

Program EVALPLOT
(Version 2015-2)

by

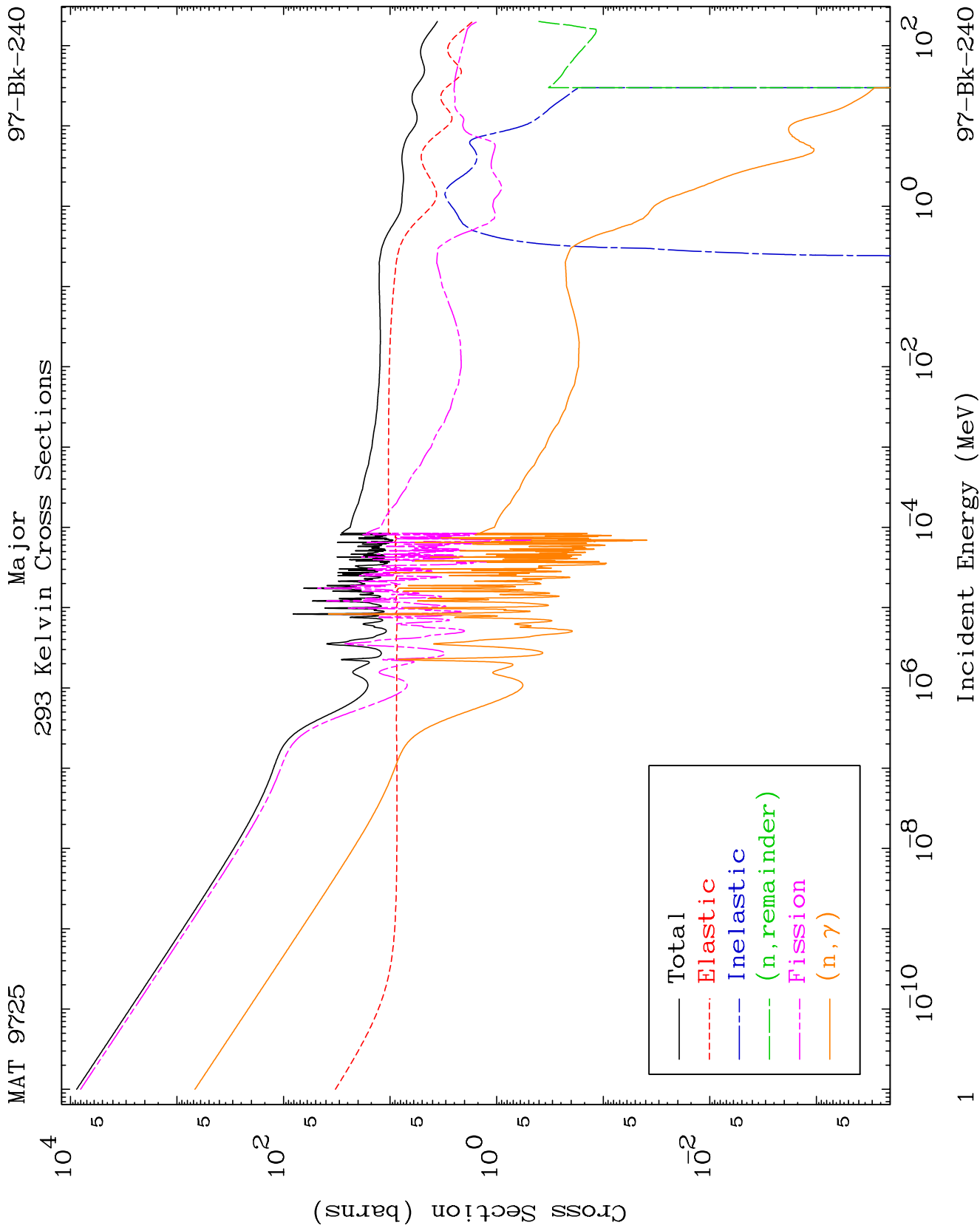
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(Present Contact Information)

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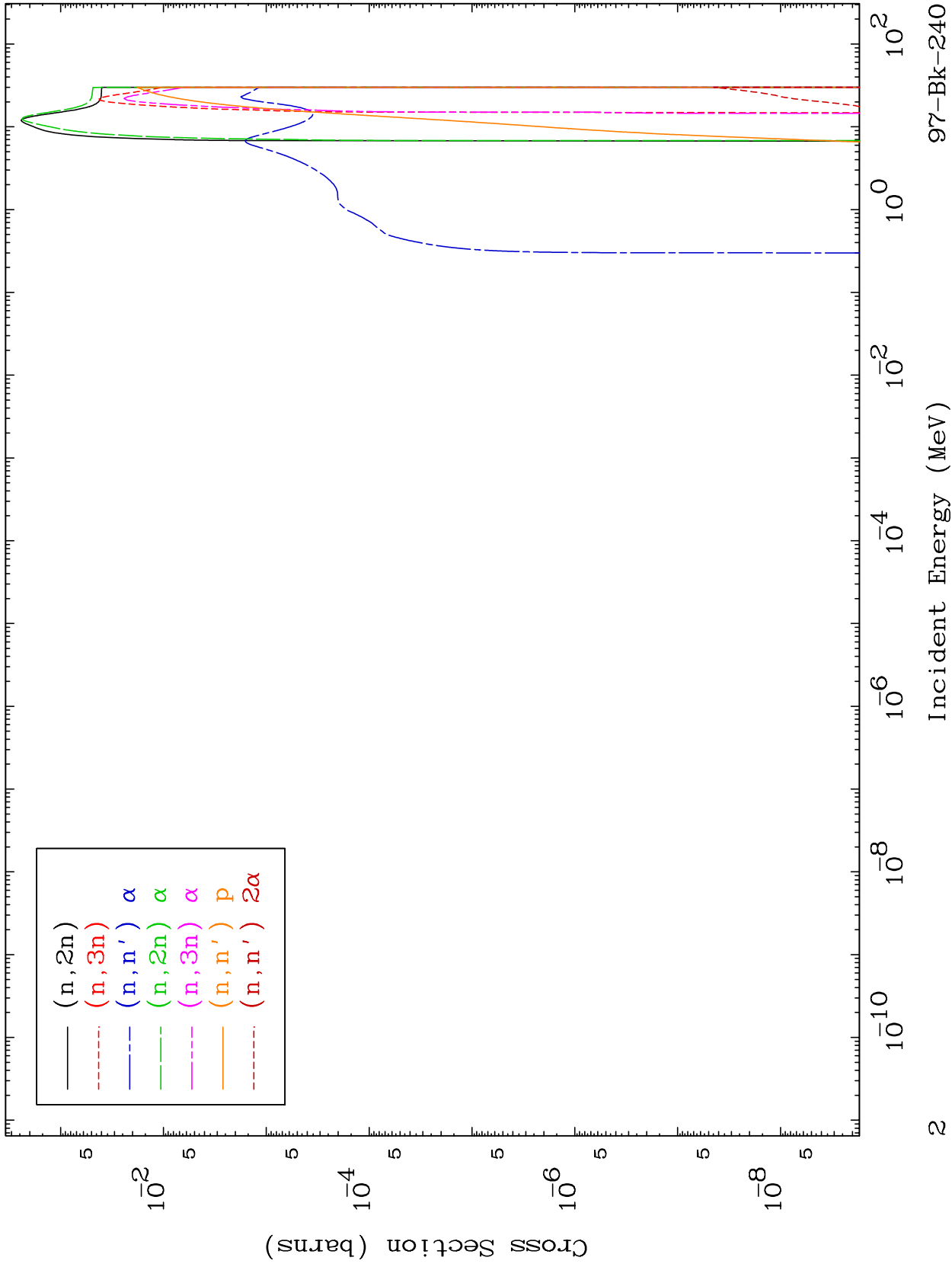
Press Mouse Button to Start



