

Program EVALPLOT
(Version 2015-2)

by

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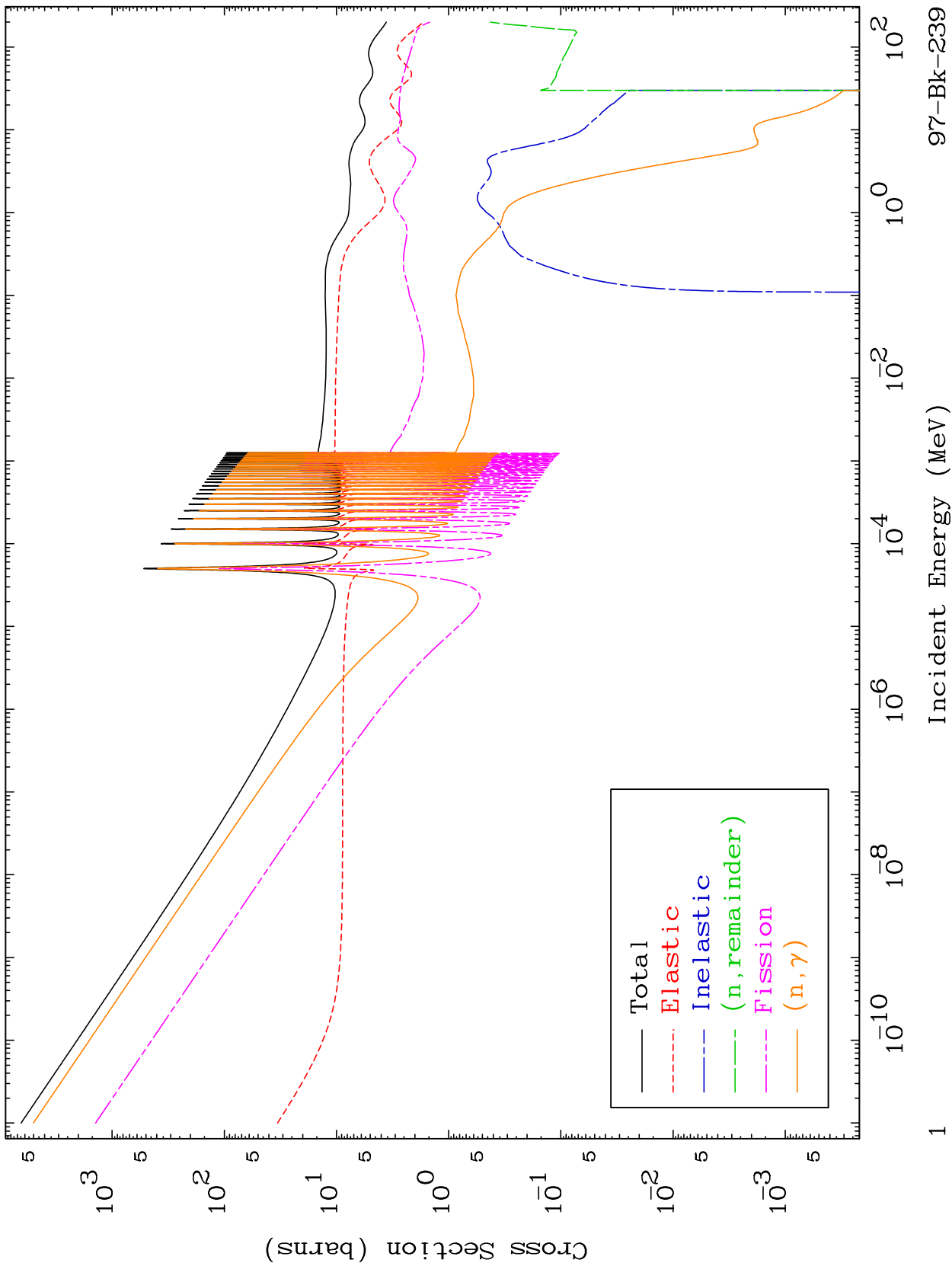
Press Mouse Button to Start

