

Program EVALPLOT
(Version 2015-2)

by

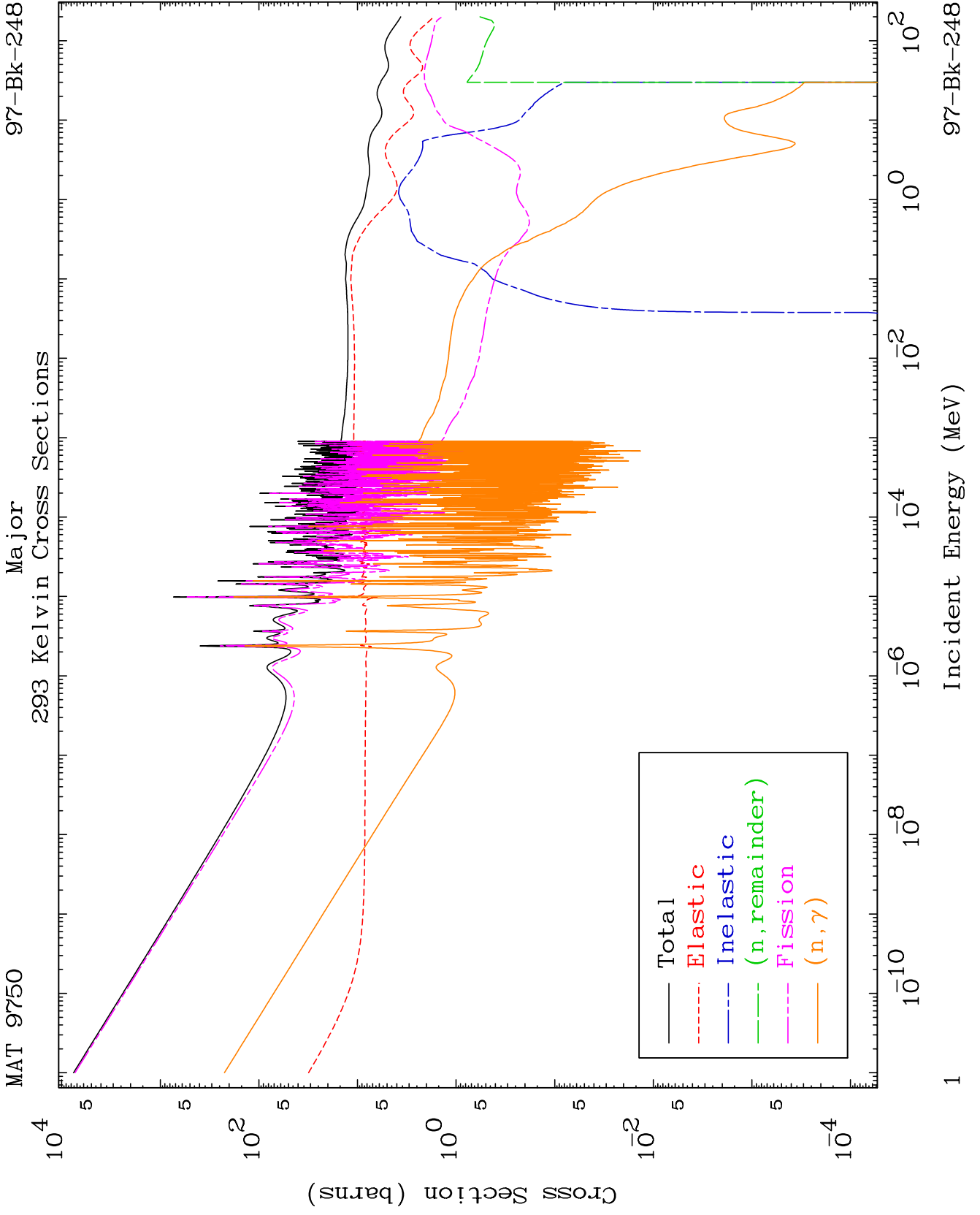
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(Present Contact Information)

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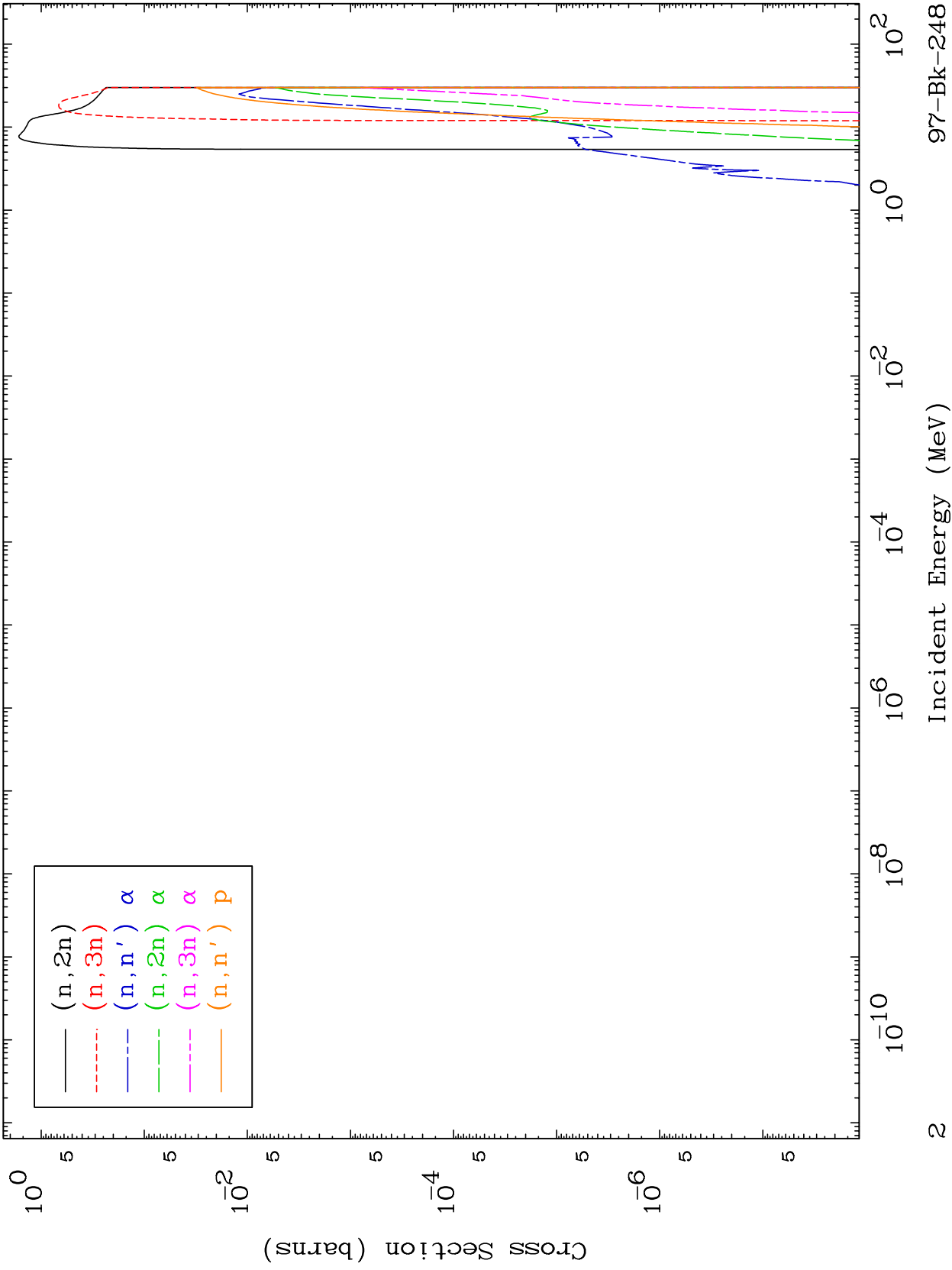
Press Mouse Button to Start



