

Program EVALPLOT
(Version 2015-2)

by

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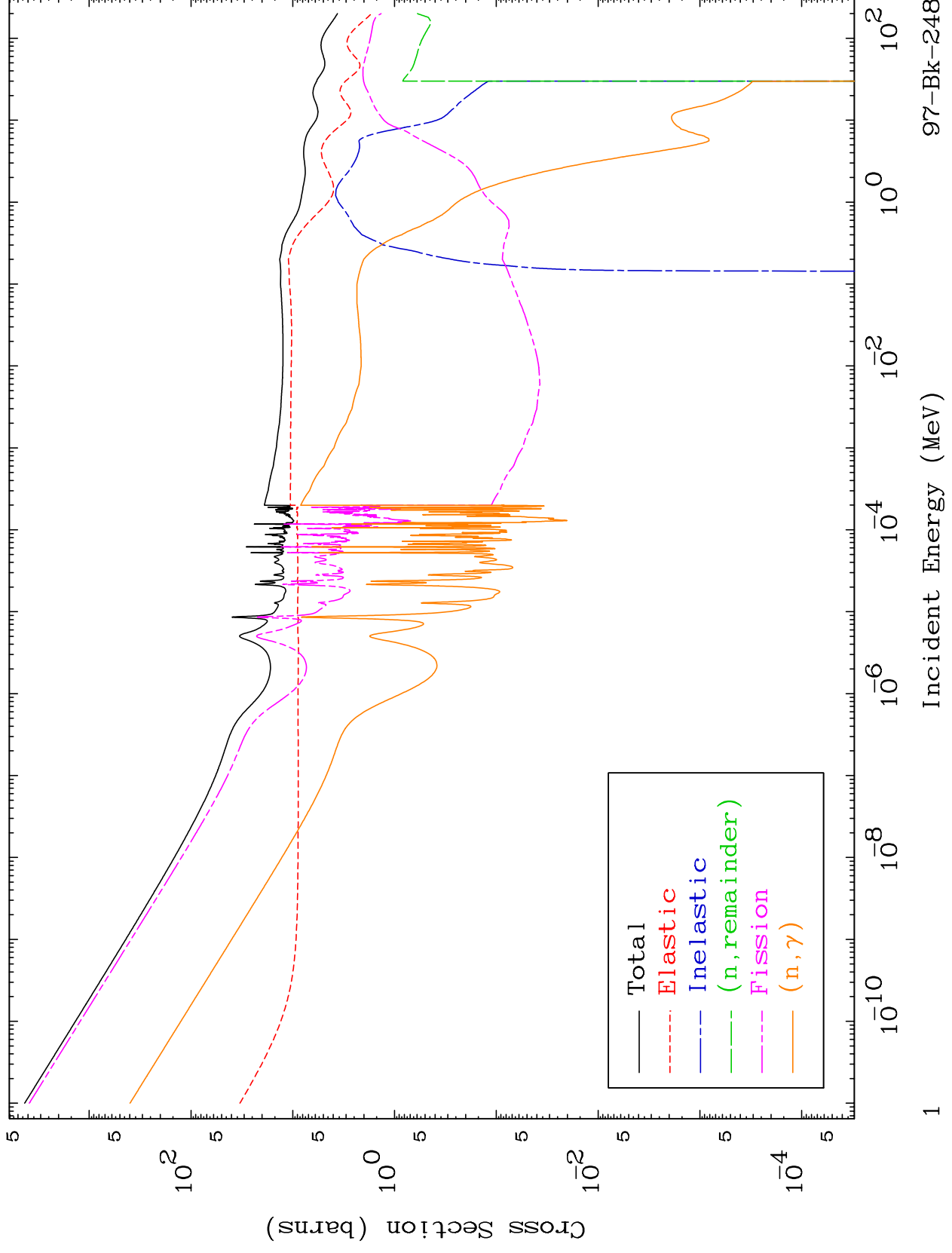
Press Mouse Button to Start