MAT 9722

Major

293 Kelvin Cross Sections

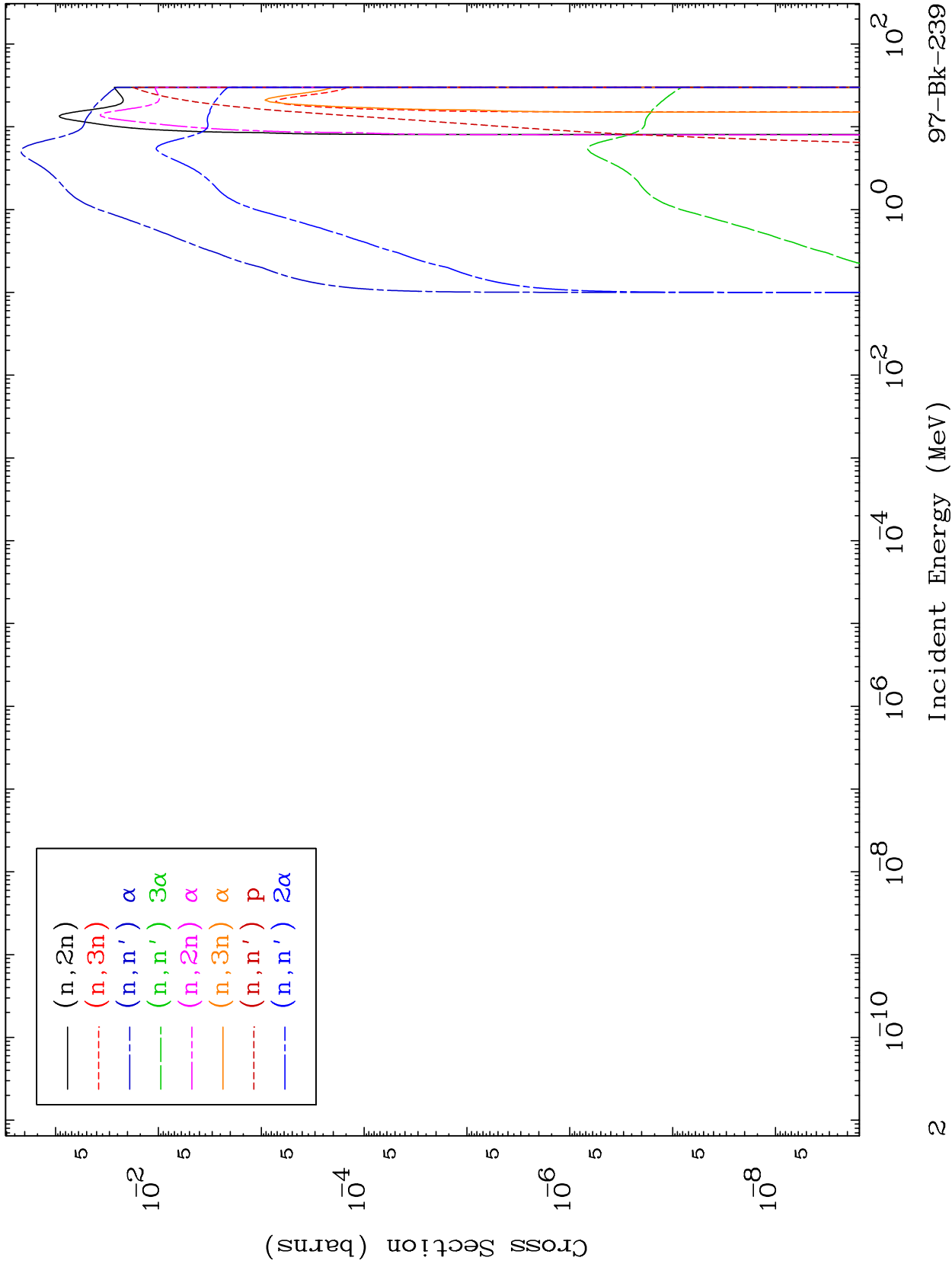
97-Bk-239



MAT 9722

Neutron Production
293 Kelvin Cross Sections

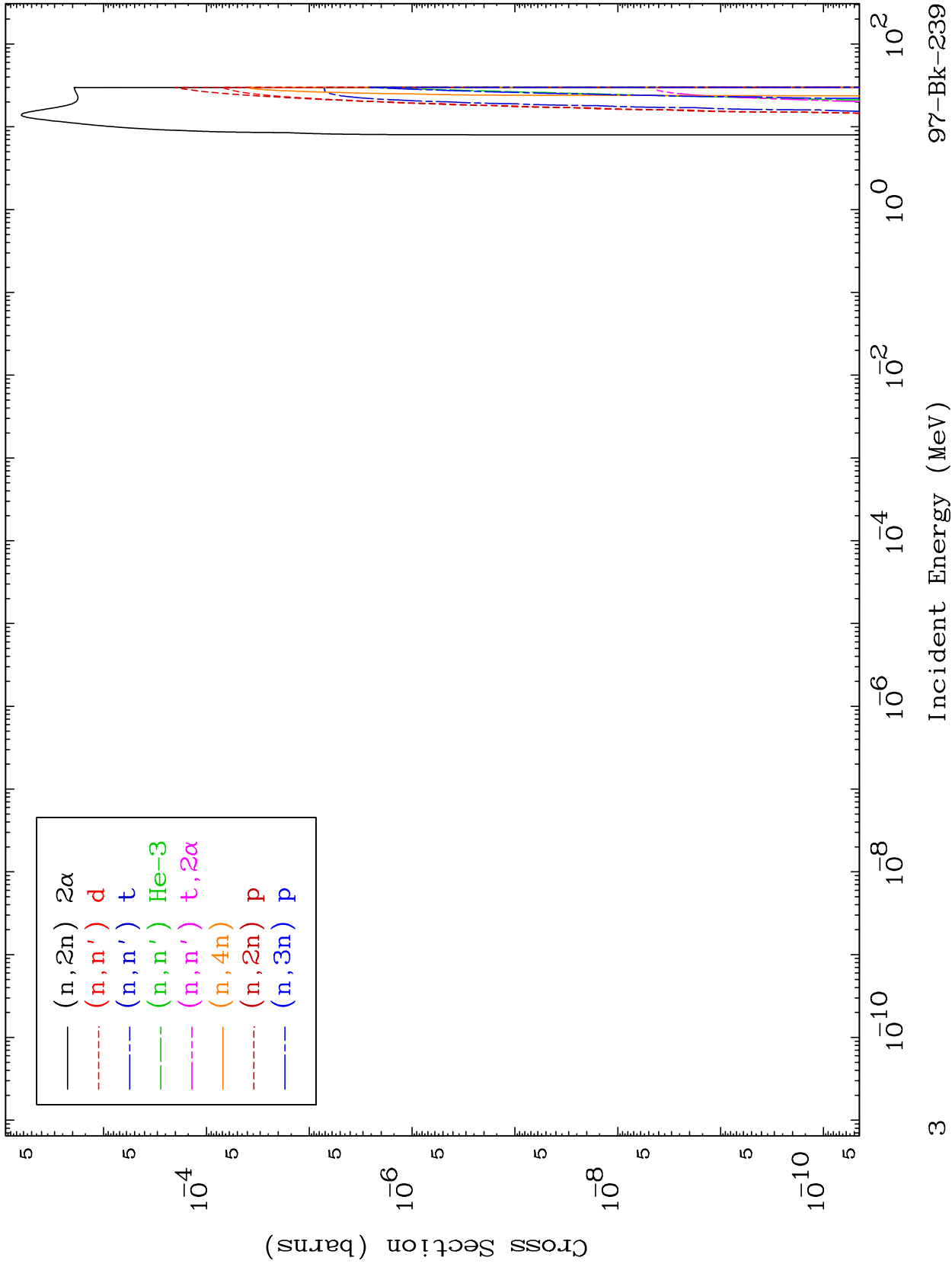
97-Bk-239



MAT 9722

Neutron Production
293 Kelvin Cross Sections

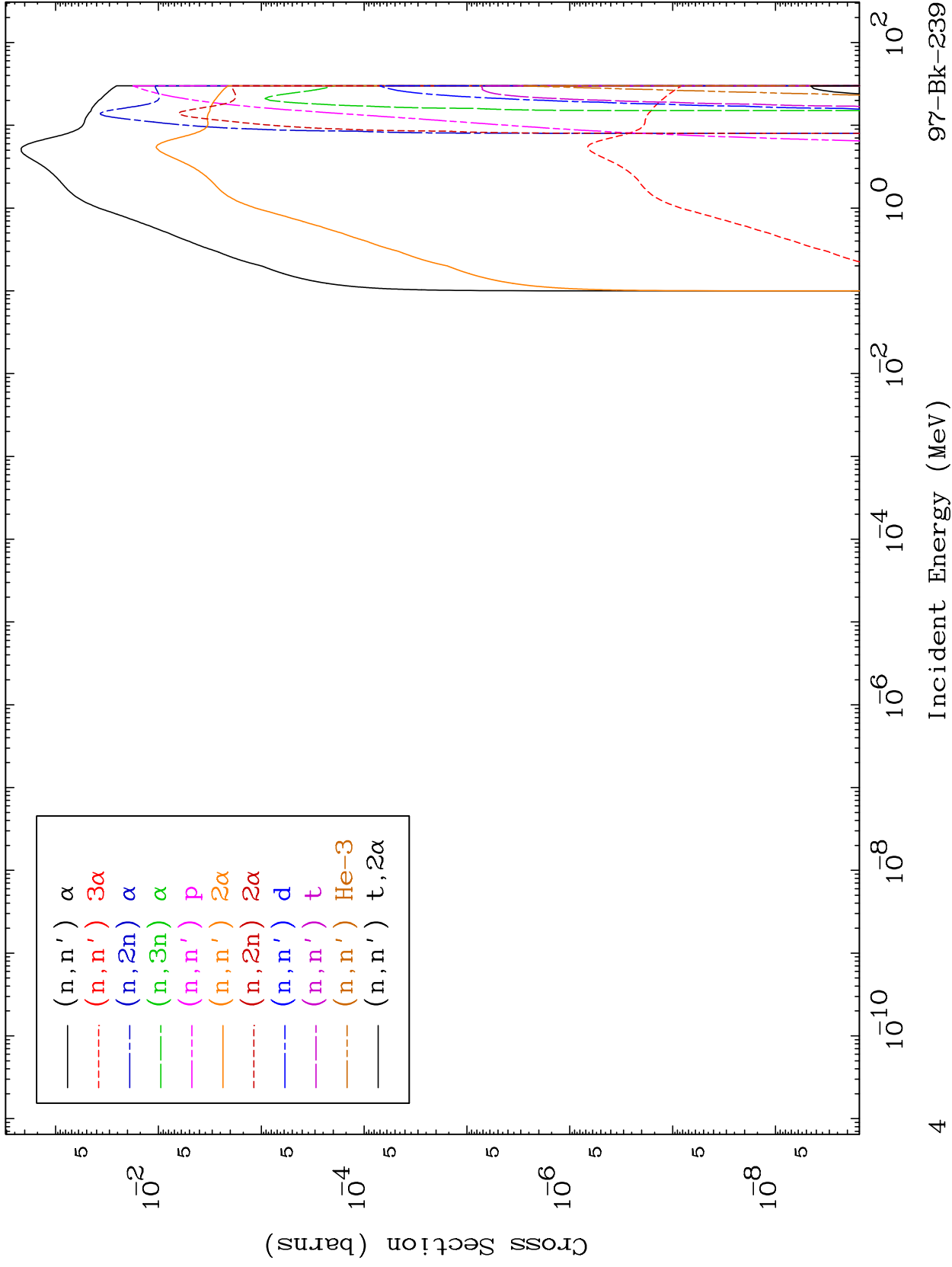
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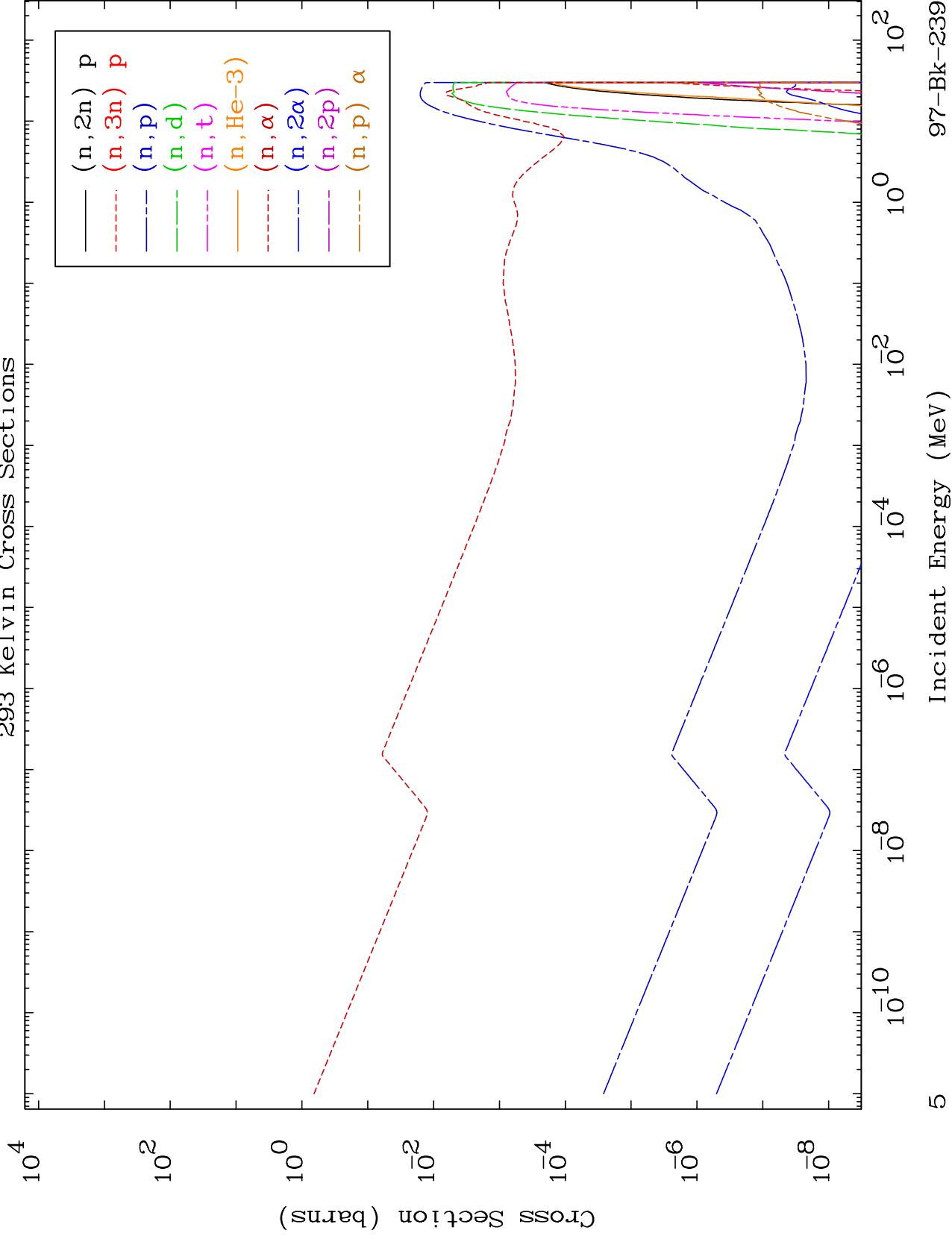


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Charged Particle
293 Kelvin Cross Sections

97-Bk-239

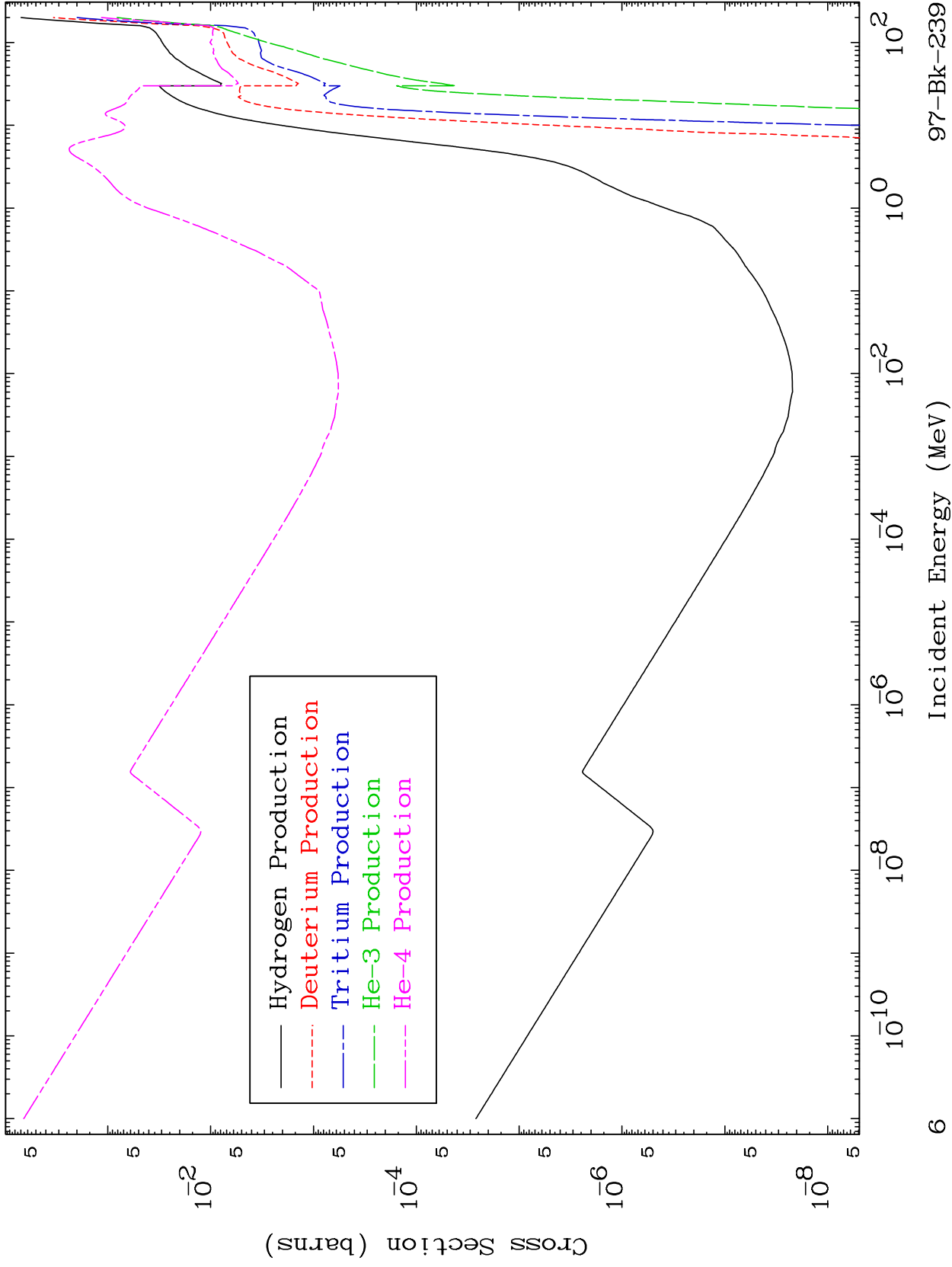




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Particle Production
293 Kelvin Cross Sections