MAT 9750

Neutron Production
293 Kelvin Cross Sections

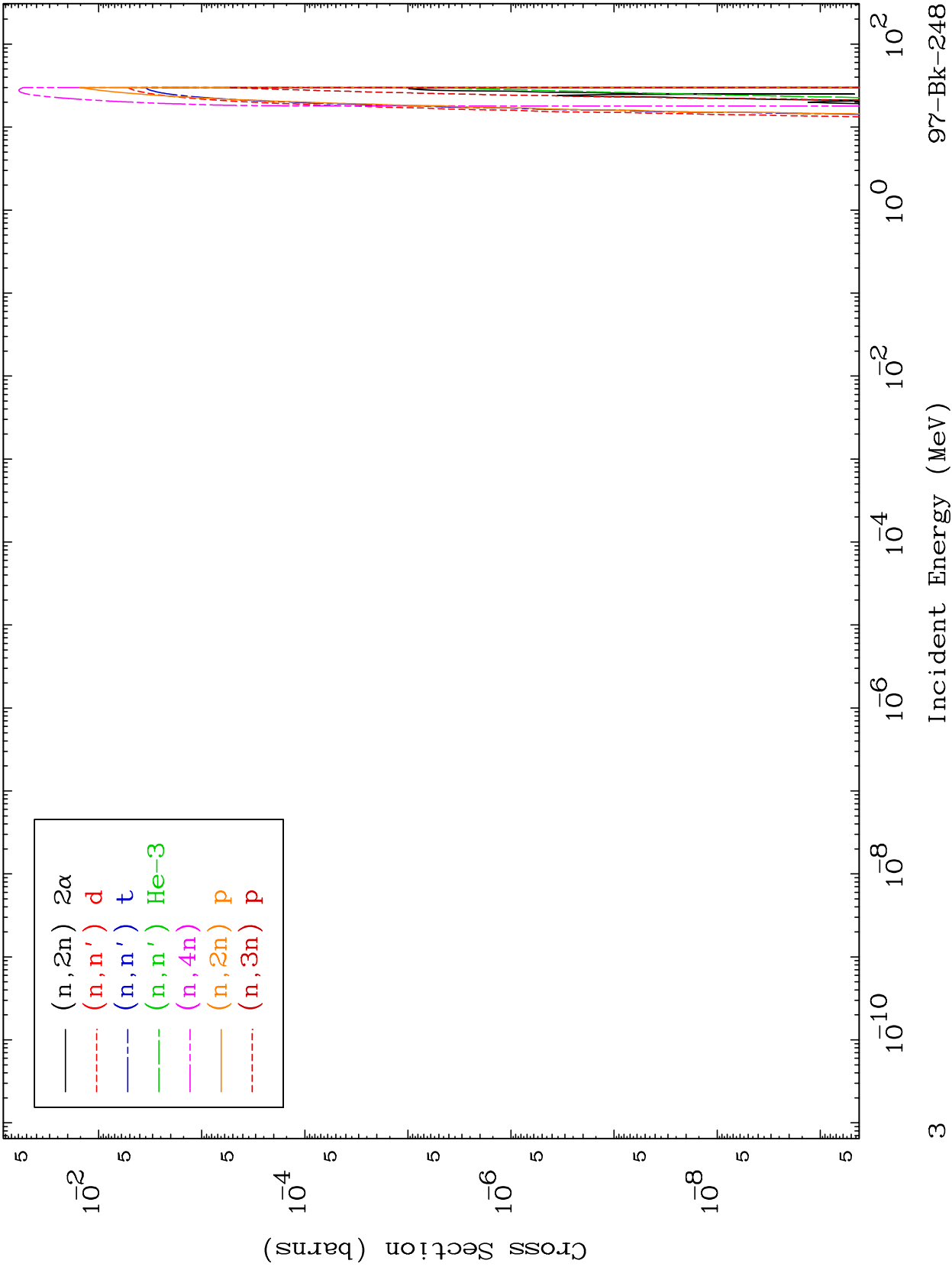
97-Bk-248



MAT 9750

Neutron Production
293 Kelvin Cross Sections

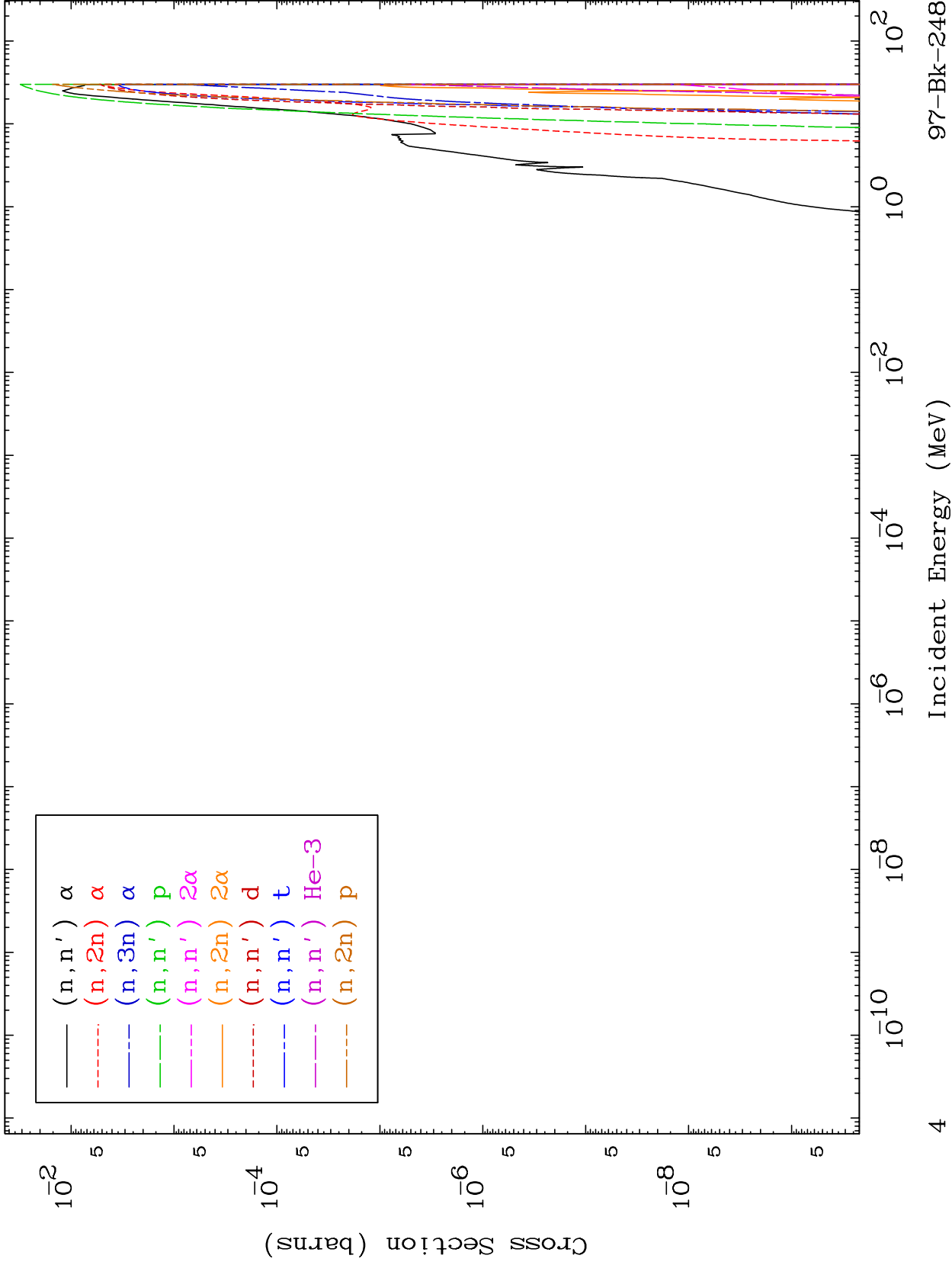
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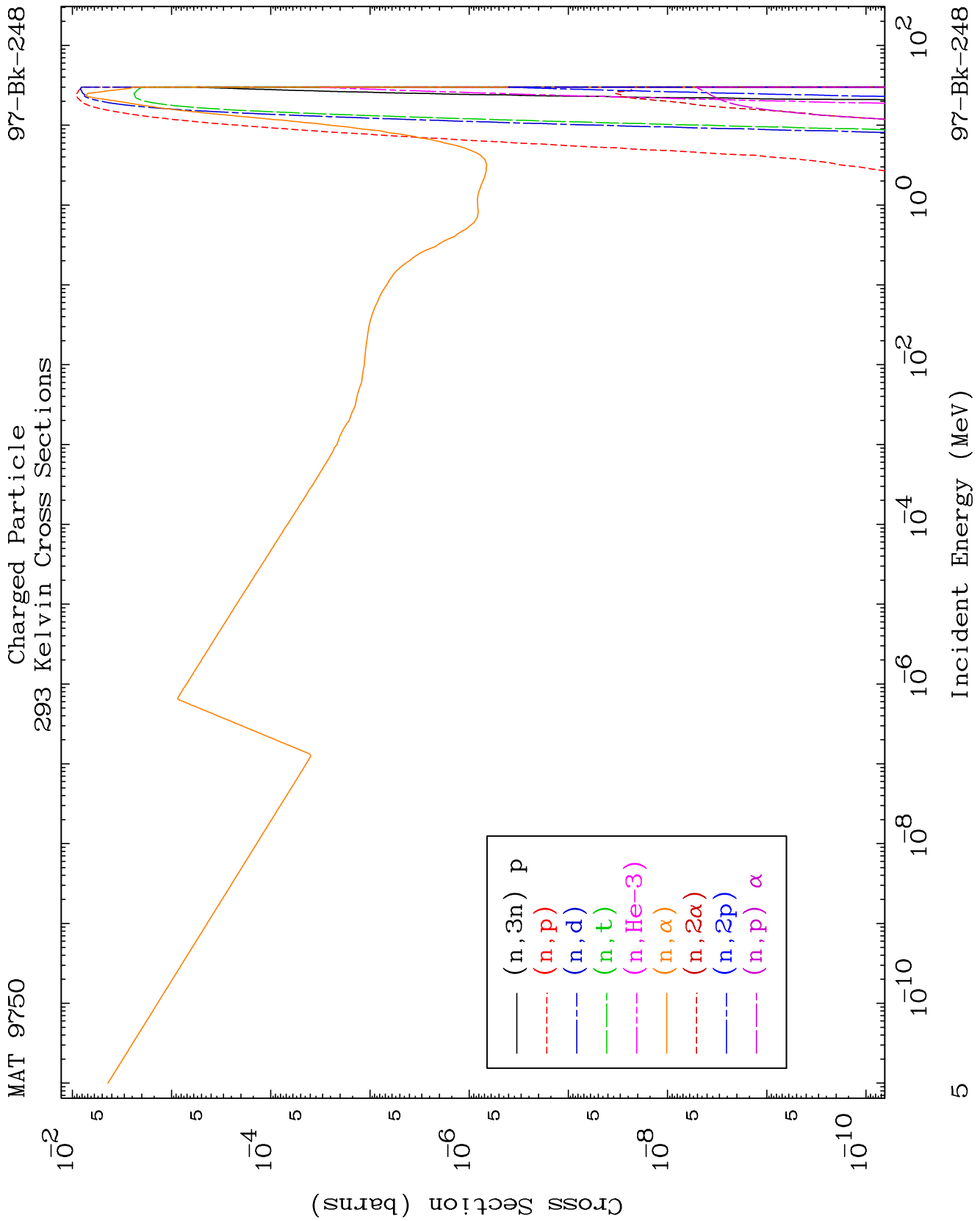


