

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

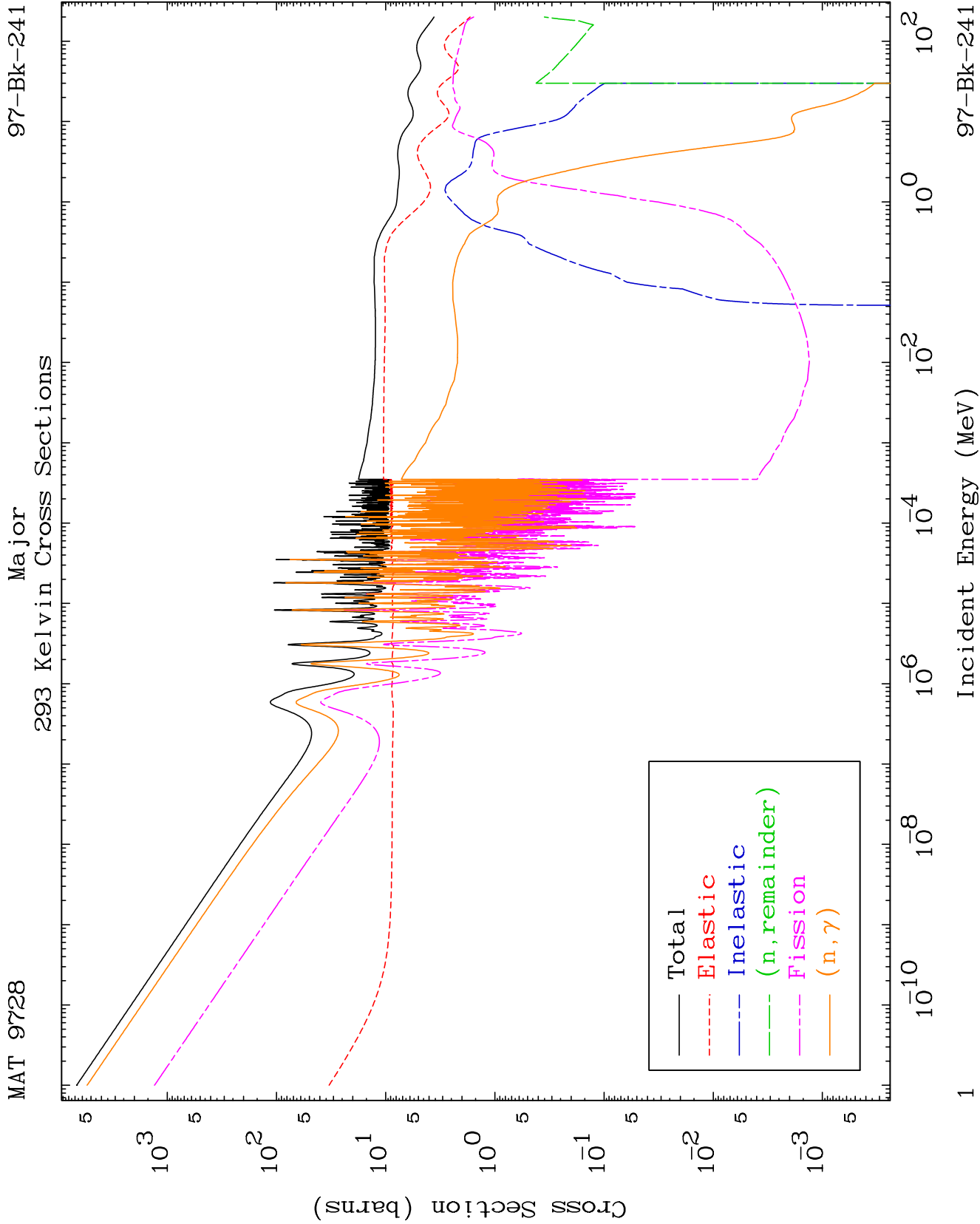
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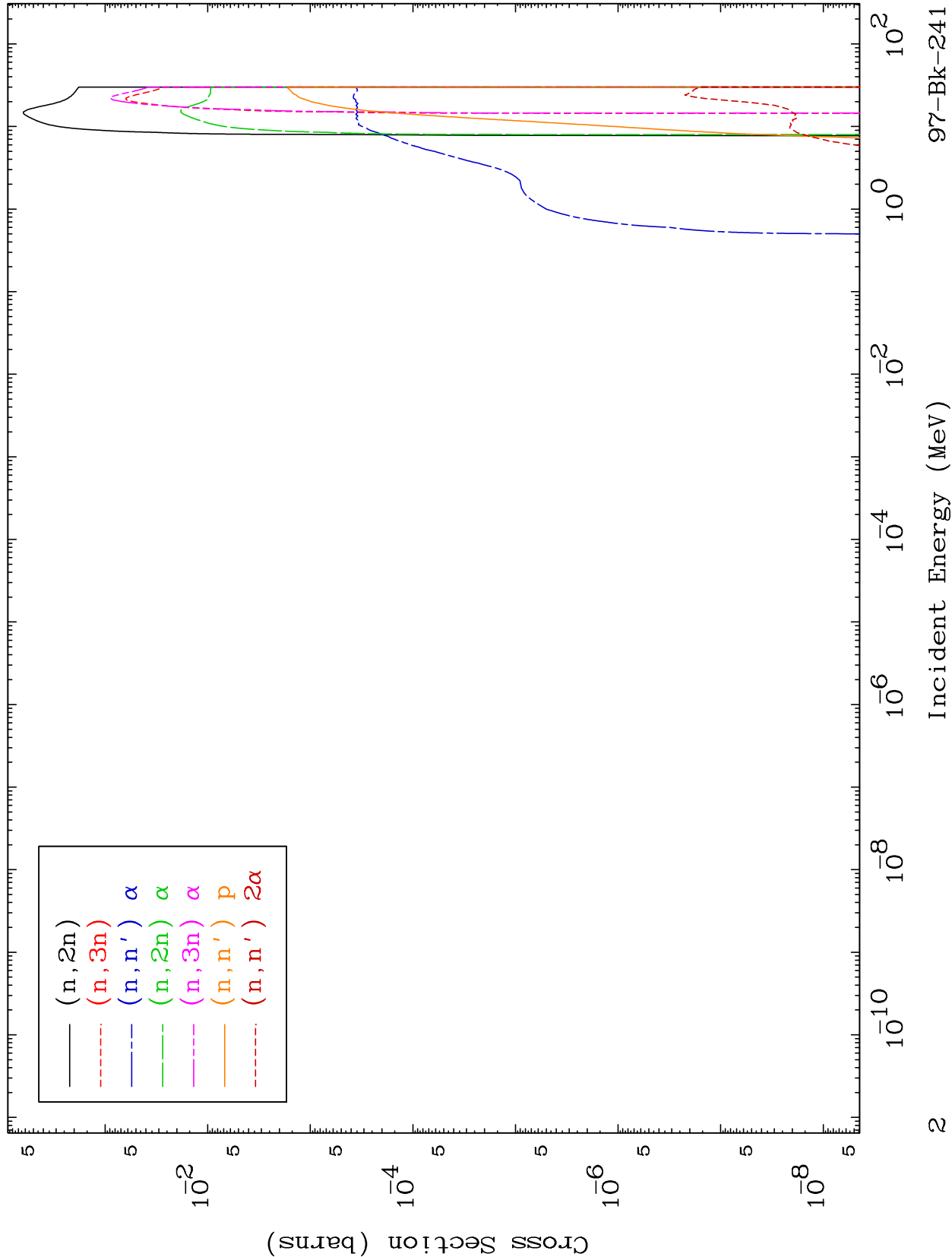
Press Mouse Button to Start