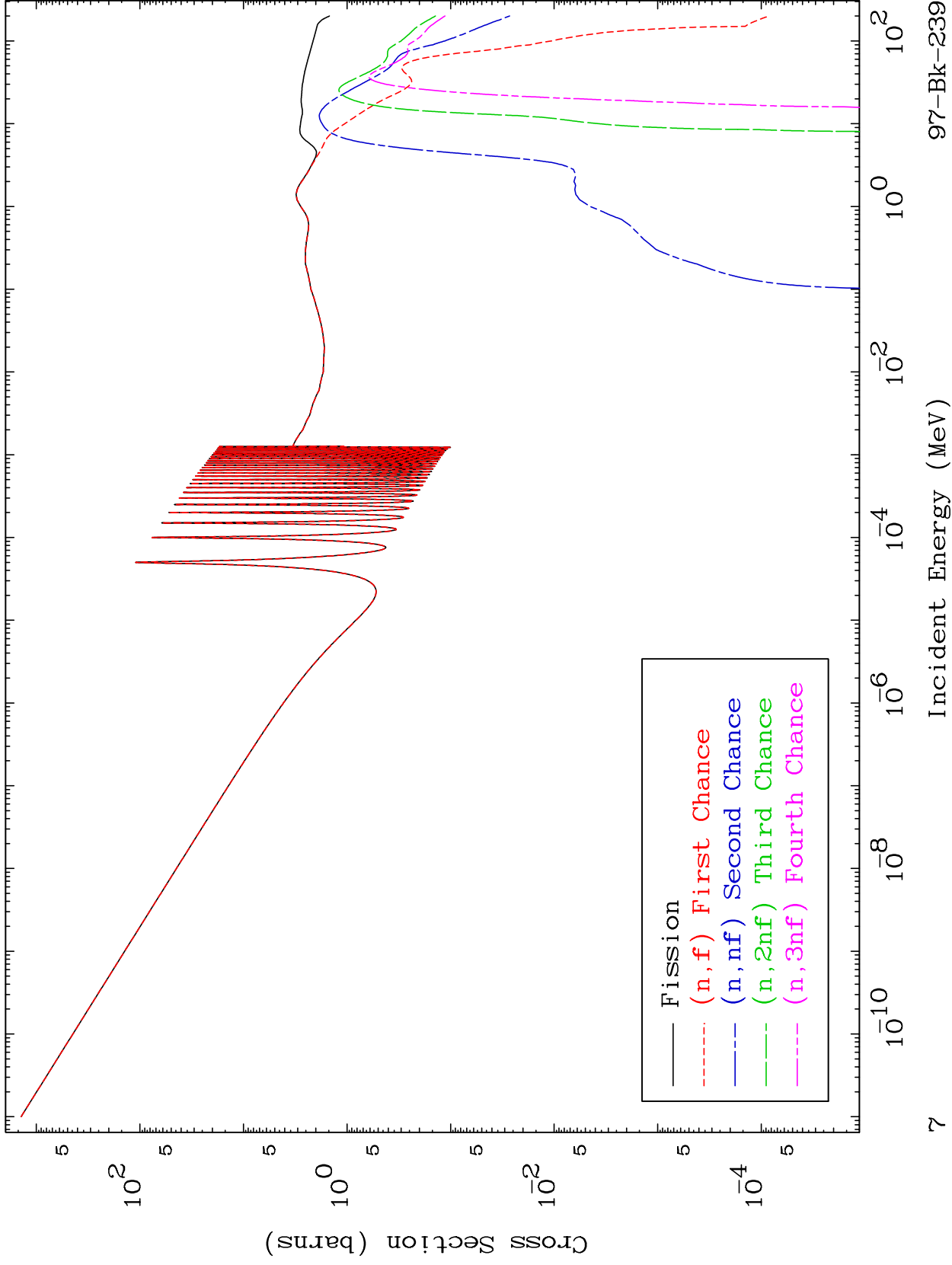
97-Bk-239



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Fission
293 Kelvin Cross Sections

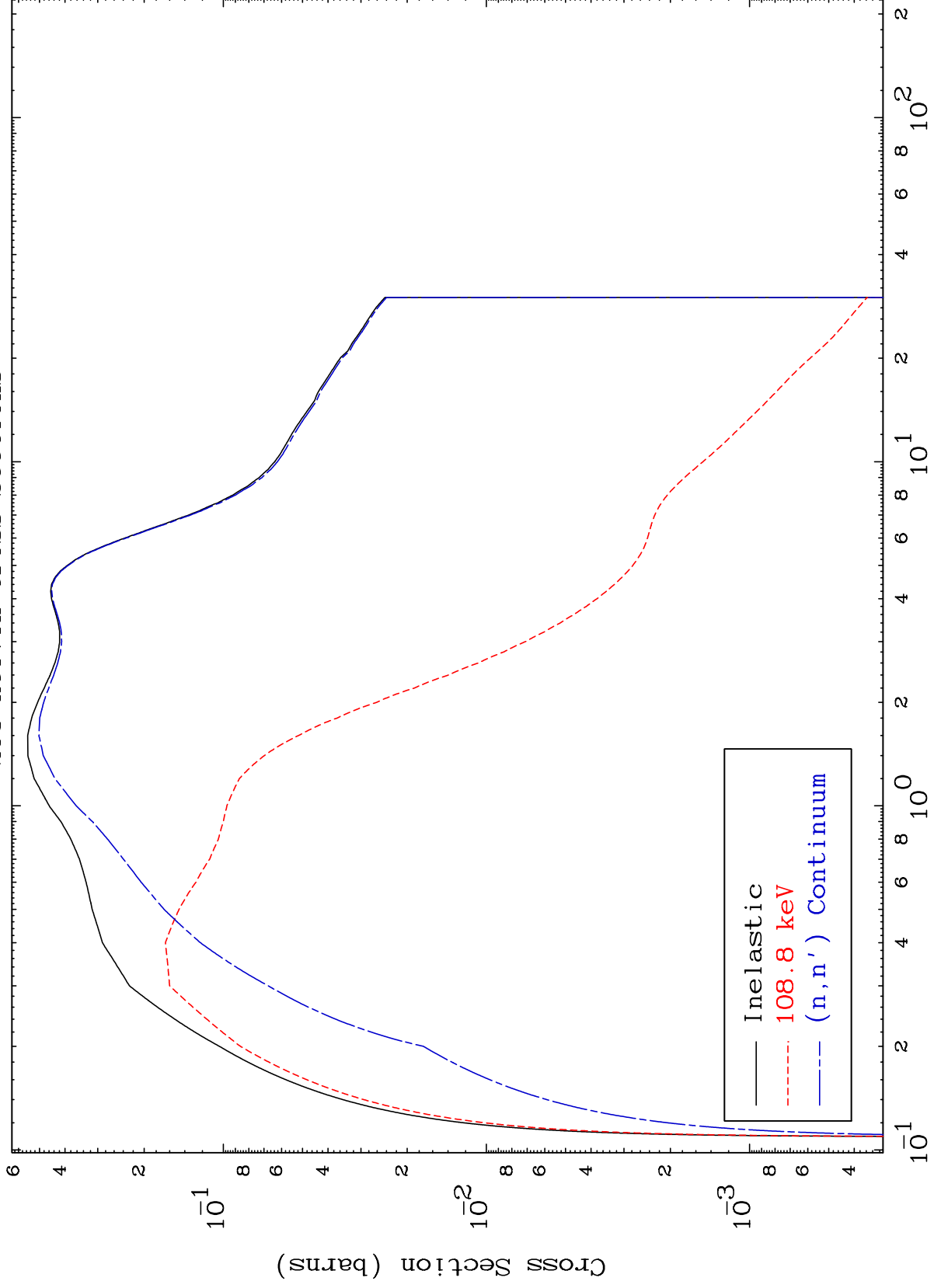
97-Bk-239



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293 Kelvin Cross Sections
(n,n') Level

97-Bk-239



97-Bk-239

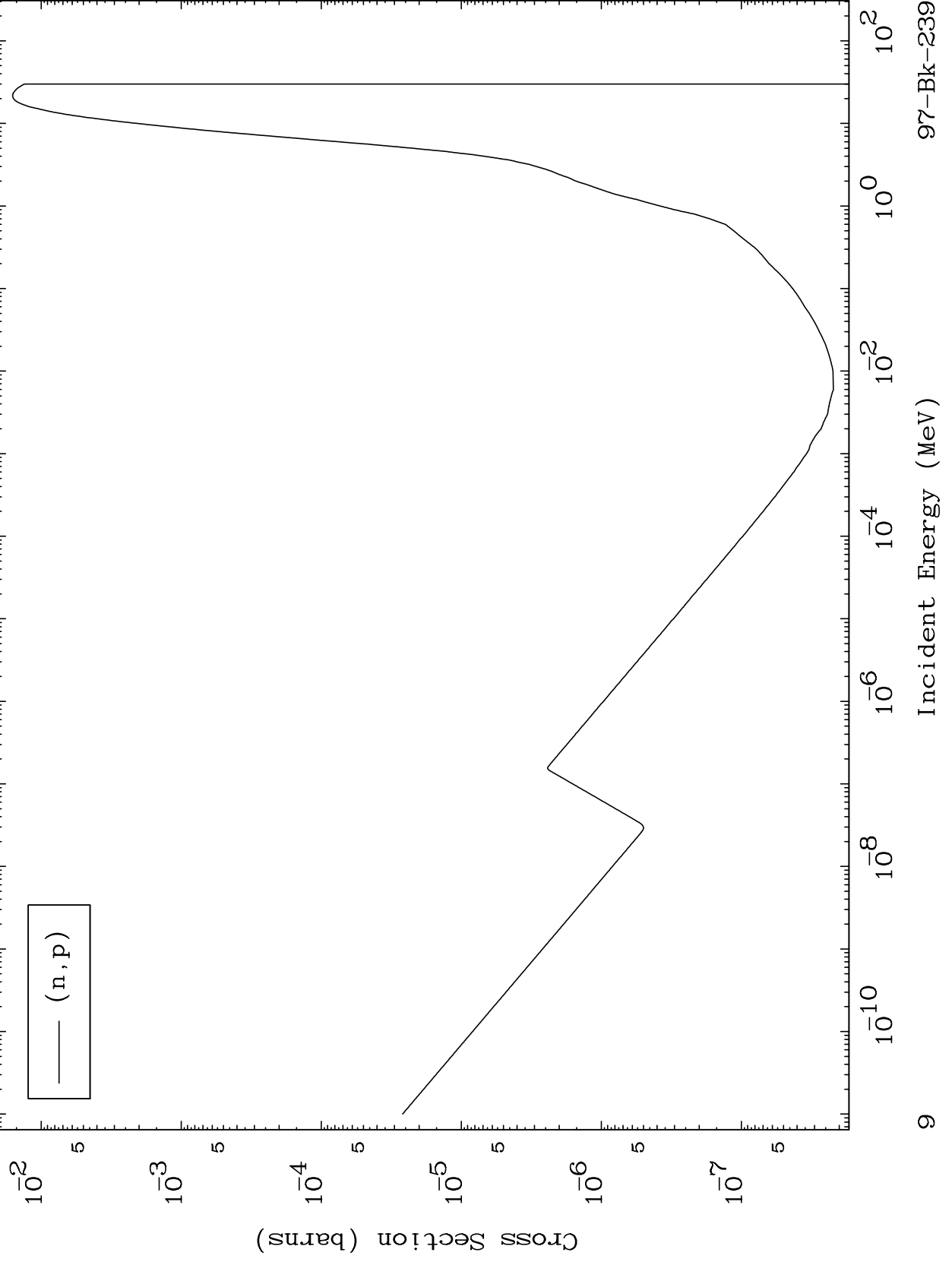
Incident Energy (MeV)

8

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(n,p) Levels
293 Kelvin Cross Sections

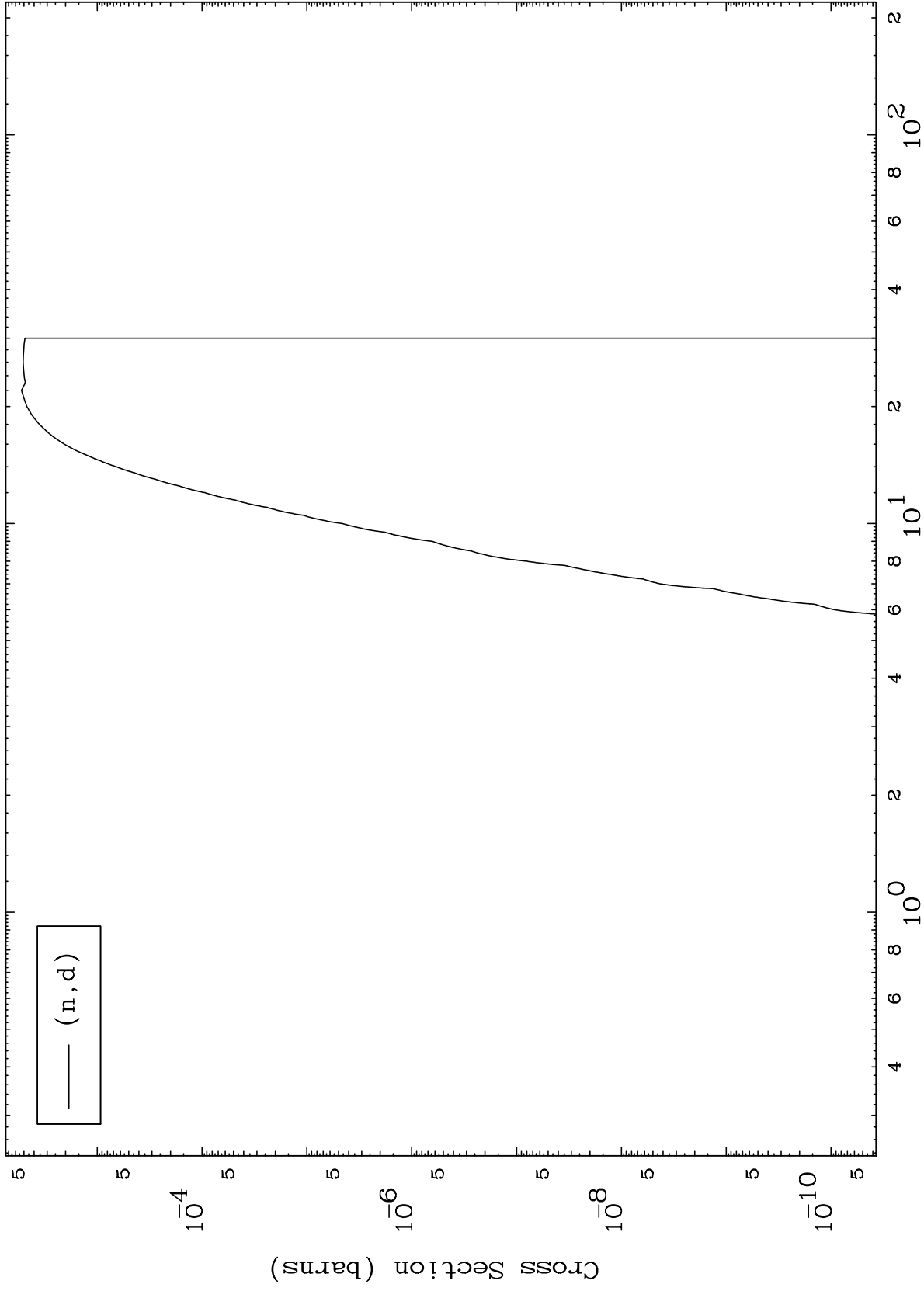
97-Bk-239



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(n,d) Levels
293 Kelvin Cross Sections