MAT 9750

Charged Particle
293 Kelvin Cross Sections

97-Bk-248

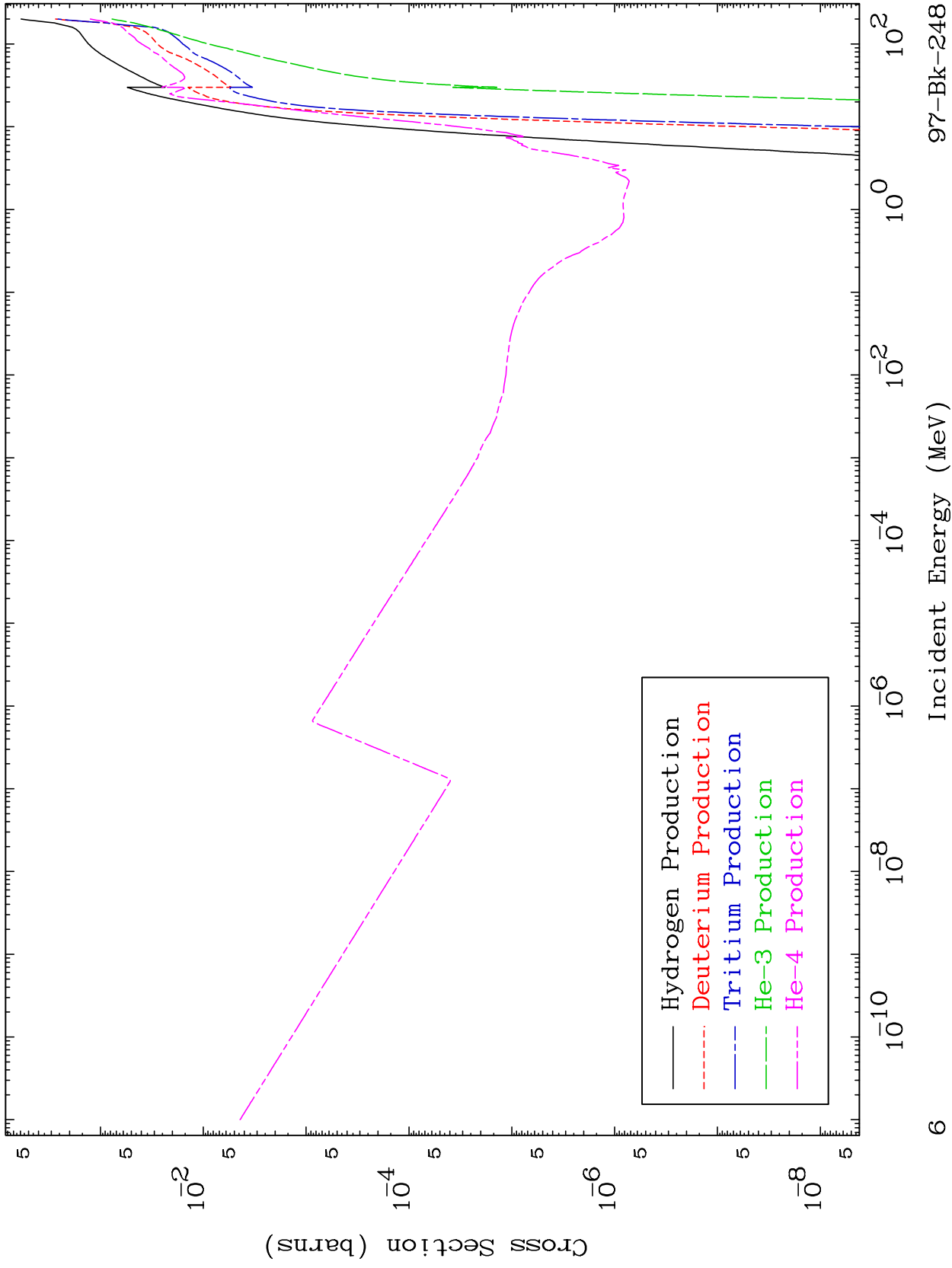


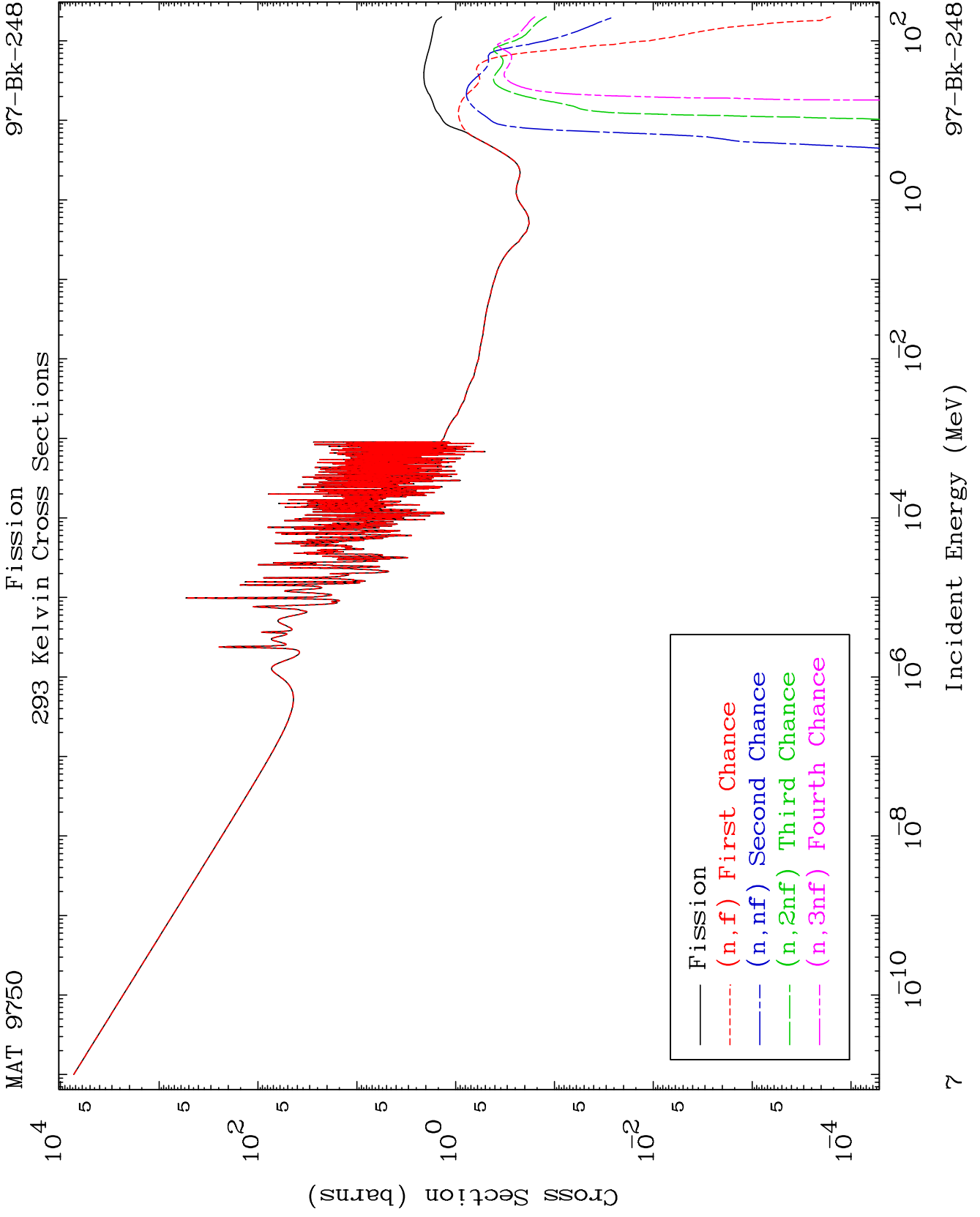


MAT 9750

Particle Production
293 Kelvin Cross Sections

97-Bk-248

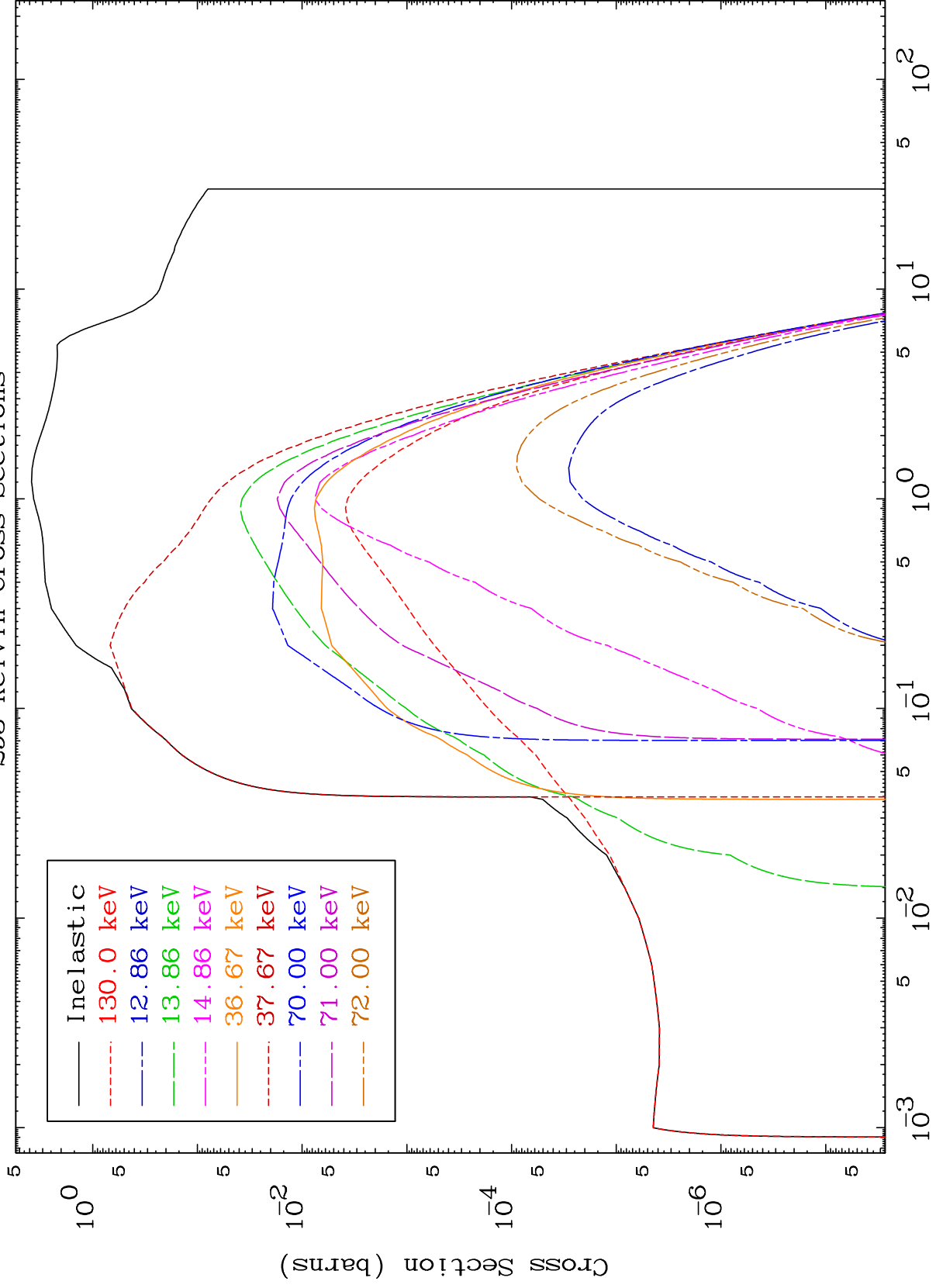




MAT 9750

(n,n') Level
293 Kelvin Cross Sections

97-Bk-248



8

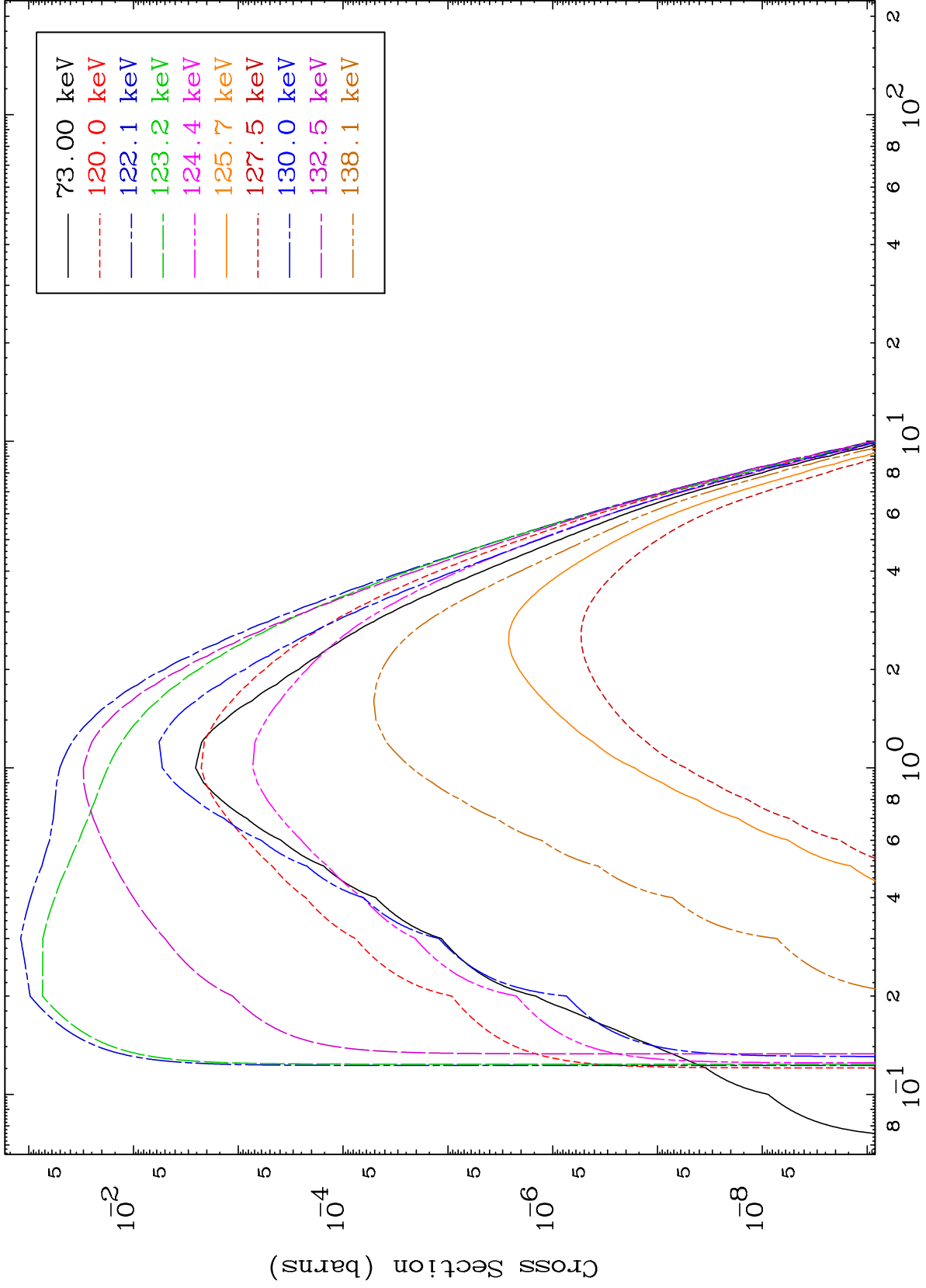
Incident Energy (MeV)

