

Program EVALPLOT
(Version 2015-2)

by

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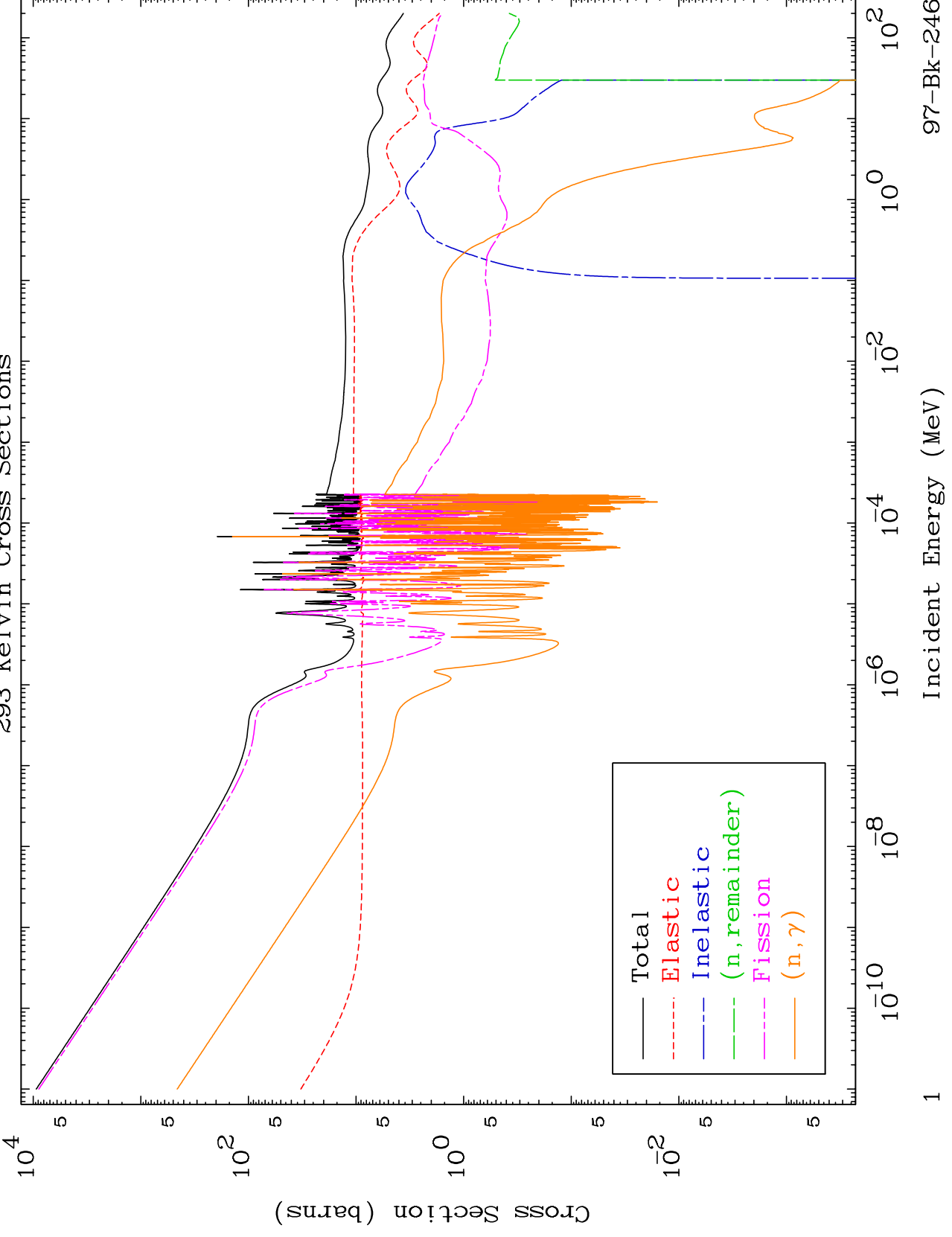
Press Mouse Button to Start

MAT 9743

Major

293 Kelvin Cross Sections

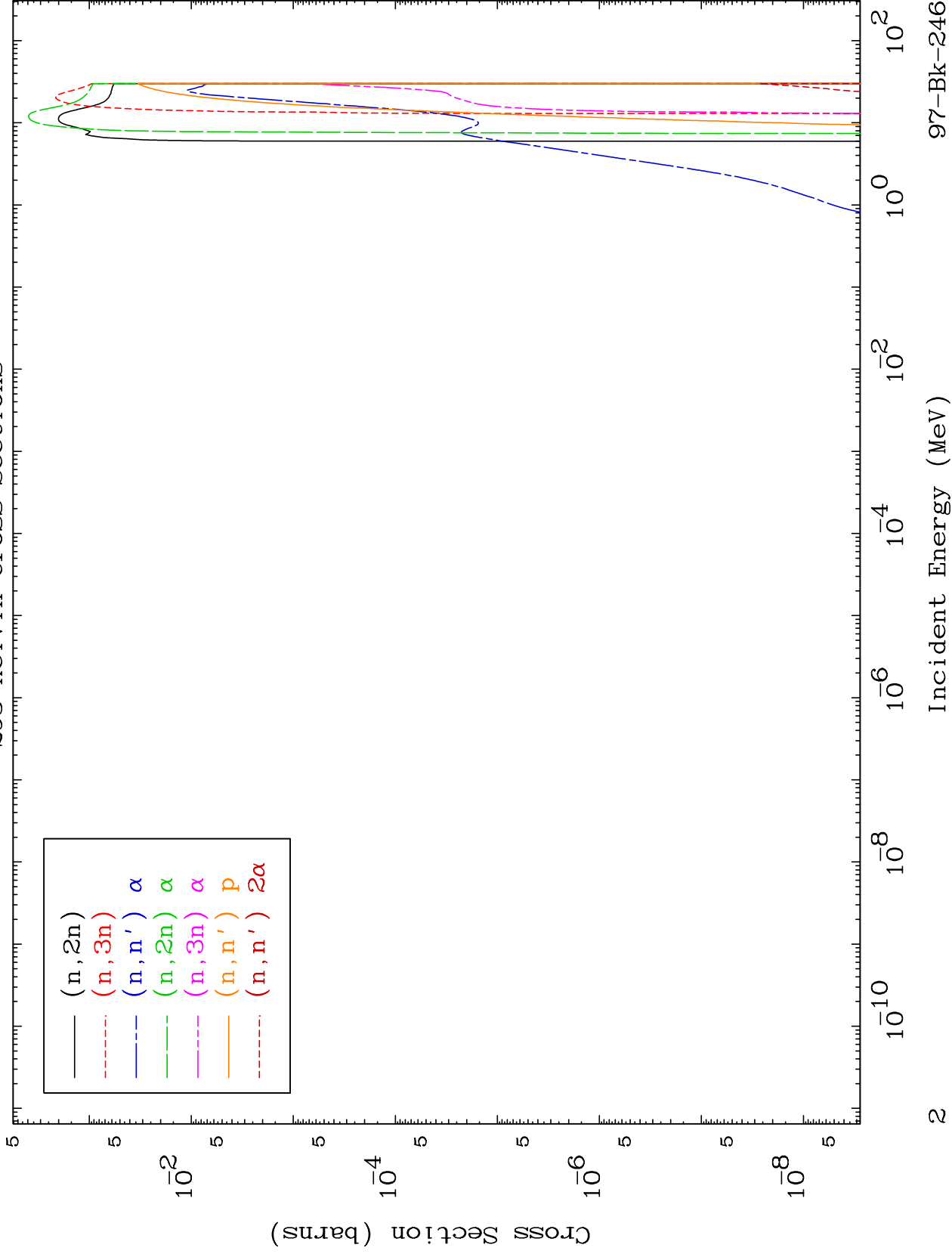
97-Bk-246



MAT 9743

Neutron Production
293 Kelvin Cross Sections

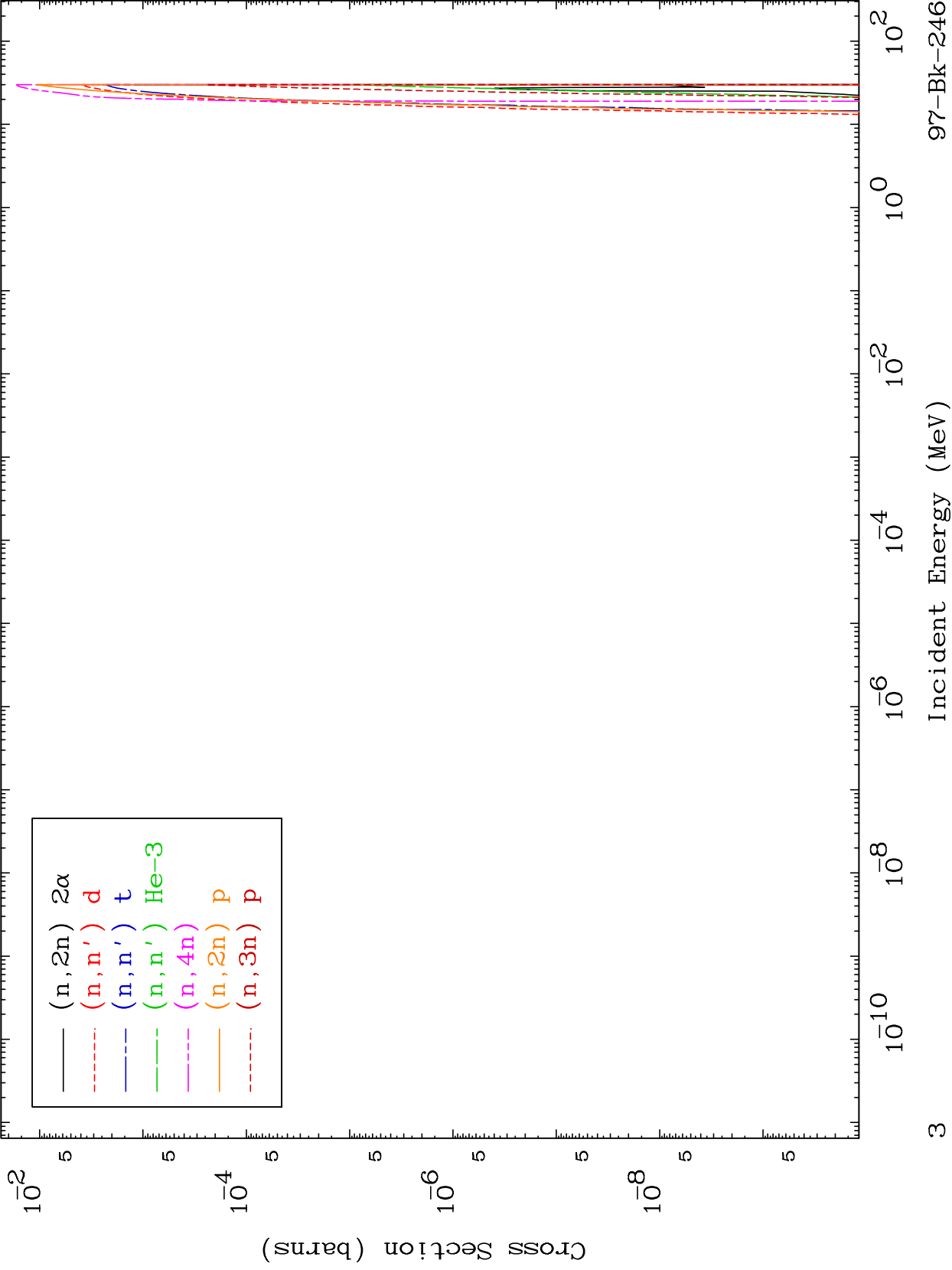
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MAT 9743