MAT 9749

Major

293 Kelvin Cross Sections

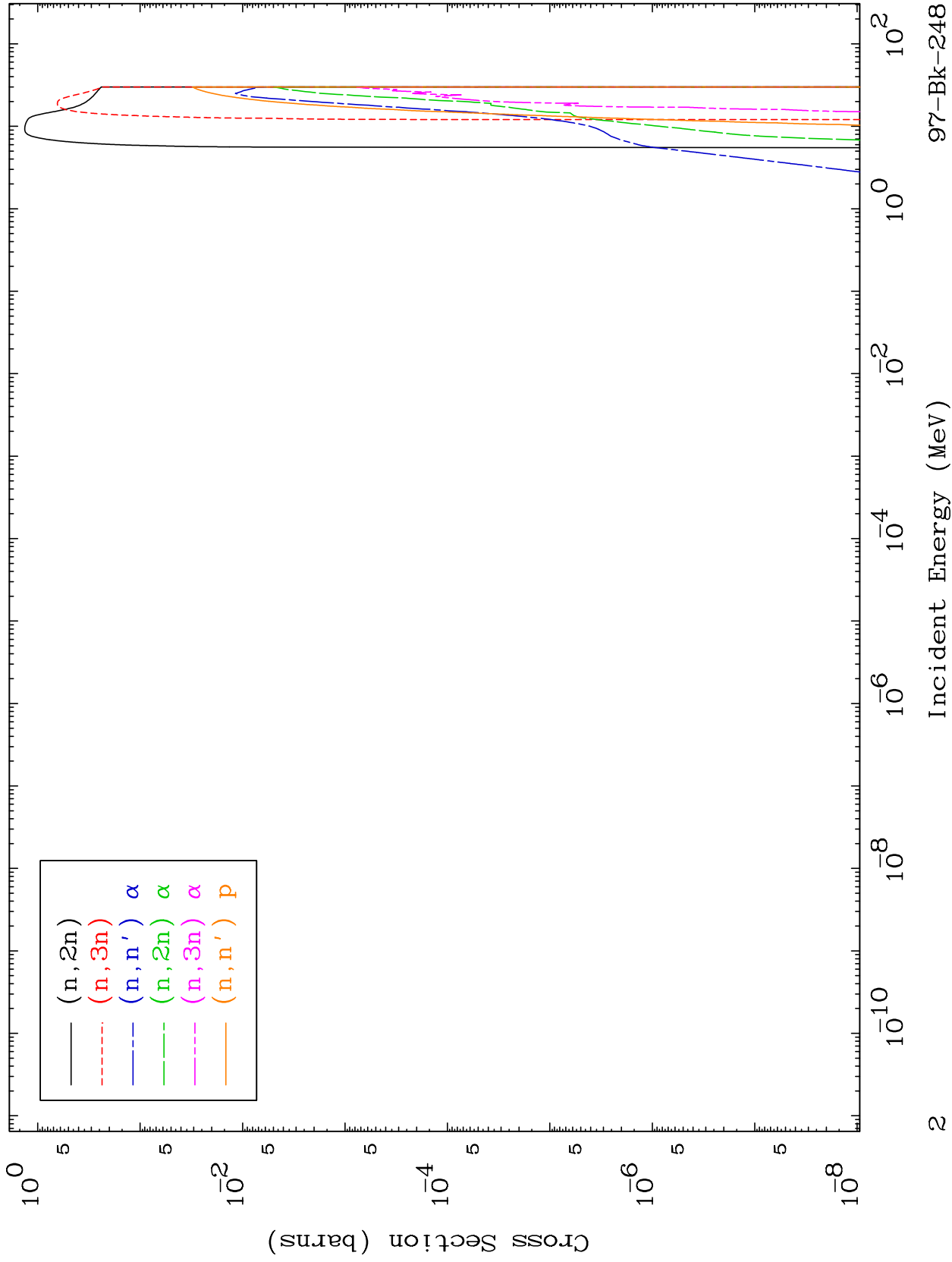
97-Bk-248



MAT 9749

Neutron Production
293 Kelvin Cross Sections

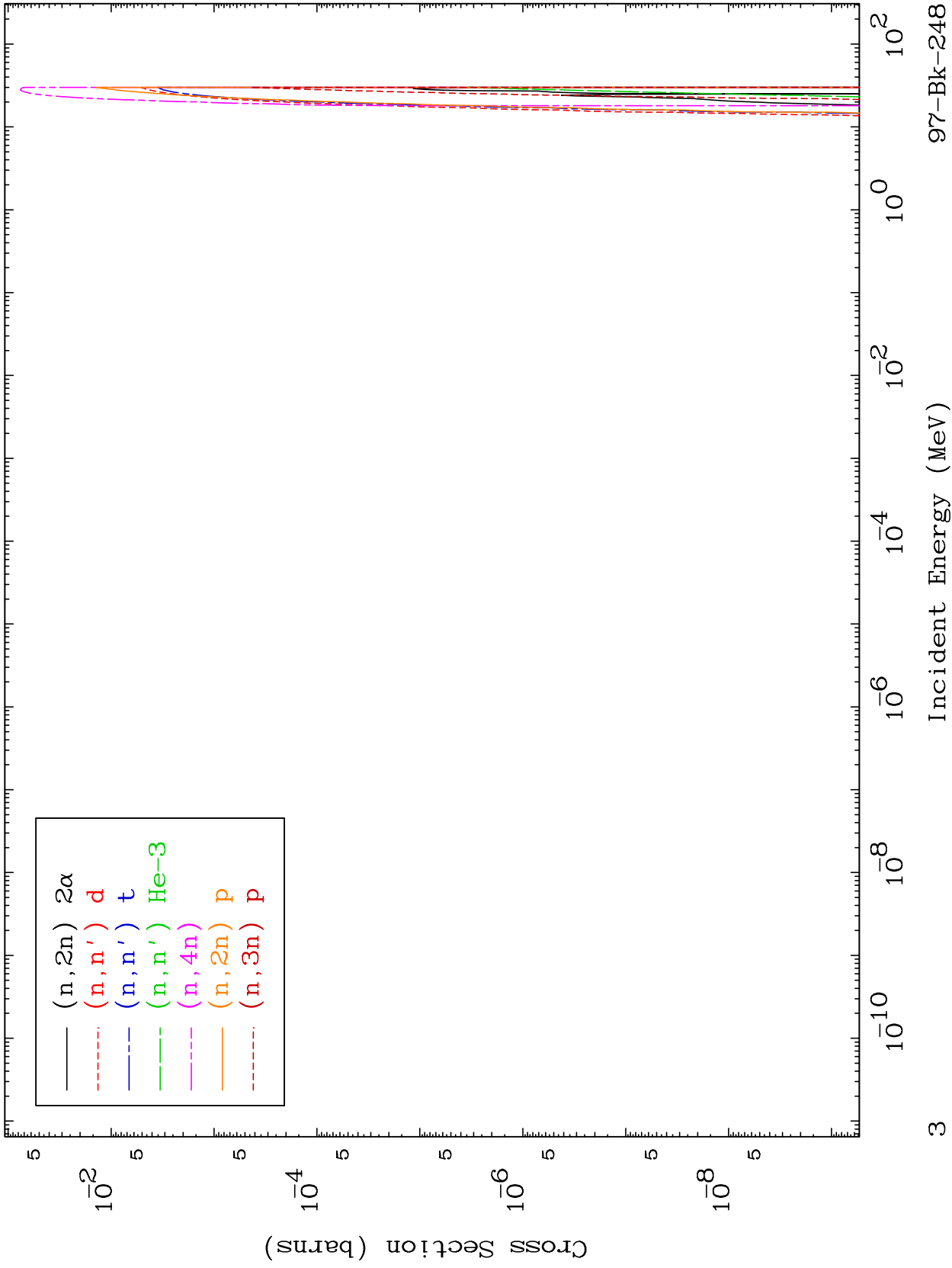
97-Bk-248



MAT 9749

Neutron Production
293 Kelvin Cross Sections

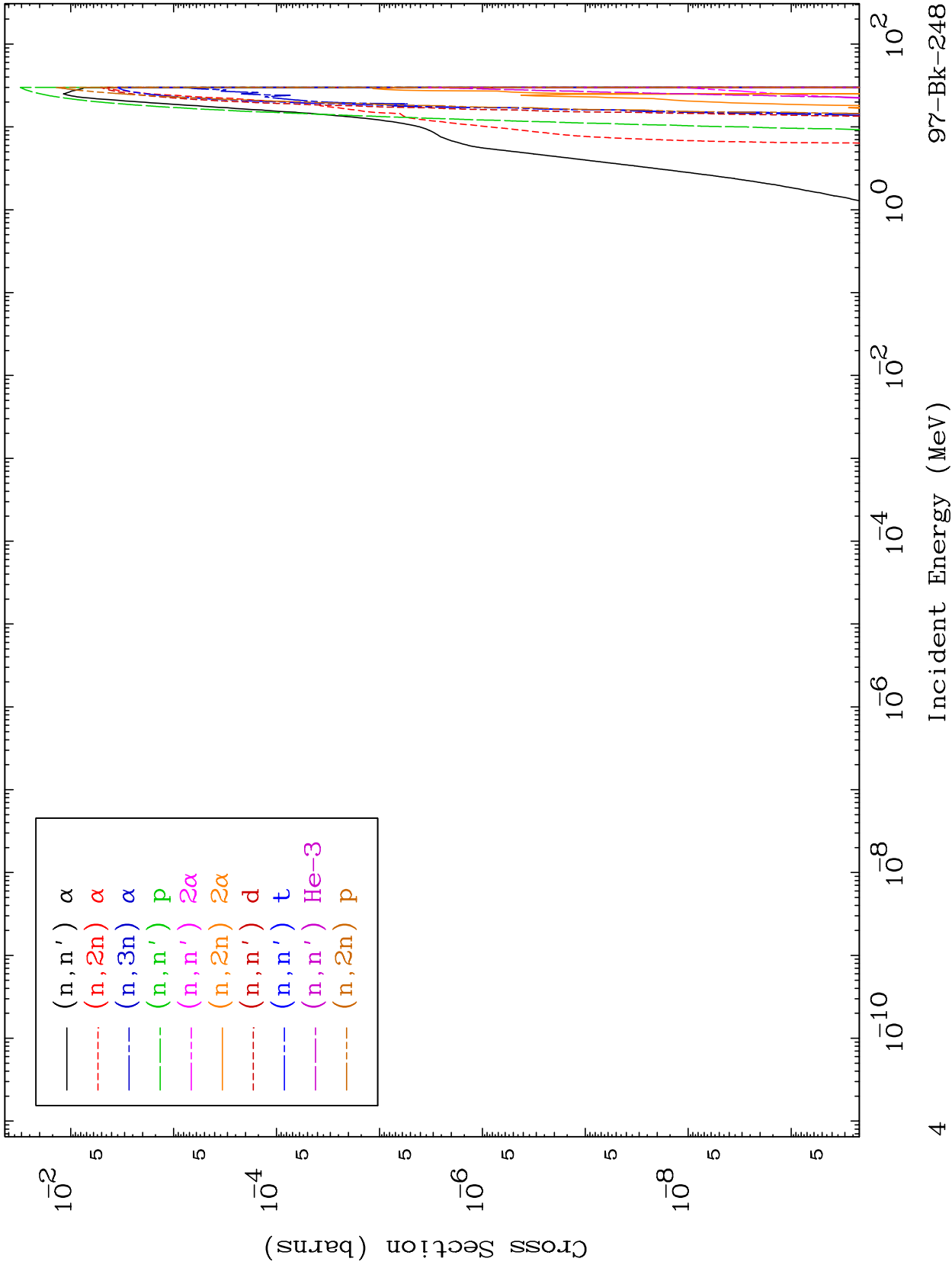
97-Bk-248



MAT 9749

Charged Particle
293 Kelvin Cross Sections

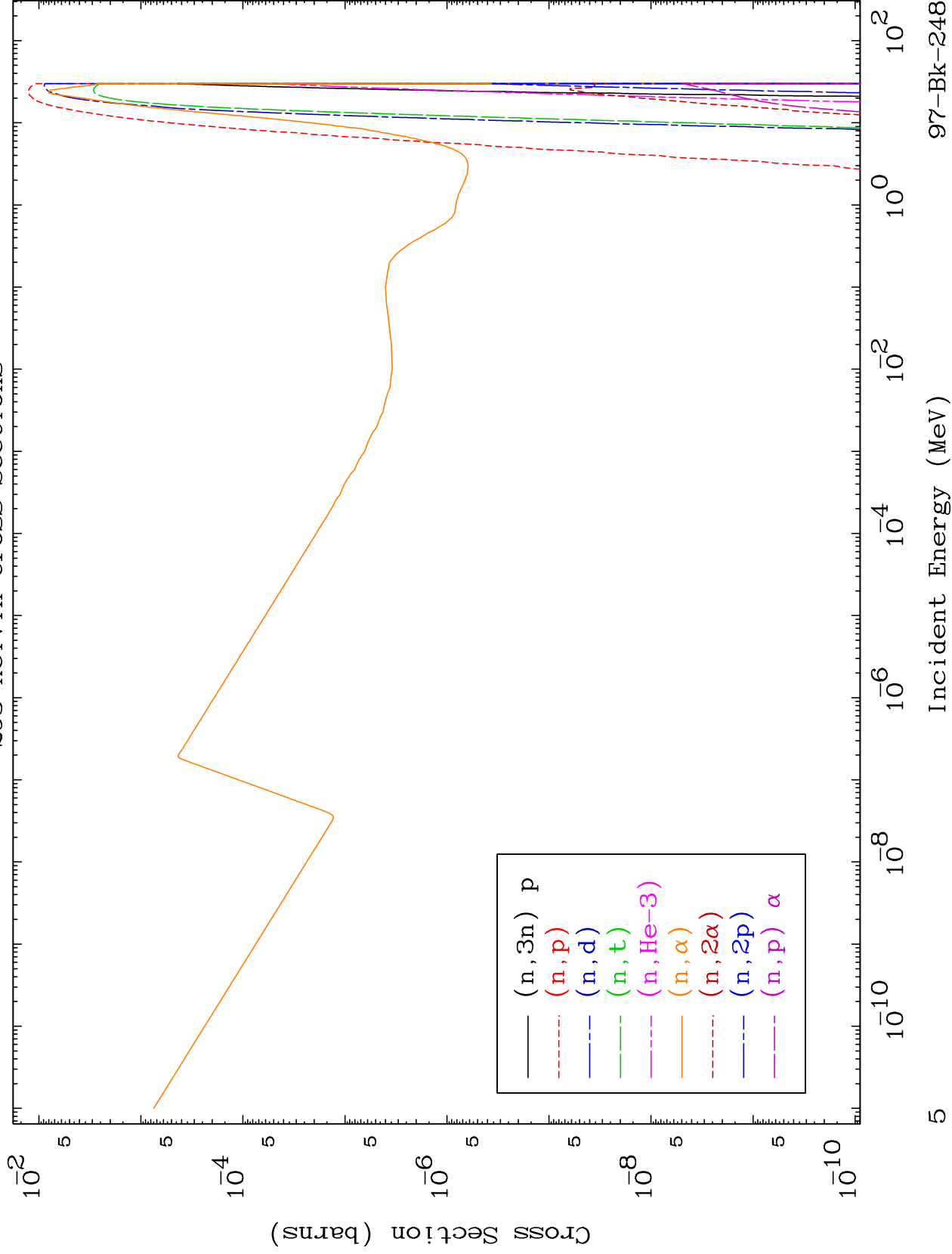
97-Bk-248



MAT 9749

Charged Particle
293 Kelvin Cross Sections

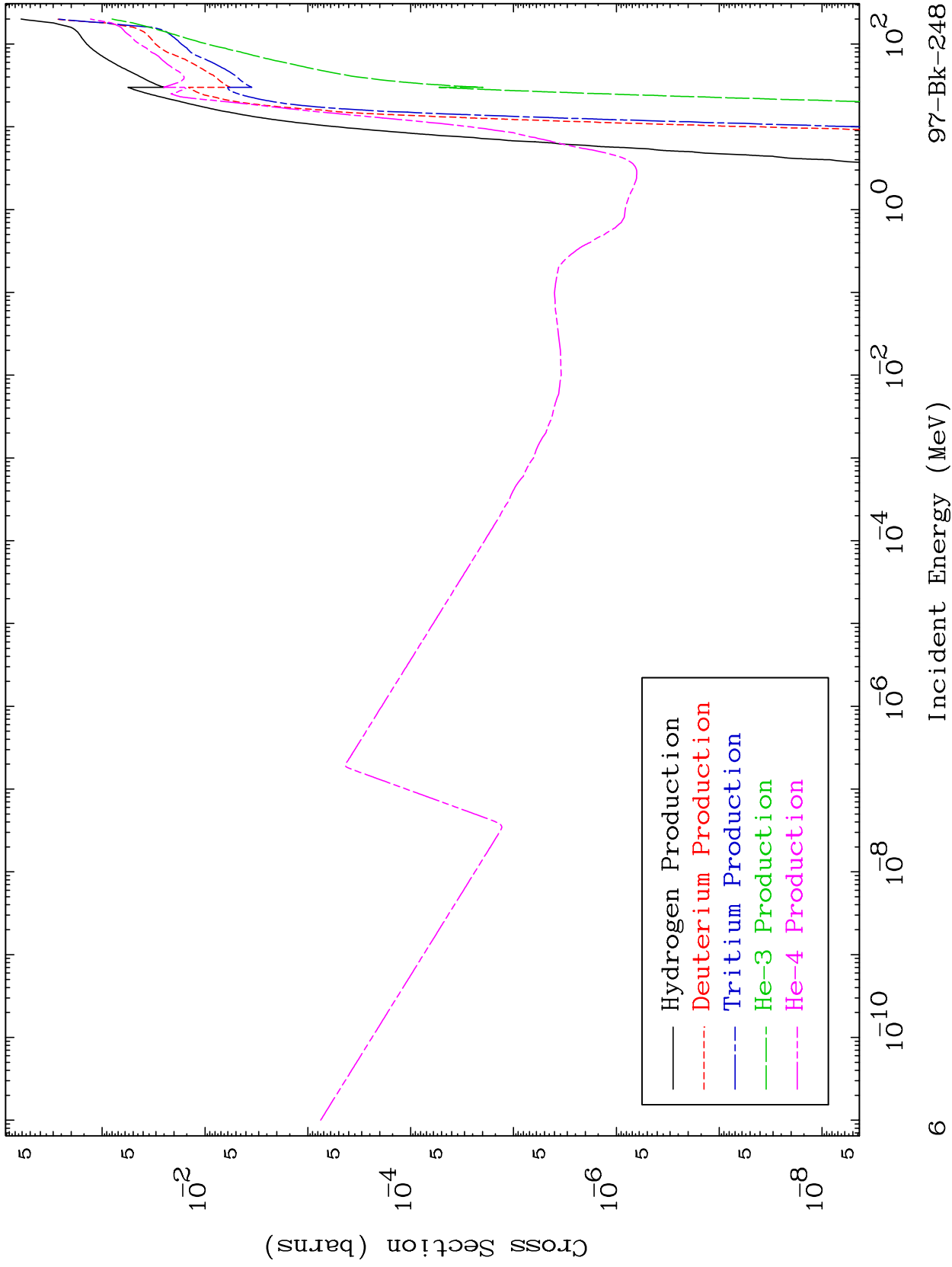
97-Bk-248



MAT 9749

Particle Production
293 Kelvin Cross Sections

