

Program EVALPLOT  
(Version 2015-2)

by

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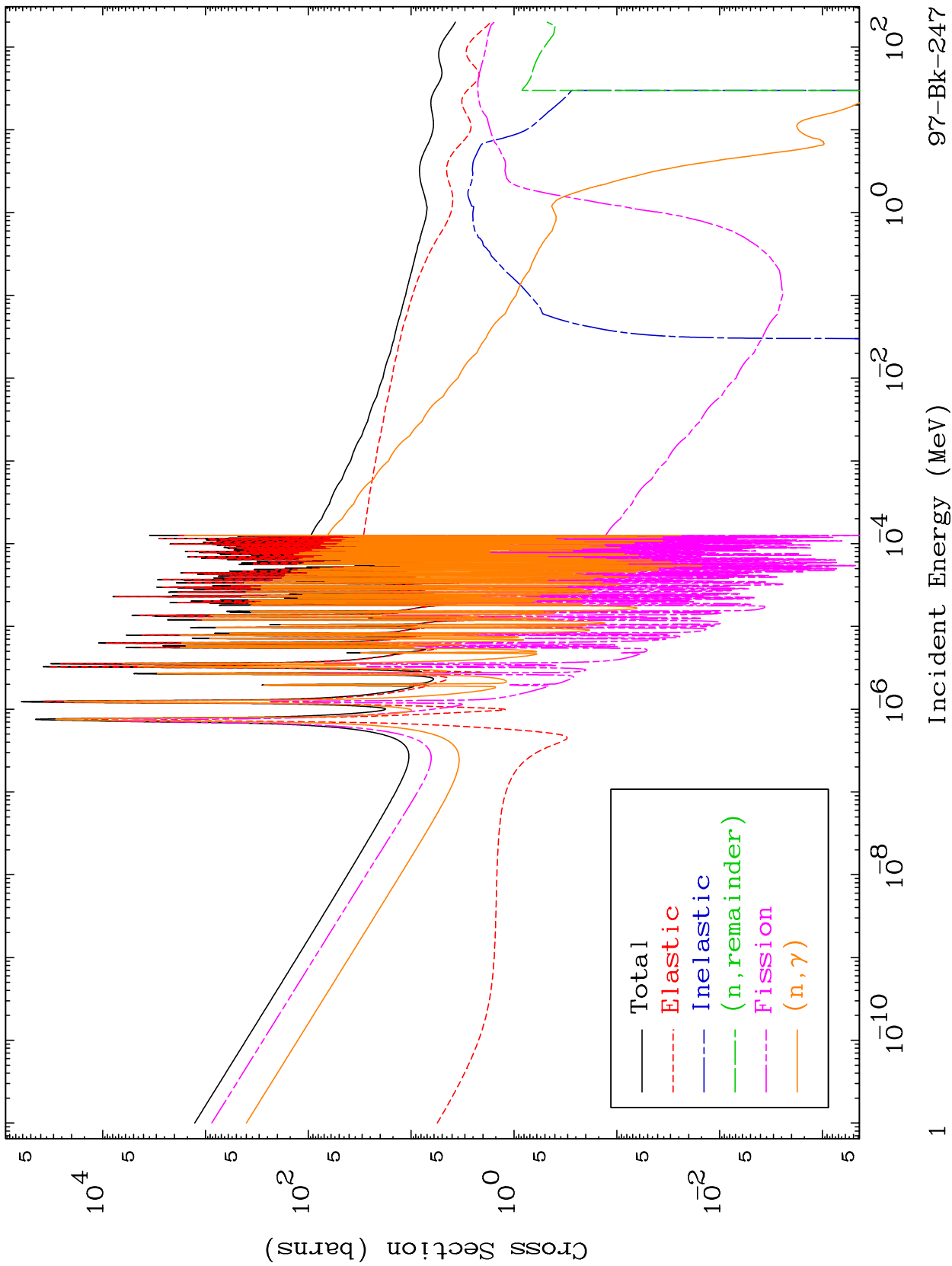
Press Mouse Button to Start