Neutron Production
293 Kelvin Cross Sections

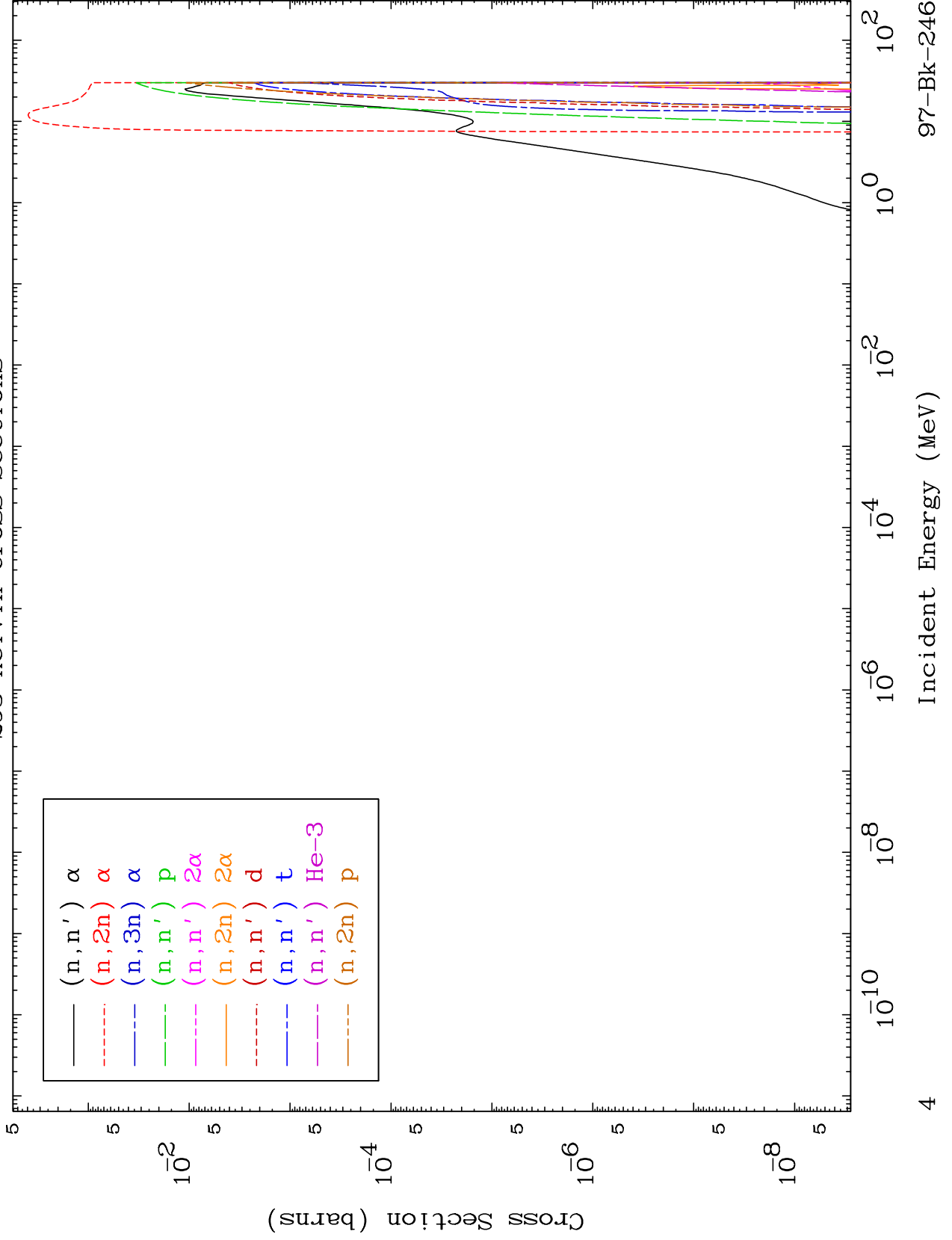
97-Bk-246



MAT 9743

Charged Particle
293 Kelvin Cross Sections

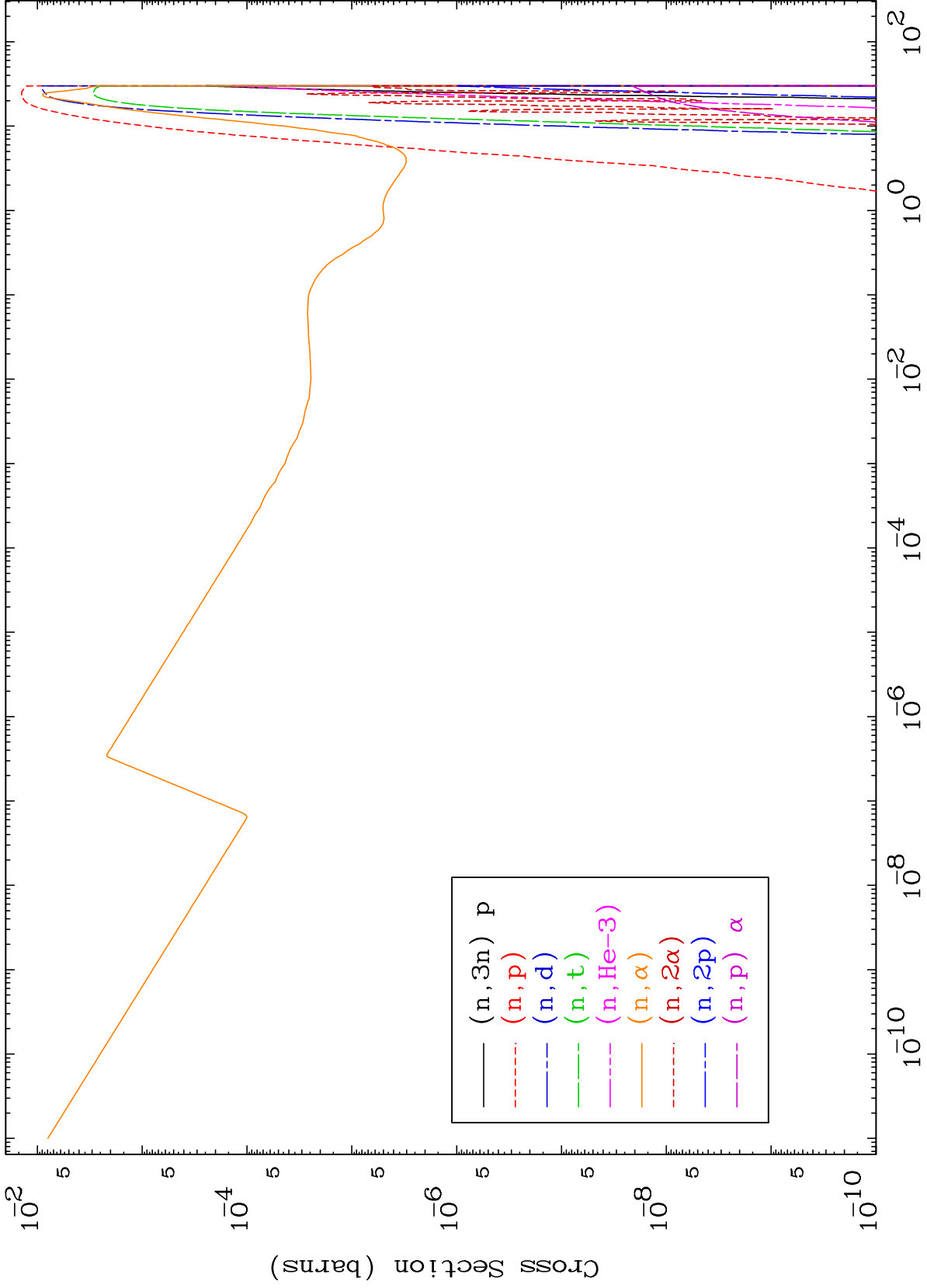
97-Bk-246



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Charged Particle
293 Kelvin Cross Sections

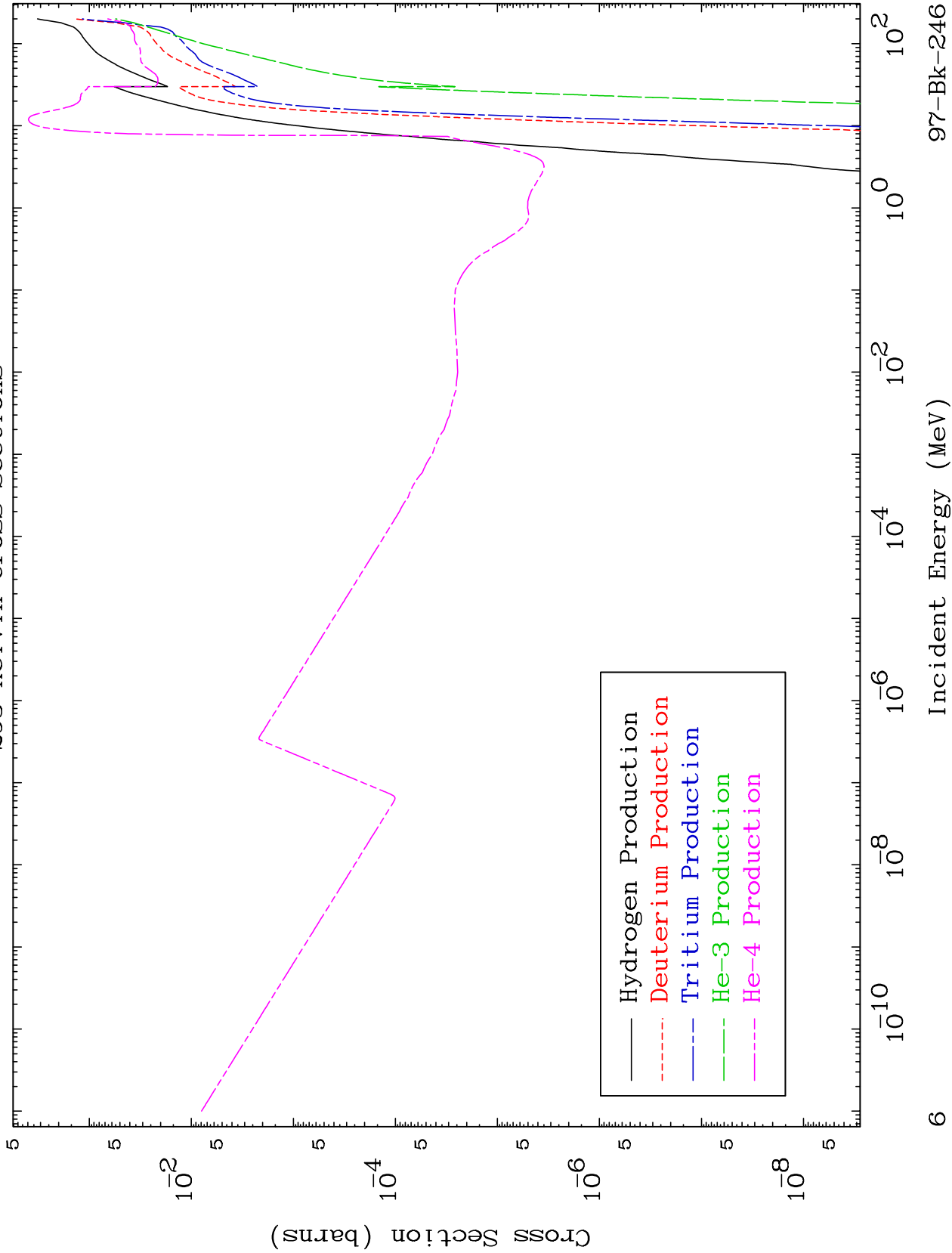
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Particle Production
293 Kelvin Cross Sections

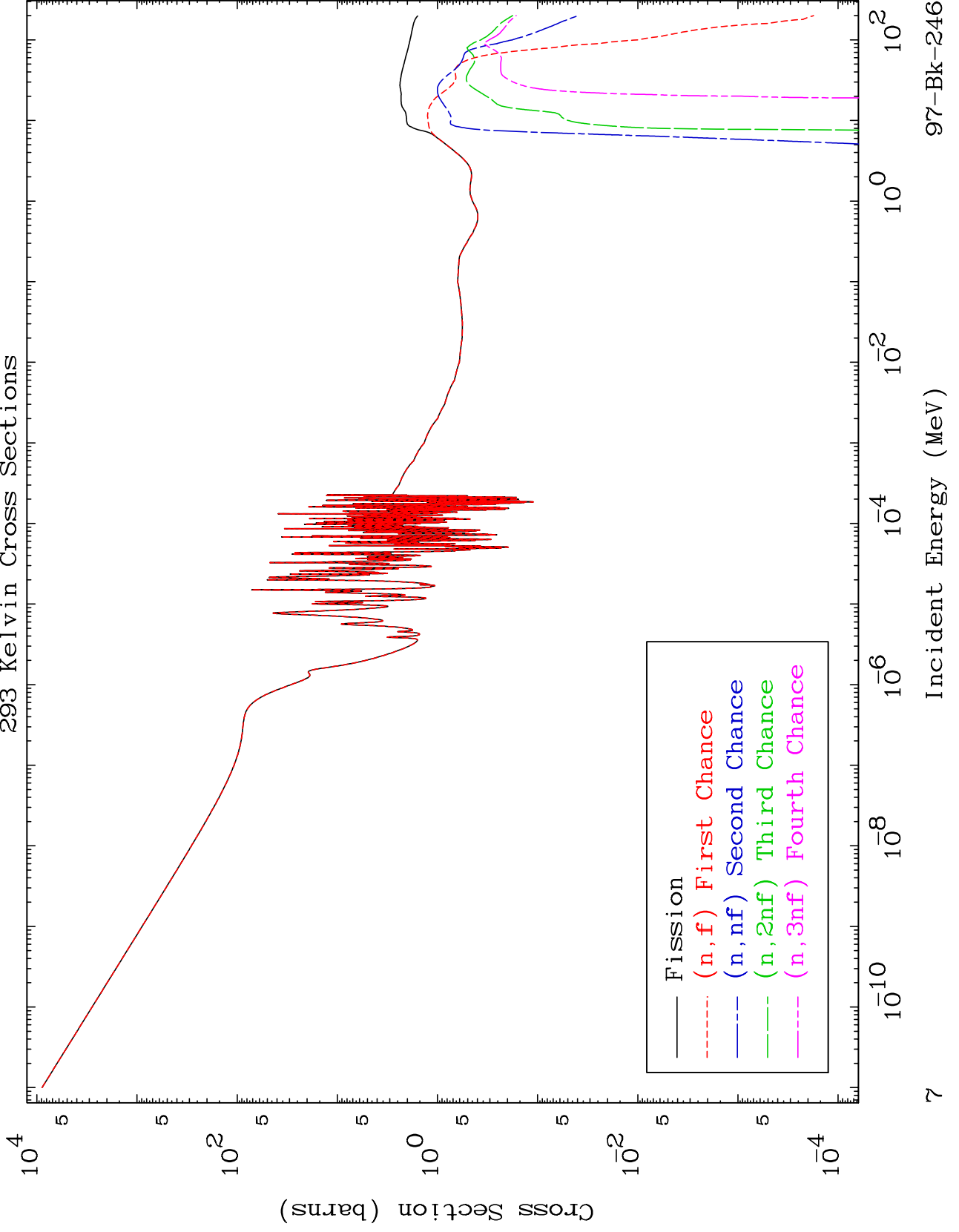
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Fission
293 Kelvin Cross Sections