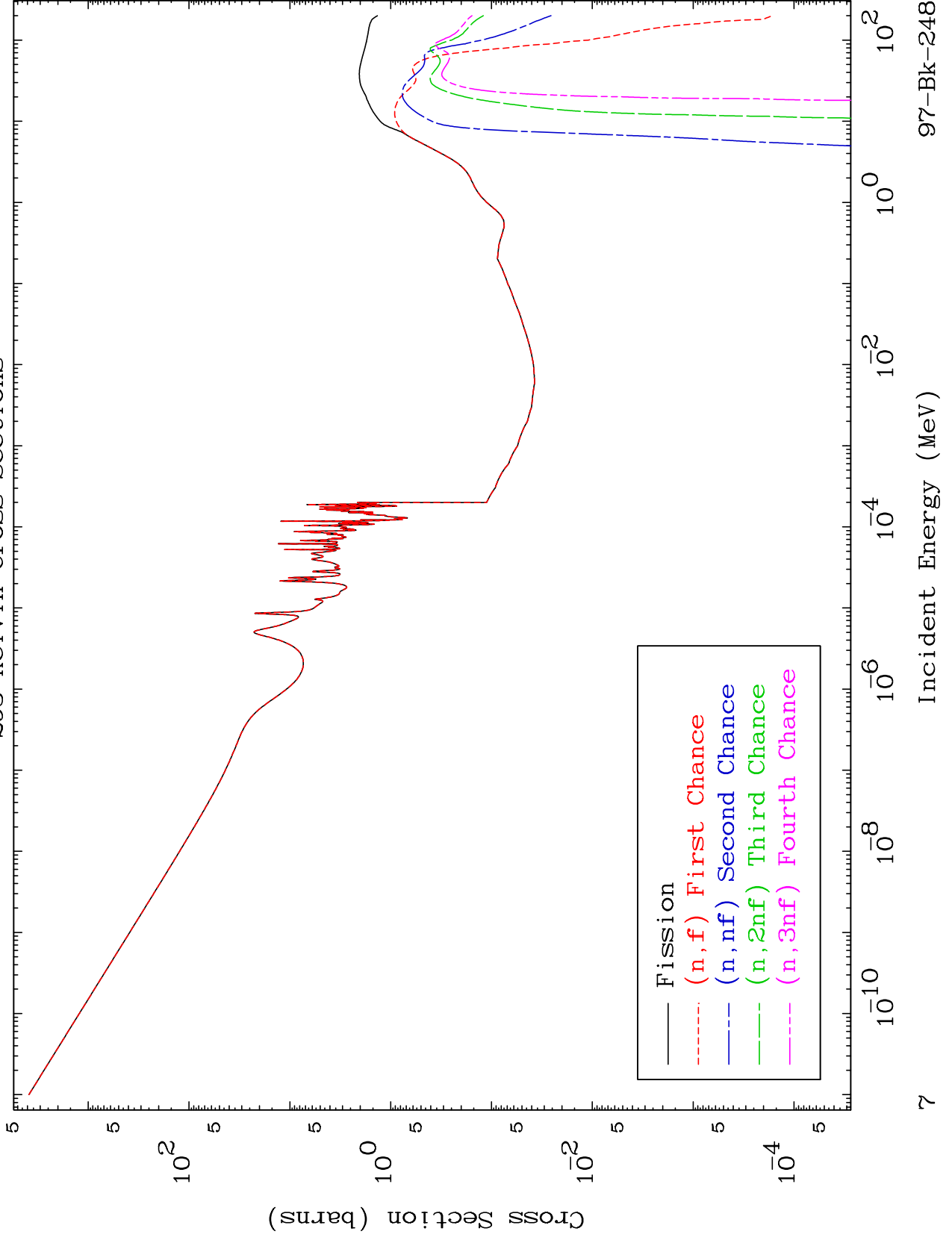
97-Bk-248



MAT 9749

Fission
293 Kelvin Cross Sections

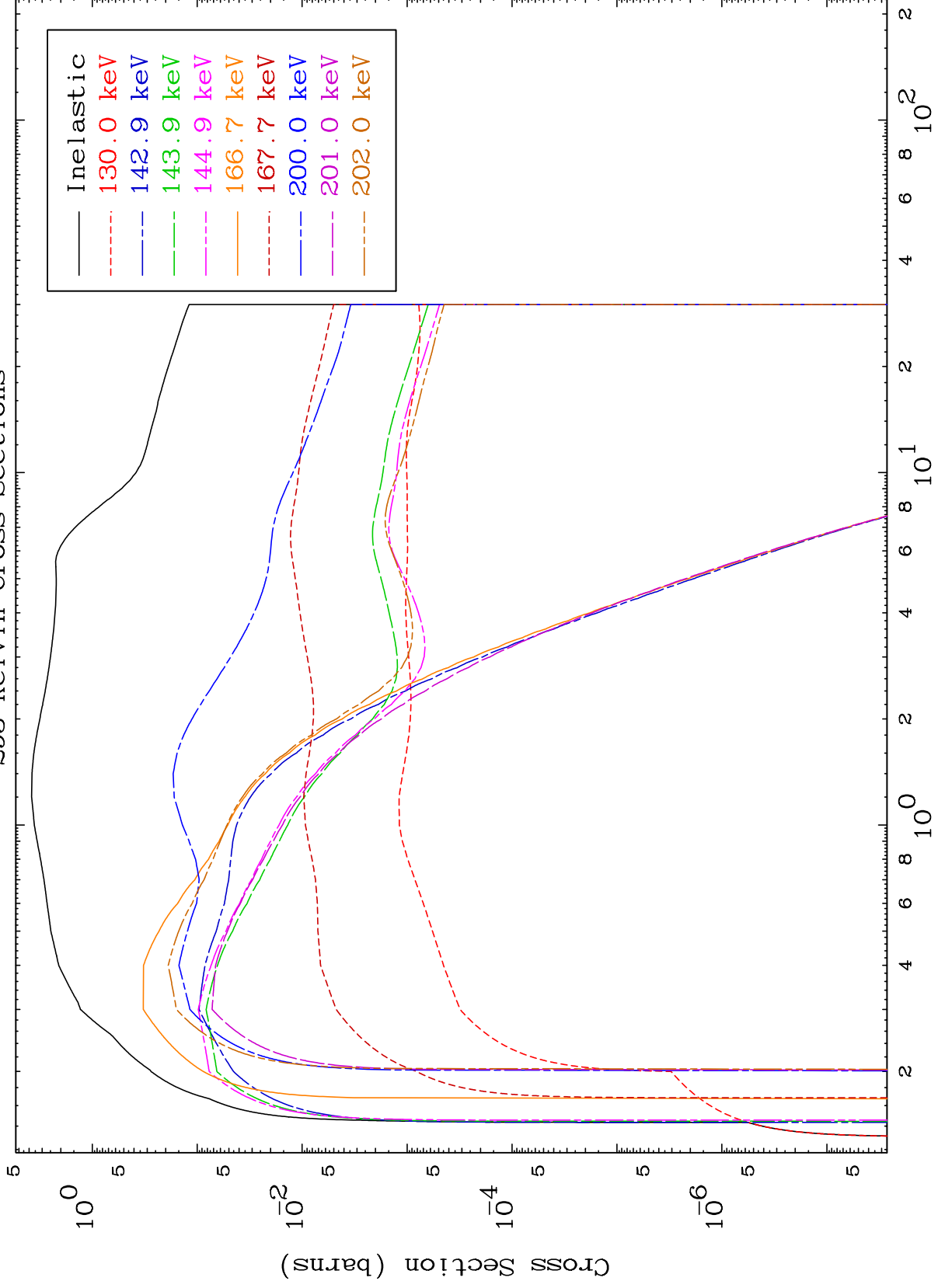
97-Bk-248



MAT 9749

(n,n') Level
293 Kelvin Cross Sections

97-Bk-248



8

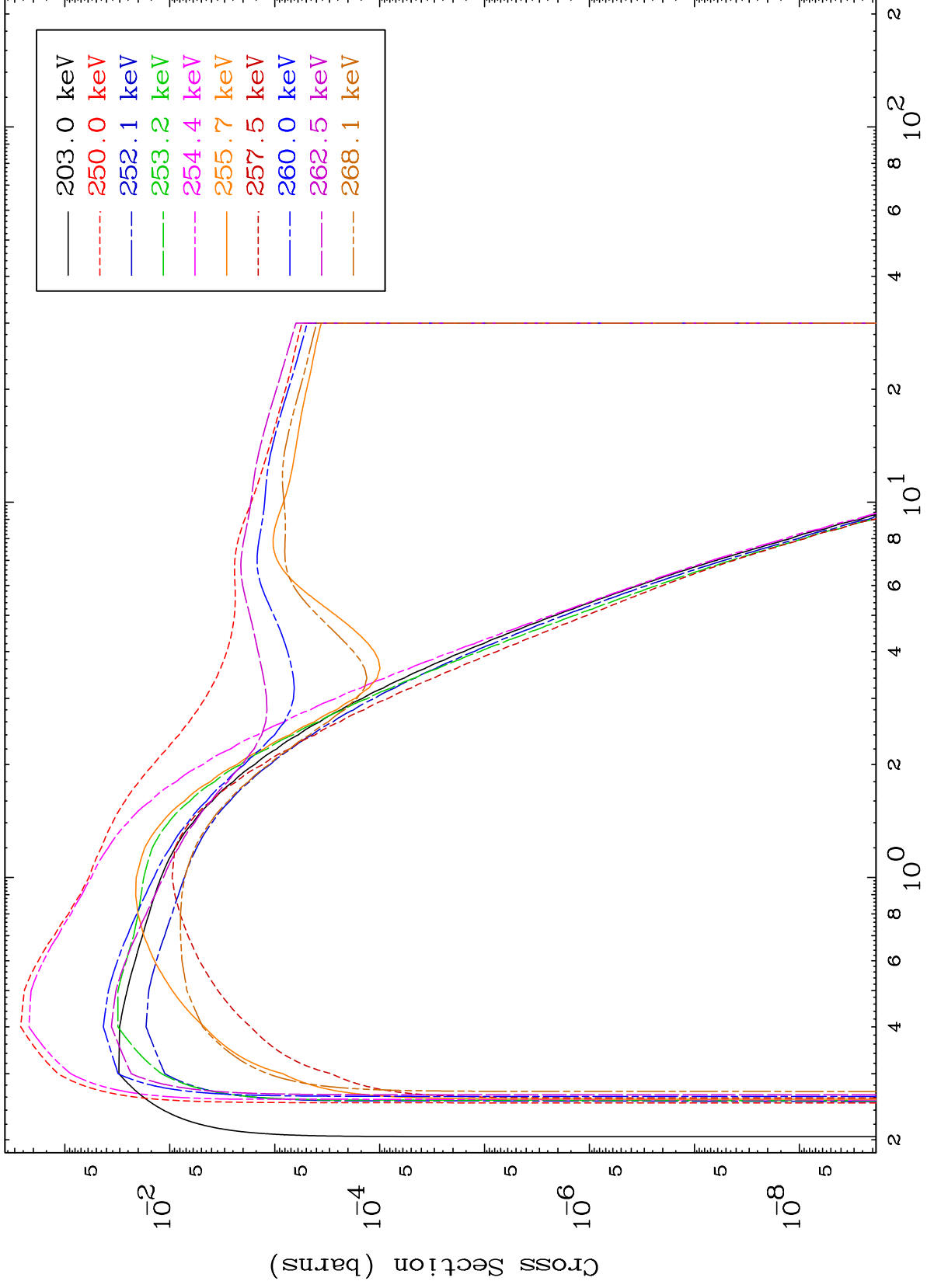
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,n') Level
293 Kelvin Cross Sections

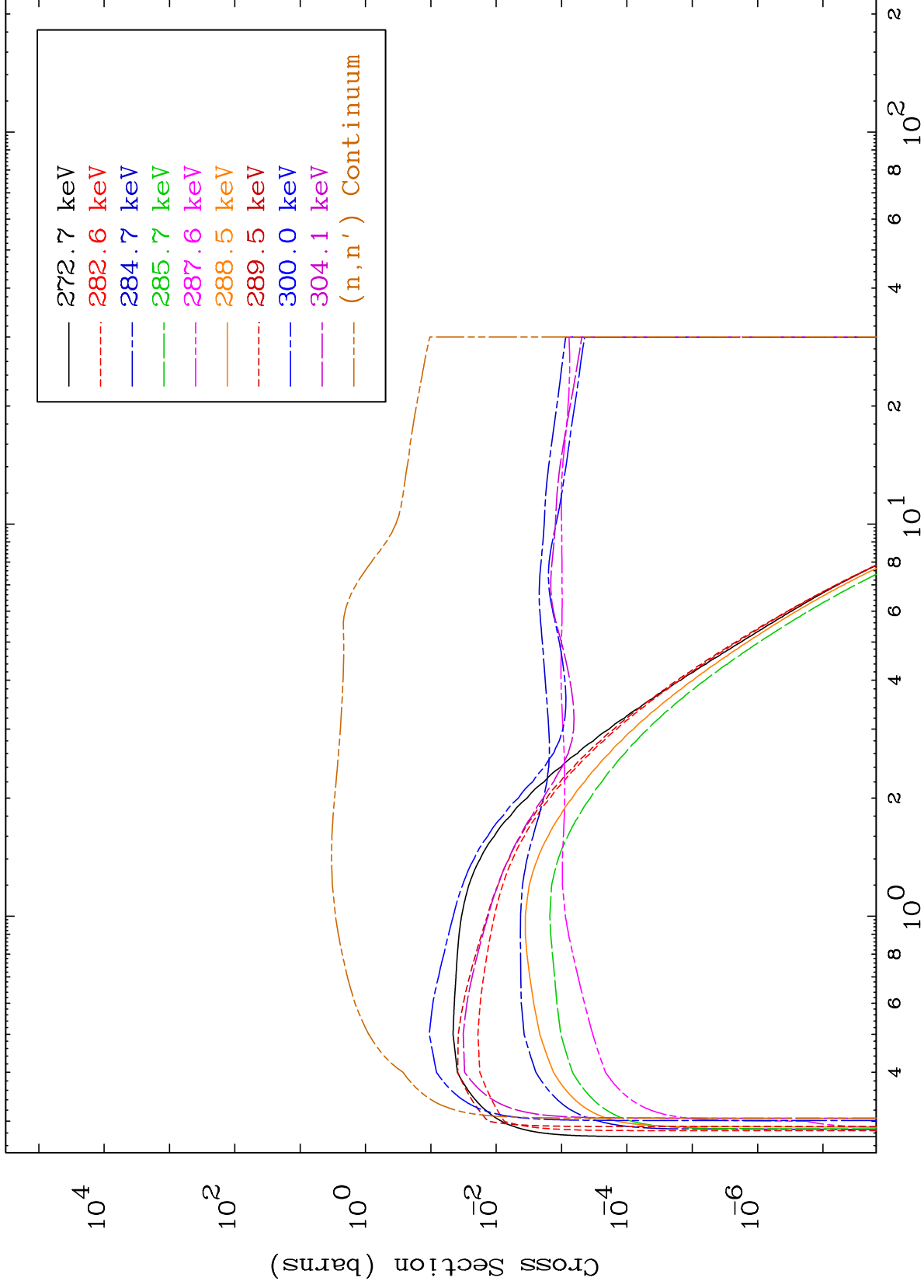
97-Bk-248



MAT 9749

(n,n') Level
293 Kelvin Cross Sections

97-Bk-248



10

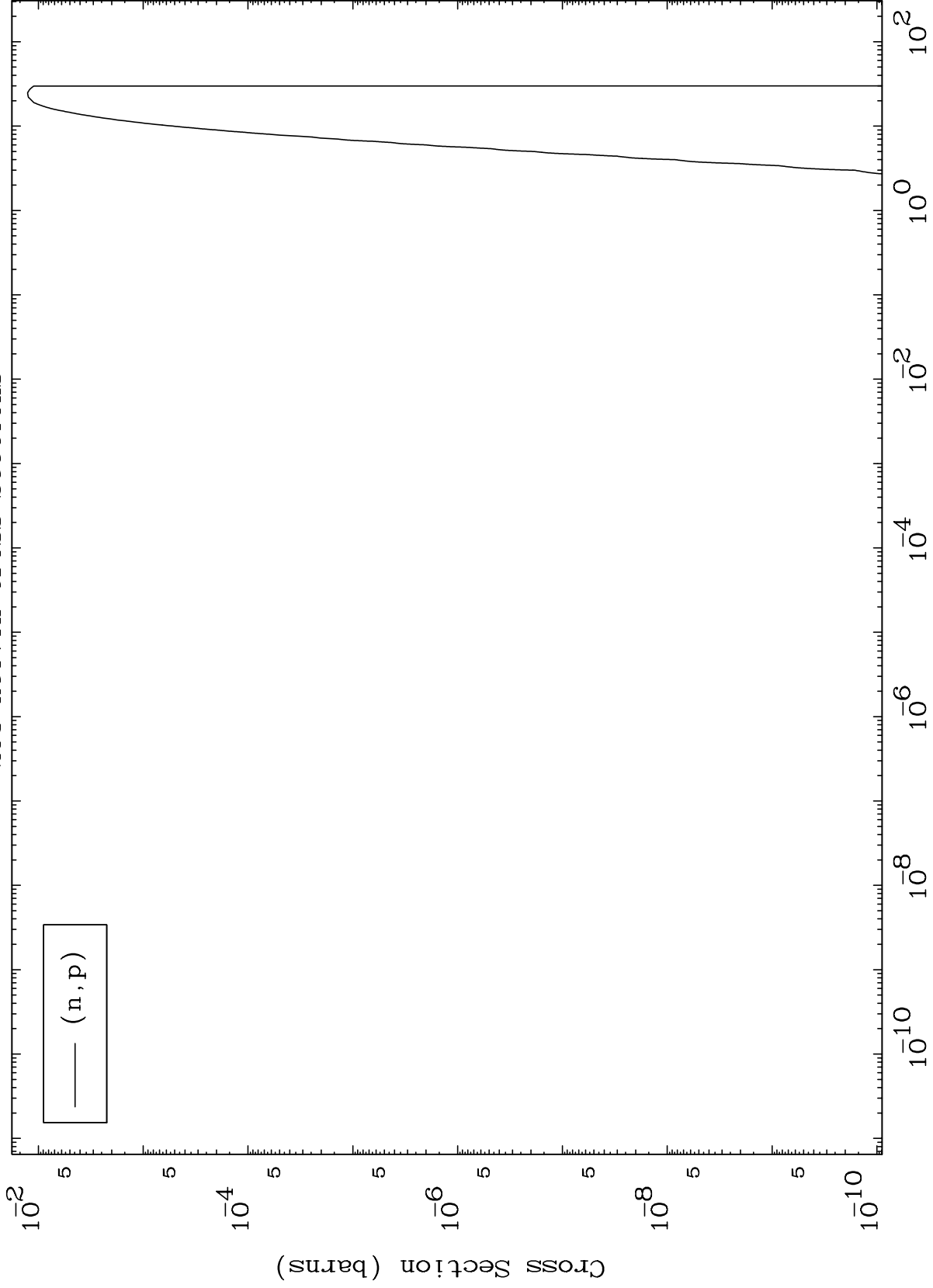
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,p) Levels
293 Kelvin Cross Sections