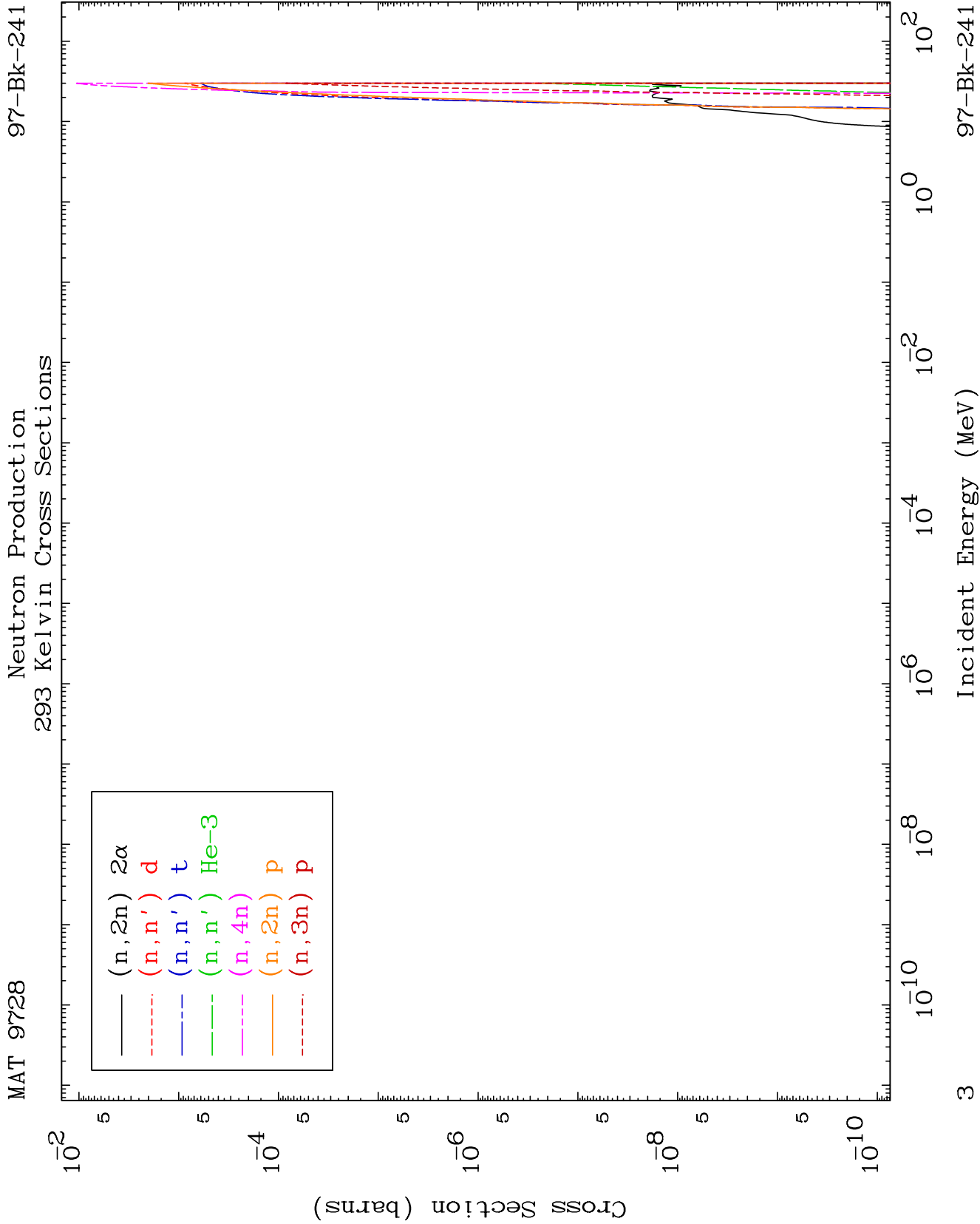


MAT 9728

Neutron Production
293 Kelvin Cross Sections

97-Bk-241

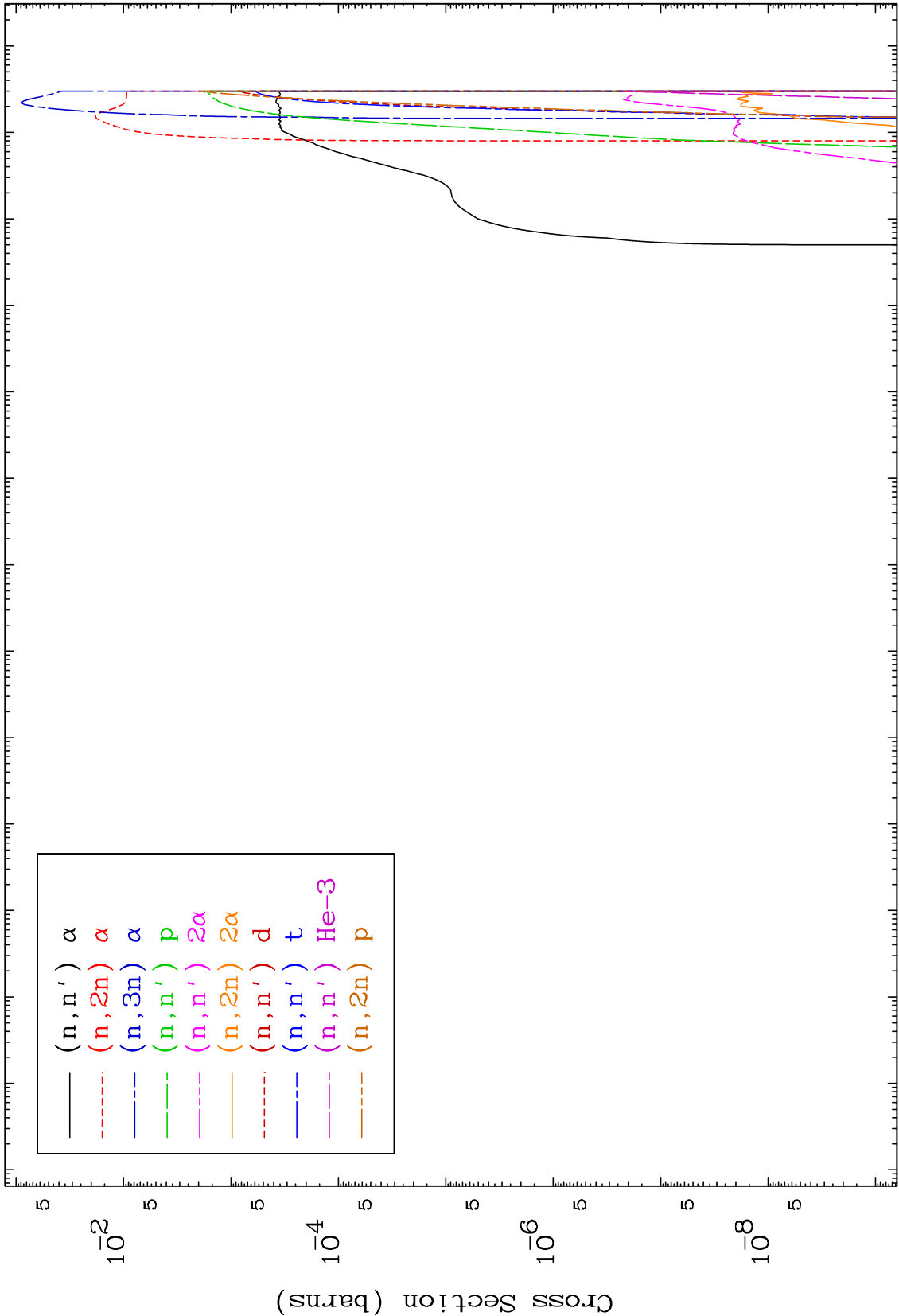


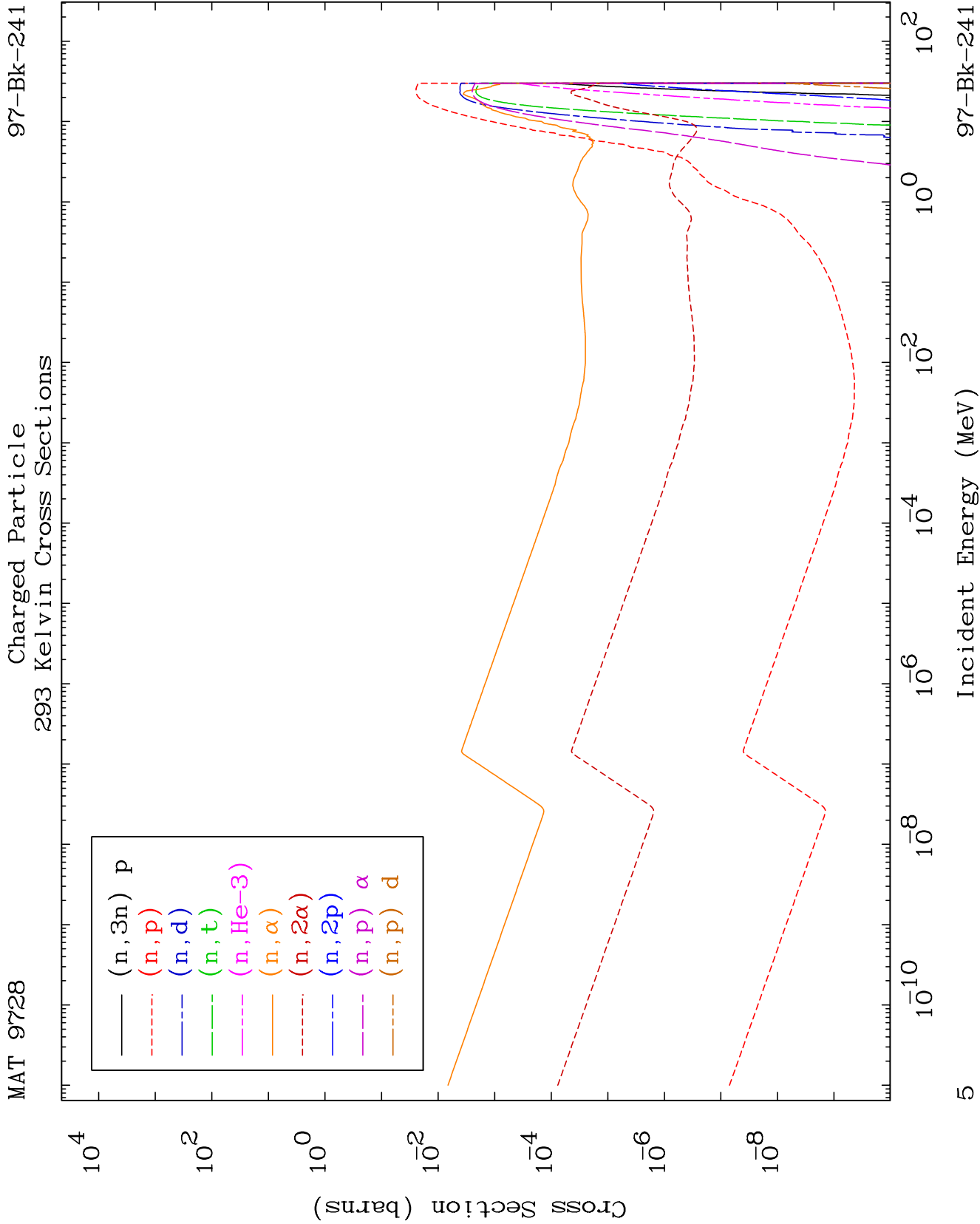


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Charged Particle
293 Kelvin Cross Sections

97-Bk-241

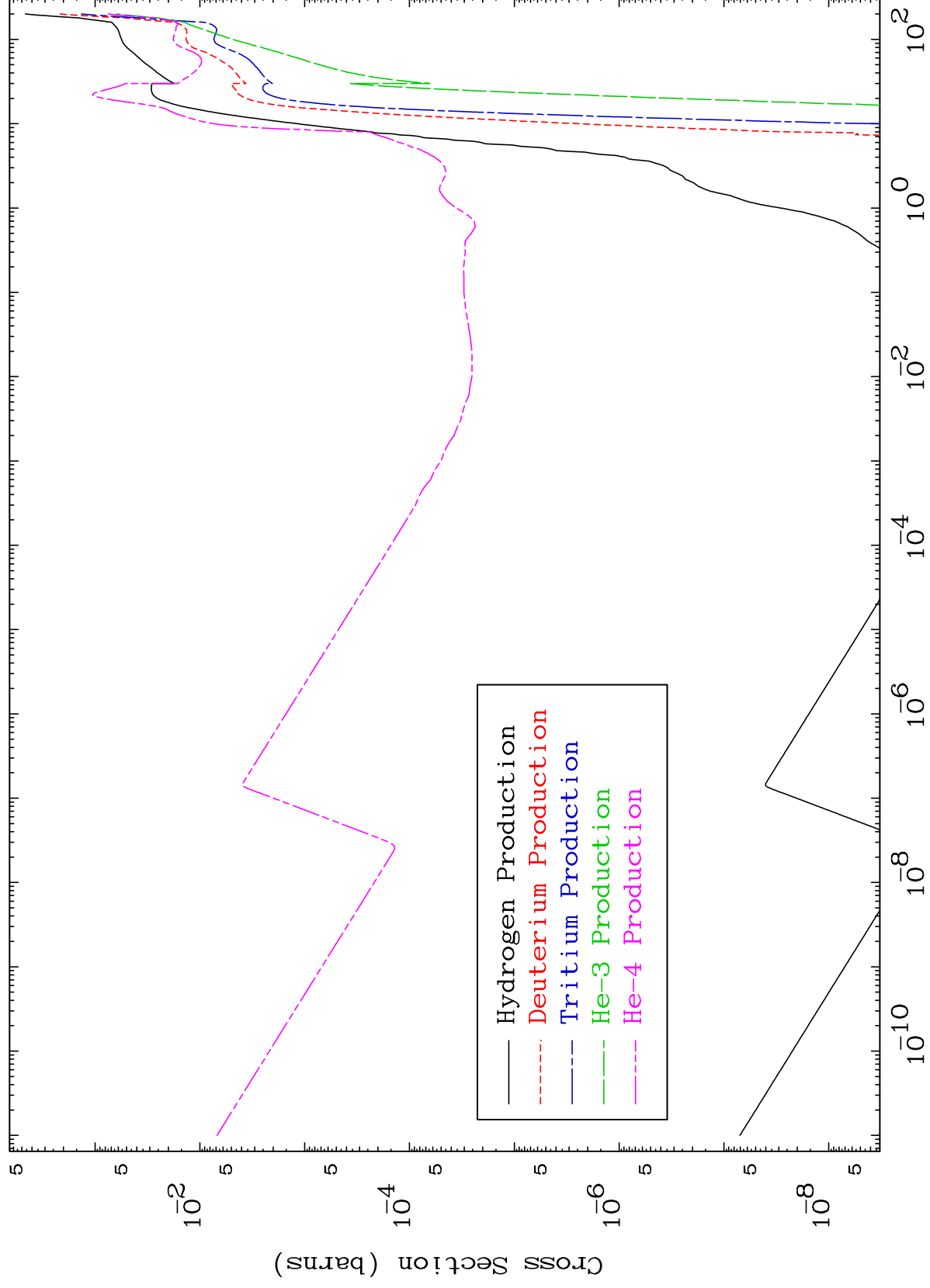




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Particle Production
293 Kelvin Cross Sections