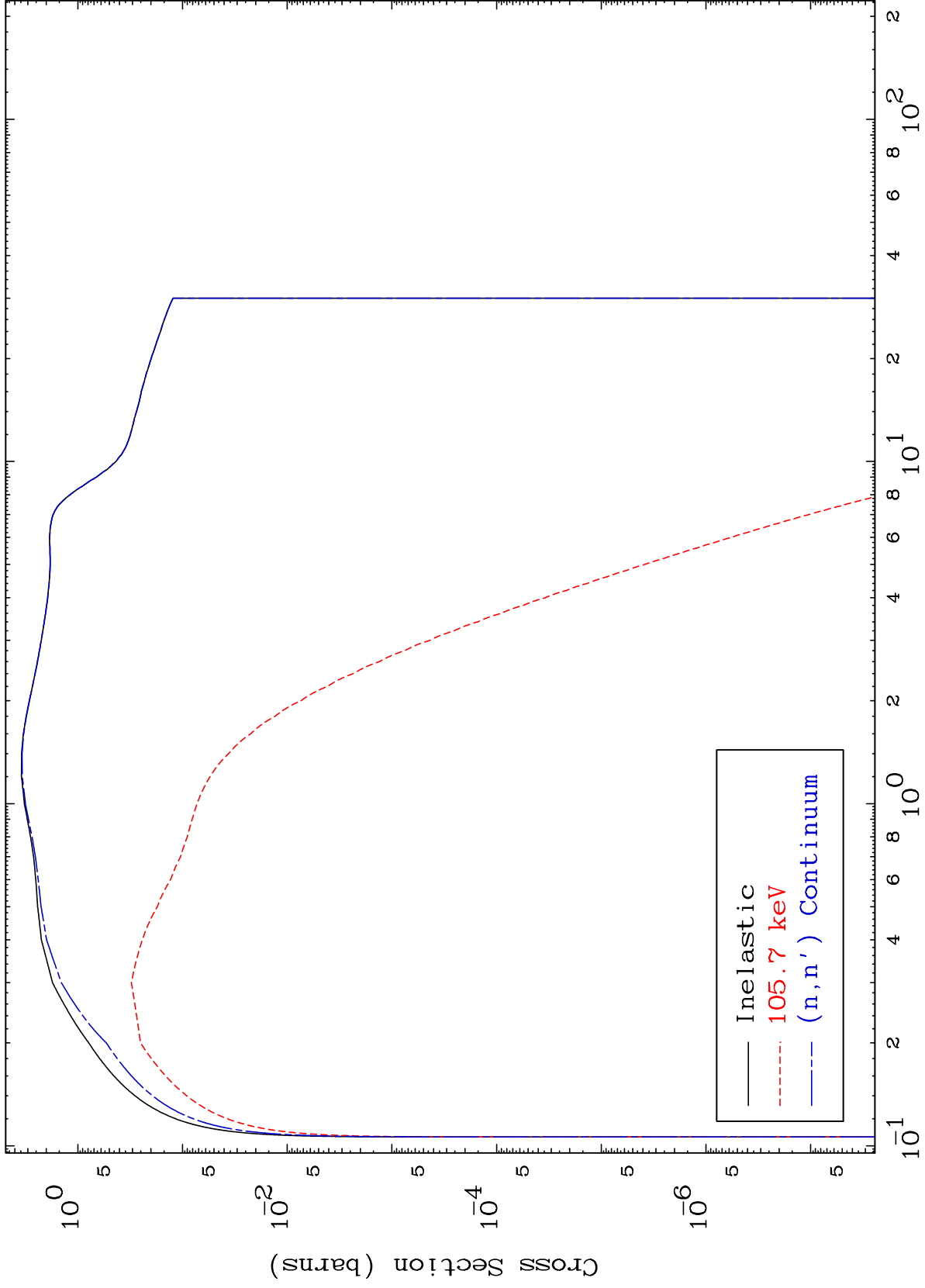
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(n,n') Level
293 Kelvin Cross Sections

97-Bk-246



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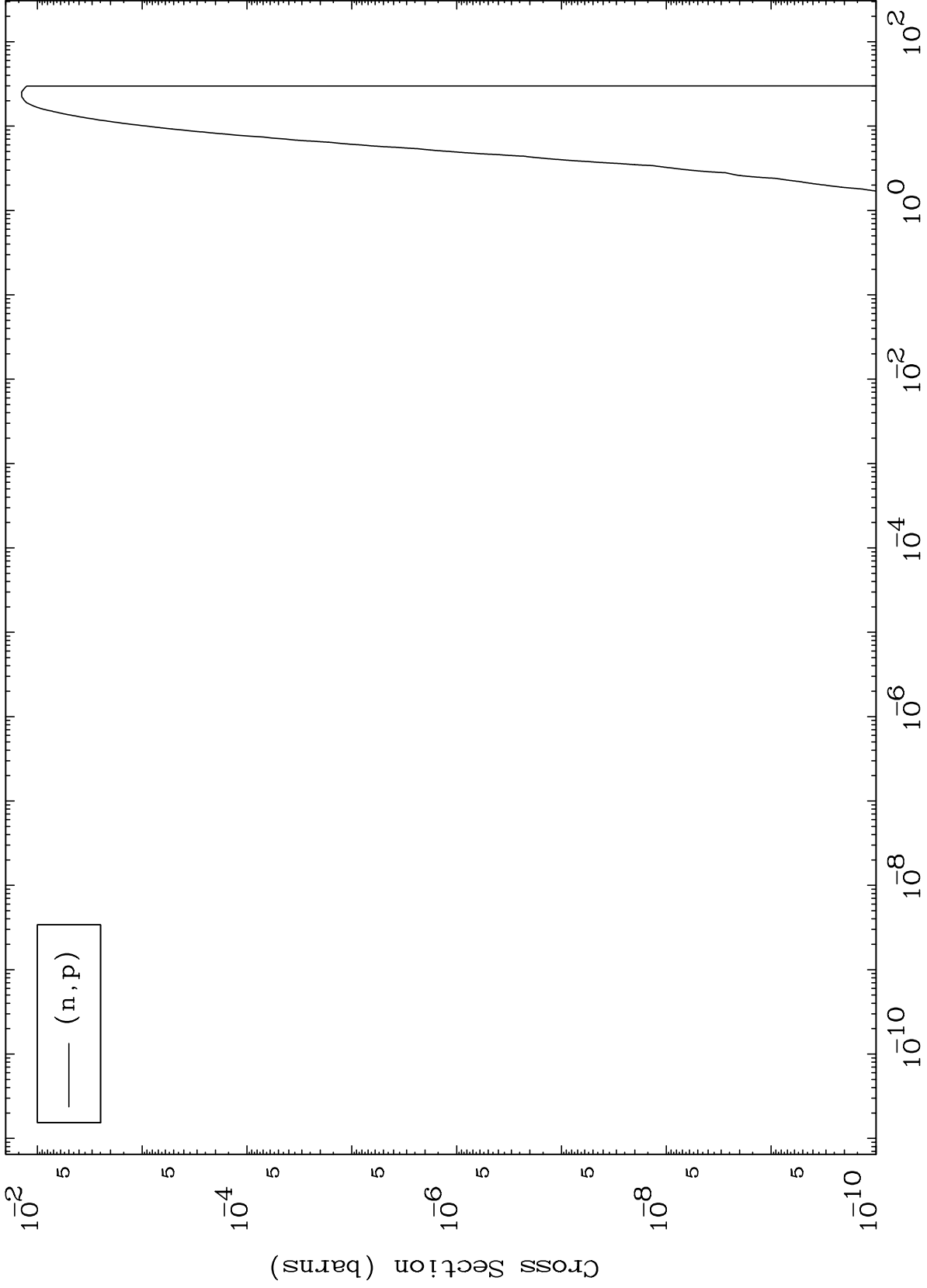
Incident Energy (MeV)

97-Bk-246

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(n,p) Levels
293 Kelvin Cross Sections

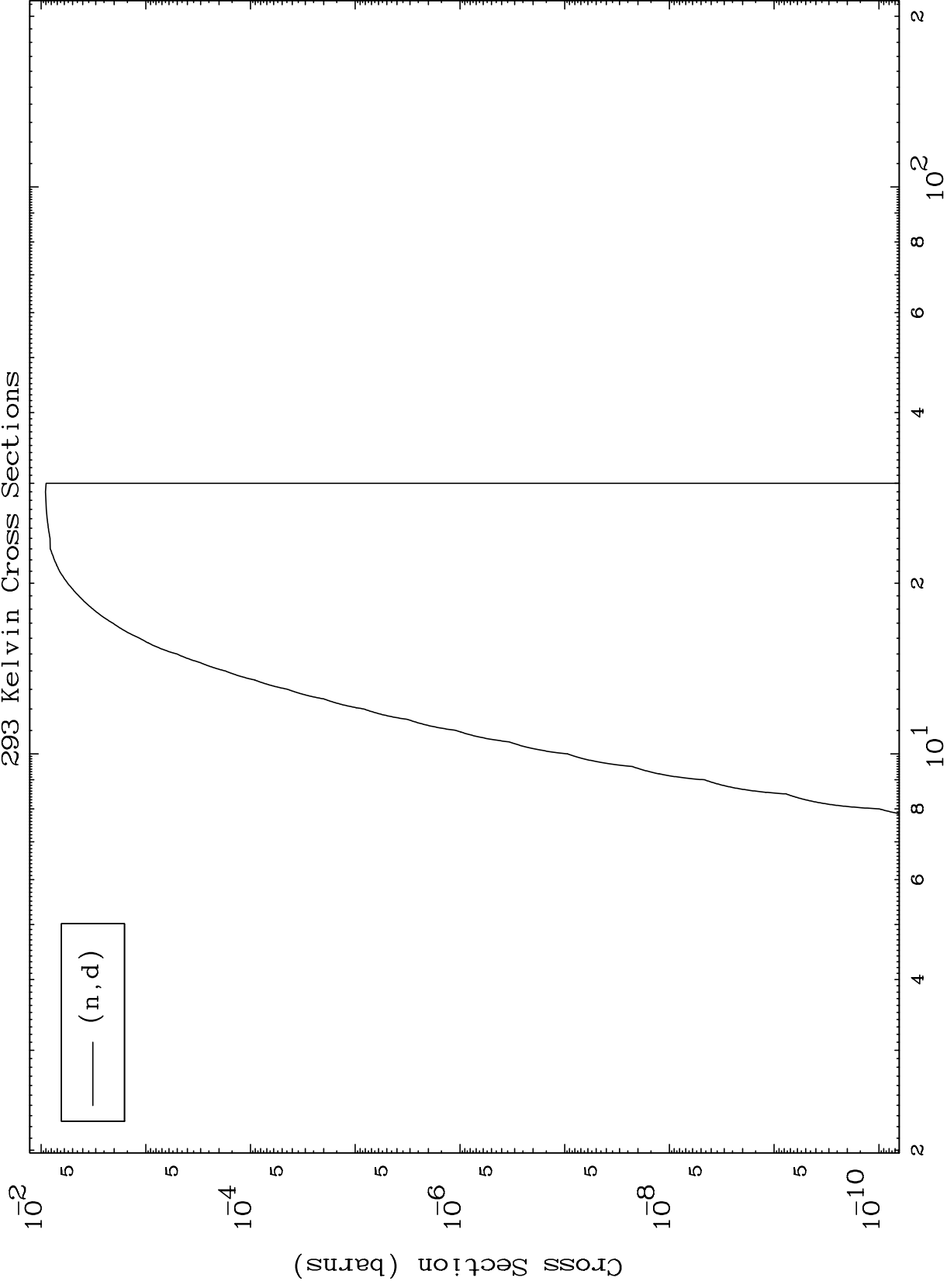
97-Bk-246



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(n,d) Levels
293 Kelvin Cross Sections