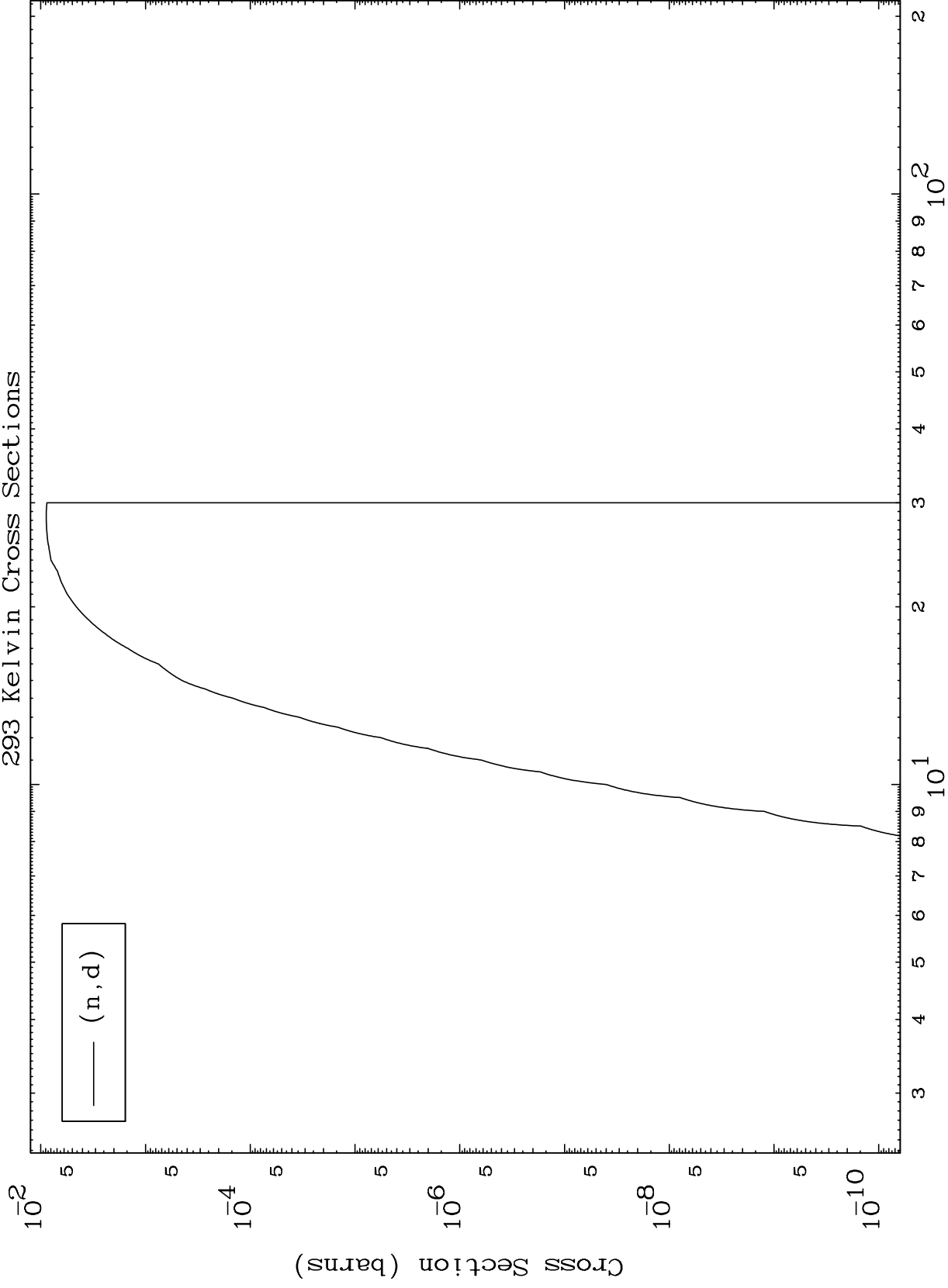
97-Bk-248



MAT 9749

(n,d) Levels
293 Kelvin Cross Sections

97-Bk-248



12

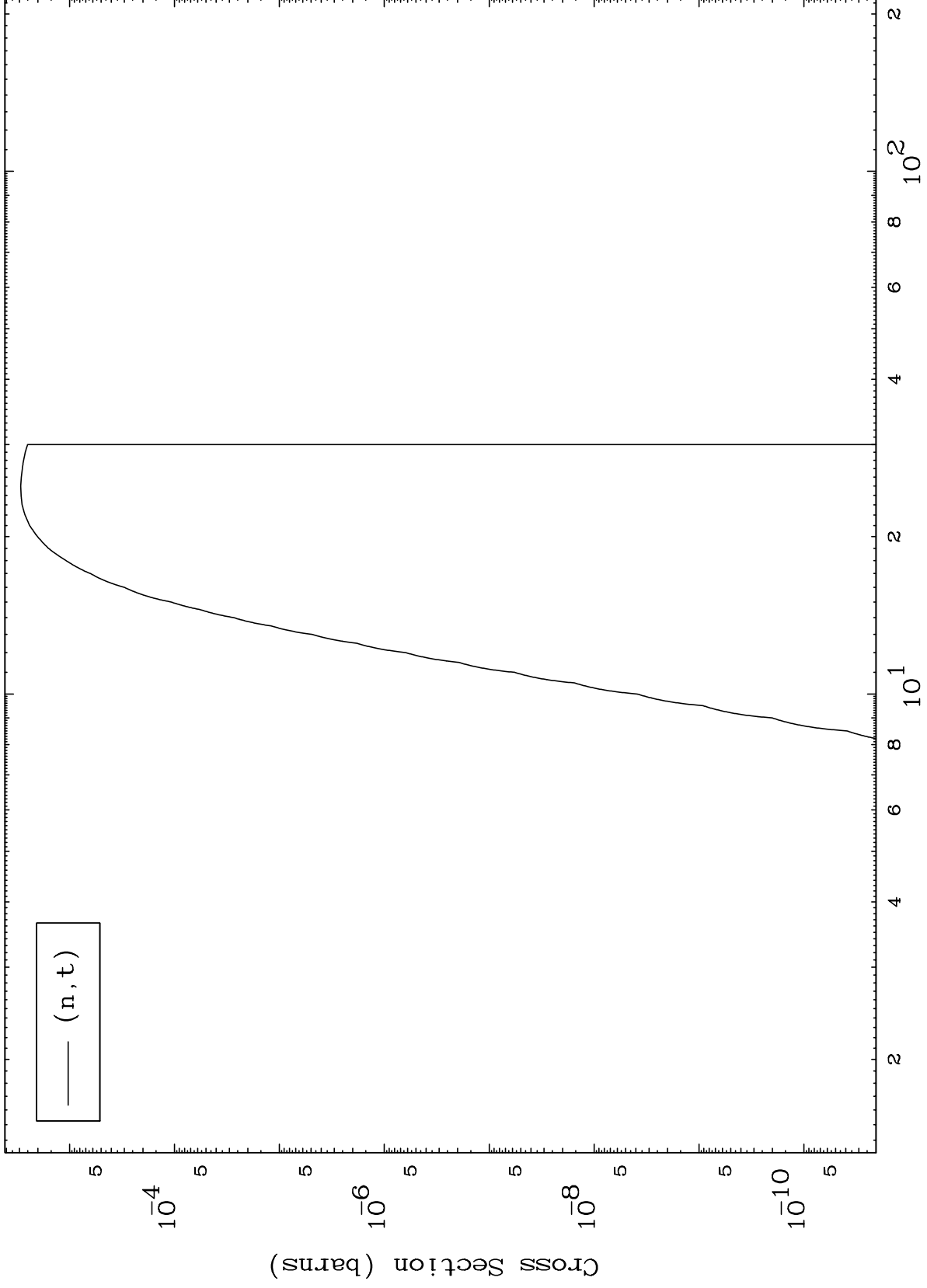
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-248



13

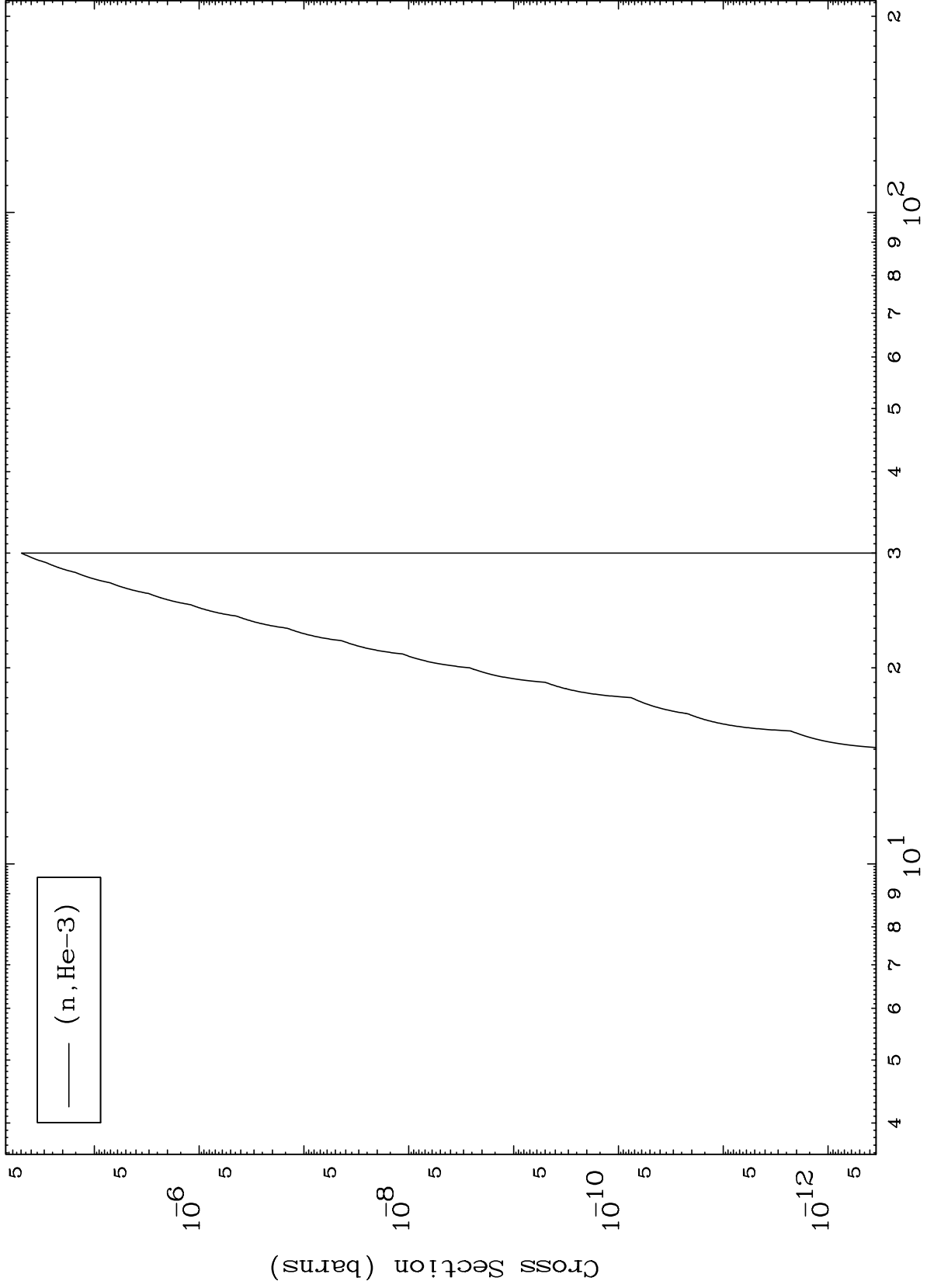
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-248



14

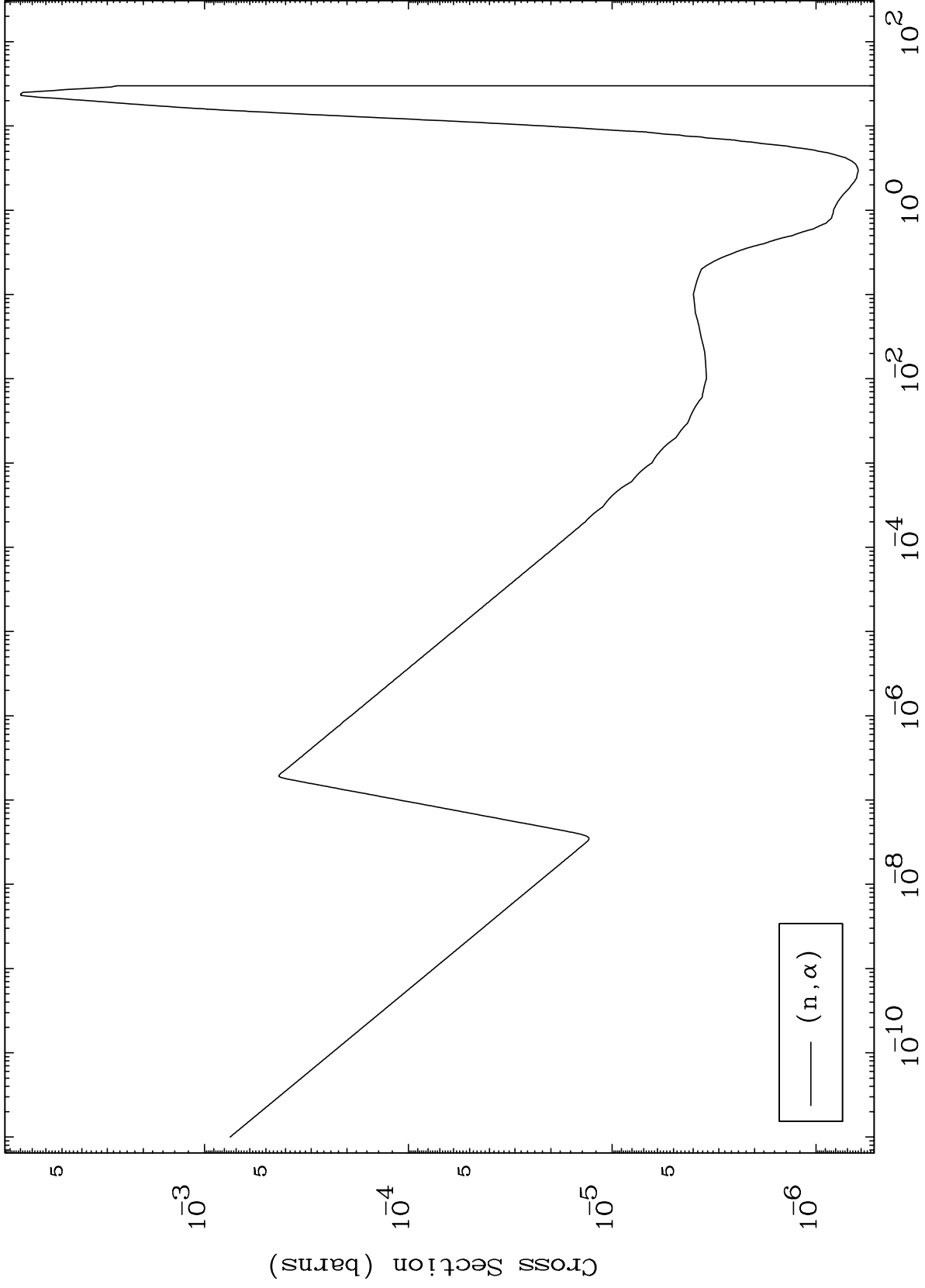
Incident Energy (MeV)

97-Bk-248

MAT 9749

(n, α) Levels
293 Kelvin Cross Sections

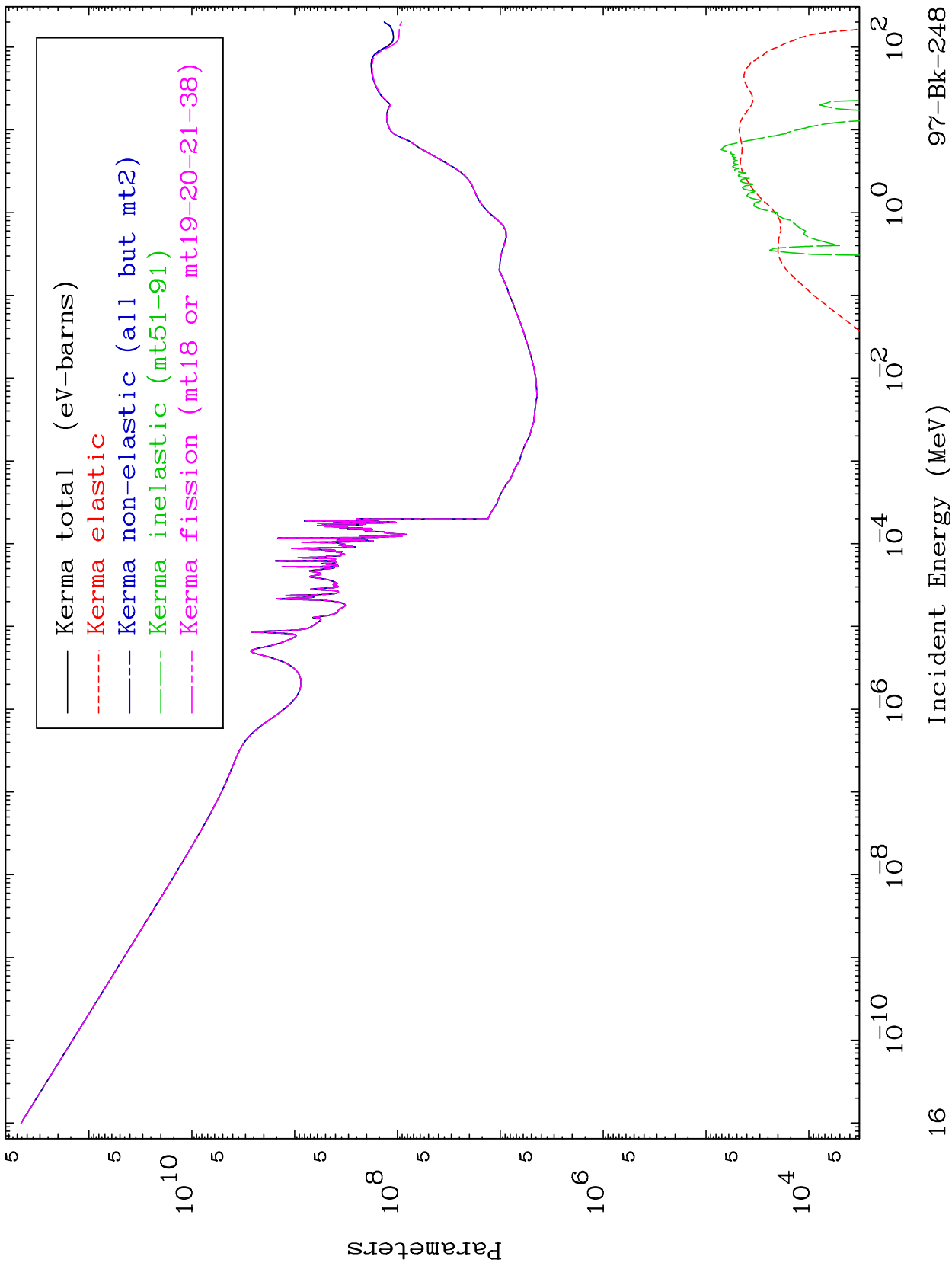
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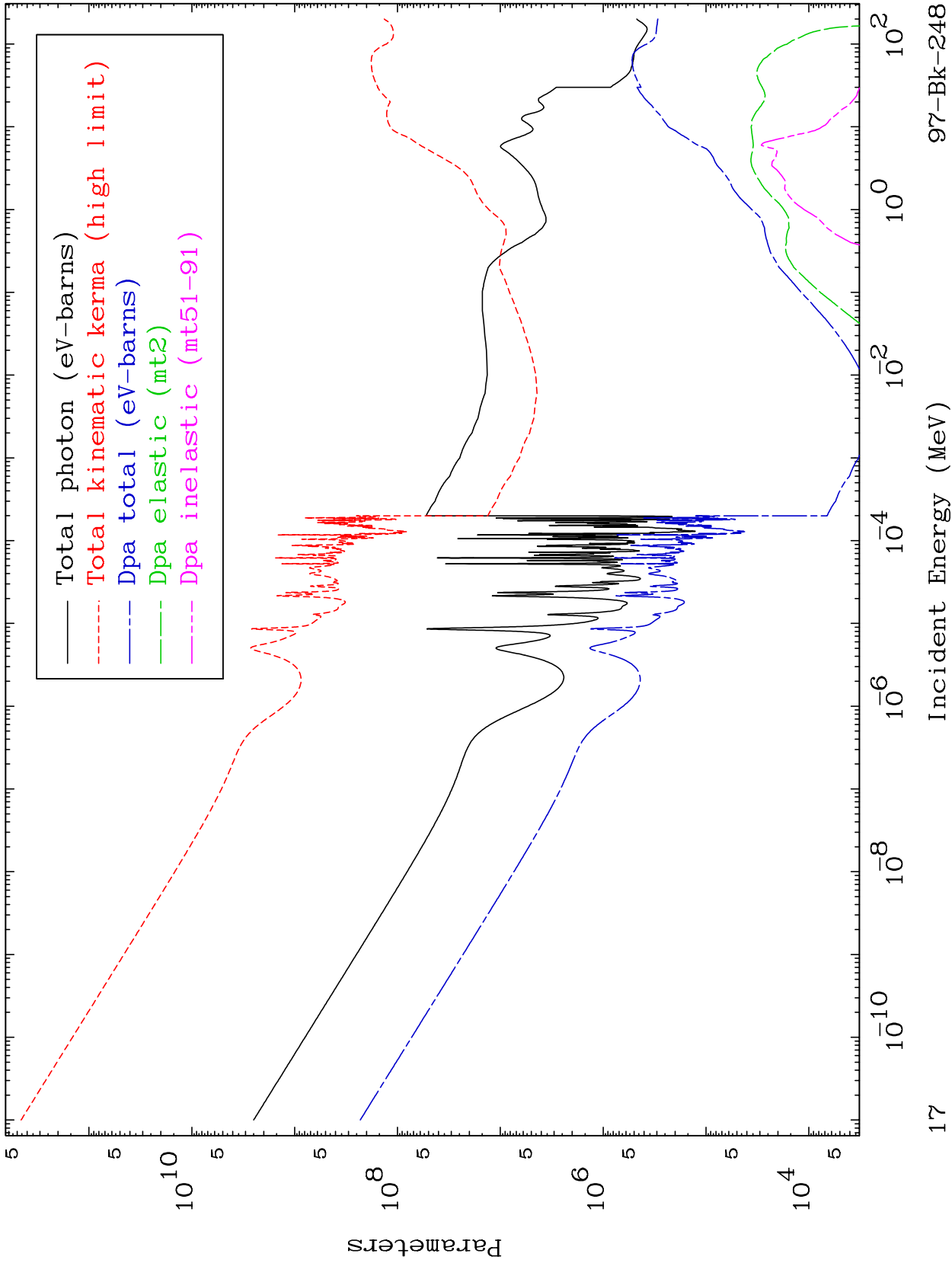