MAT 9725

Neutron Production
293 Kelvin Cross Sections

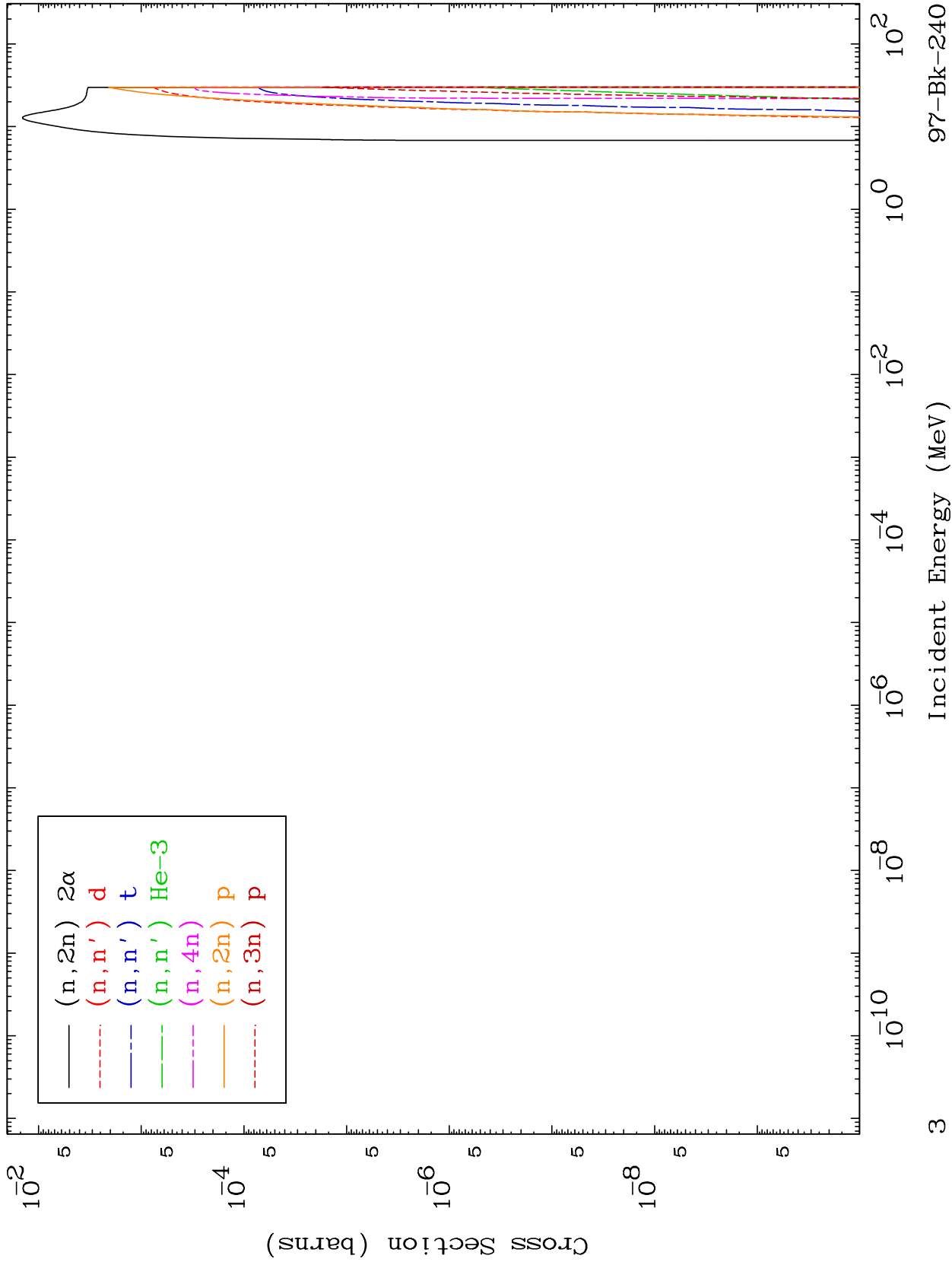
97-Bk-240



MAT 9725

Neutron Production
293 Kelvin Cross Sections

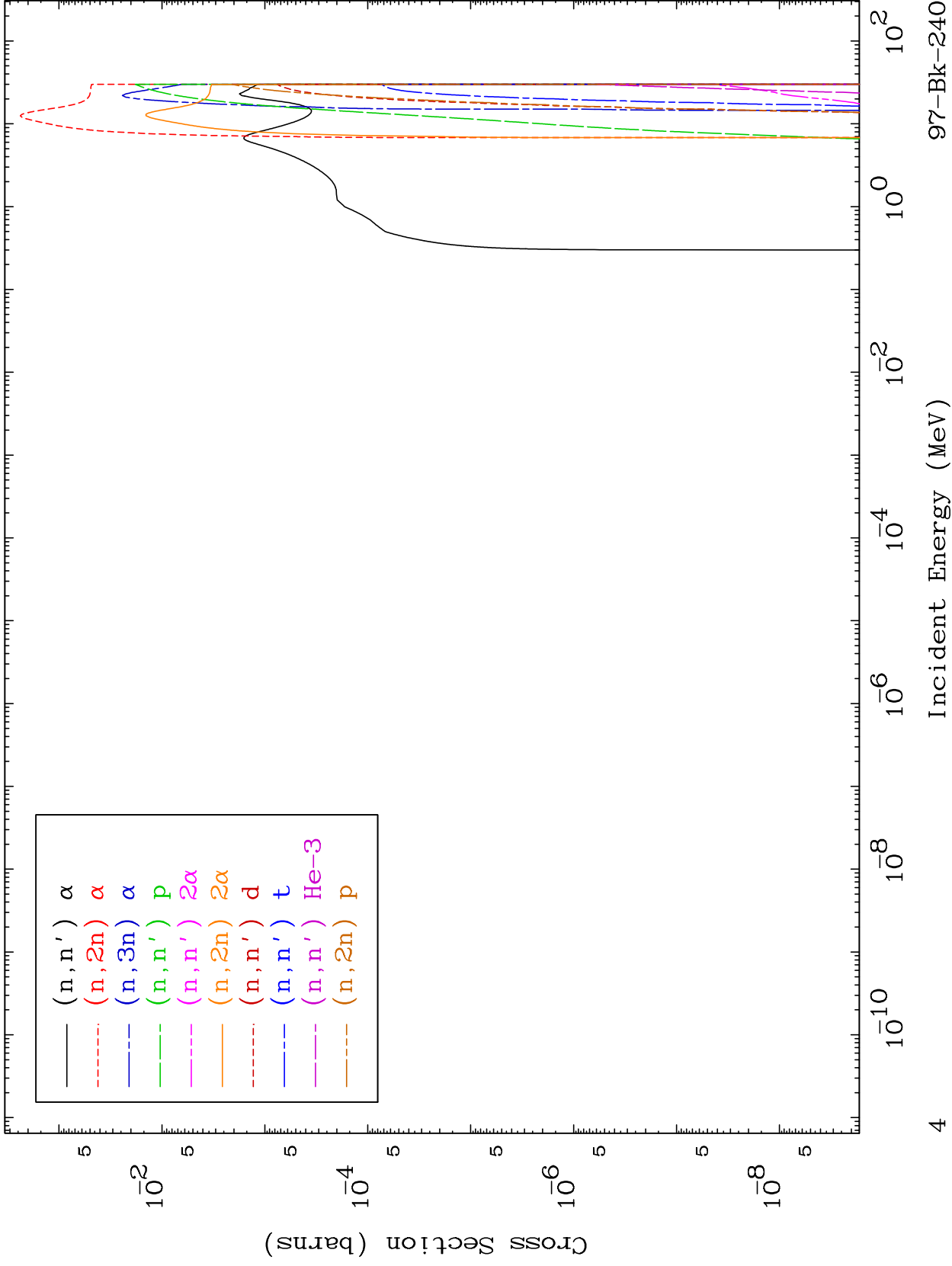
97-Bk-240



MAT 9725

Charged Particle
293 Kelvin Cross Sections

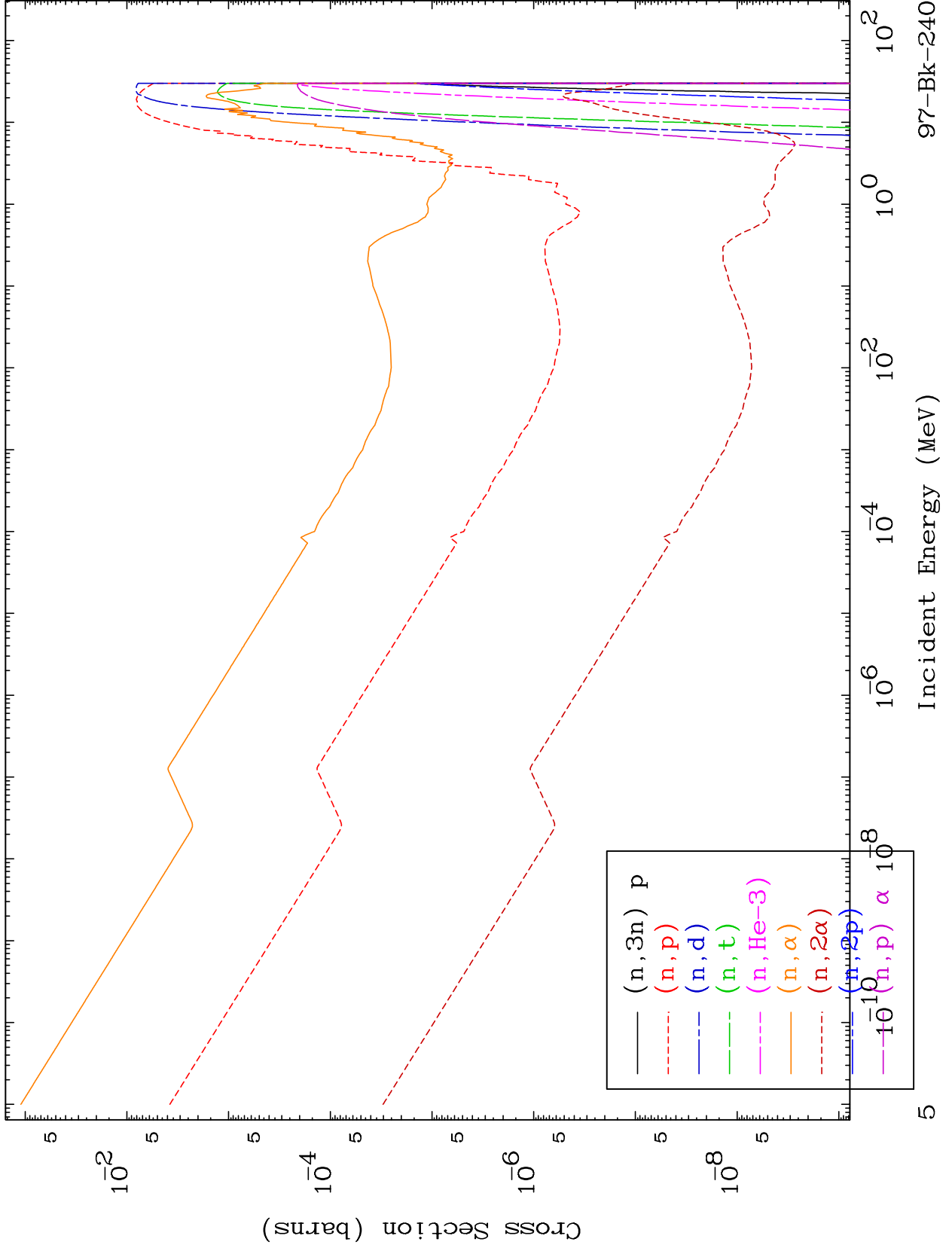
97-Bk-240



MAT 9725

Charged Particle
293 Kelvin Cross Sections

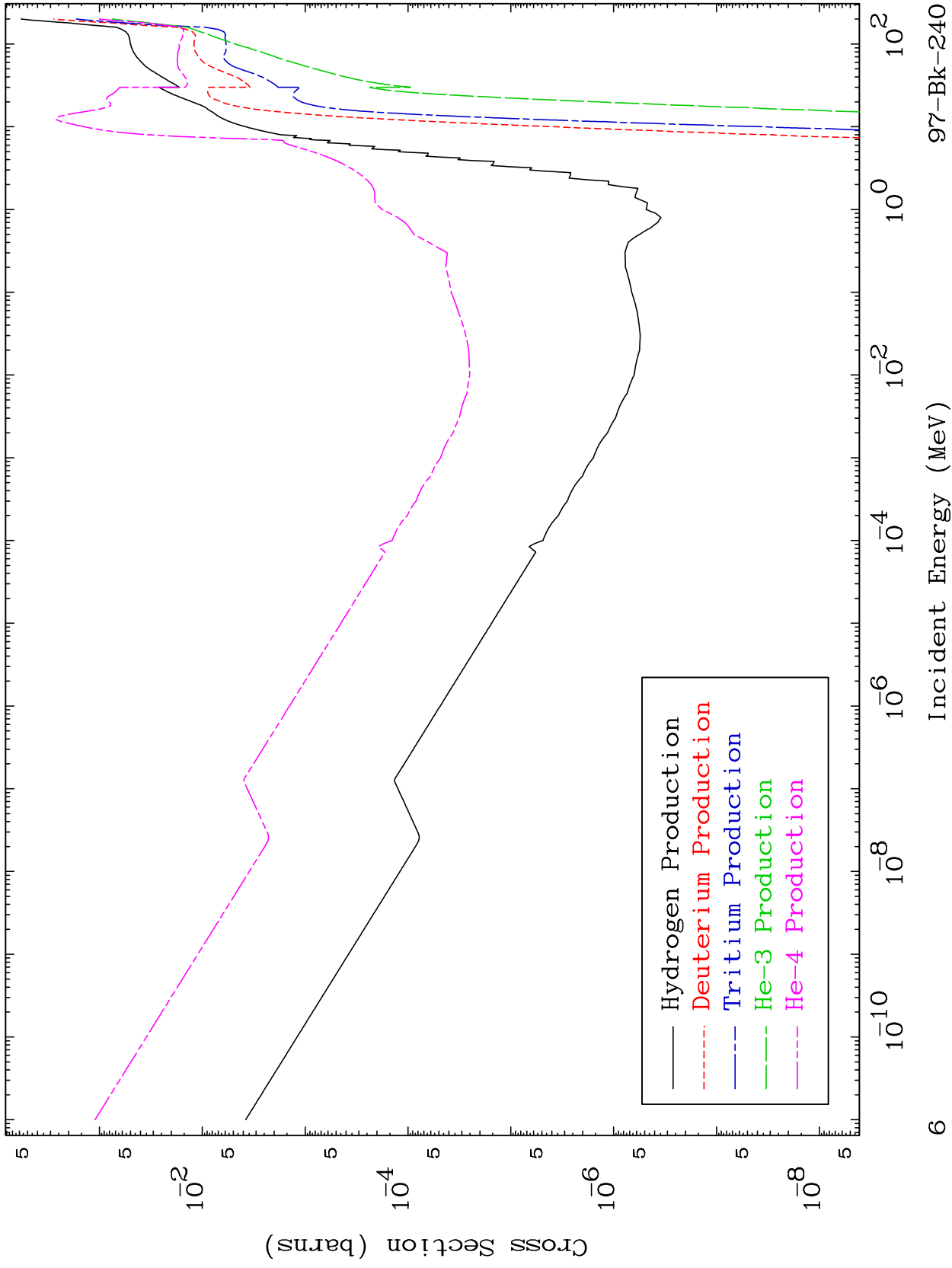
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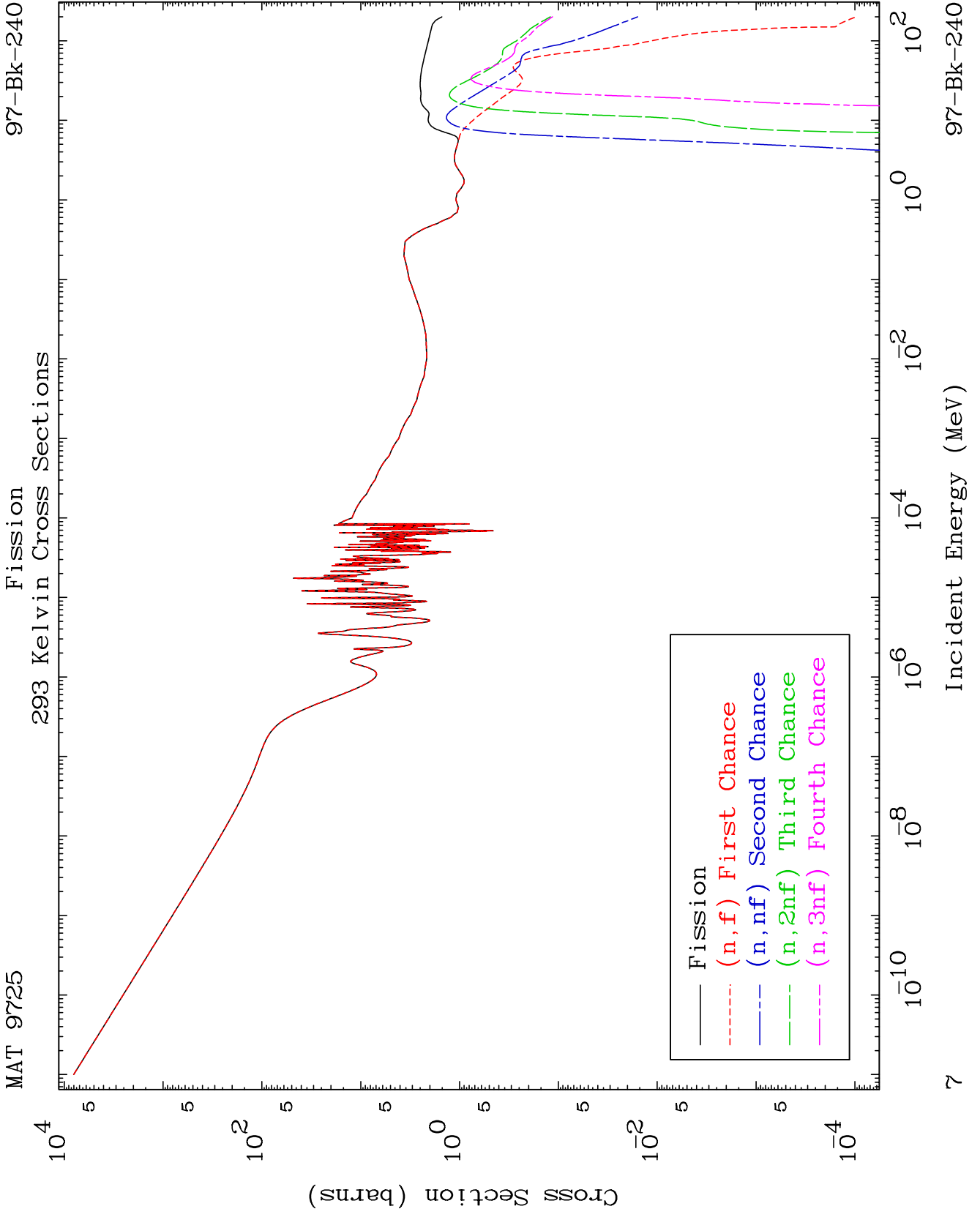


