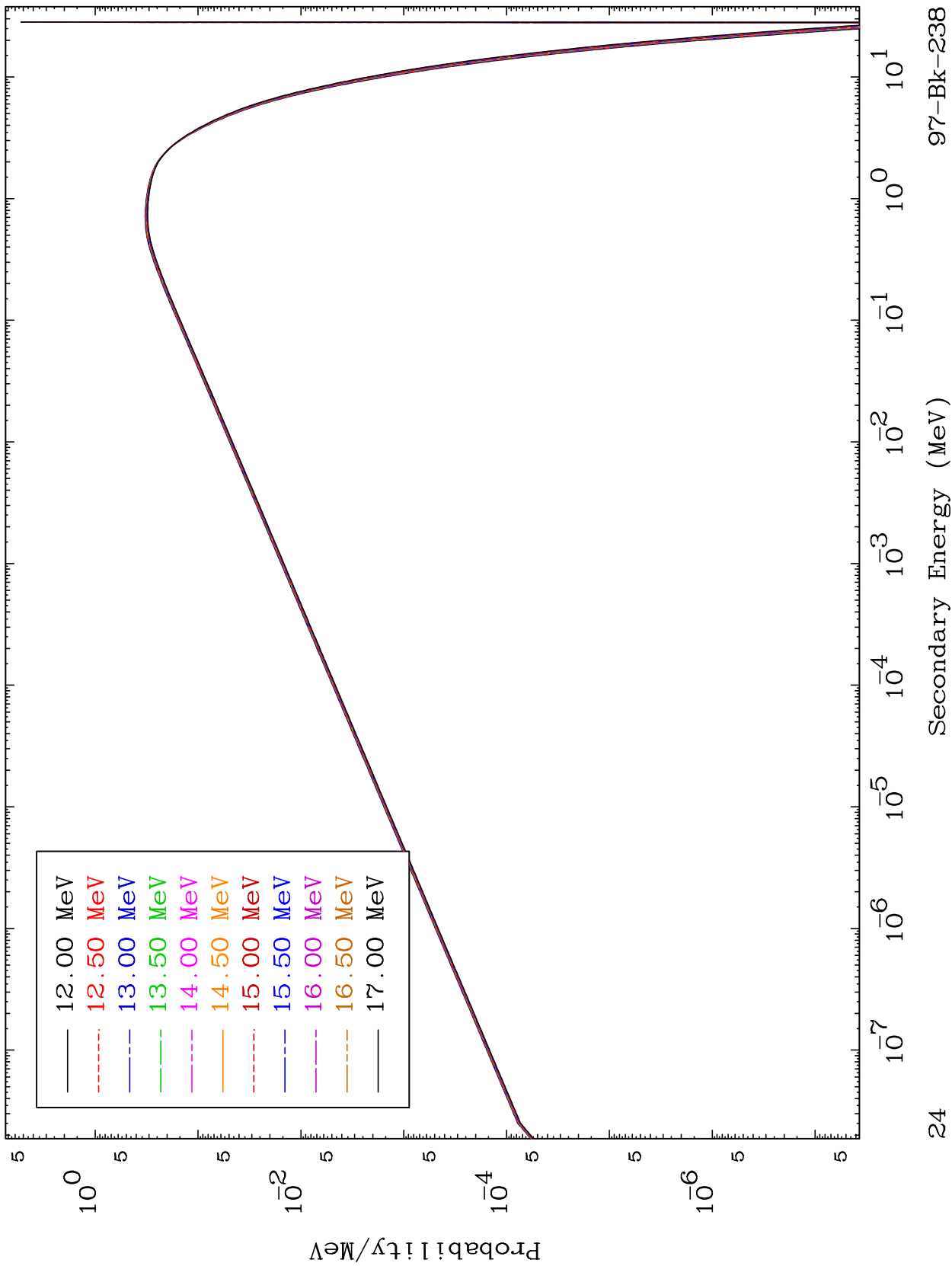


Fission Energy Distribution

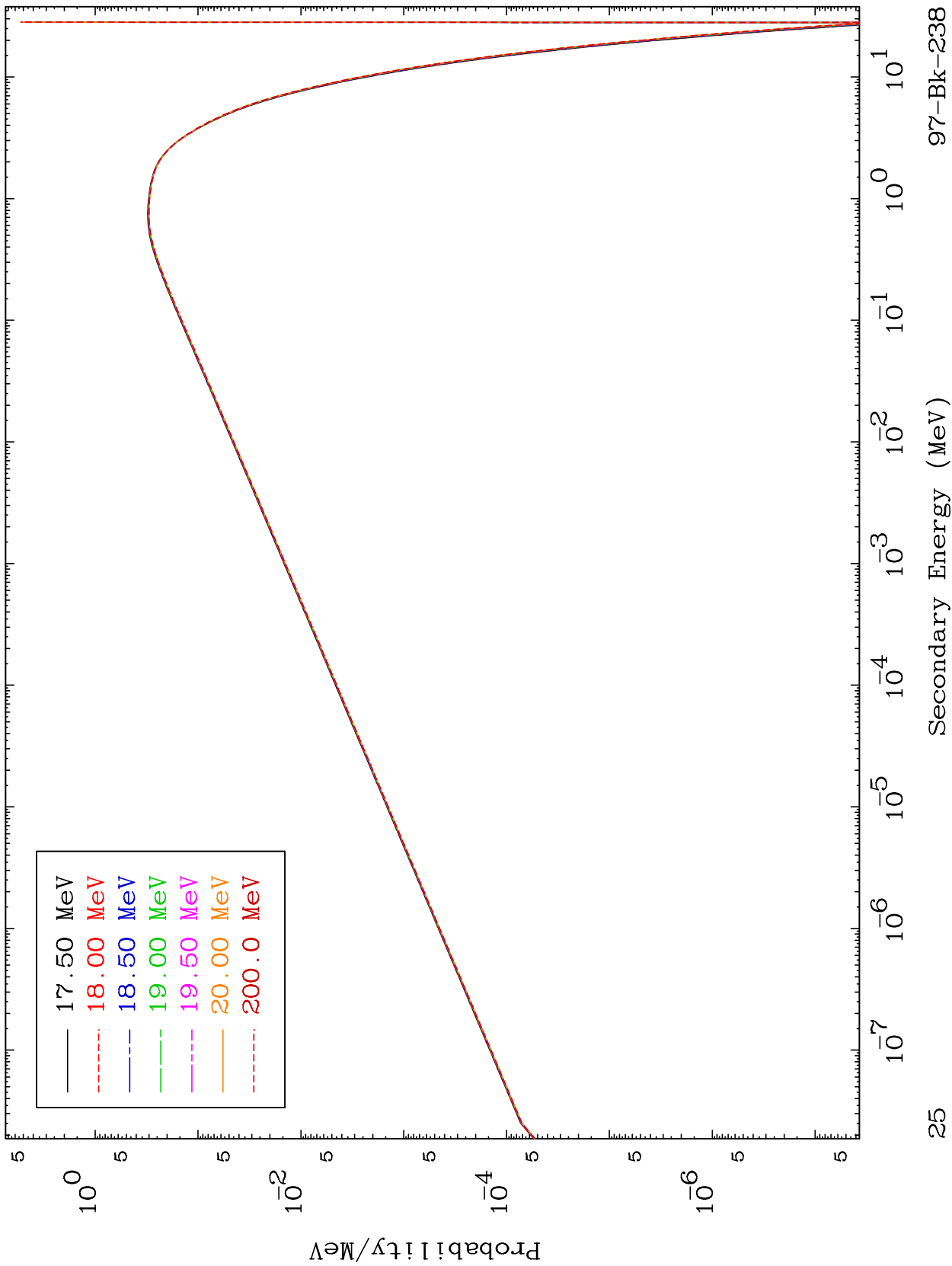




Fission  
