

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

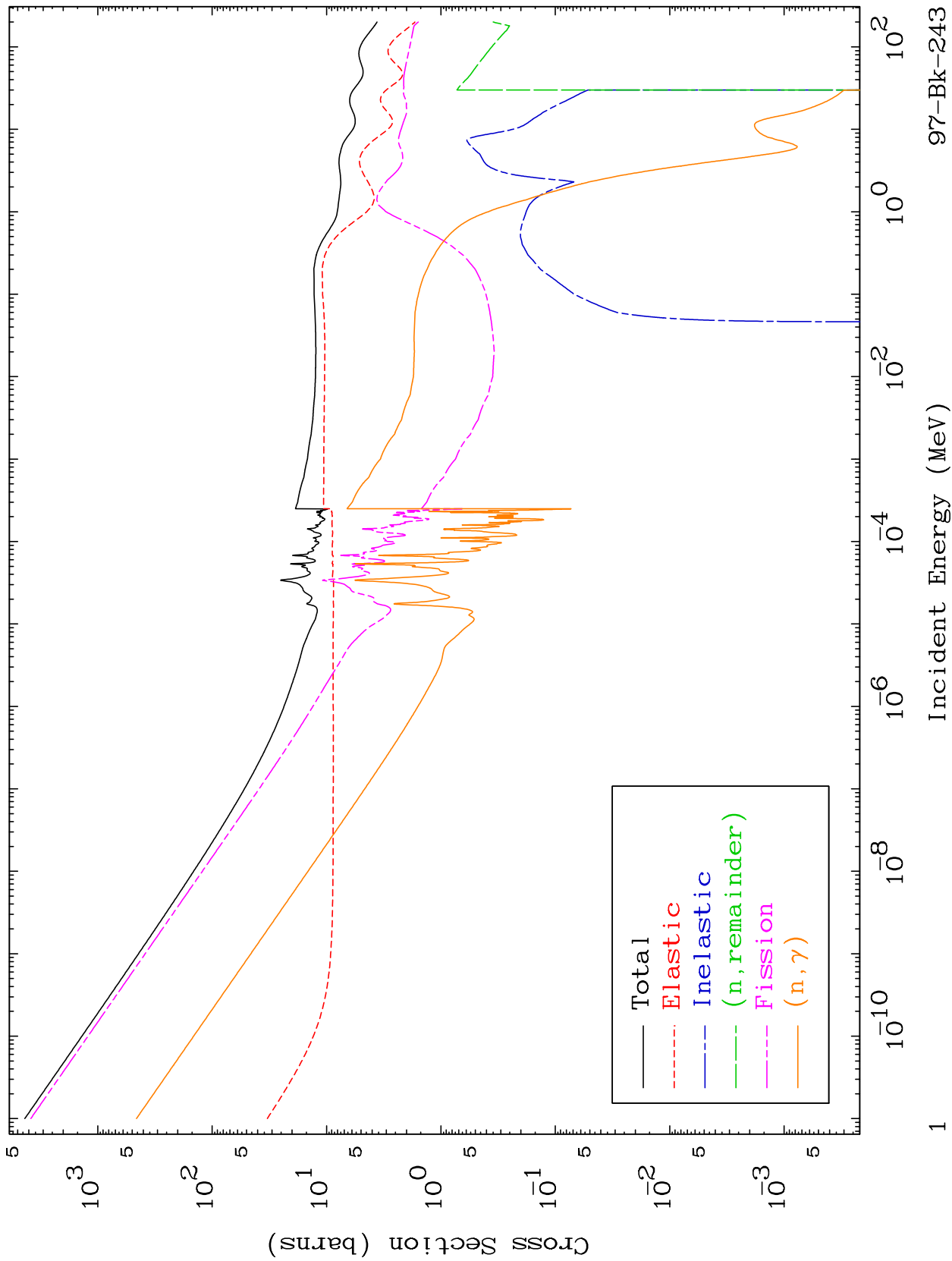
Press Mouse Button to Start

MAT 9734

Major

293 Kelvin Cross Sections

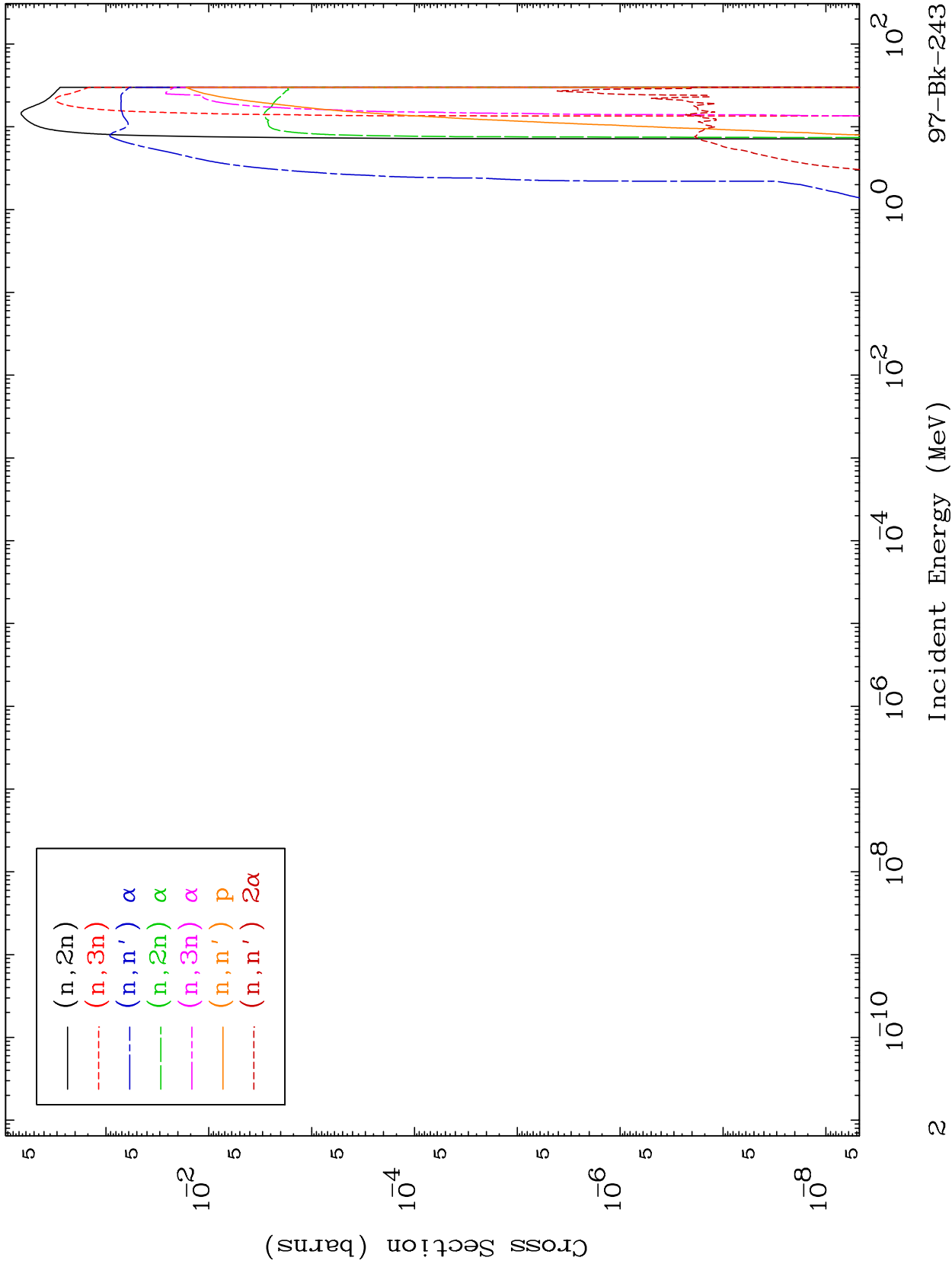
97-Bk-243



MAT 9734

Neutron Production
293 Kelvin Cross Sections

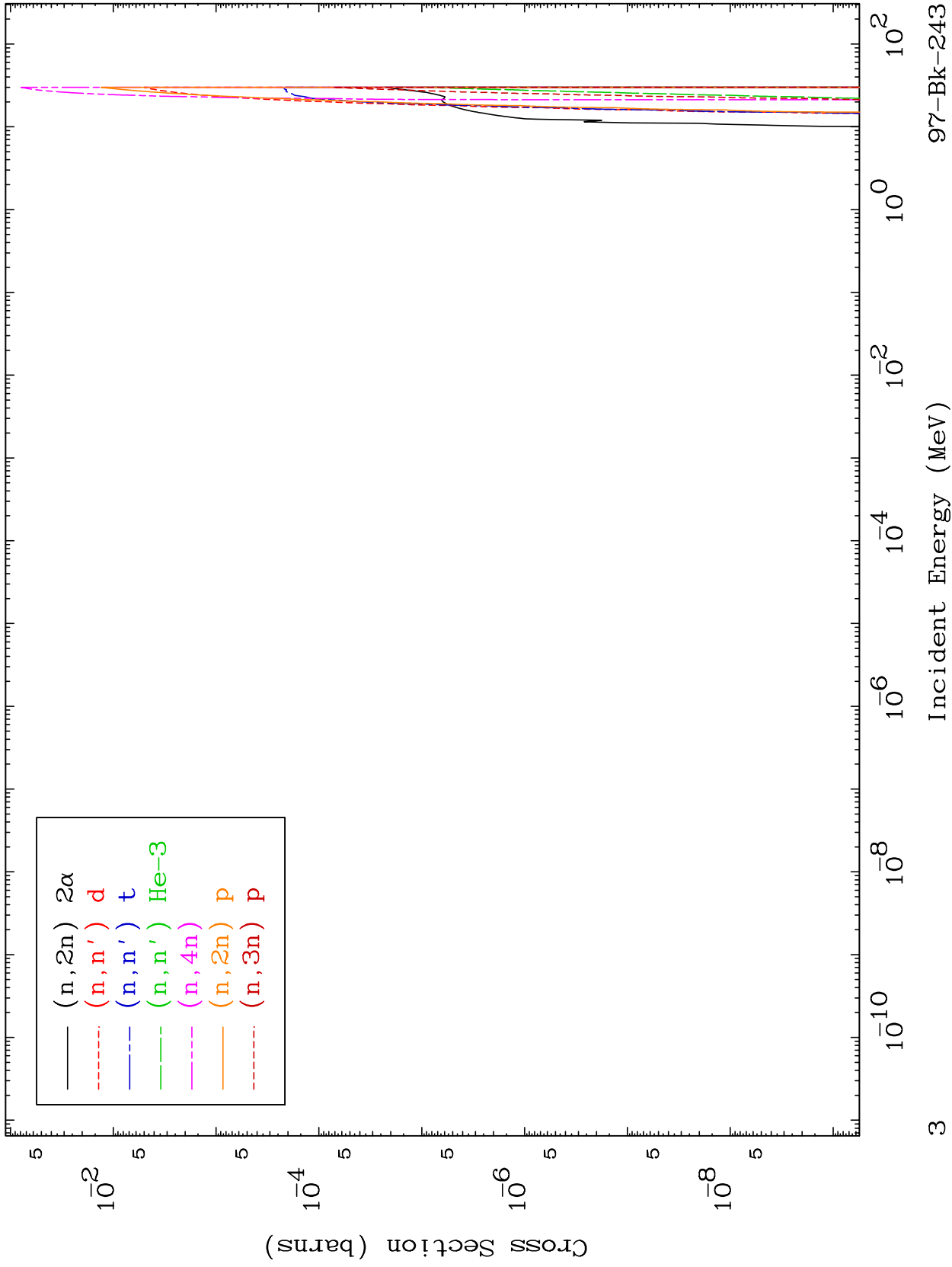
97-Bk-243



MAT 9734

Neutron Production
293 Kelvin Cross Sections