97-Bk-248

MAT 9750

(n,n') Level
293 Kelvin Cross Sections

97-Bk-248



9

Incident Energy (MeV)

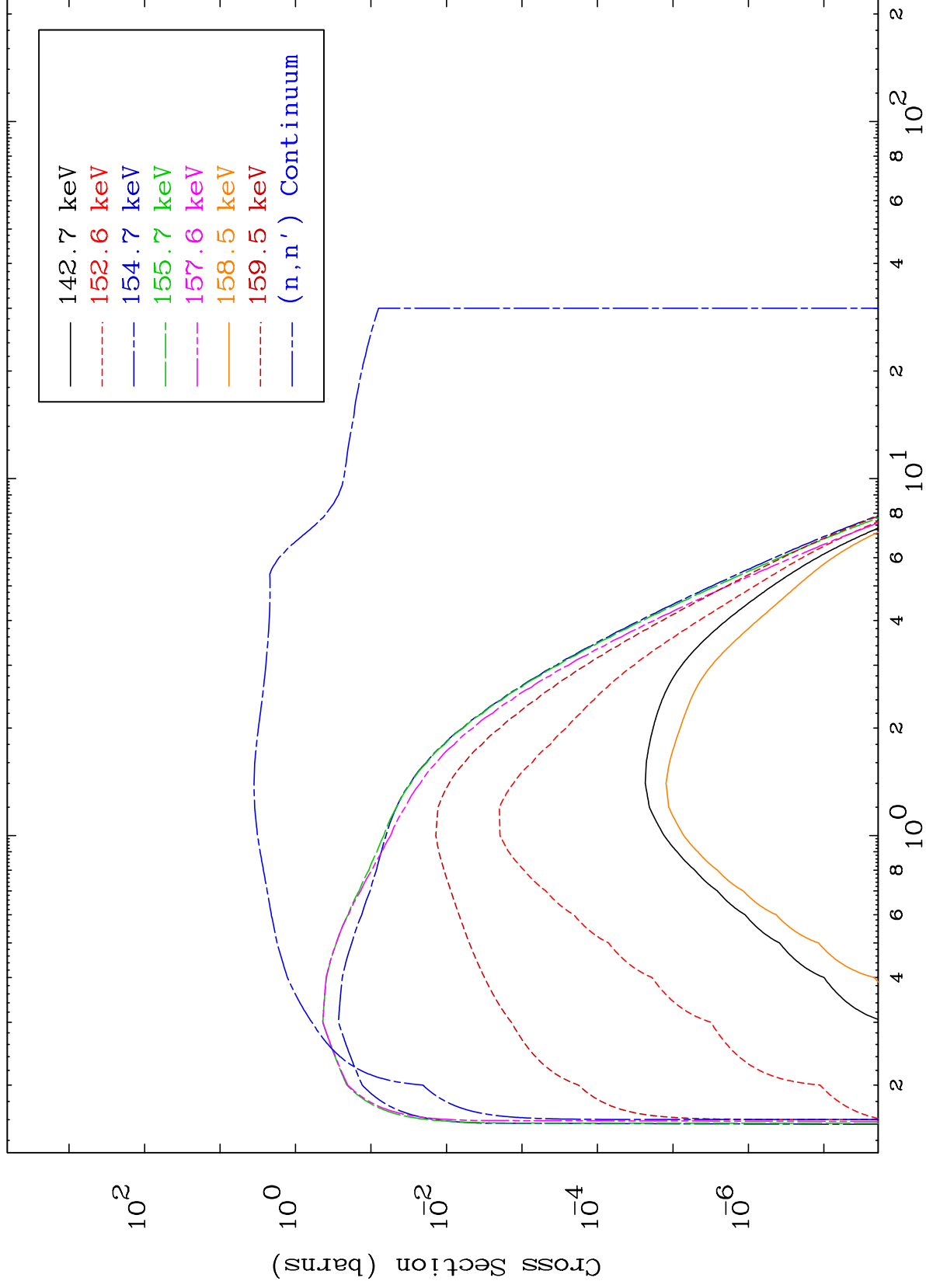
97-Bk-248

MAT 9750

(n,n') Level

97-Bk-248

293 Kelvin Cross Sections



10

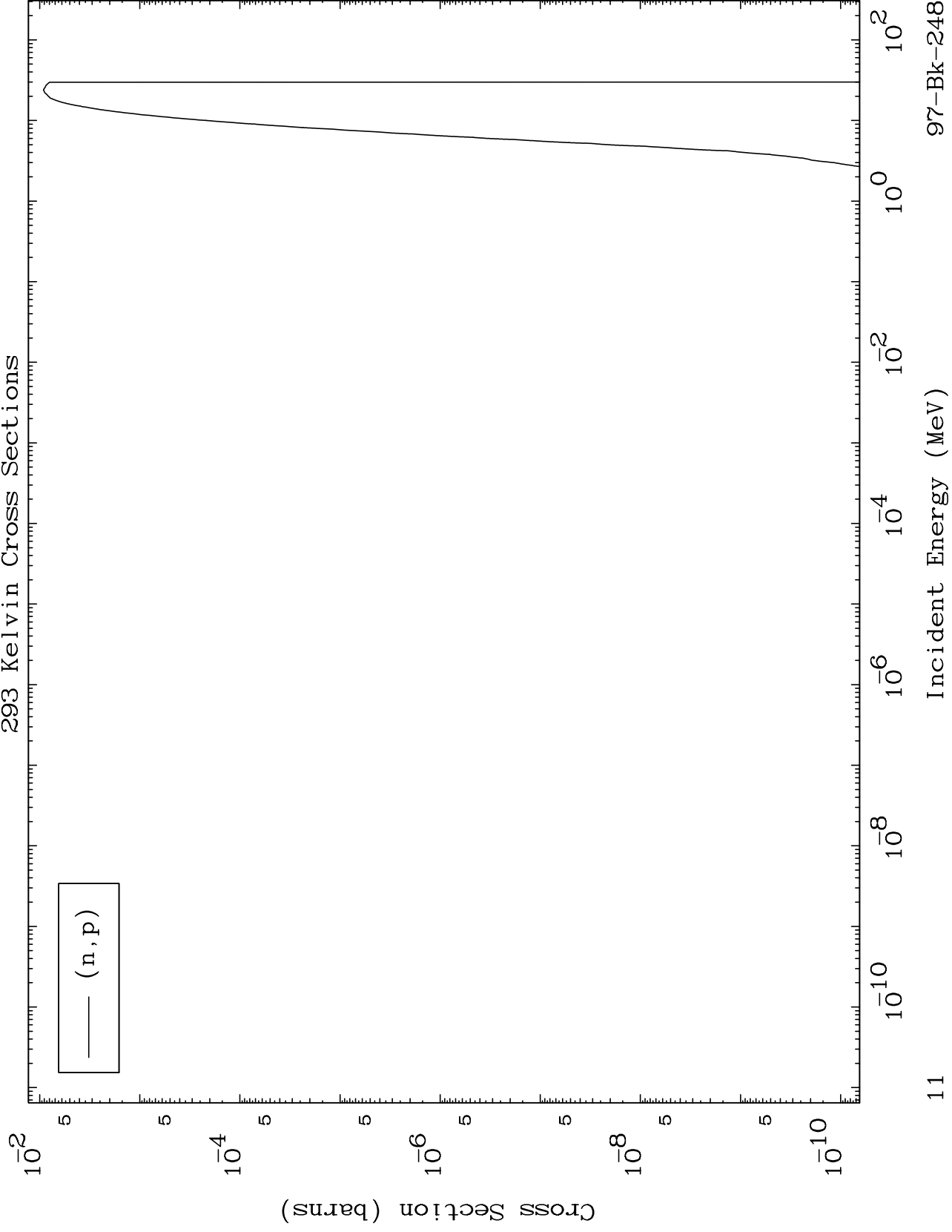
Incident Energy (MeV)