97-Bk-239



10

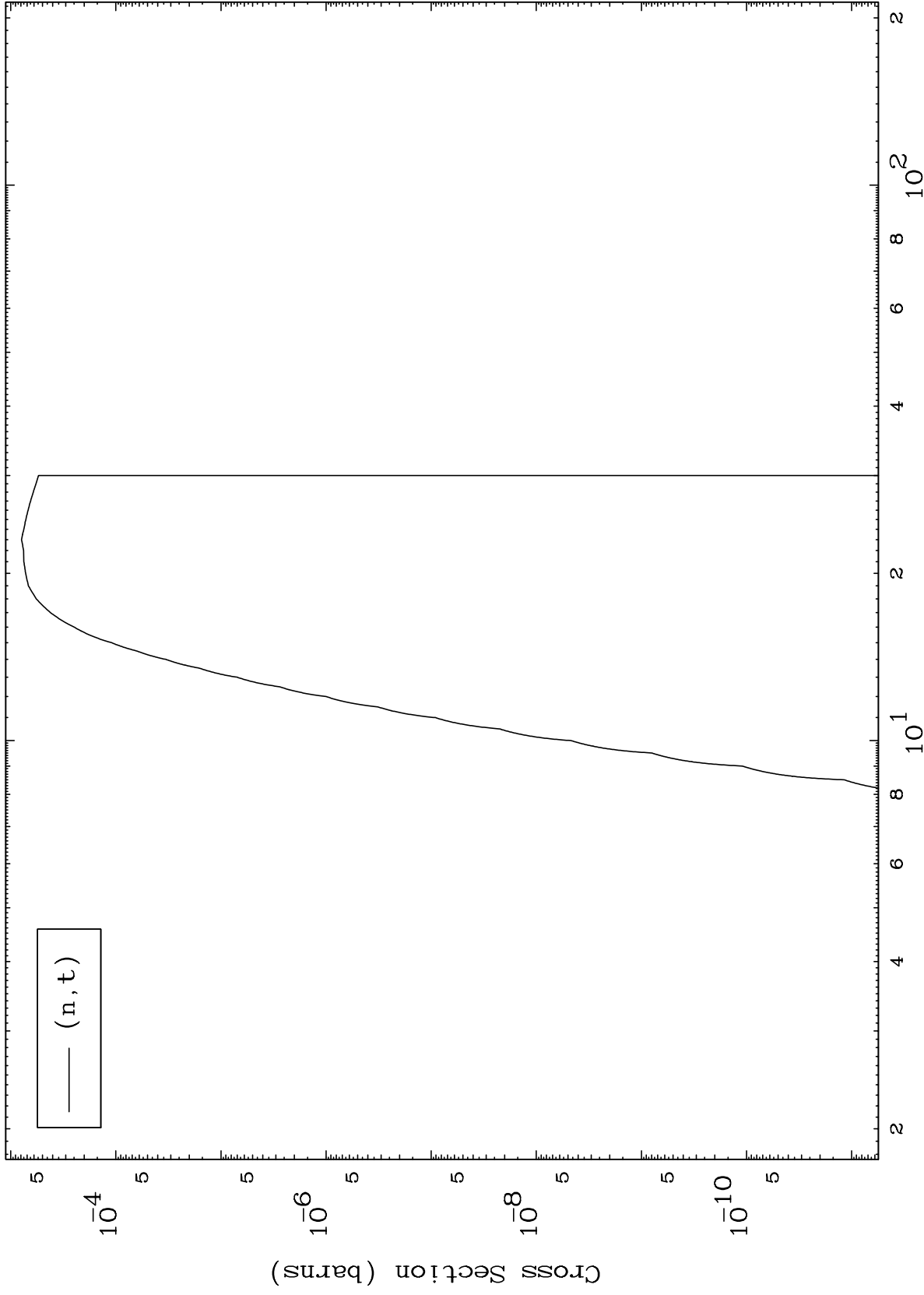
Incident Energy (MeV)

97-Bk-239

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(n,t) Levels
293 Kelvin Cross Sections

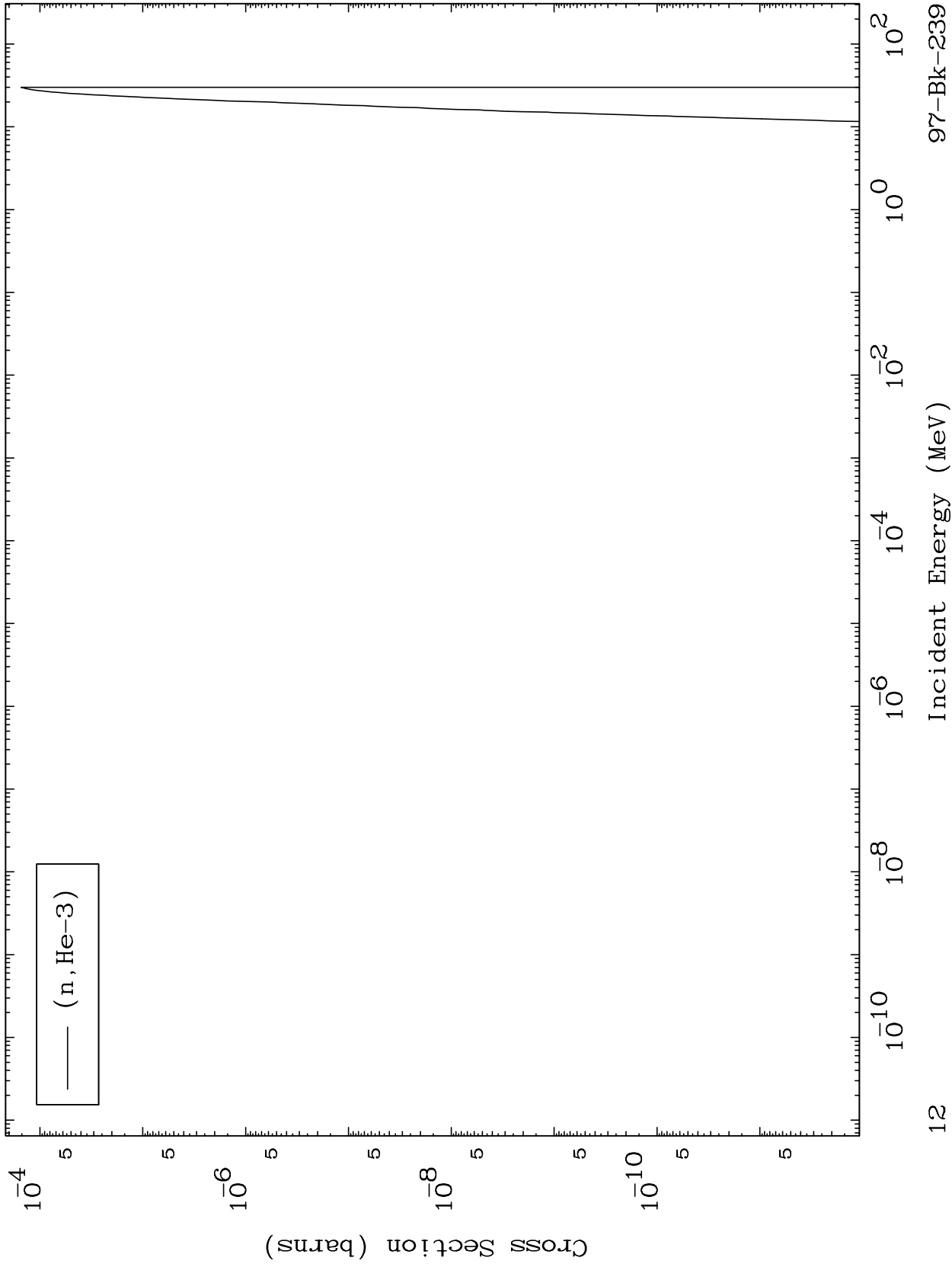
97-Bk-239



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(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-239