MAT 9746

Major

293 Kelvin Cross Sections

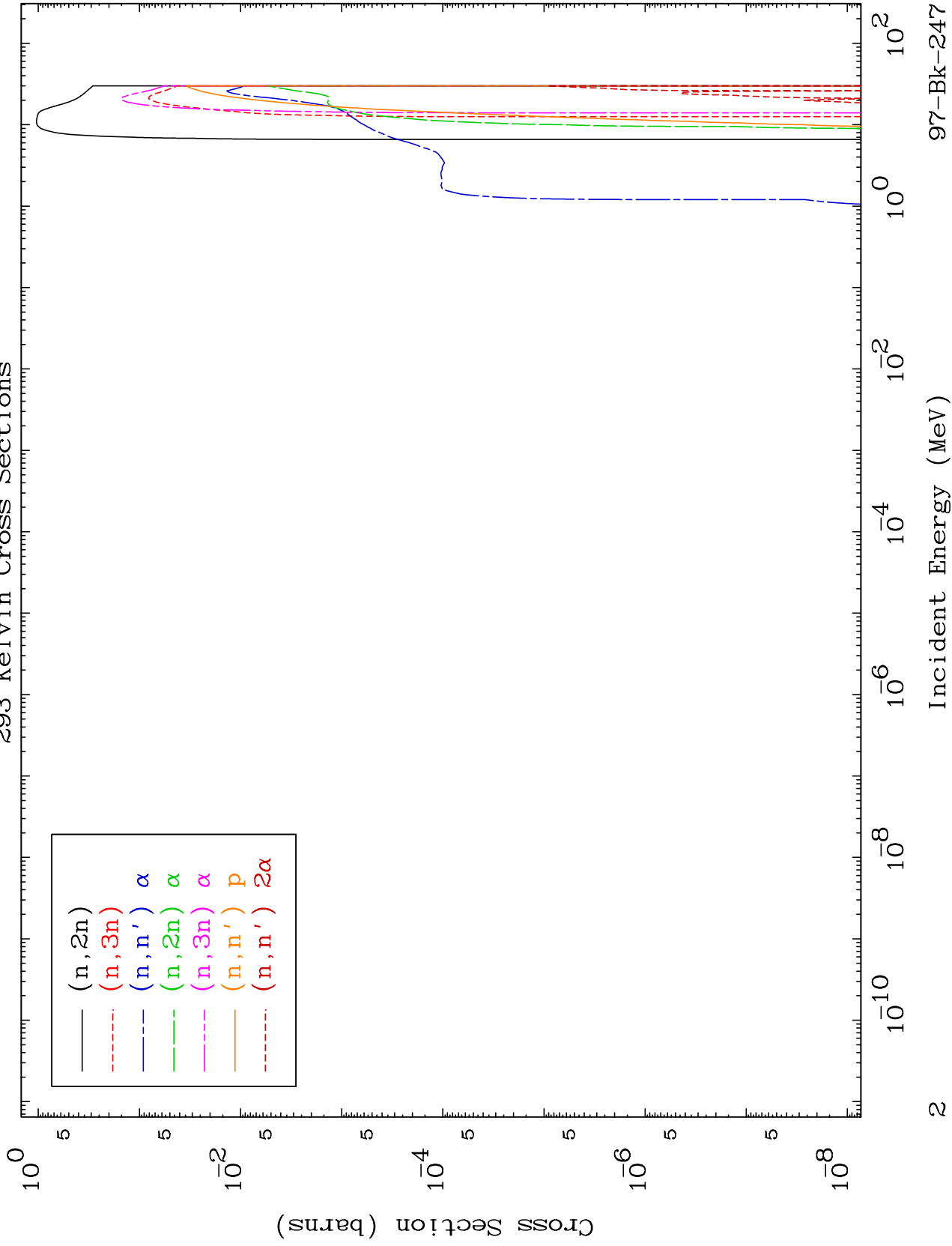
97-Bk-247



MAT 9746

Neutron Production  
293 Kelvin Cross Sections

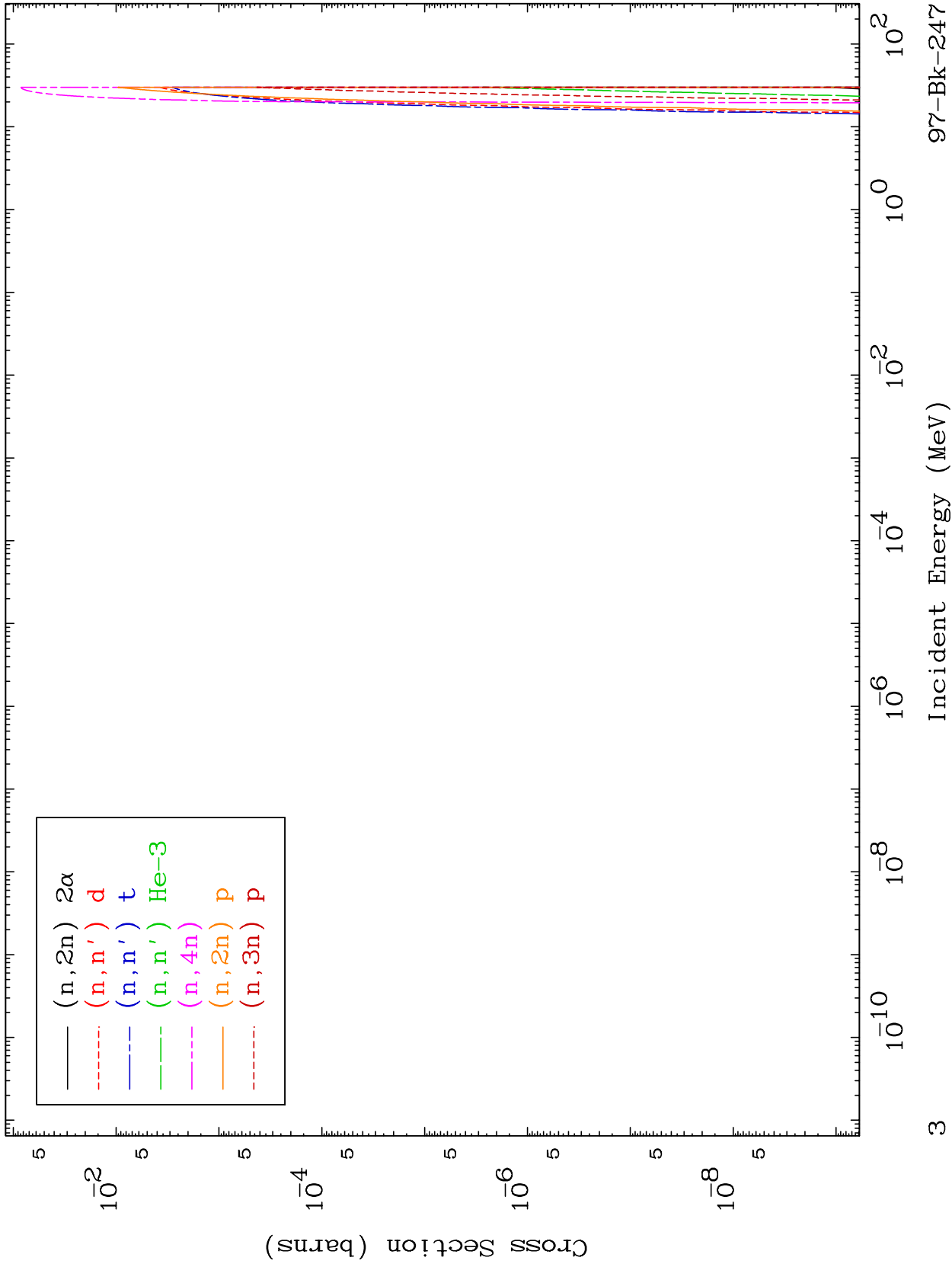
97-Bk-247



MAT 9746

Neutron Production  
293 Kelvin Cross Sections

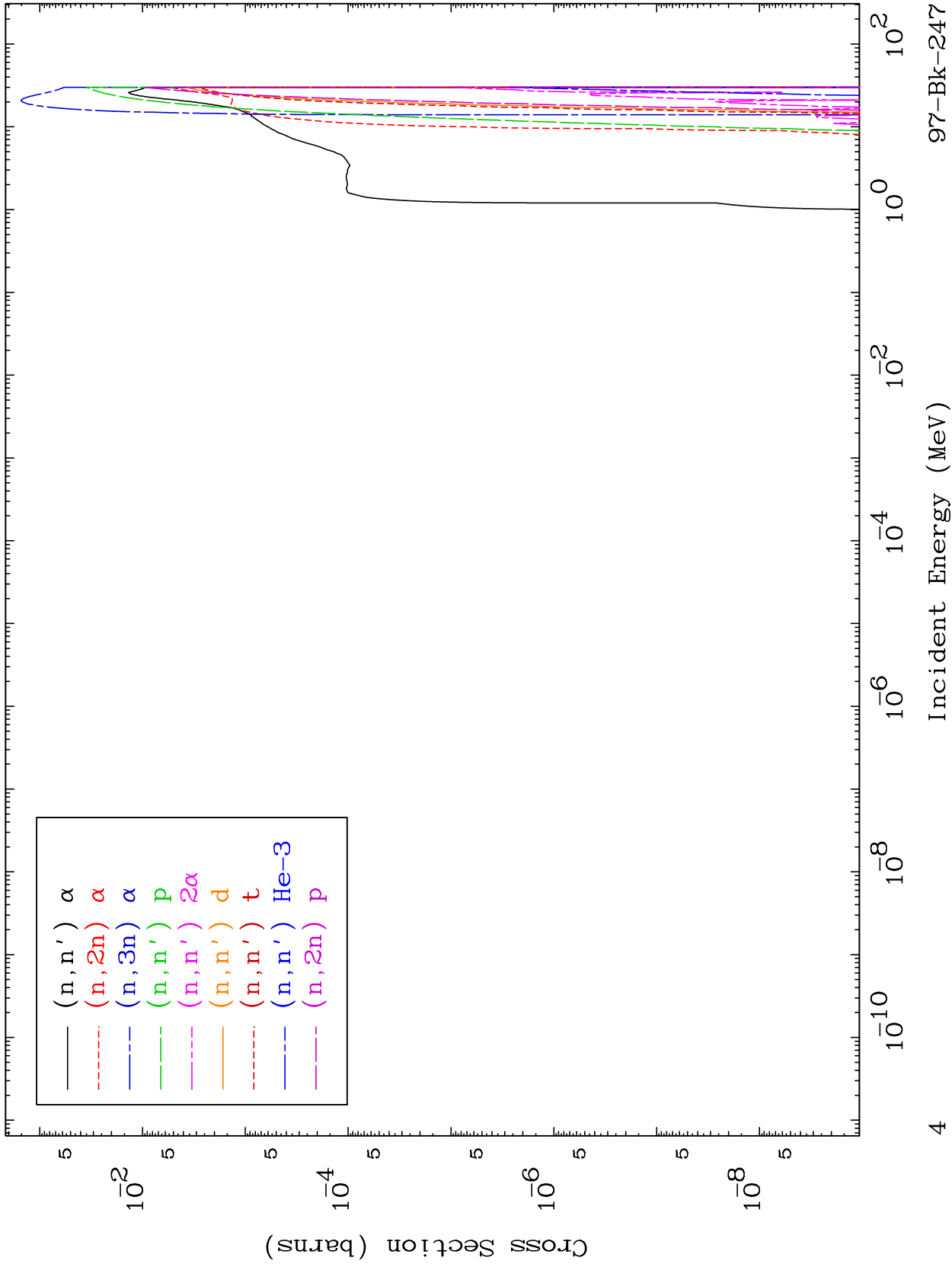
97-Bk-247



MAT 9746

Charged Particle  
293 Kelvin Cross Sections

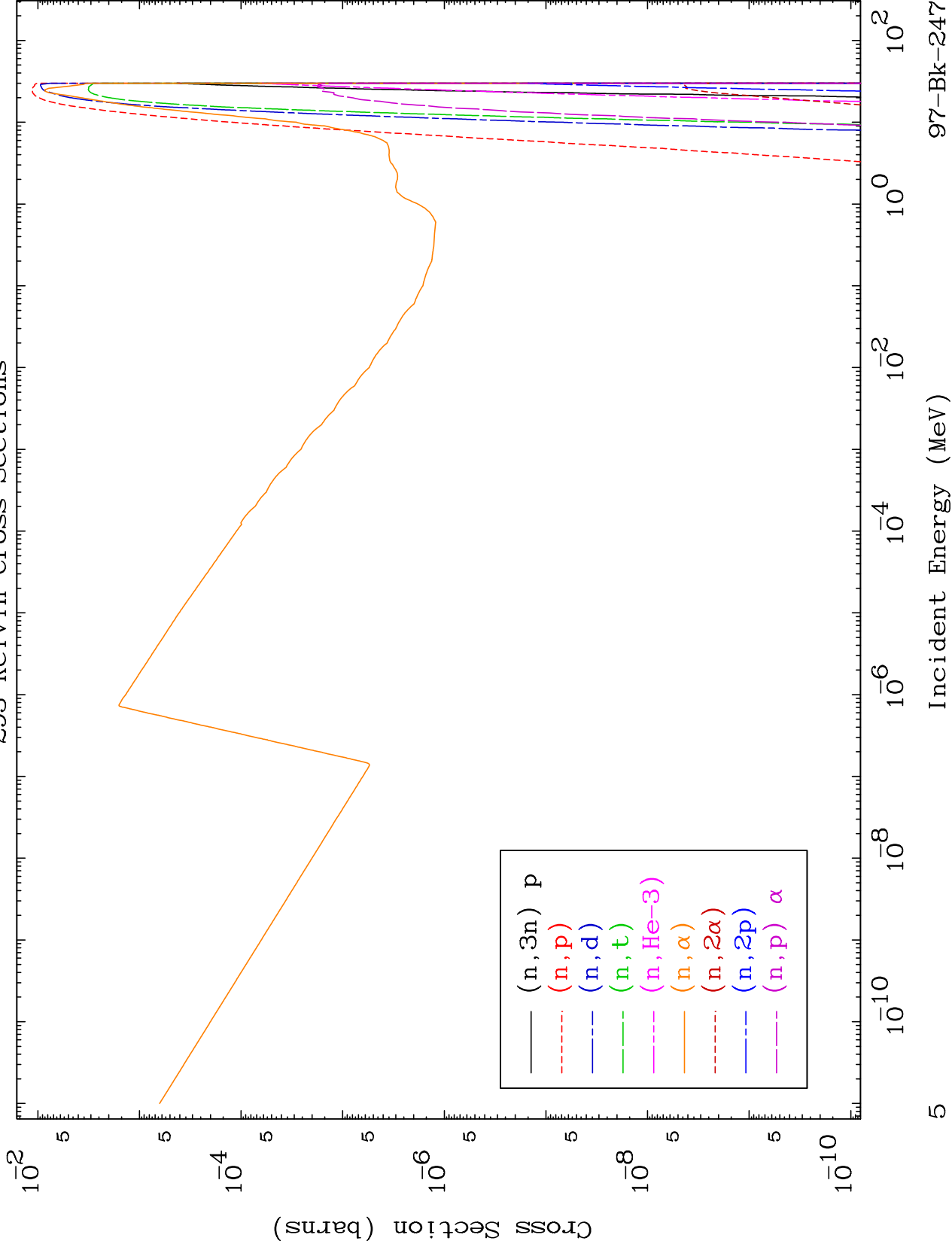
97-Bk-247



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Charged Particle  
293 Kelvin Cross Sections

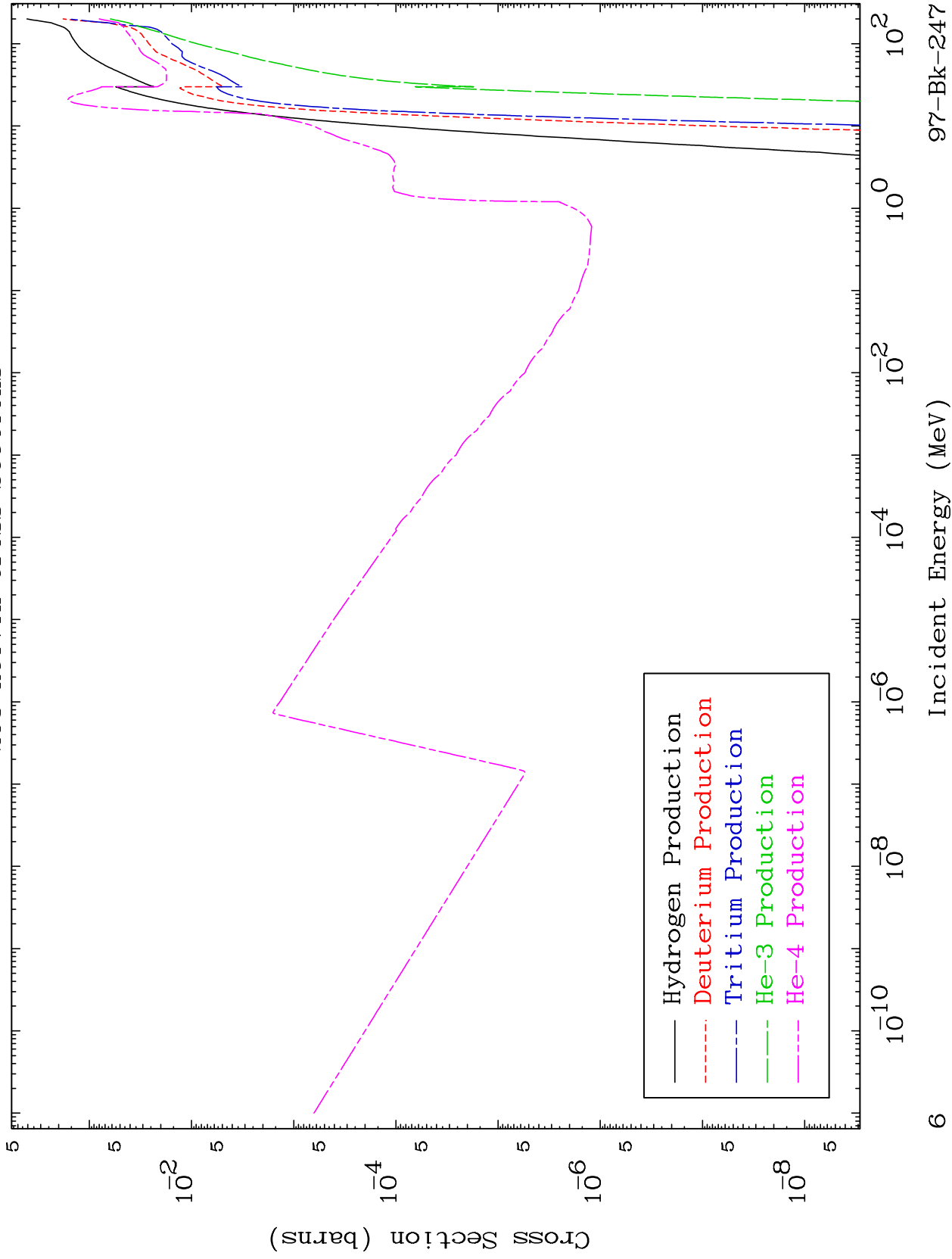
97-Bk-247



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Particle Production  
293 Kelvin Cross Sections

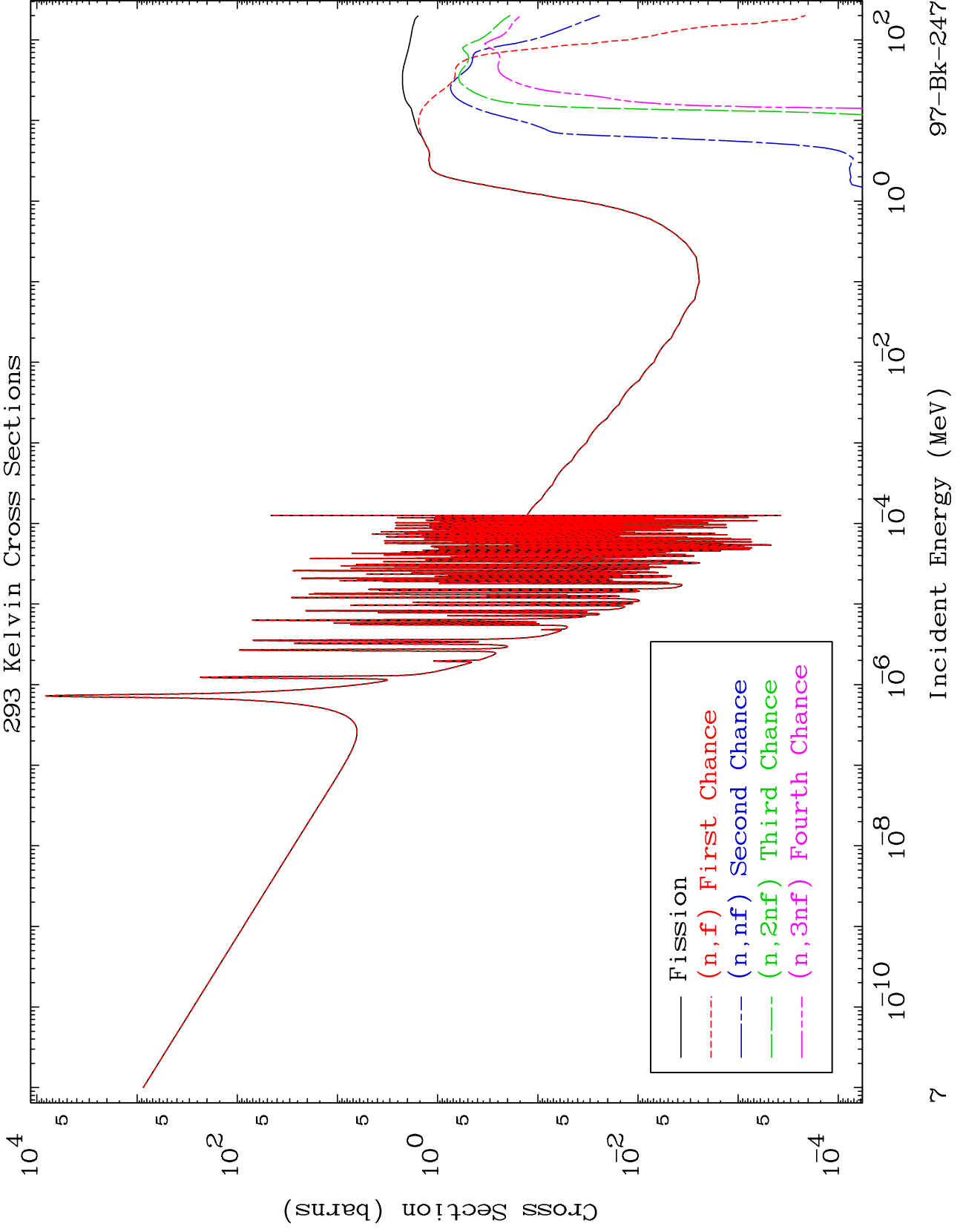
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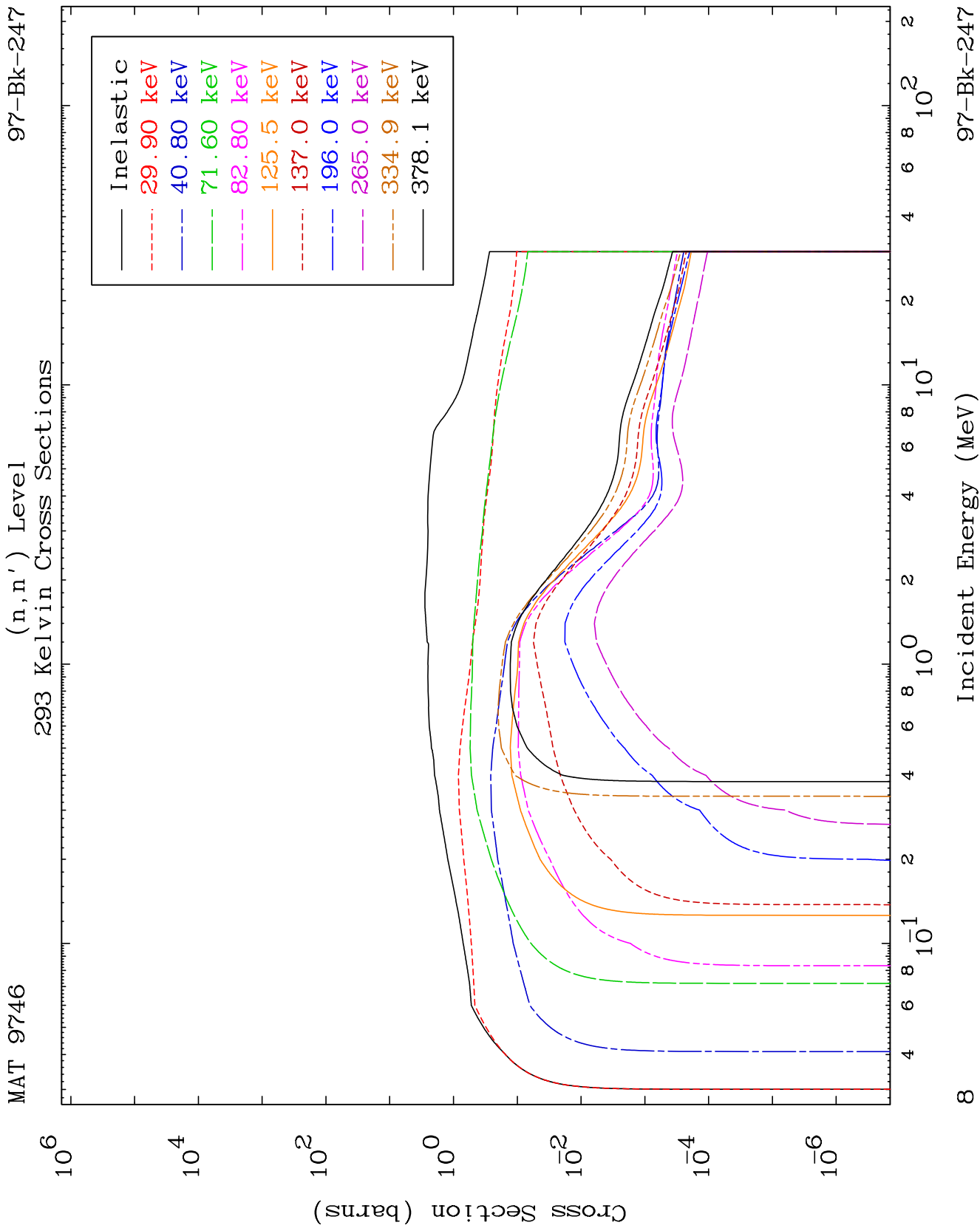
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Fission  
293 Kelvin Cross Sections