97-Bk-241

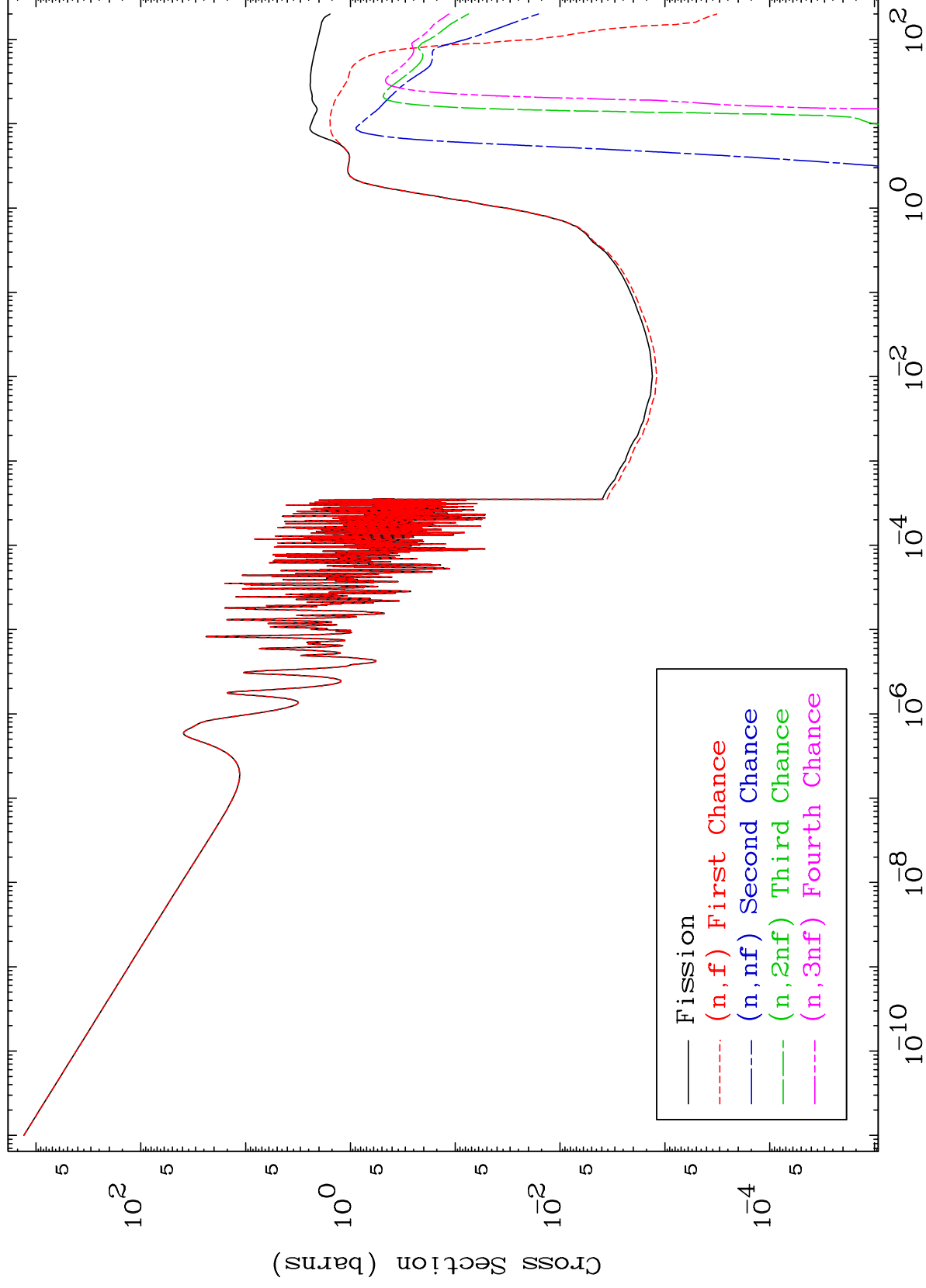


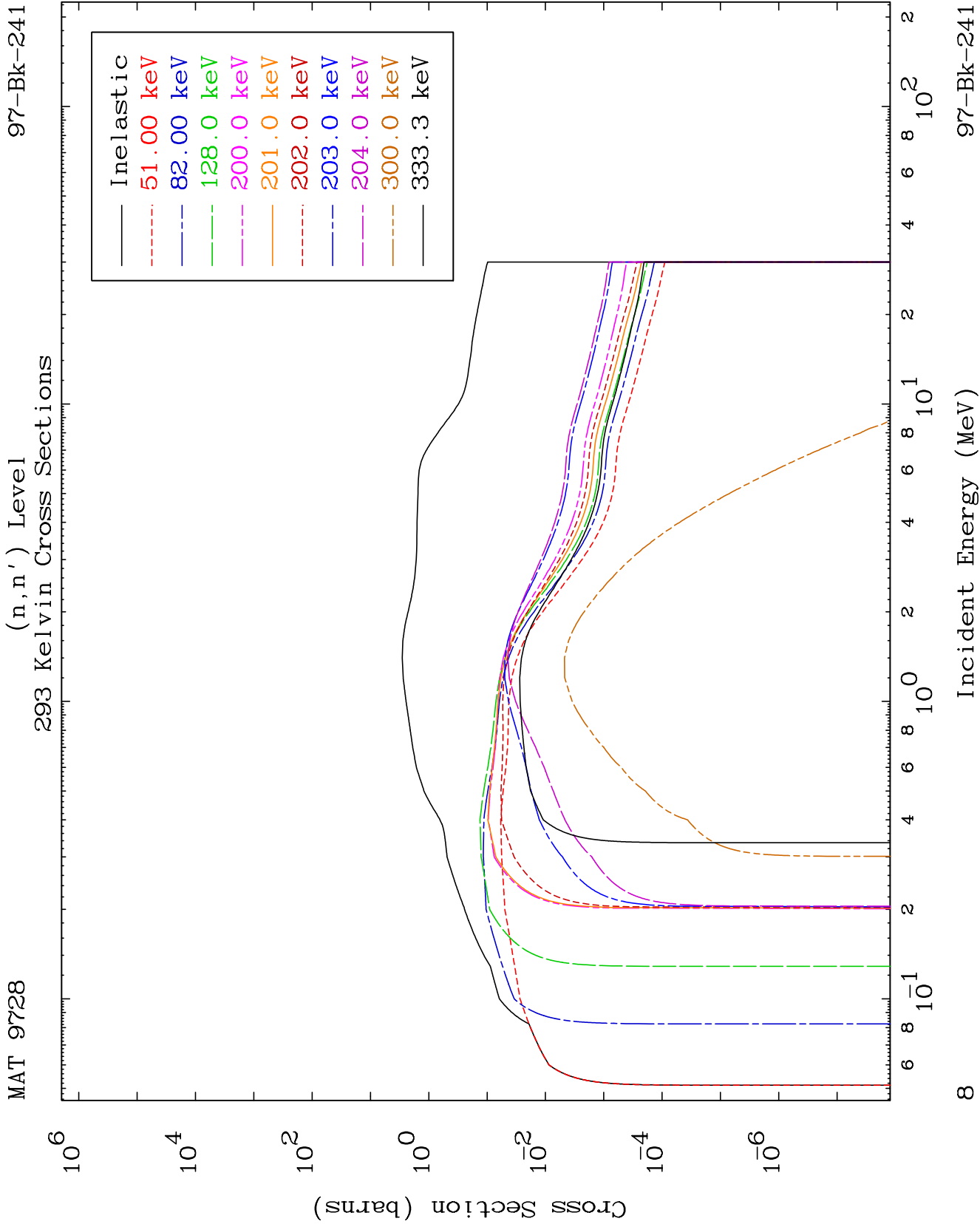
MAT 9728

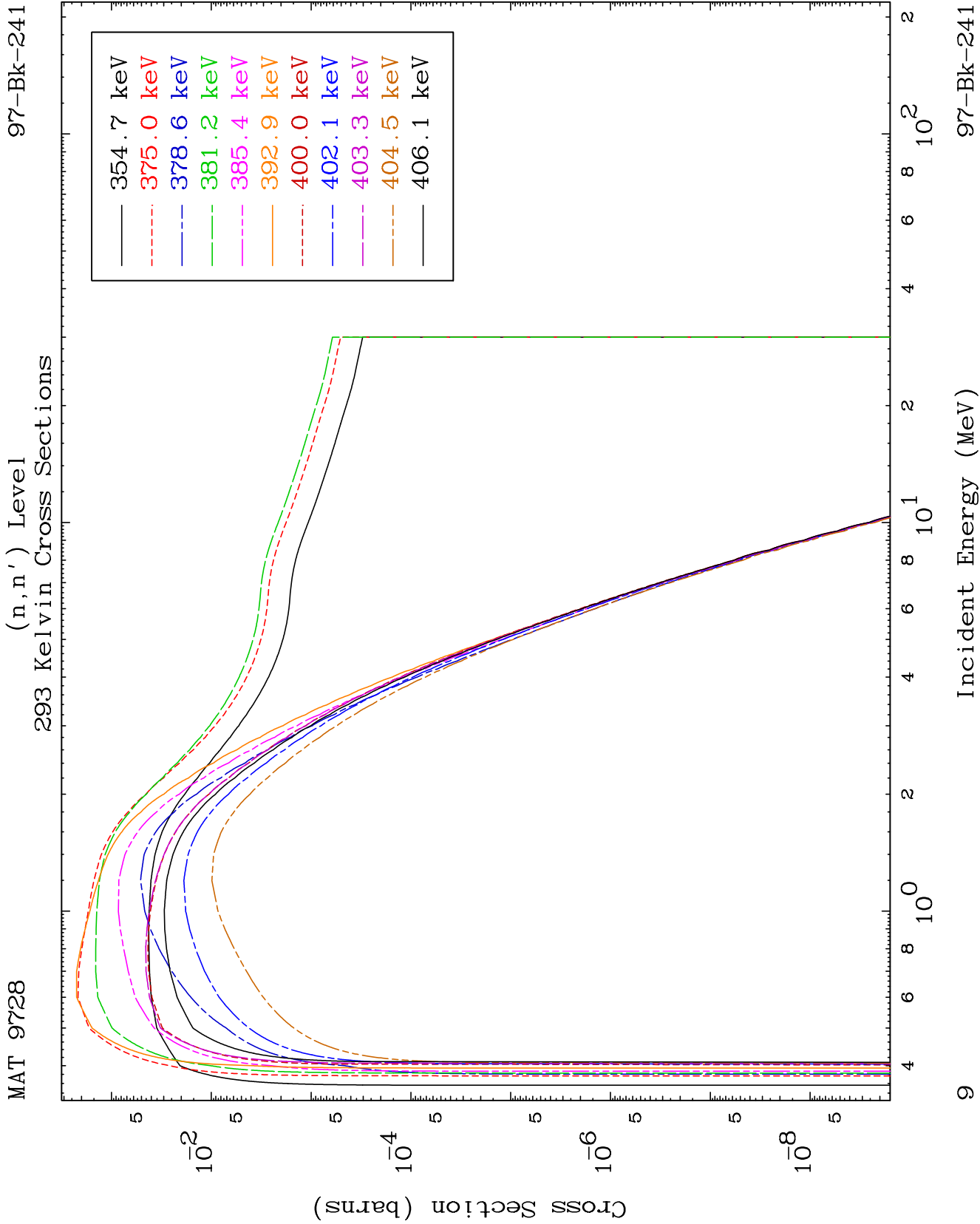
Fission

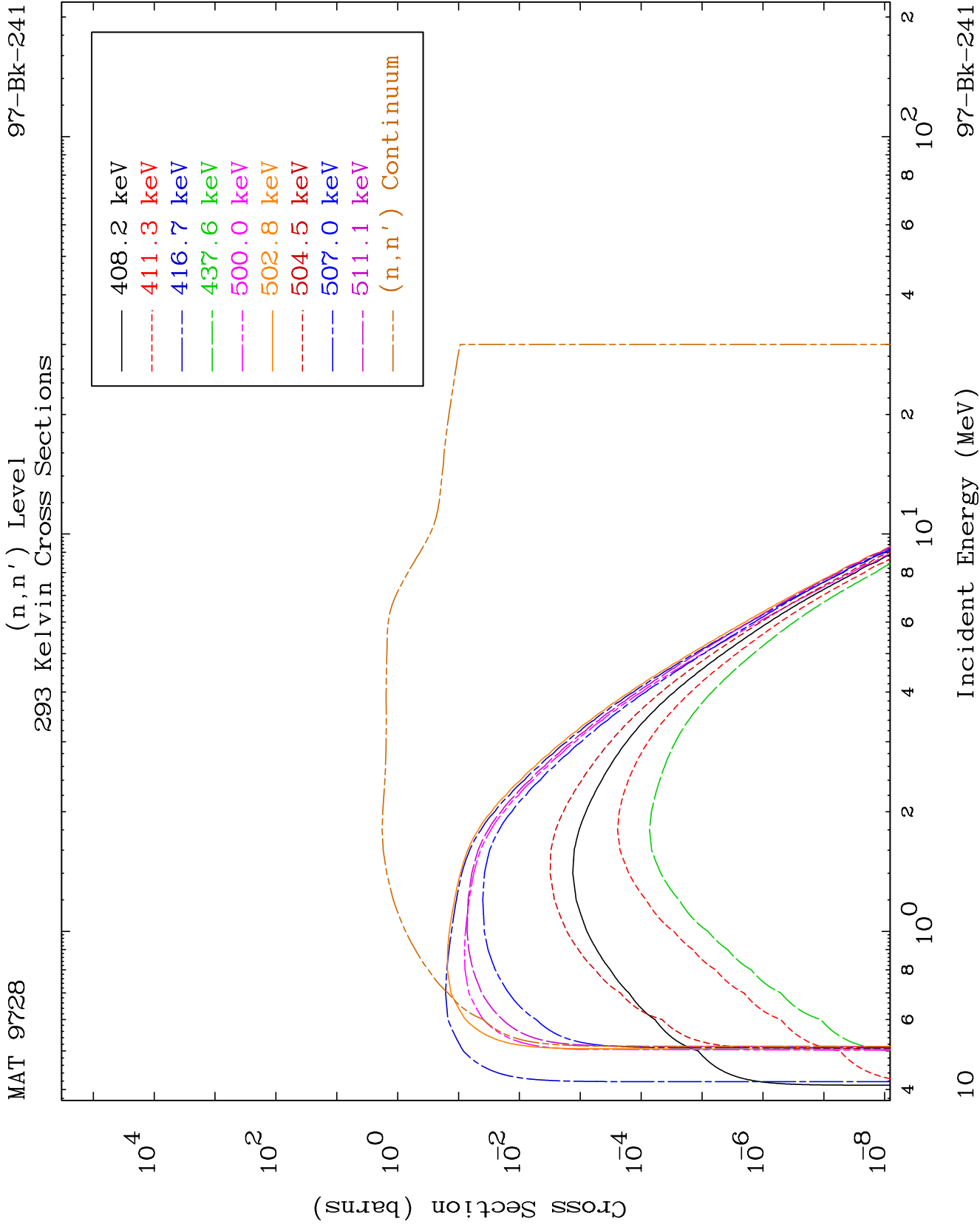
97-Bk-241

293 Kelvin Cross Sections





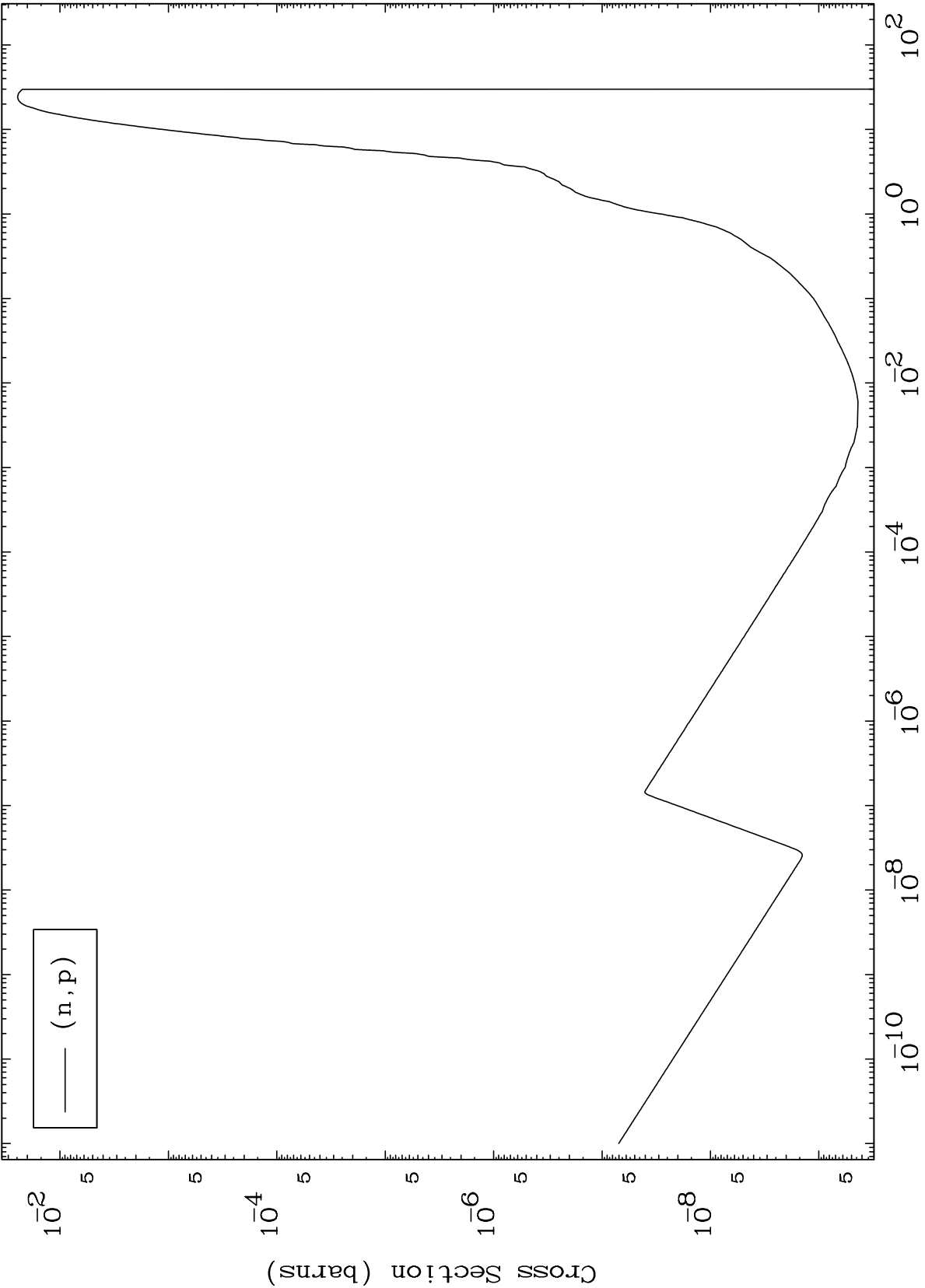




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(n,p) Levels
293 Kelvin Cross Sections