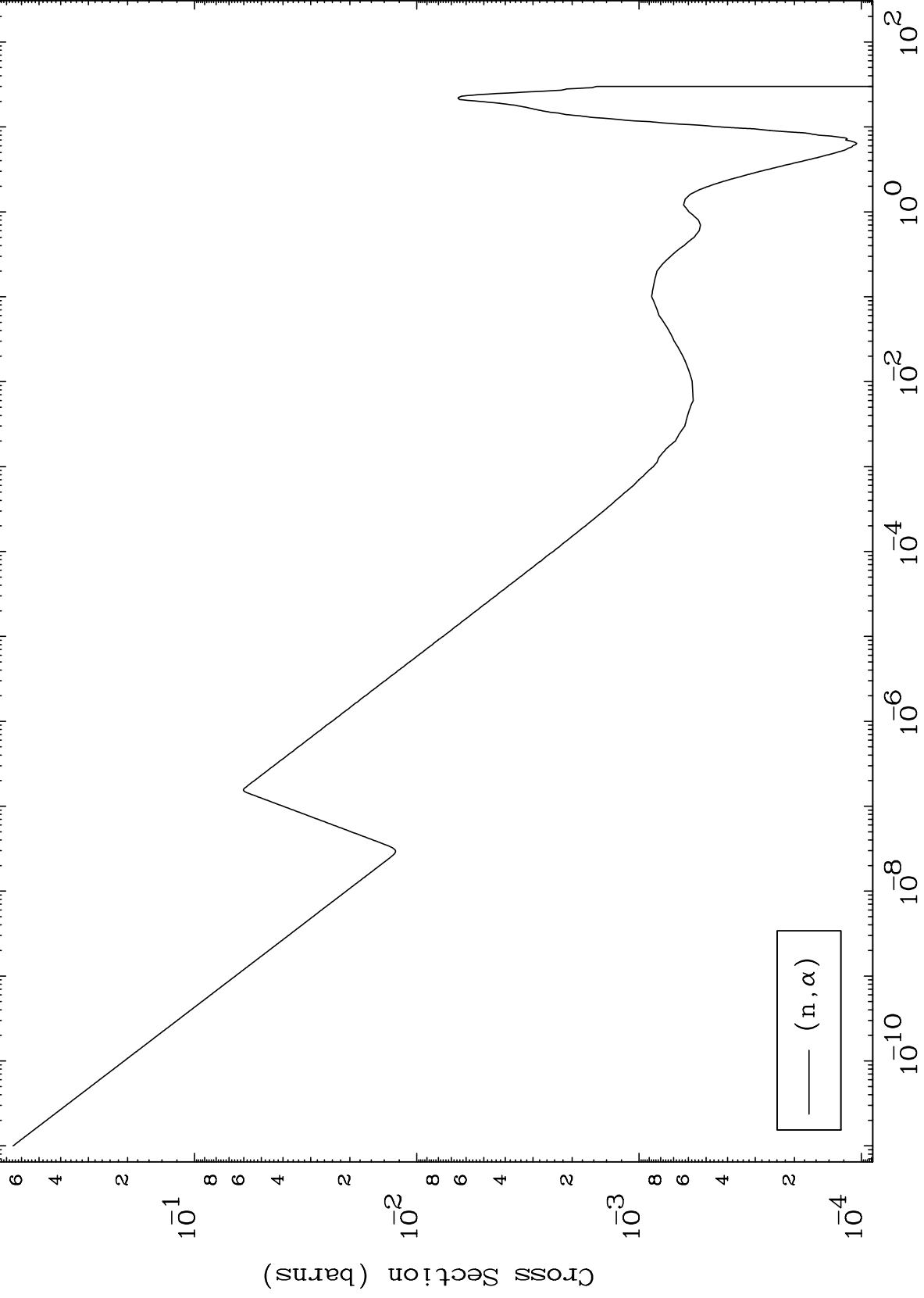


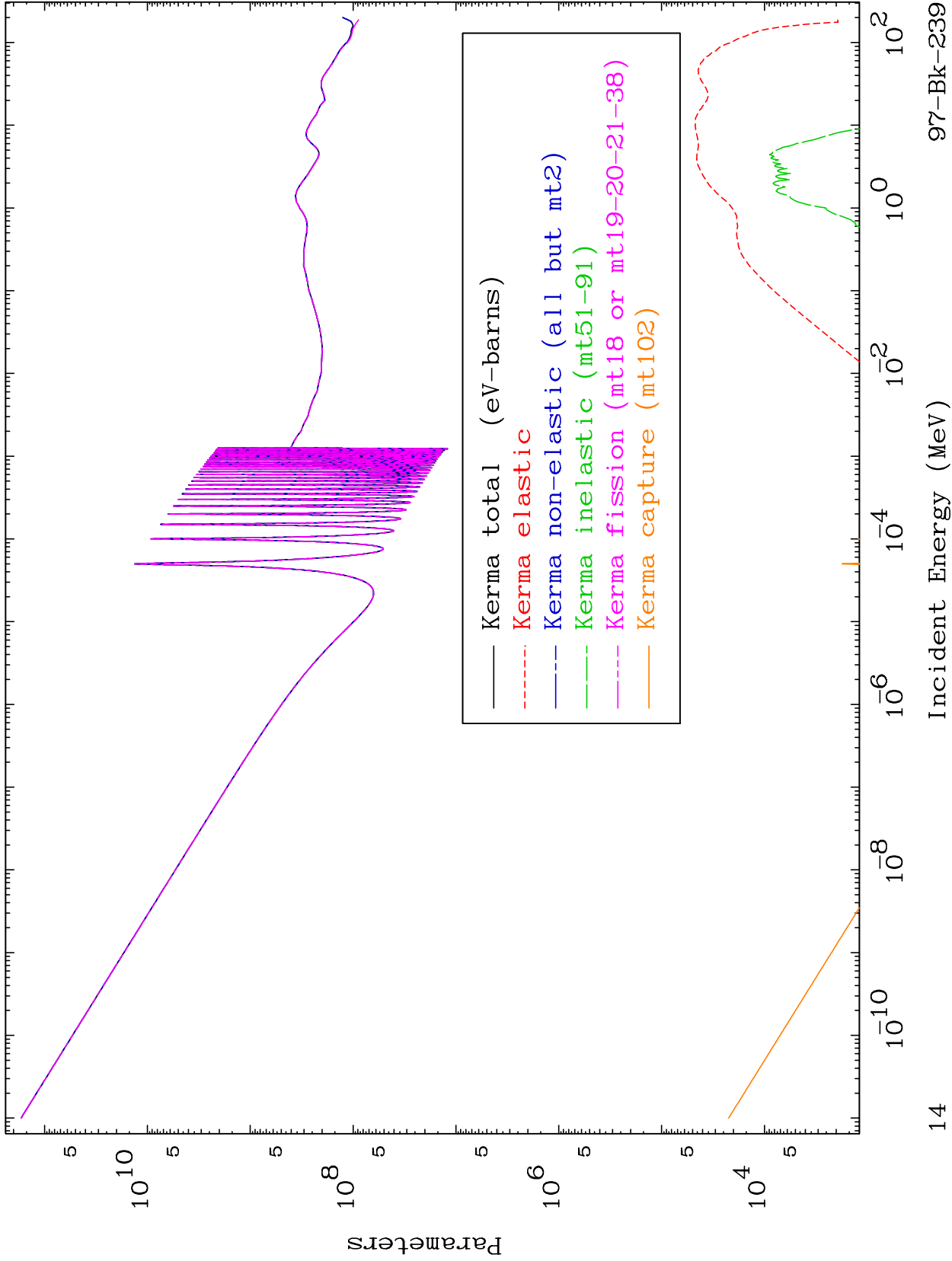
(n, He-3)

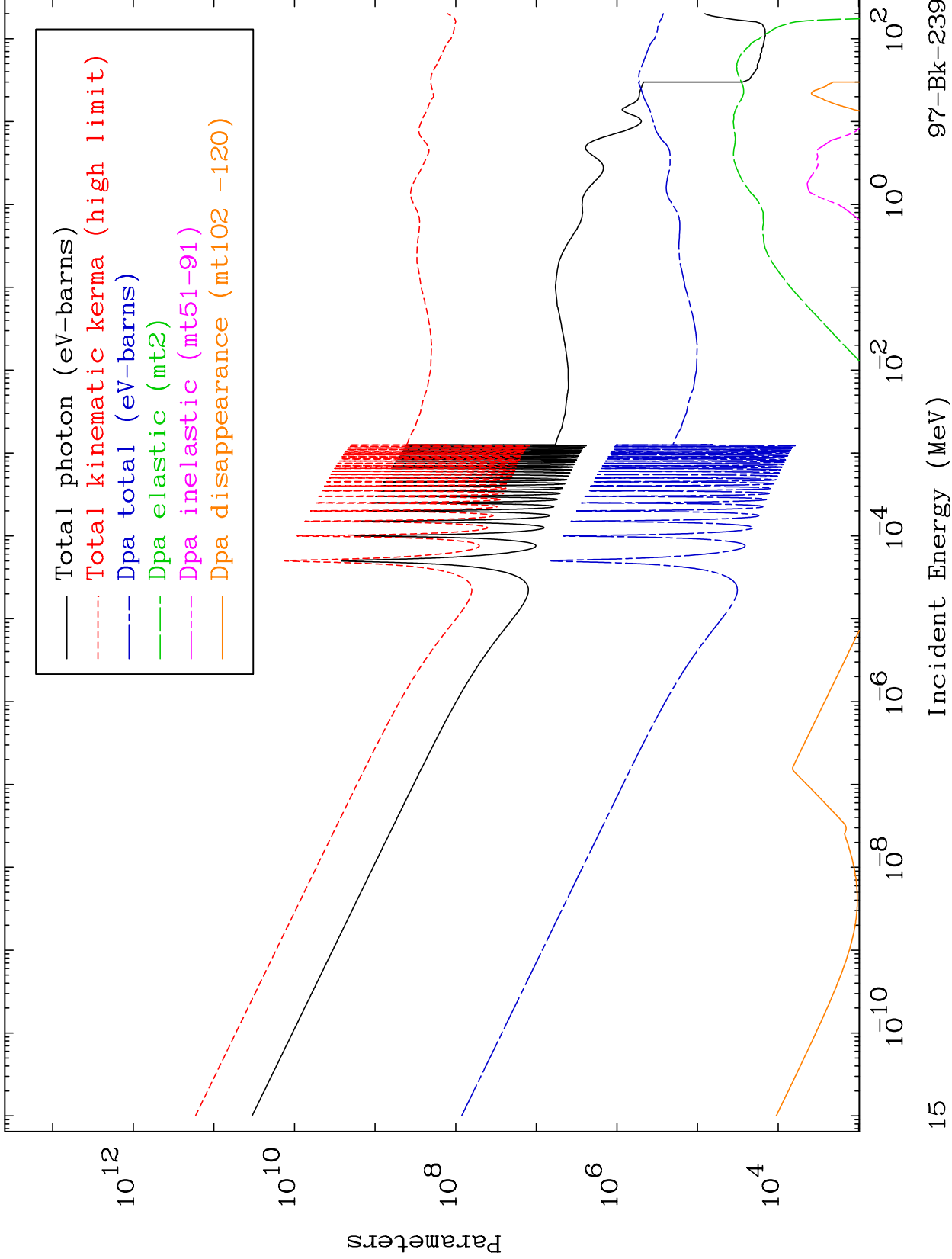
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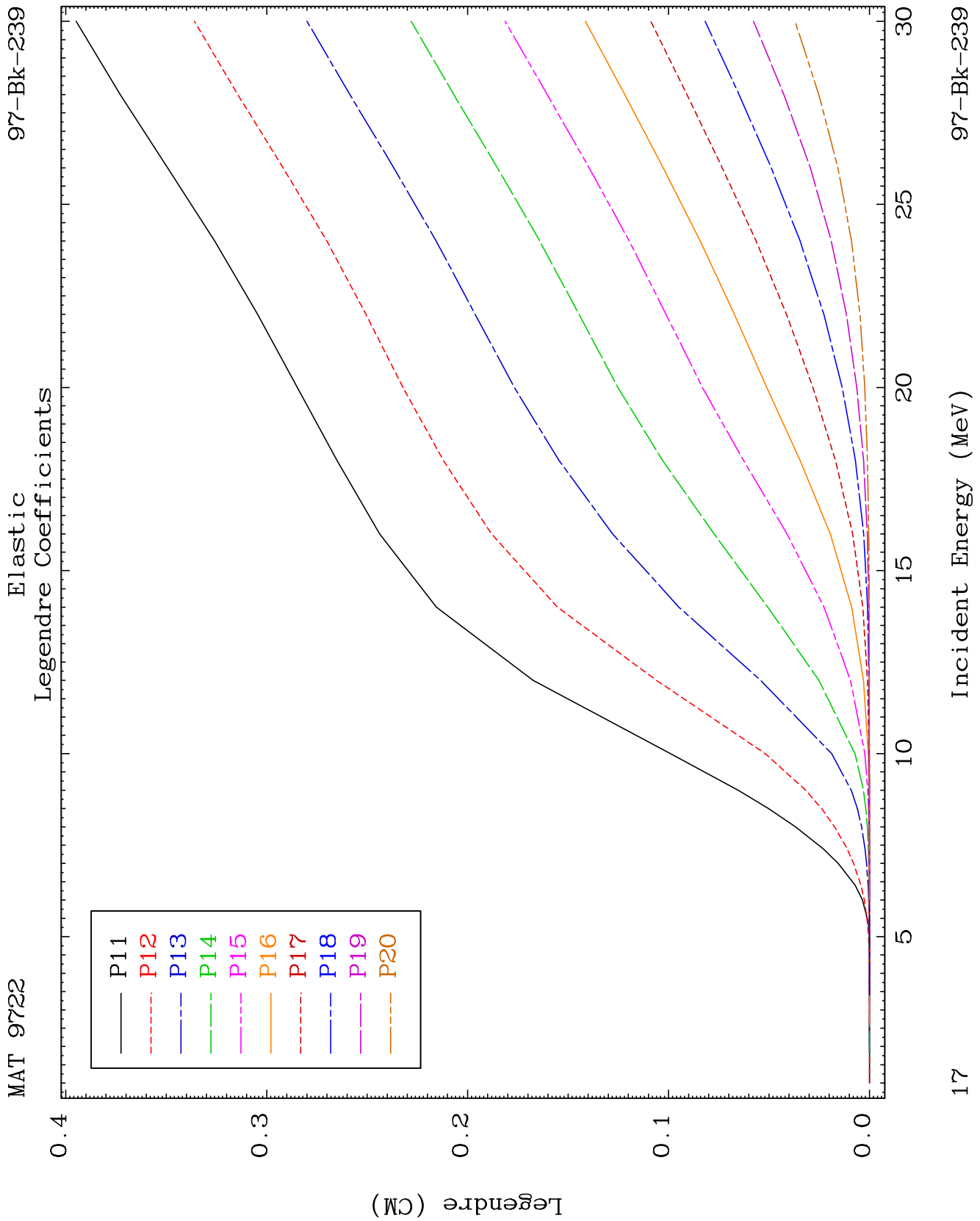
(n, α) Levels
293 Kelvin Cross Sections