97-Bk-247



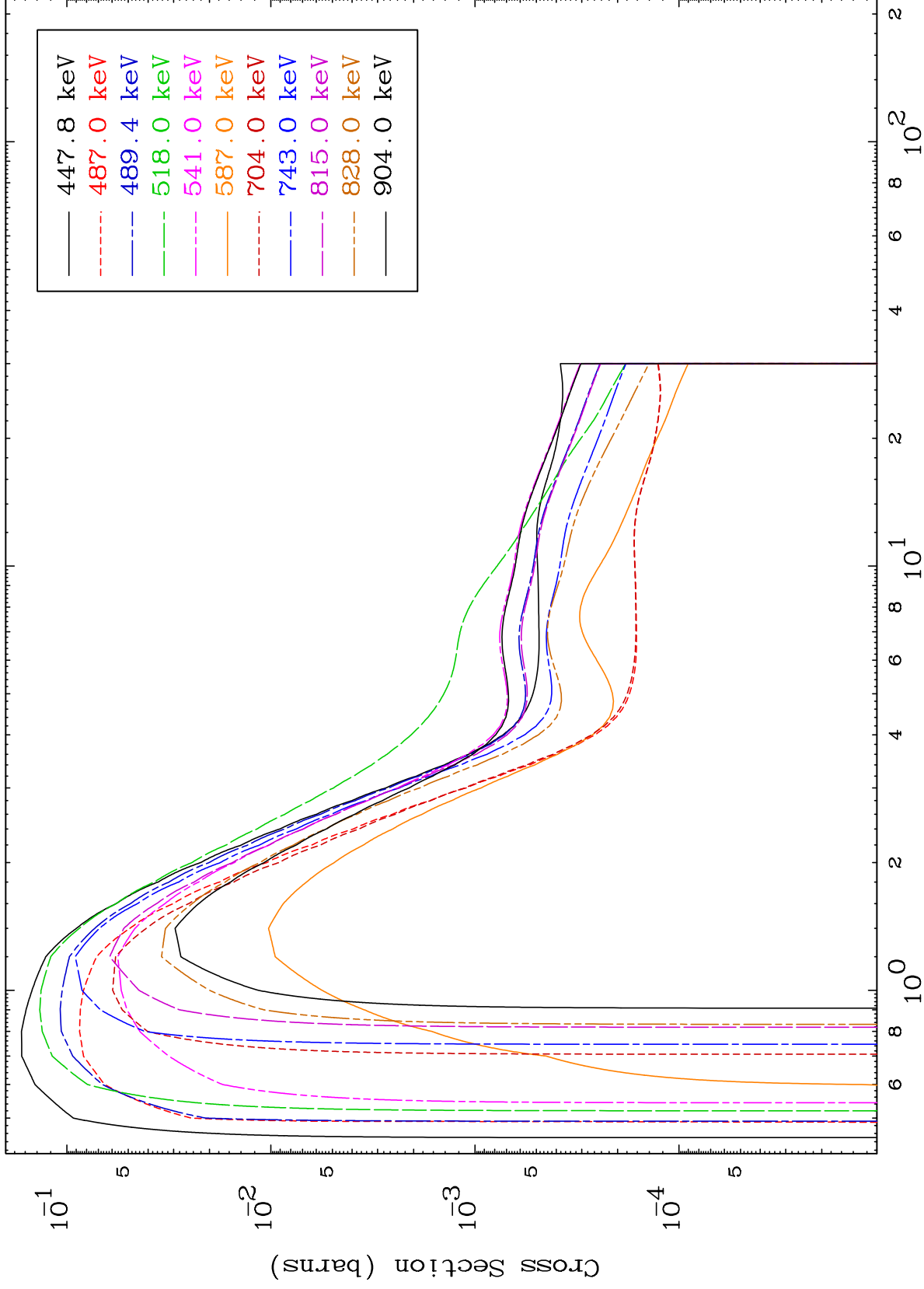




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(n,n') Level  
293 Kelvin Cross Sections

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9

Incident Energy (MeV)

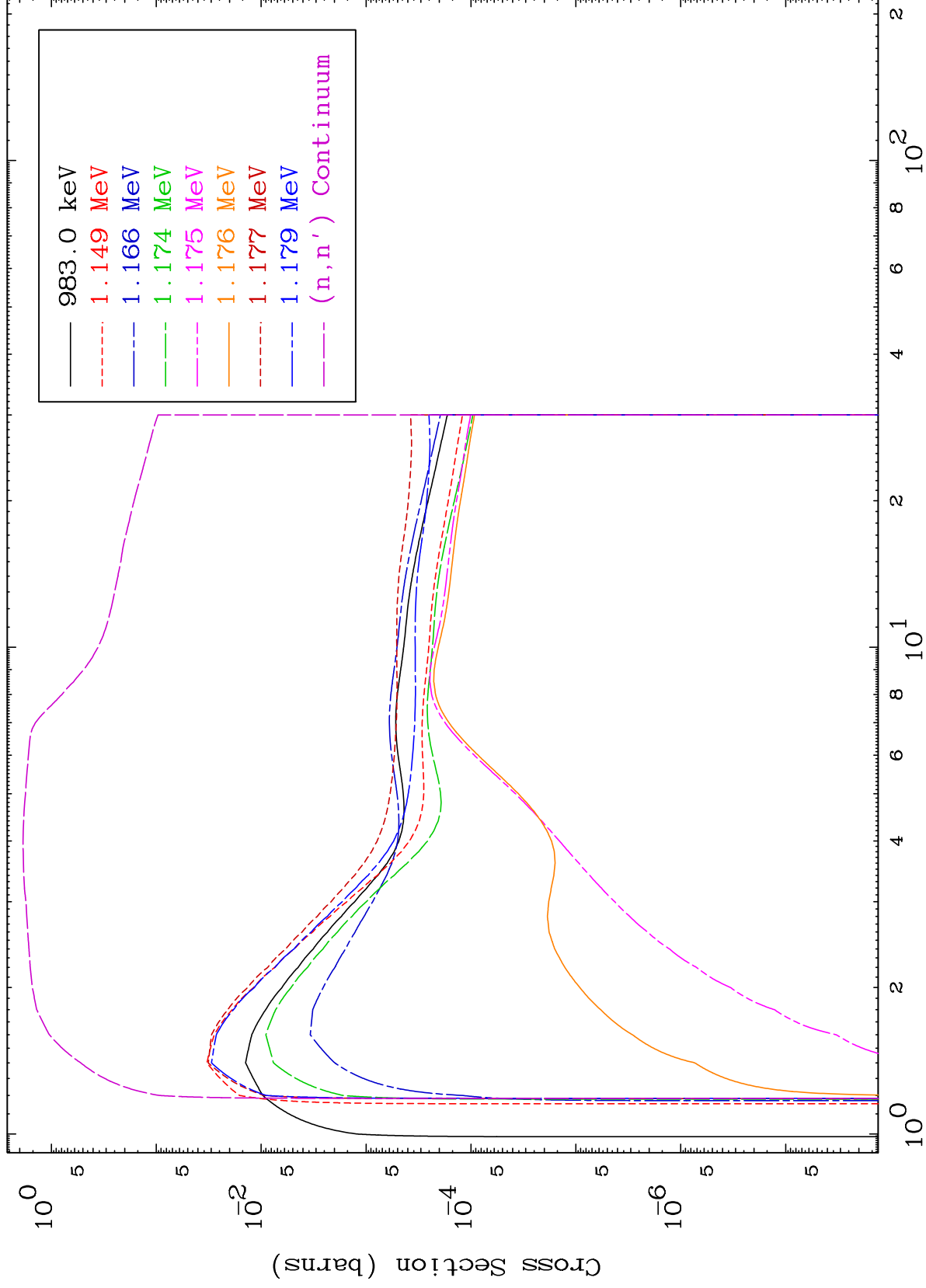
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(n,n') Level

97-Bk-247

293 Kelvin Cross Sections



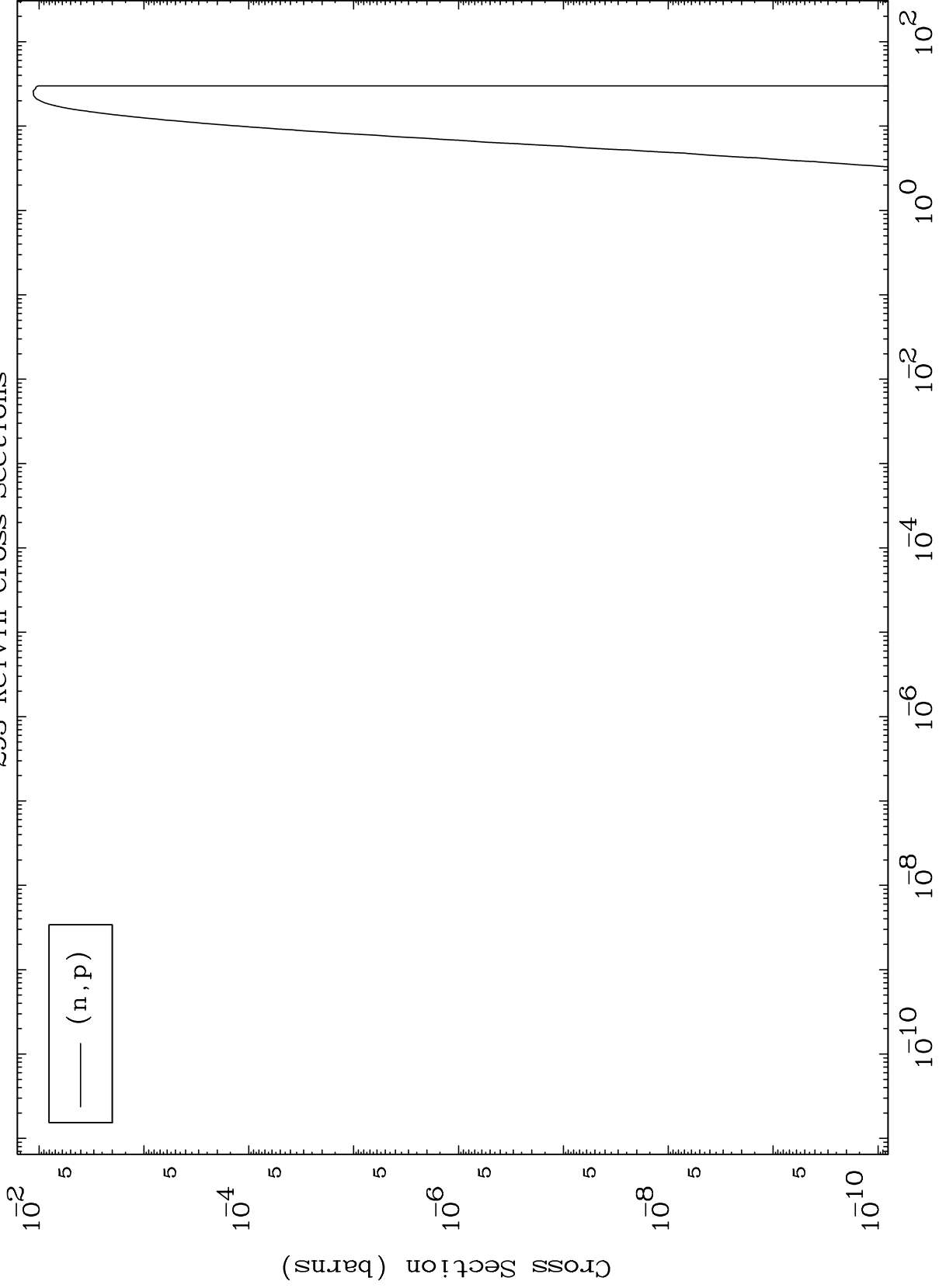
Incident Energy (MeV)

97-Bk-247

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(n,p) Levels  
293 Kelvin Cross Sections

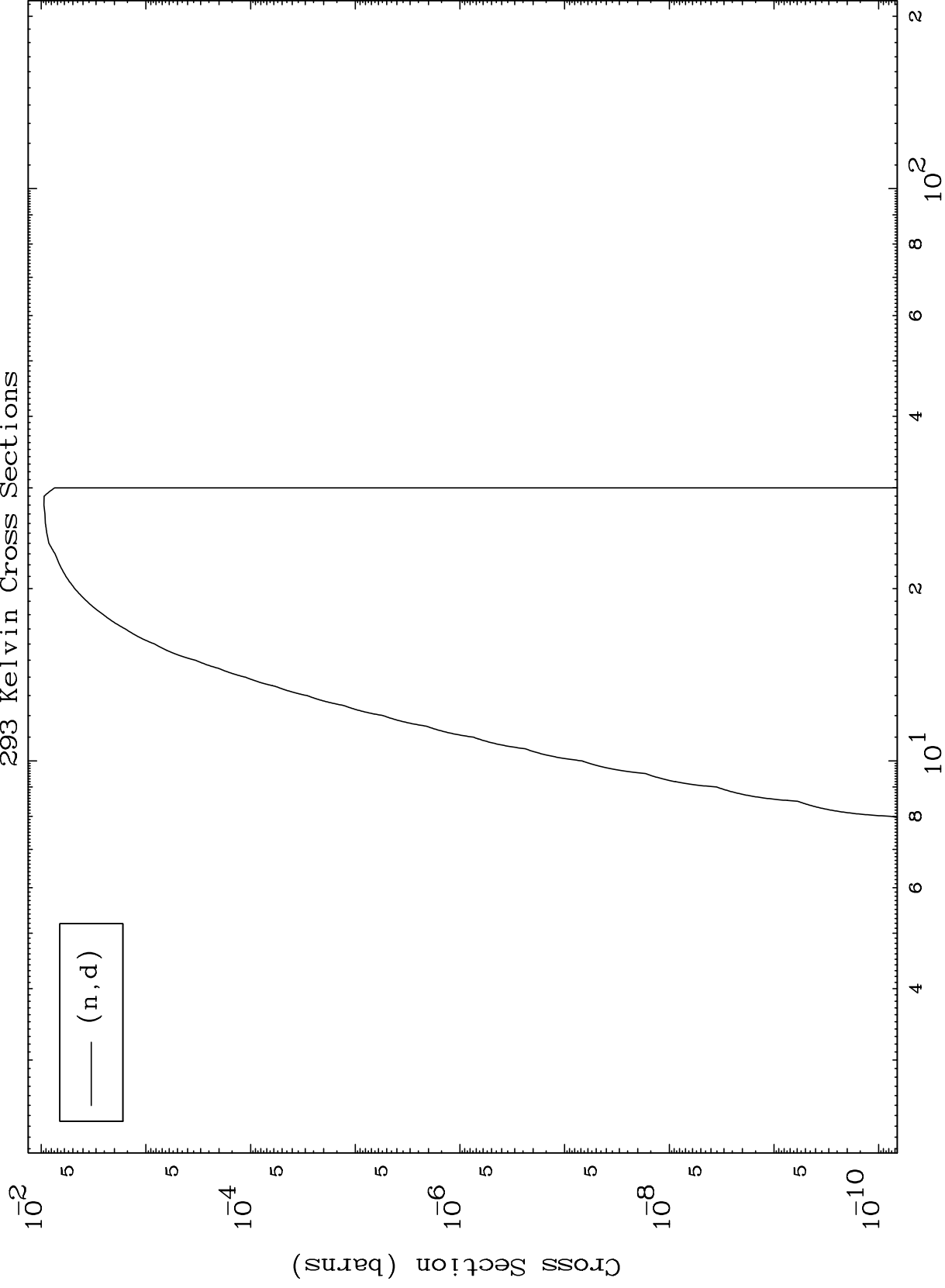
97-Bk-247



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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-247