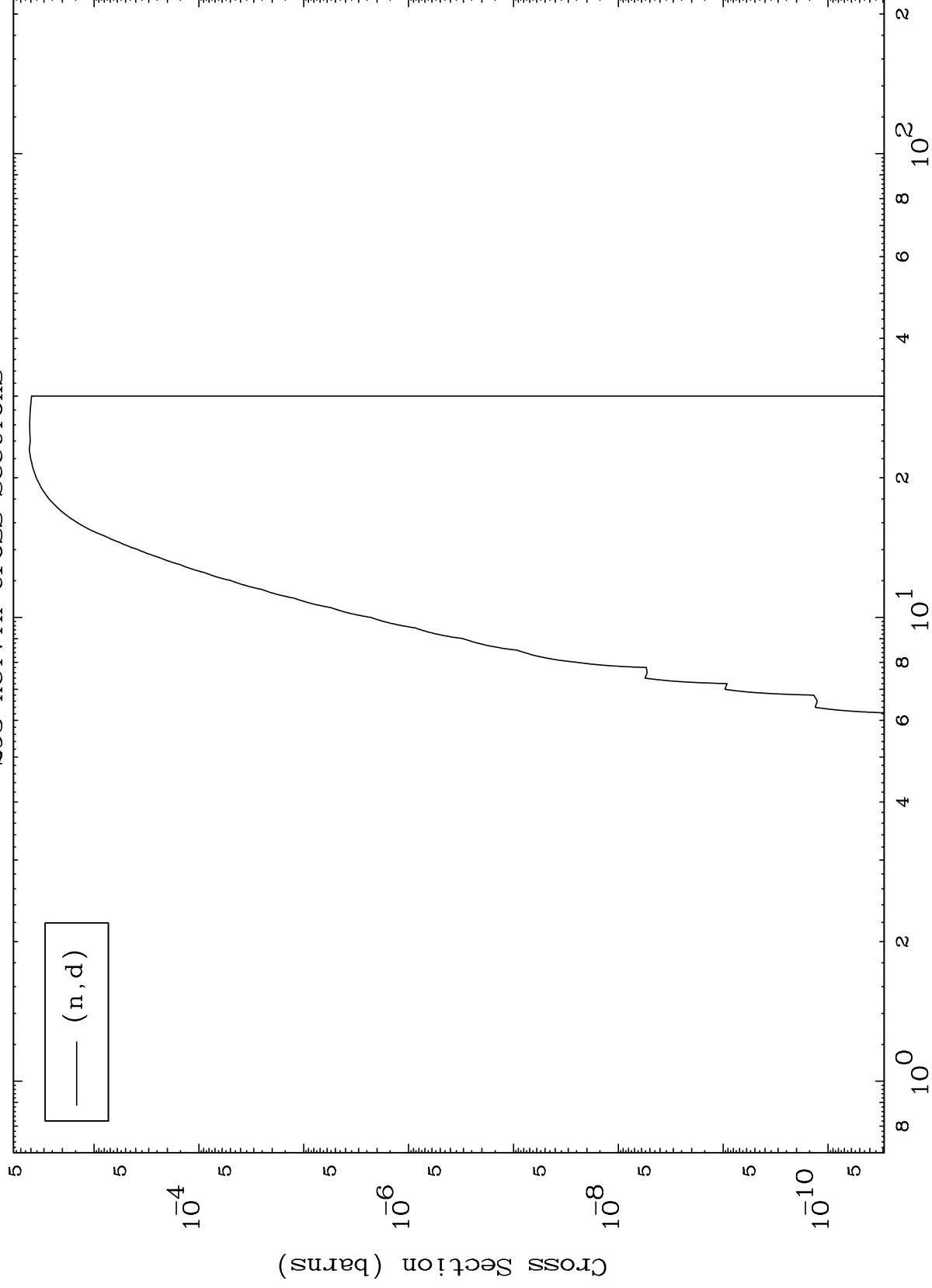
97-Bk-241



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97-Bk-241

(n,d) Levels
293 Kelvin Cross Sections



97-Bk-241

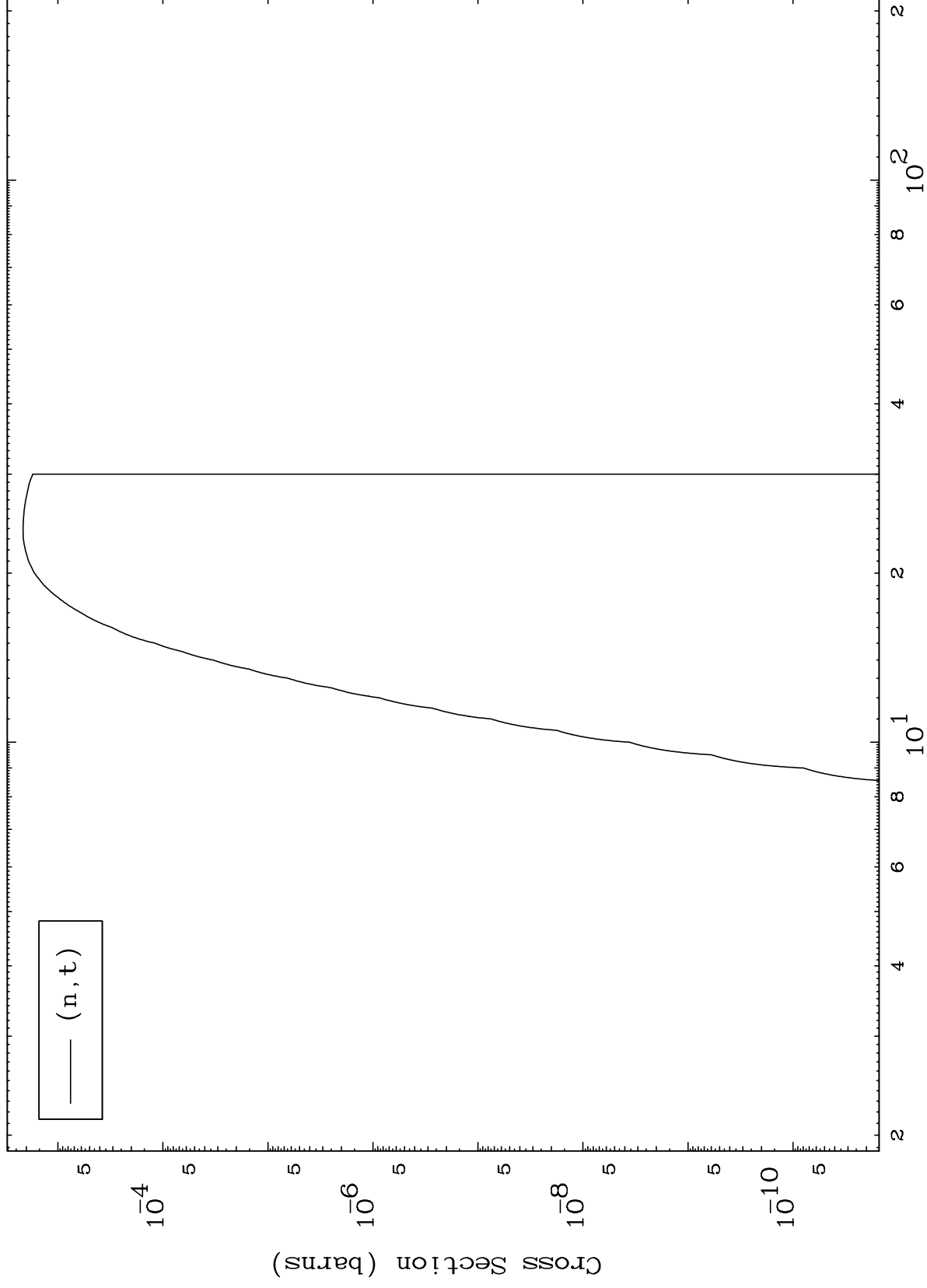
Incident Energy (MeV)

12

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97-Bk-241

(n,t) Levels
293 Kelvin Cross Sections



97-Bk-241

Incident Energy (MeV)