Energy Distribution



Fission  
Energy Distribution

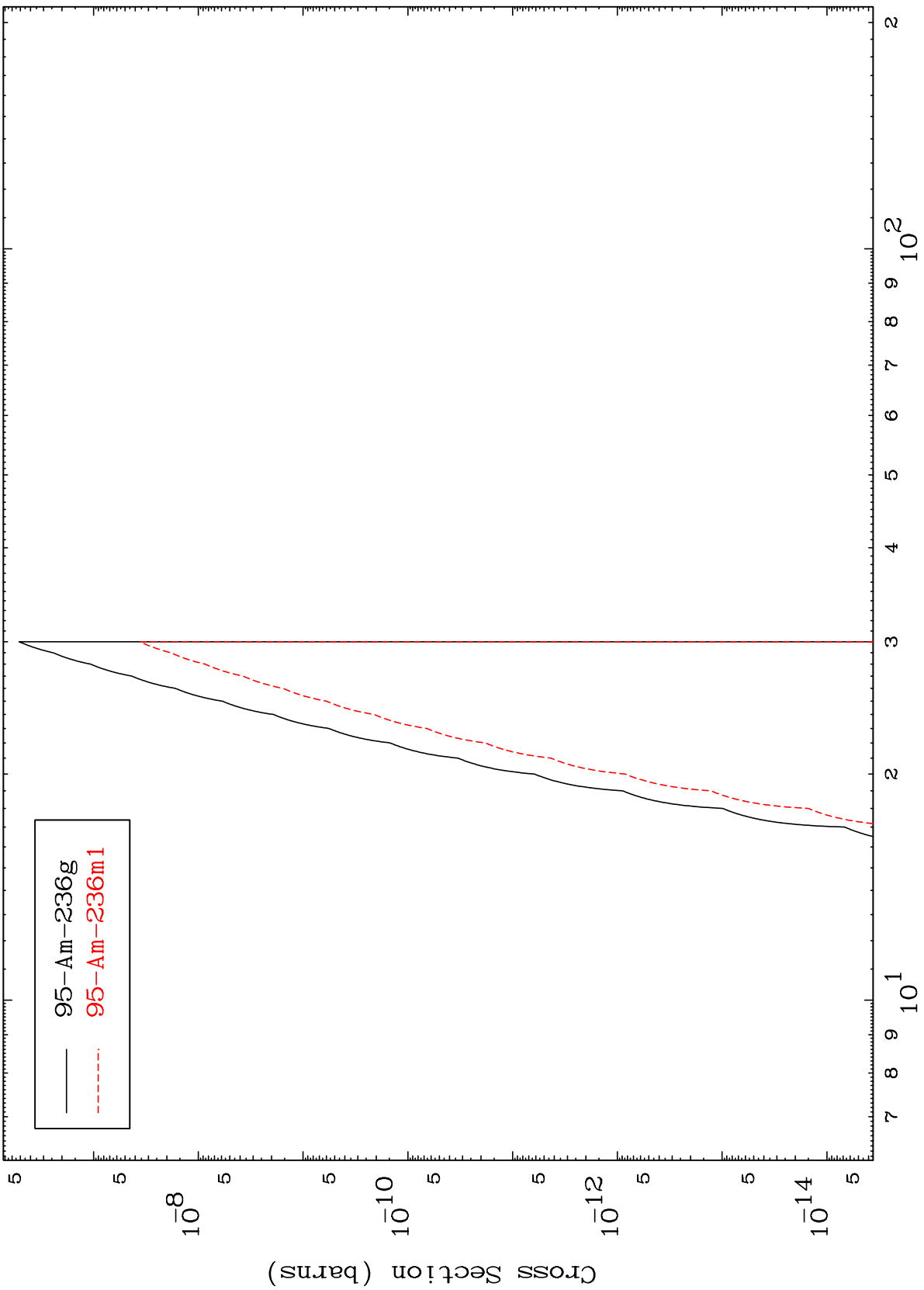


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(n,2n) p

97-Bk-238

Radionuclide Production Cross Section



95-Am-236g  
95-Am-236m1

26

Incident Energy (MeV)