97-Bk-239





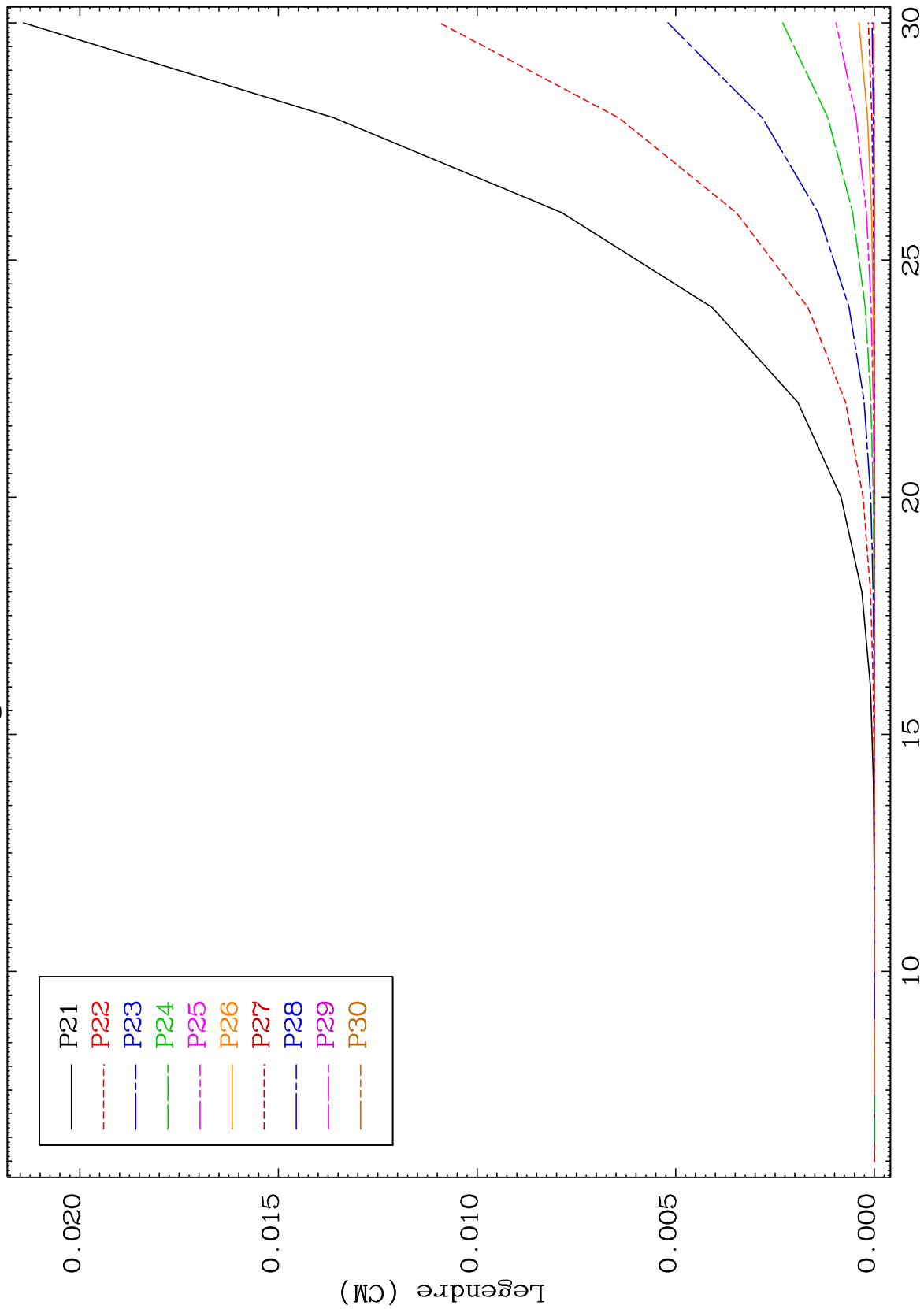




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Elastic Legendre Coefficients

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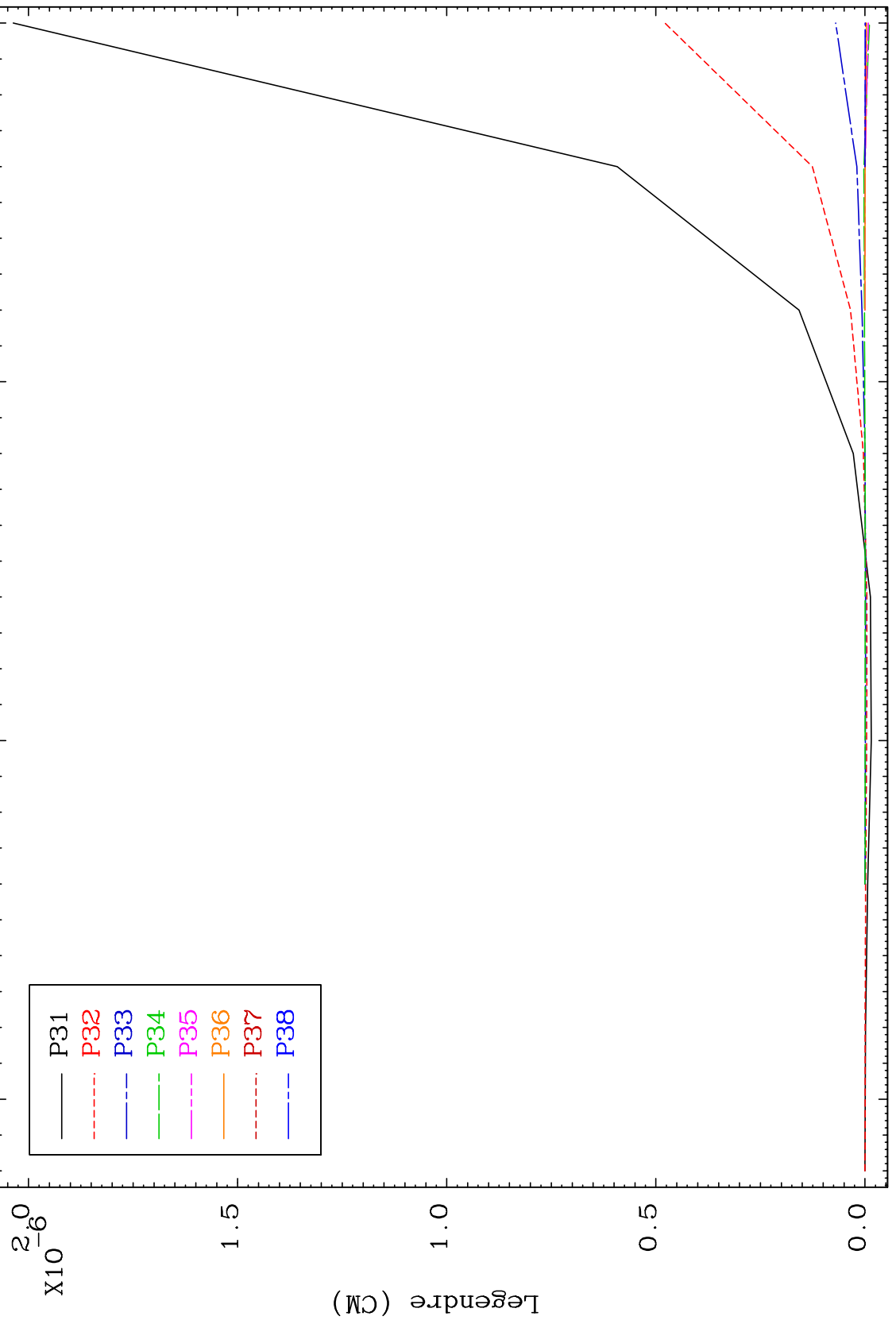
Incident Energy (MeV)

97-Bk-239

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Elastic
Legendre Coefficients

97-Bk-239

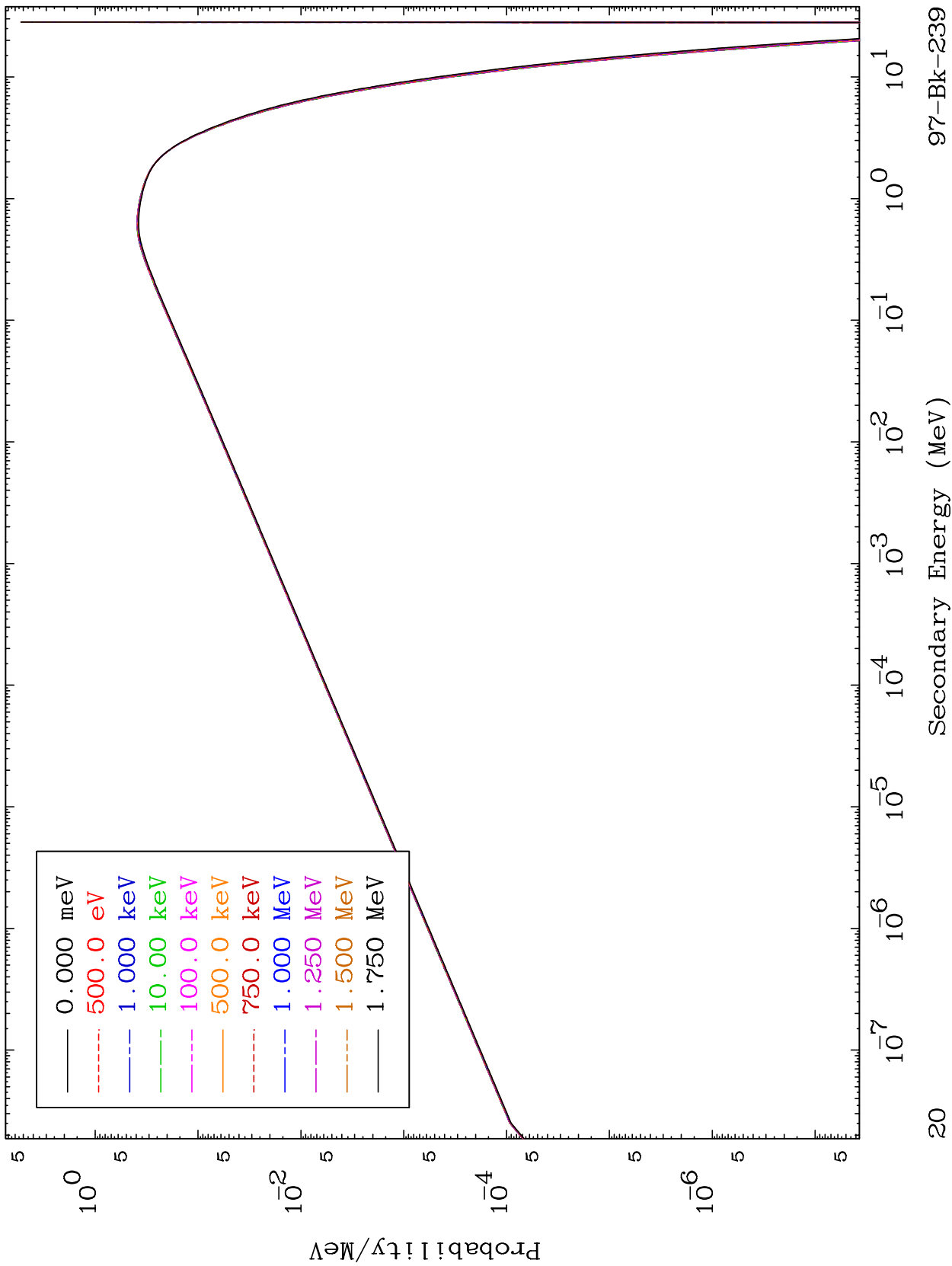


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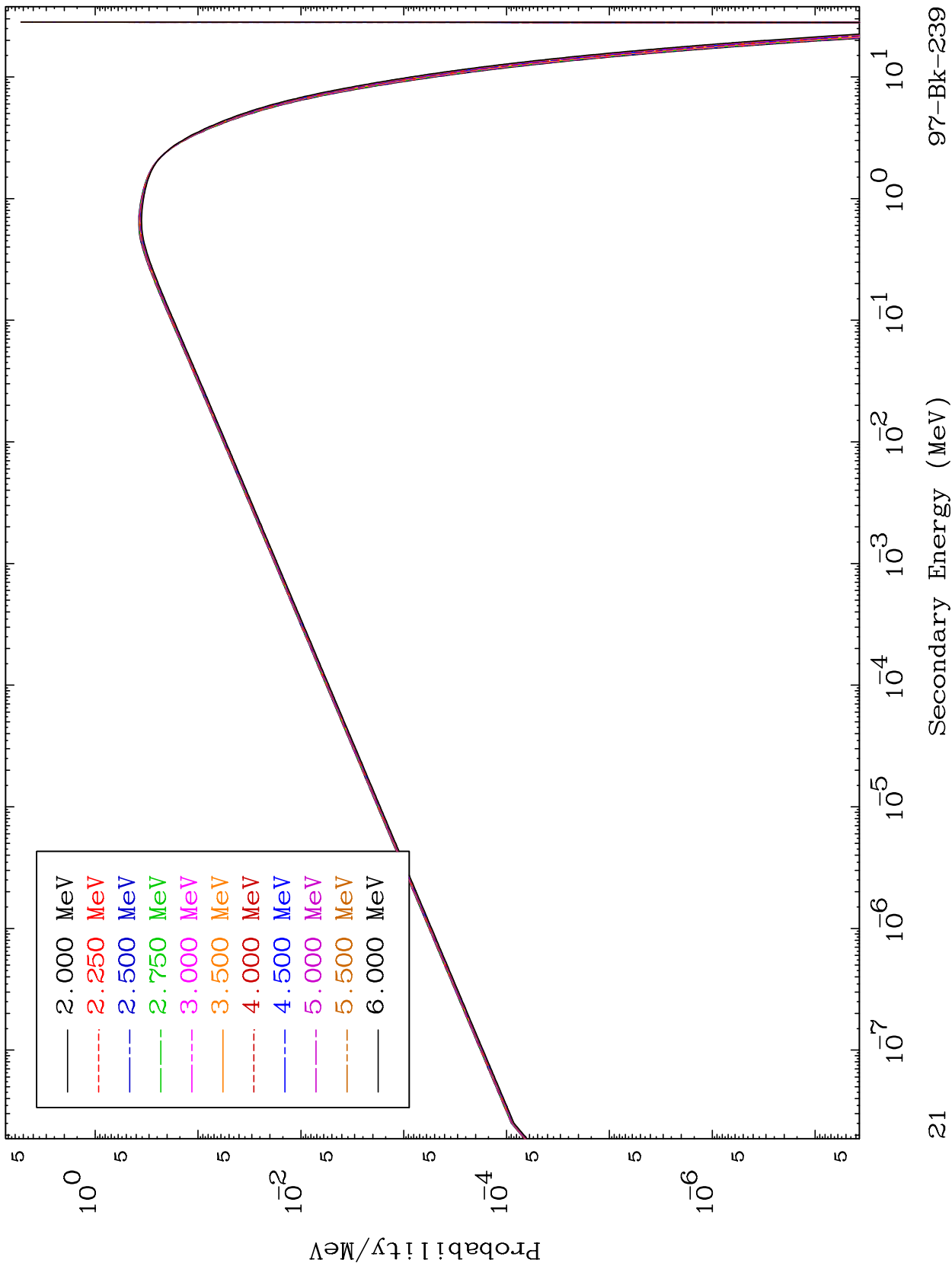
Incident Energy (MeV)