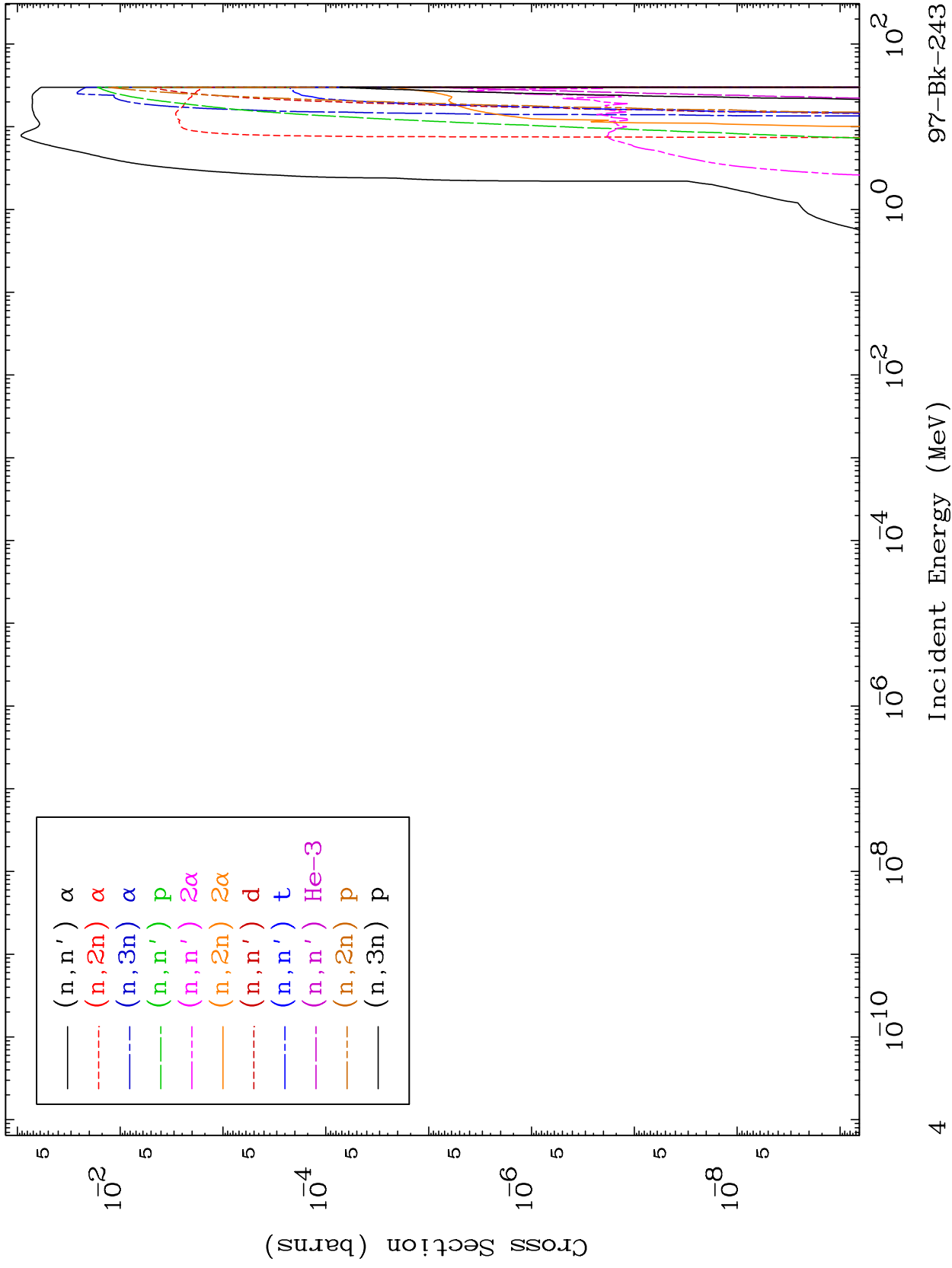
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MAT 9734

Charged Particle
293 Kelvin Cross Sections

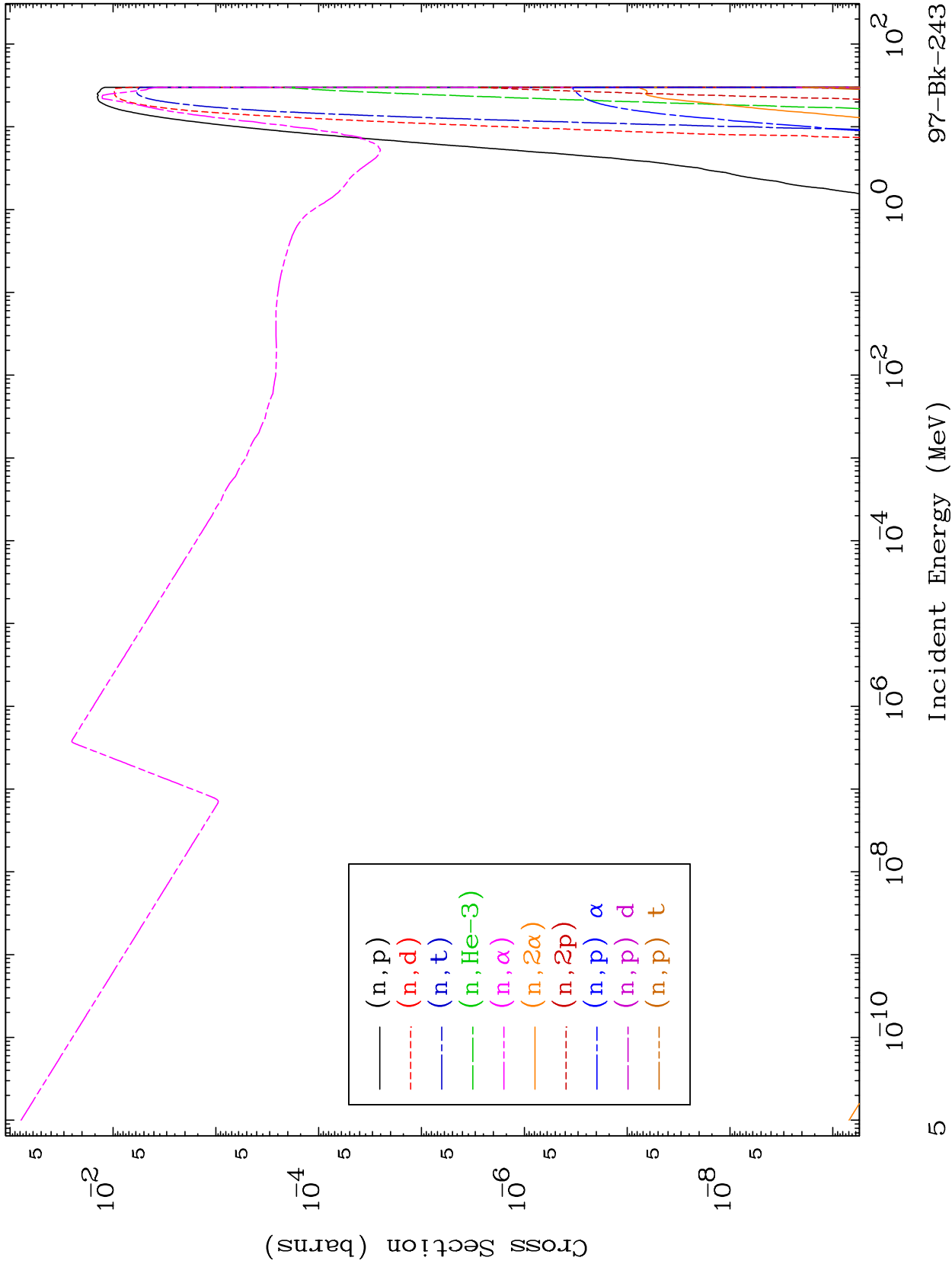
97-Bk-243



MAT 9734

Charged Particle
293 Kelvin Cross Sections

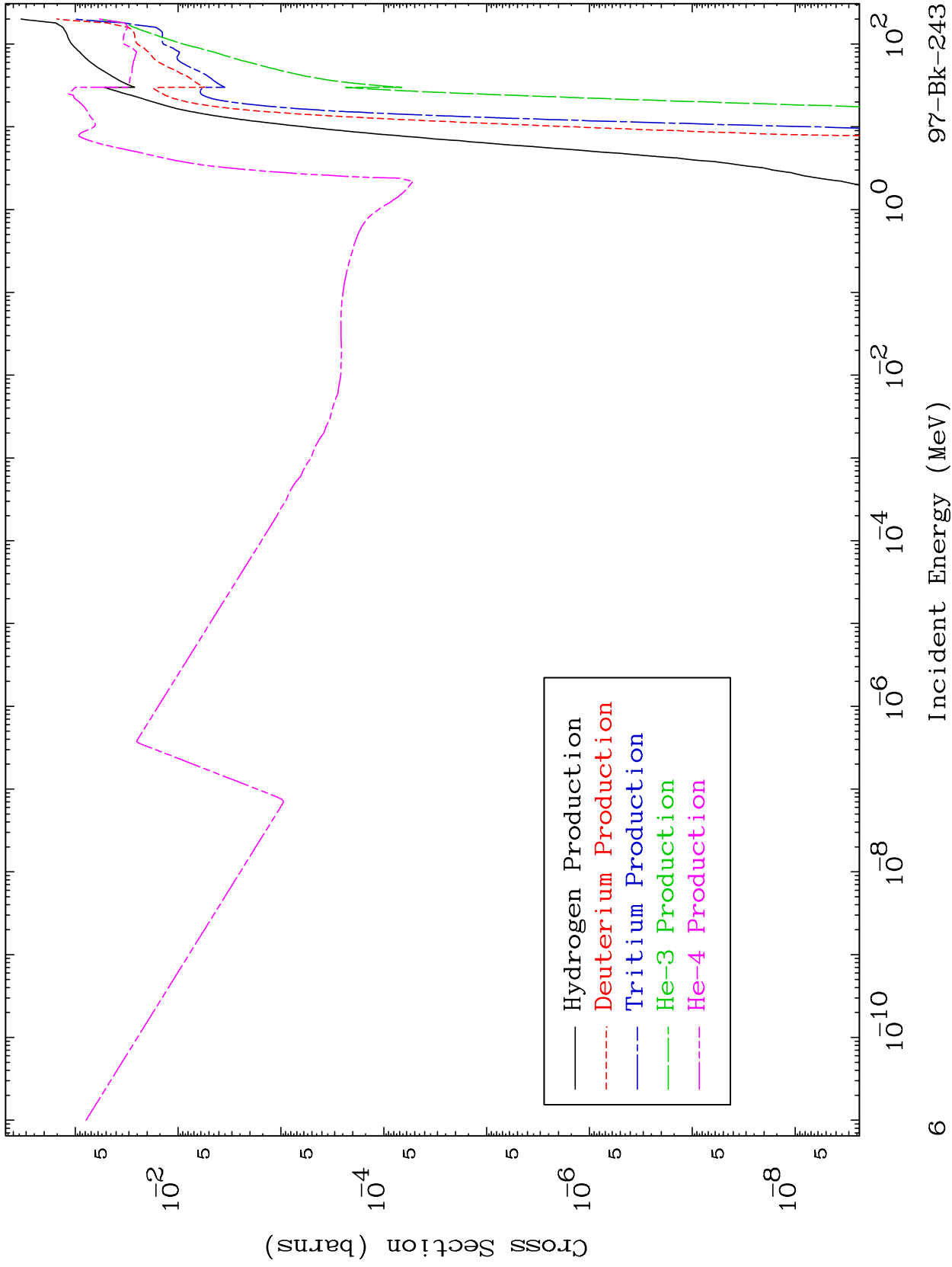
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Particle Production
293 Kelvin Cross Sections