12

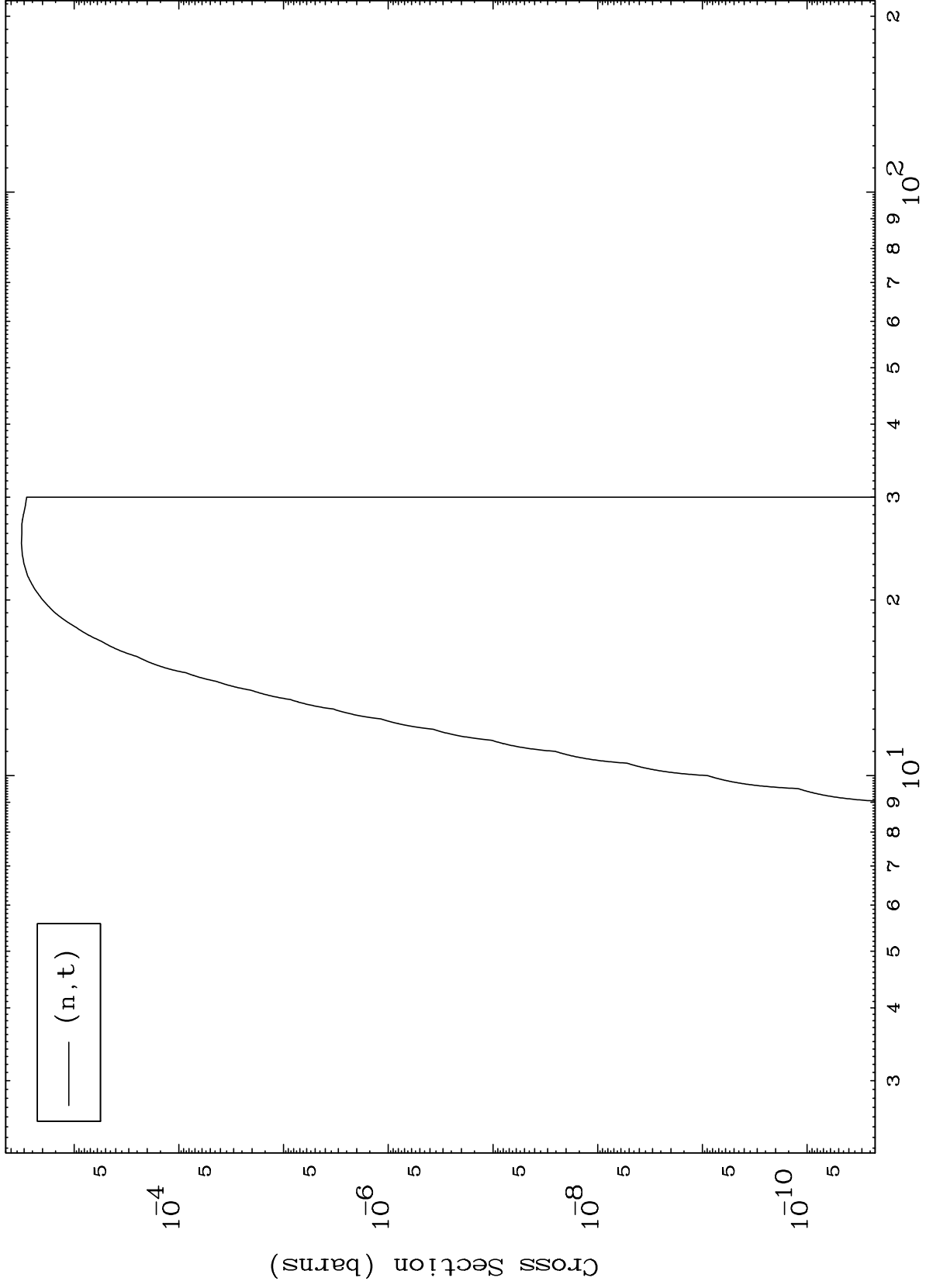
Incident Energy (MeV)

97-Bk-247

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(n,t) Levels  
293 Kelvin Cross Sections

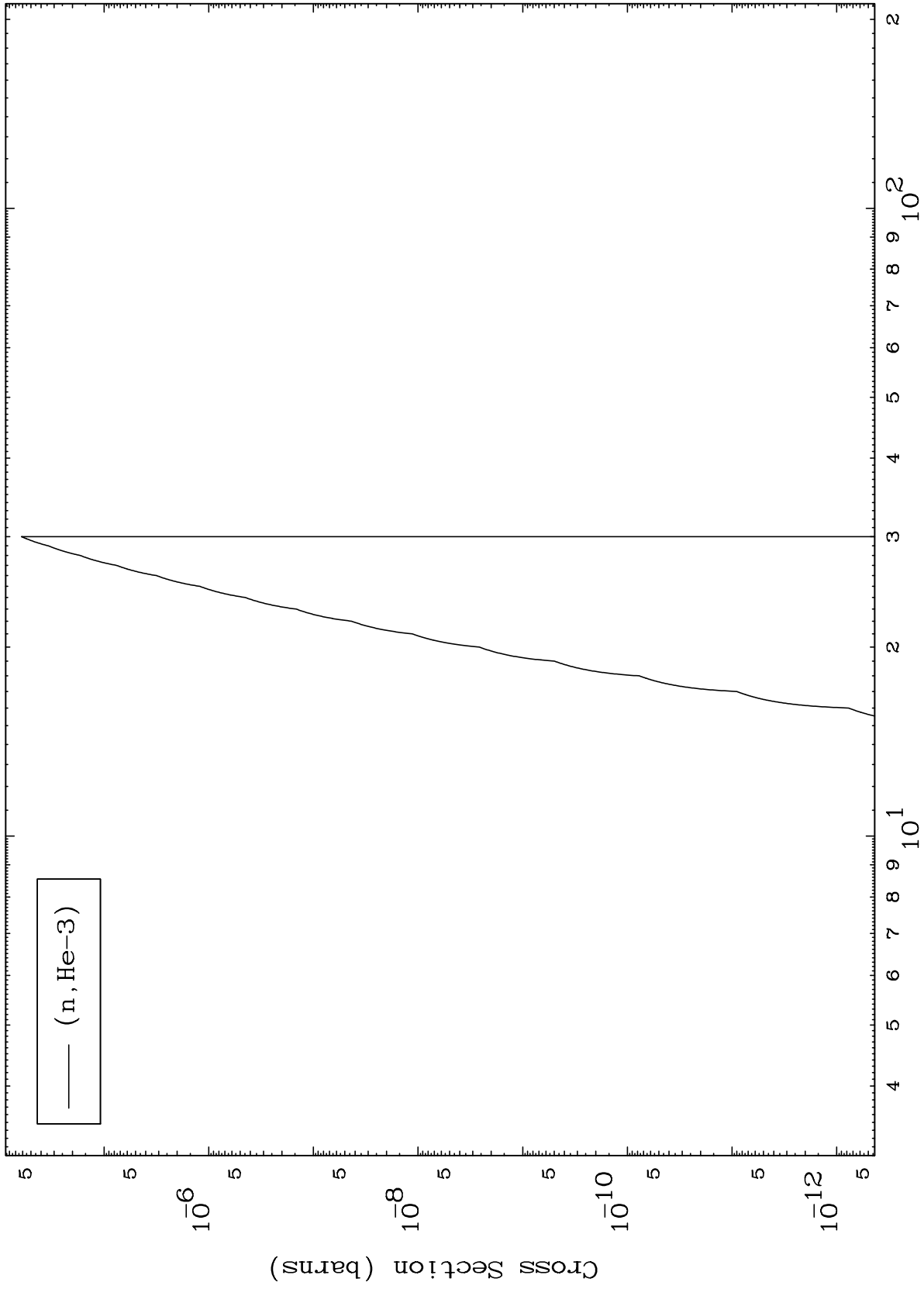
97-Bk-247



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(n,He3) Levels  
293 Kelvin Cross Sections

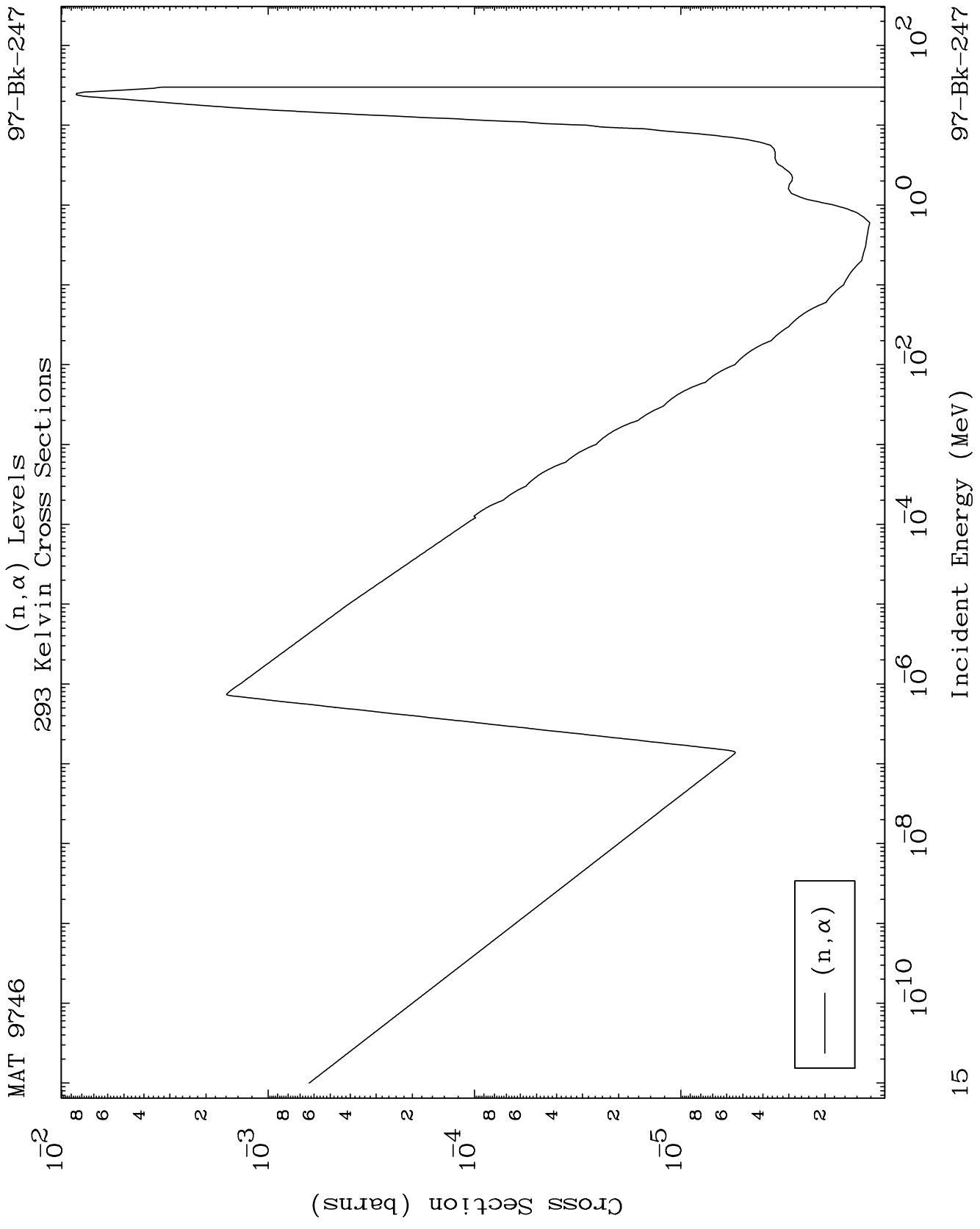
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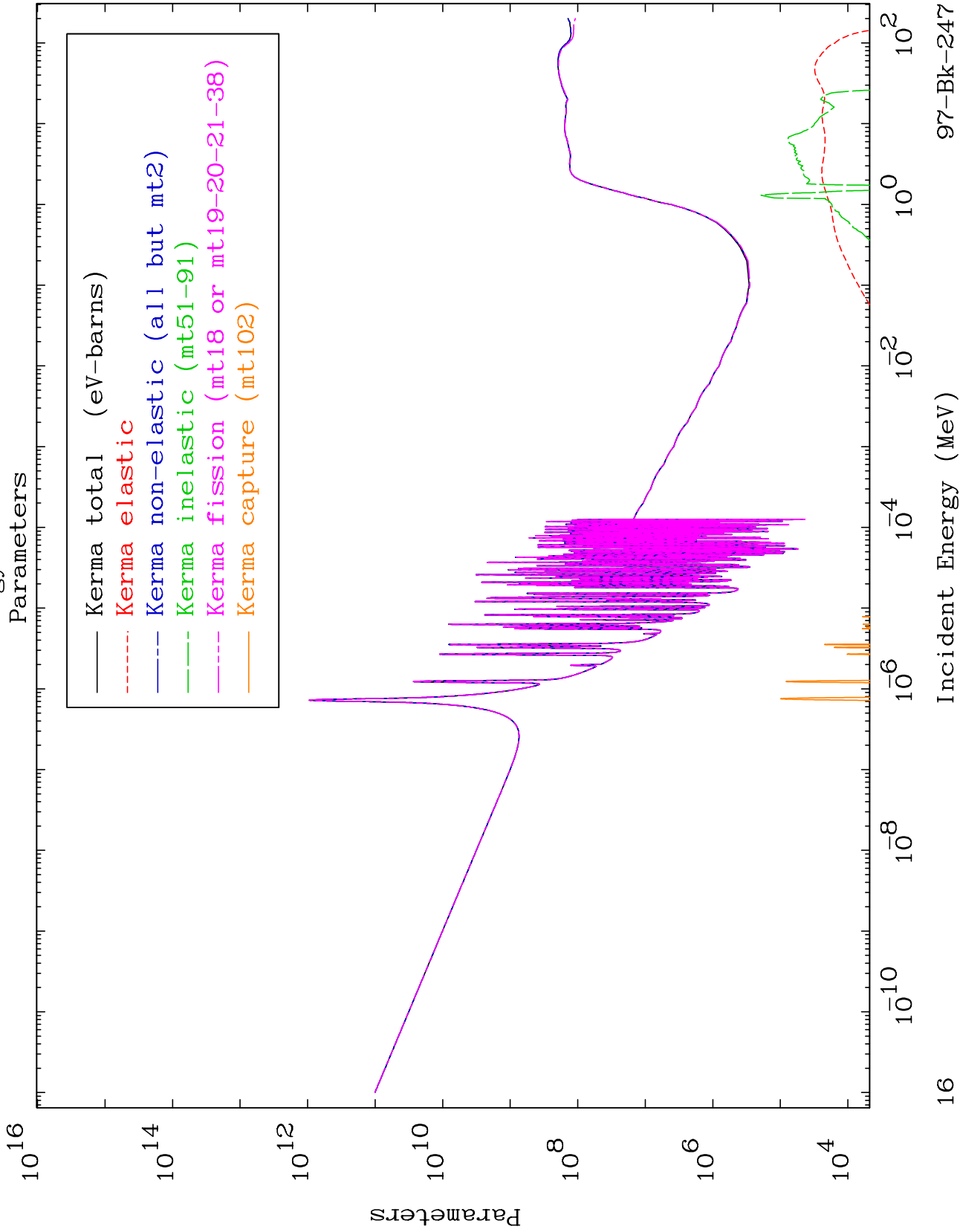
14