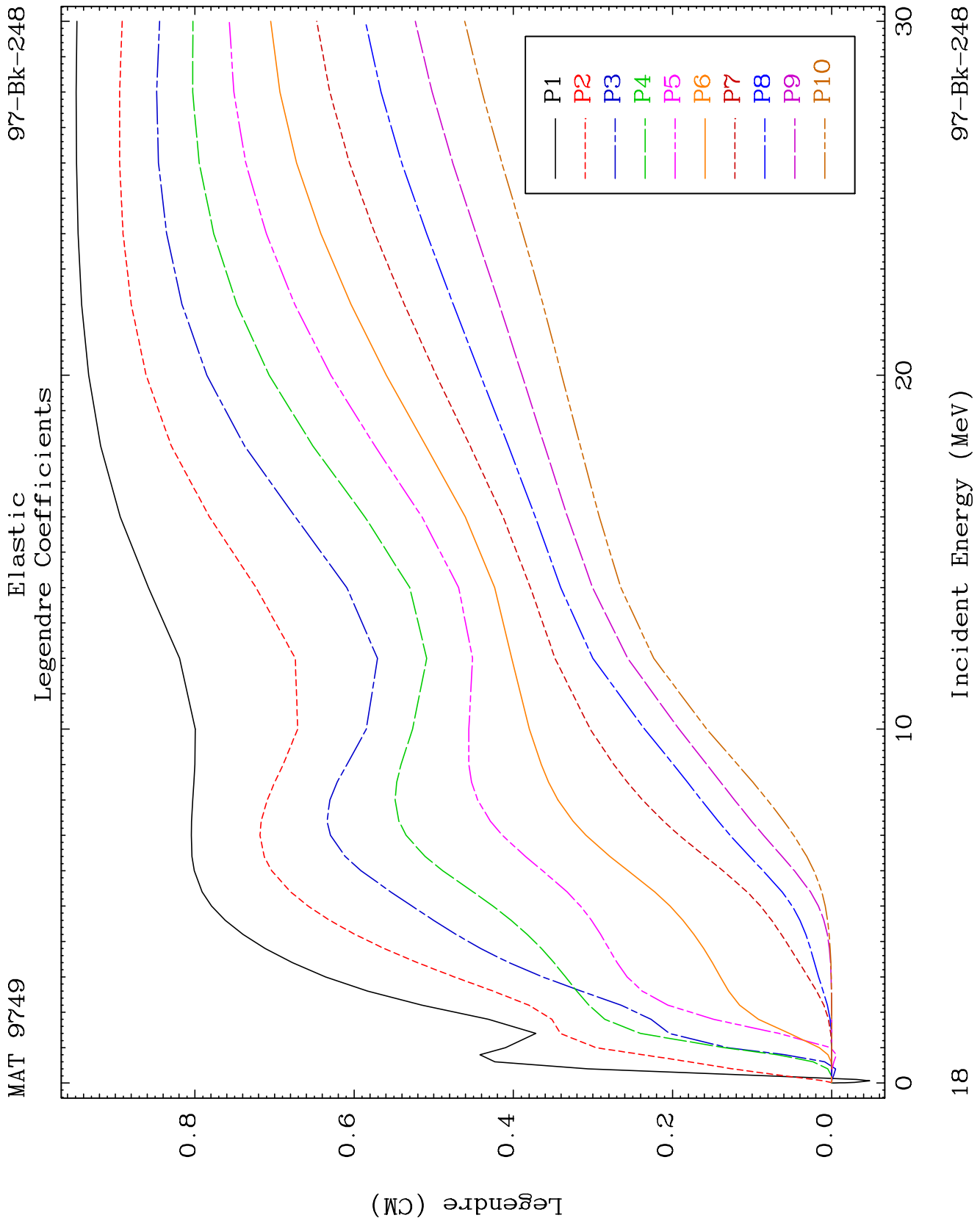
15

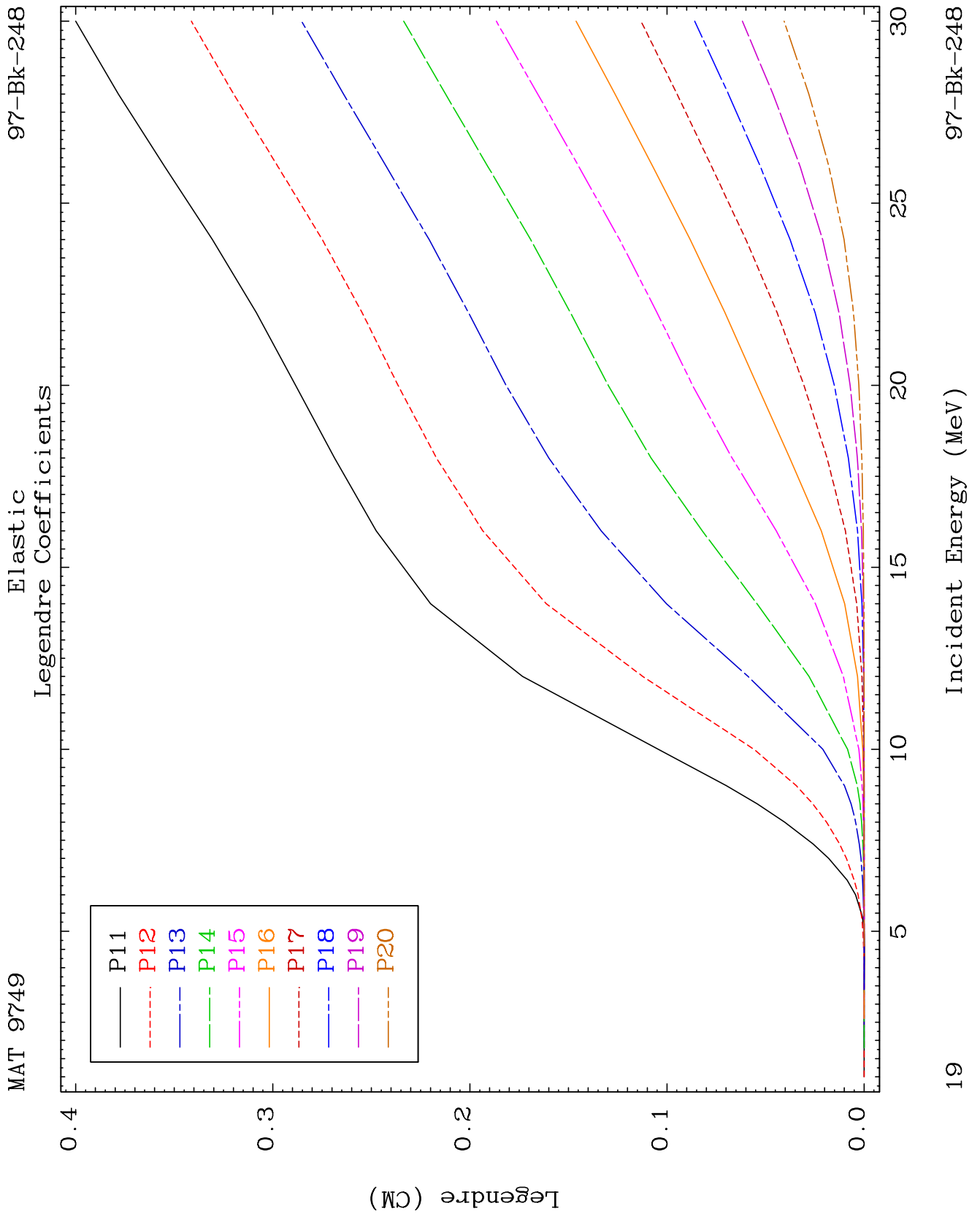
Incident Energy (MeV)

97-Bk-248





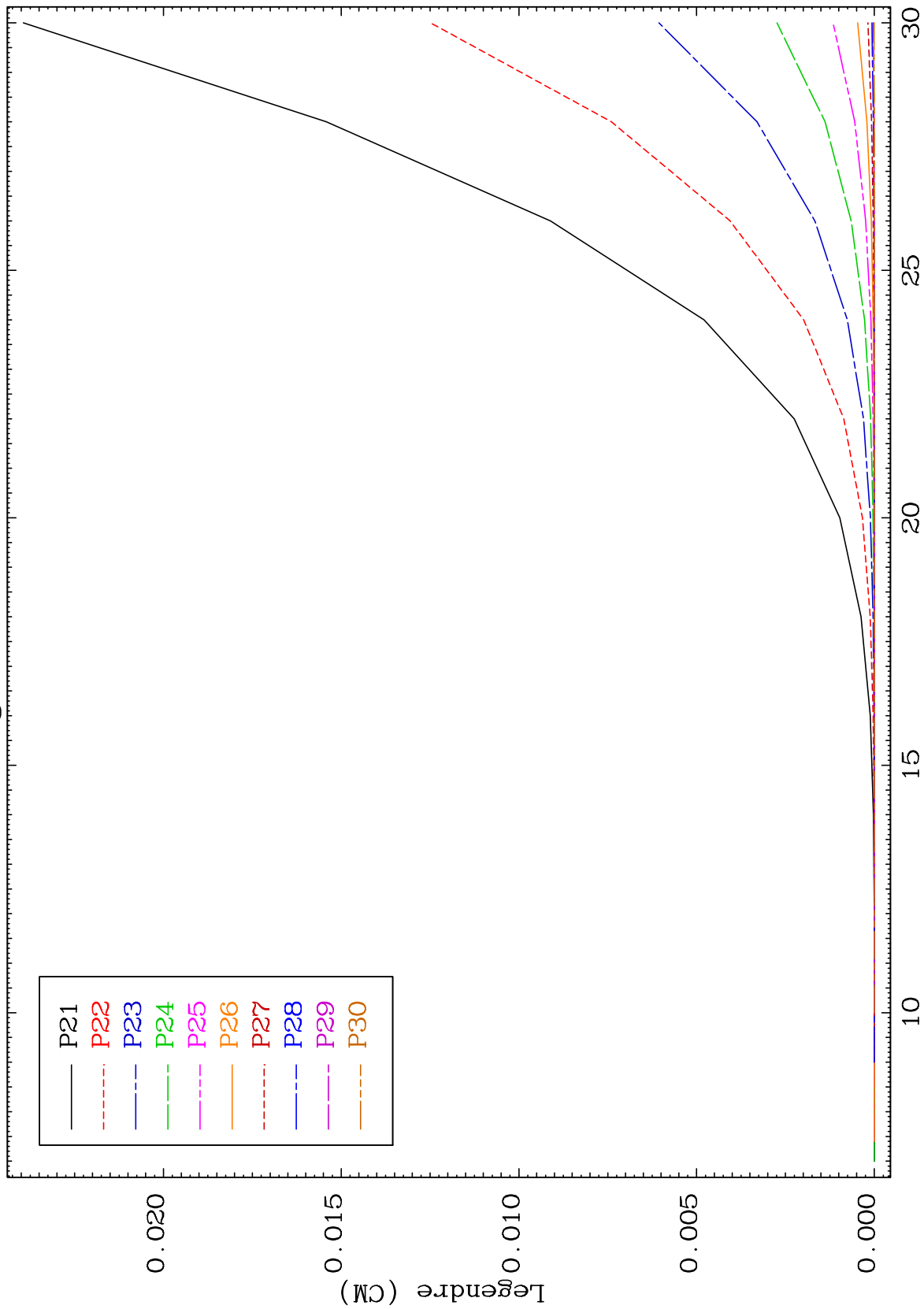




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Elastic Legendre Coefficients

97-Bk-248



20

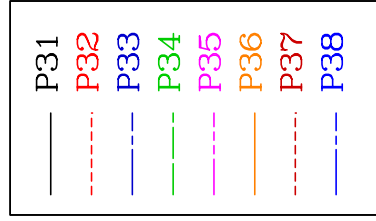
Incident Energy (MeV)