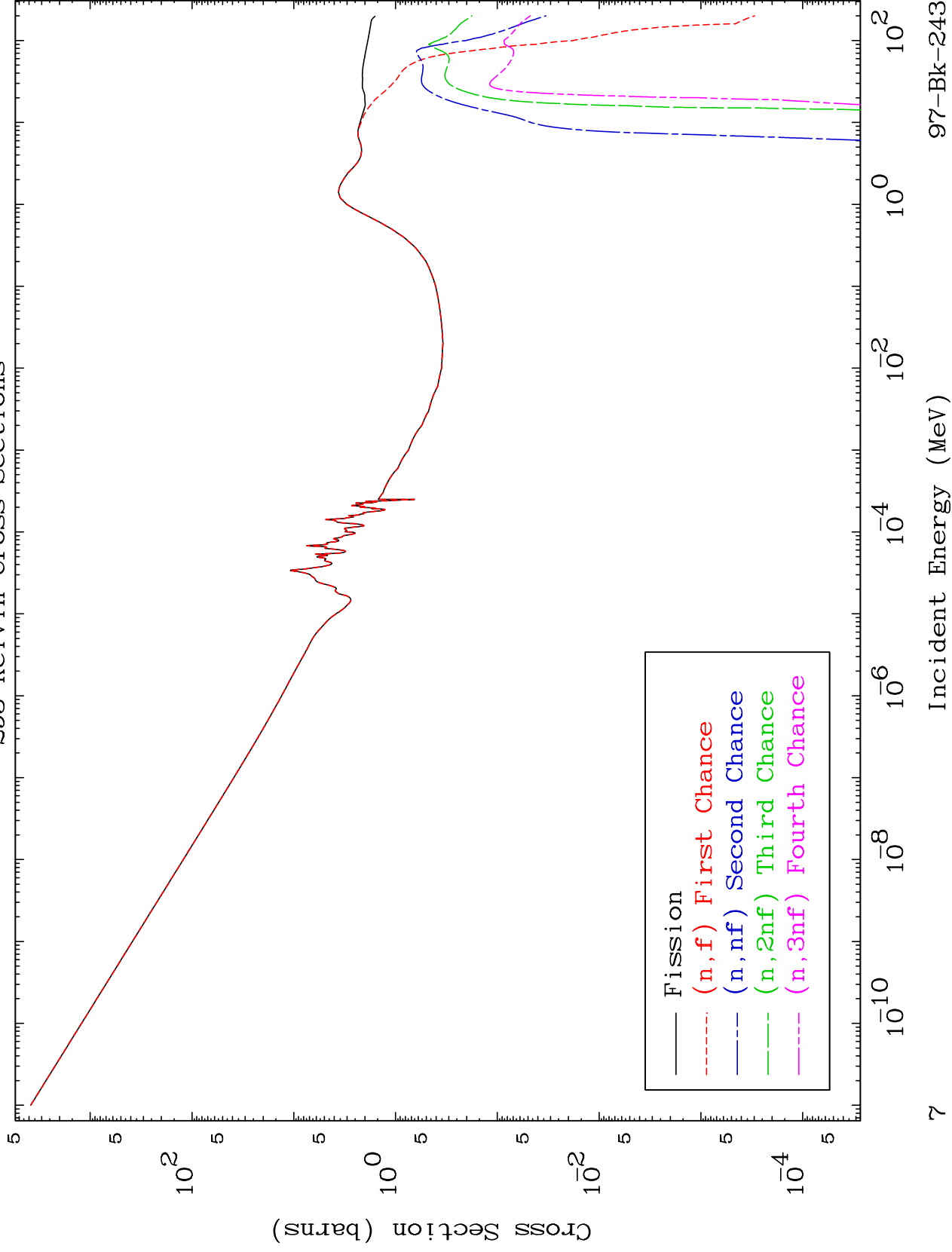
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Fission
293 Kelvin Cross Sections

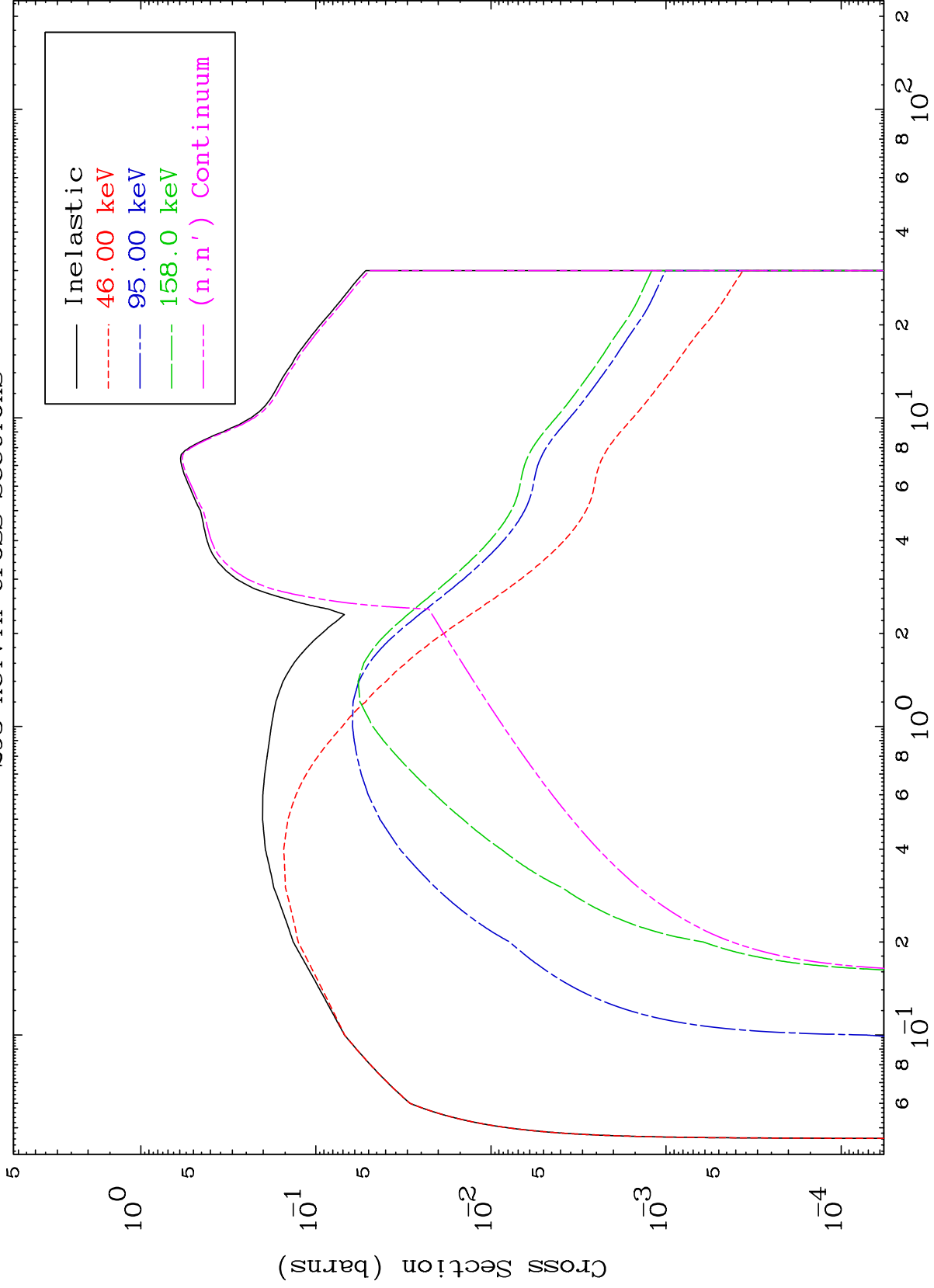
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MAT 9734

(n,n') Level
293 Kelvin Cross Sections

97-Bk-243



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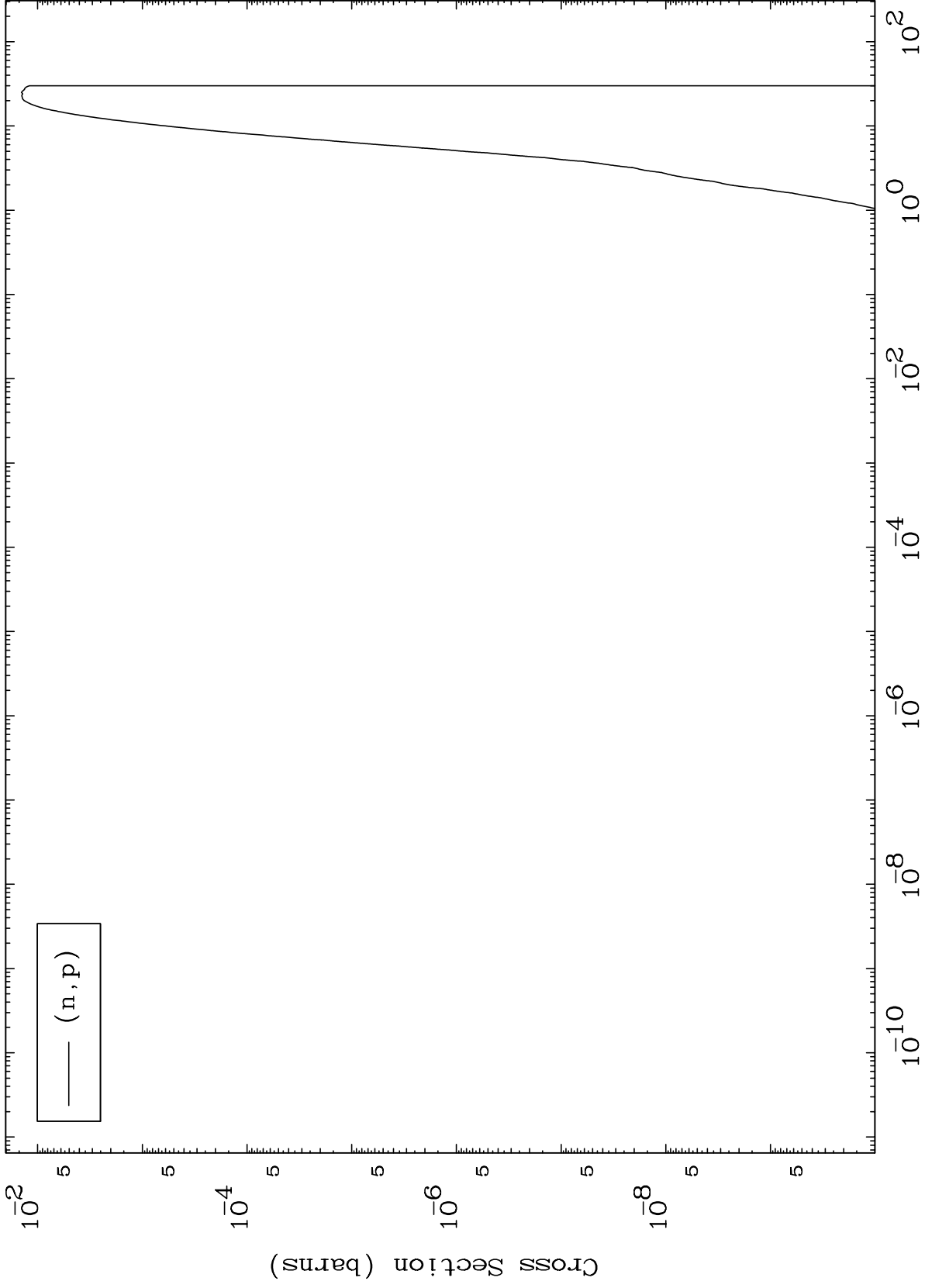
Incident Energy (MeV)

97-Bk-243

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(n,p) Levels
293 Kelvin Cross Sections

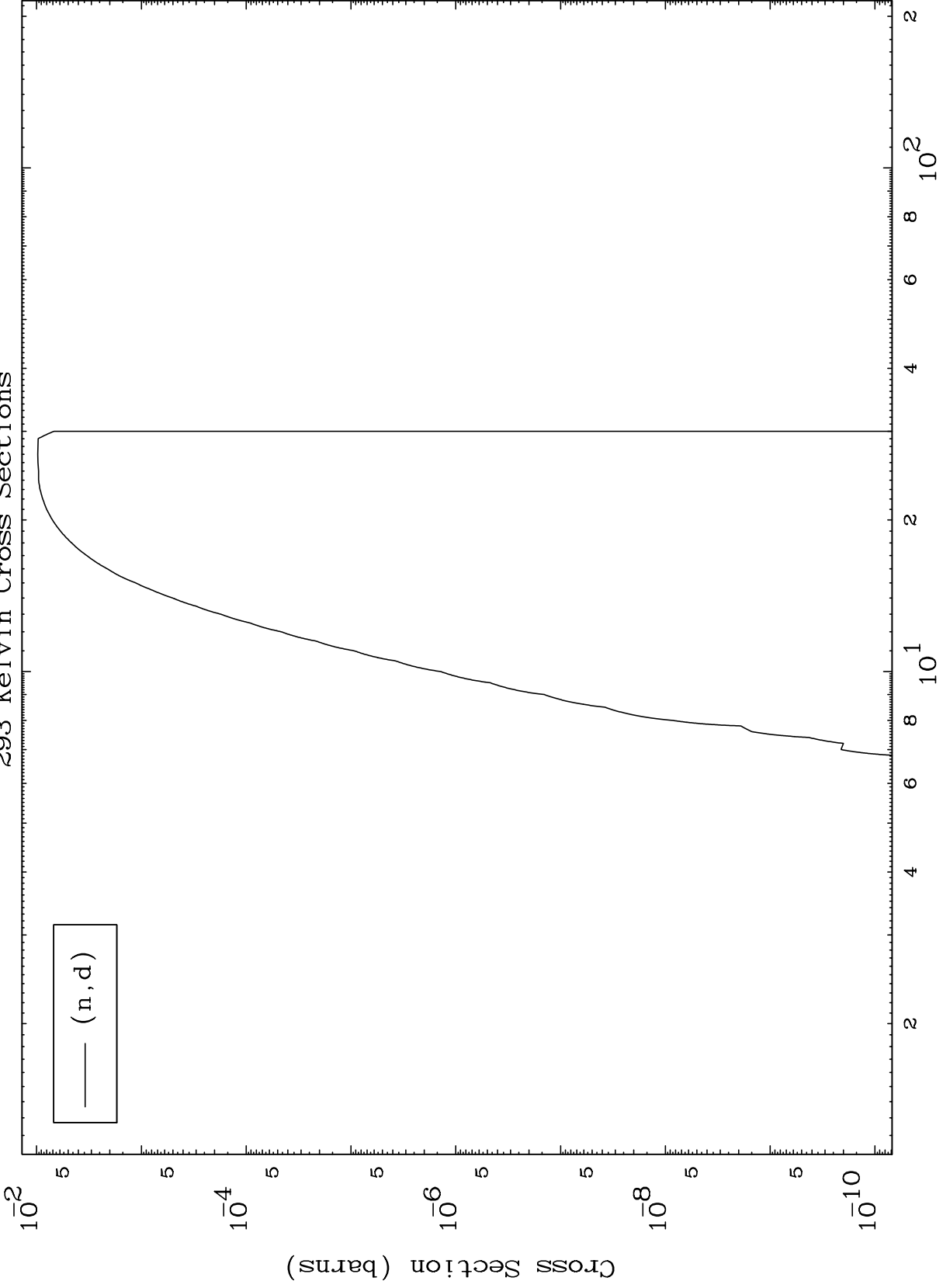
97-Bk-243



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(n,d) Levels
293 Kelvin Cross Sections