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Particle Production
293 Kelvin Cross Sections

97-Bk-240

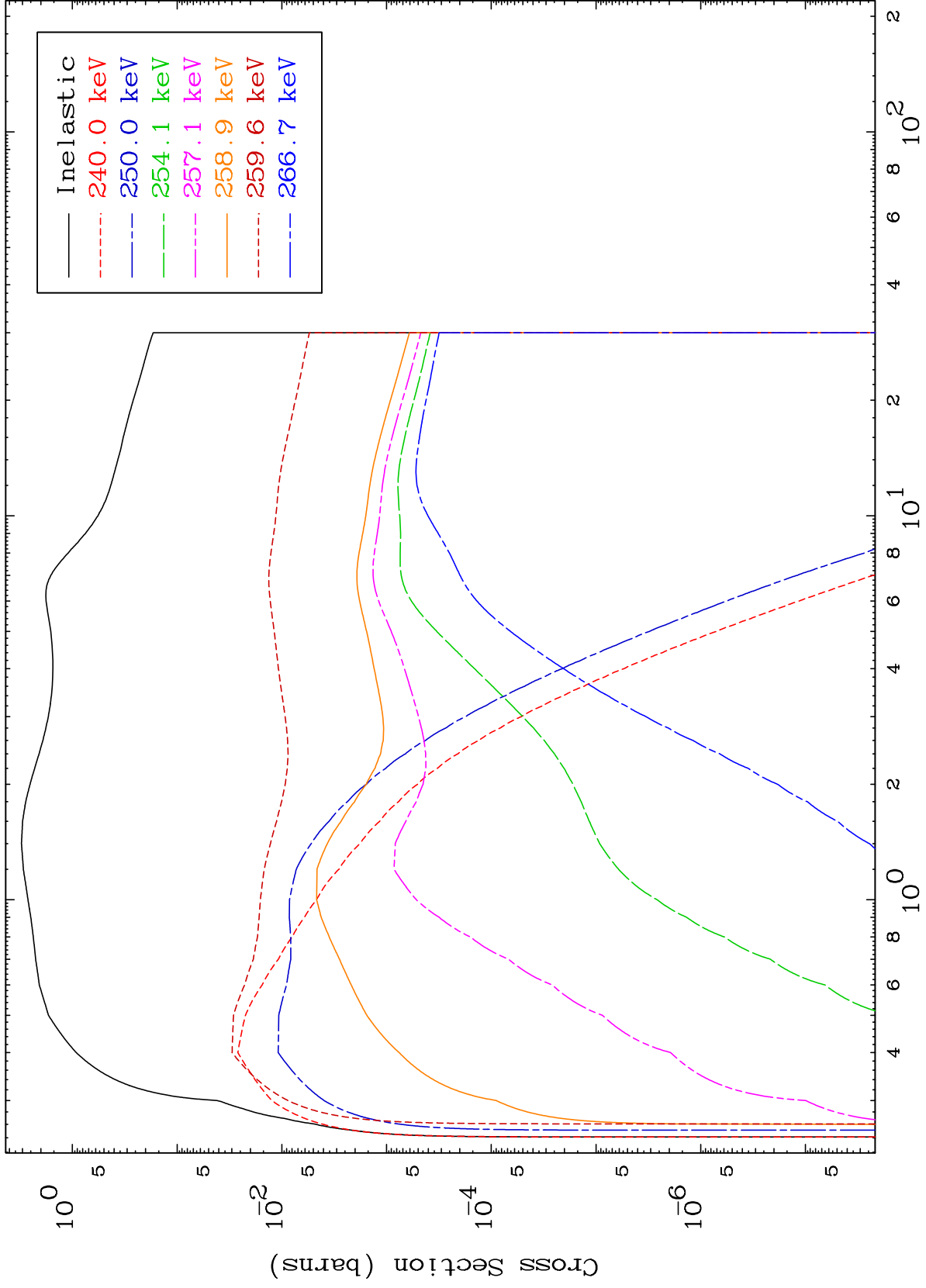




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(n,n') Level
293 Kelvin Cross Sections

97-Bk-240



8

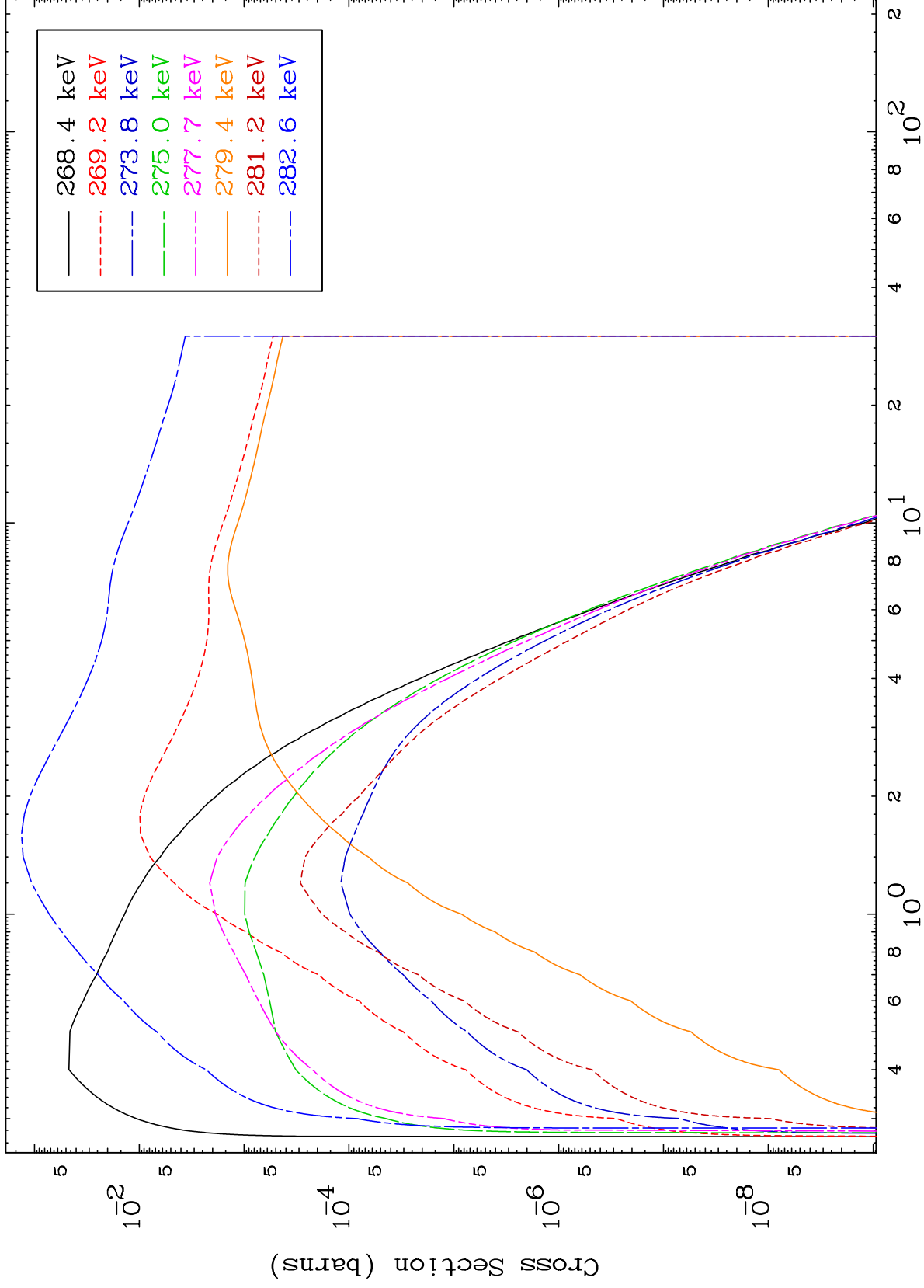
Incident Energy (MeV)

97-Bk-240

MAT 9725

(n,n') Level
293 Kelvin Cross Sections

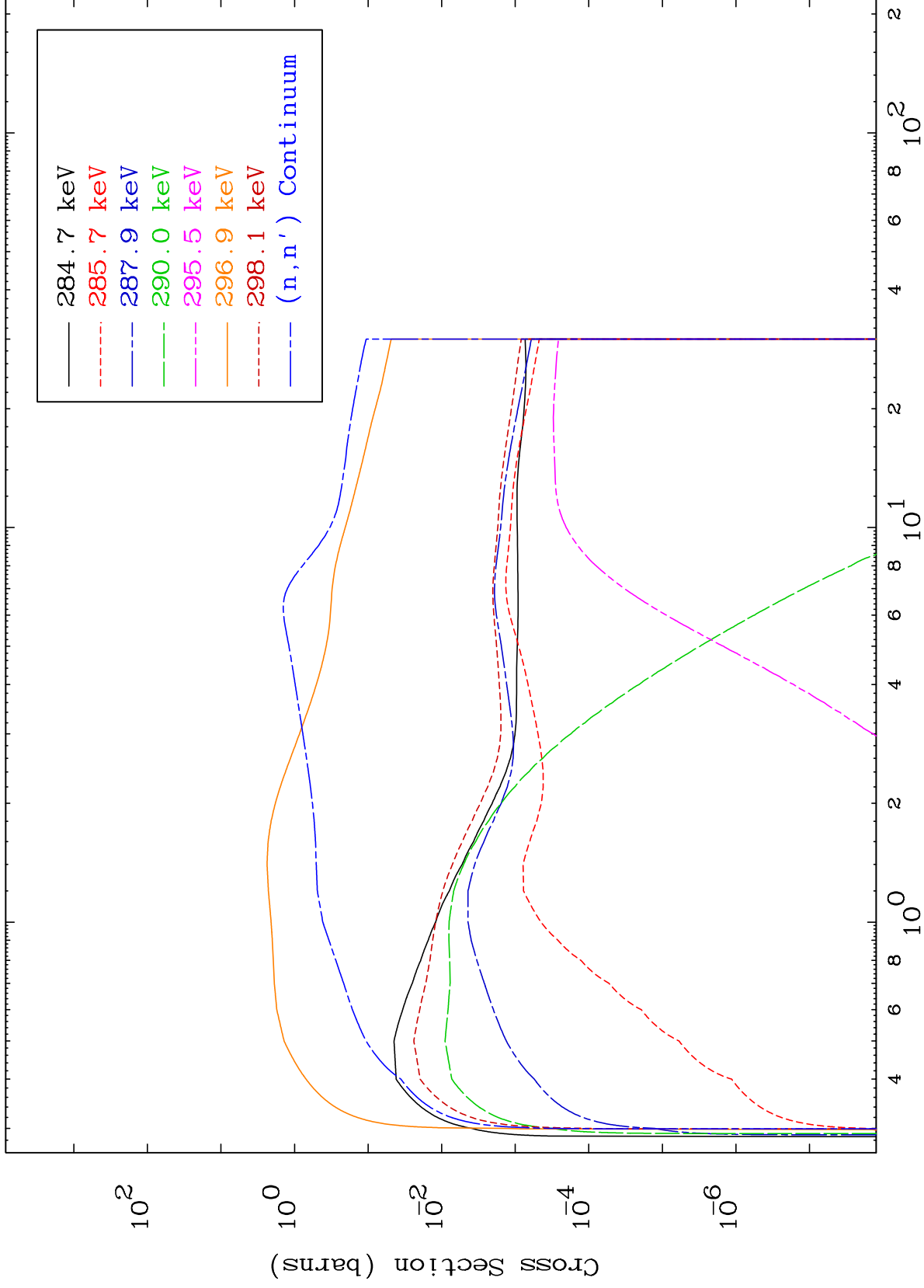
97-Bk-240



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(n,n') Level
293 Kelvin Cross Sections