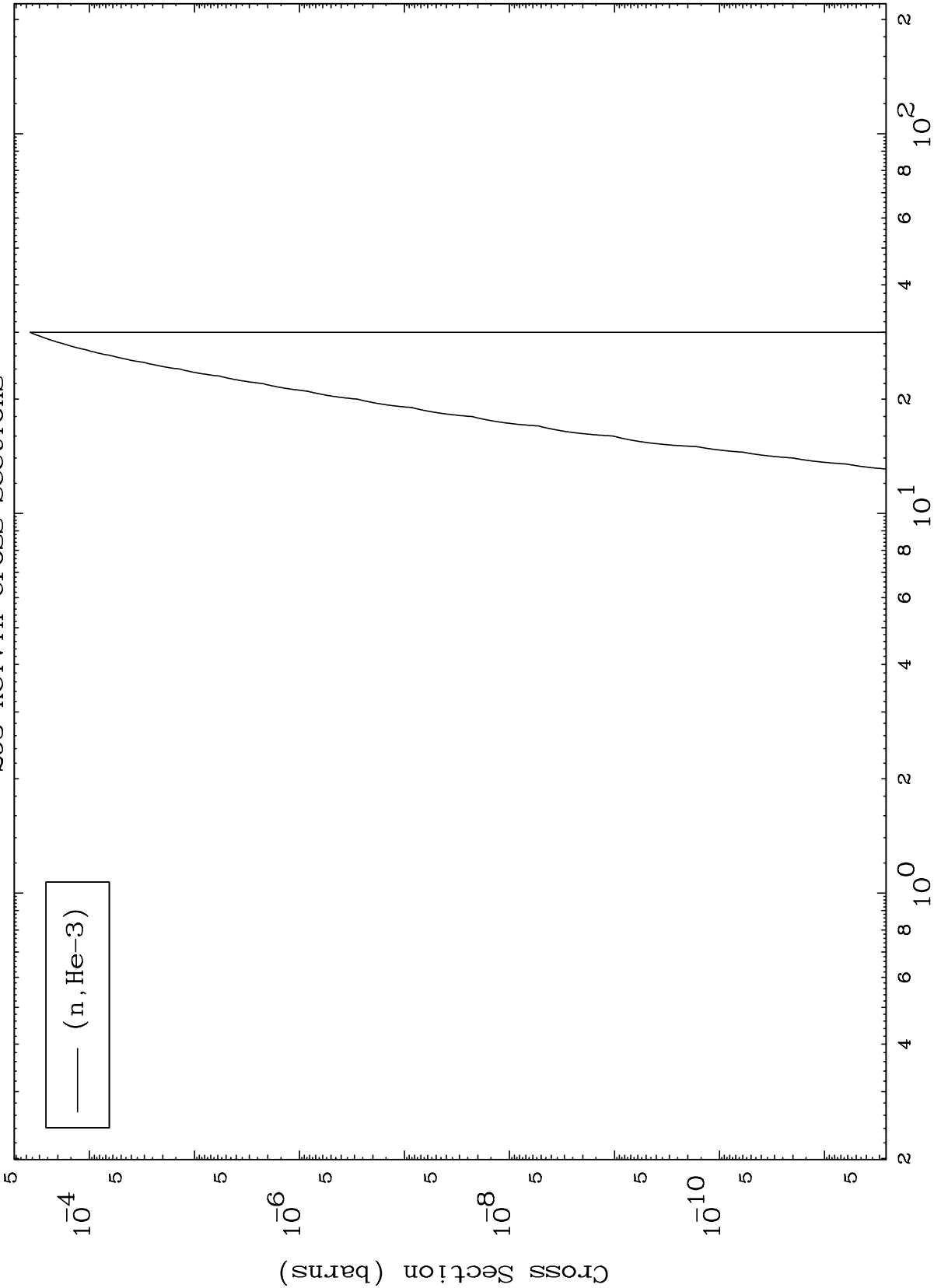
13

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(n,He3) Levels

97-Bk-241

293 Kelvin Cross Sections



Incident Energy (MeV)

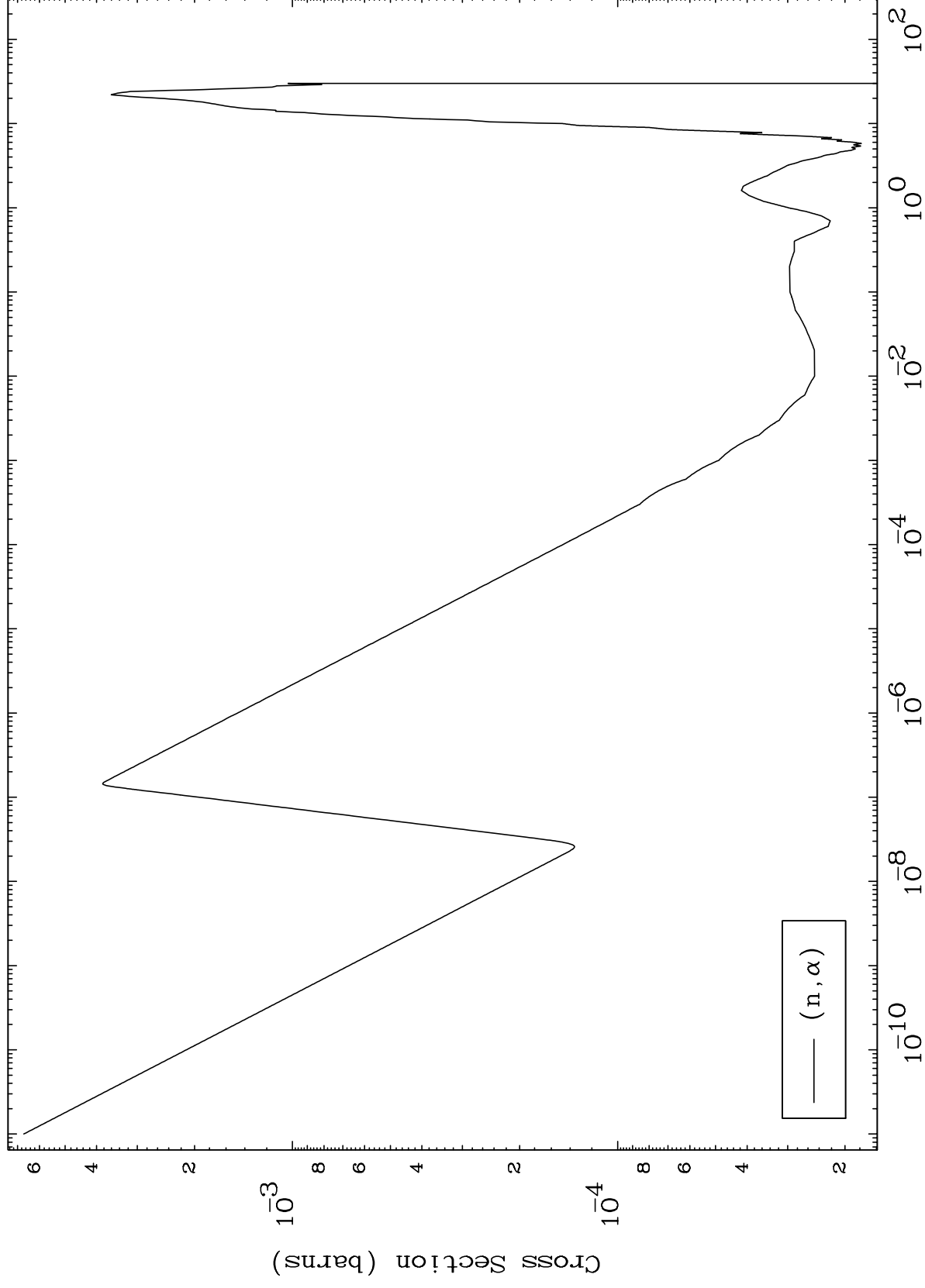
97-Bk-241

14

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(n, α) Levels
293 Kelvin Cross Sections

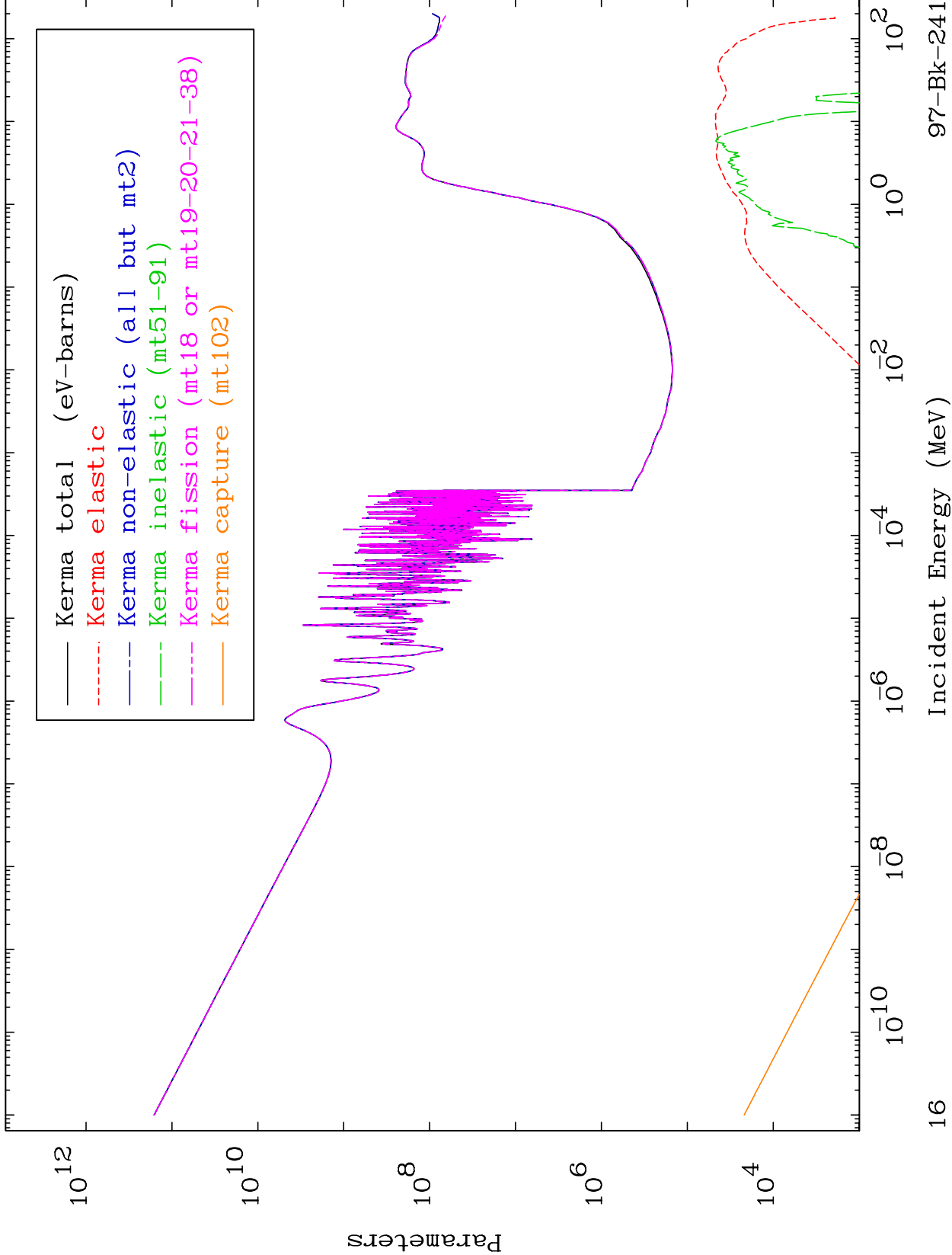
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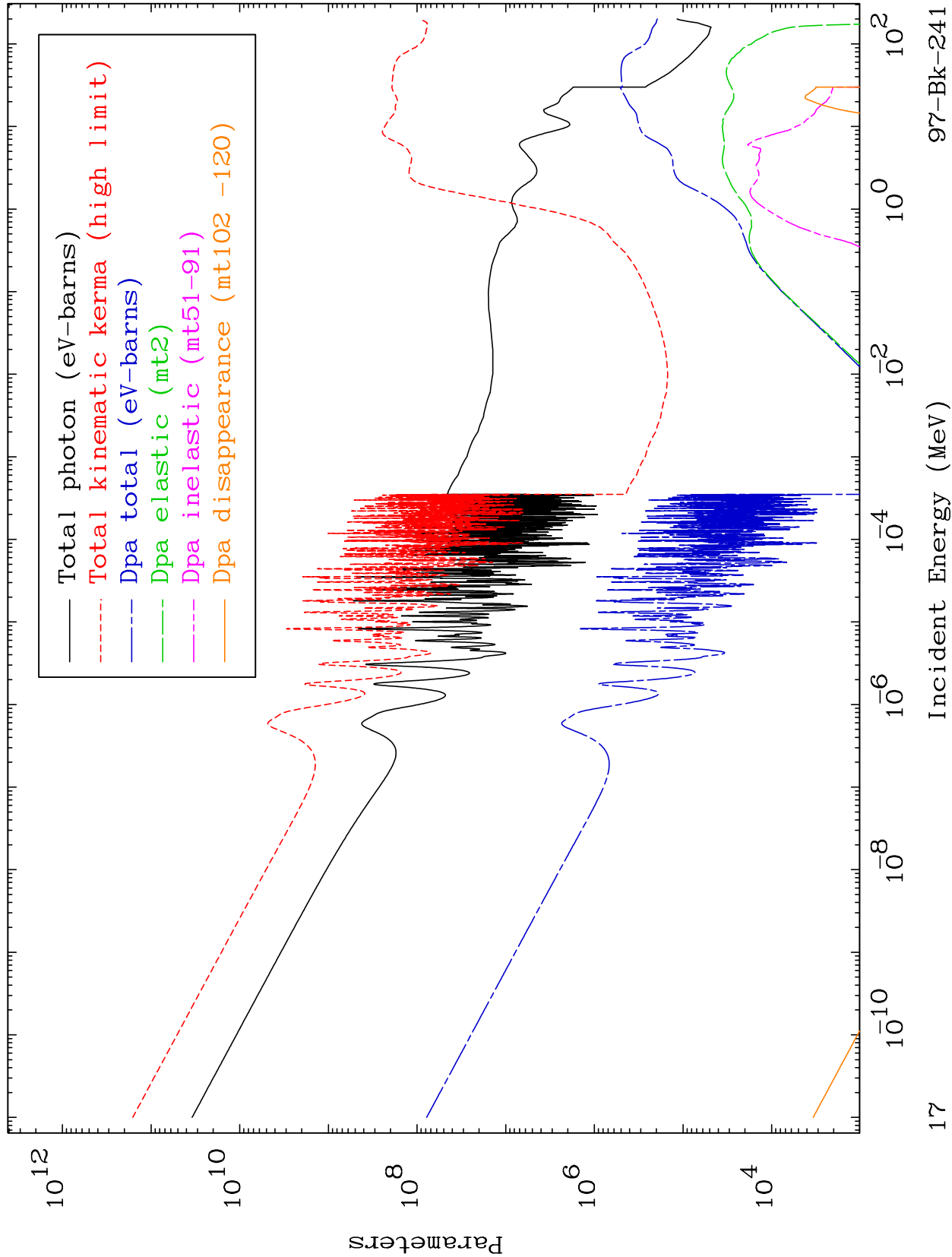