97-Bk-243



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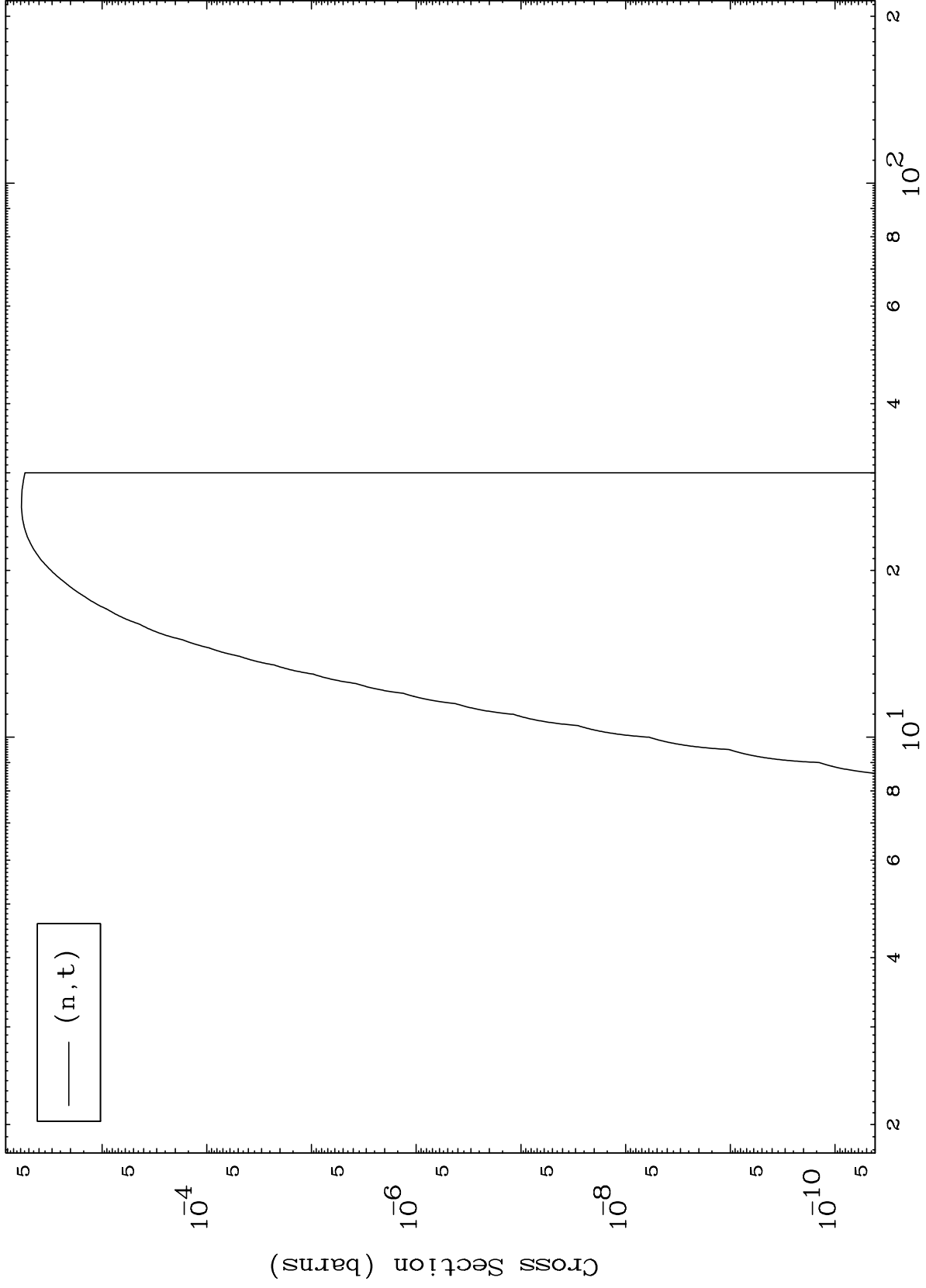
Incident Energy (MeV)

97-Bk-243

MAT 9734

(n,t) Levels
293 Kelvin Cross Sections

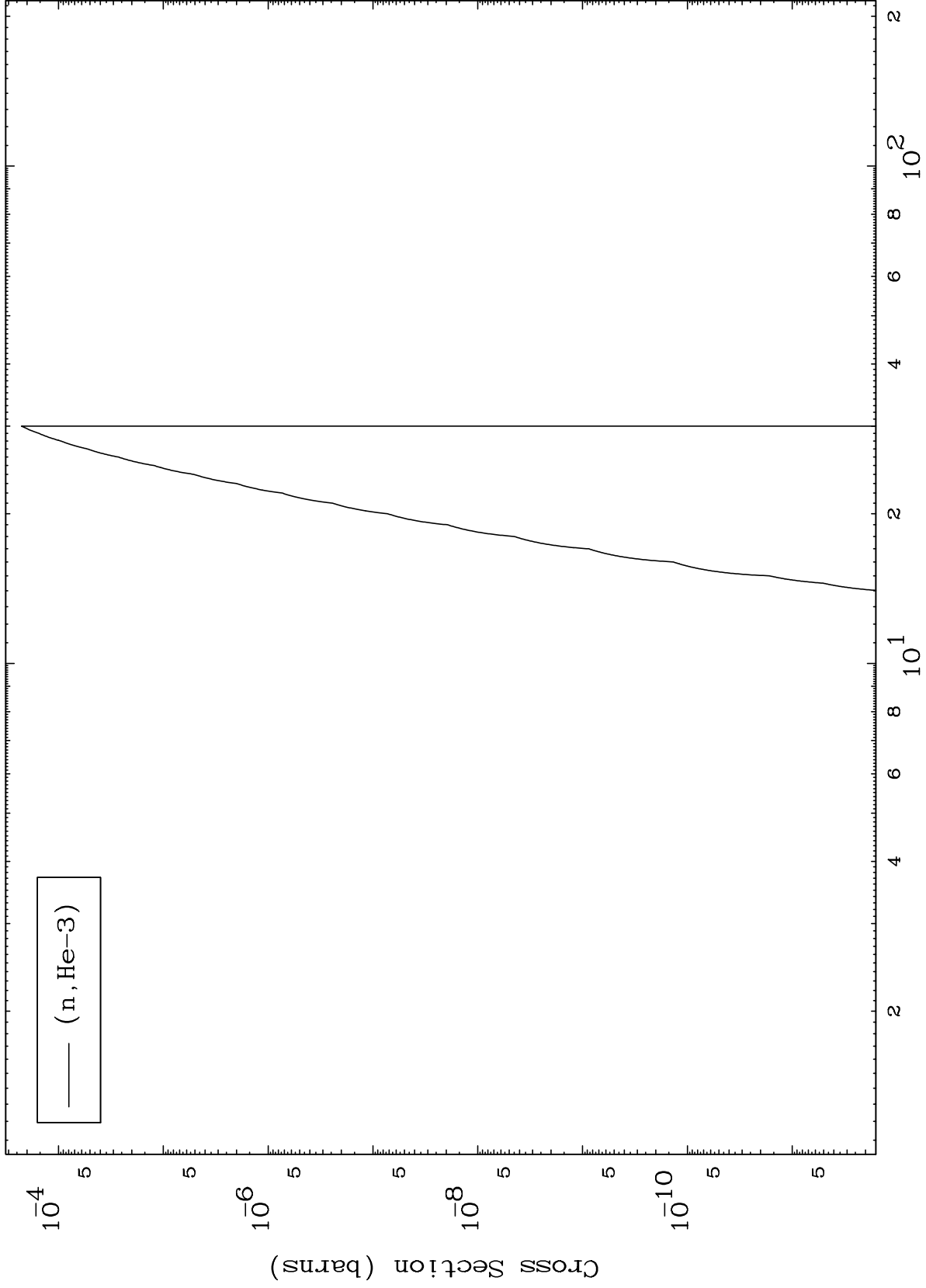
97-Bk-243



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(n,He3) Levels
293 Kelvin Cross Sections