97-Bk-240



10

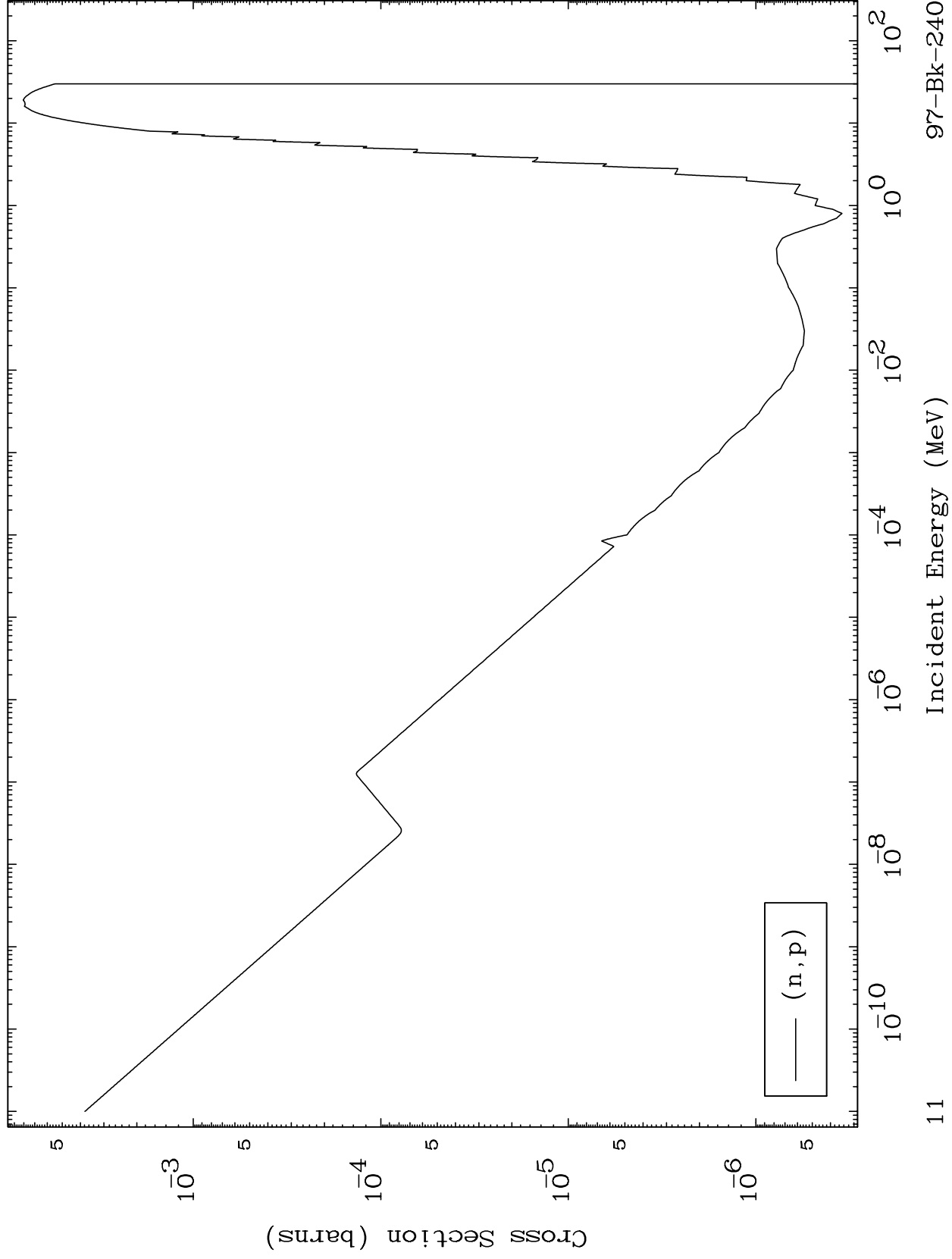
Incident Energy (MeV)

97-Bk-240

MAT 9725

(n,p) Levels
293 Kelvin Cross Sections

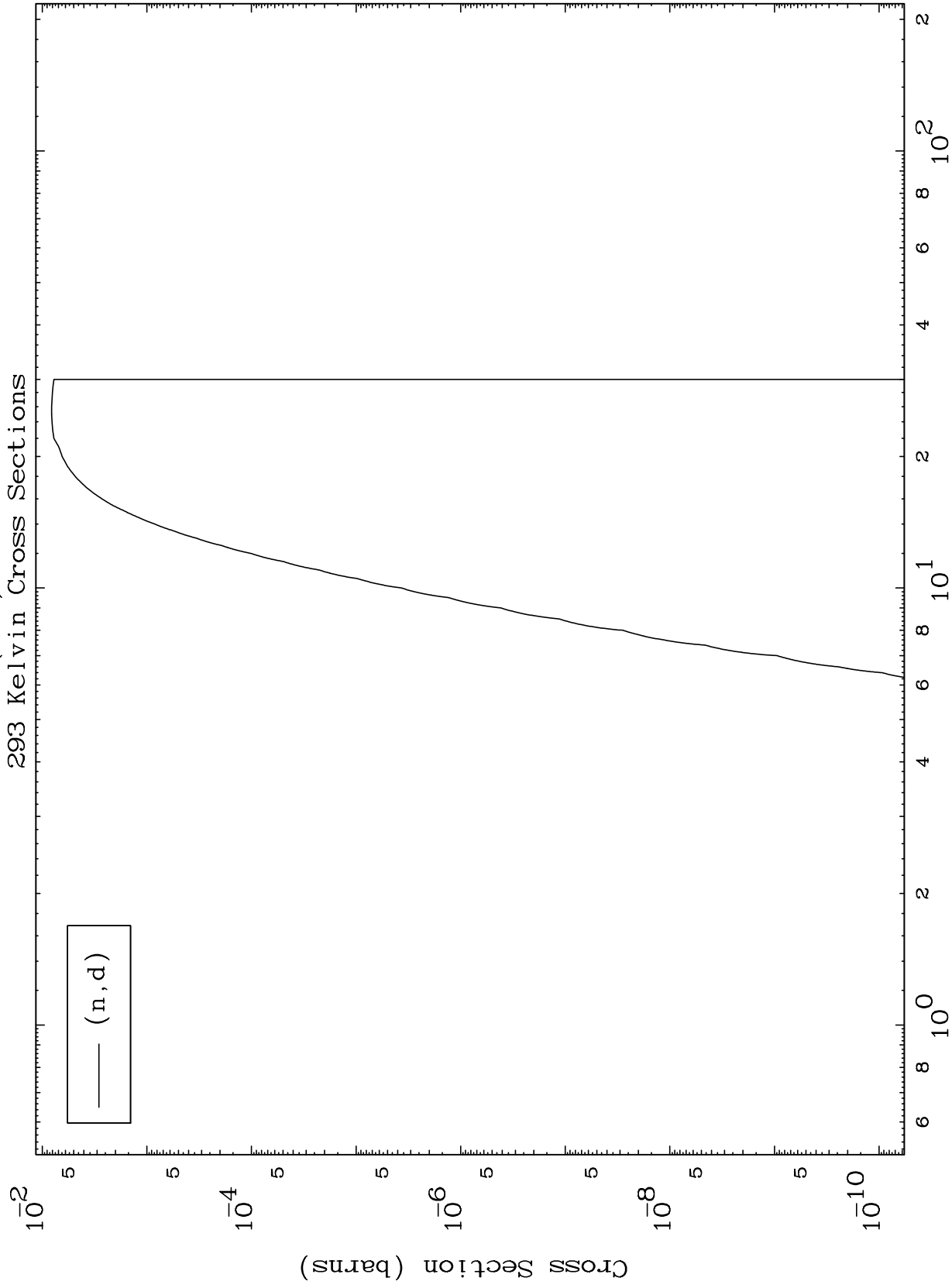
97-Bk-240



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(n,d) Levels

97-Bk-240



12

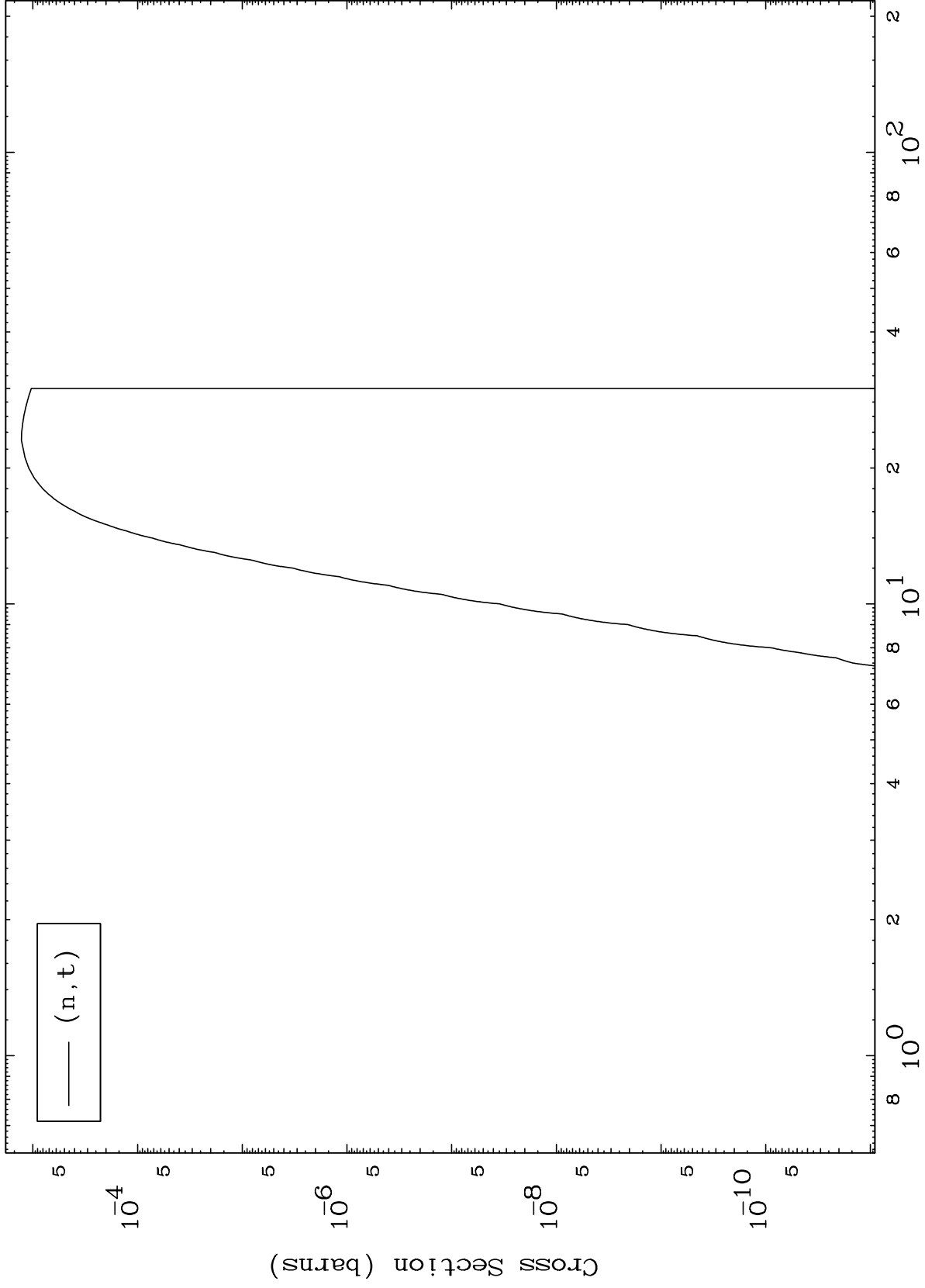
Incident Energy (MeV)