97-Bk-248

MAT 9749

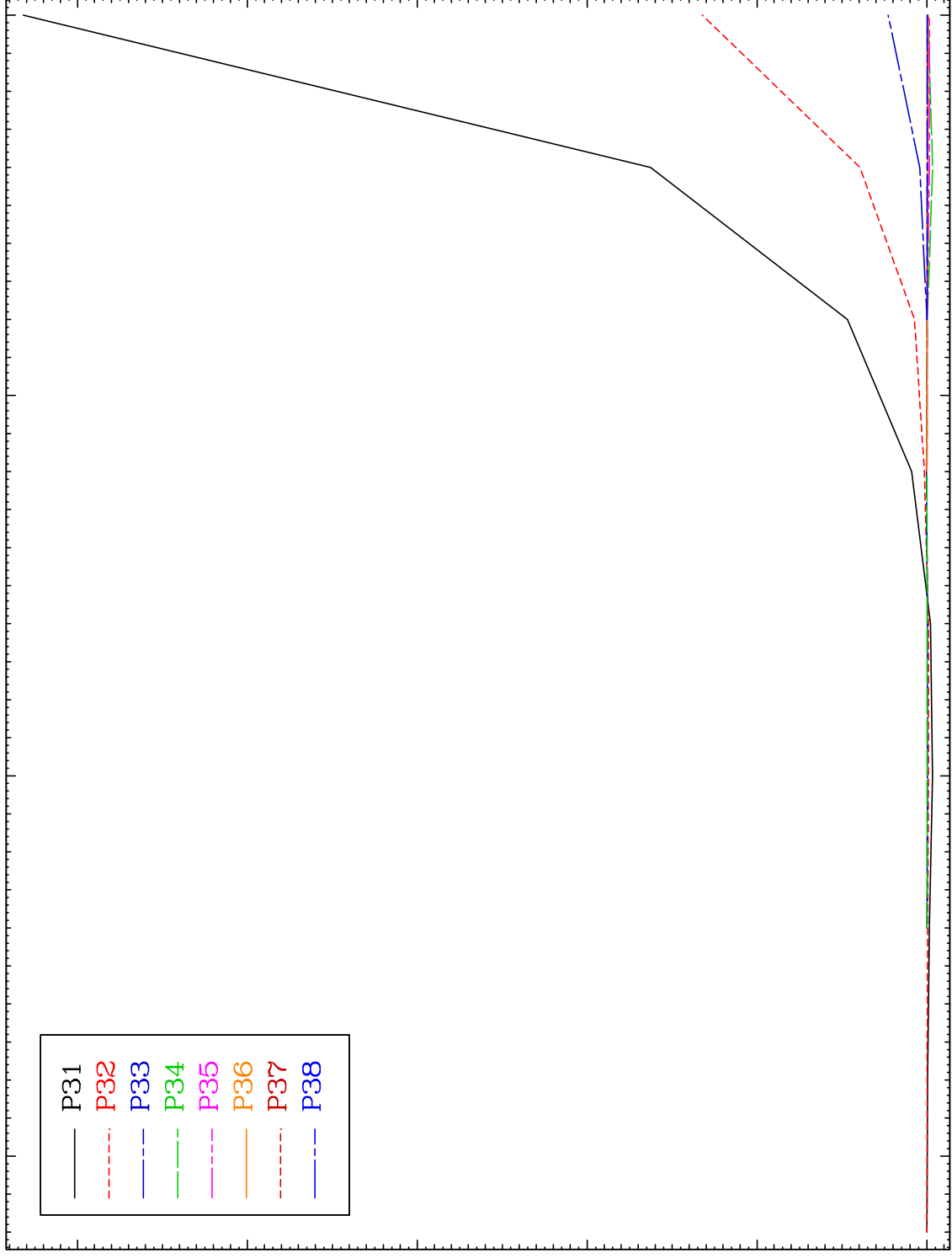
Elastic Legendre Coefficients

97-Bk-248



$\times 10^{-6}$

Legendre (CM)



30

25

20

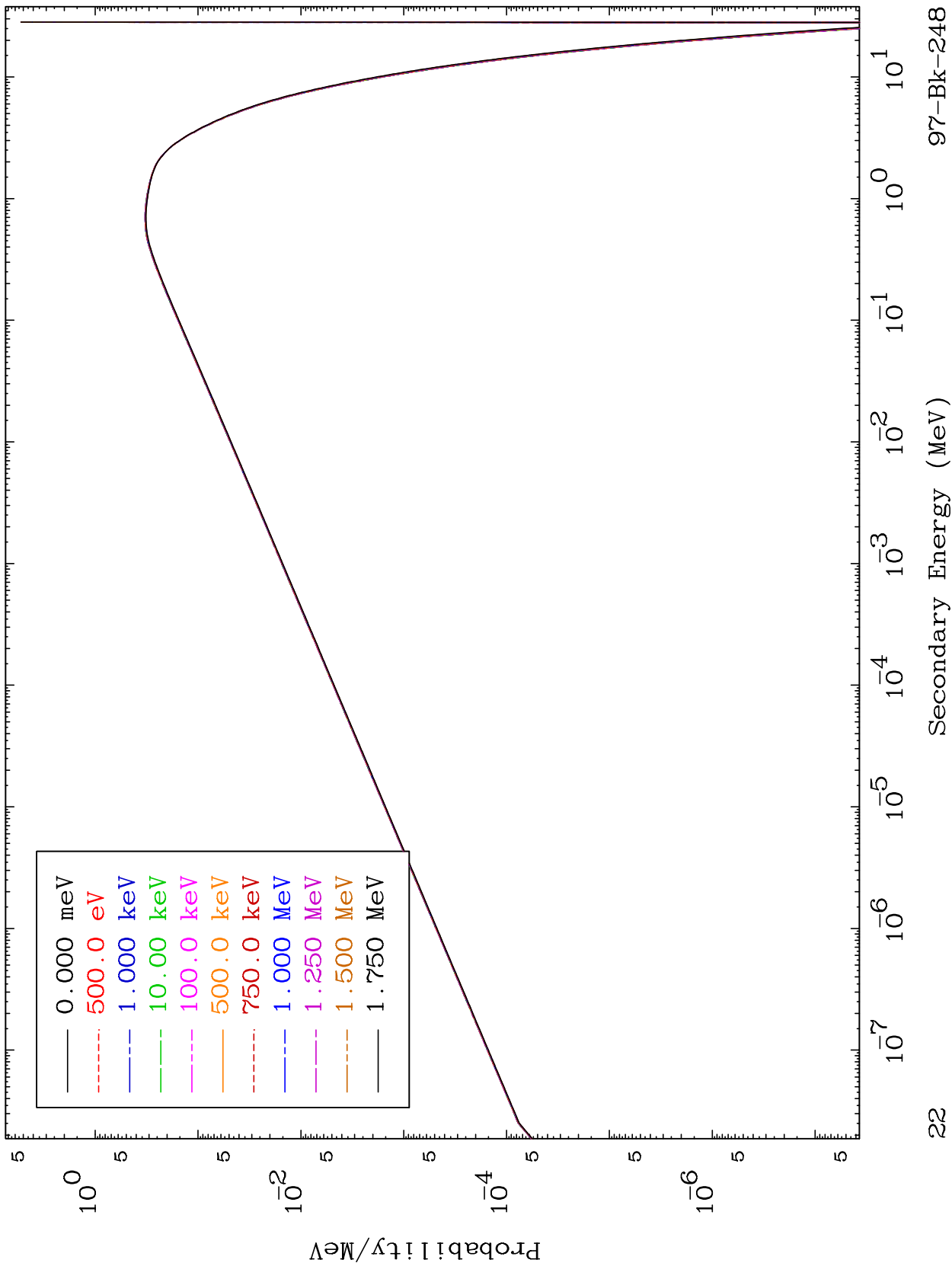
15

21

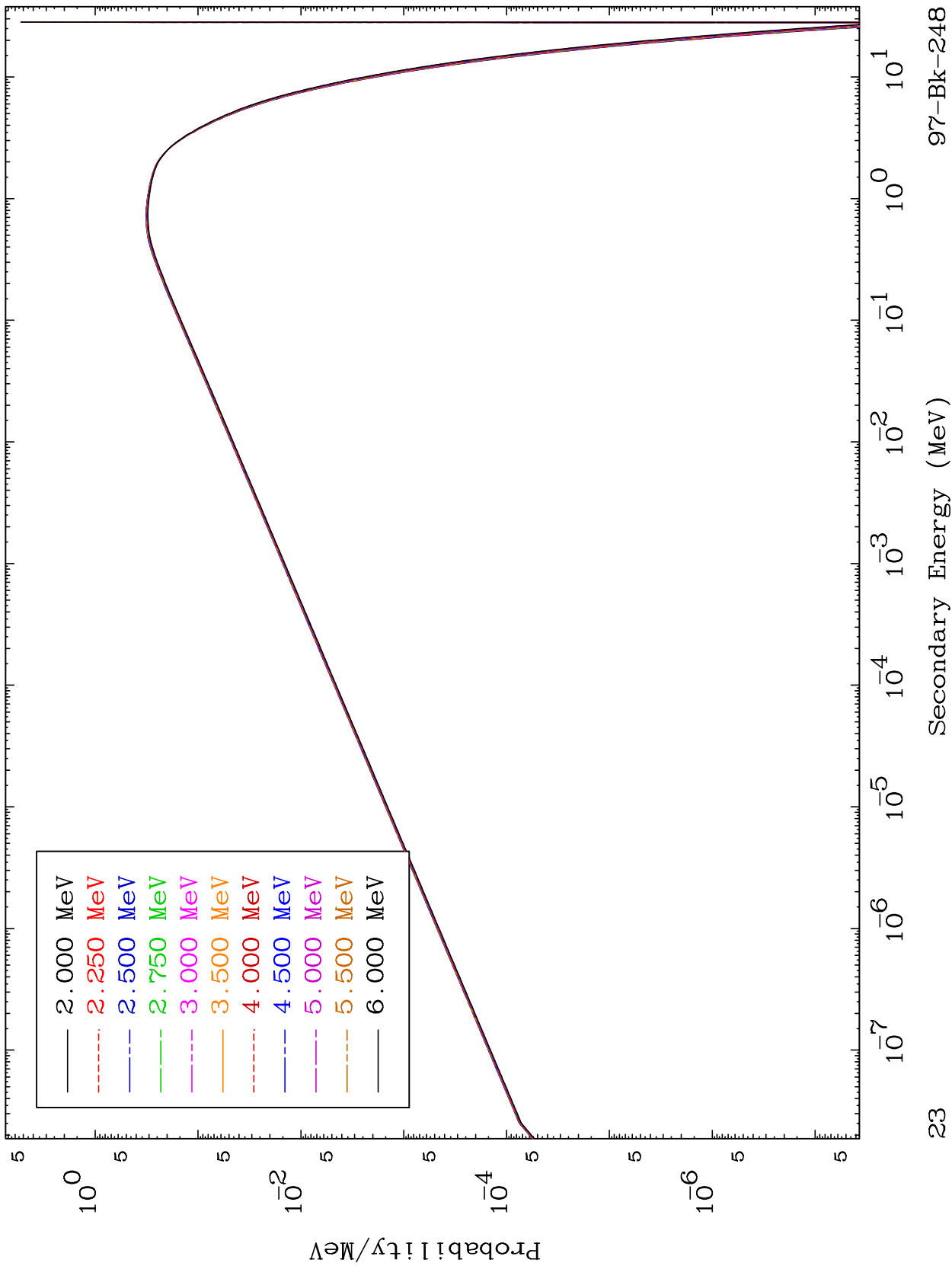
Incident Energy (MeV)