97-Bk-246



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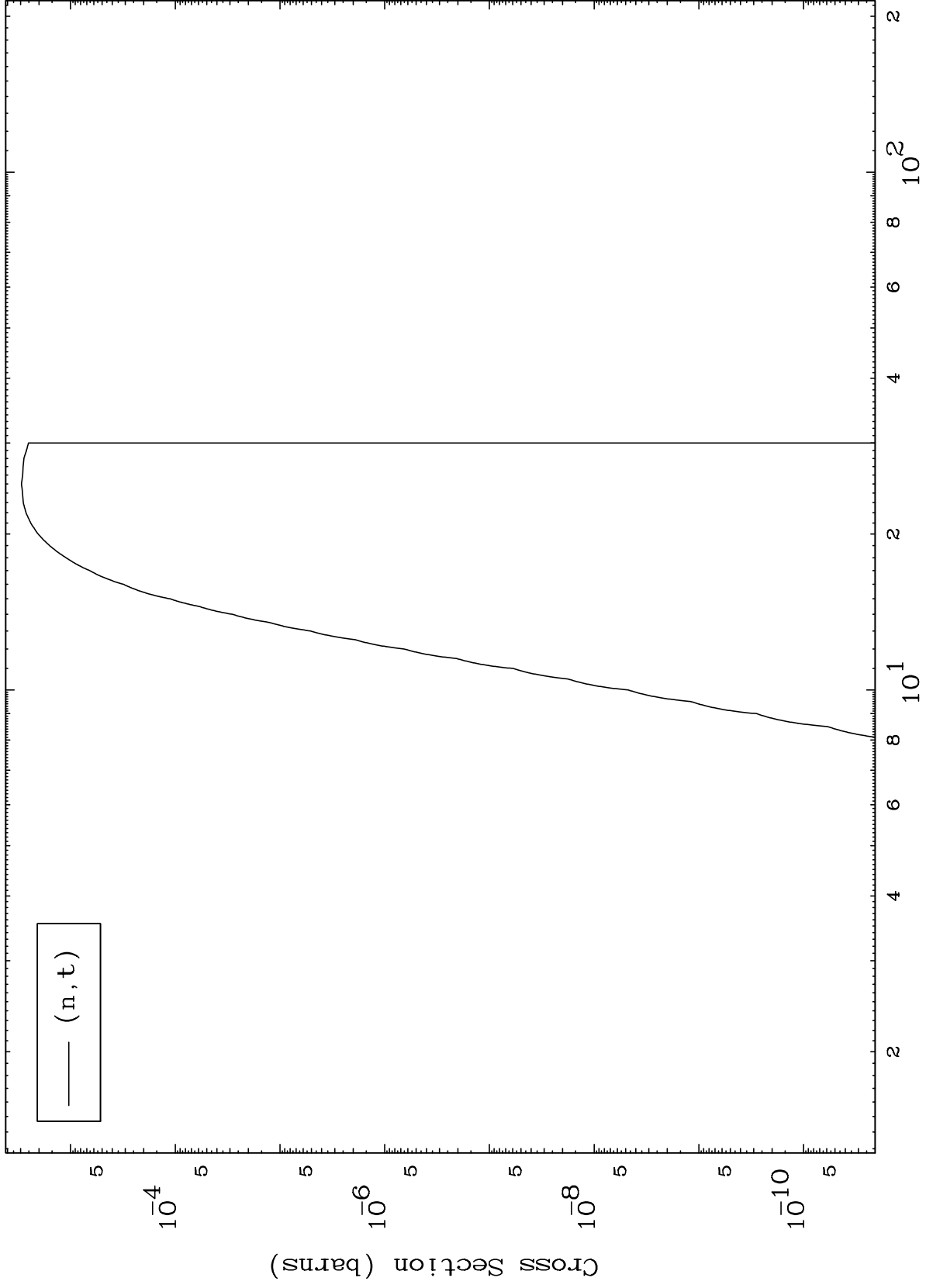
Incident Energy (MeV)

97-Bk-246

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(n,t) Levels
293 Kelvin Cross Sections

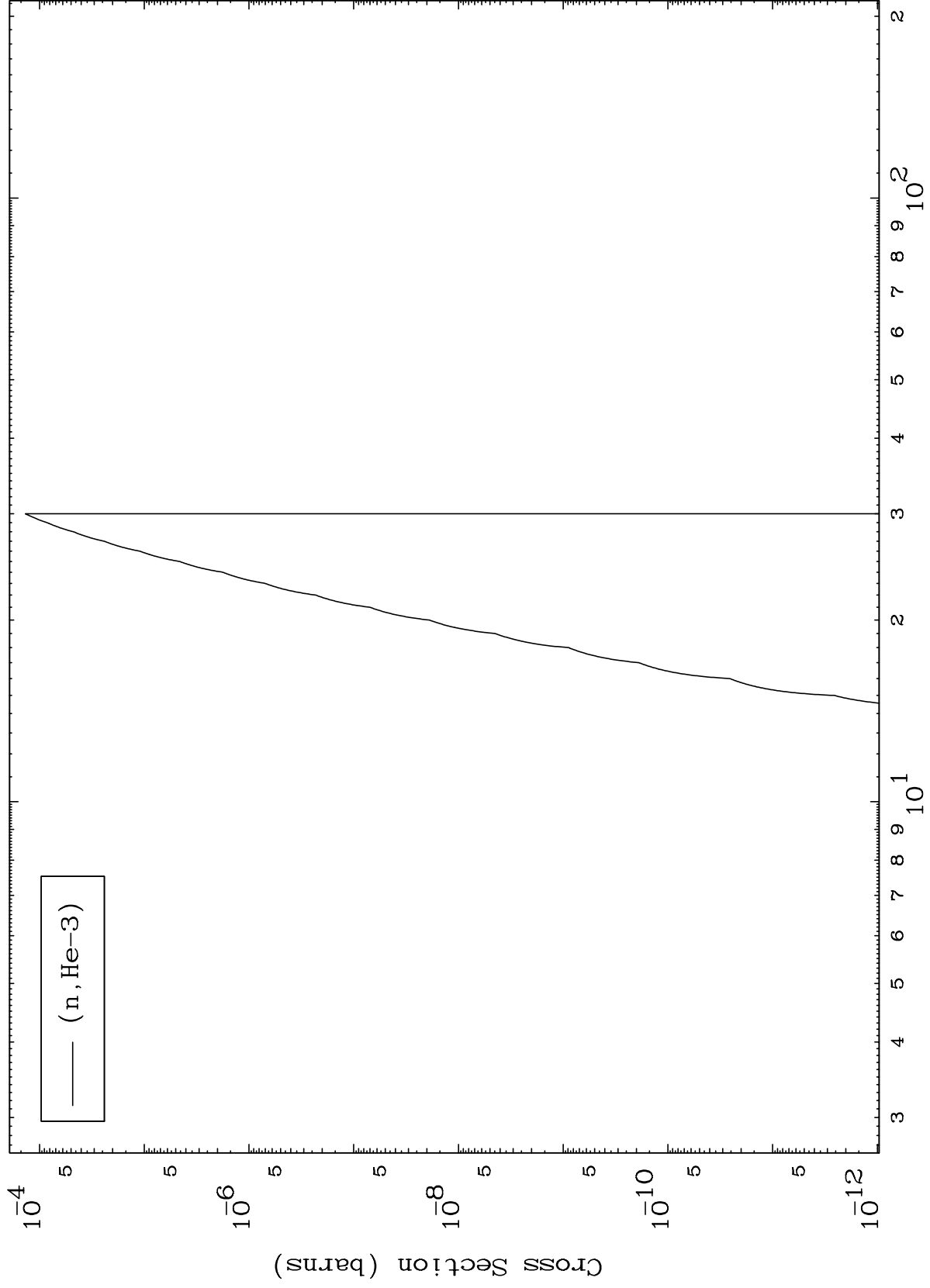
97-Bk-246



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(n,He3) Levels
293 Kelvin Cross Sections

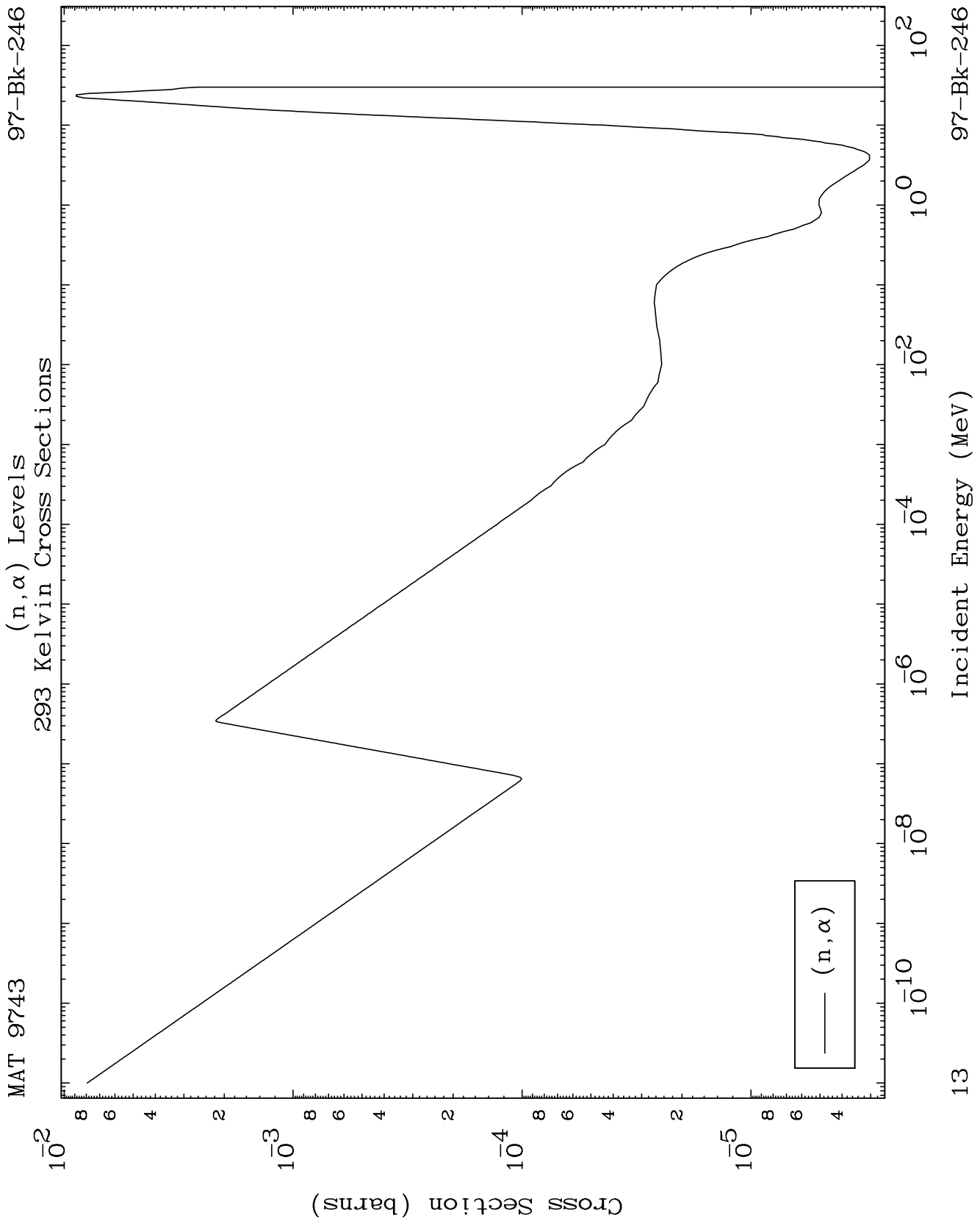
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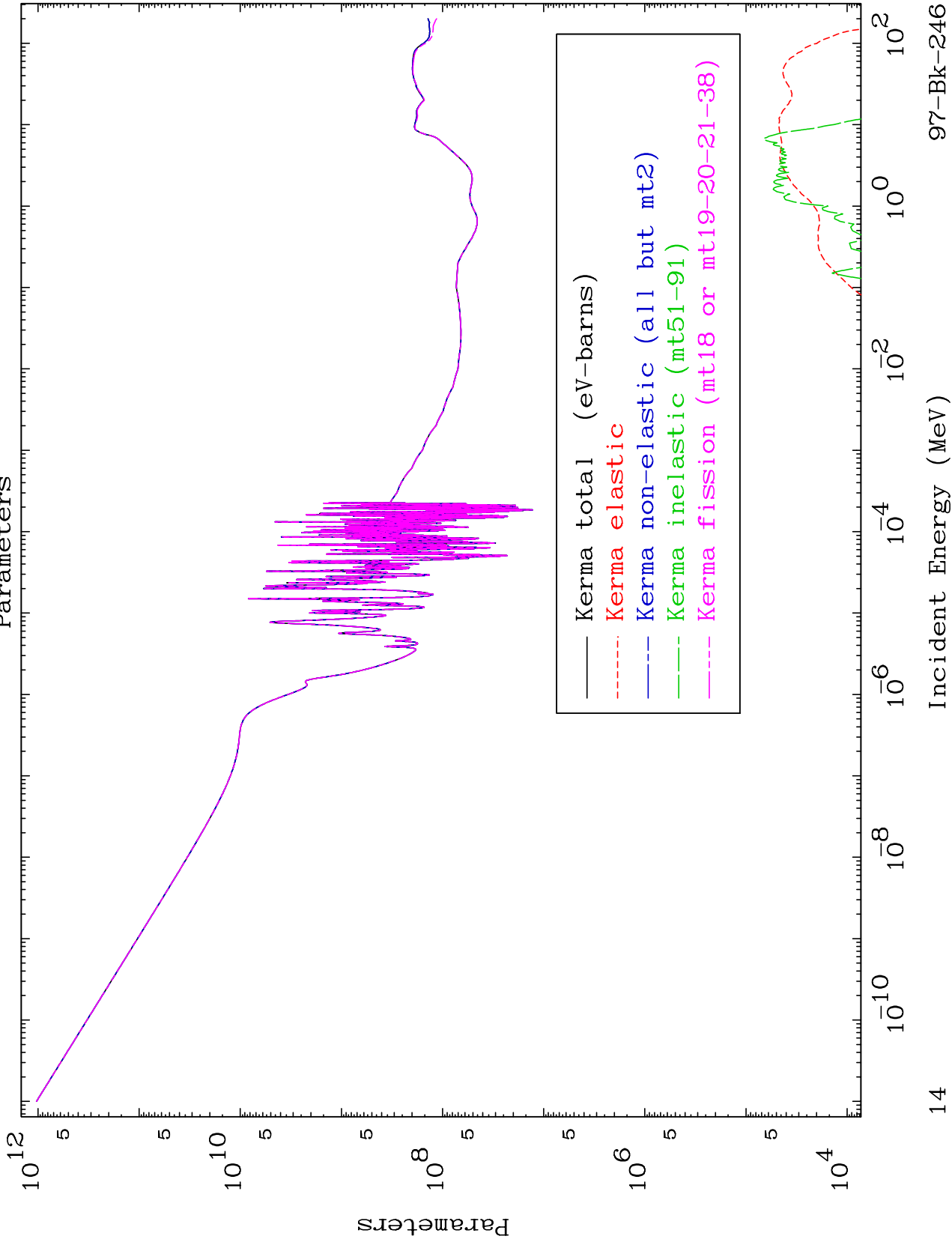