97-Bk-240

MAT 9725

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-240



13

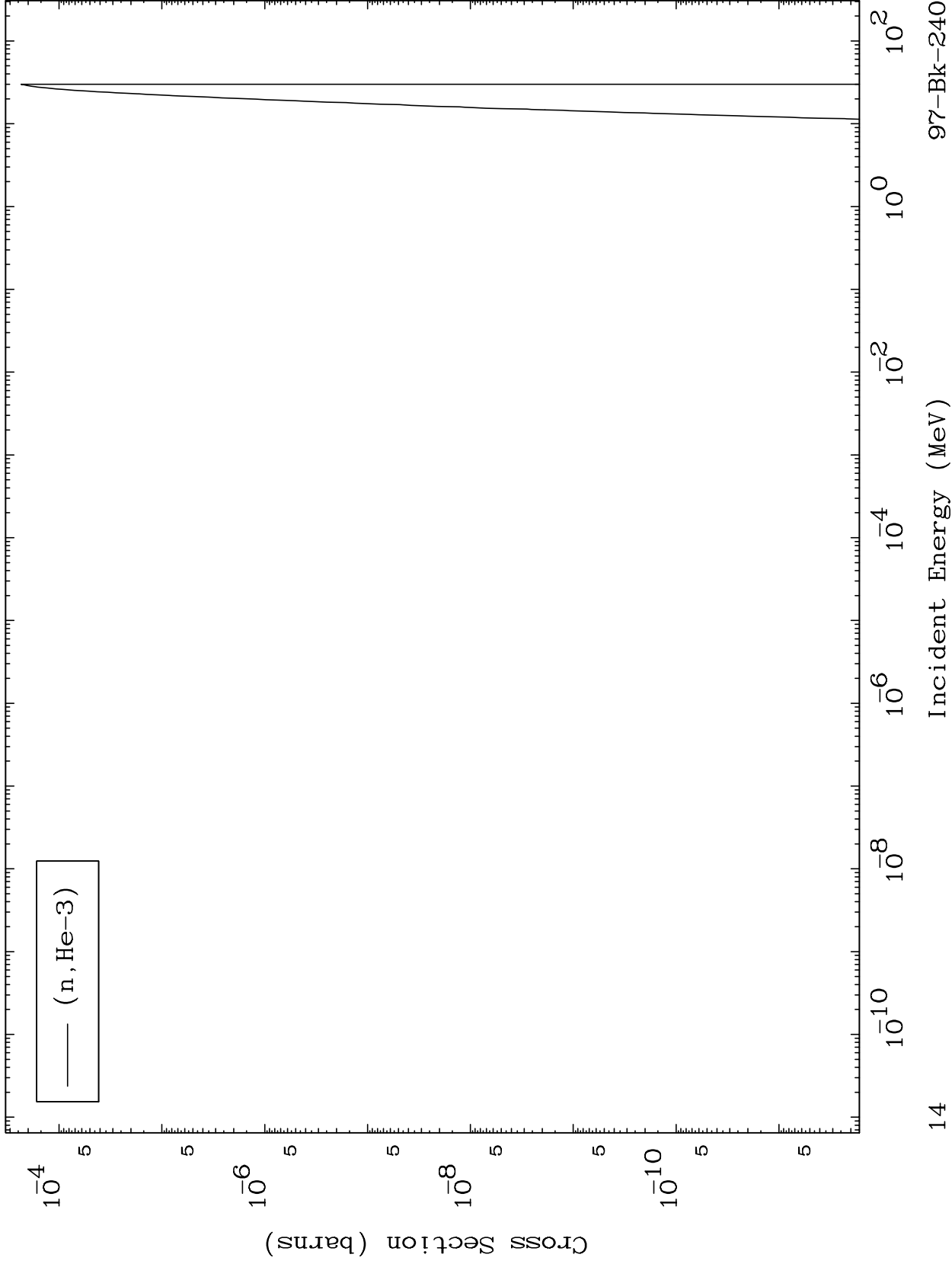
Incident Energy (MeV)

97-Bk-240

MAT 9725

(n,He3) Levels
293 Kelvin Cross Sections

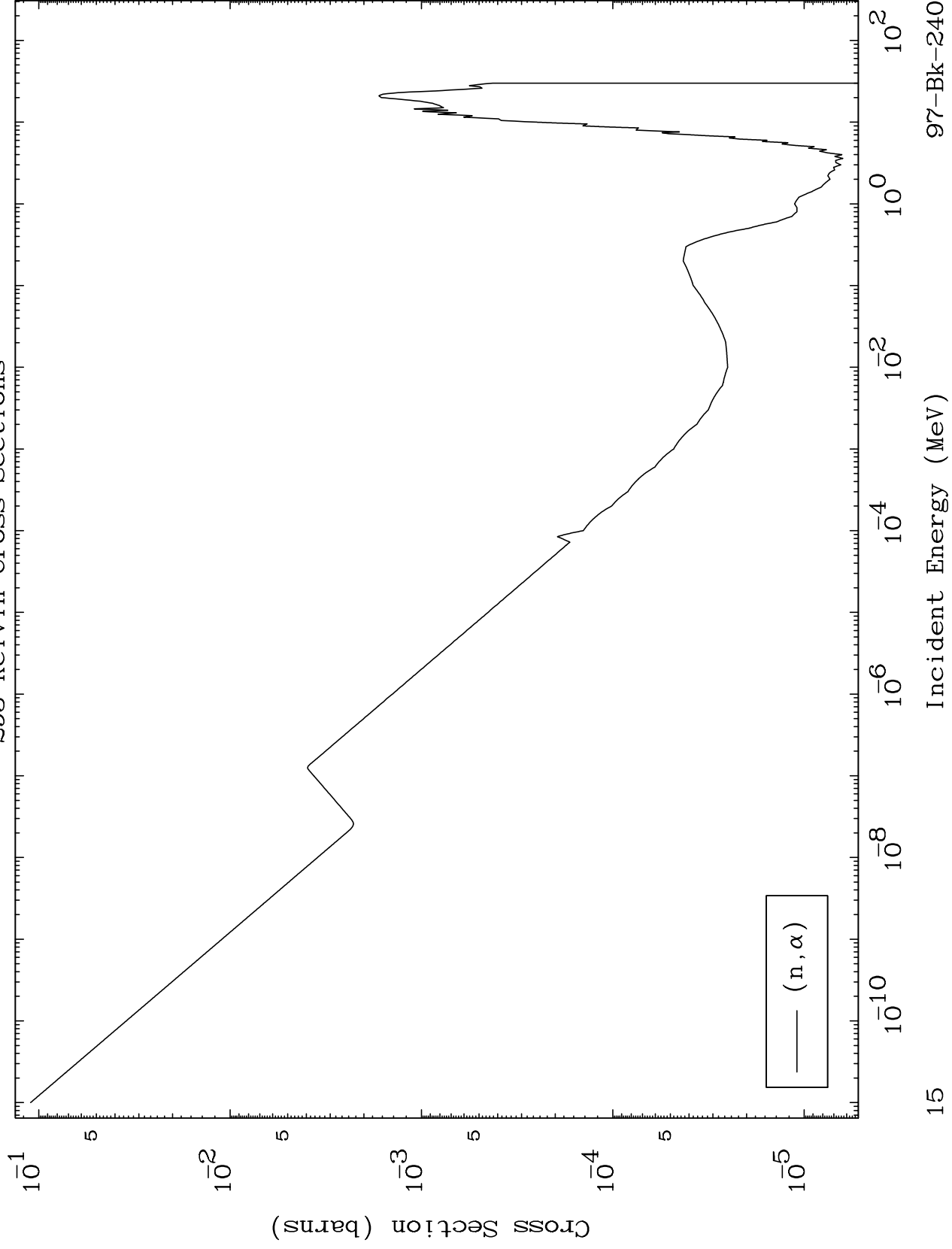
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MAT 9725