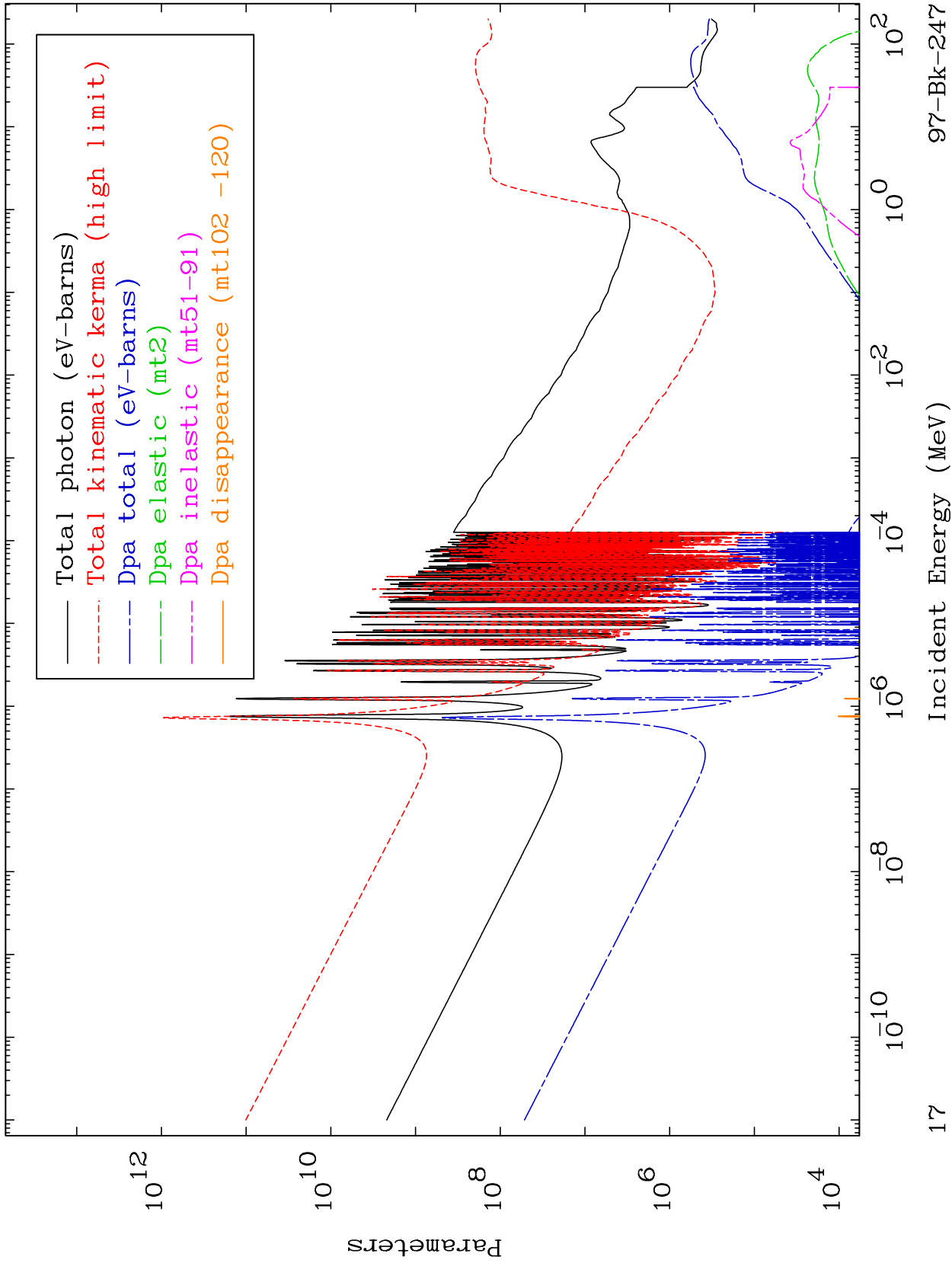
Incident Energy (MeV)

97-Bk-247





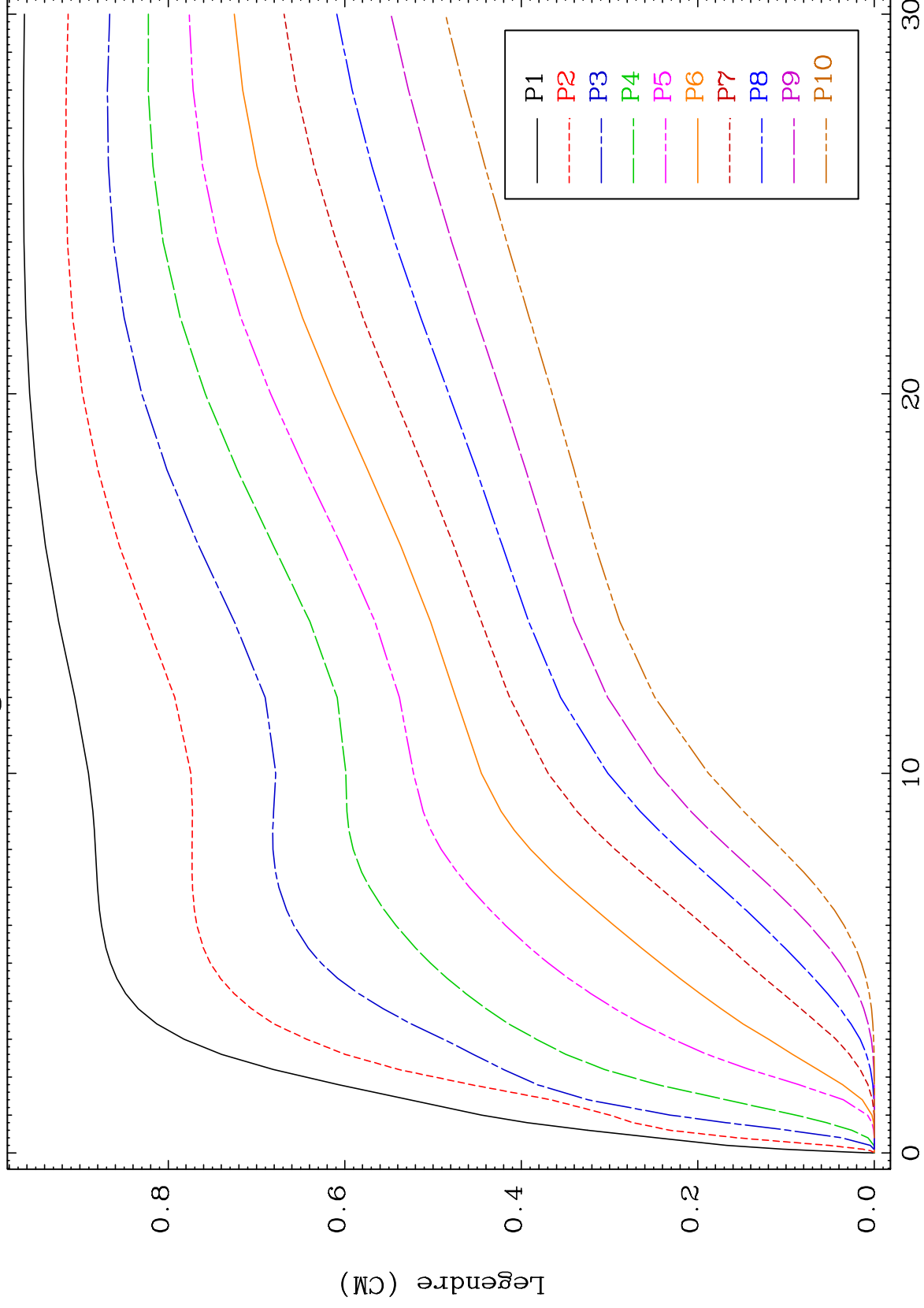




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Elastic Legendre Coefficients

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18

Incident Energy (MeV)

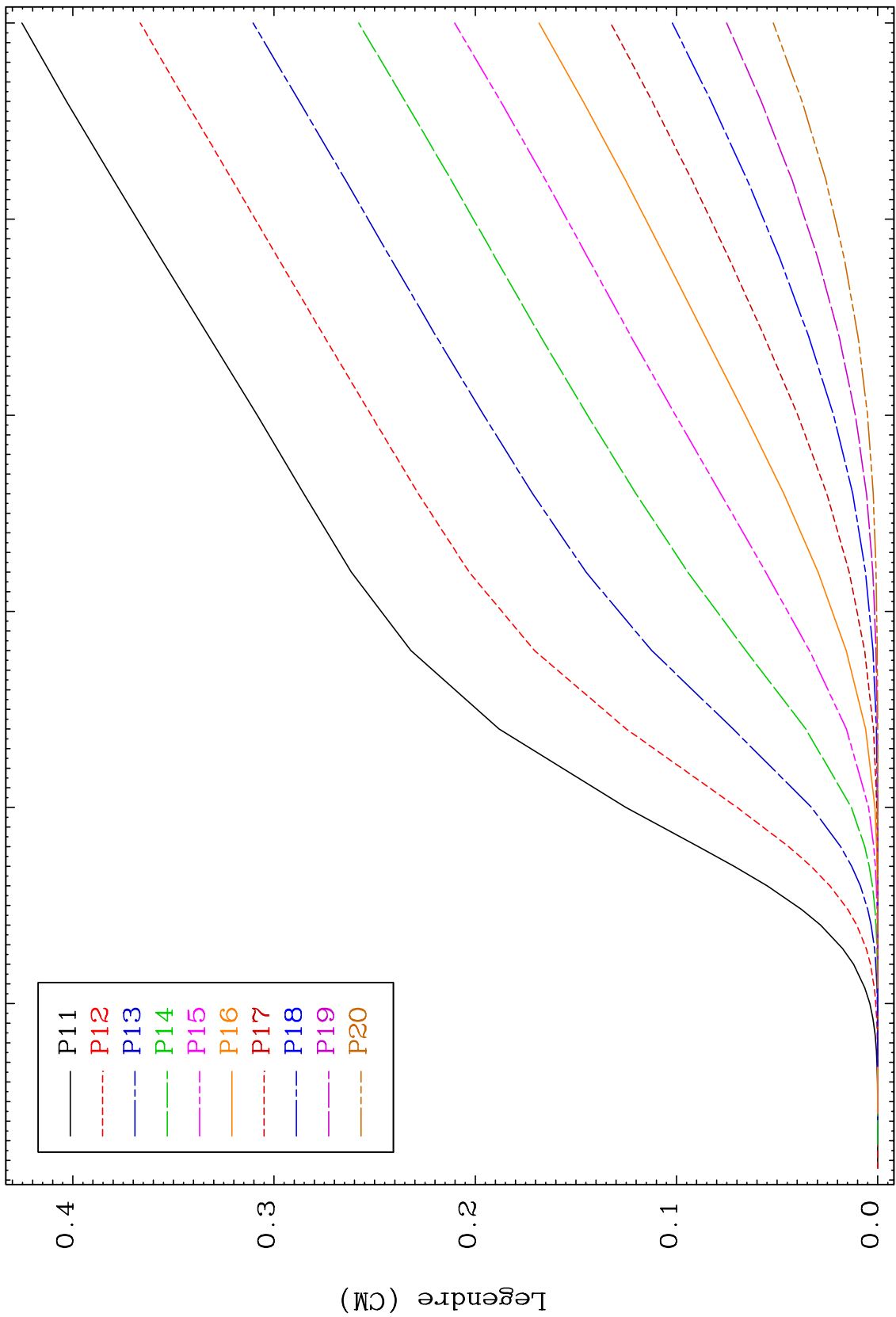
97-Bk-247

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Elastic Legendre Coefficients

97-Bk-247



19

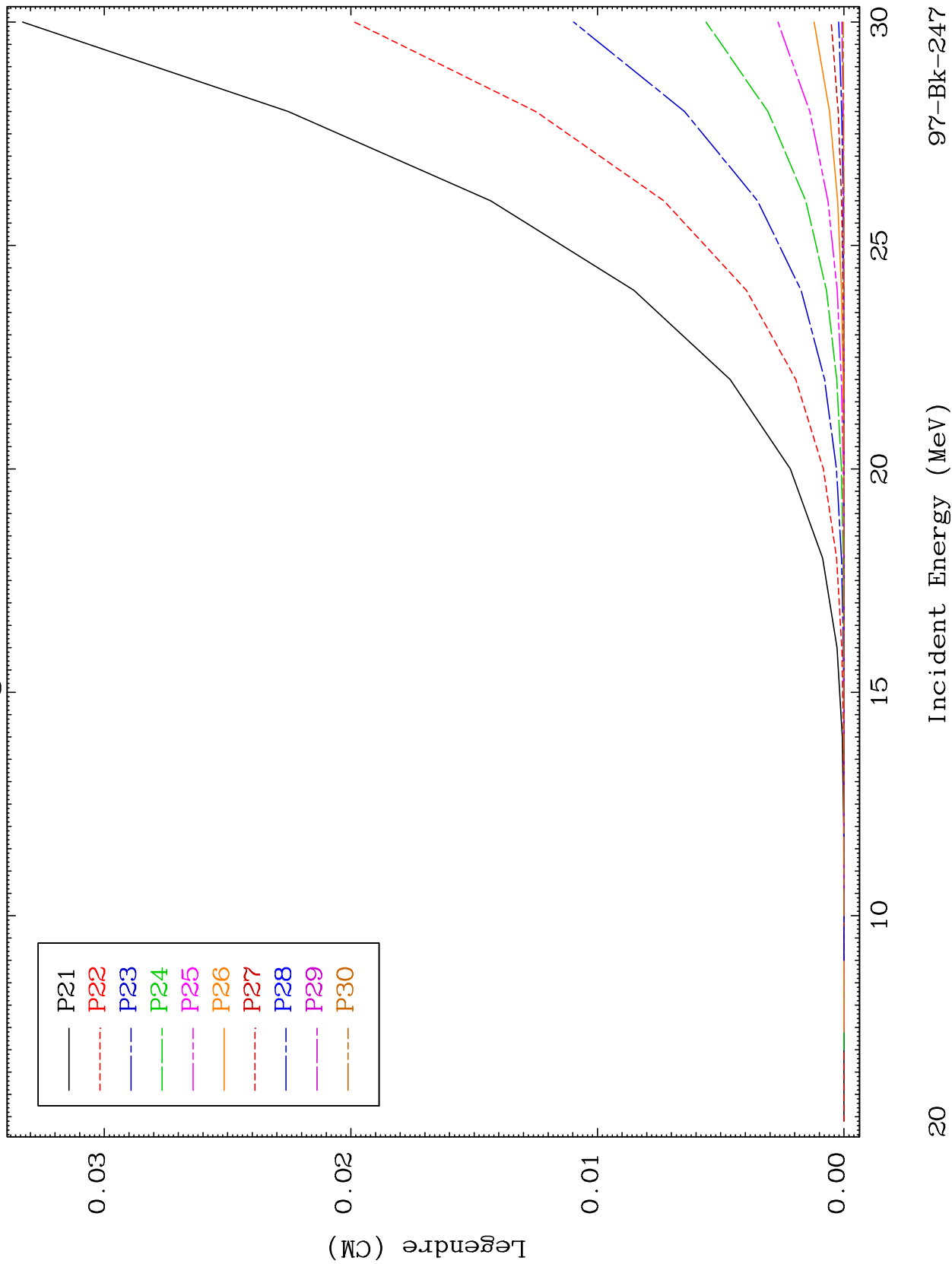
97-Bk-247

Incident Energy (MeV)

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### Elastic Legendre Coefficients

97-Bk-247



20

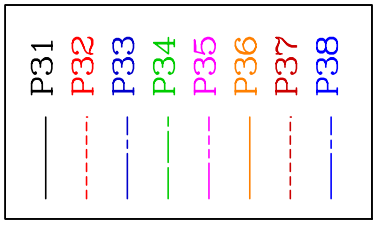
Incident Energy (MeV)