97-Bk-239

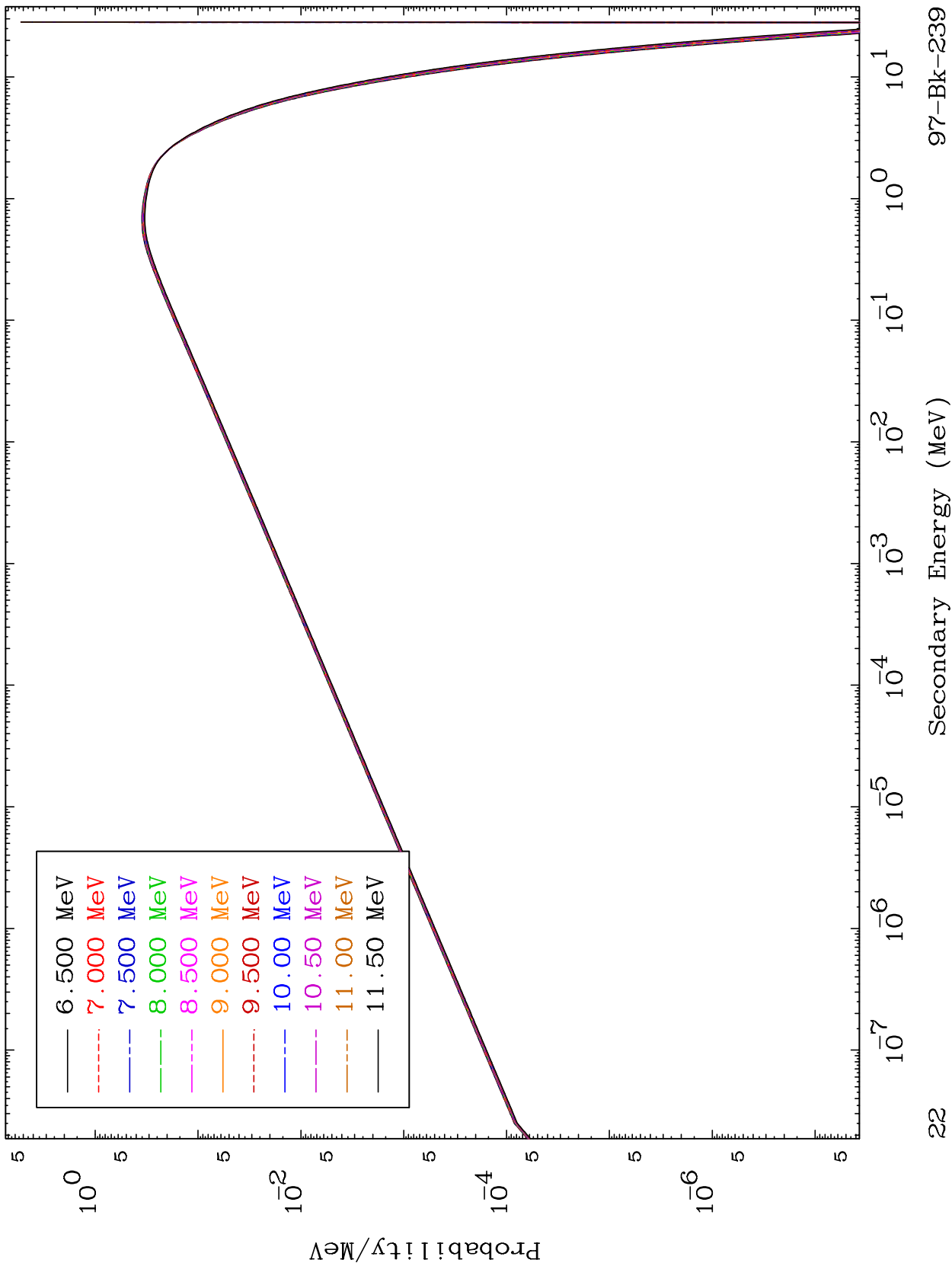
Fission Energy Distribution



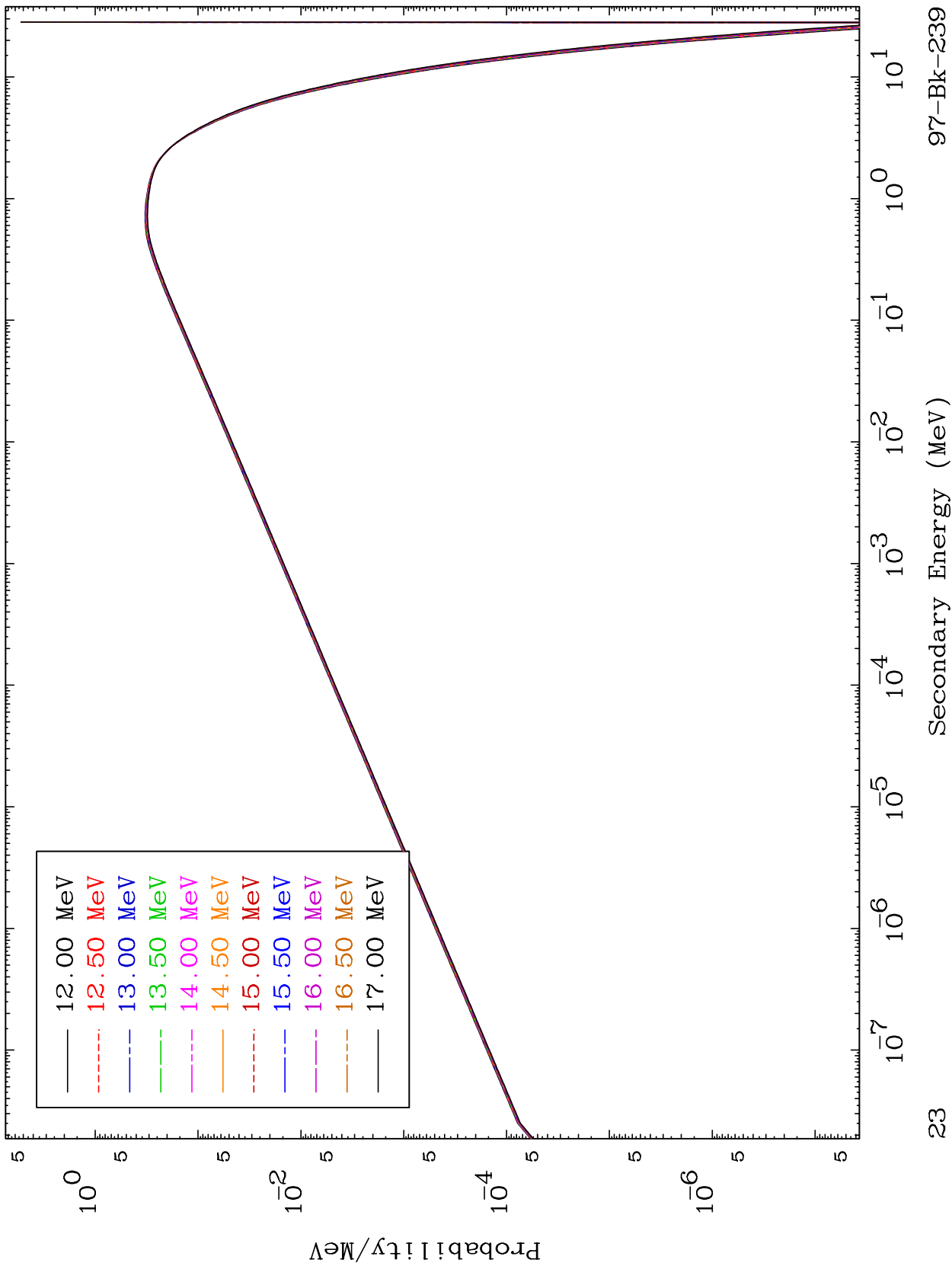
Fission Energy Distribution



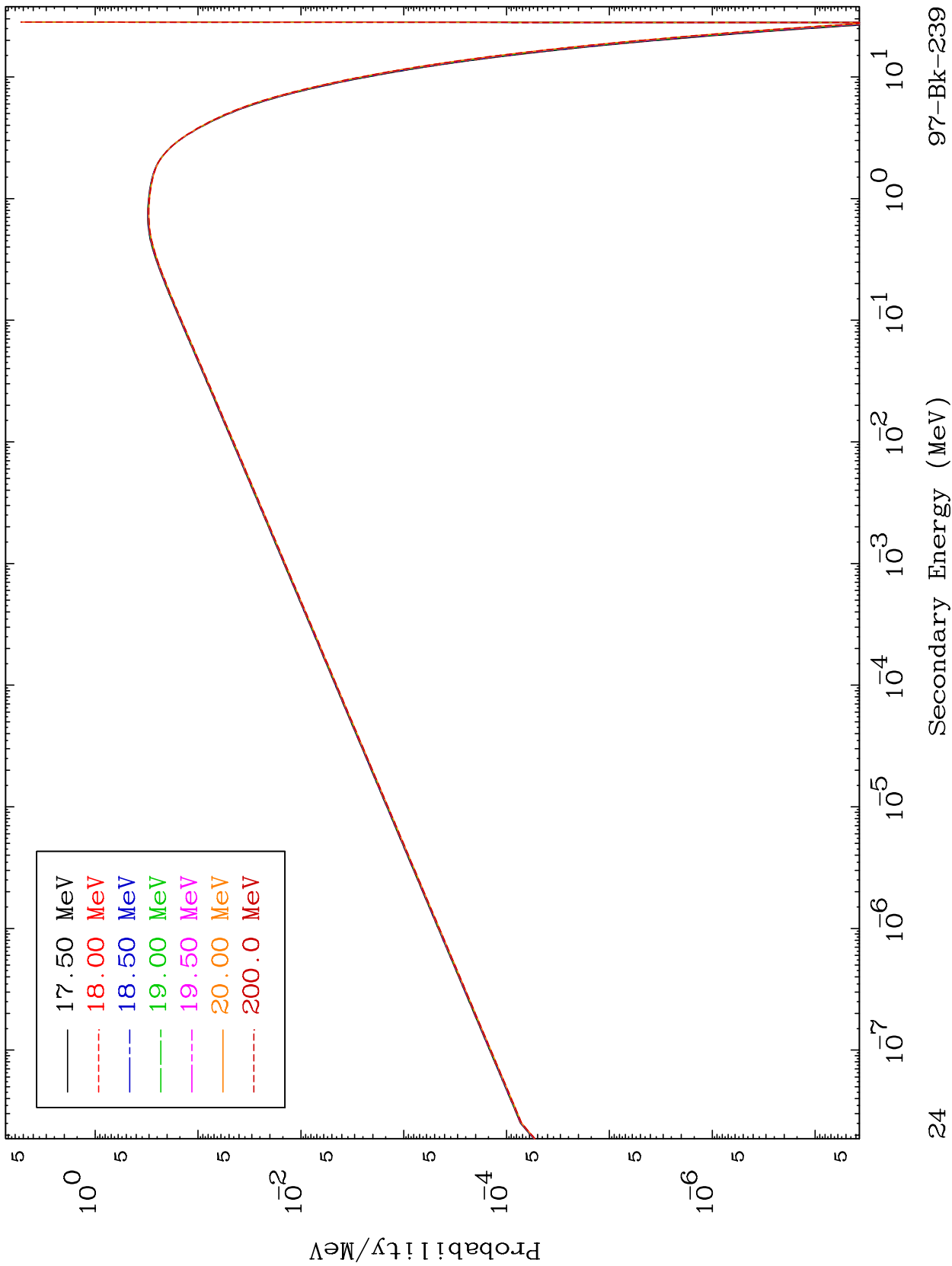
Fission Energy Distribution



Fission Energy Distribution



Fission
Energy Distribution

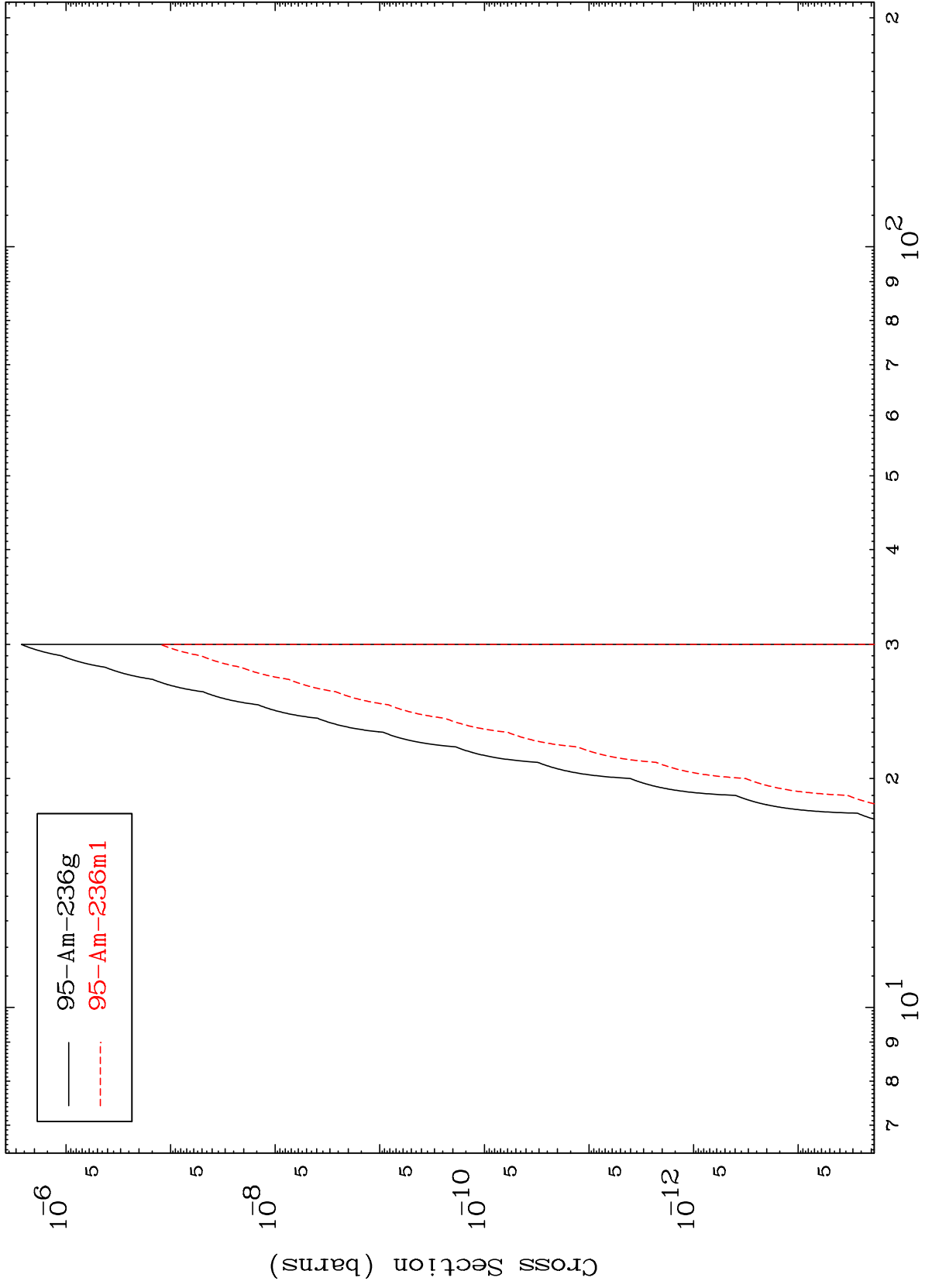


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(n,n') He-3

97-Bk-239

Radionuclide Production Cross Section



— 95-Am-236g