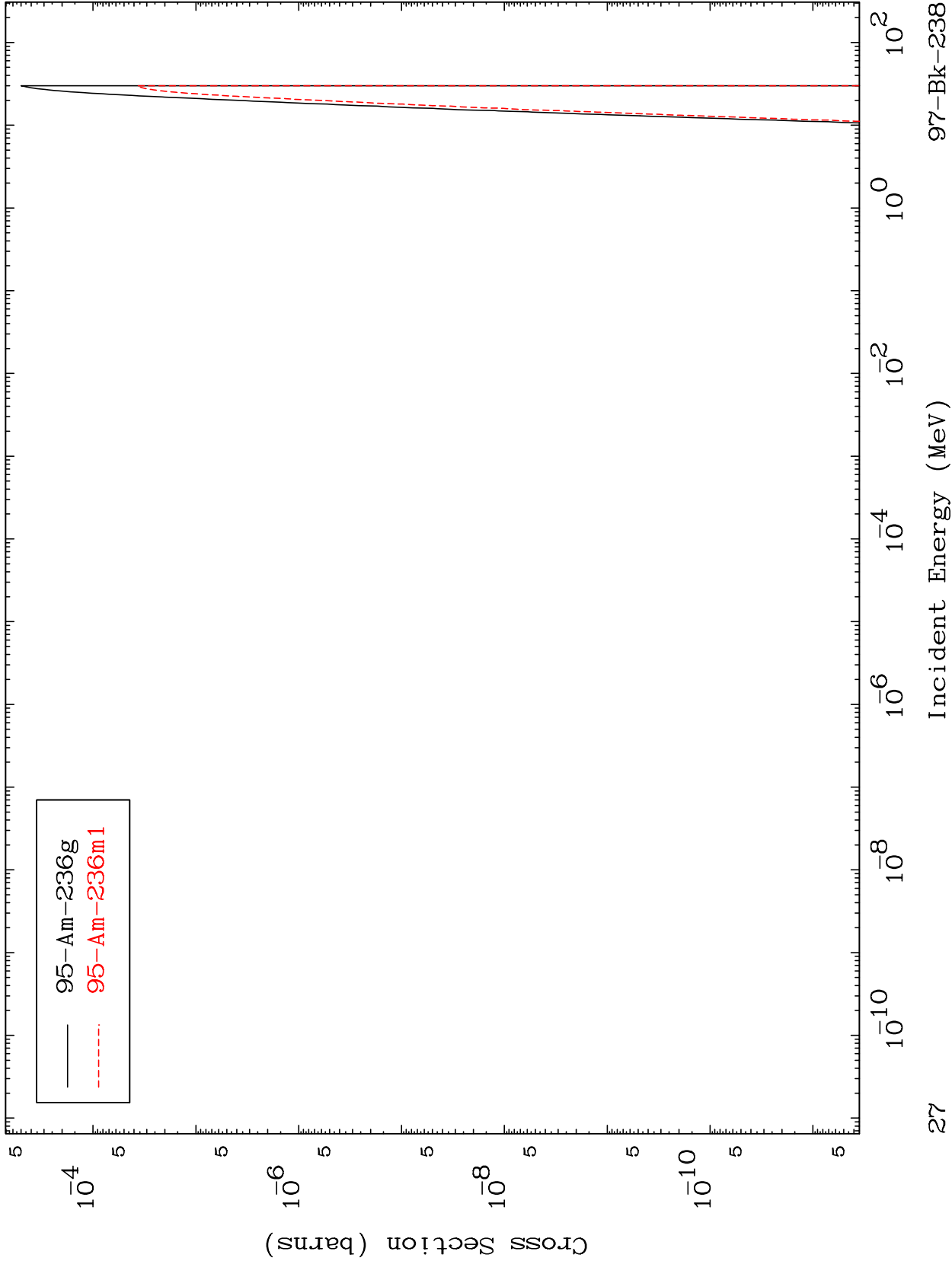
97-Bk-238

MAT 9719

(n,He-3)