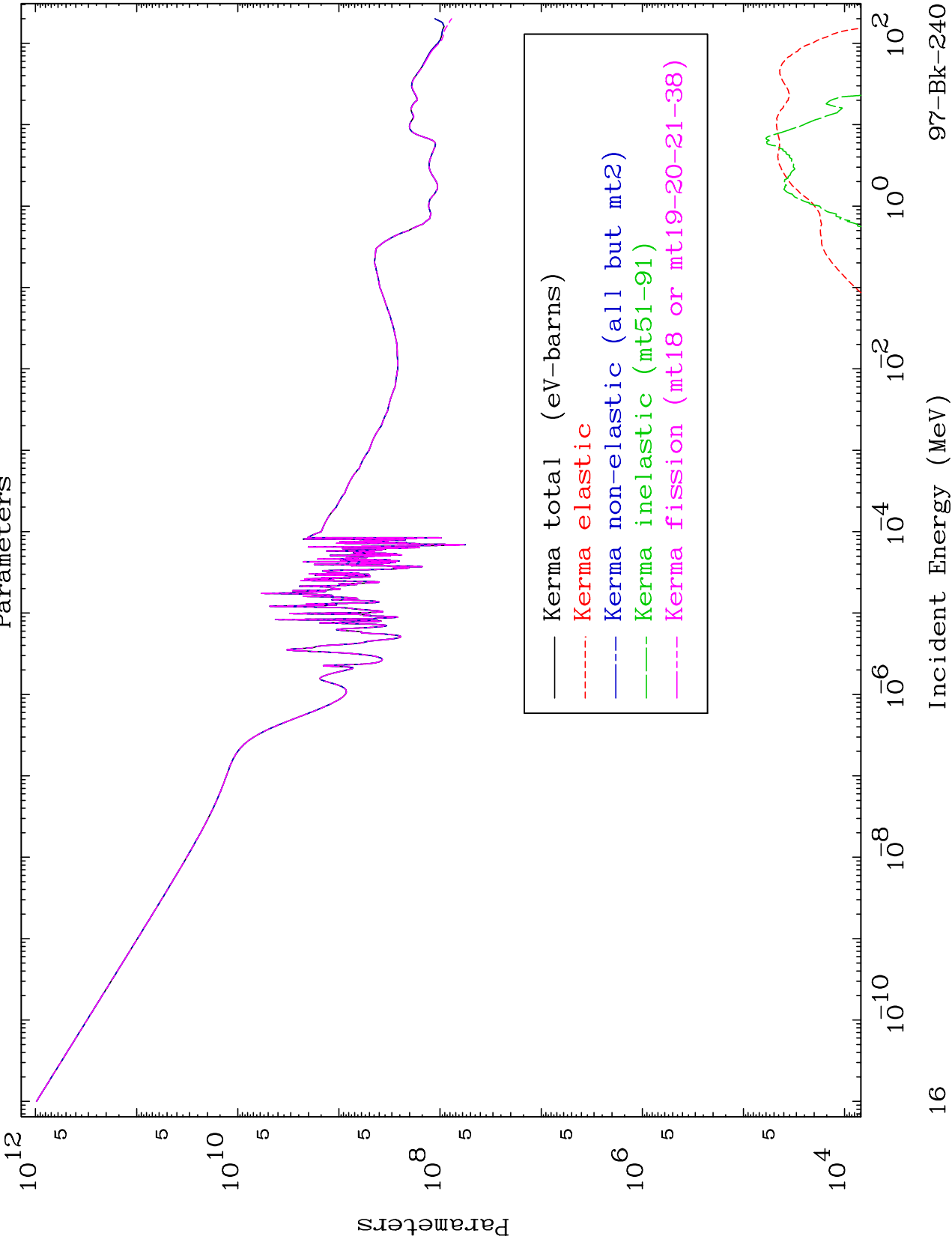
(n, α) Levels
293 Kelvin Cross Sections

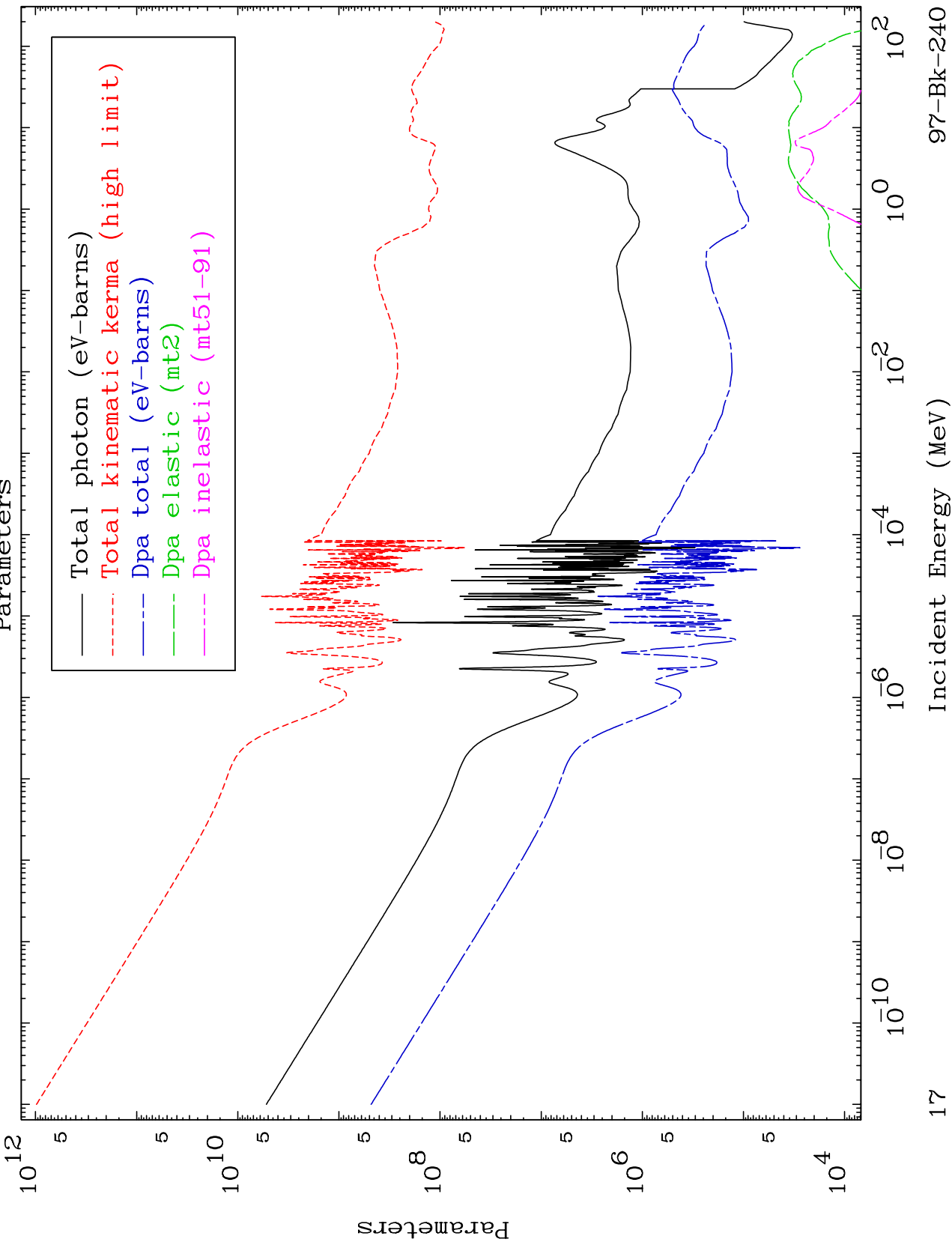
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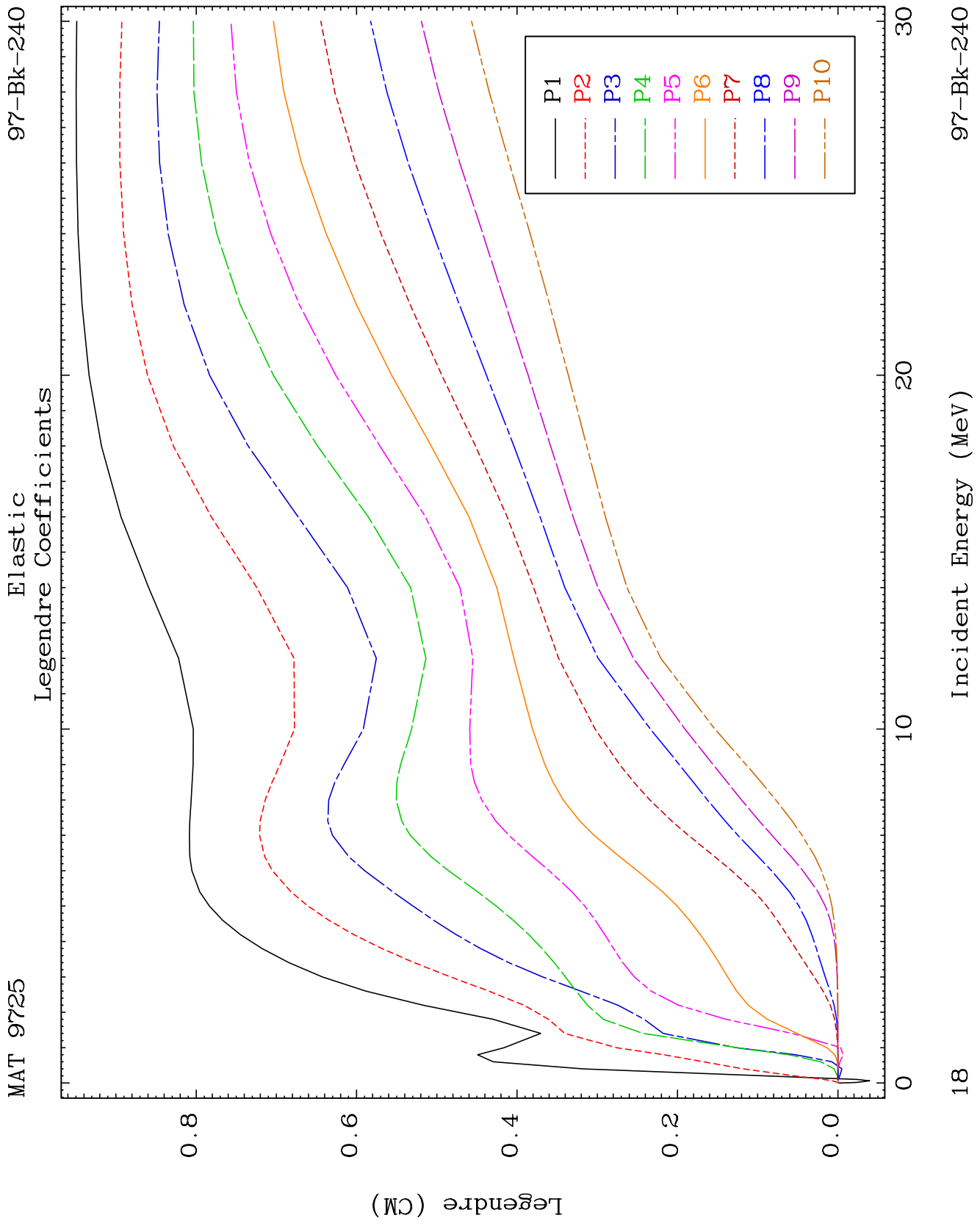


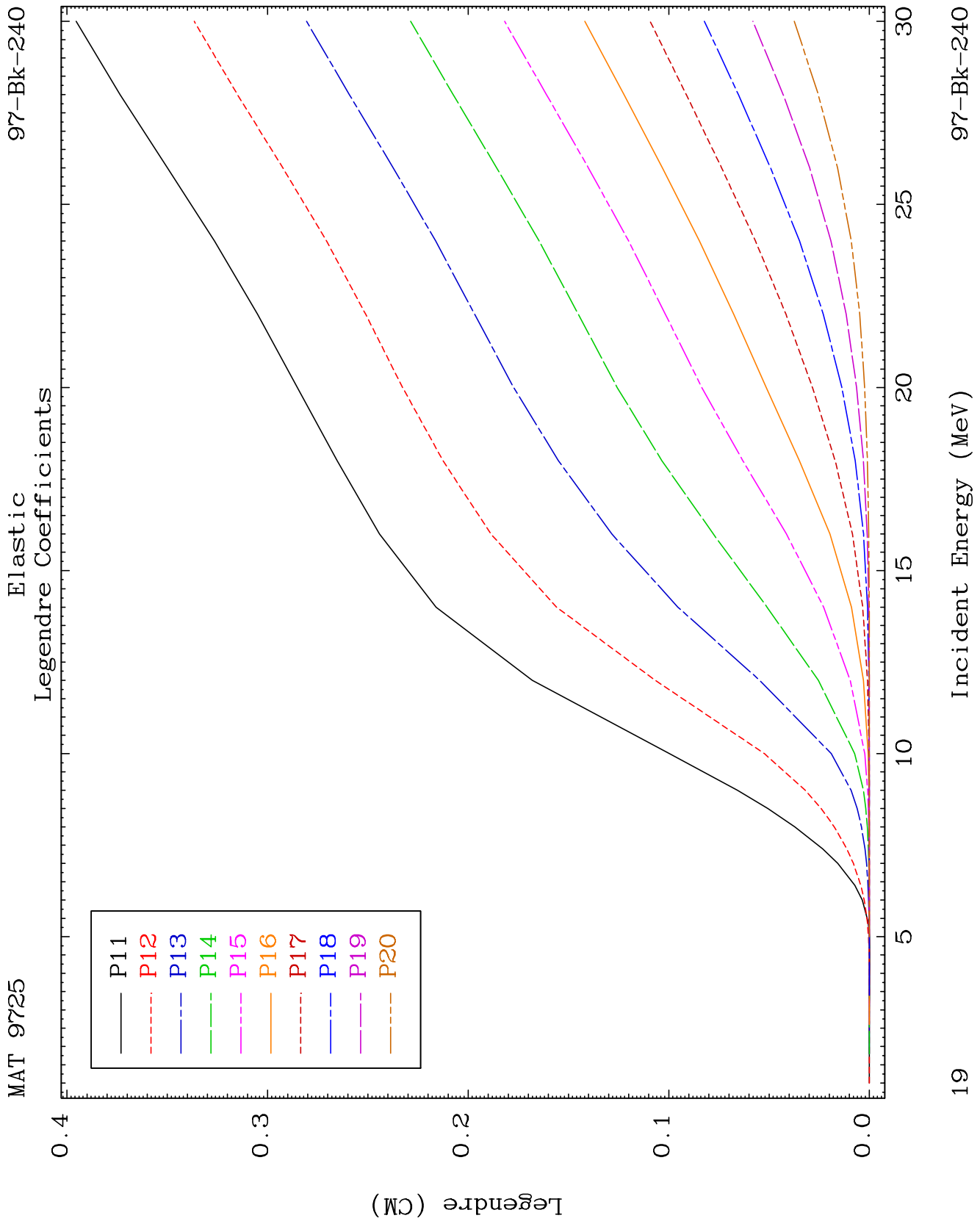
15

97-Bk-240





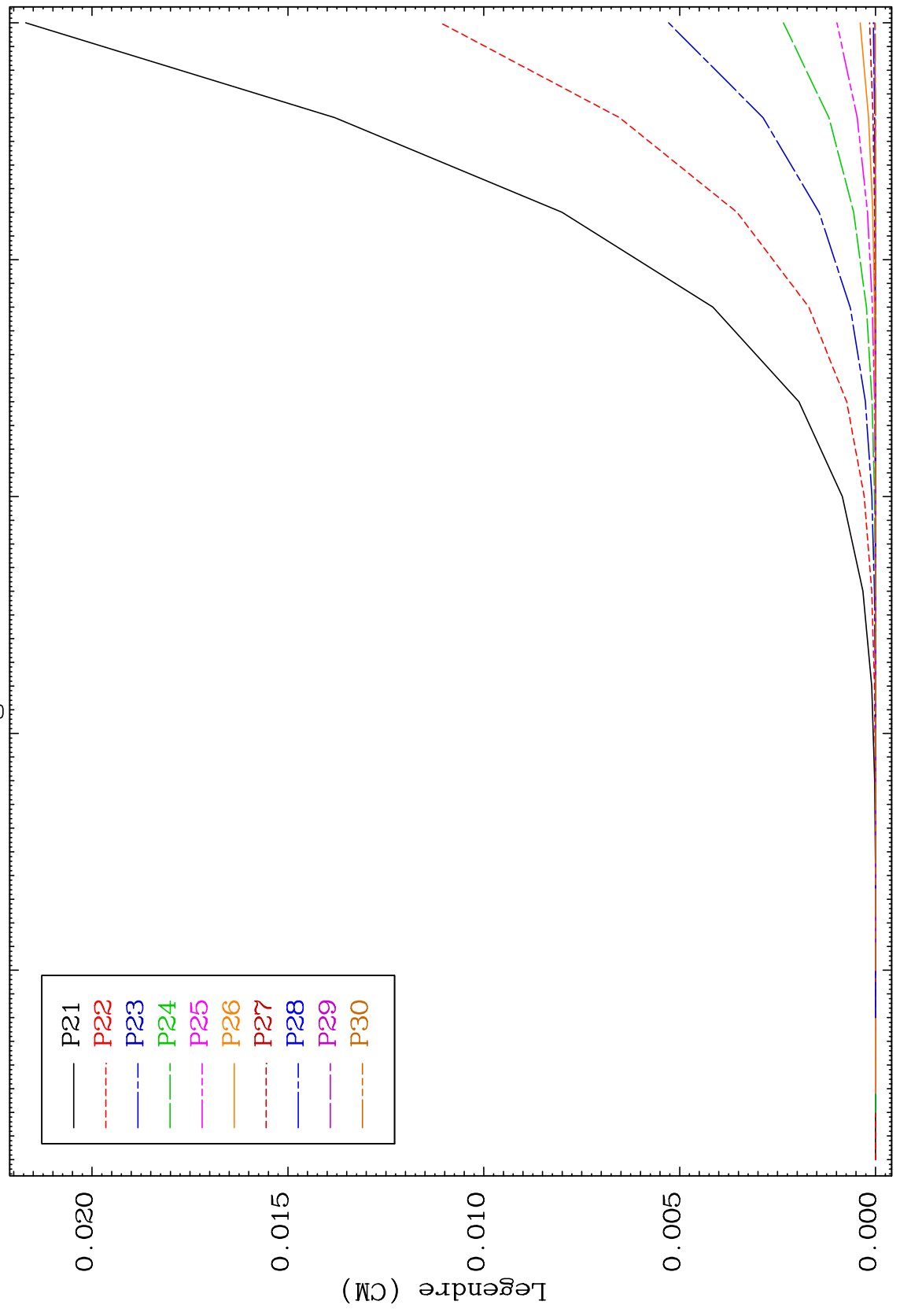




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Elastic Legendre Coefficients