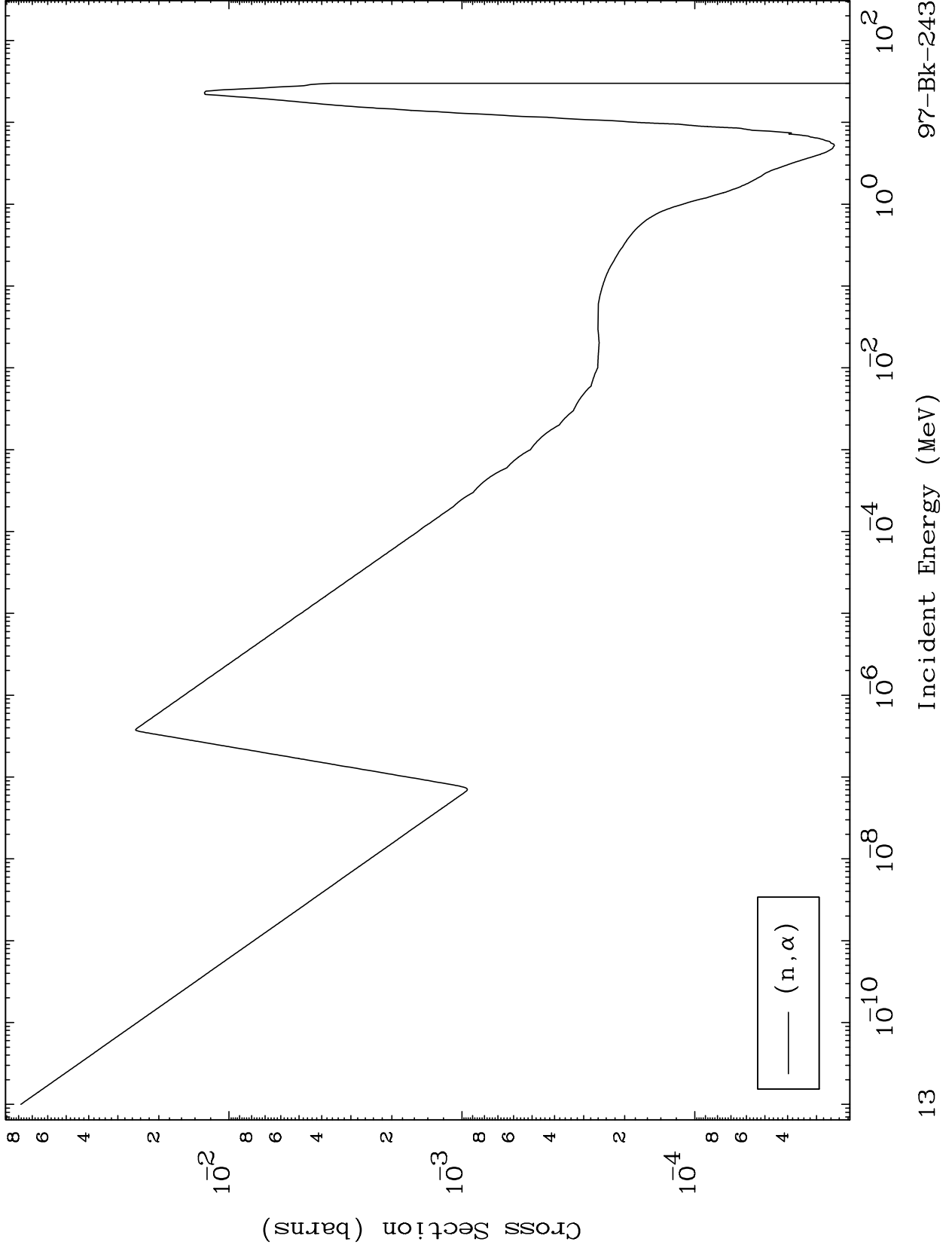
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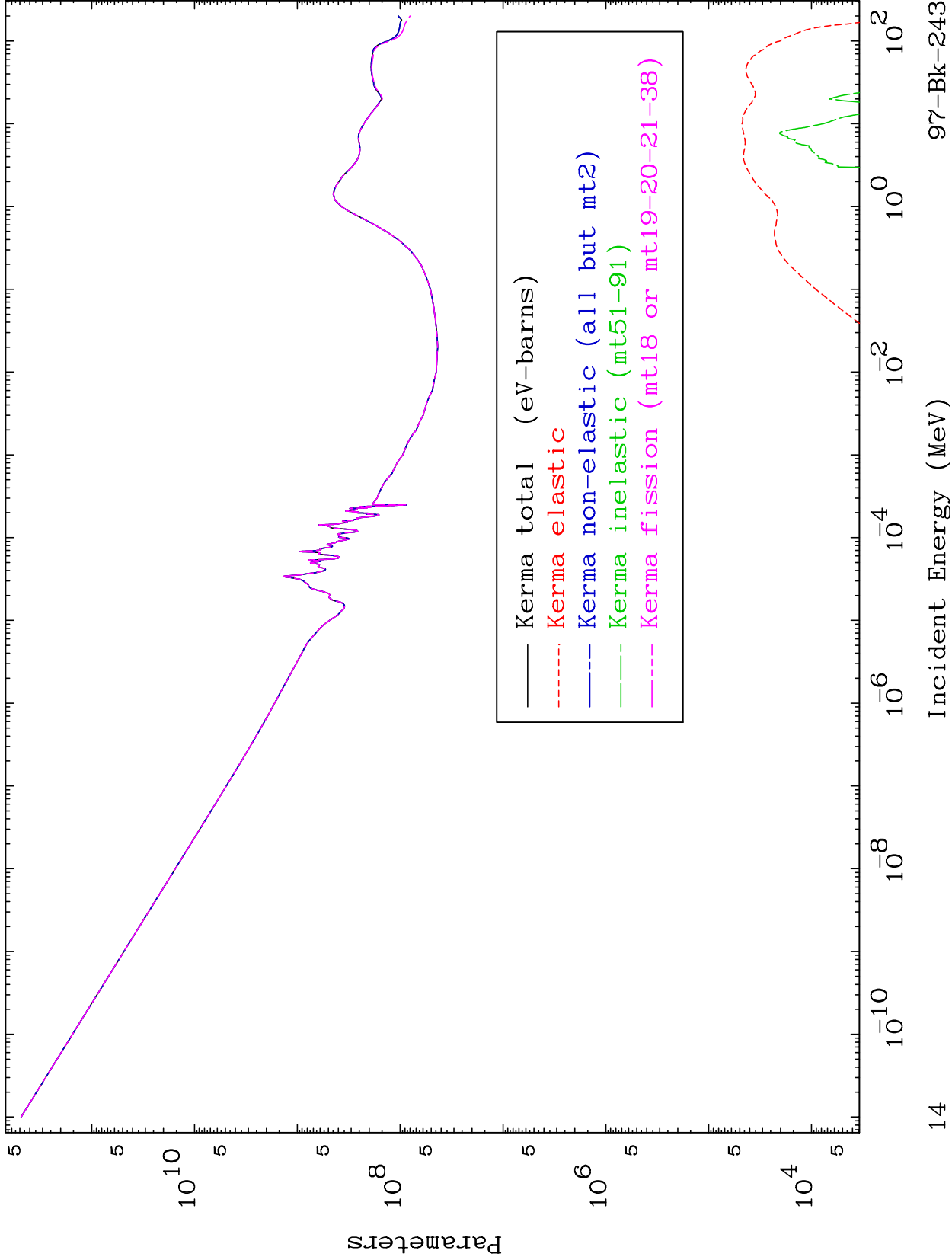