97-Bk-248

MAT 9750

(n,p) Levels
293 Kelvin Cross Sections

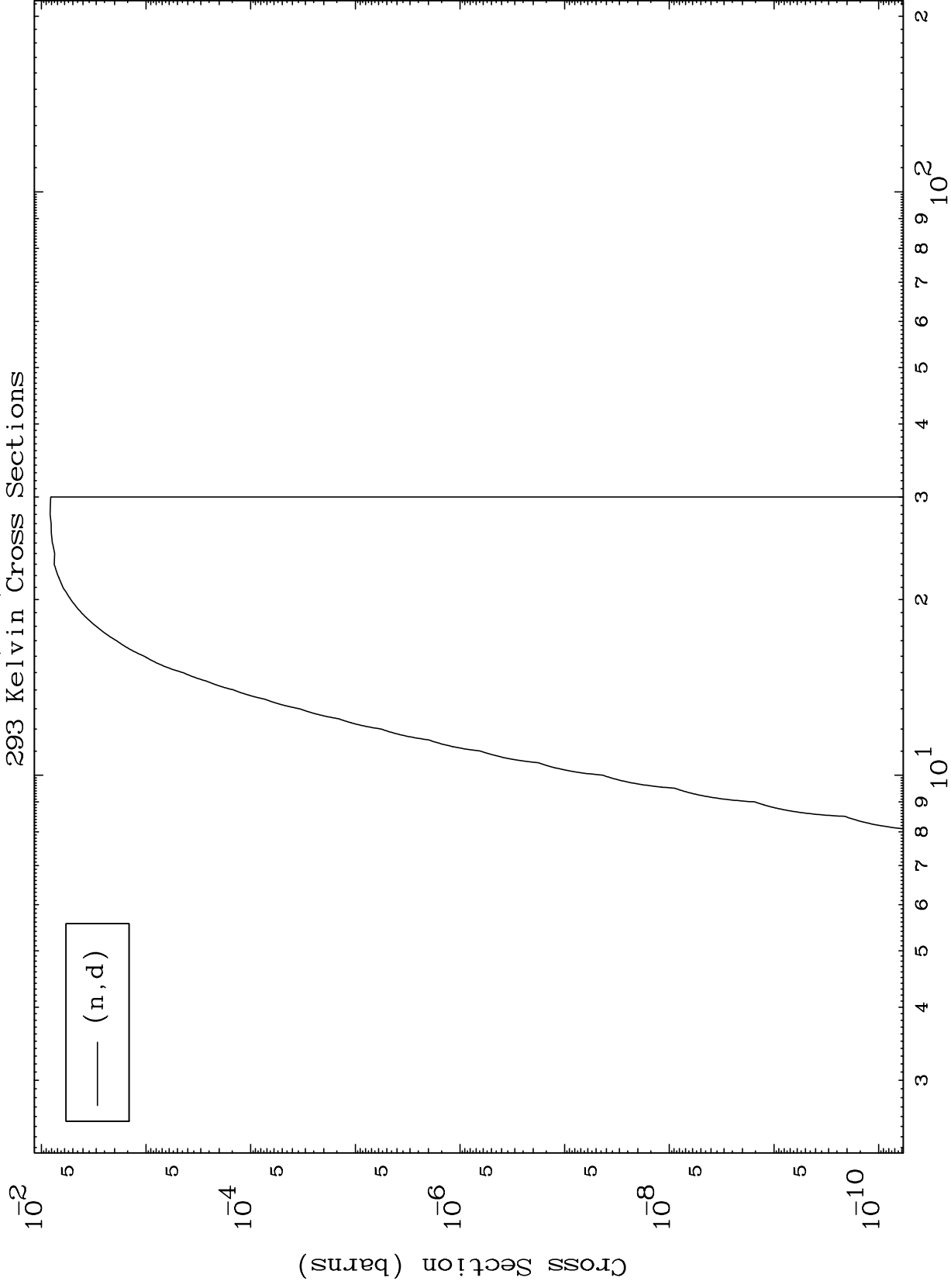
97-Bk-248



MAT 9750

(n,d) Levels
293 Kelvin Cross Sections

97-Bk-248



12

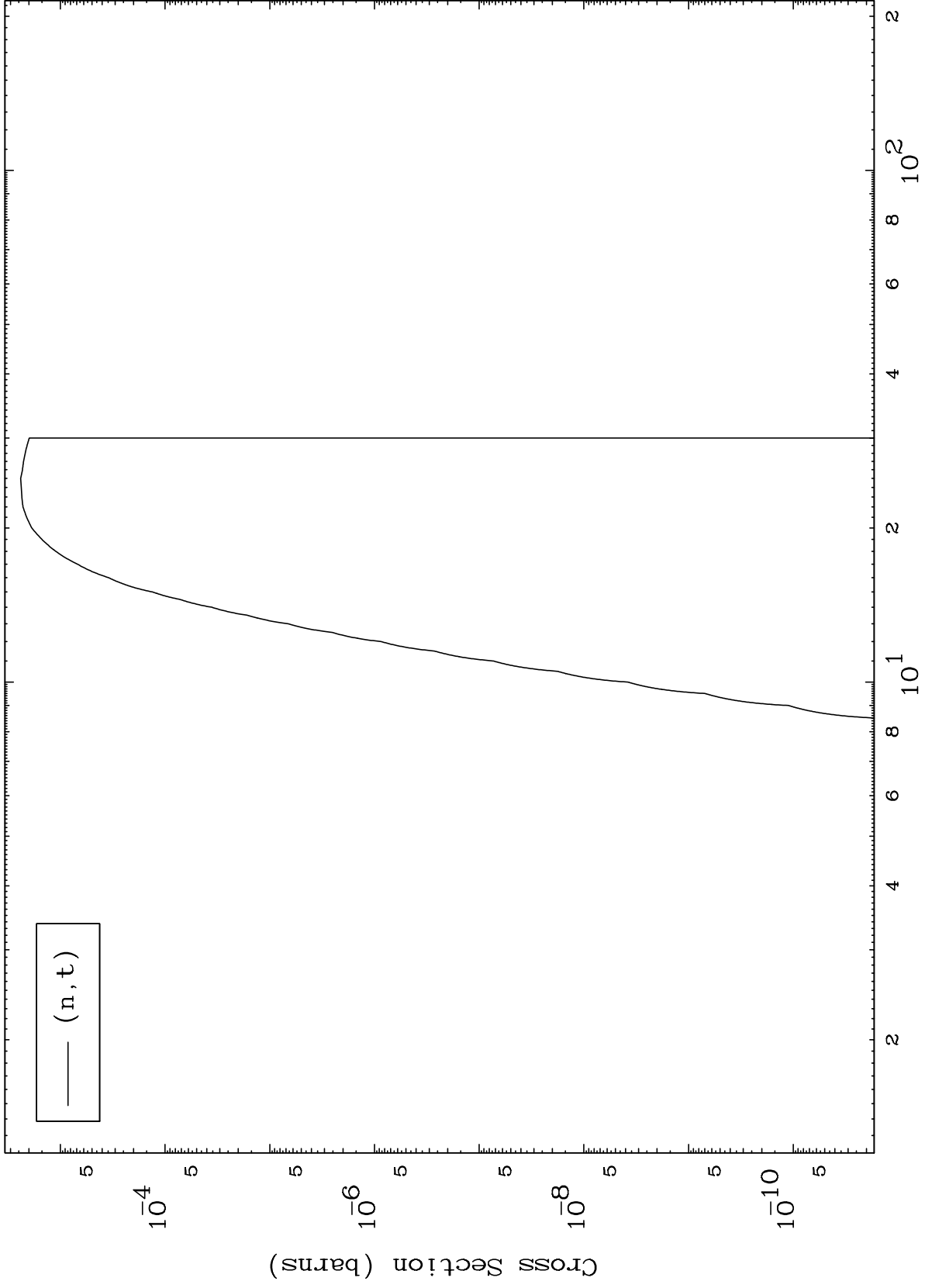
Incident Energy (MeV)

97-Bk-248

MAT 9750

(n,t) Levels
293 Kelvin Cross Sections

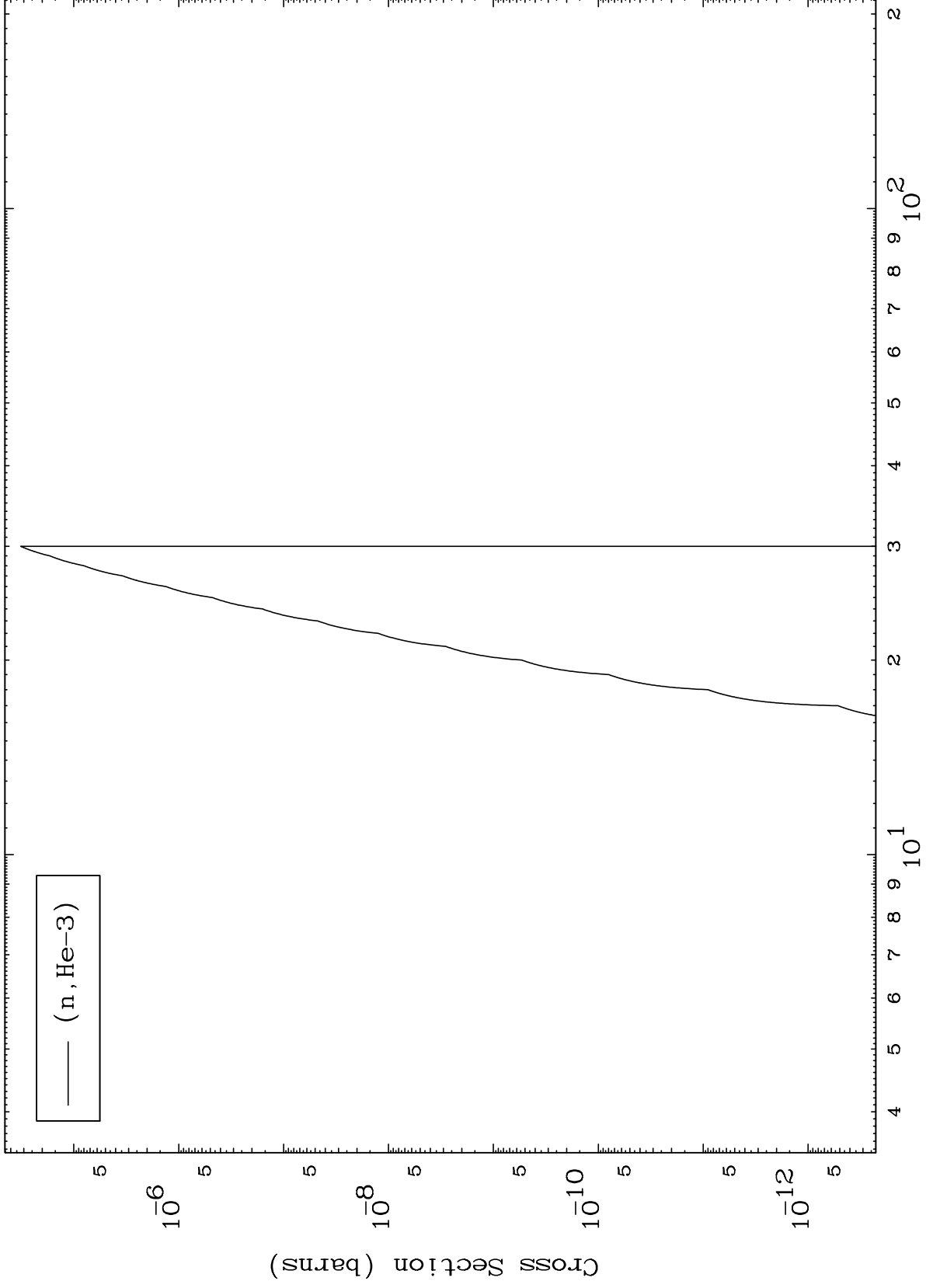
97-Bk-248



MAT 9750

(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-248



14

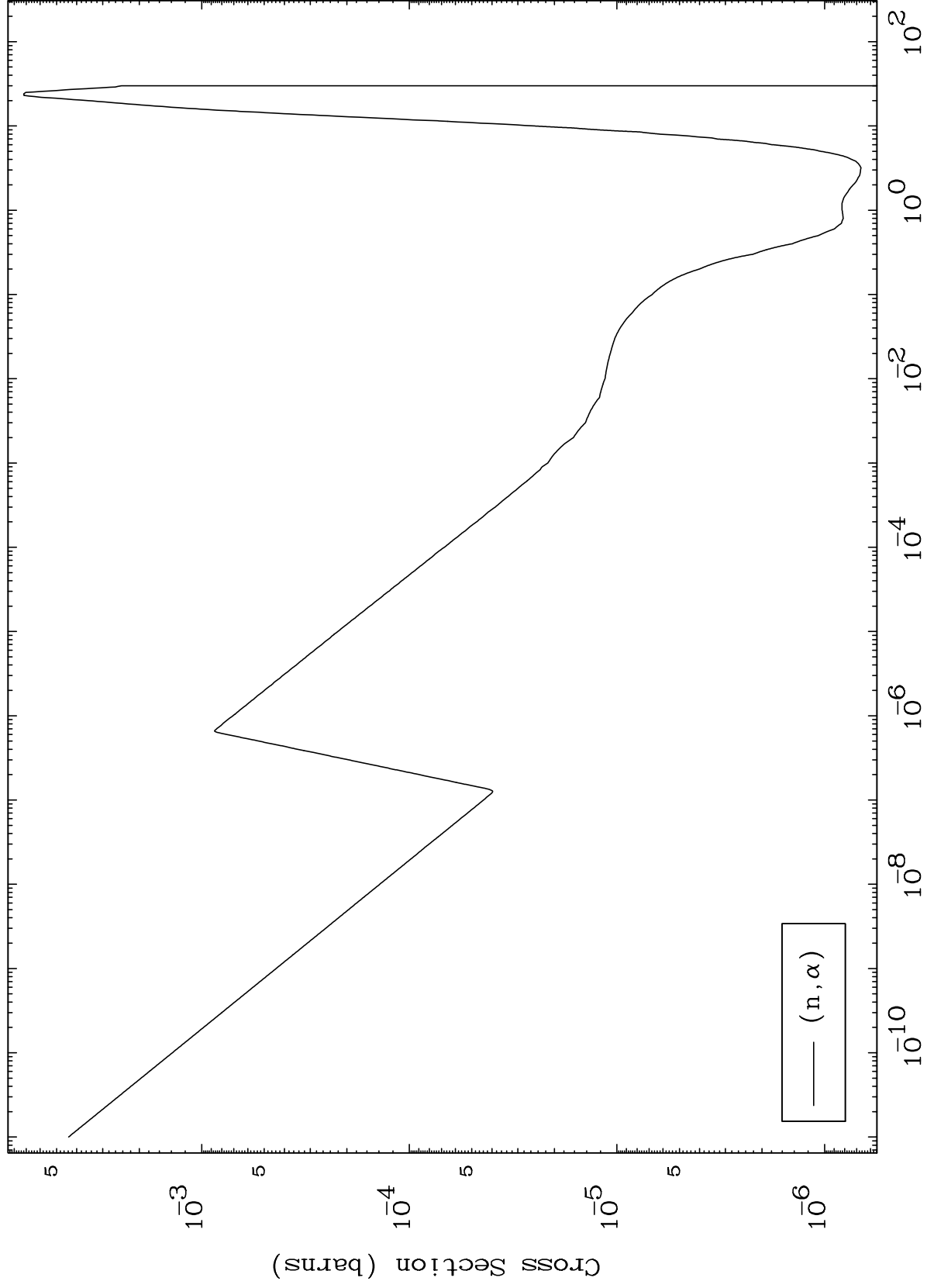
Incident Energy (MeV)