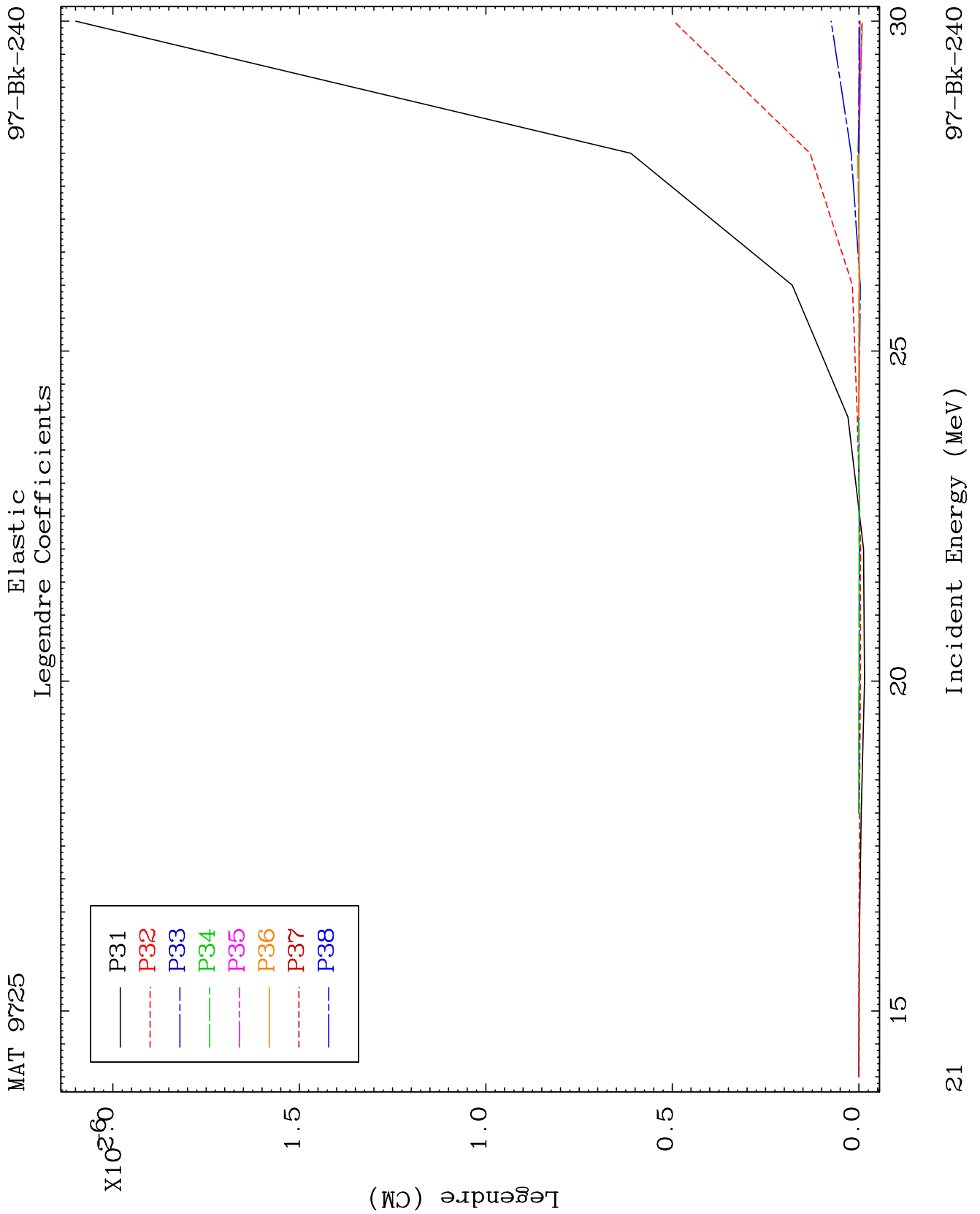
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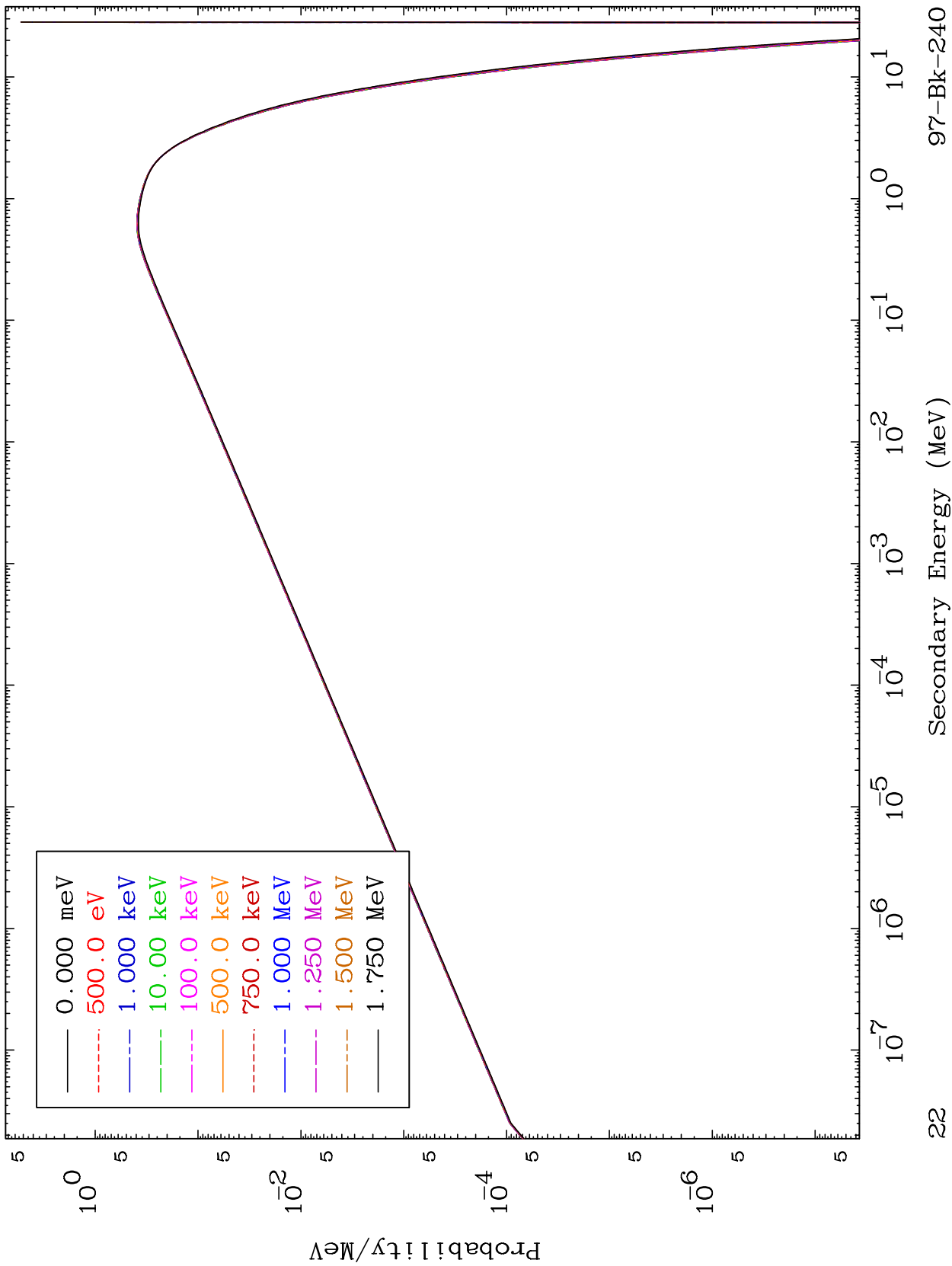
97-Bk-240

Incident Energy (MeV)