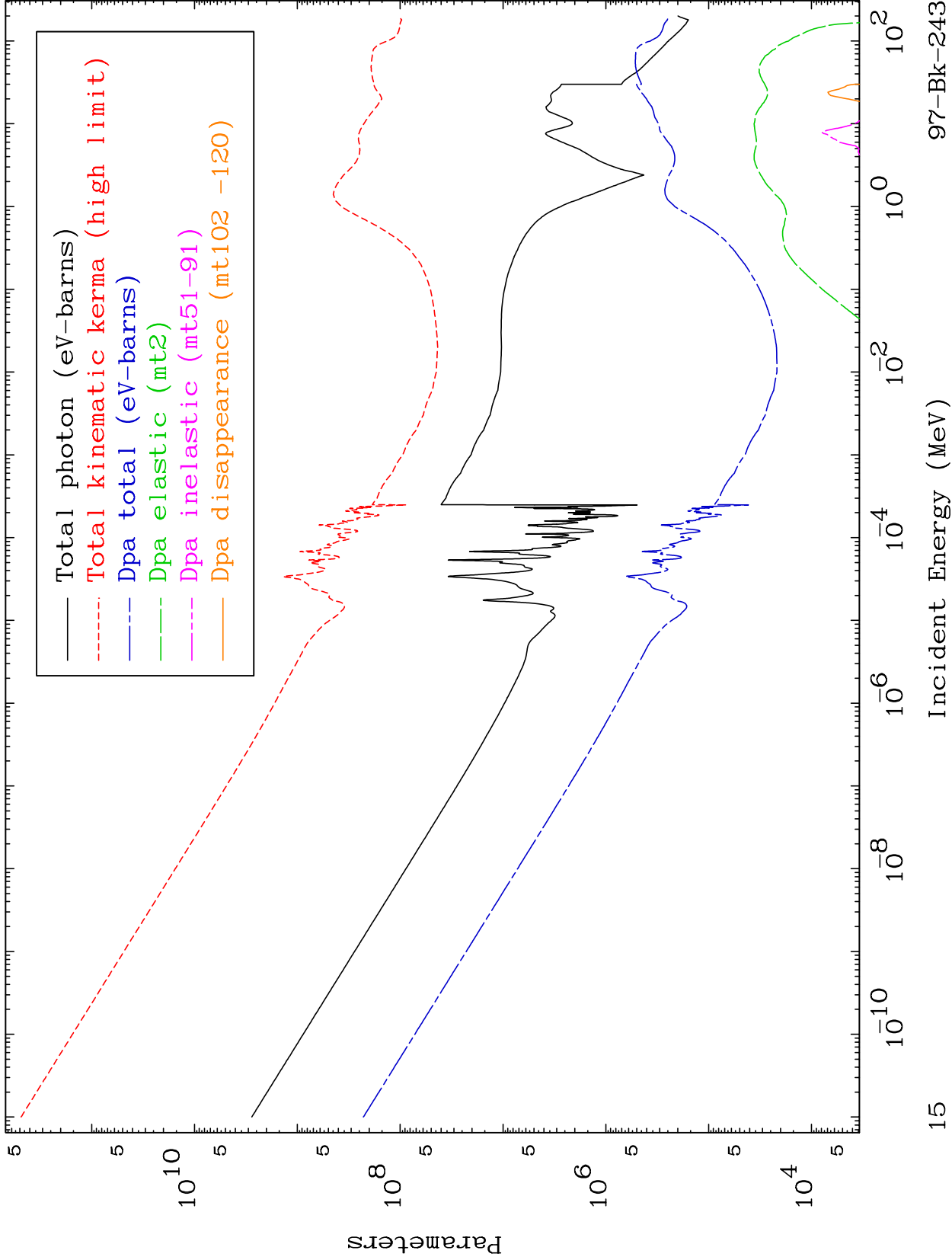
MAT 9734

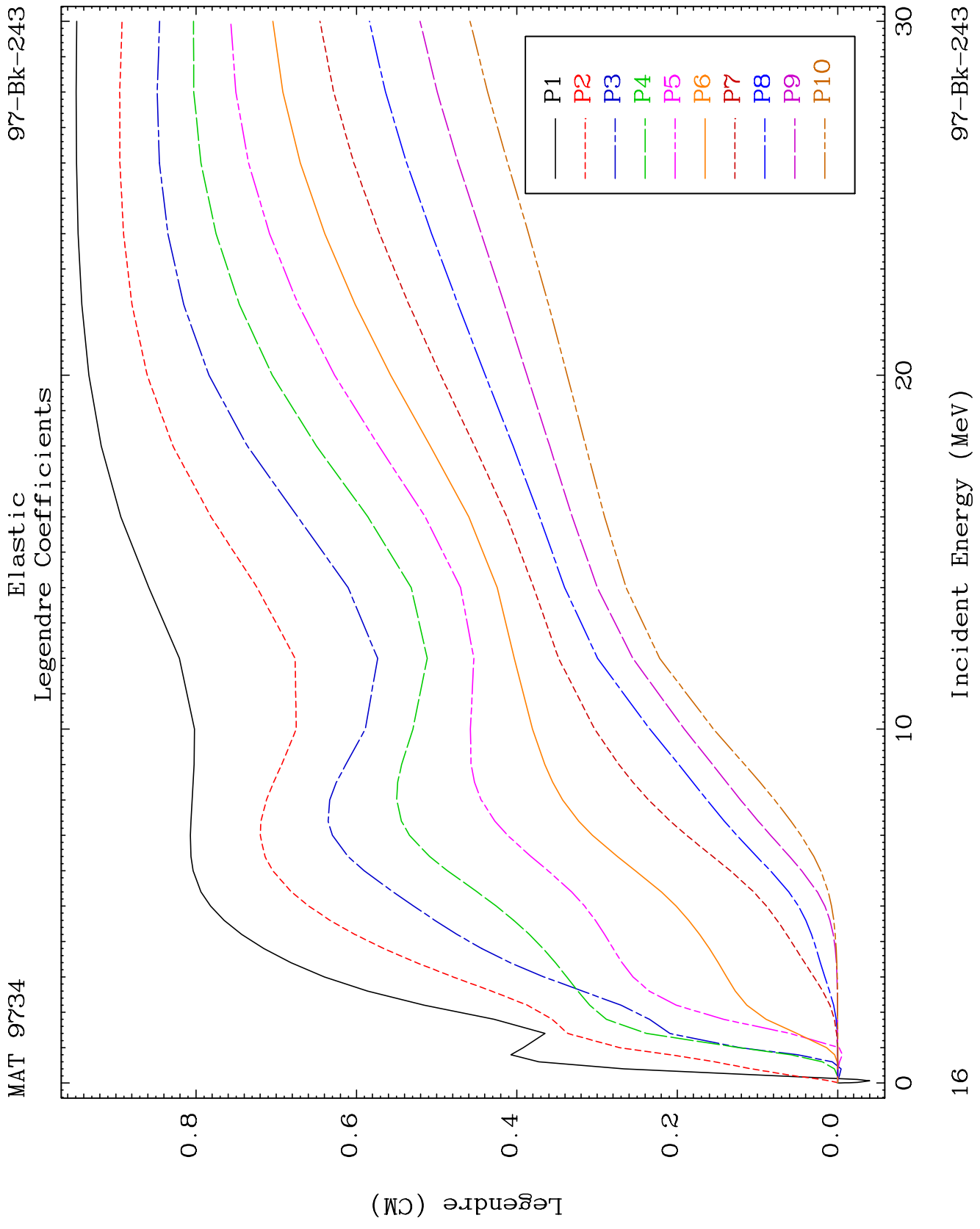
(n,α) Levels
293 Kelvin Cross Sections

97-Bk-243





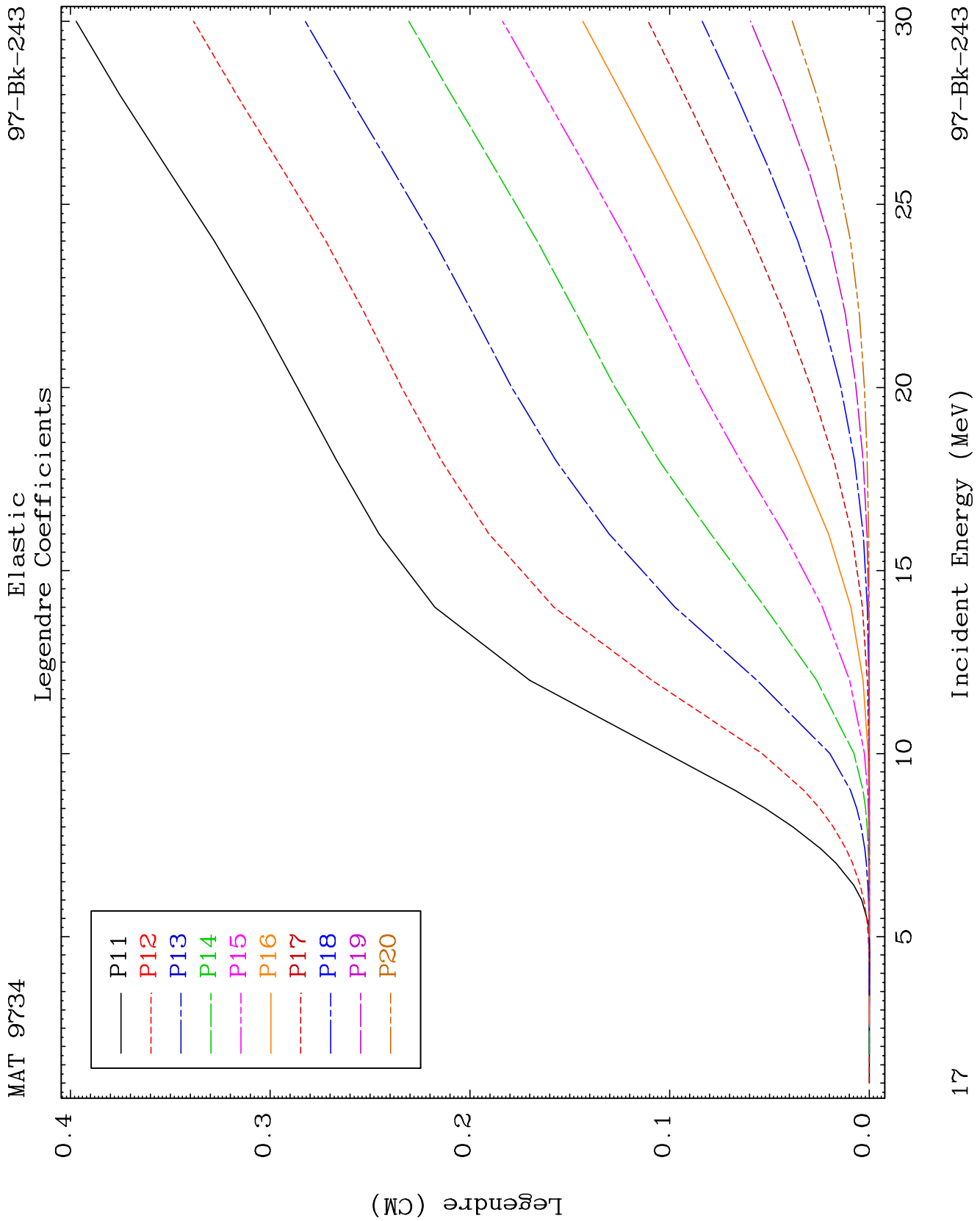




97-Bk-243

Incident Energy (MeV)

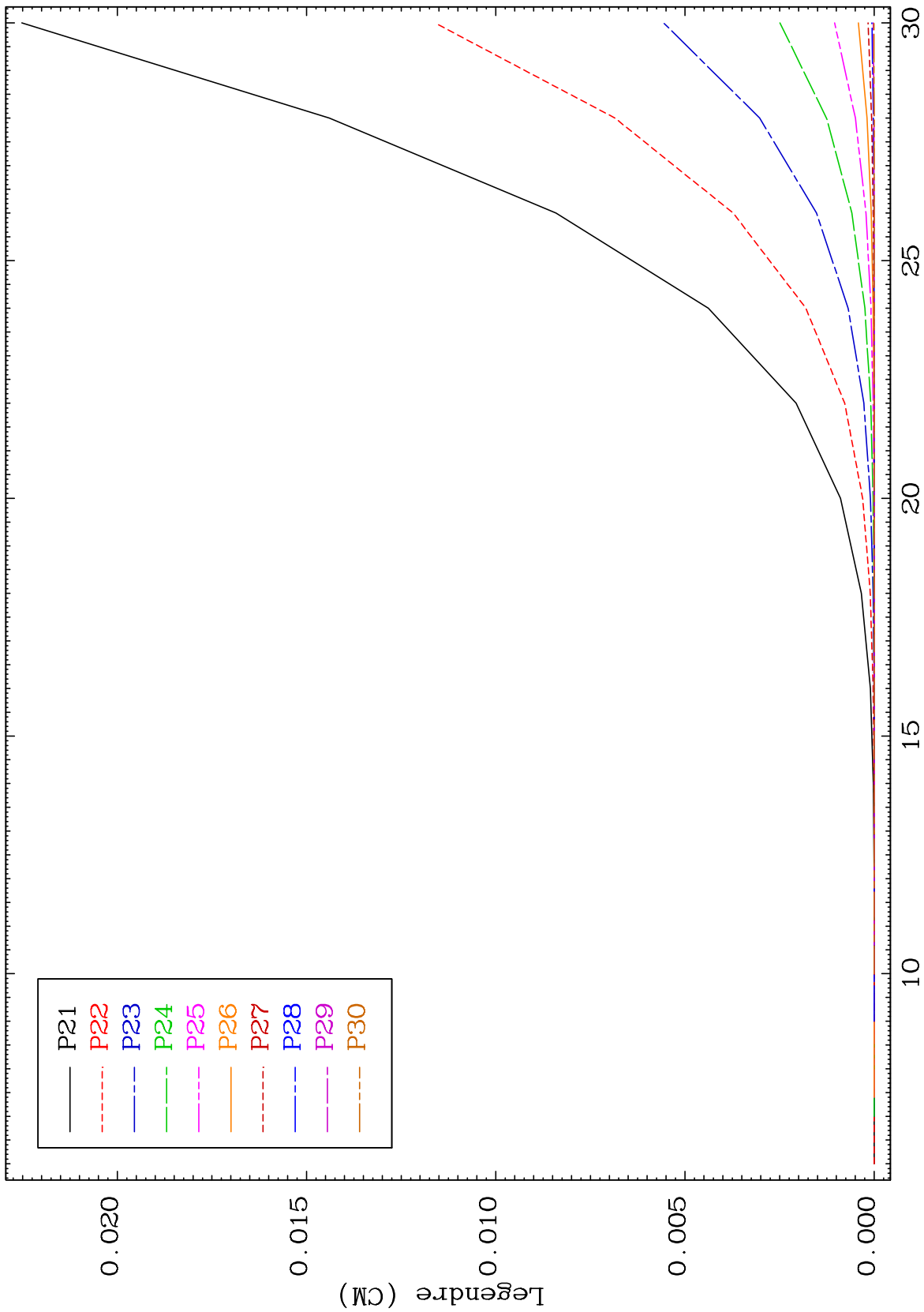
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Elastic Legendre Coefficients

97-Bk-243



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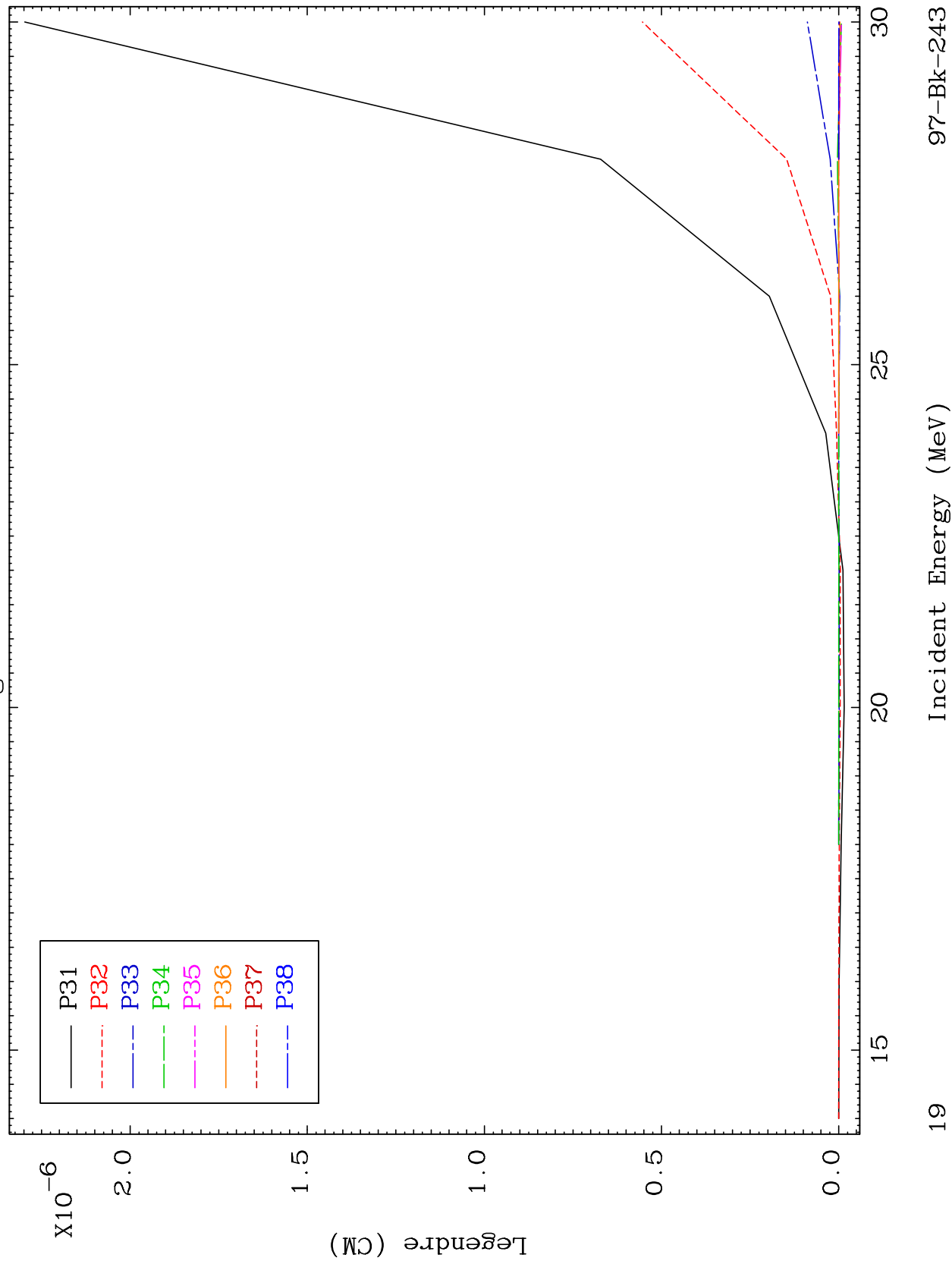
Incident Energy (MeV)