97-Bk-248

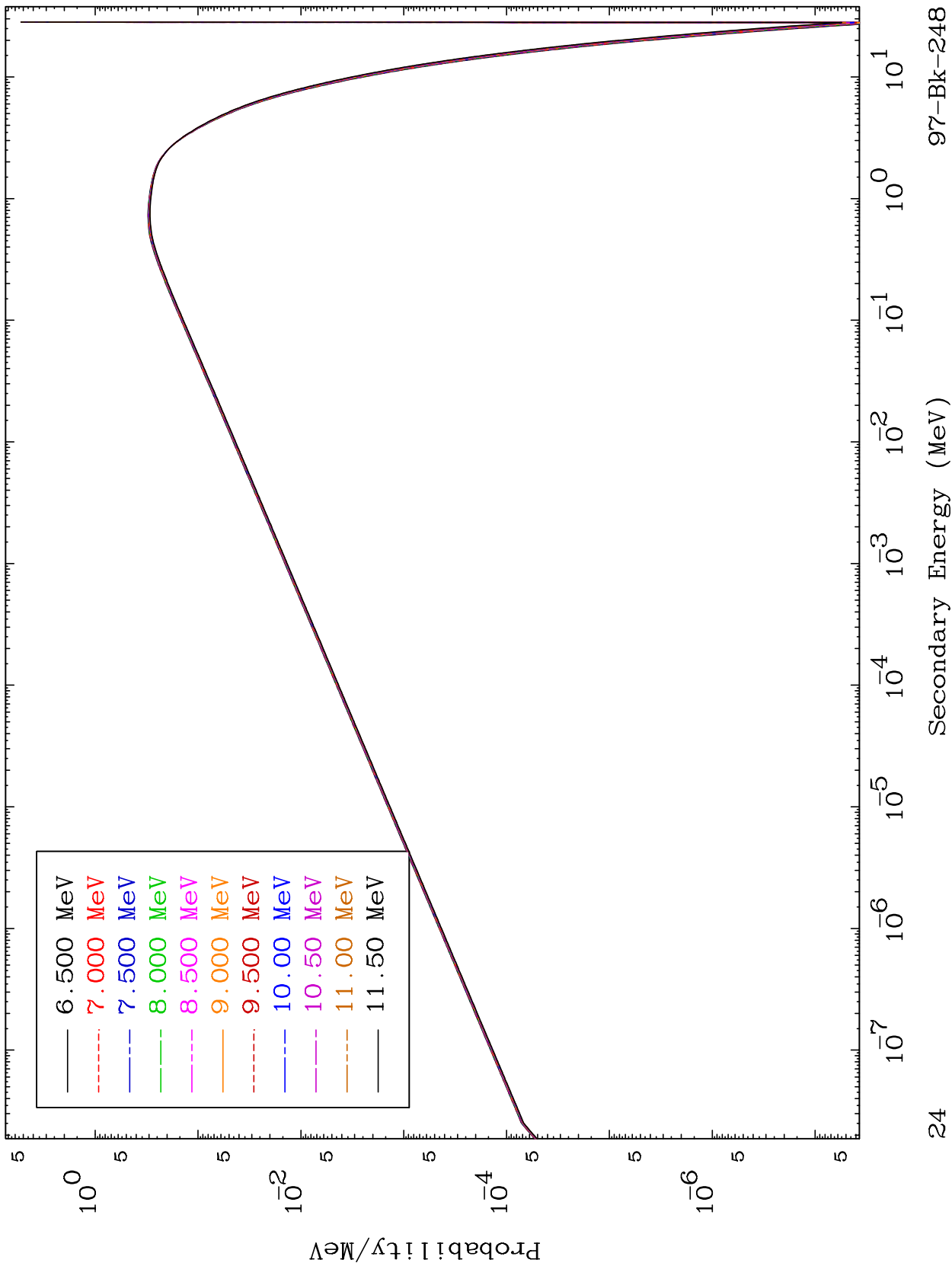
Fission Energy Distribution



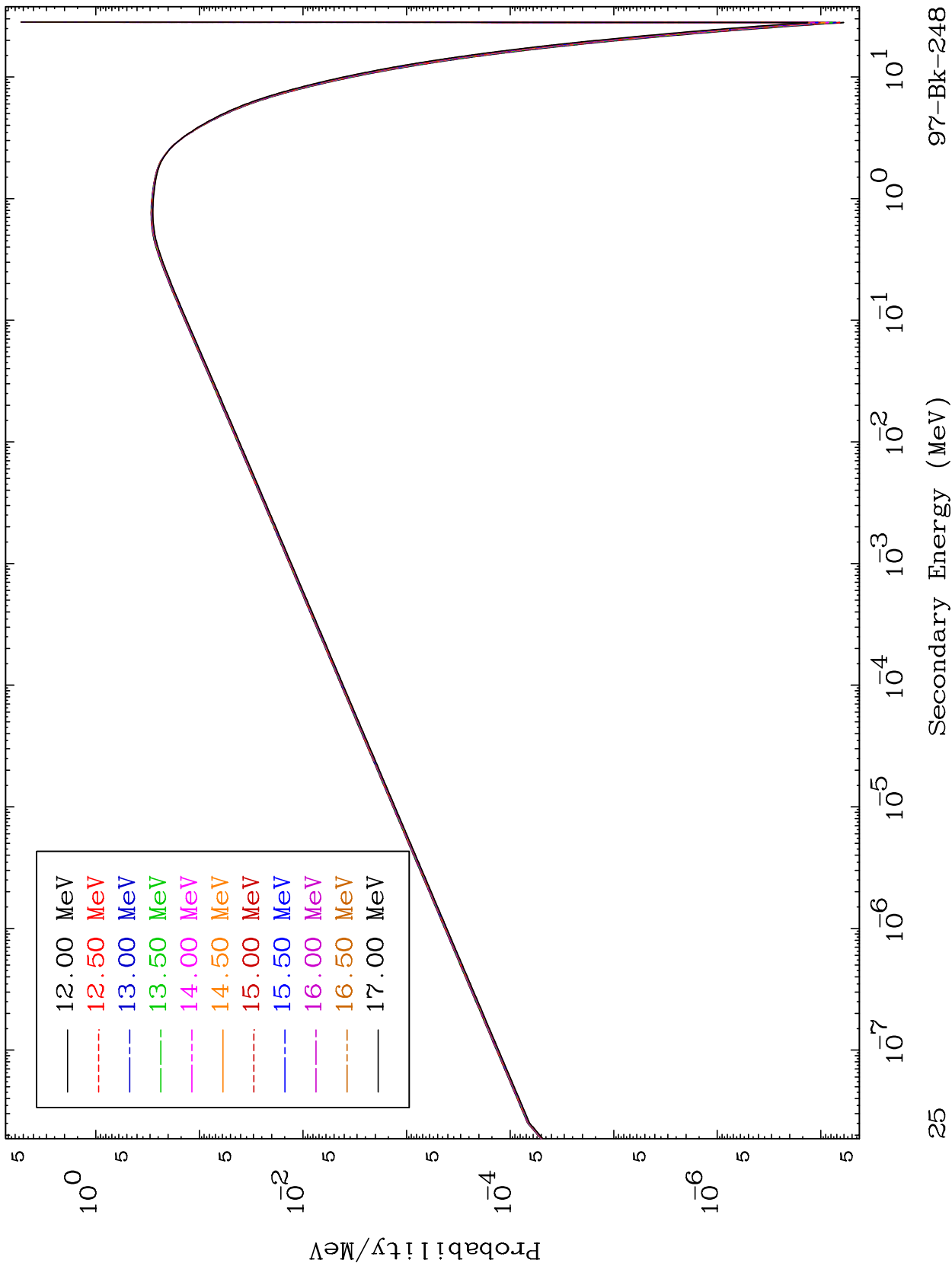
Fission Energy Distribution



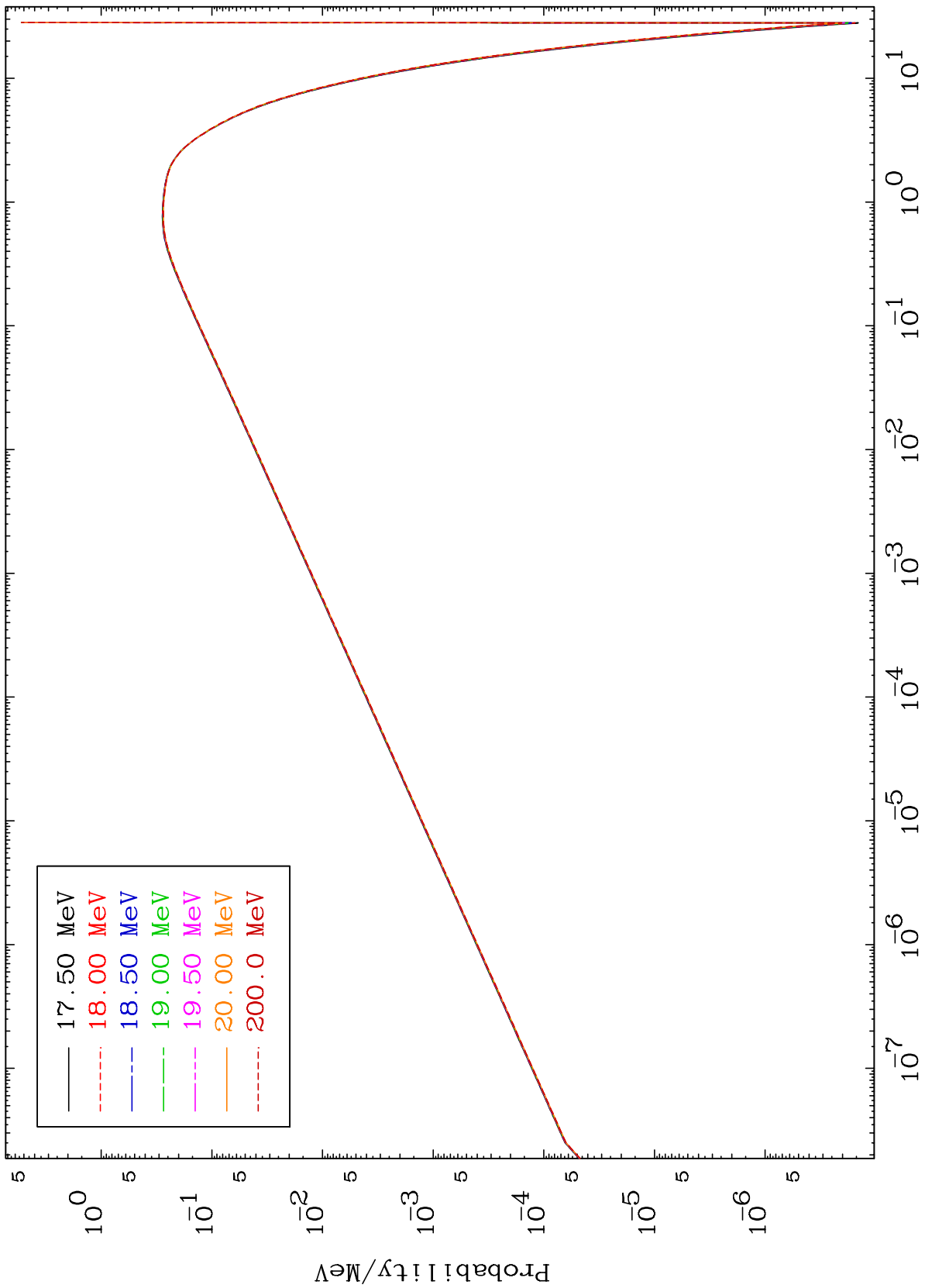
Fission Energy Distribution



Fission
Energy Distribution



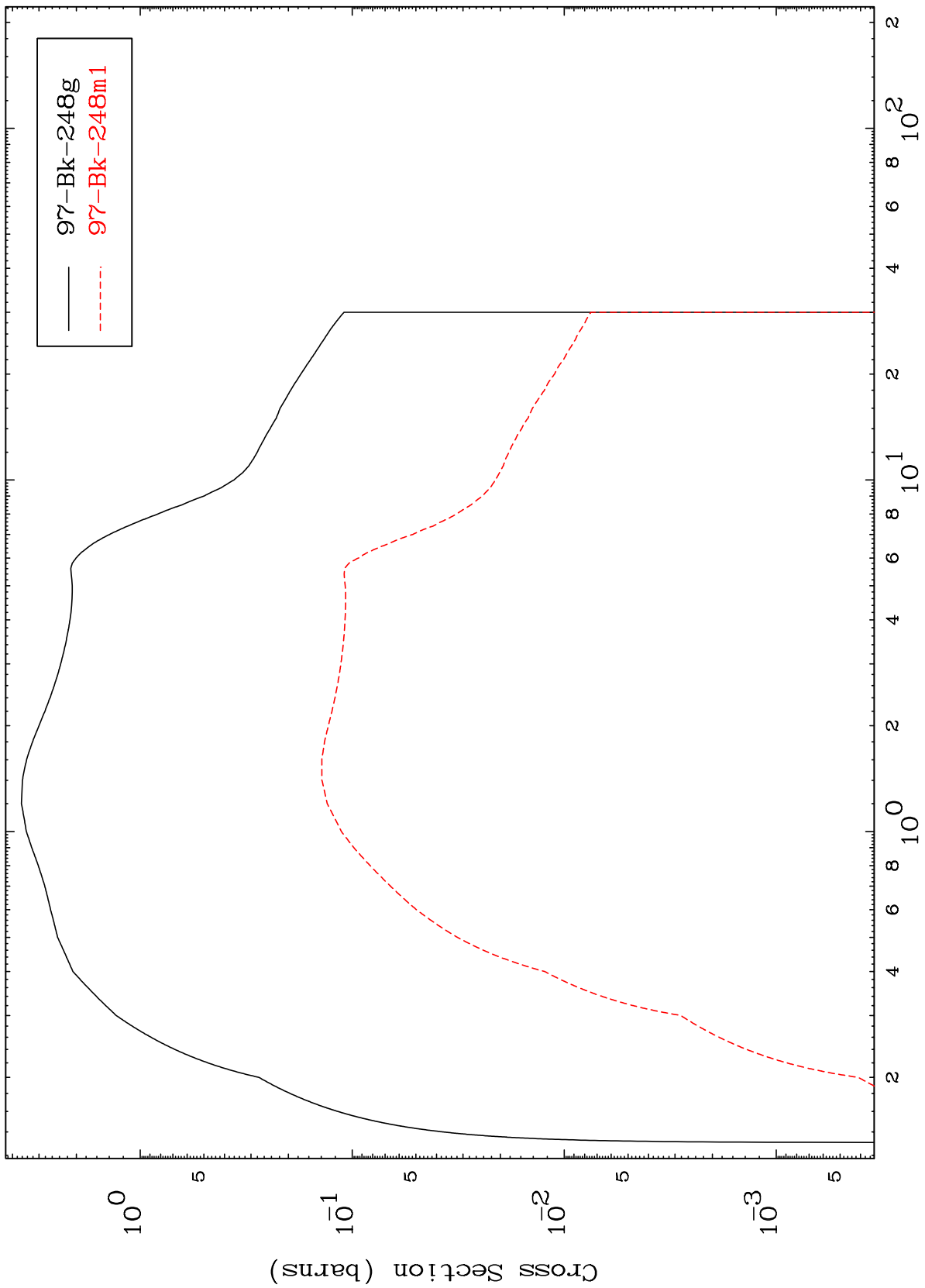
Fission
Energy Distribution



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97-Bk-248

Inelastic
Radionuclide Production Cross Section



— 97-Bk-248g
- - - 97-Bk-248m1

27

97-Bk-248

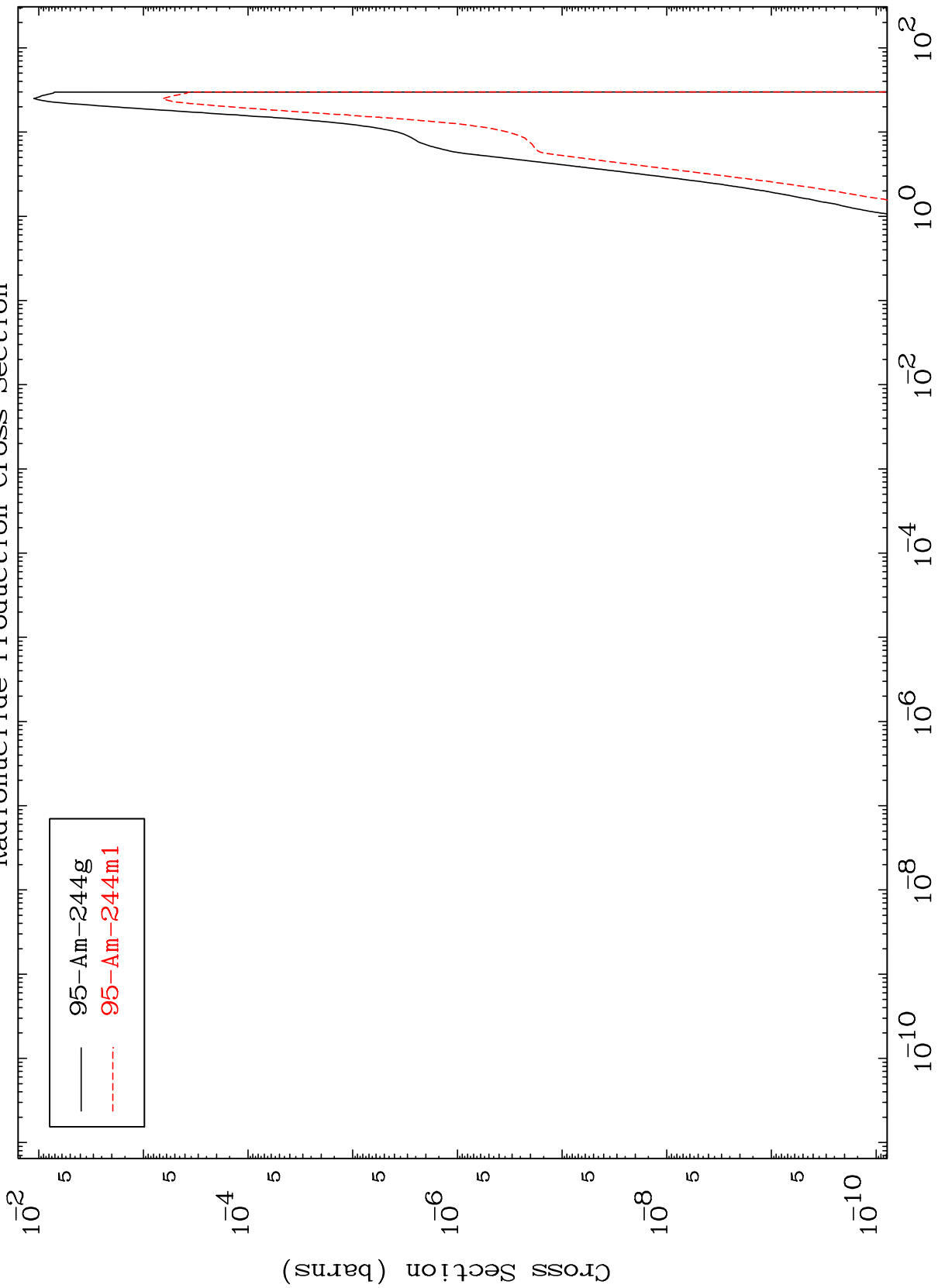
Incident Energy (MeV)