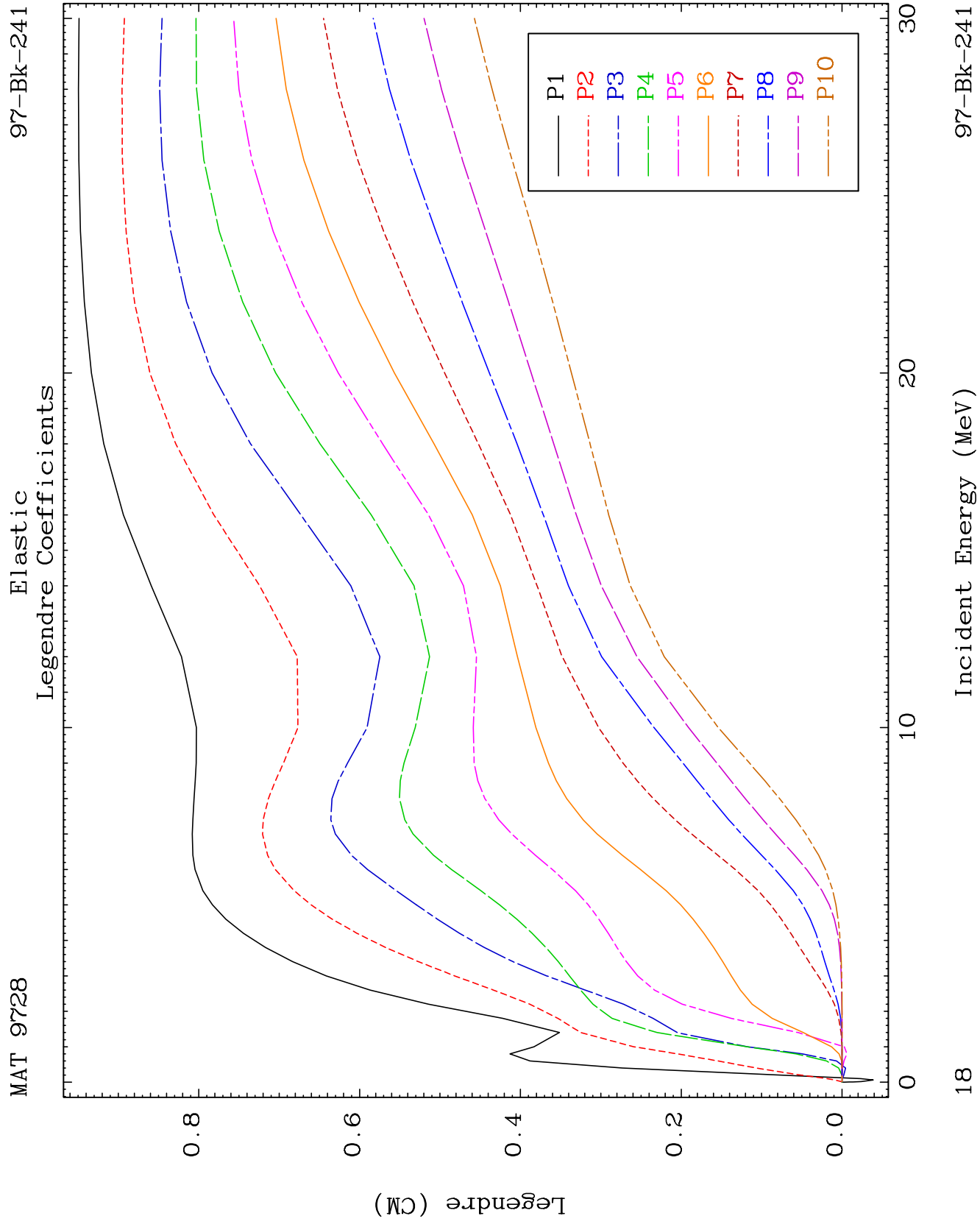
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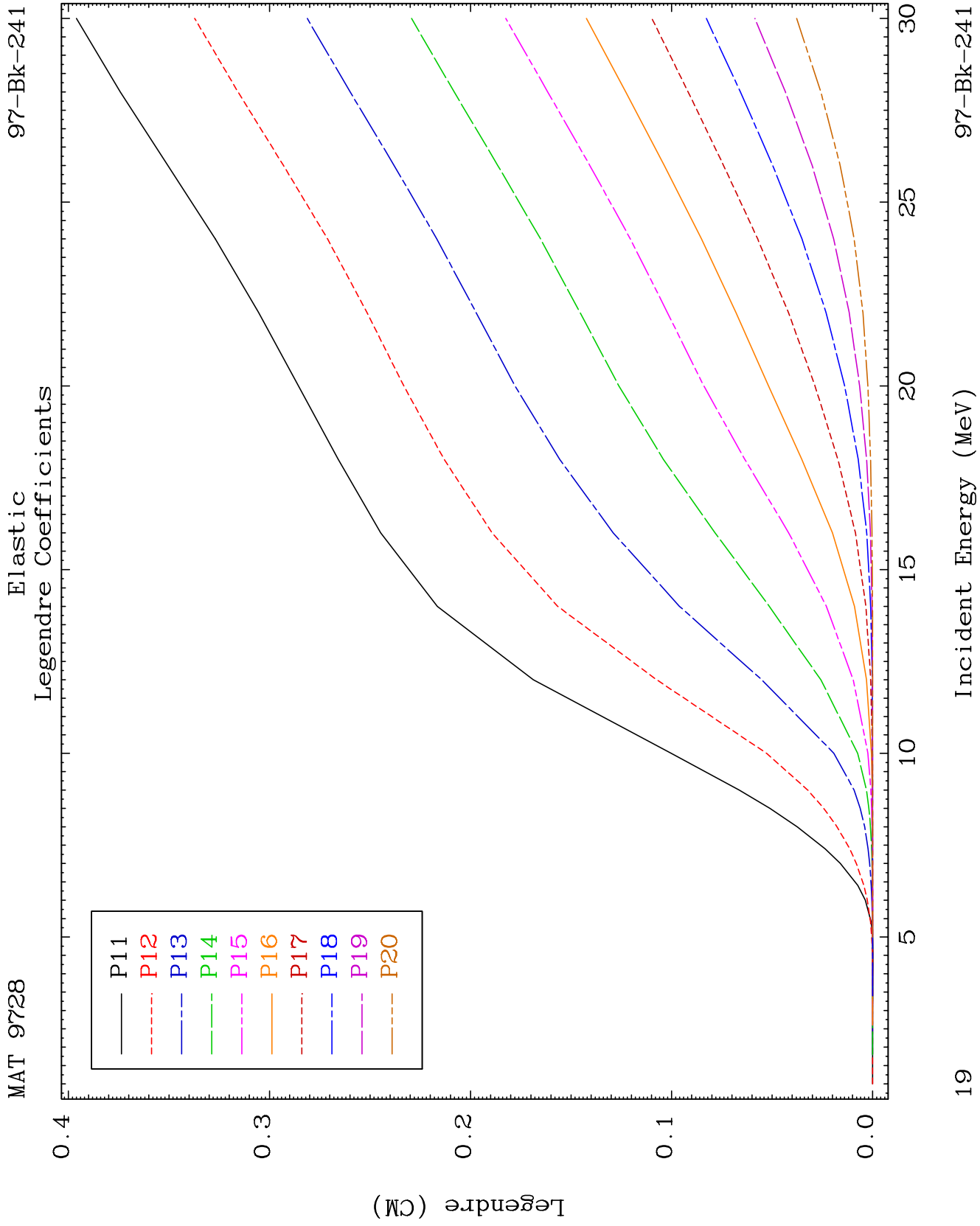
Incident Energy (MeV)

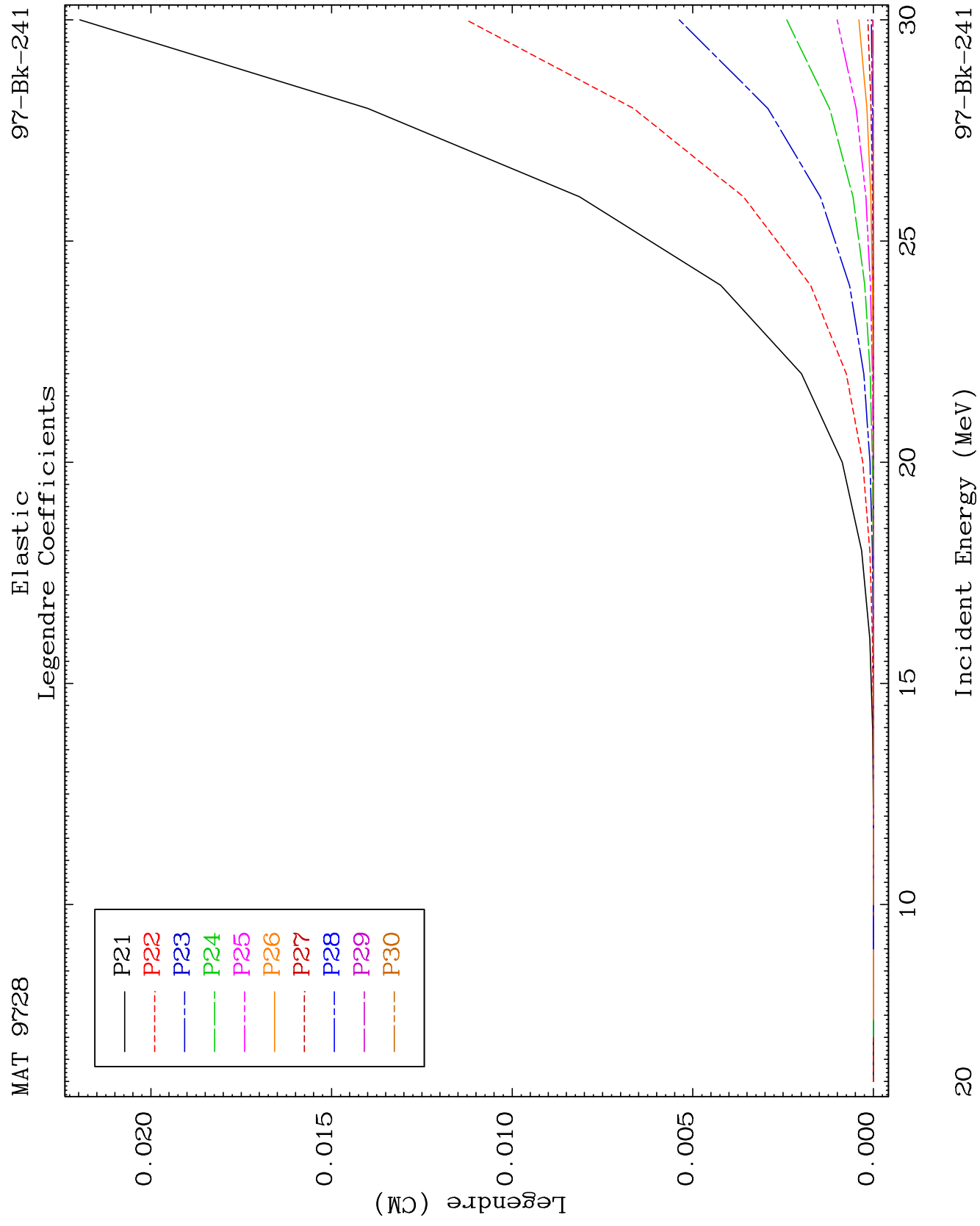
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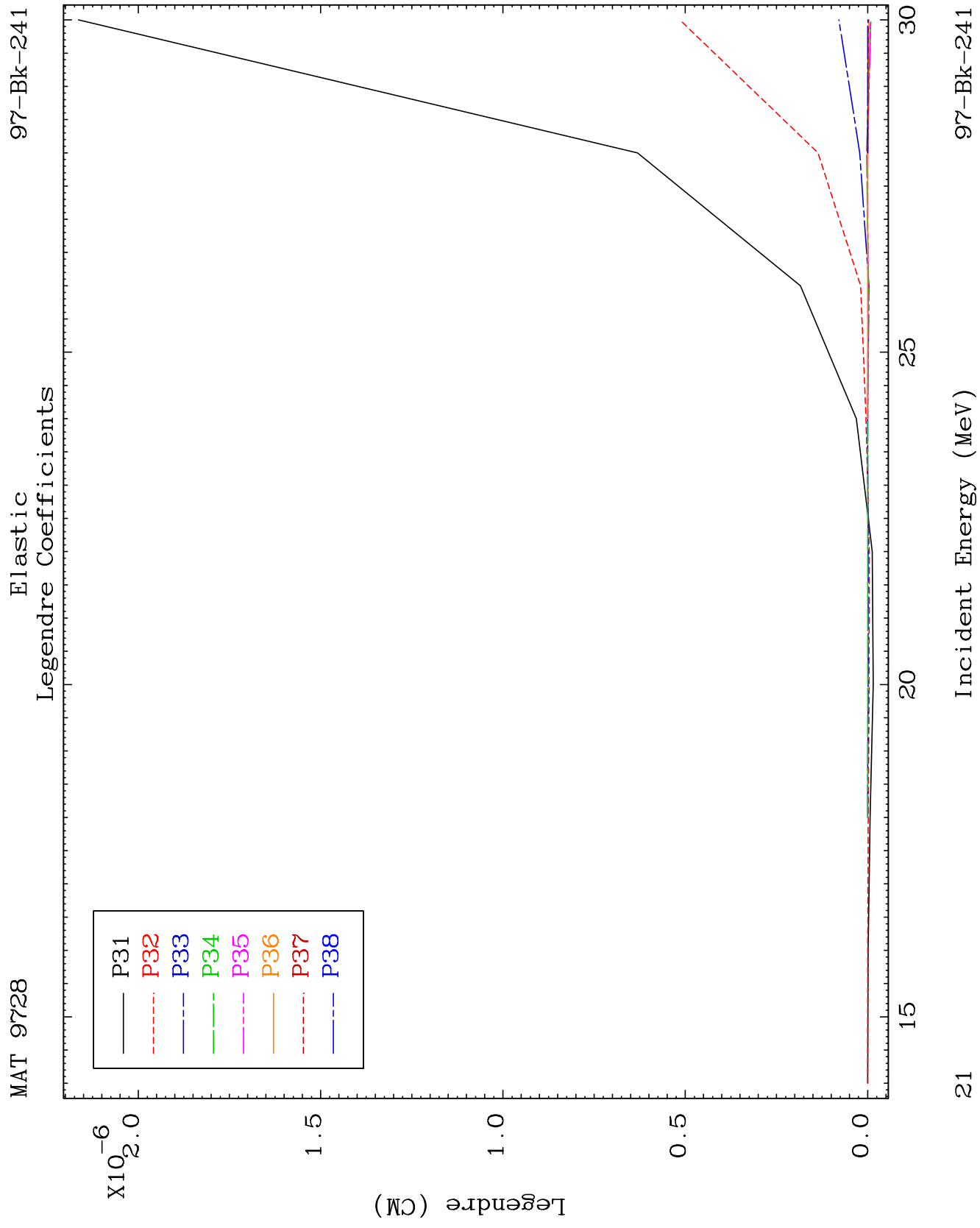