97-Bk-248

MAT 9750

(n,α) Levels
293 Kelvin Cross Sections

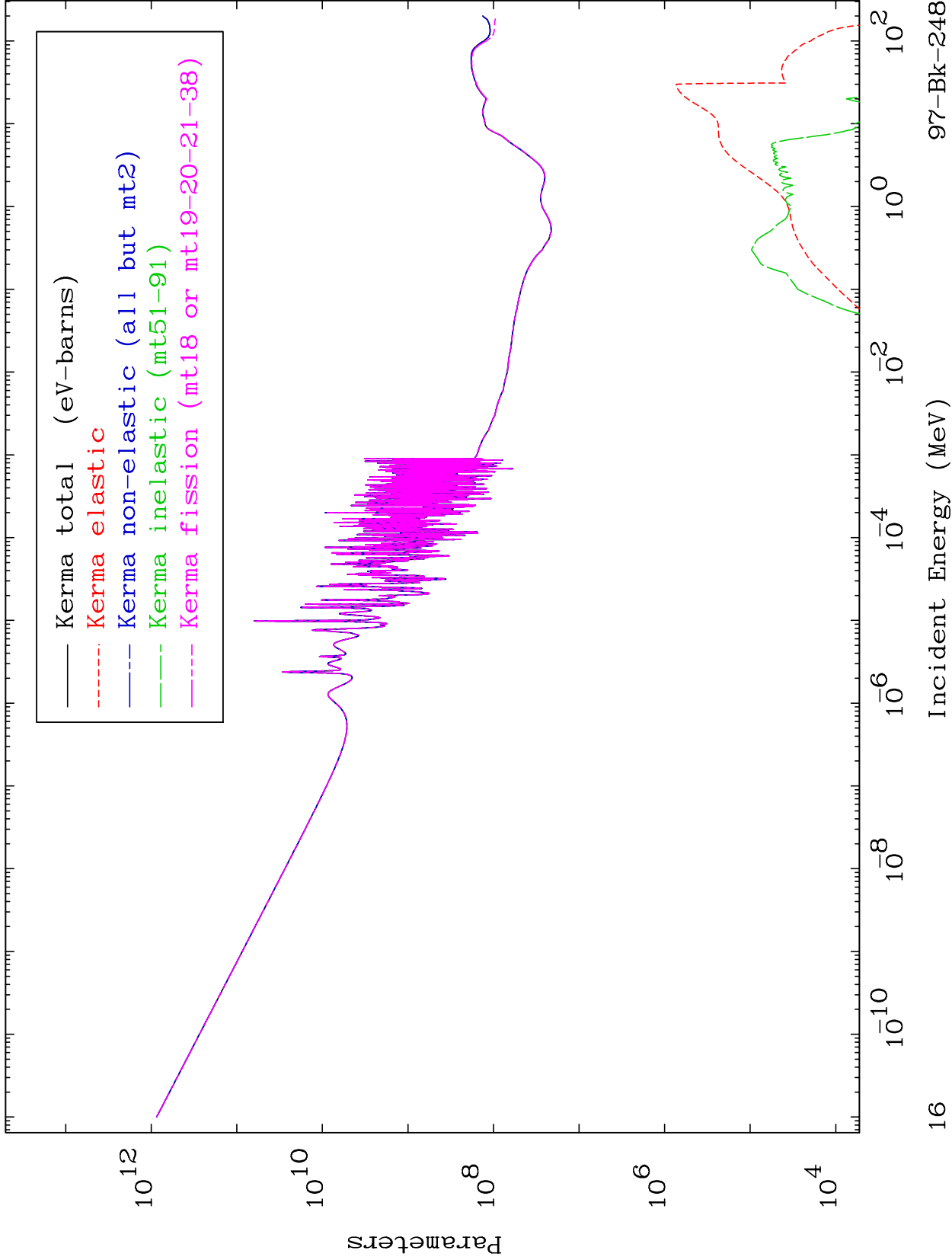
97-Bk-248

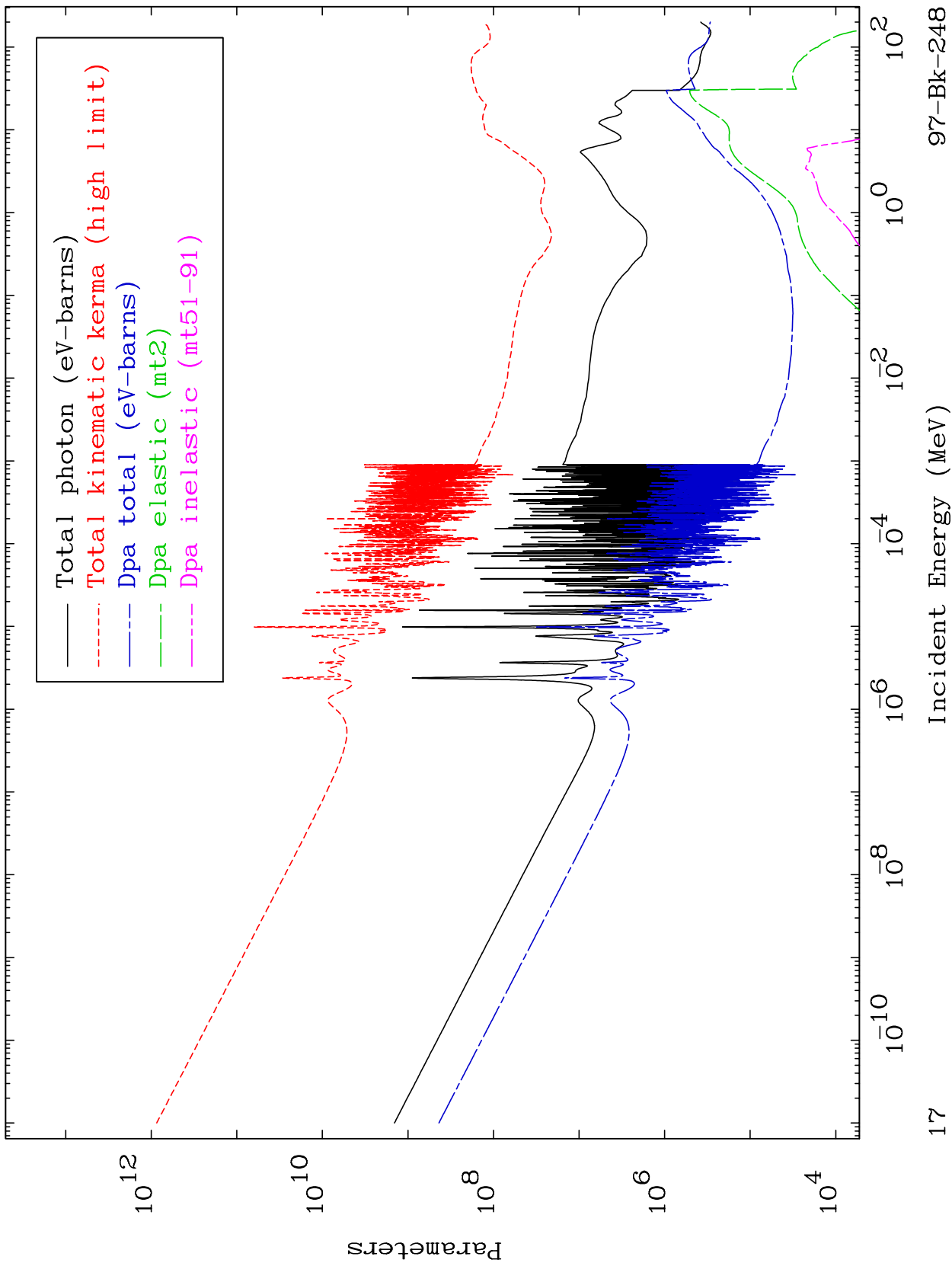


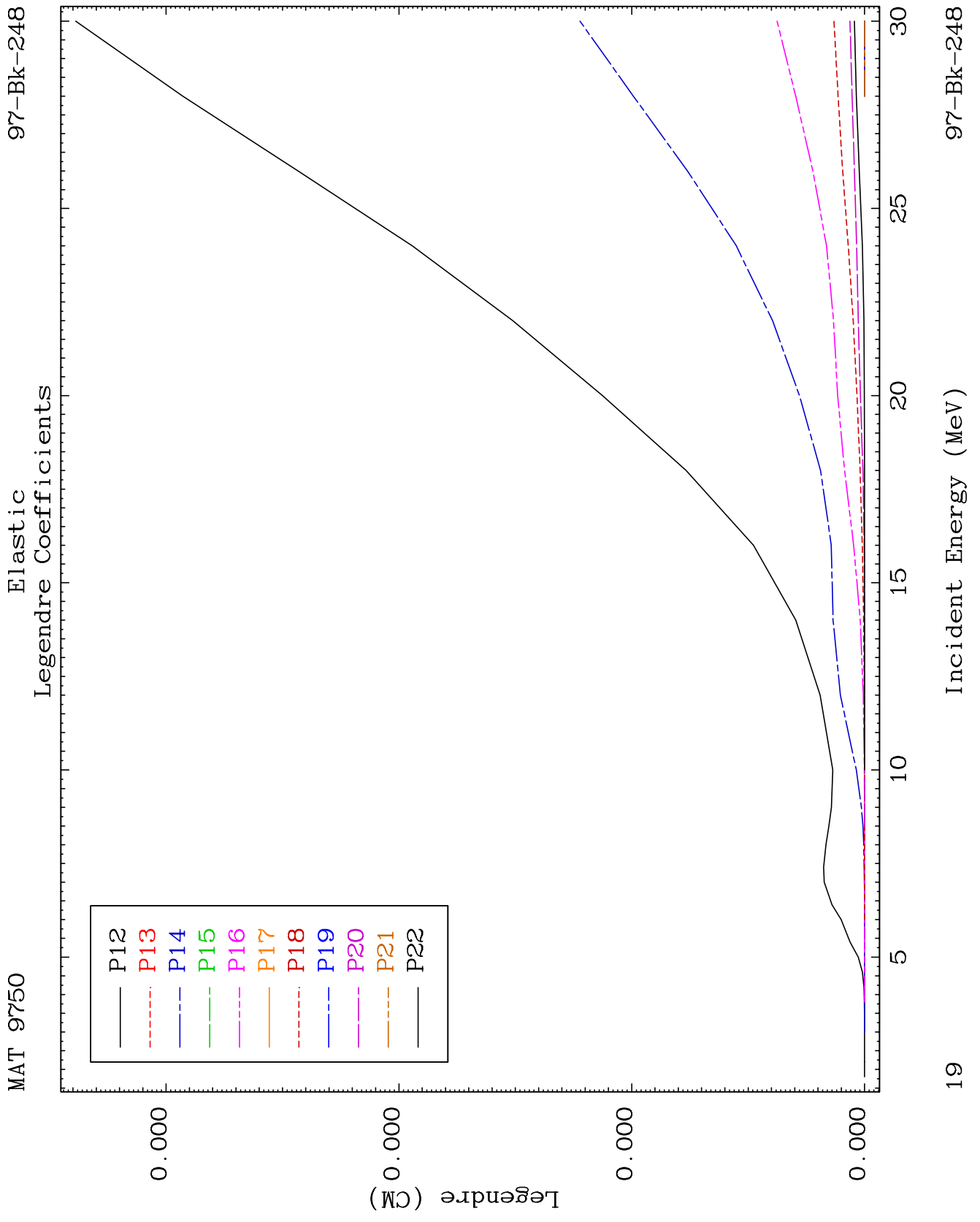
15

Incident Energy (MeV)