MAT 9749

(n,n') α

97-Bk-248

Radionuclide Production Cross Section

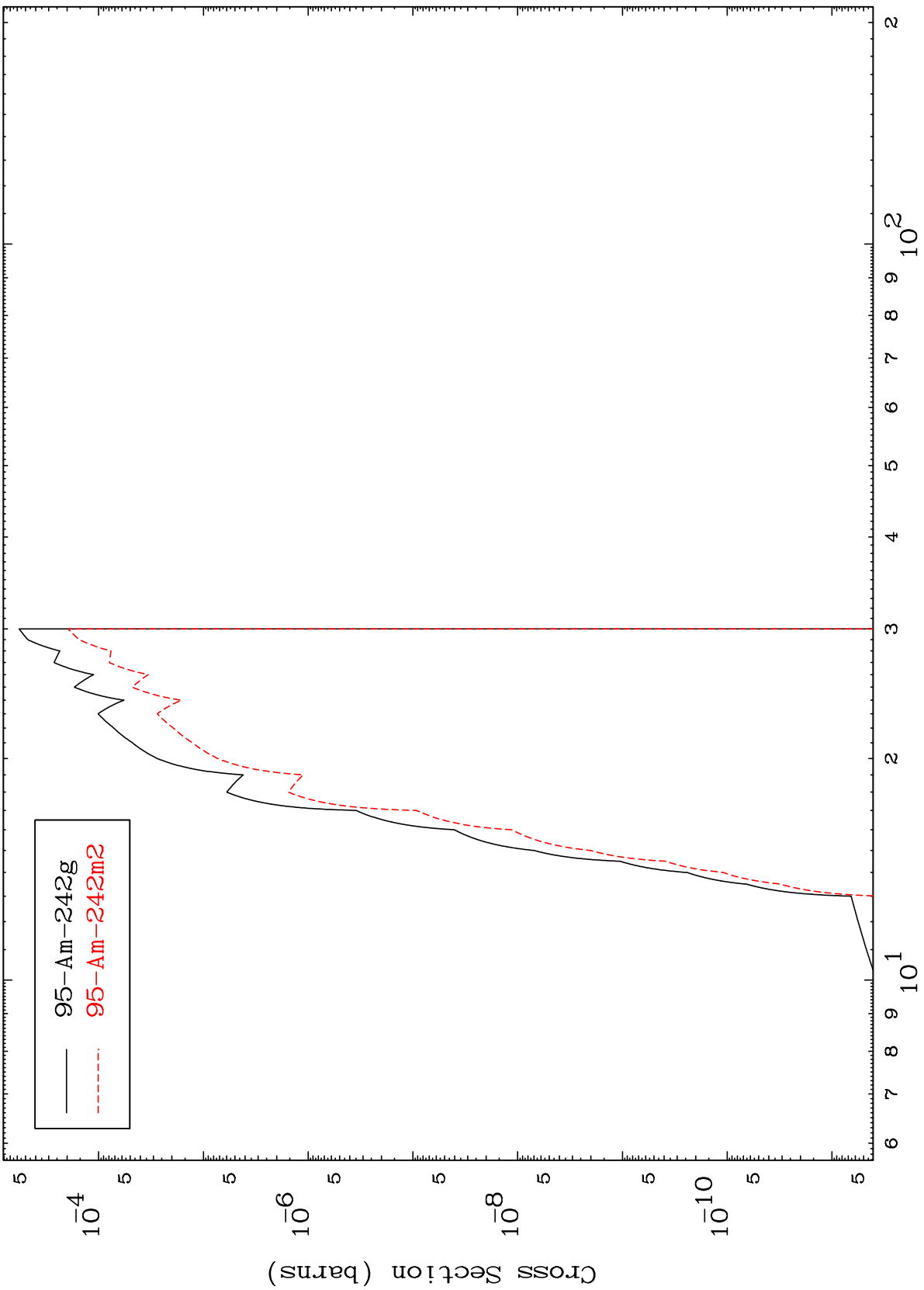


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(n,3n) α

97-Bk-248

Radionuclide Production Cross Section



29

Incident Energy (MeV)

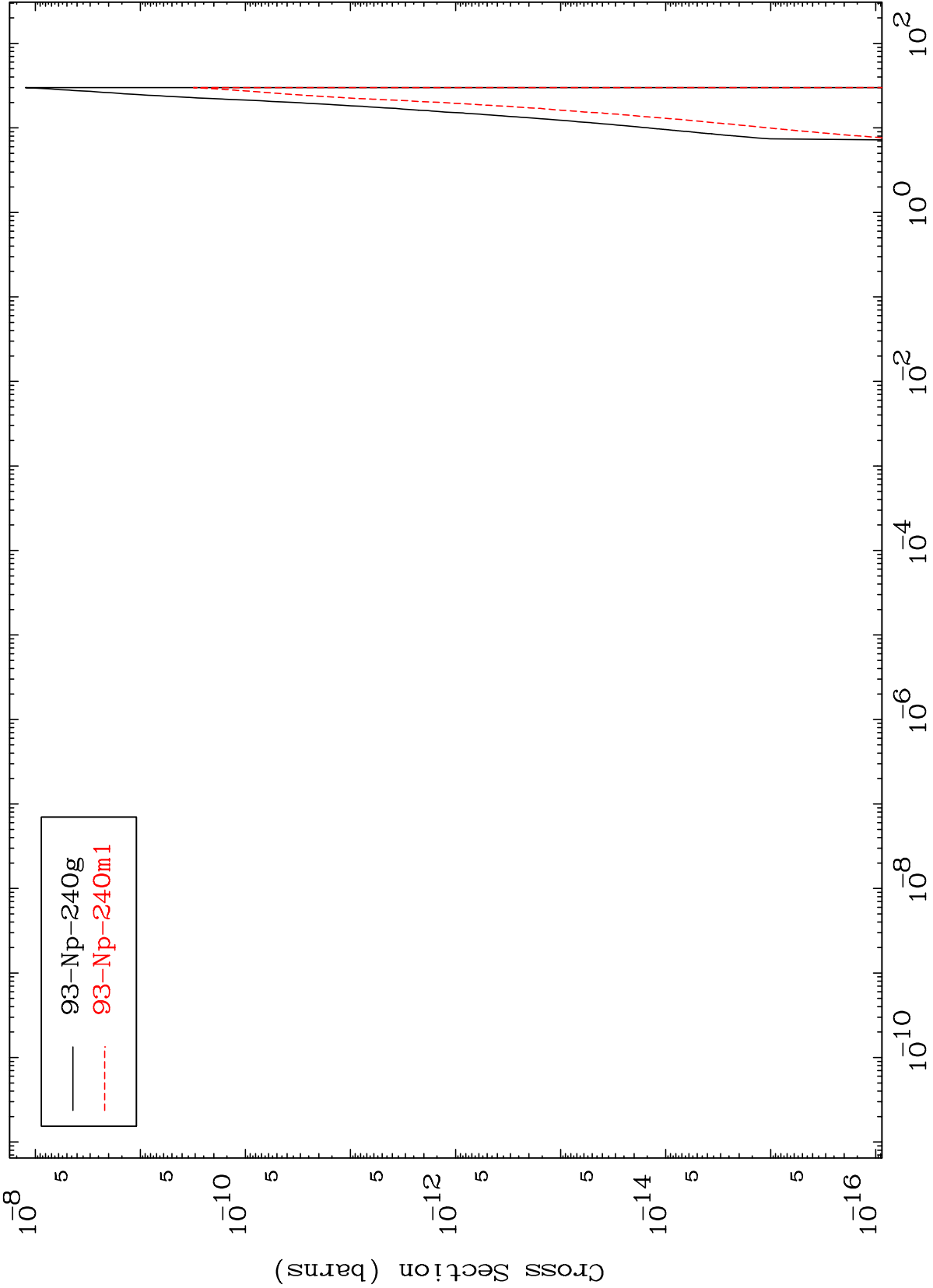
97-Bk-248

MAT 9749

(n,n') 2α

97-Bk-248

Radionuclide Production Cross Section



93-Np-240g
93-Np-240m1

Incident Energy (MeV)

97-Bk-248

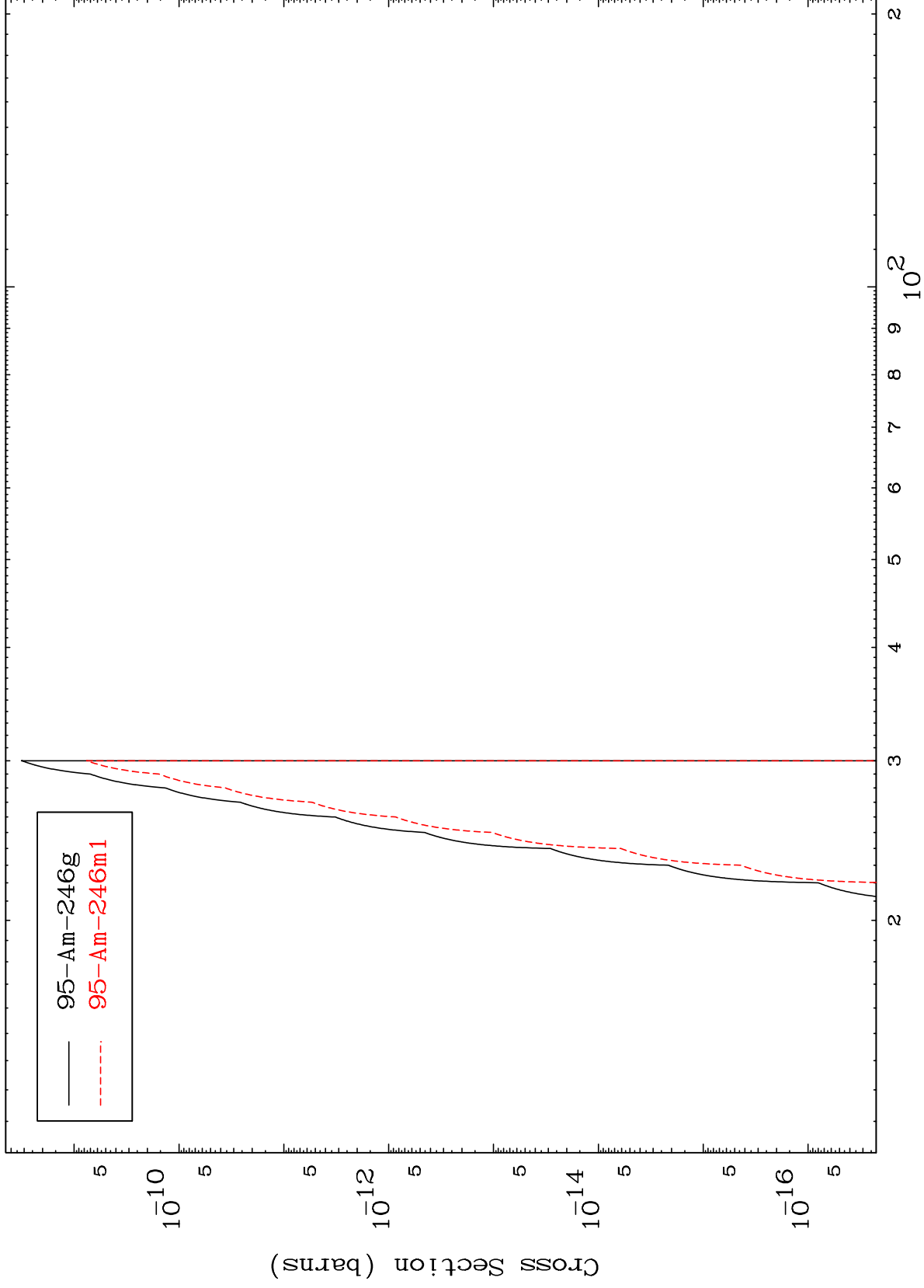
30

MAT 9749

(n,2n) p

97-Bk-248

Radionuclide Production Cross Section



31

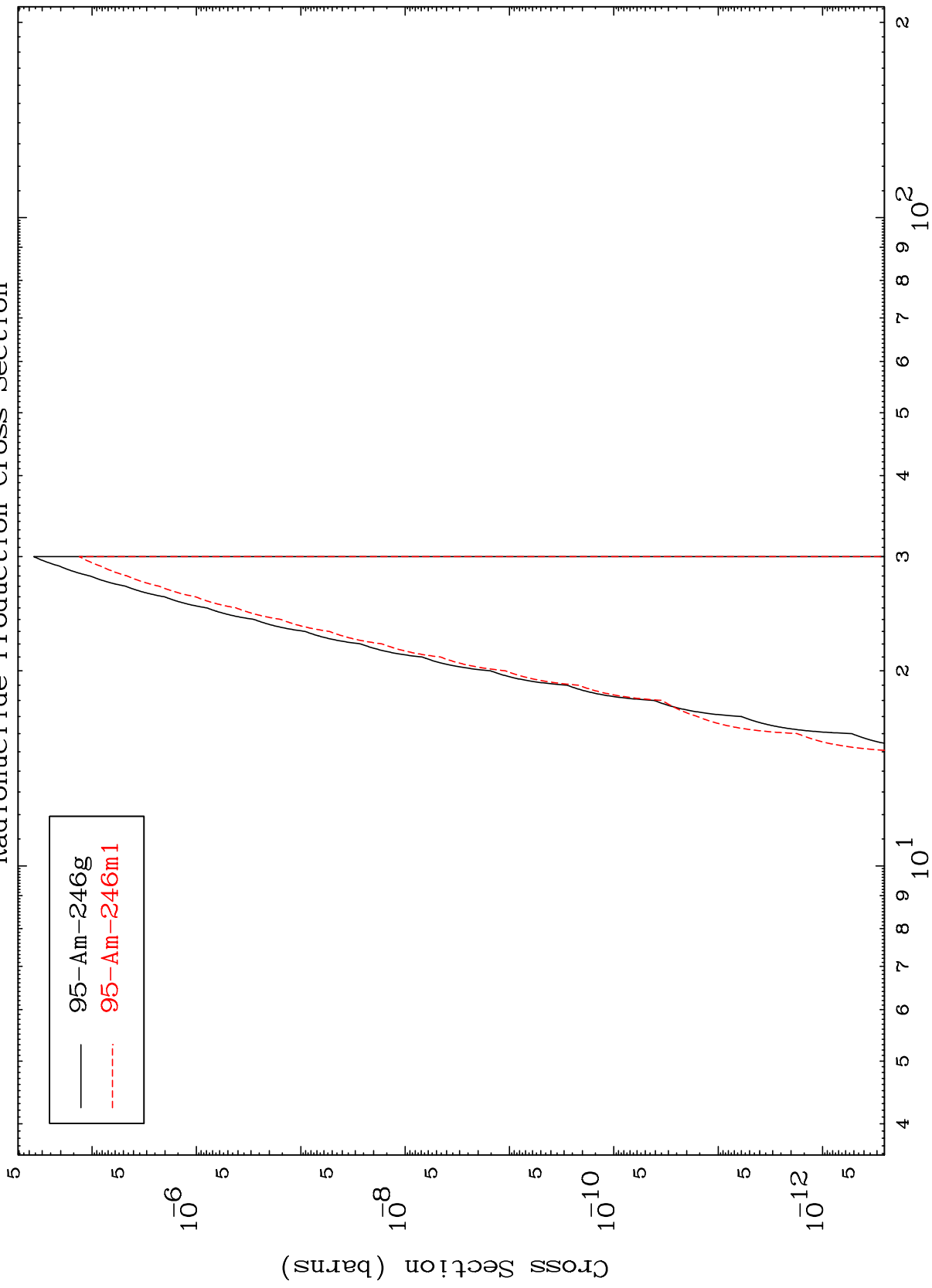
Incident Energy (MeV)

97-Bk-248

MAT 9749

97-Bk-248

Radionuclide Production Cross Section



Incident Energy (MeV)

97-Bk-248

32