- - - 95-Am-236m1

25

Incident Energy (MeV)

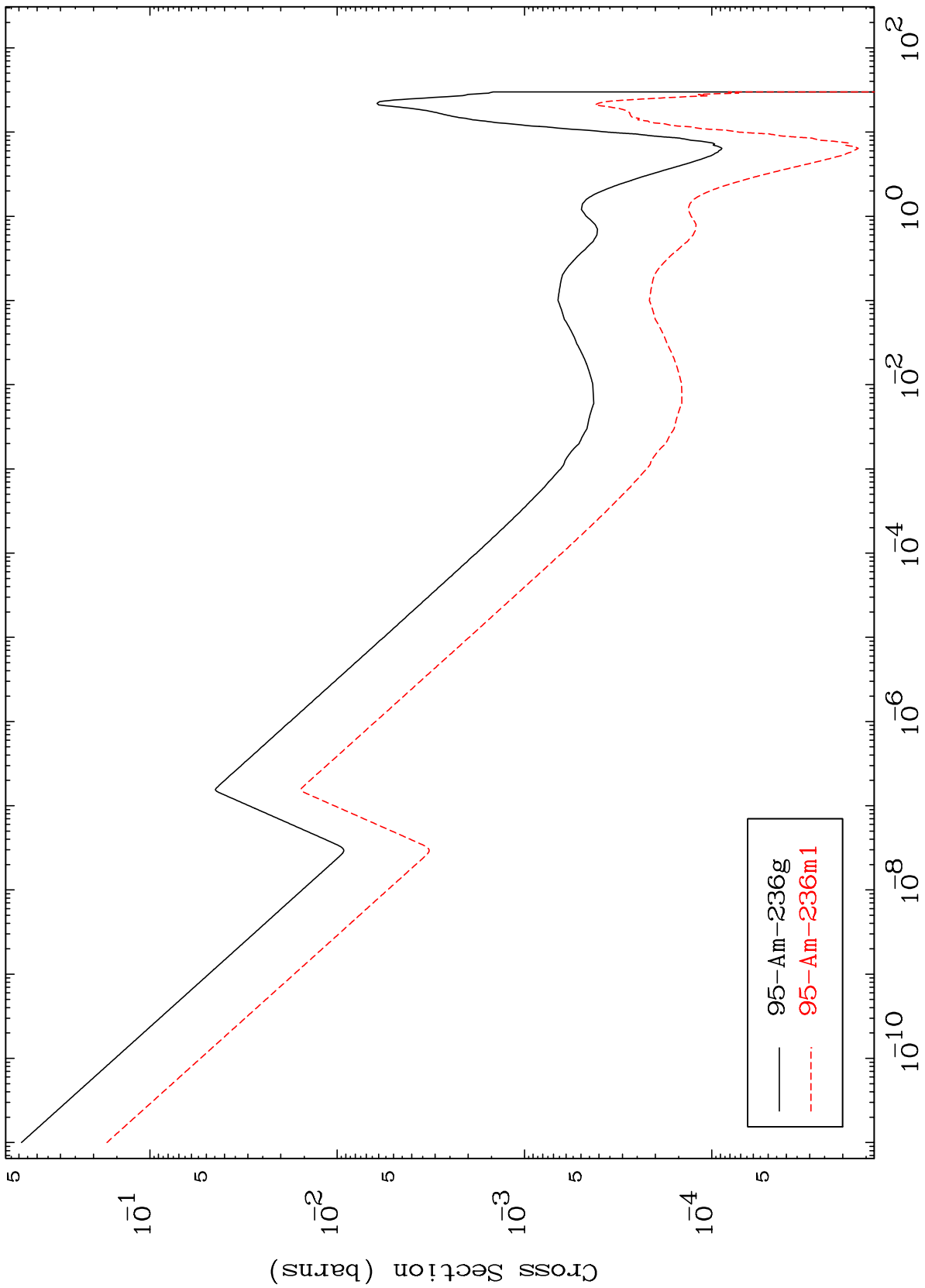
97-Bk-239

MAT 9722

97-Bk-239

Radionuclide Production Cross Section

(n, α)



— 95-Am-236g
- - - 95-Am-236m1

97-Bk-239

Incident Energy (MeV)

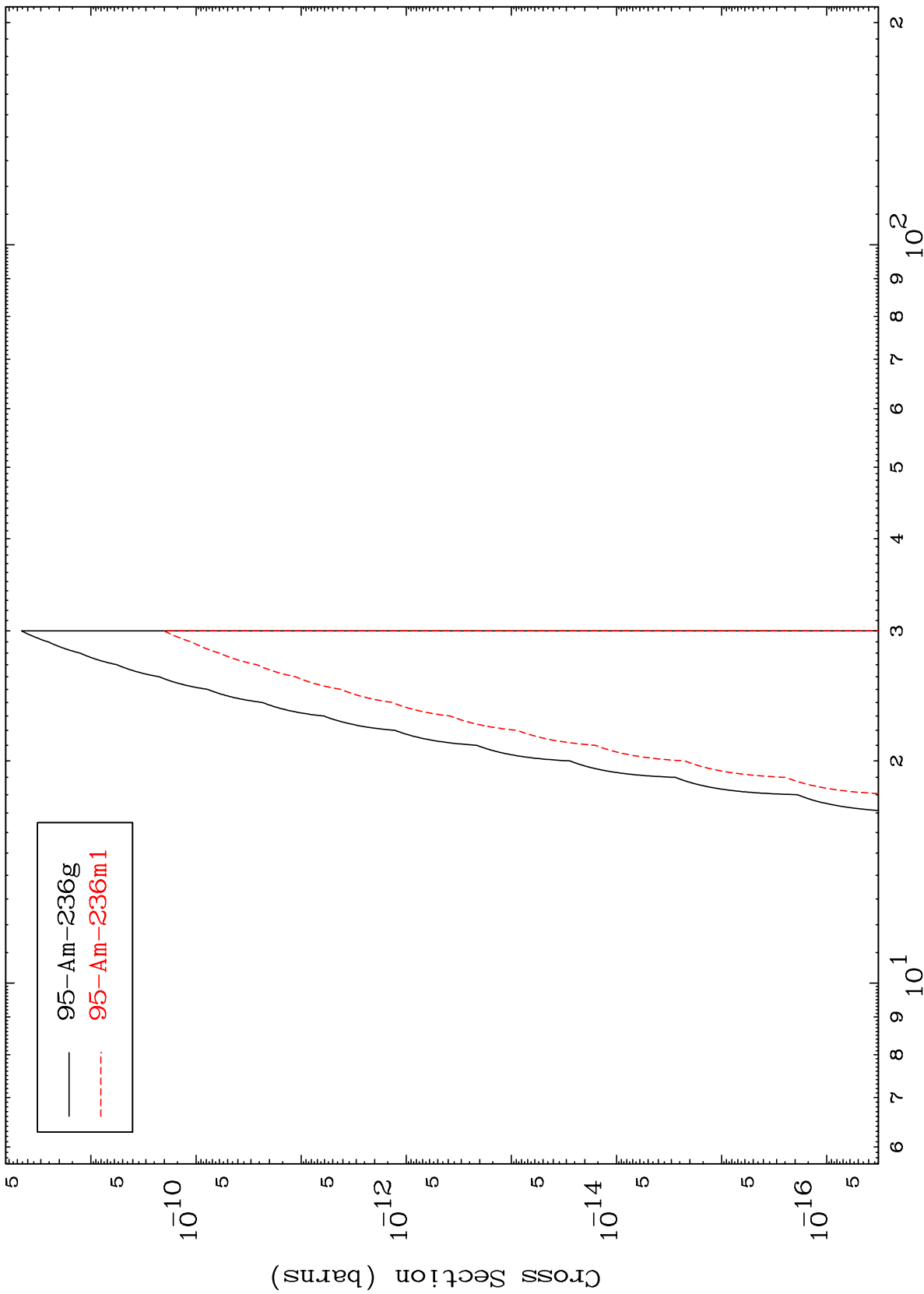
26

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(n,p) t

97-Bk-239

Radionuclide Production Cross Section



95-Am-236g
95-Am-236m1

27

Incident Energy (MeV)

97-Bk-239