97-Bk-243

MAT 9734

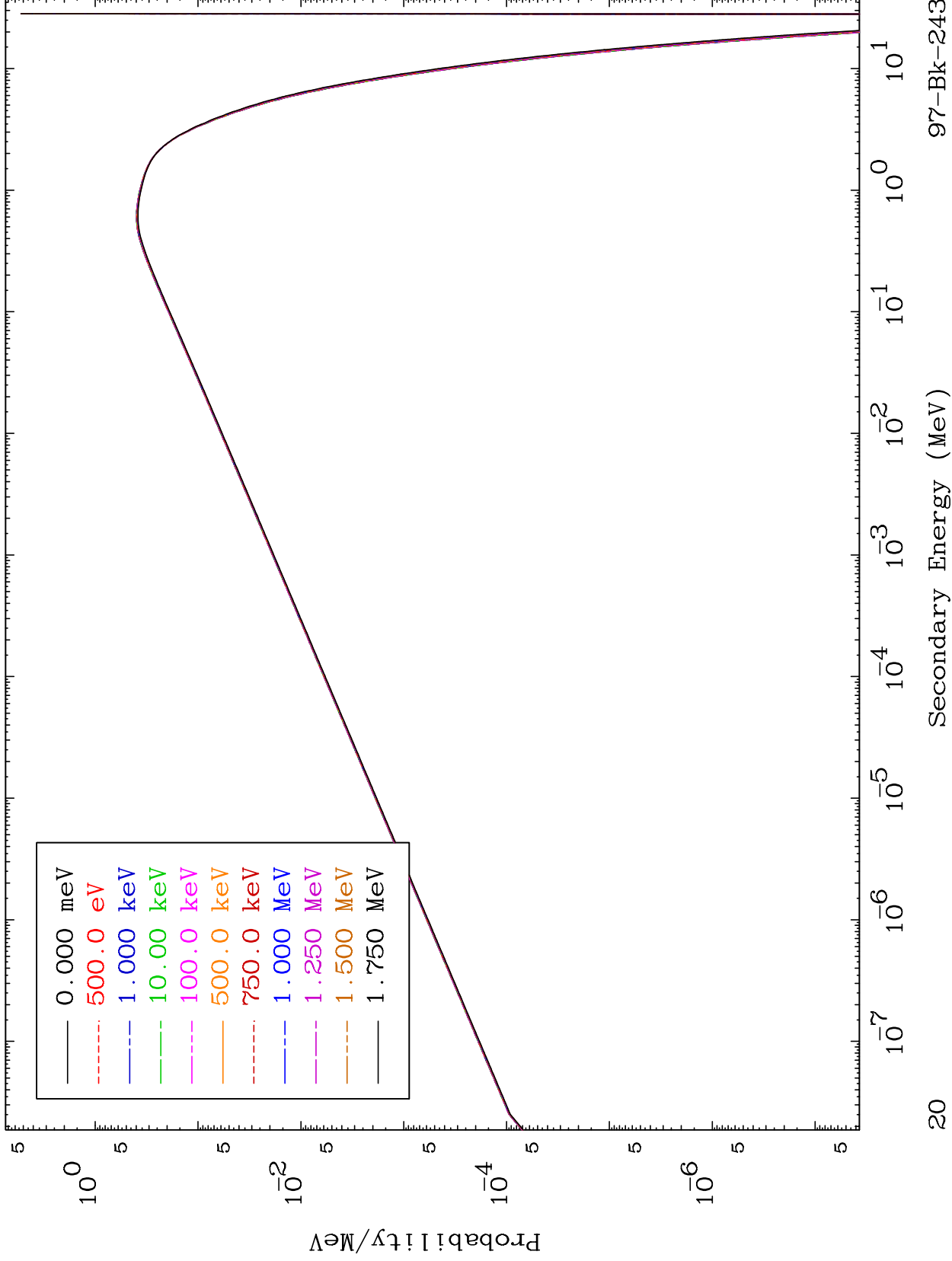
Elastic Legendre Coefficients

97-Bk-243

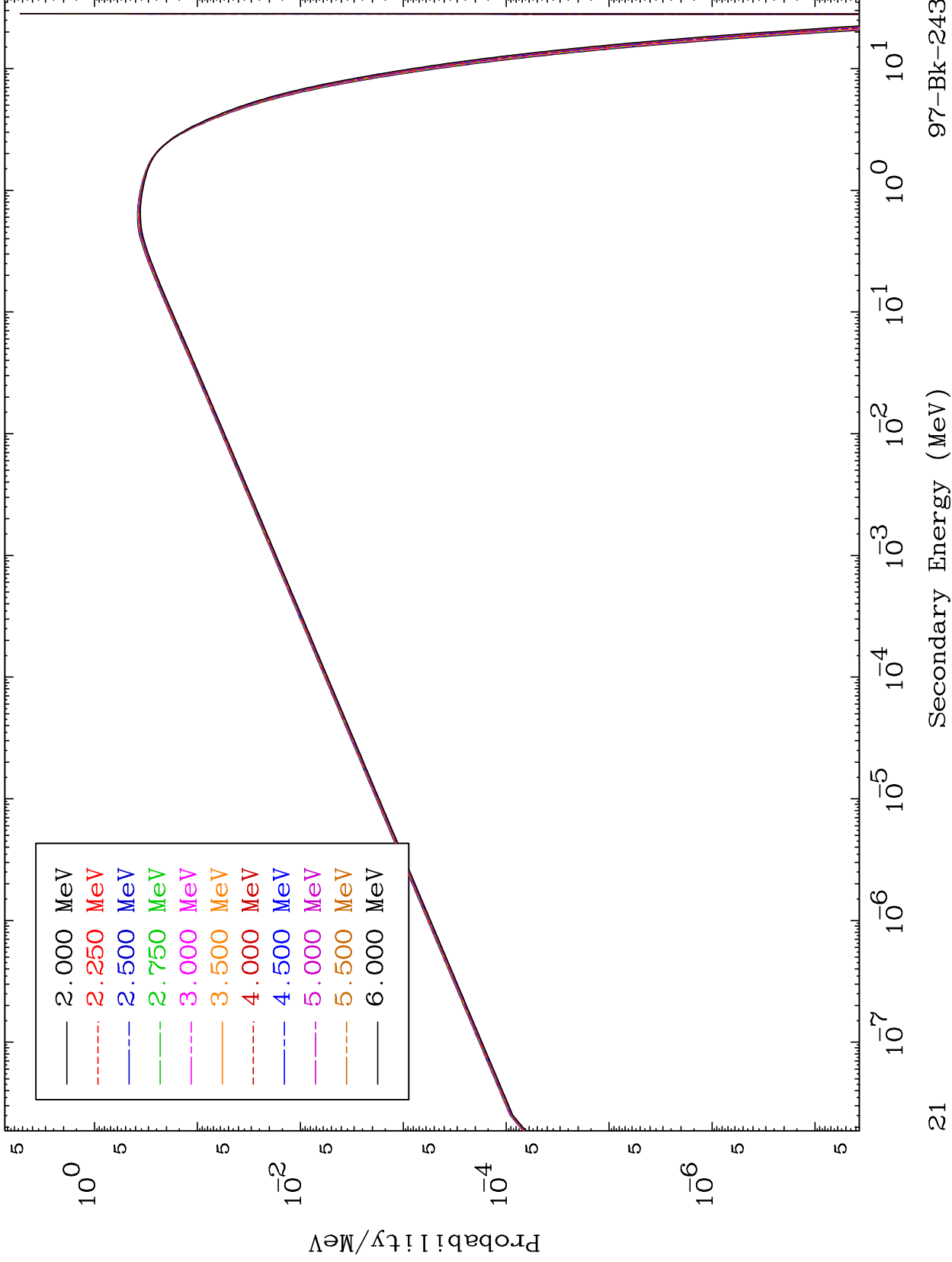


19

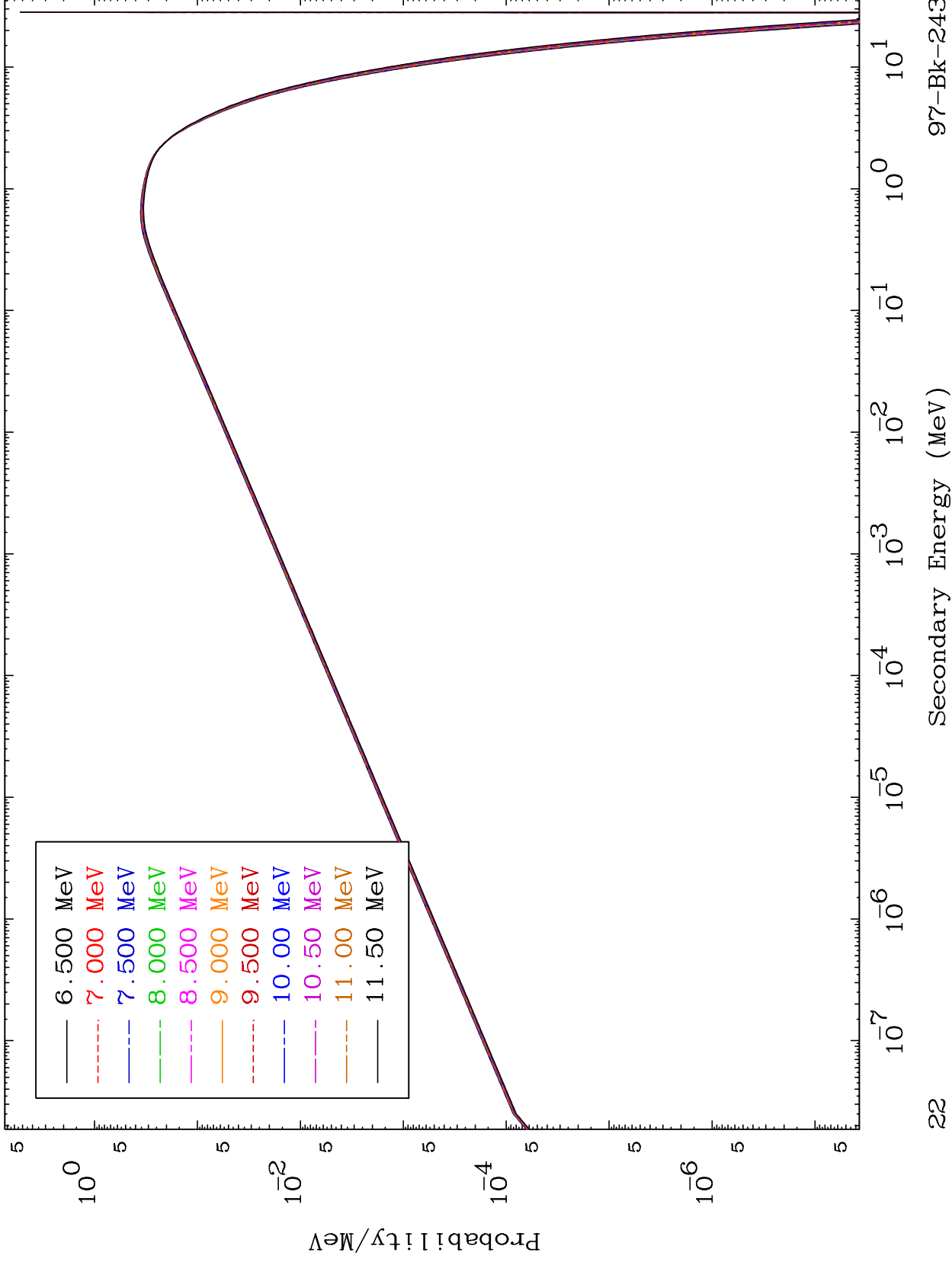
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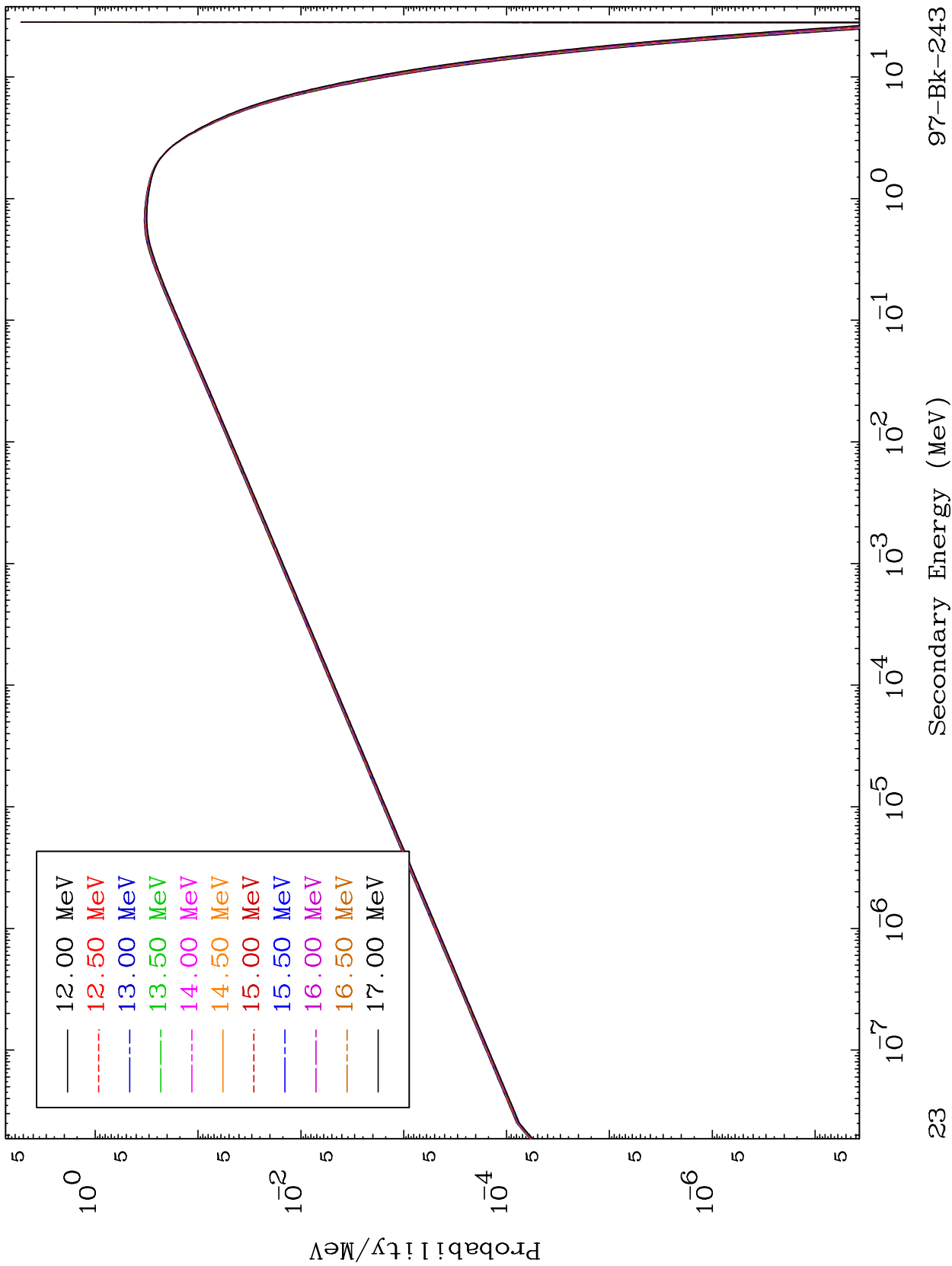
Fission Energy Distribution