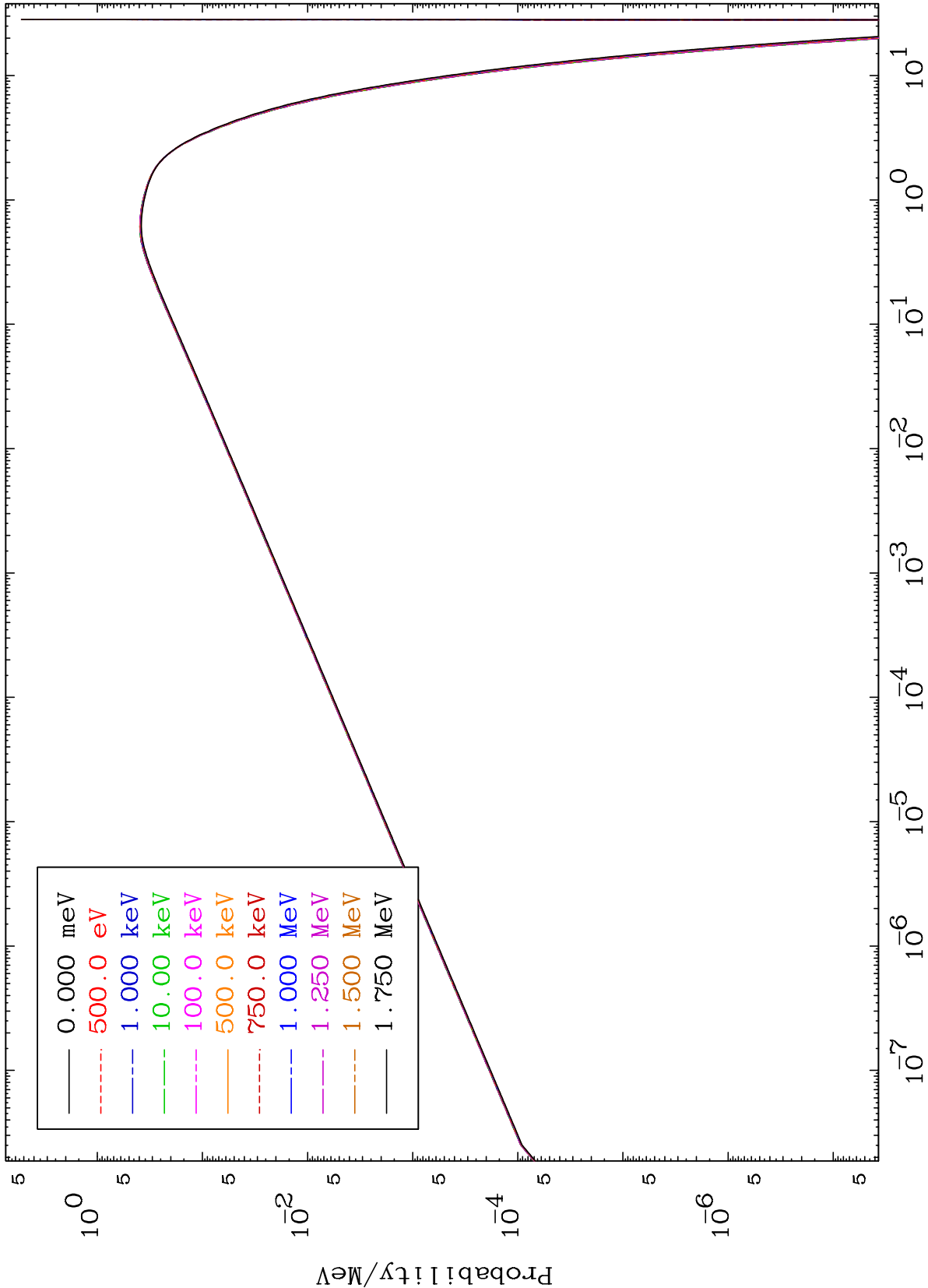


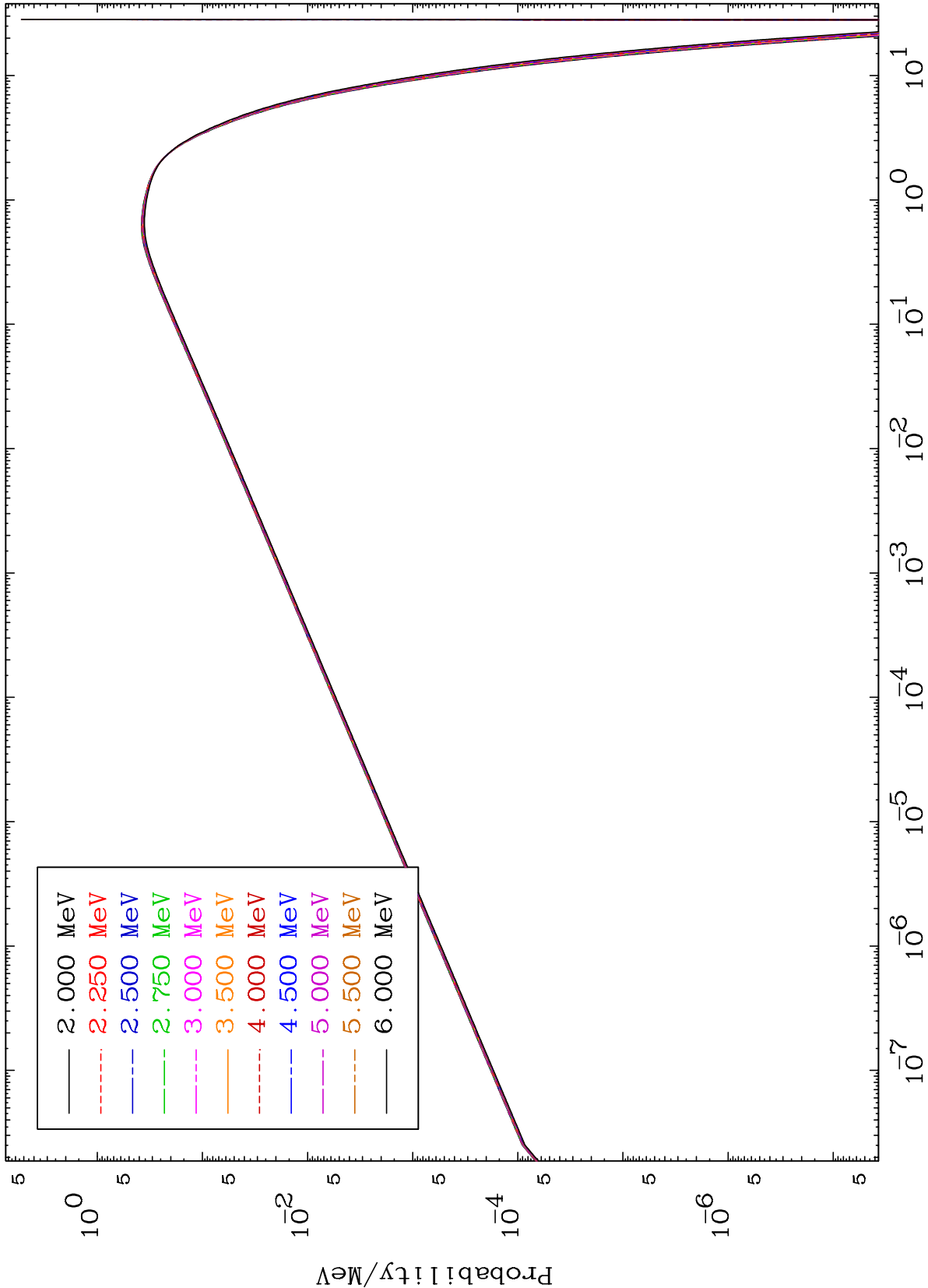




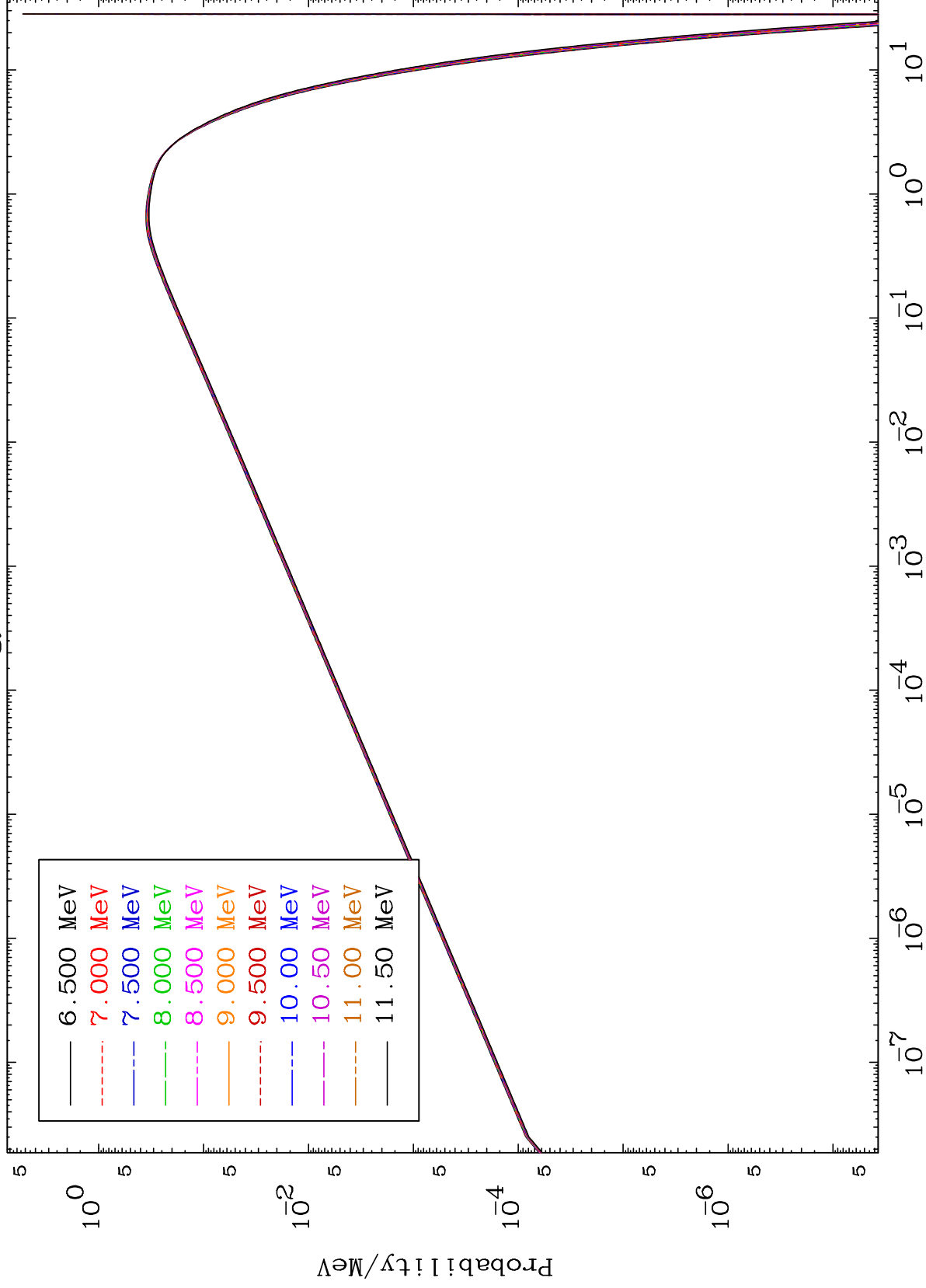


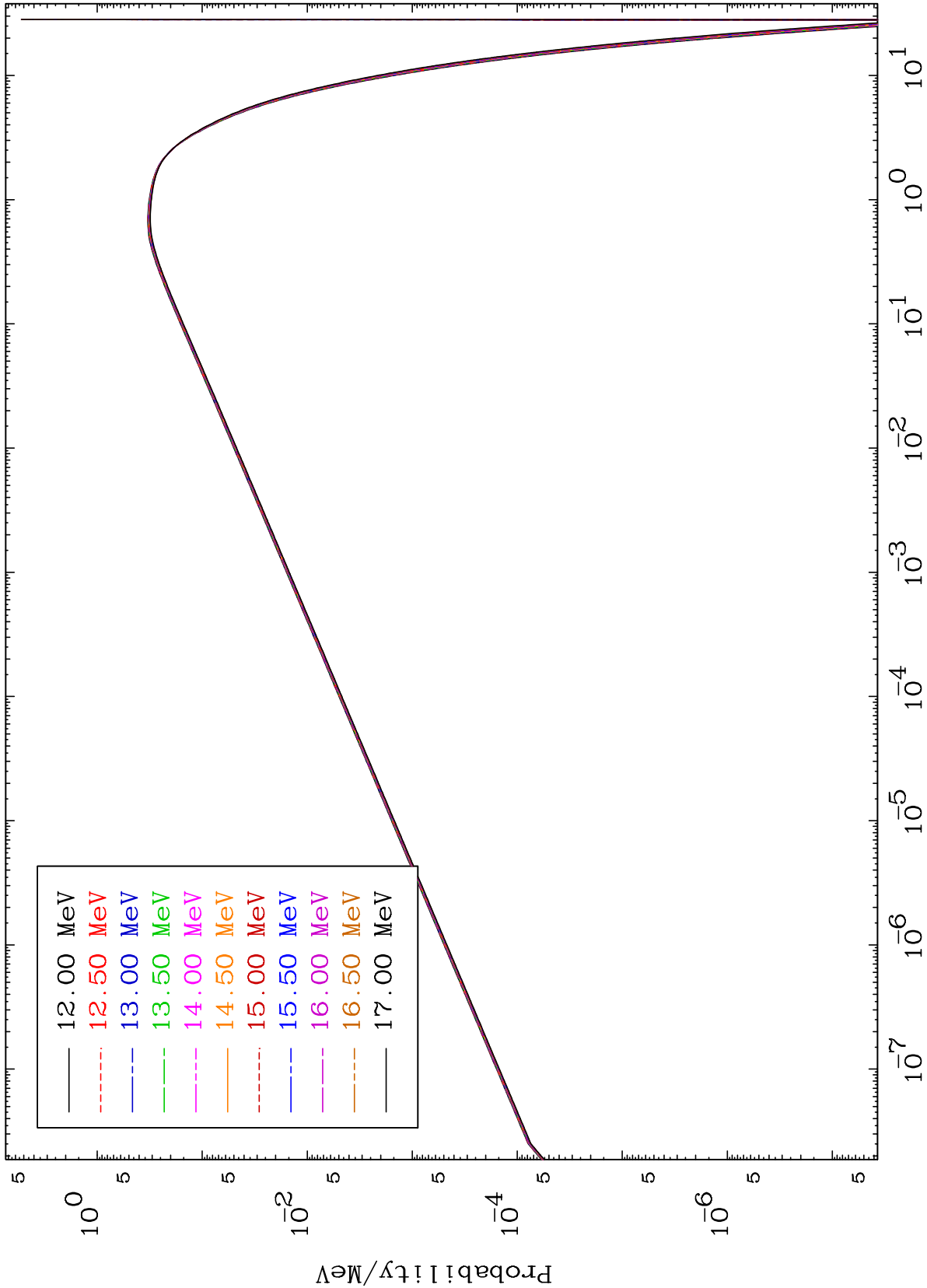




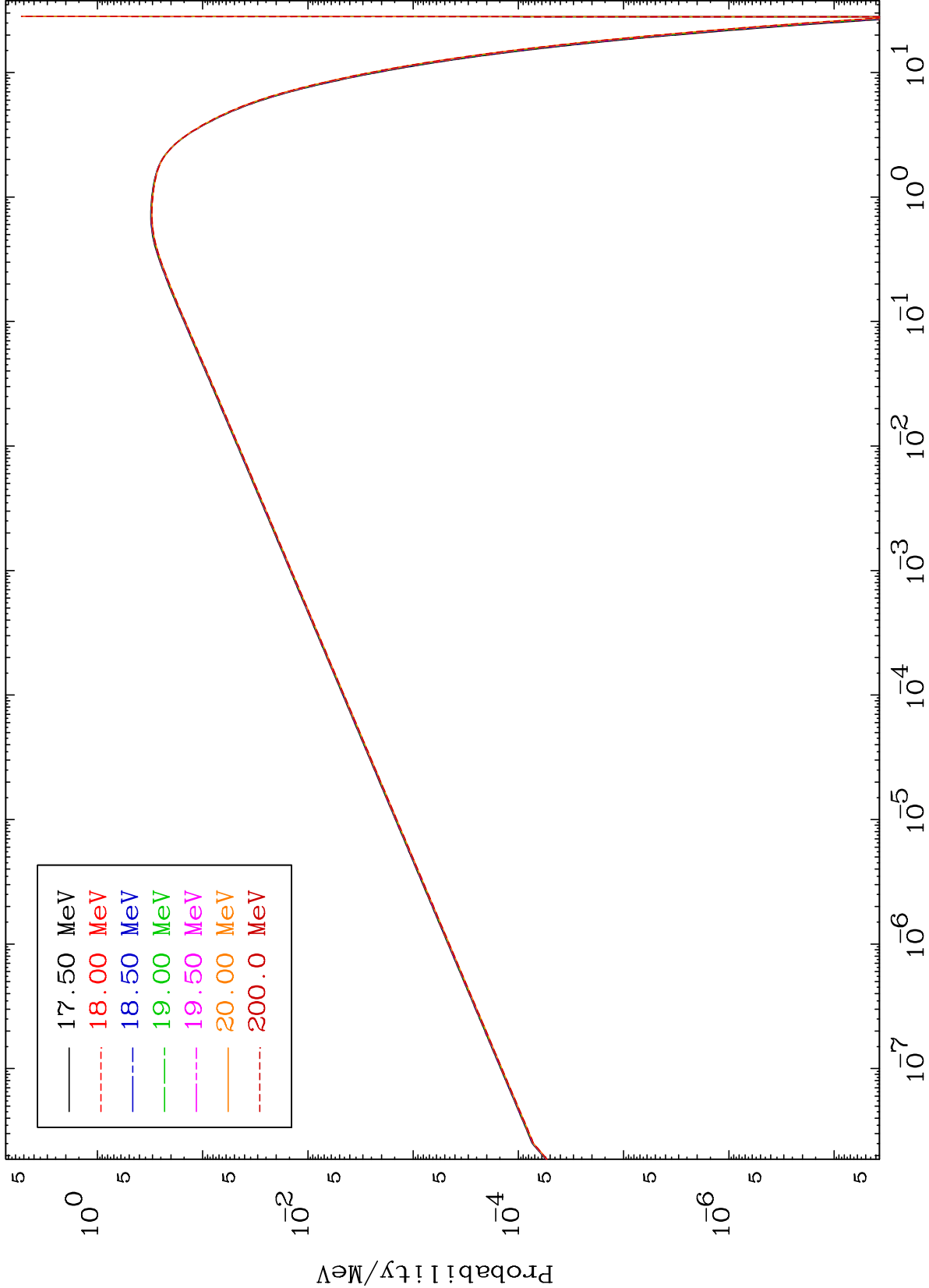


Fission
Energy Distribution