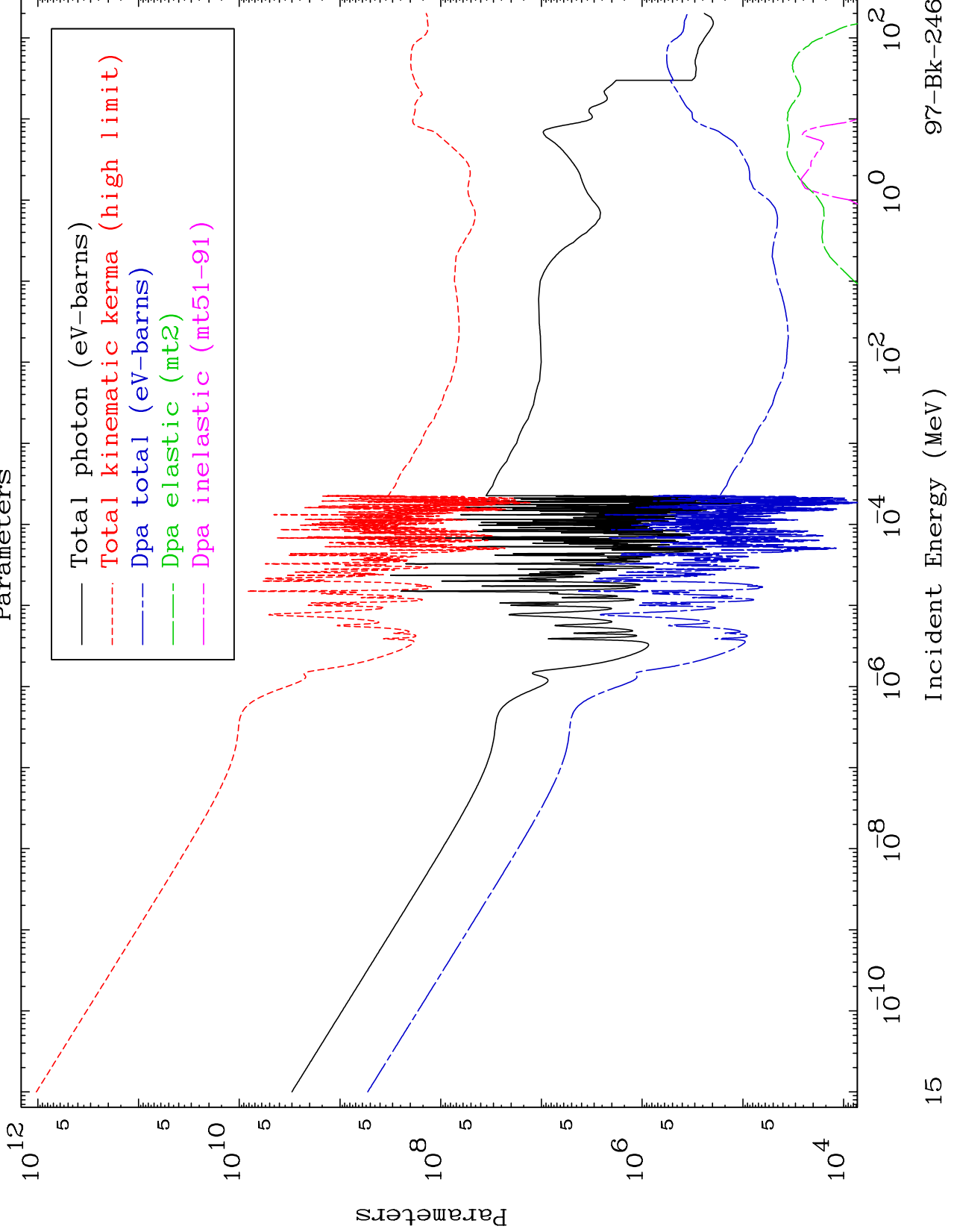
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Incident Energy (MeV)

97-Bk-246



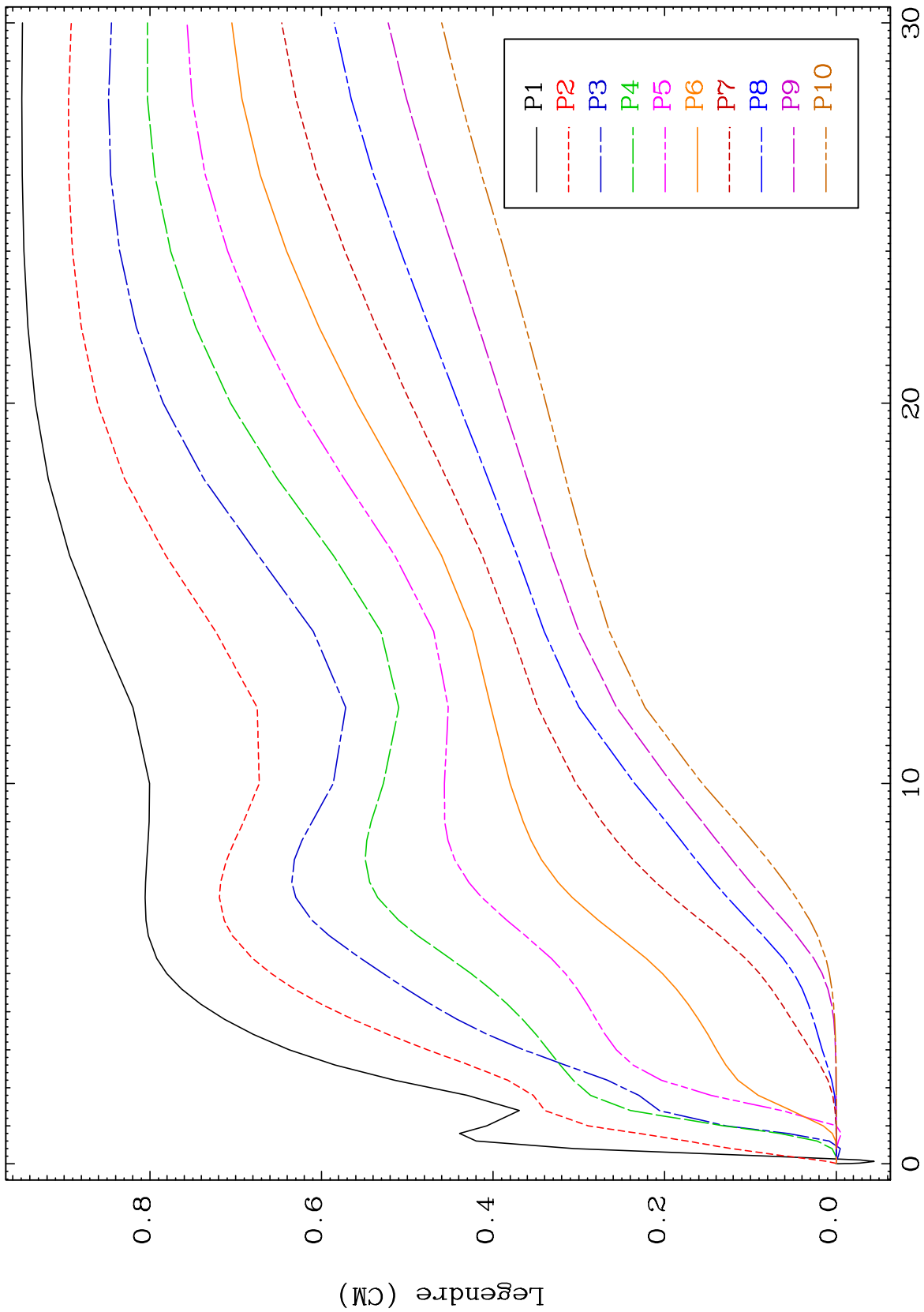




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Elastic Legendre Coefficients

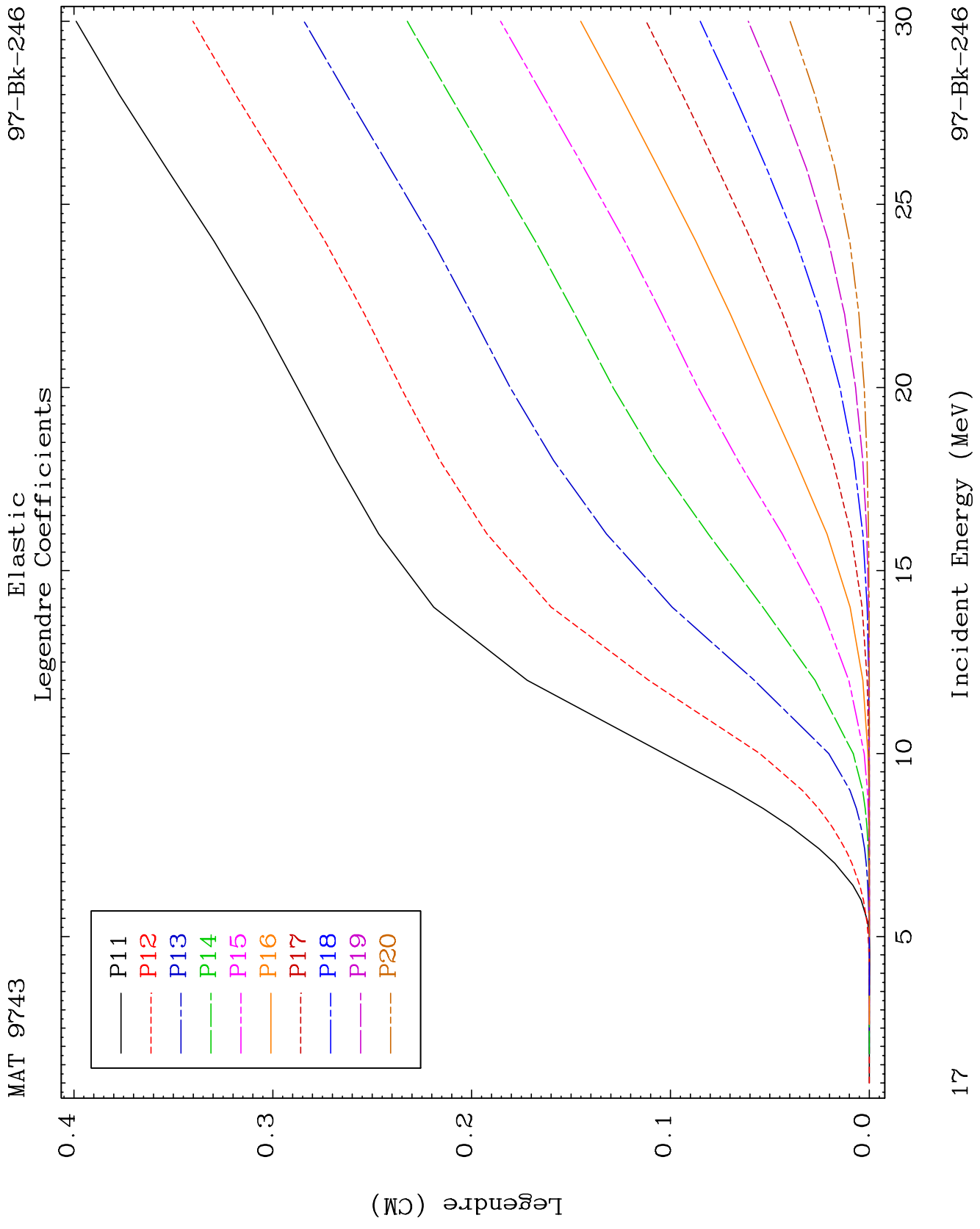
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Incident Energy (MeV)