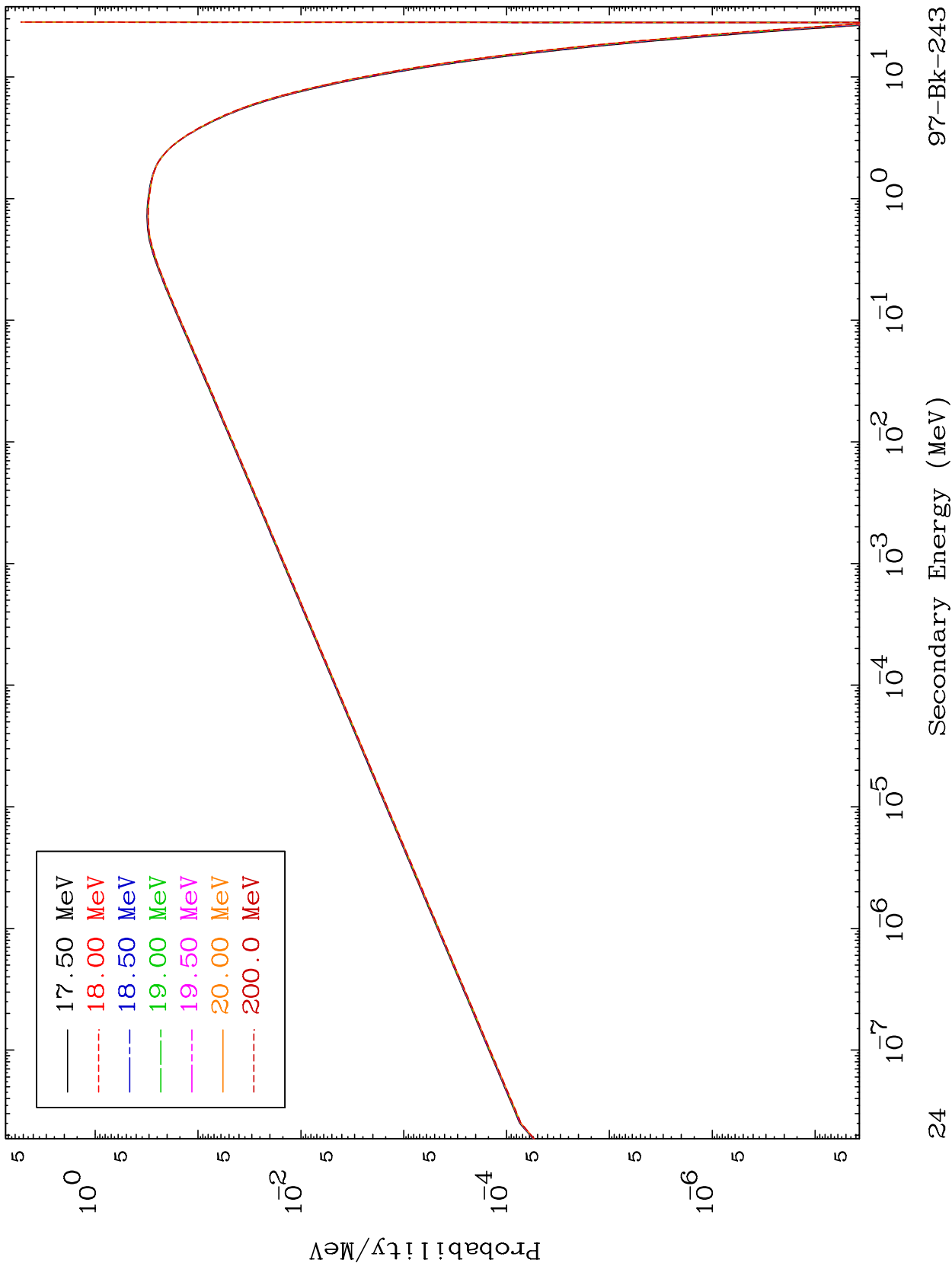
Fission Energy Distribution



Fission
Energy Distribution



Fission
Energy Distribution

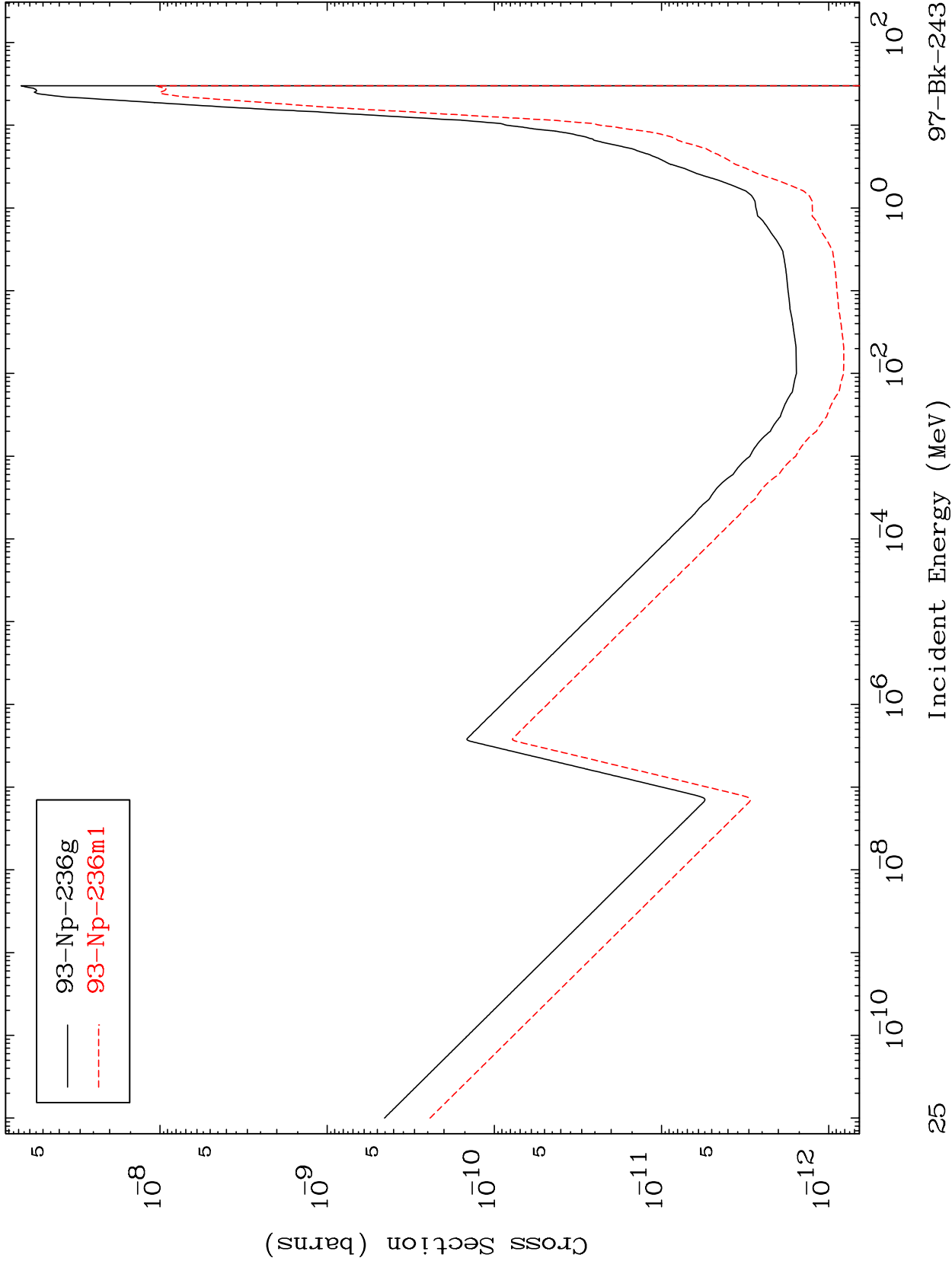


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(n,2α)

97-Bk-243

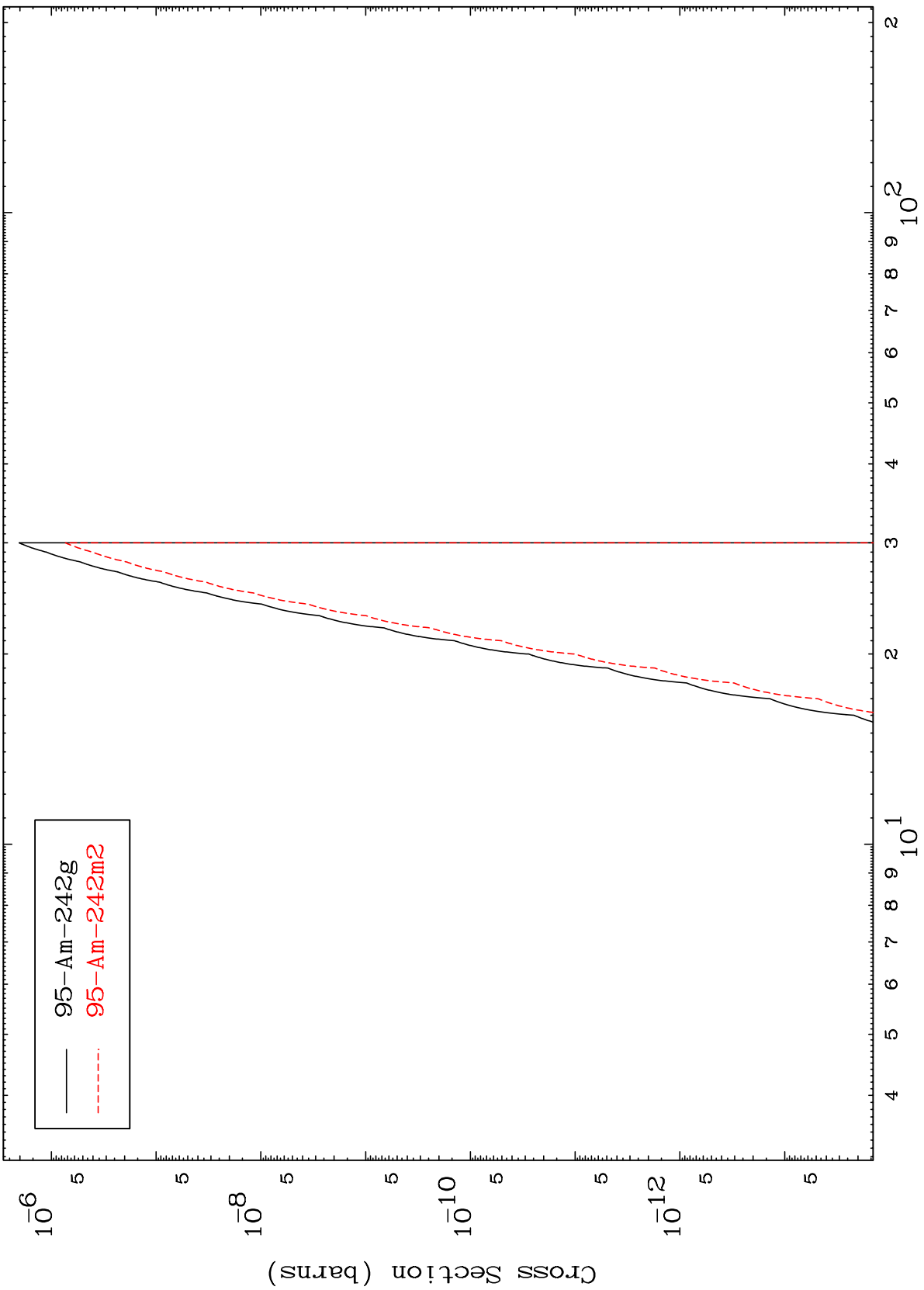
Radionuclide Production Cross Section



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97-Bk-243

Radionuclide Production Cross Section
(n,2p)



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97-Bk-243

Incident Energy (MeV)