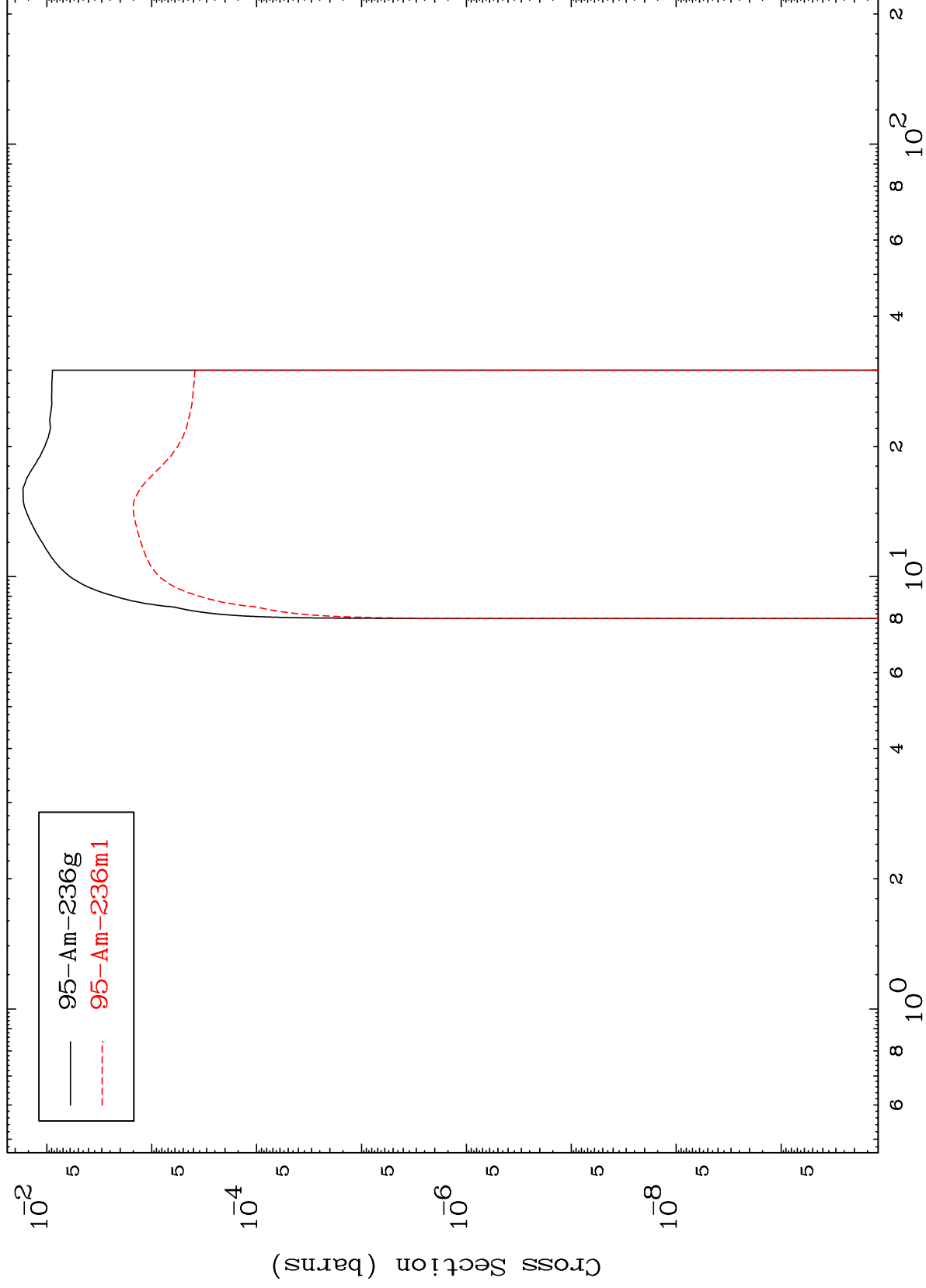
Fission
Energy Distribution



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97-Bk-241

$(n,2n) \alpha$
Radionuclide Production Cross Section



27

Incident Energy (MeV)

97-Bk-241

(n,p) α
Radionuclide Production Cross Section