97-Bk-247

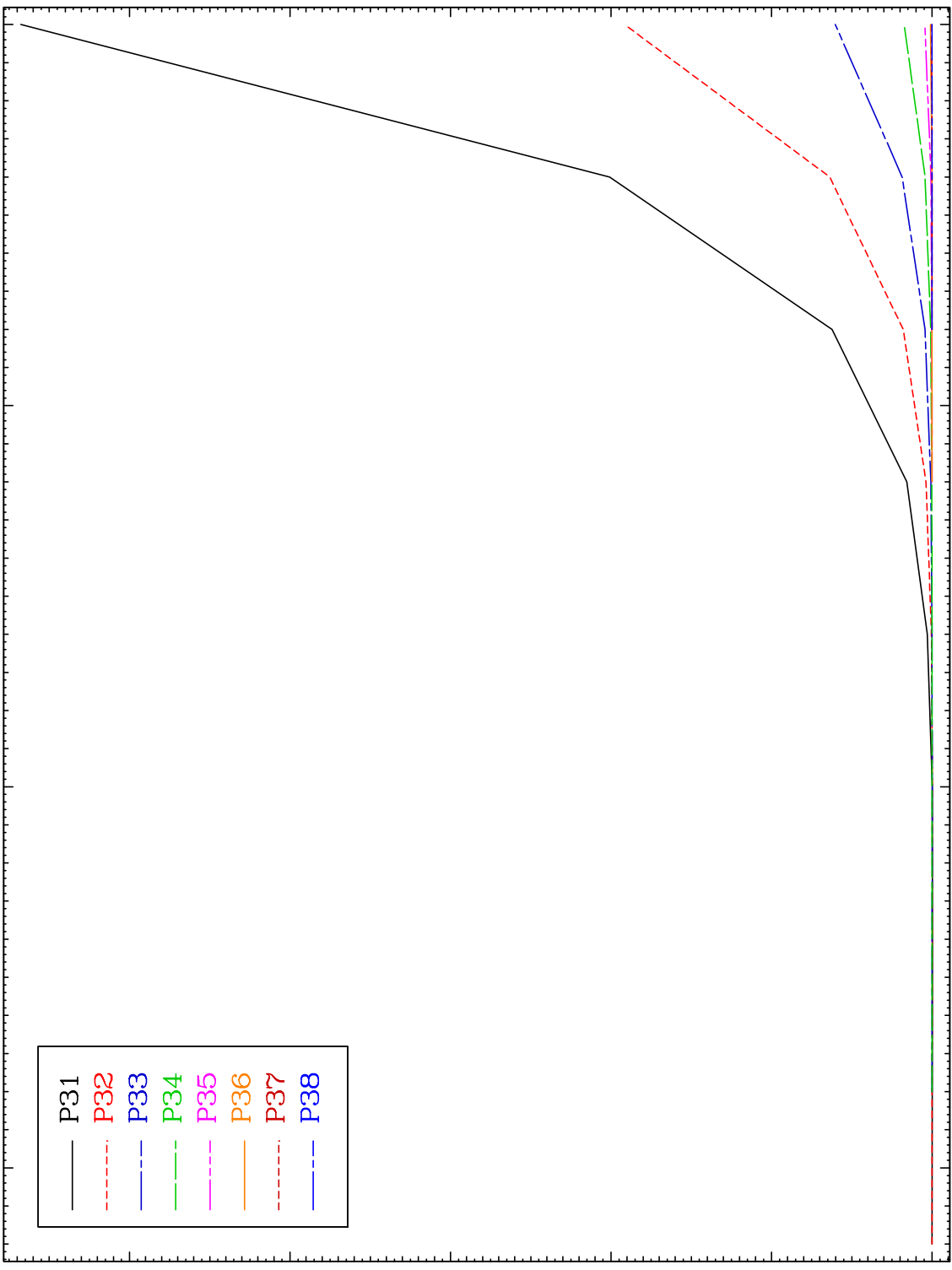
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Elastic Legendre Coefficients

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$\times 10^{-5}$   
1.0  
0.8  
0.6  
0.4  
0.2  
0.0

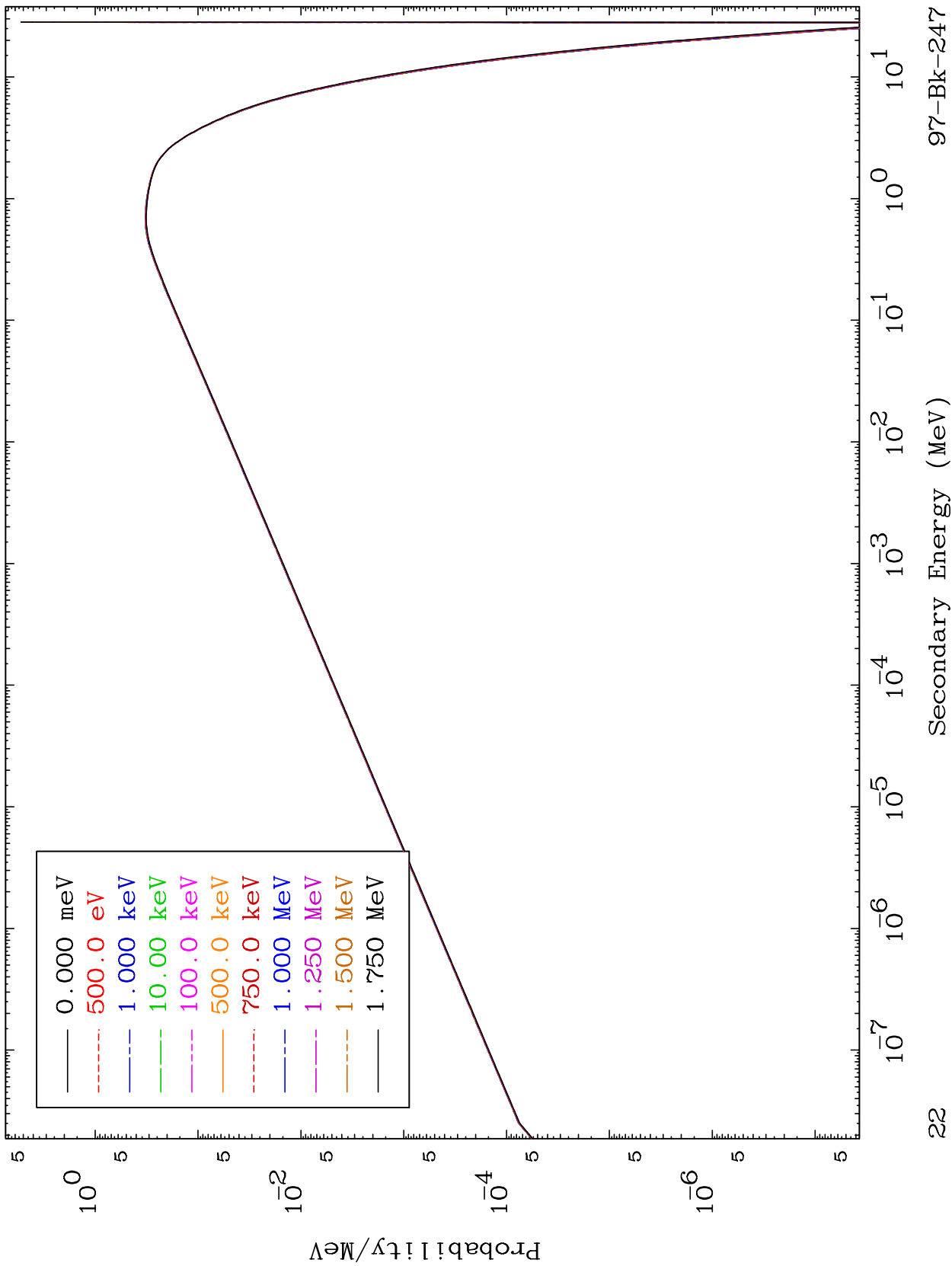


21

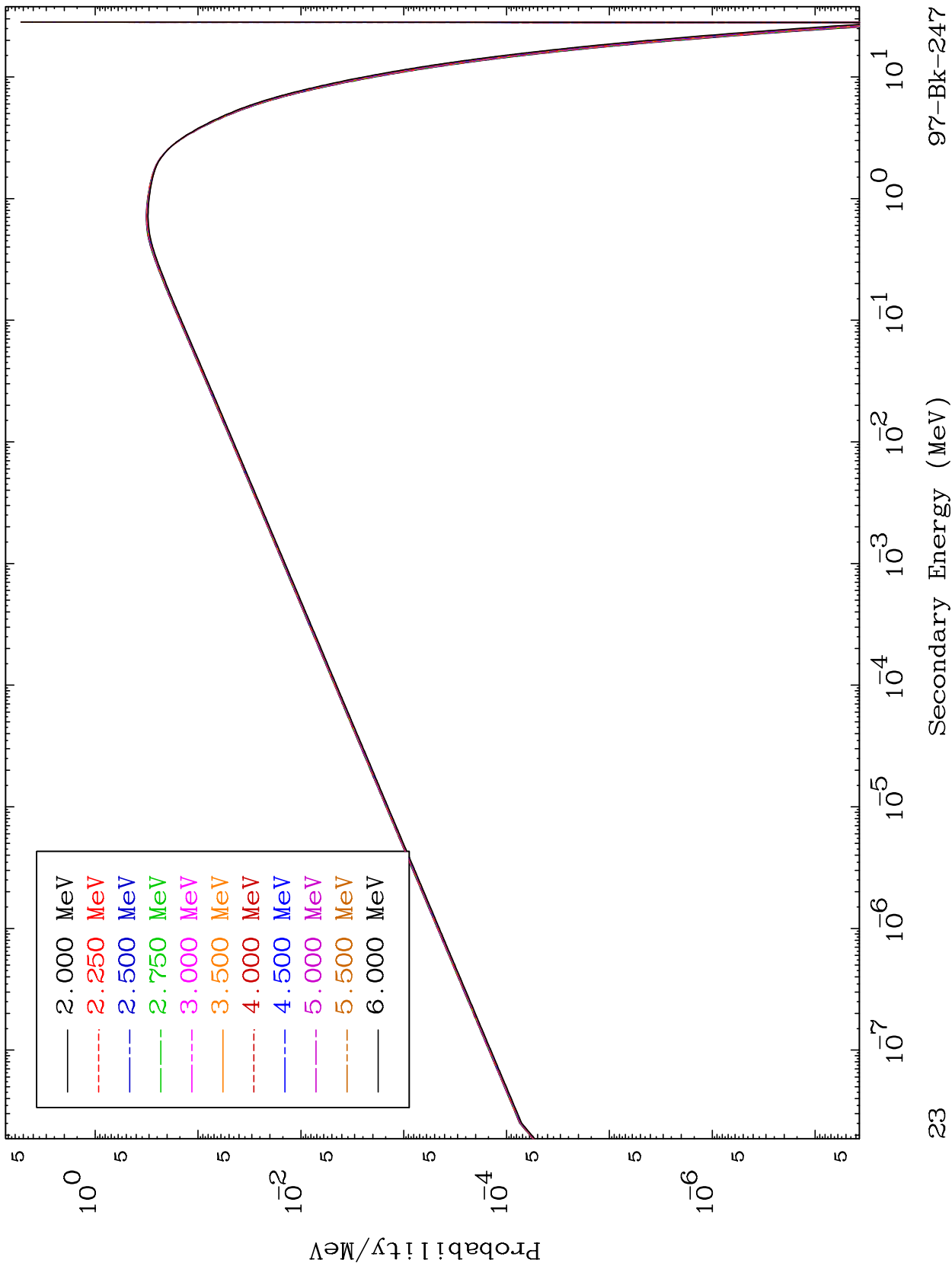
Incident Energy (MeV)