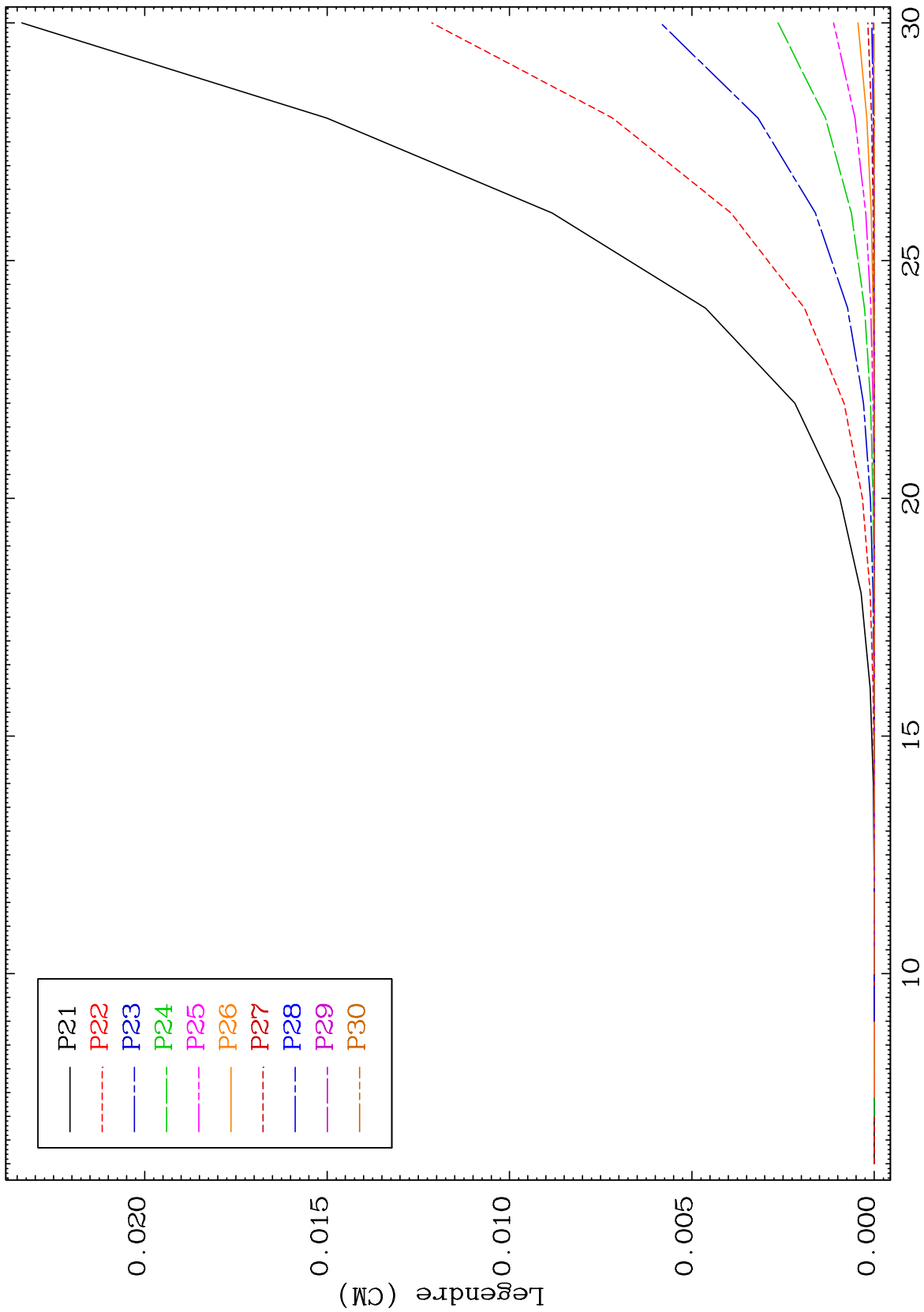
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Elastic Legendre Coefficients

97-Bk-246



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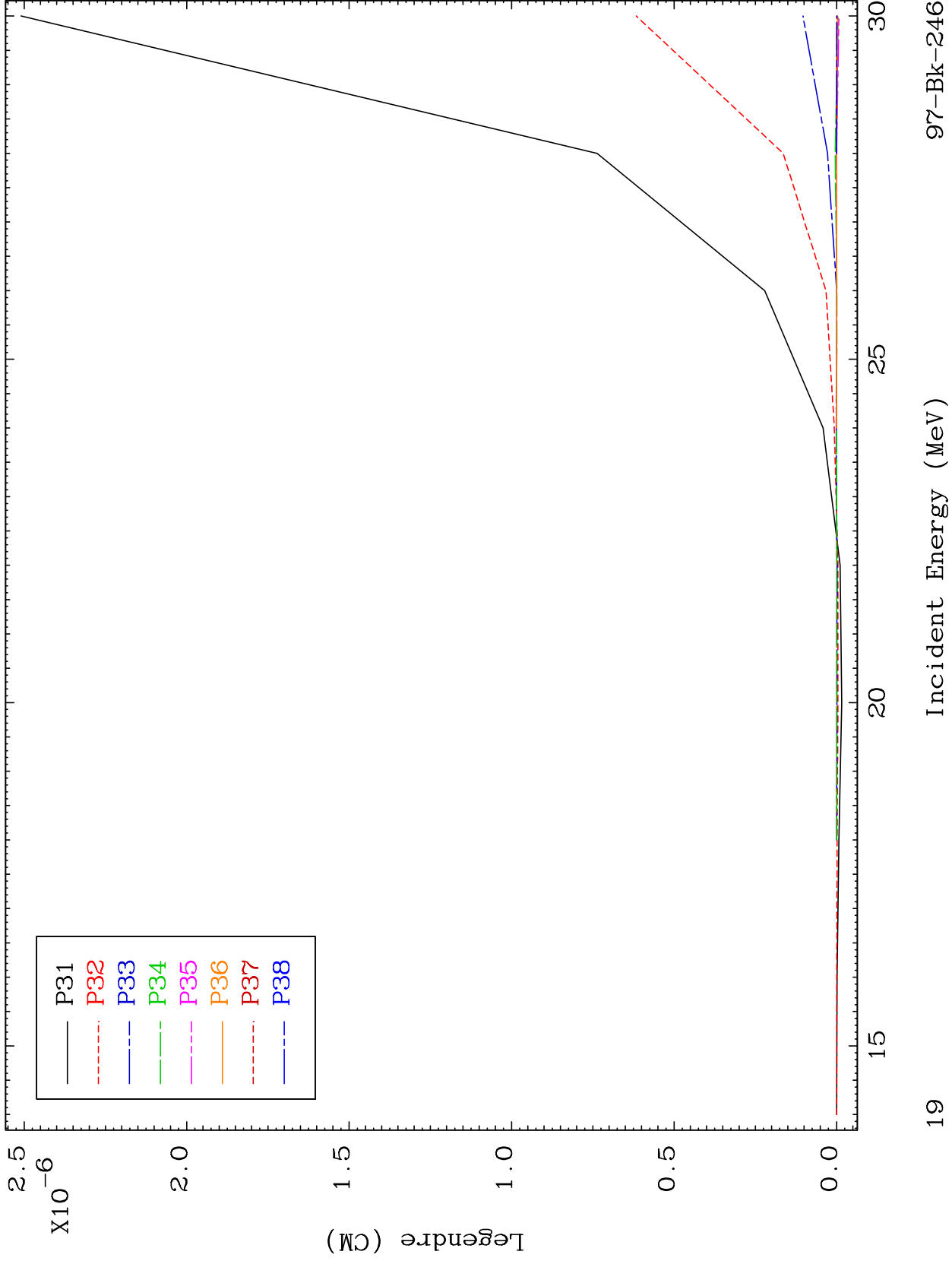
Incident Energy (MeV)

97-Bk-246

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Elastic
Legendre Coefficients