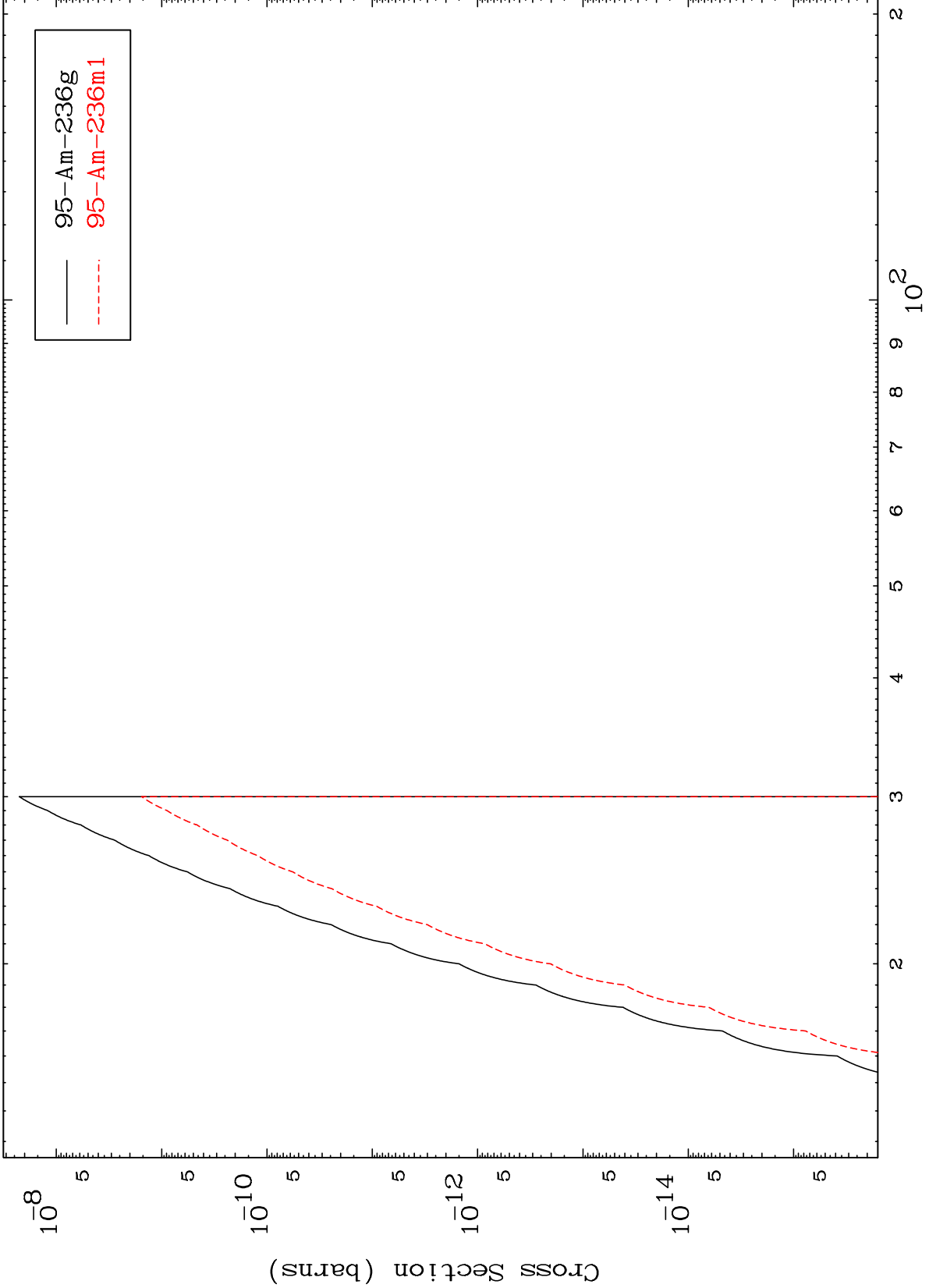
97-Bk-238

Radionuclide Production Cross Section



95-Am-236g  
95-Am-236m1

Radionuclide Production Cross Section



95-Am-236g  
95-Am-236m1