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Fission Energy Distribution

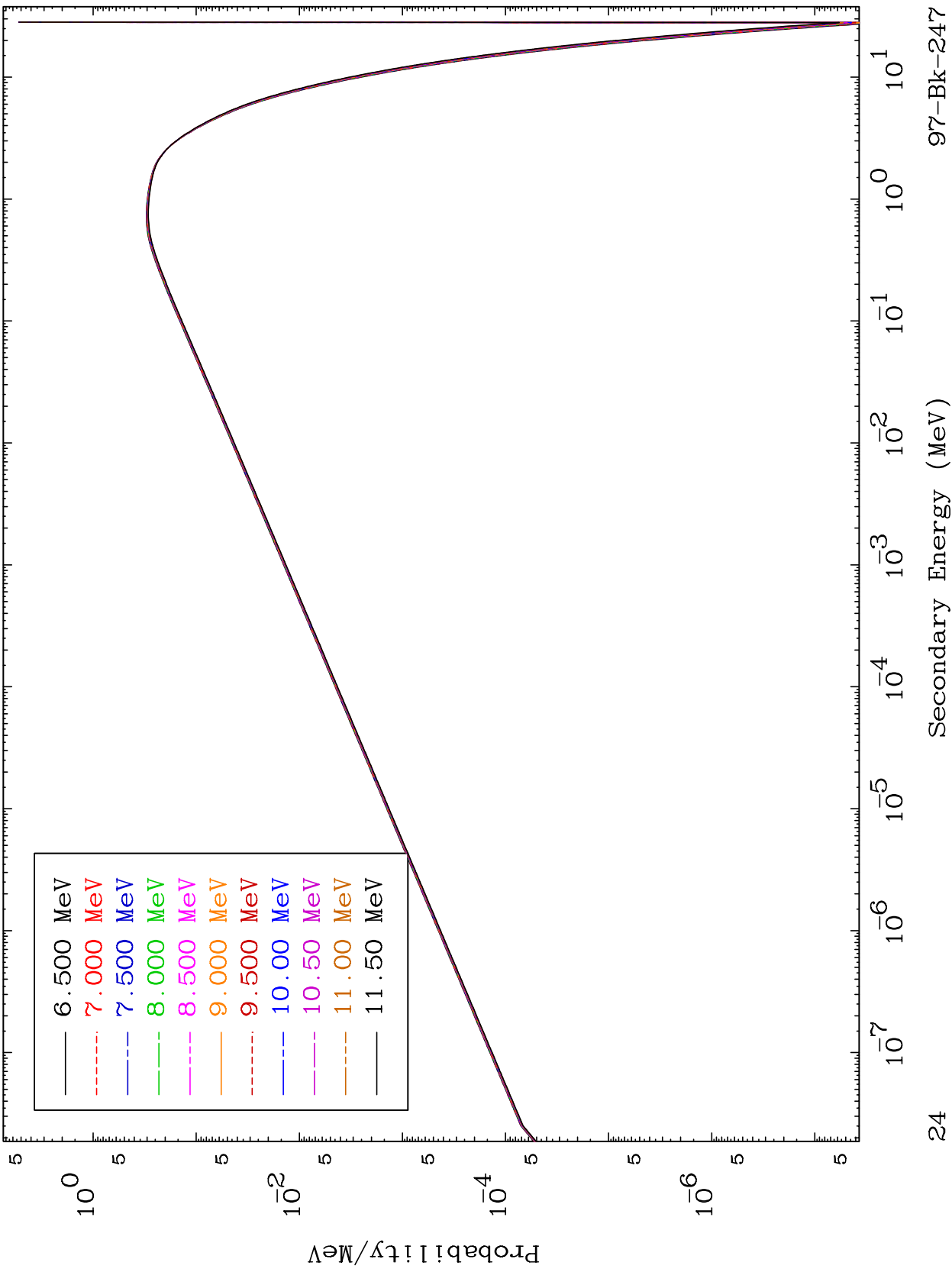


Fission Energy Distribution

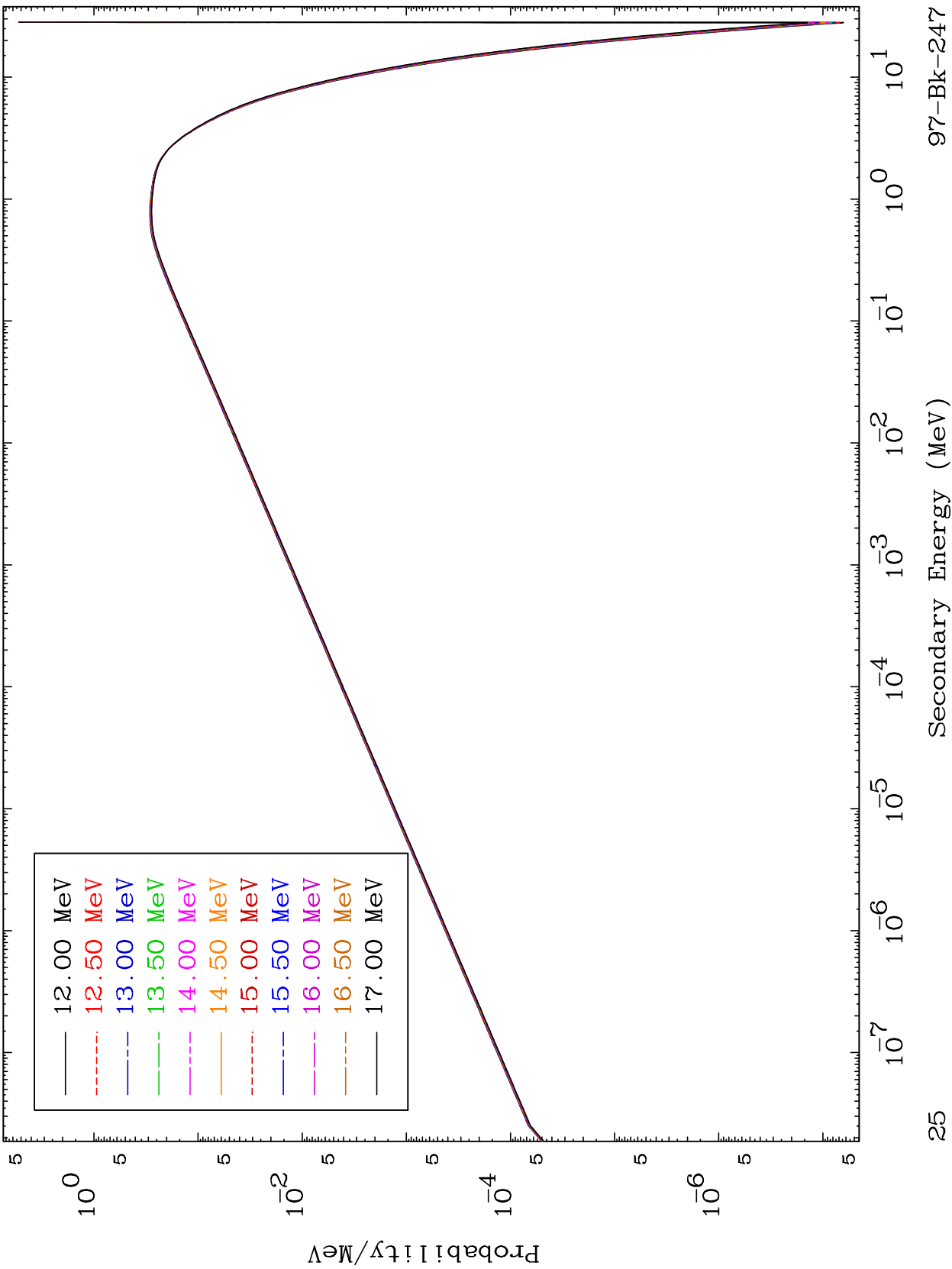




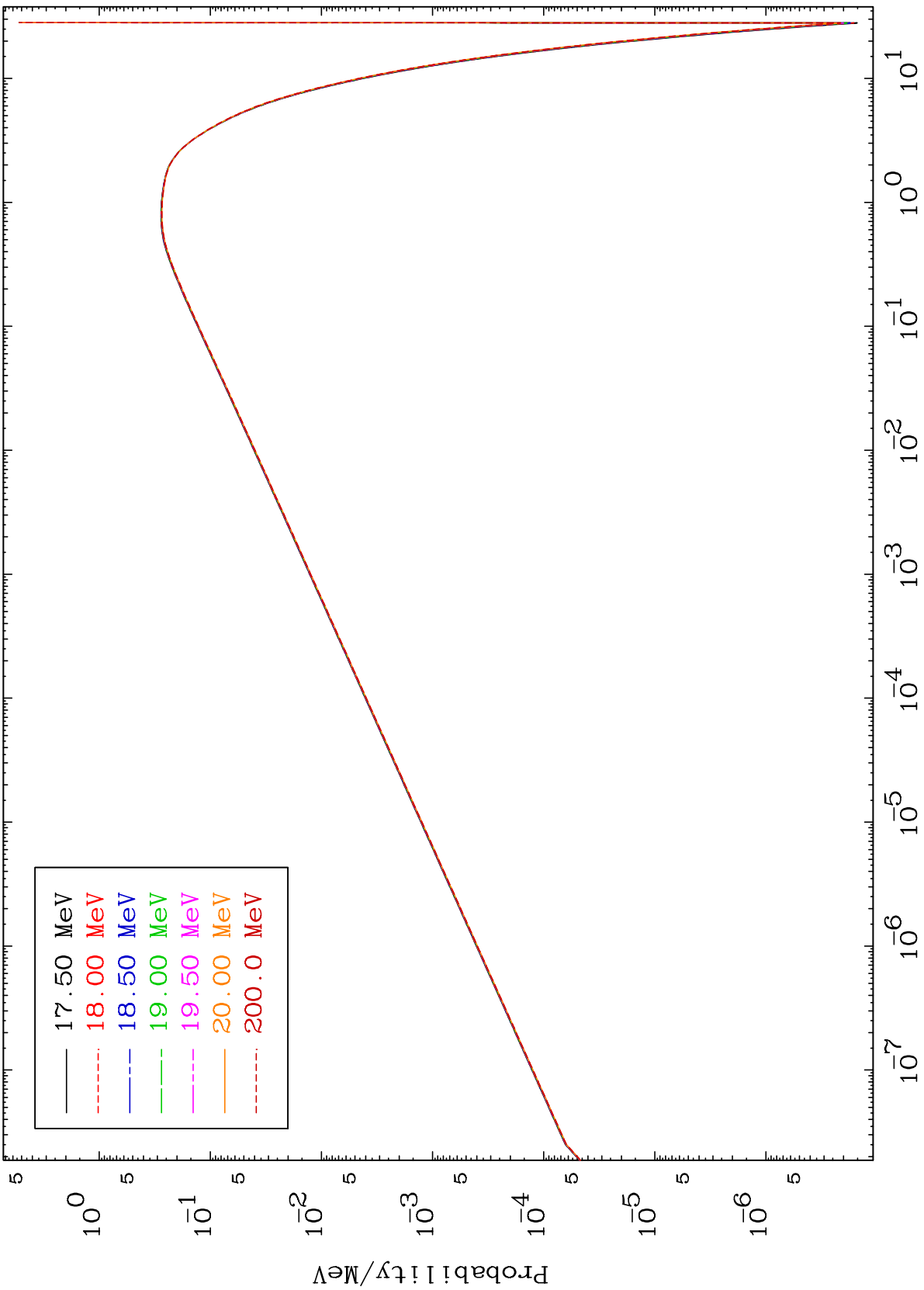
Fission Energy Distribution



Fission  
Energy Distribution



Fission Energy Distribution

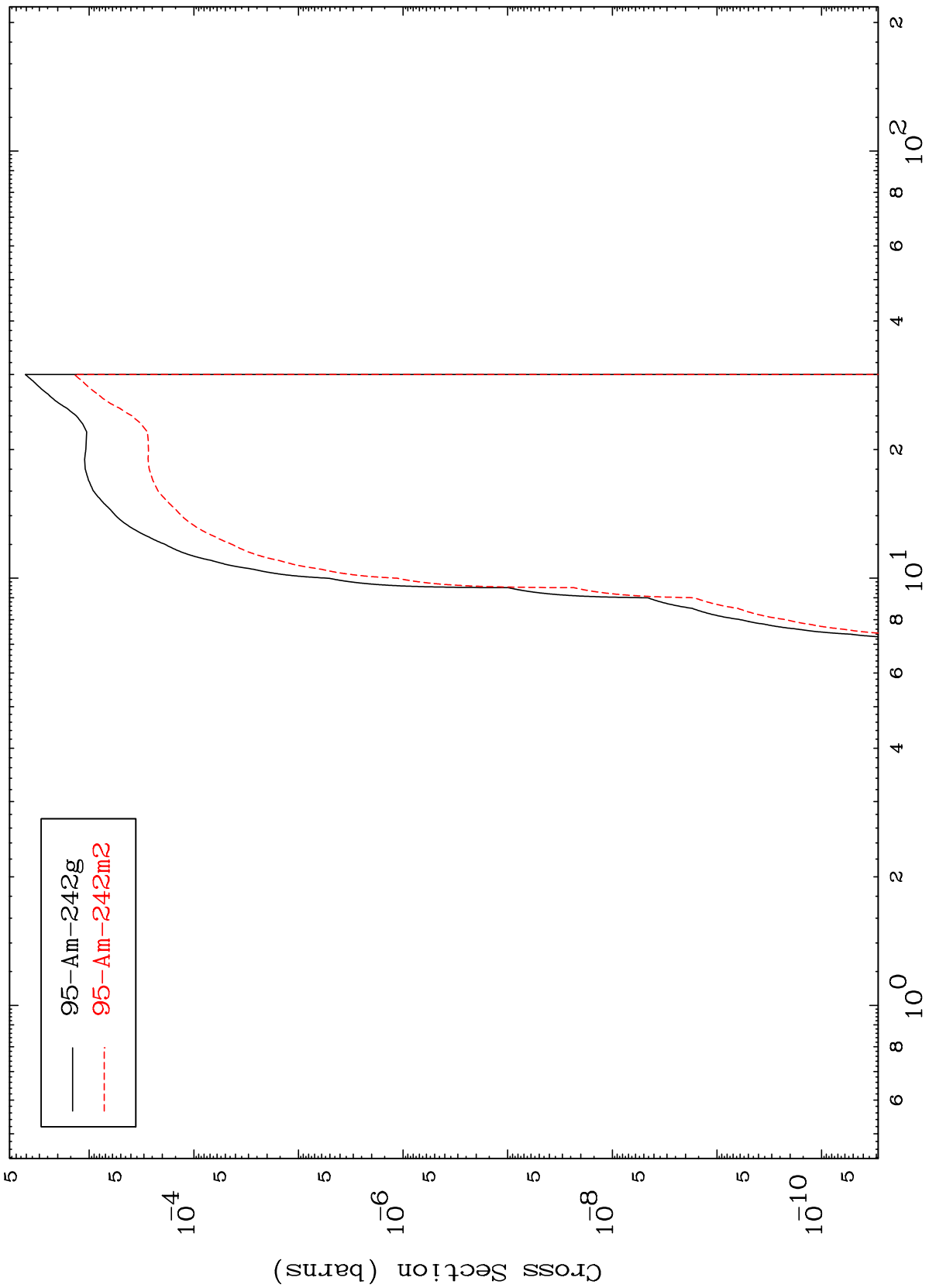


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(n,2n)  $\alpha$

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Radionuclide Production Cross Section



27

Incident Energy (MeV)

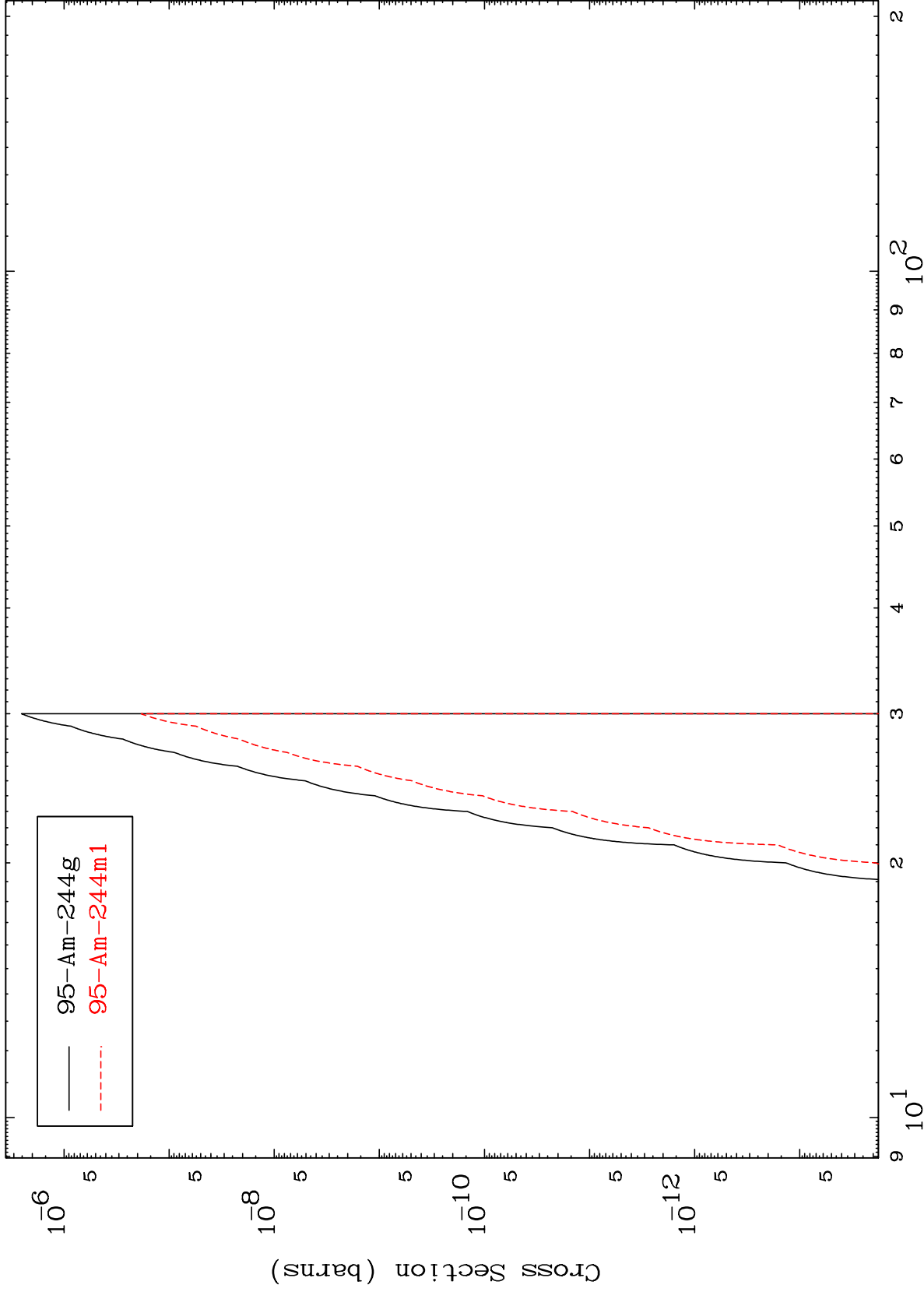
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(n,n') He-3

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Radionuclide Production Cross Section