97-Bk-246

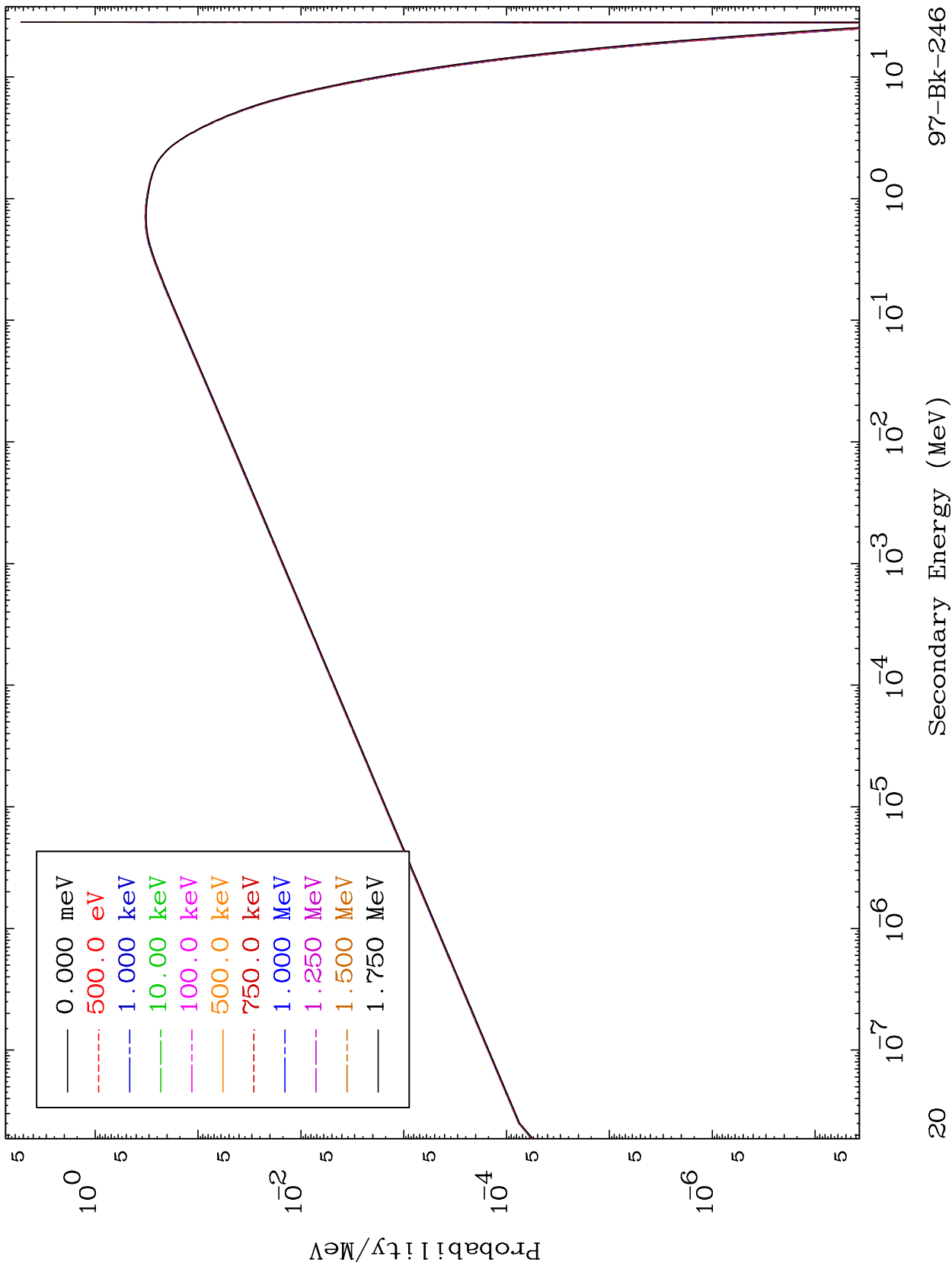


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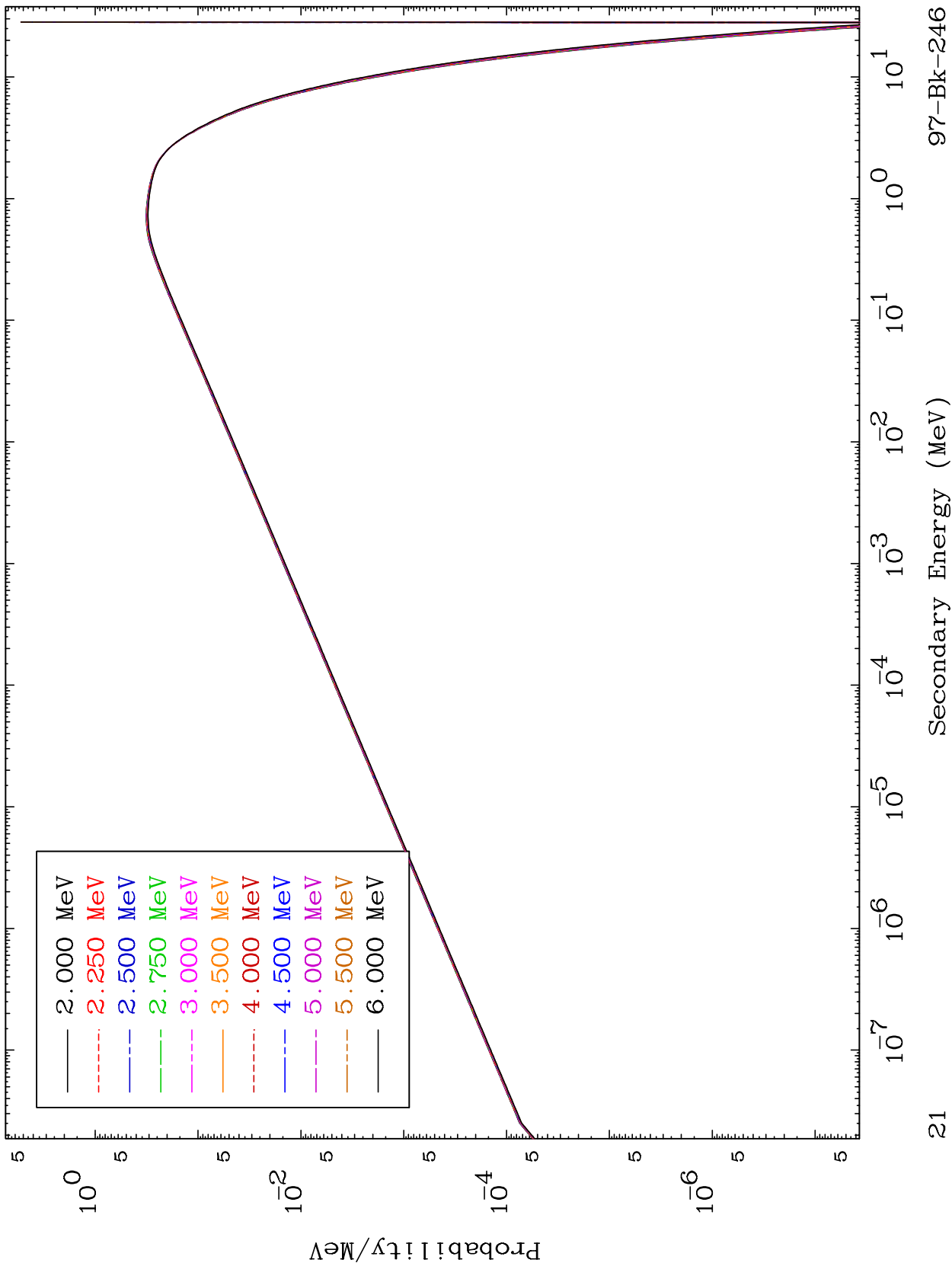
Incident Energy (MeV)