97-Bk-248

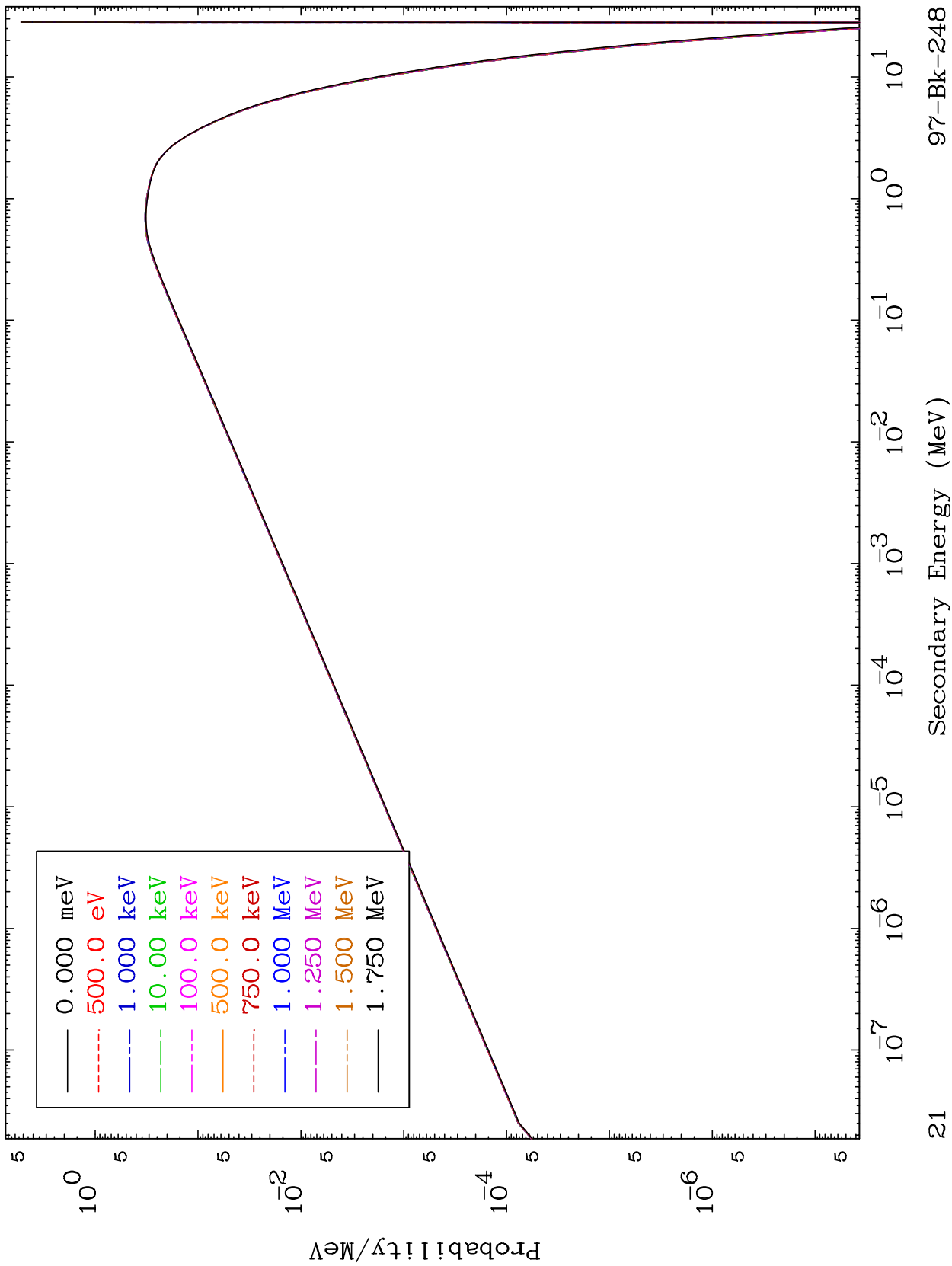




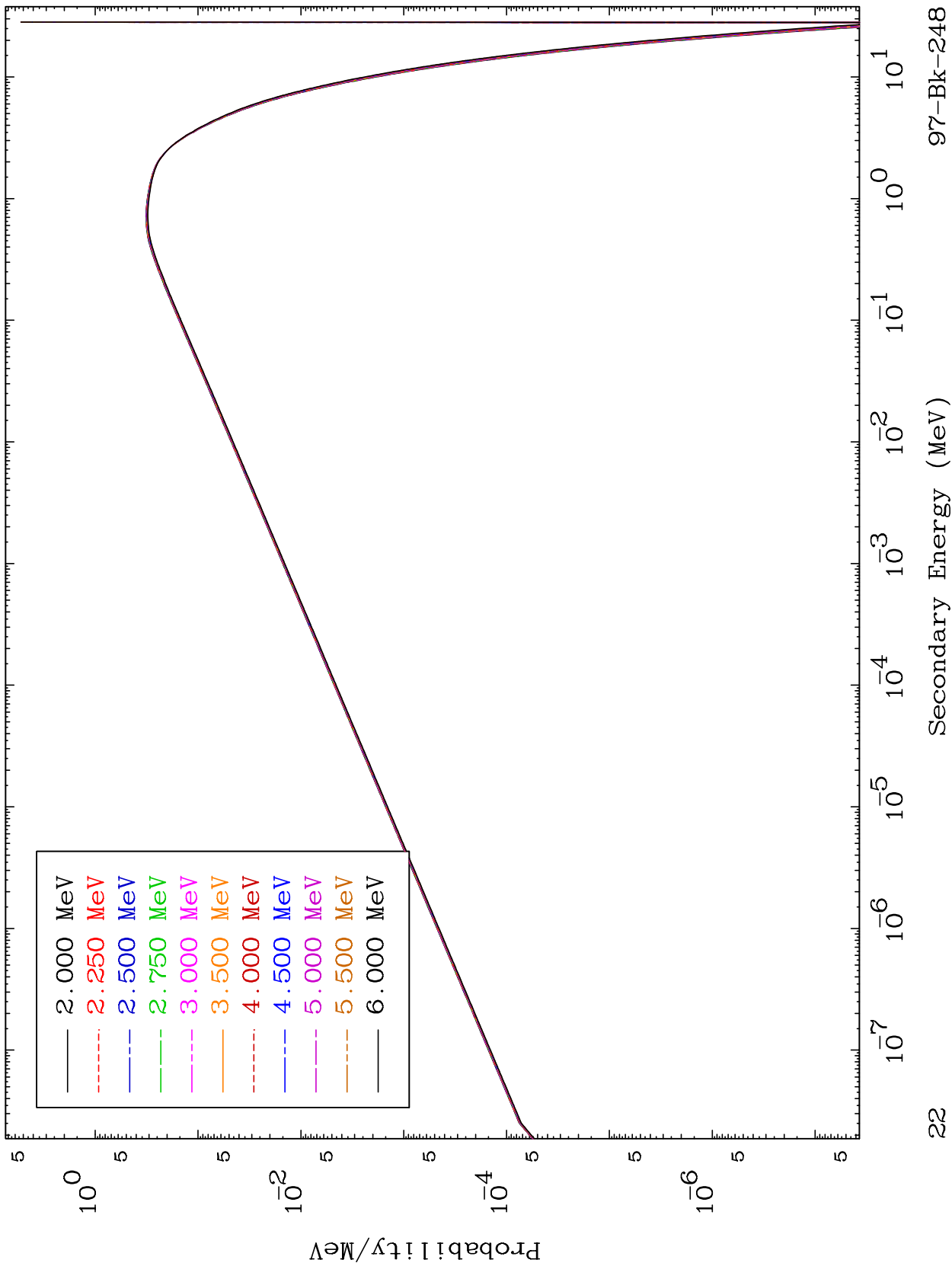


MAT 9750

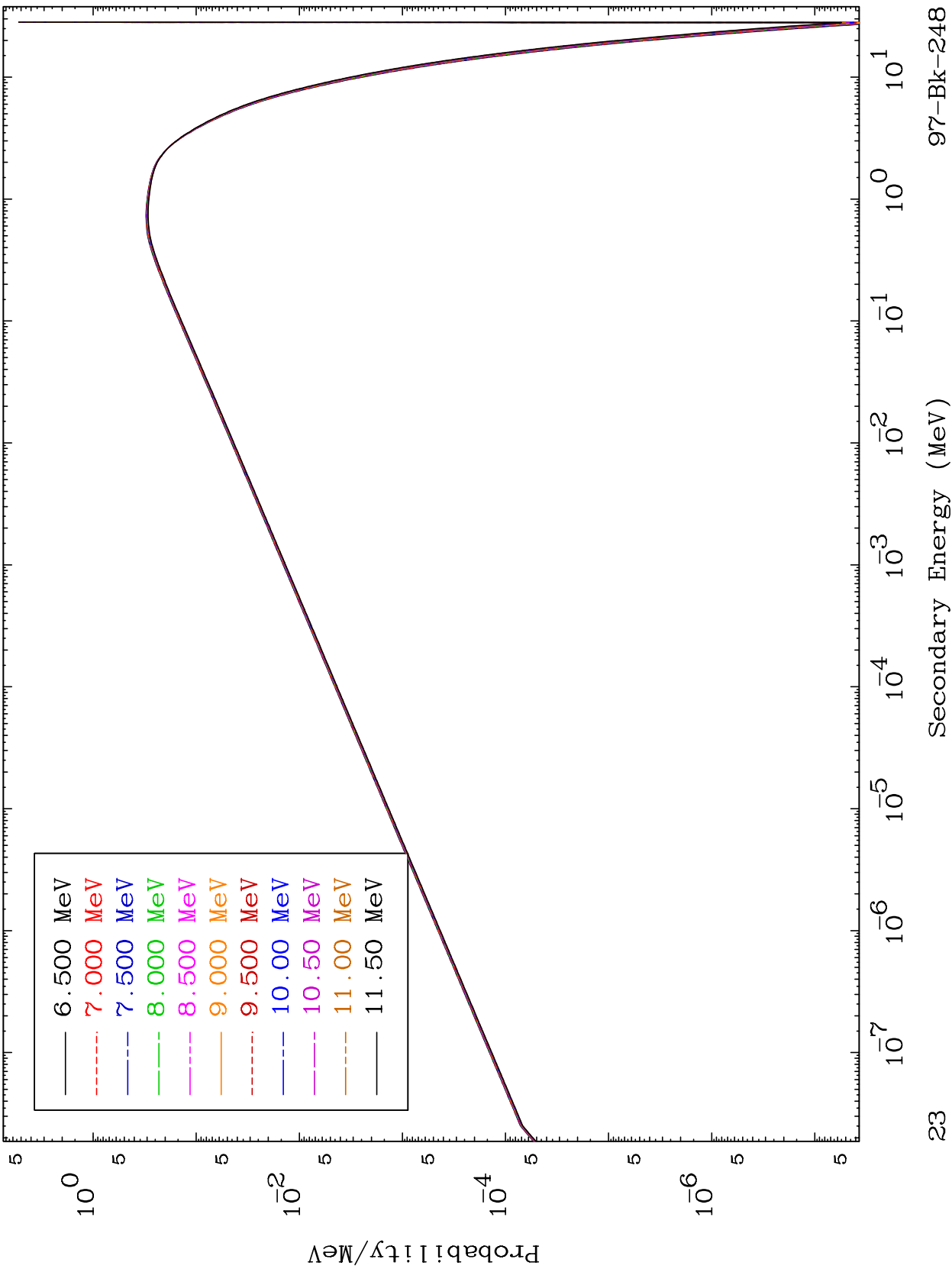
Fission Energy Distribution



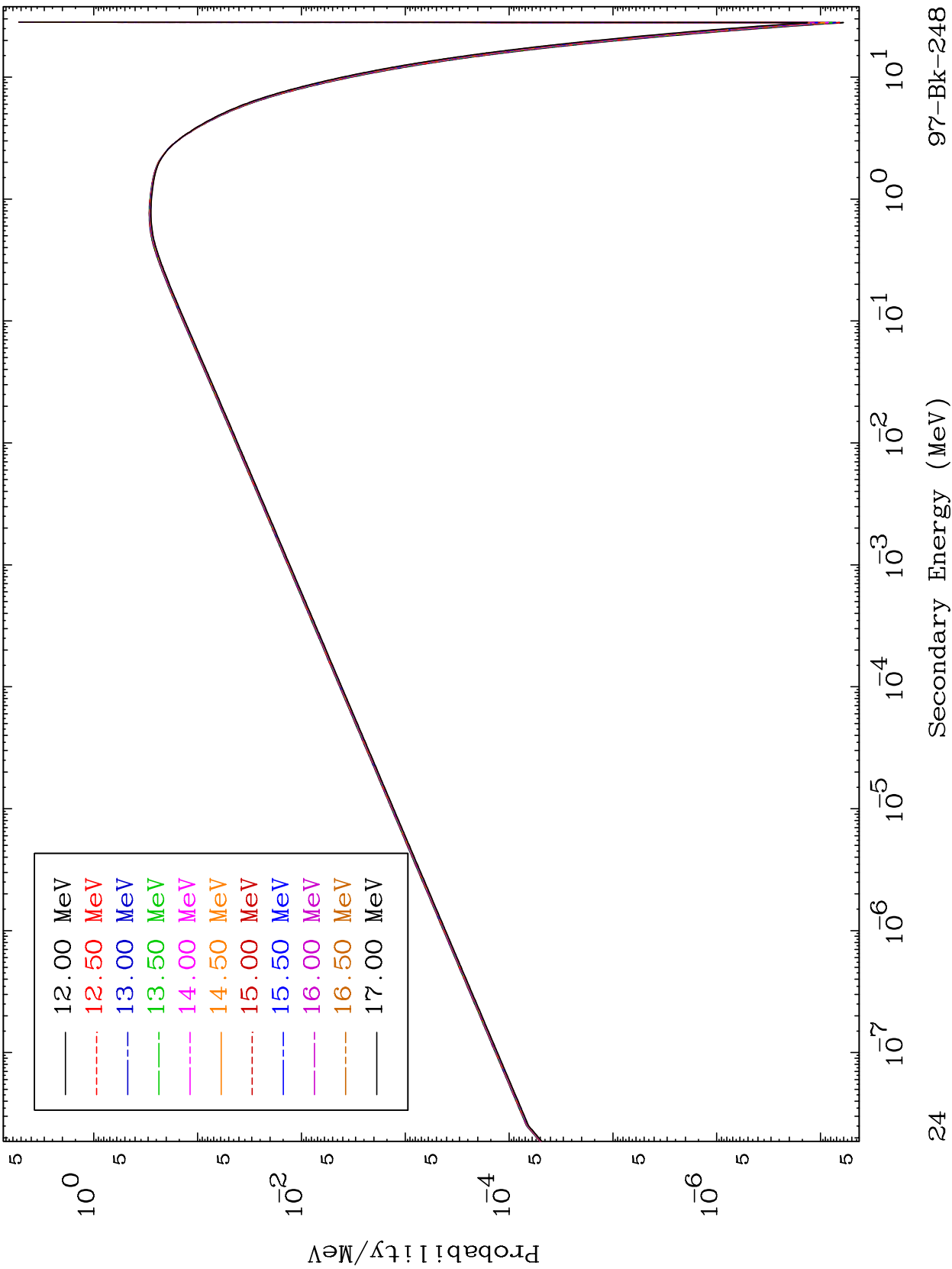
Fission Energy Distribution



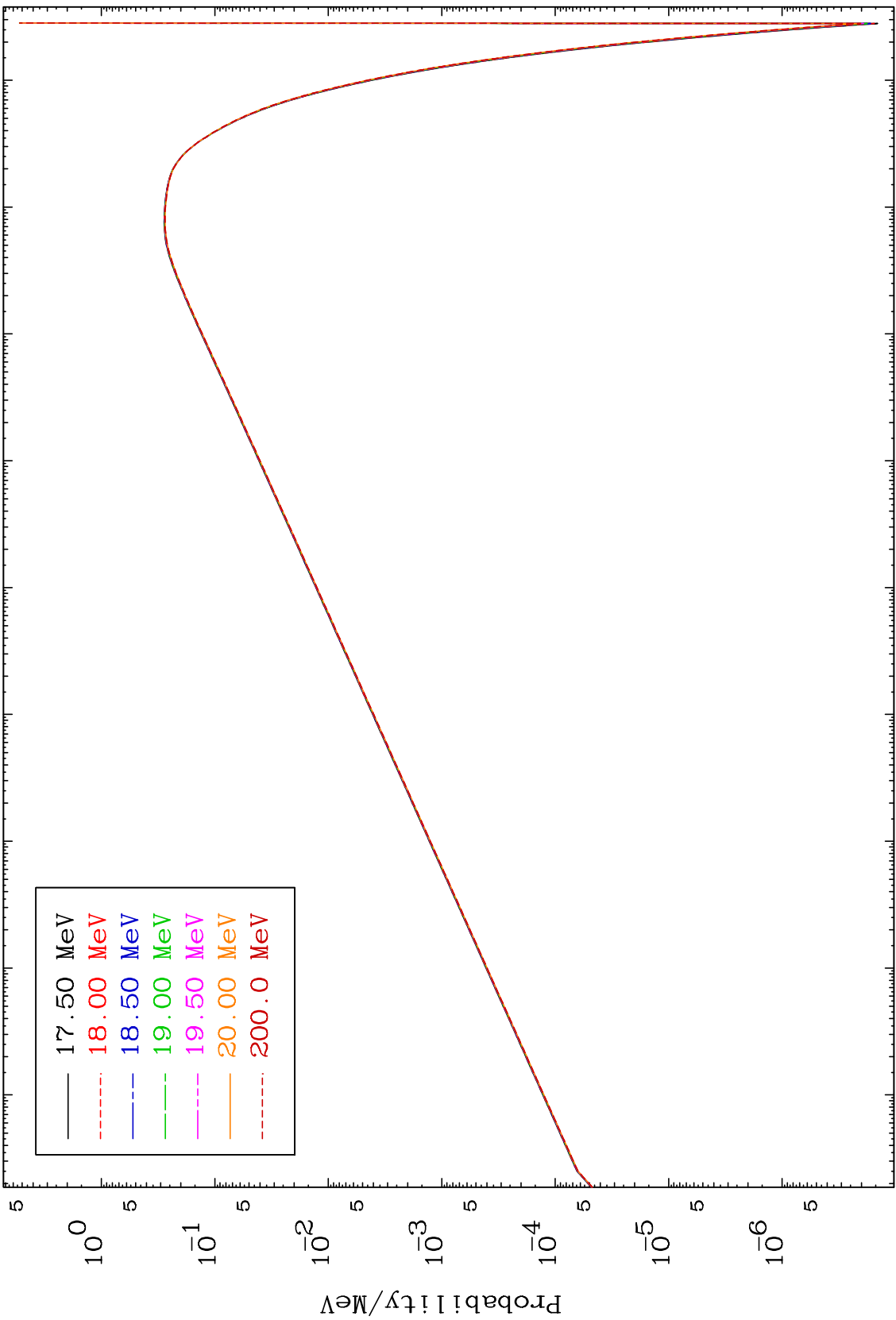