28

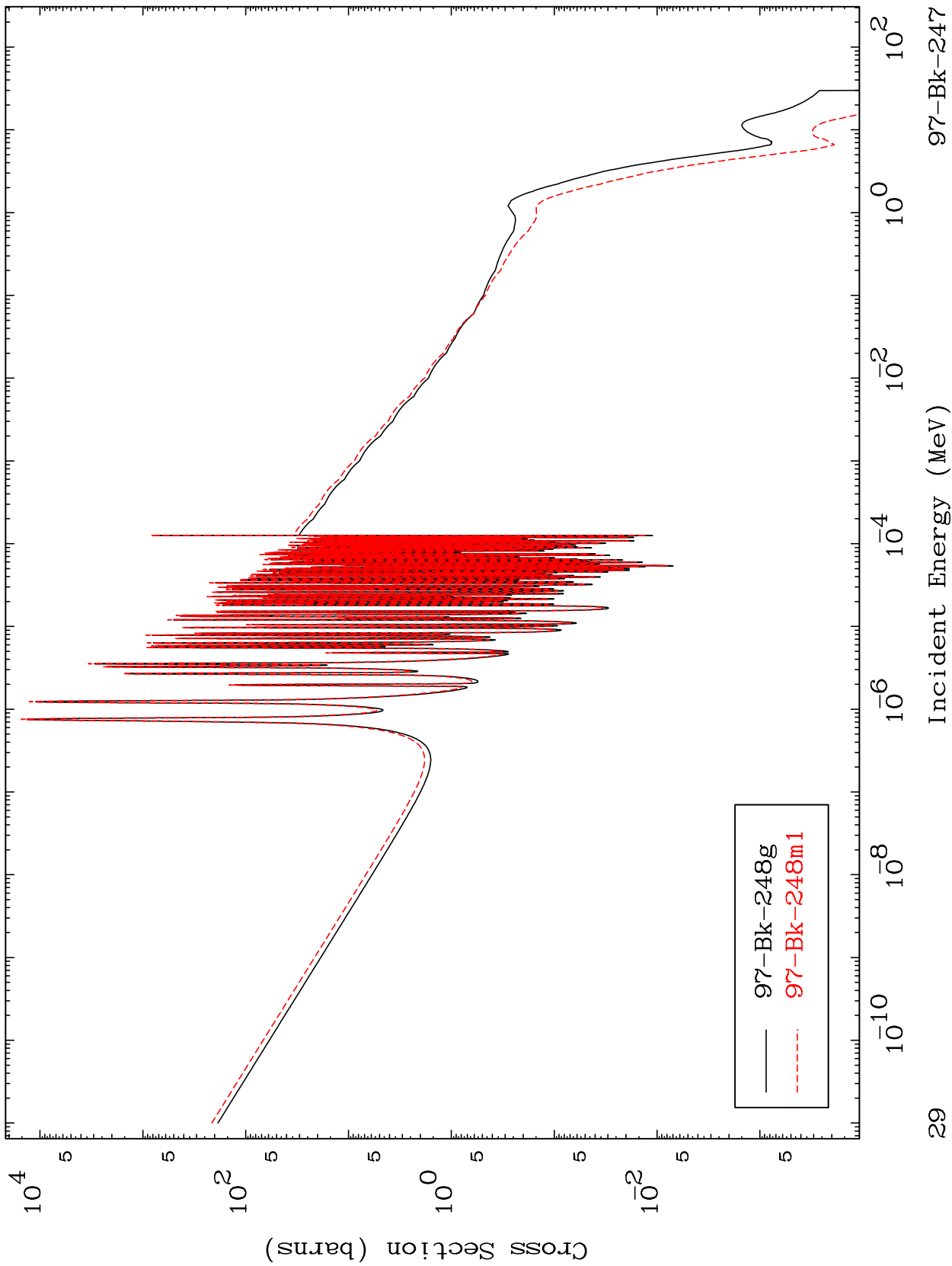
Incident Energy (MeV)

97-Bk-247

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97-Bk-247

$(n, \gamma)$   
Radionuclide Production Cross Section

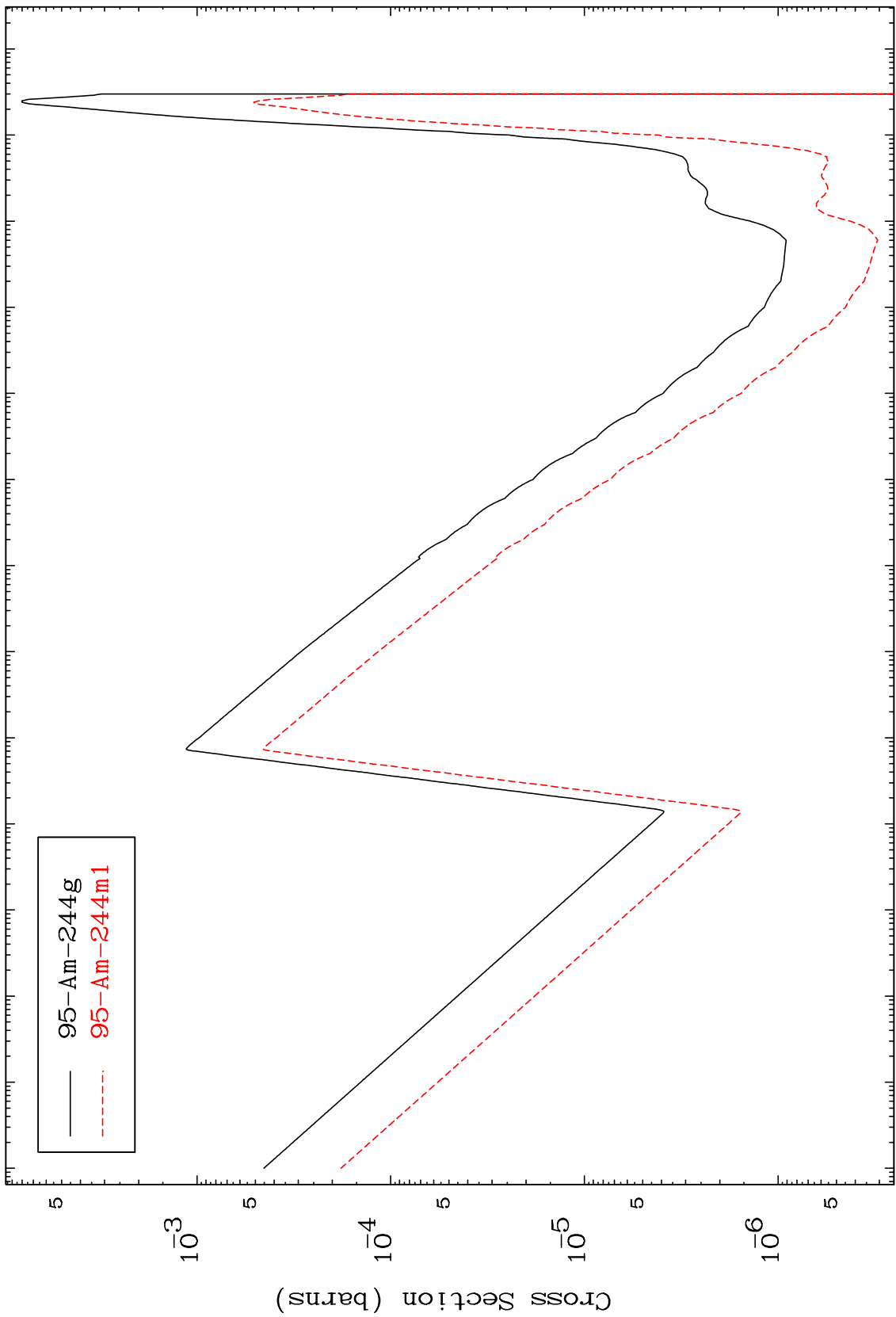
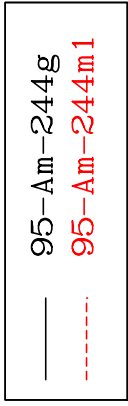


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(n,  $\alpha$ )

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Radionuclide Production Cross Section

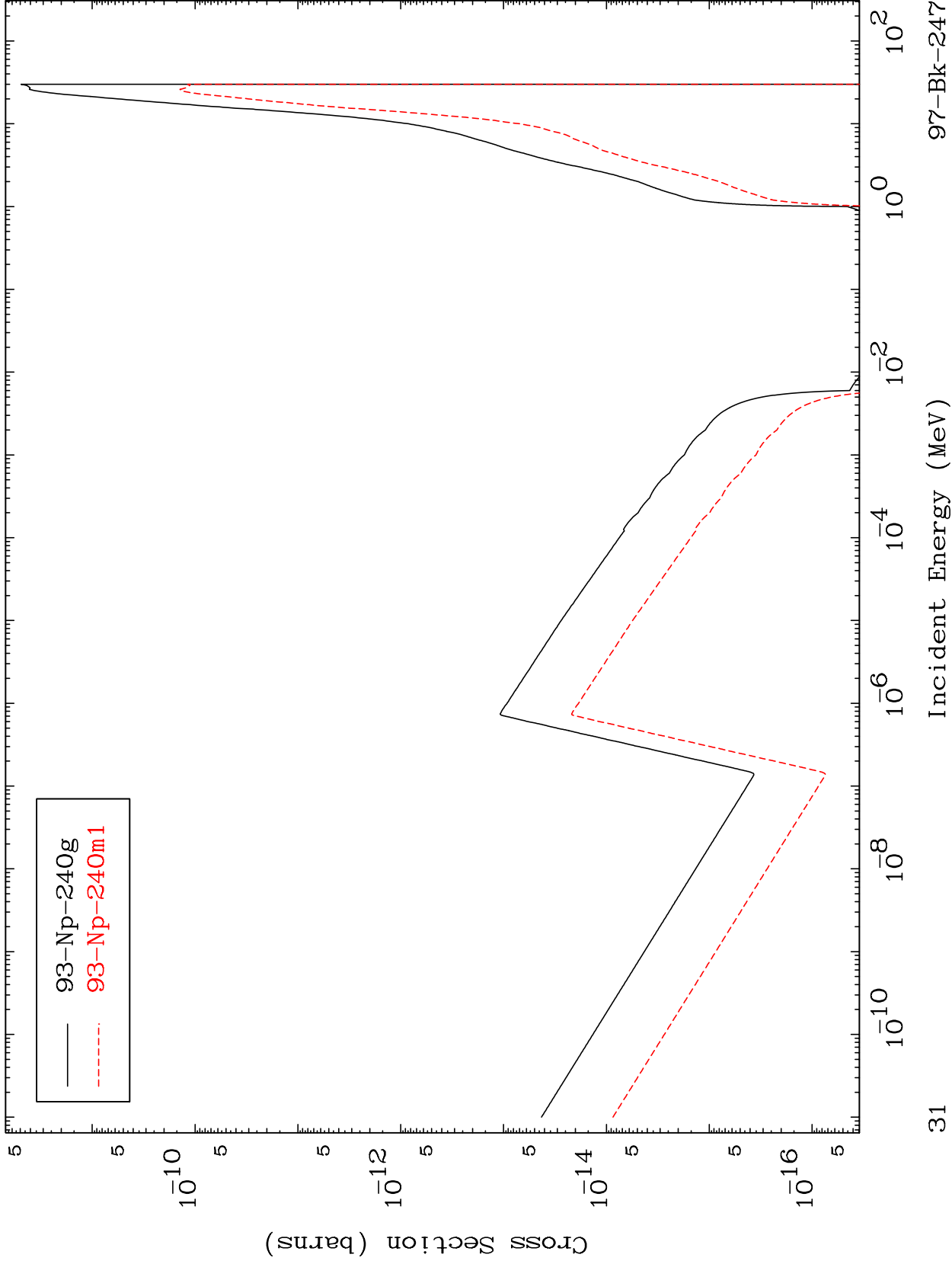


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(n,2α)

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Radionuclide Production Cross Section

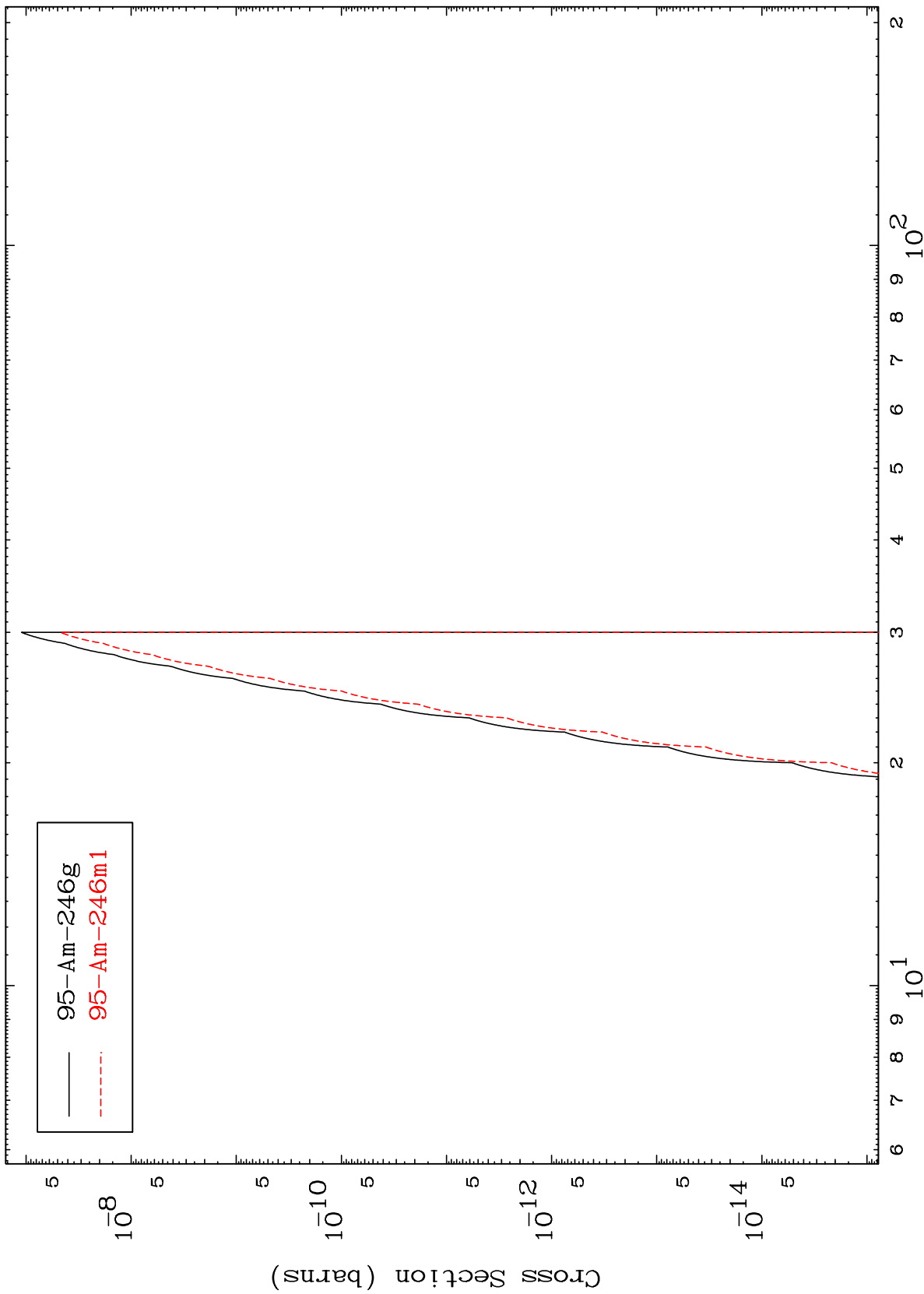




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(n,2p)  
Radionuclide Production Cross Section



32

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Incident Energy (MeV)