20



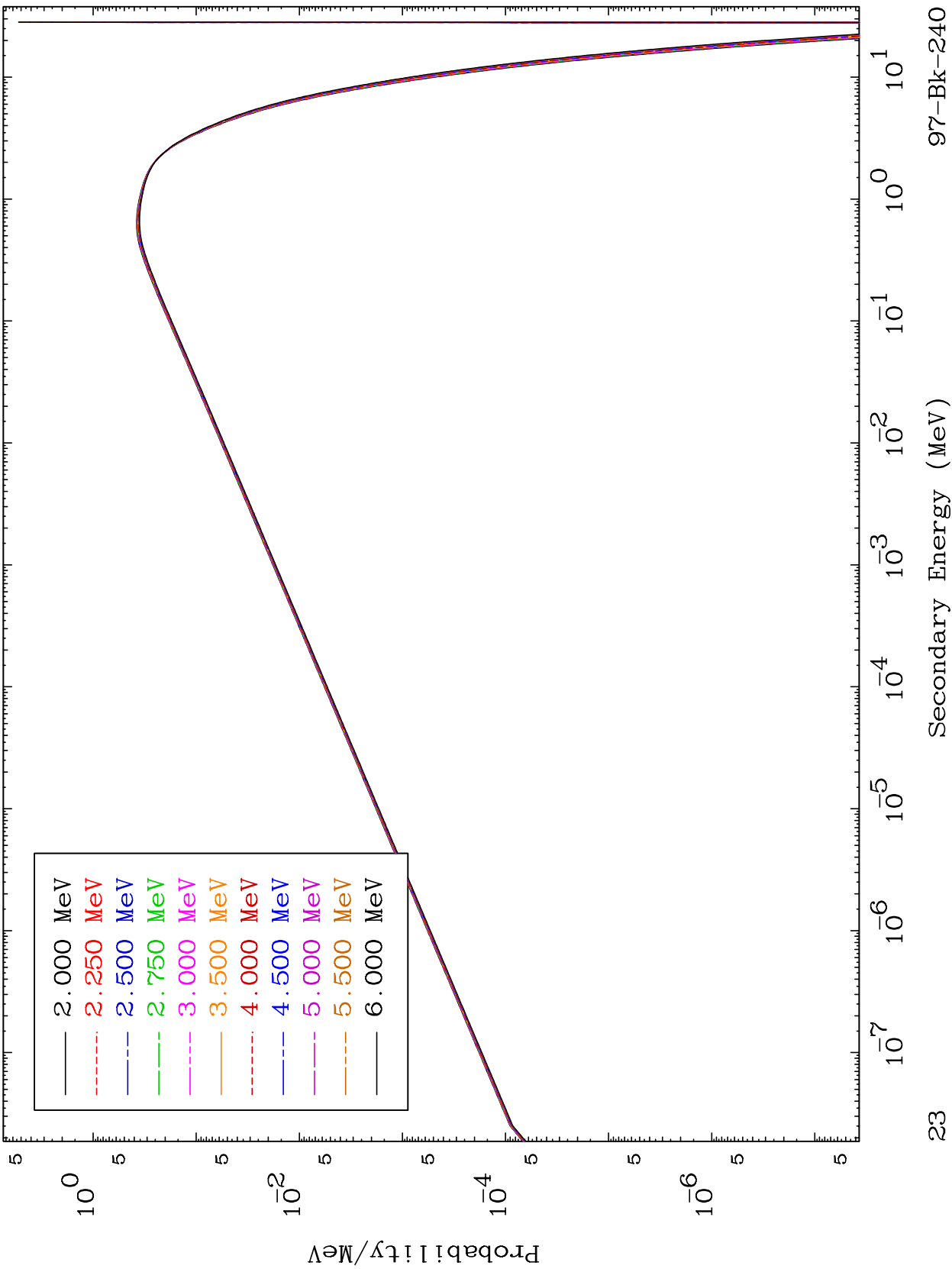
Fission Energy Distribution



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Fission Energy Distribution

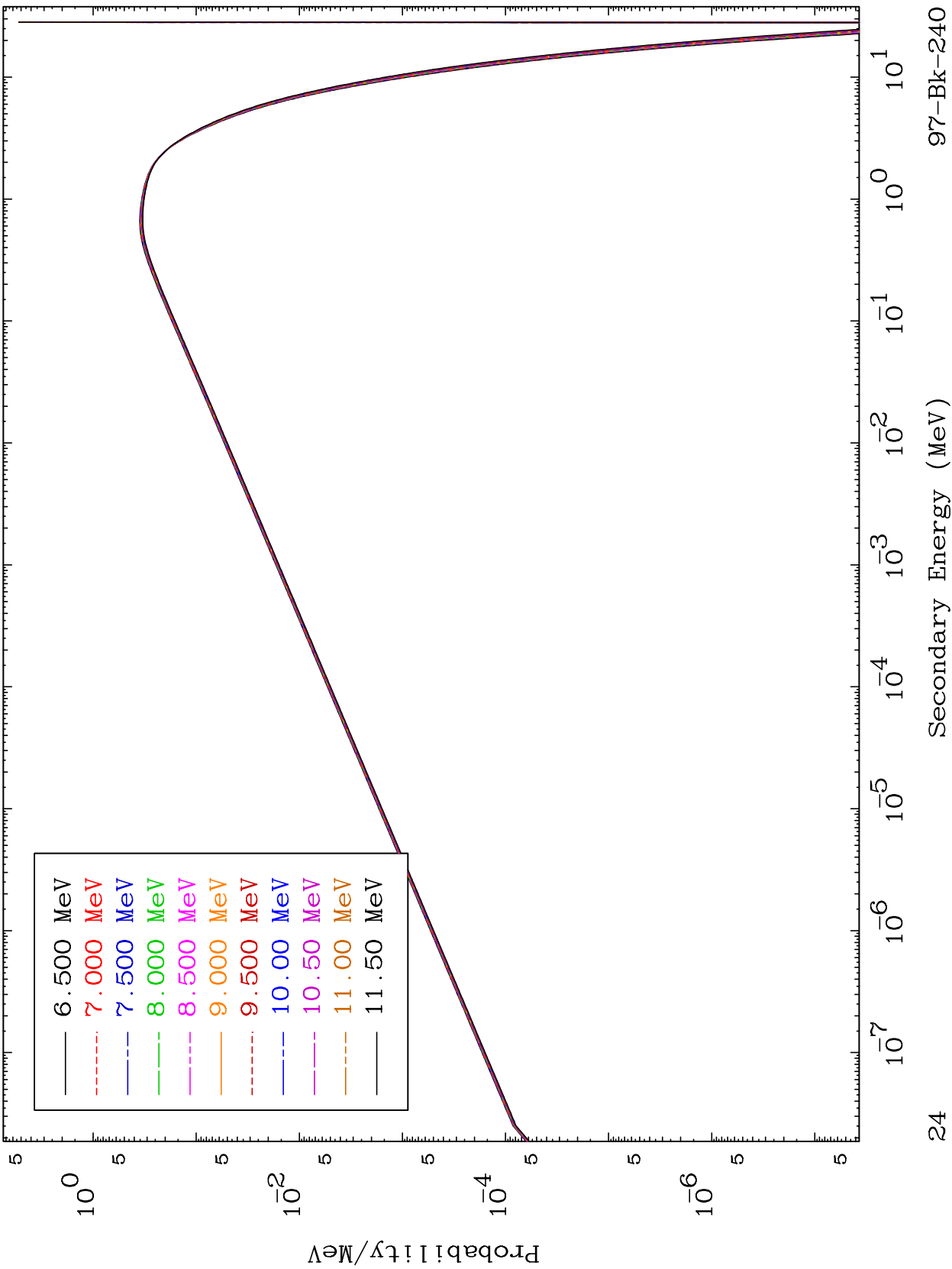
97-Bk-240



MAT 9725

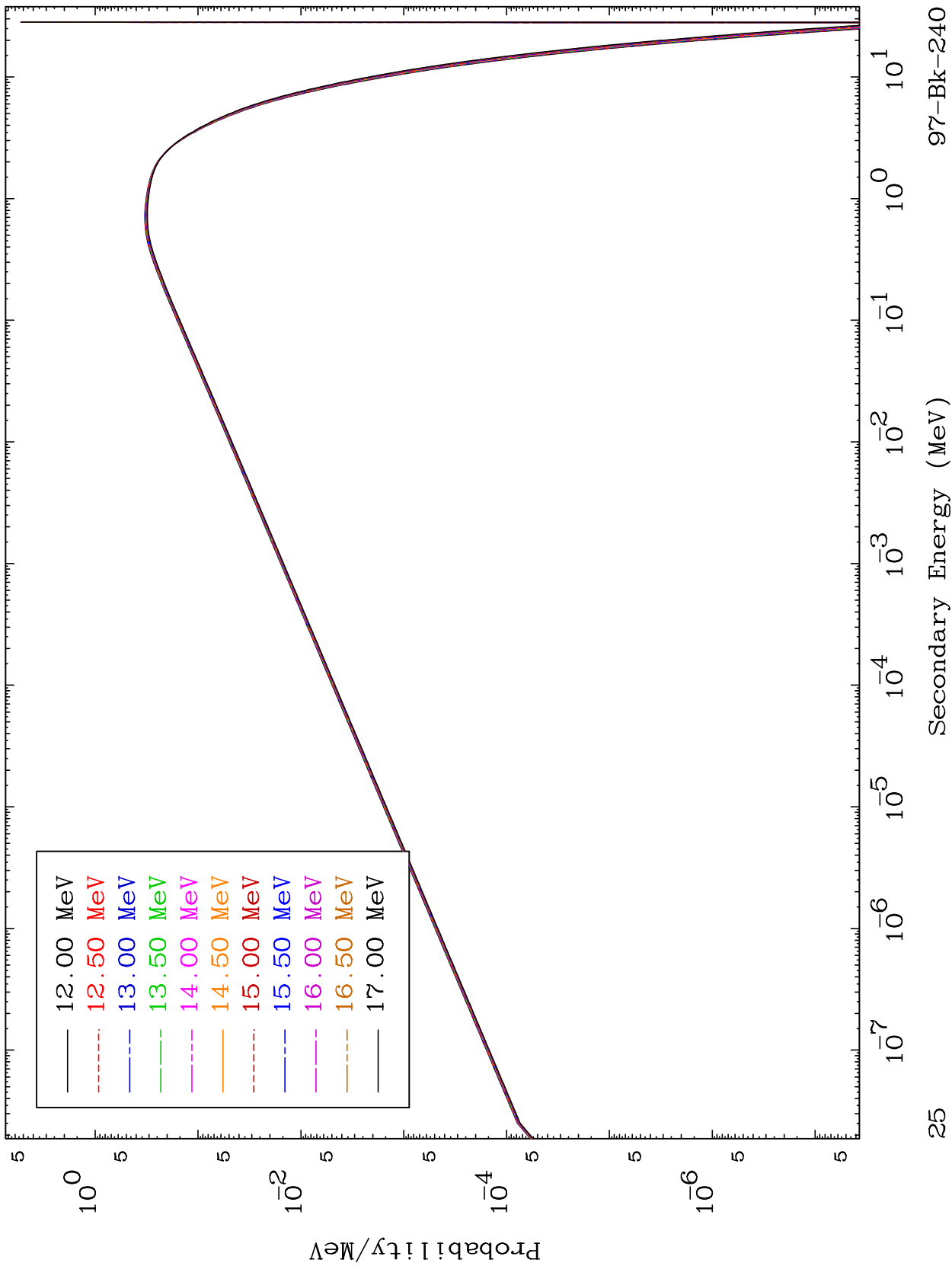
Fission Energy Distribution

97-Bk-240

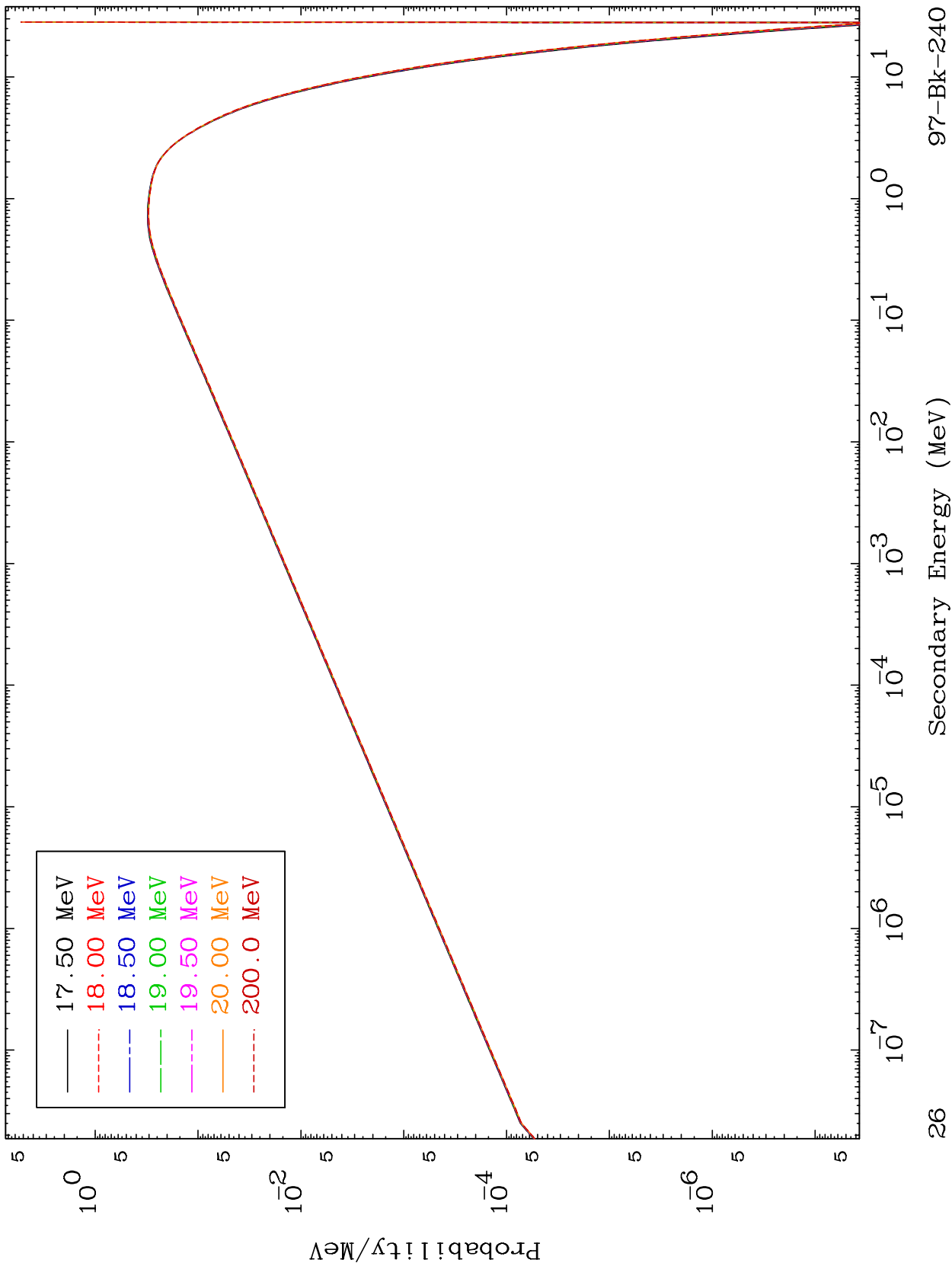


97-Bk-240

Fission Energy Distribution



Fission
Energy Distribution

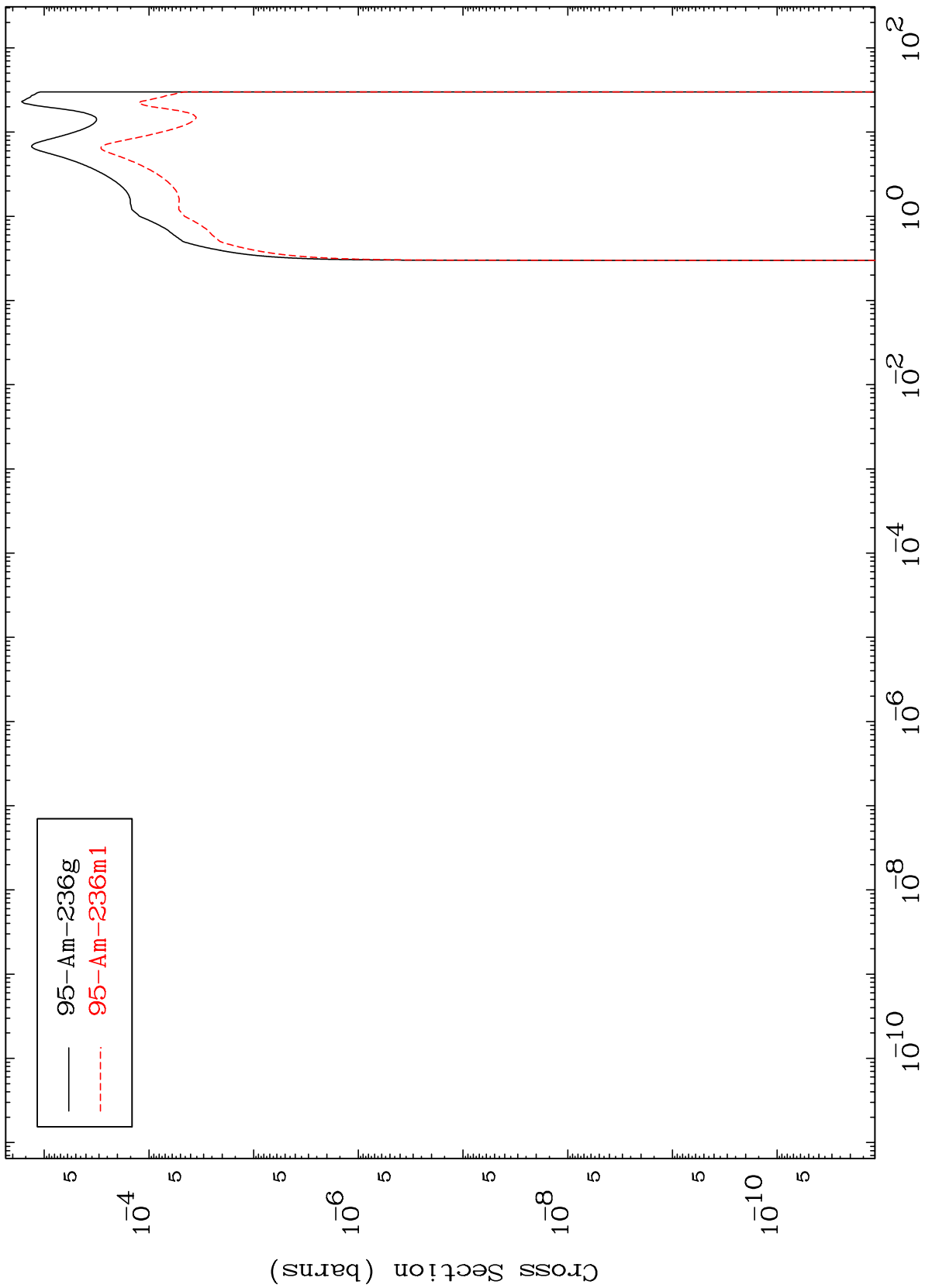


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(n,n') α

97-Bk-240

Radionuclide Production Cross Section



95-Am-236g
95-Am-236m1