97-Bk-246

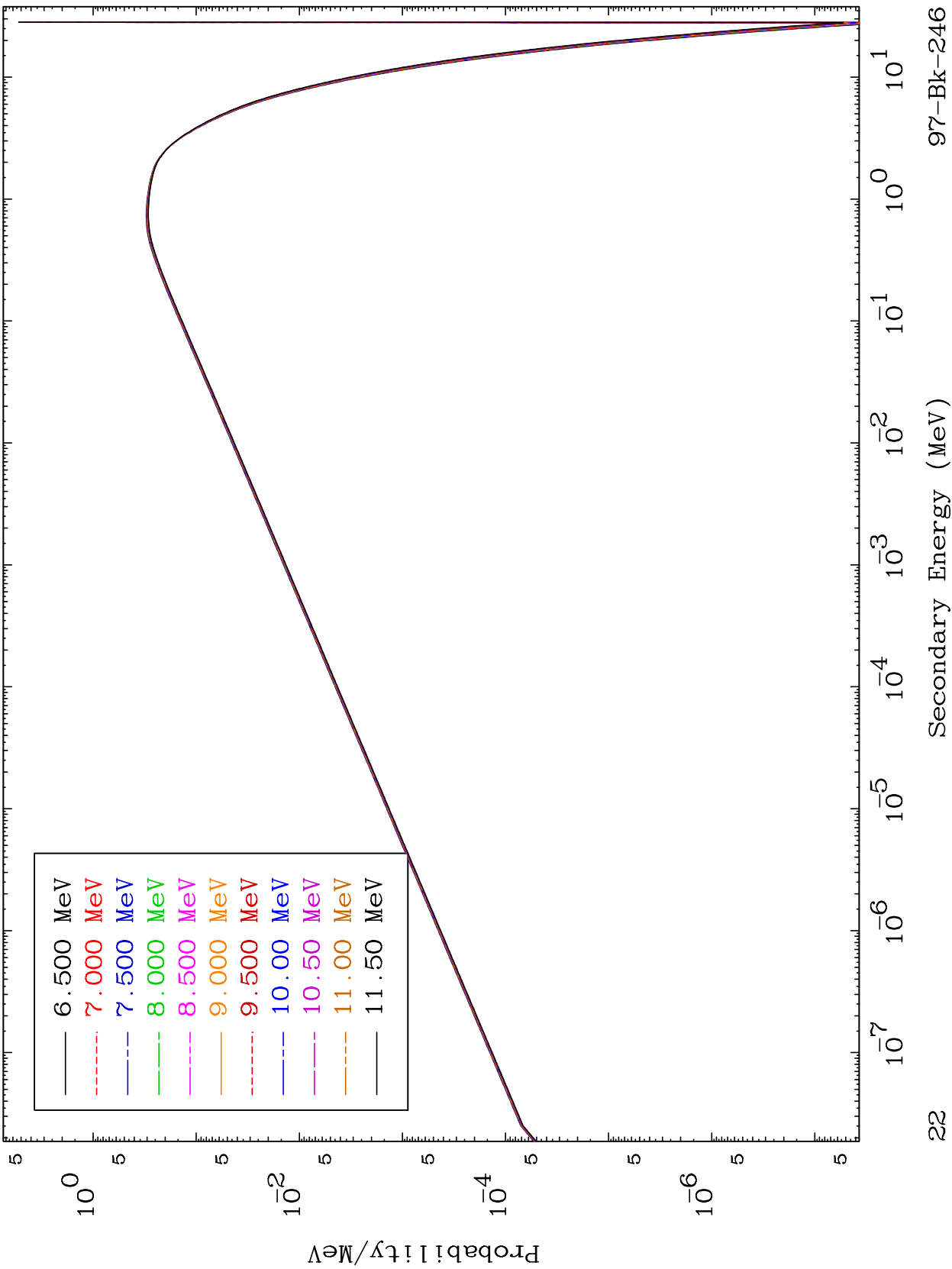
Fission Energy Distribution



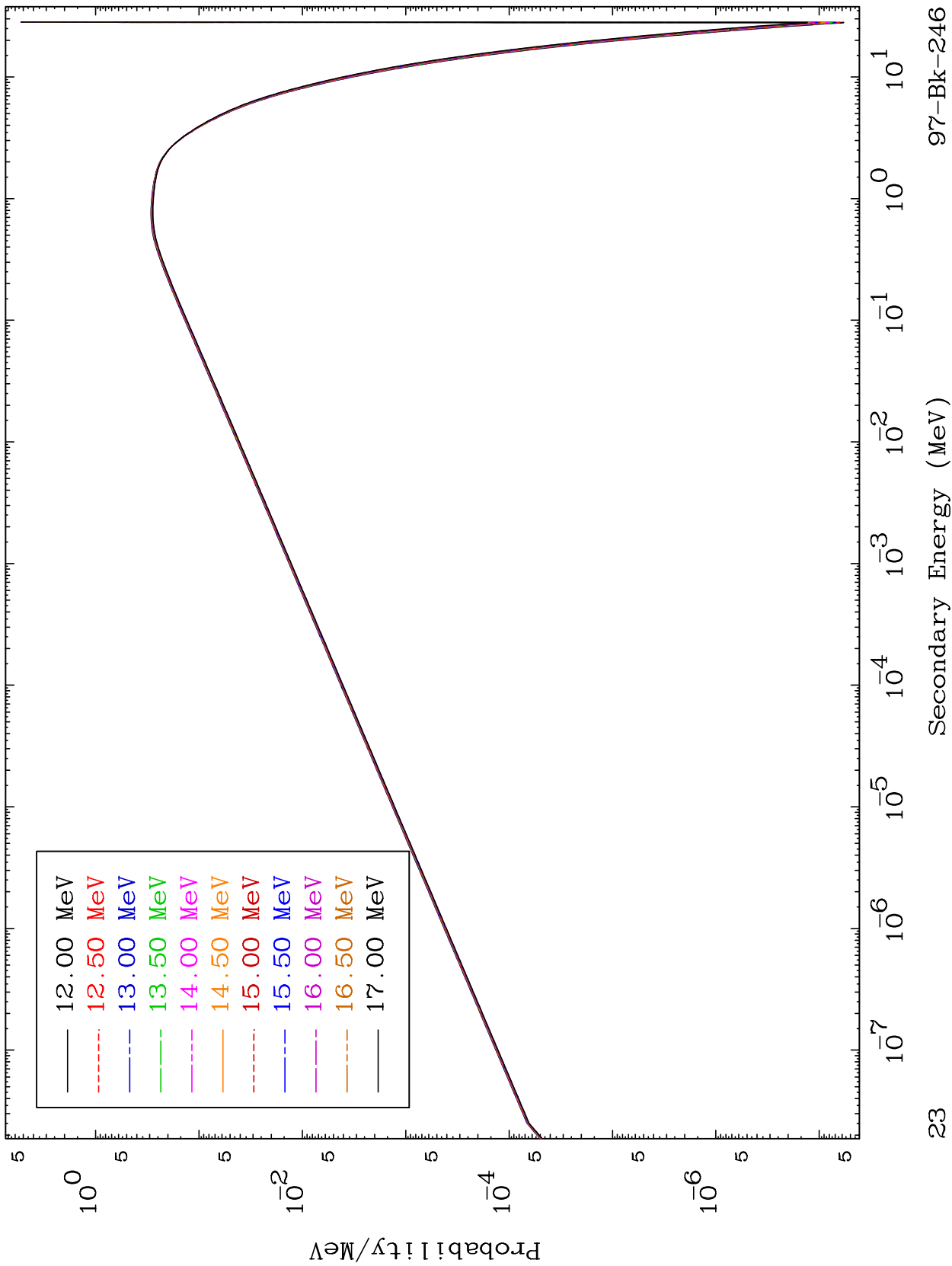
Fission Energy Distribution



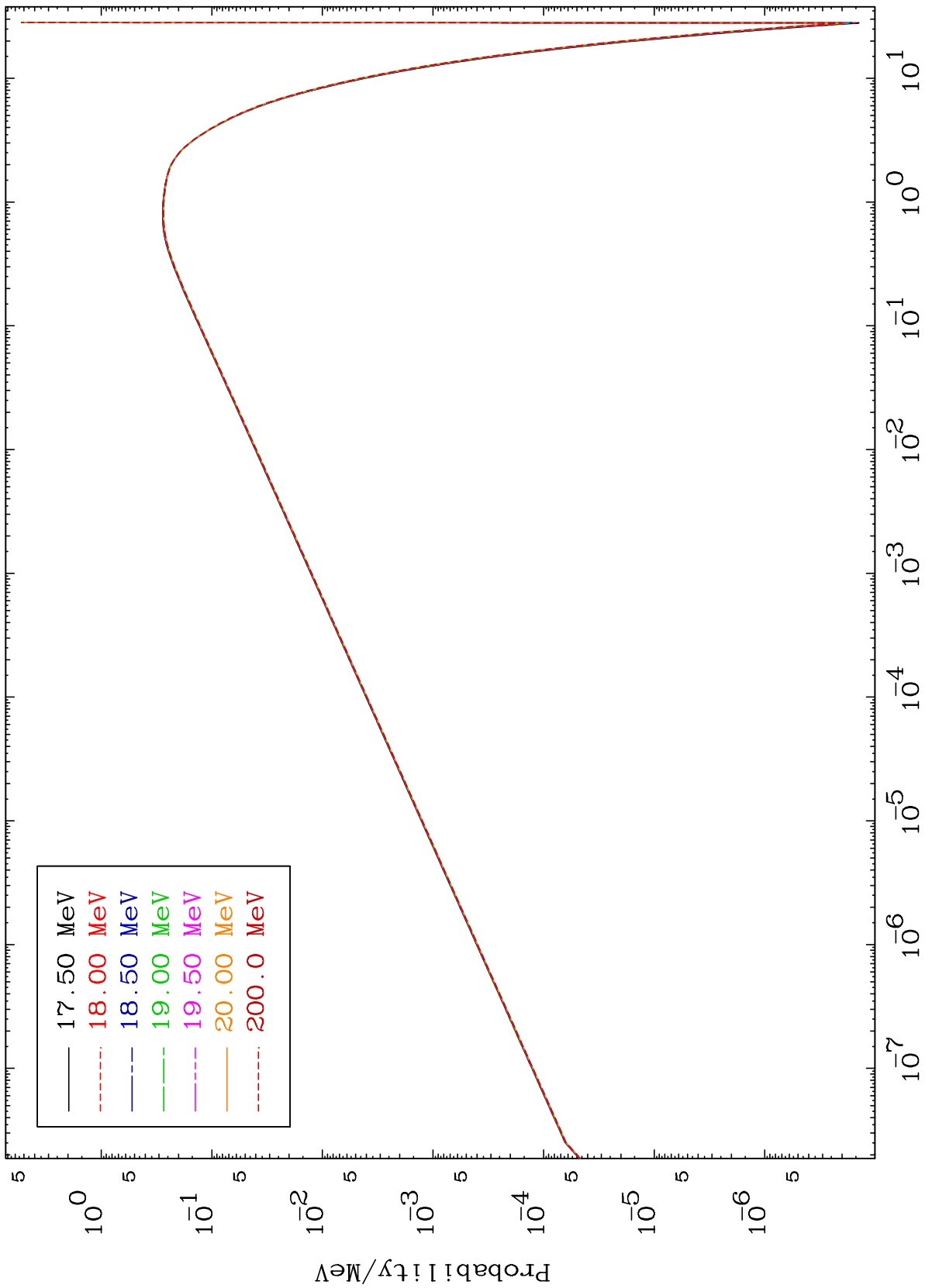
Fission Energy Distribution



Fission
Energy Distribution



Fission
Energy Distribution

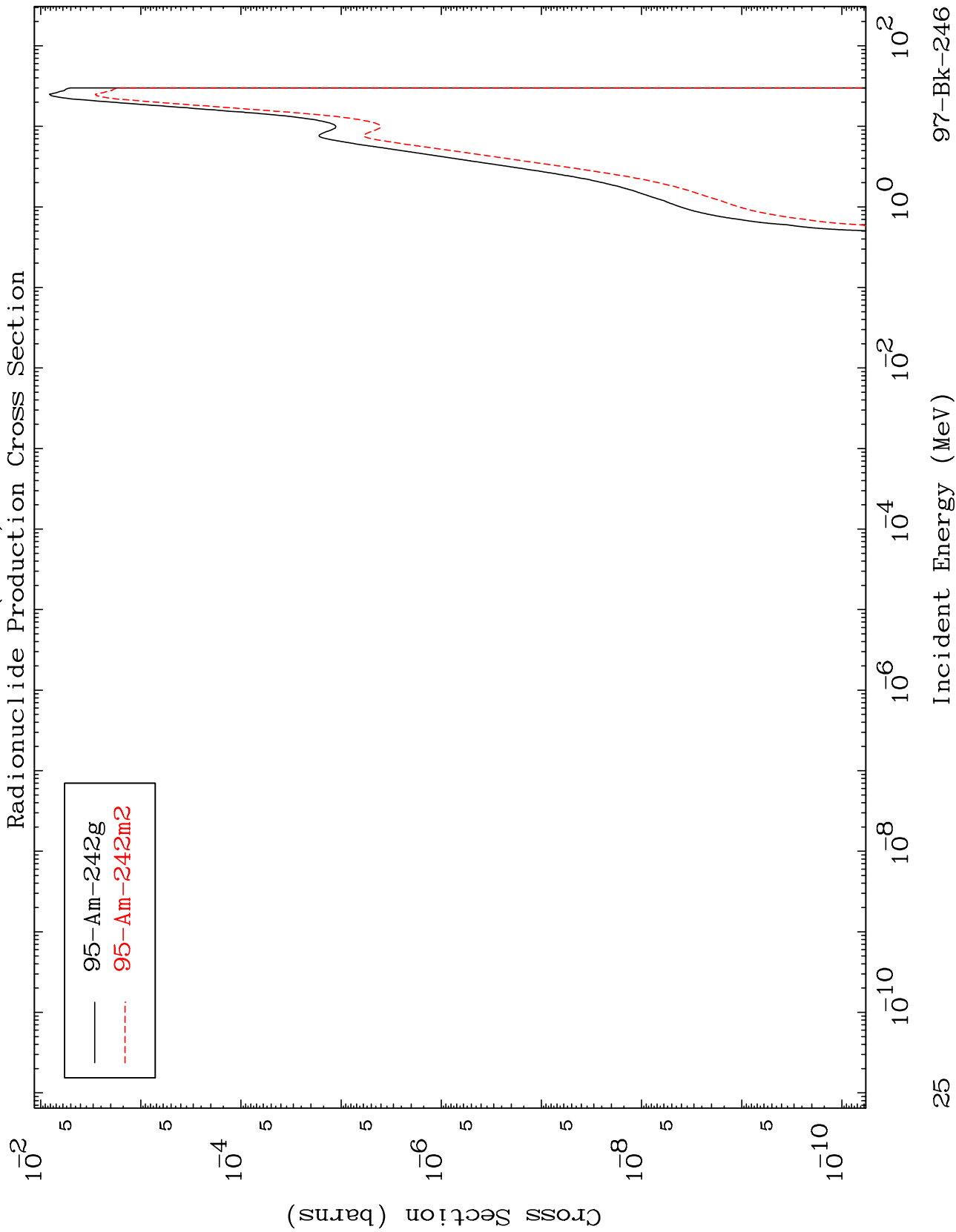


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(n,n') α

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Radionuclide Production Cross Section

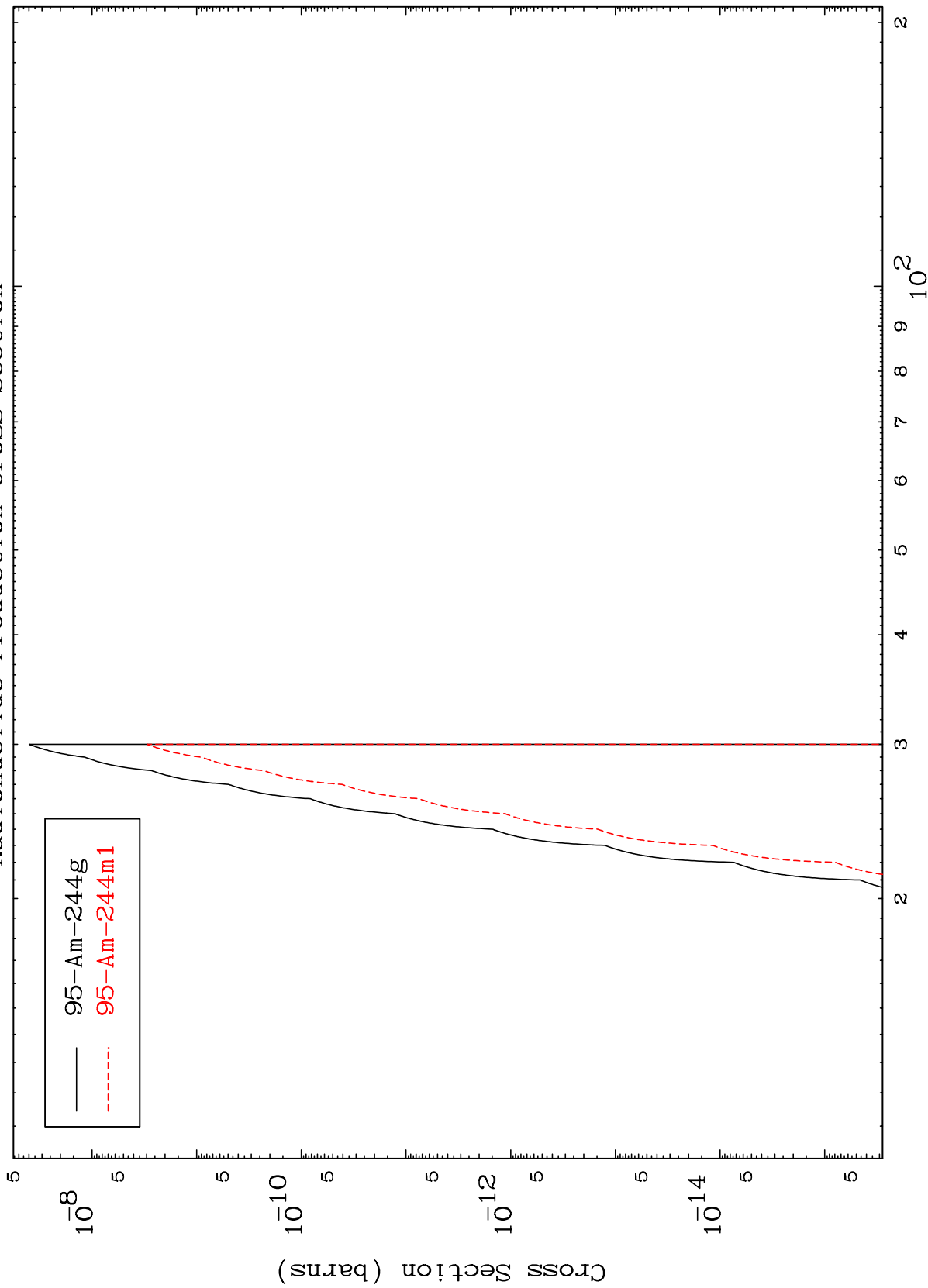


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(n,2n) p

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Radionuclide Production Cross Section



Incident Energy (MeV)

97-Bk-246

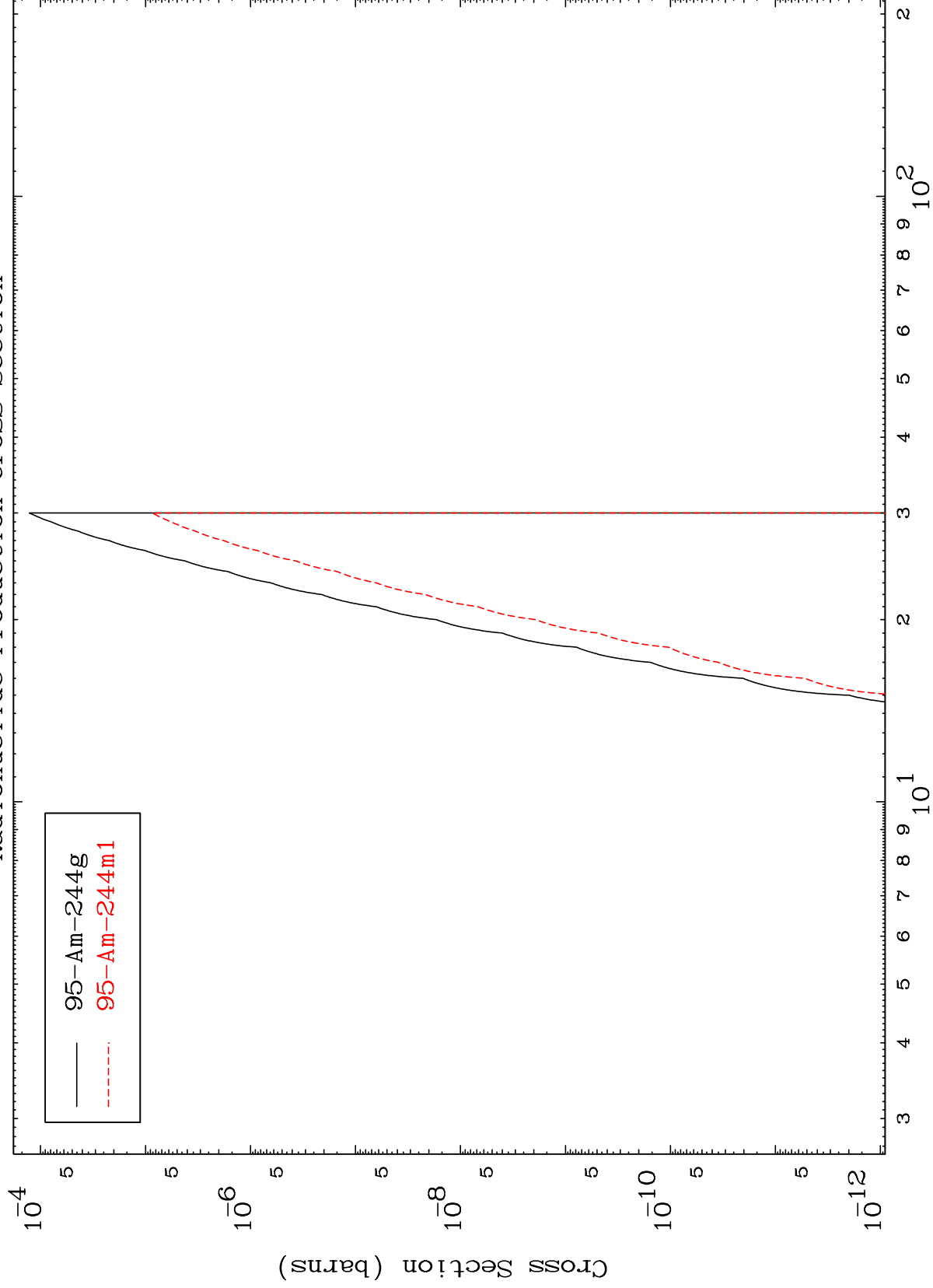
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(n,He-3)

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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