Fission Energy Distribution



Fission Energy Distribution



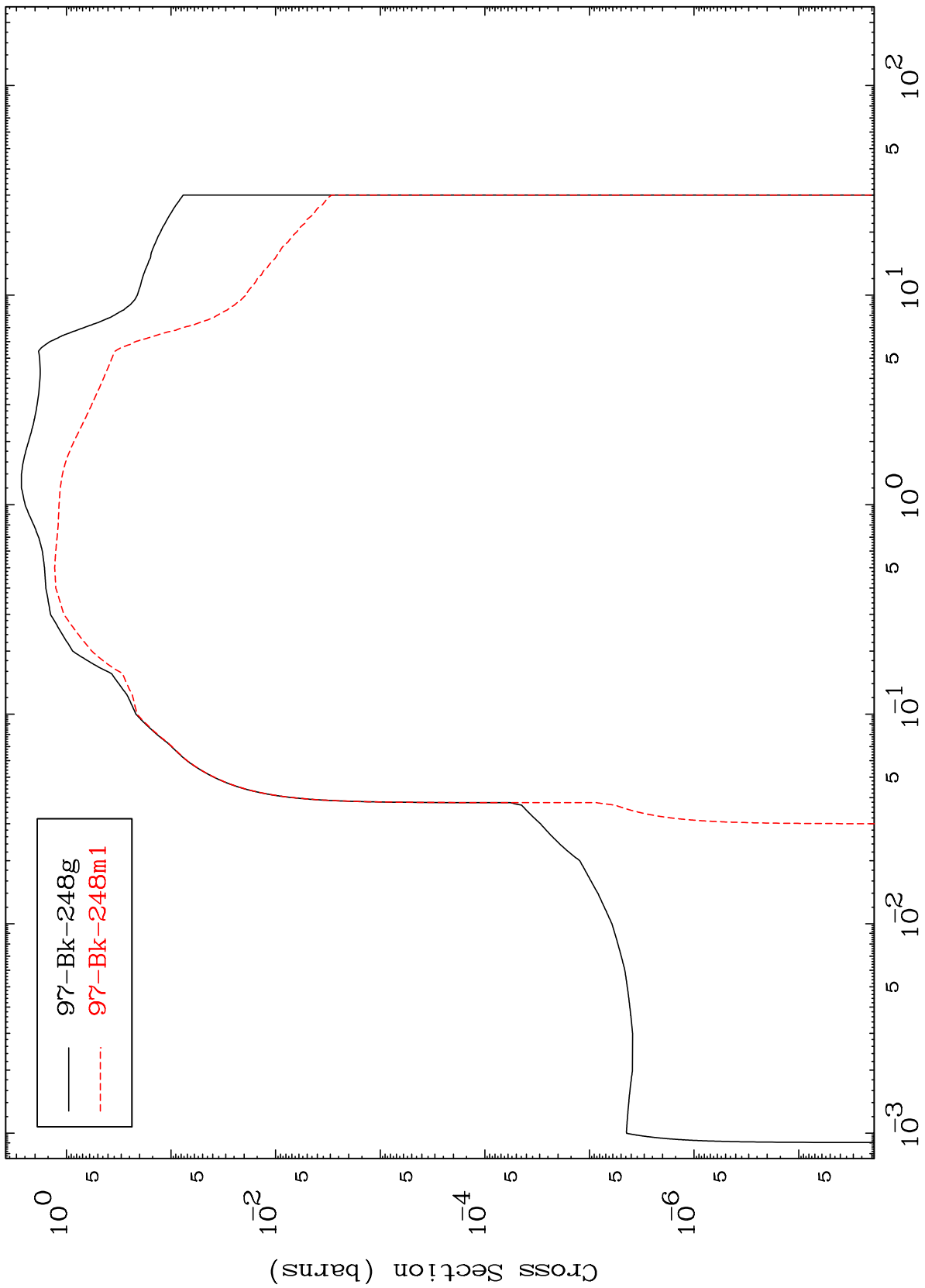
Fission Energy Distribution



MAT 9750

97-Bk-248

Radionuclide Production Cross Section



— 97-Bk-248g
- - - 97-Bk-248m1

97-Bk-248

Incident Energy (MeV)

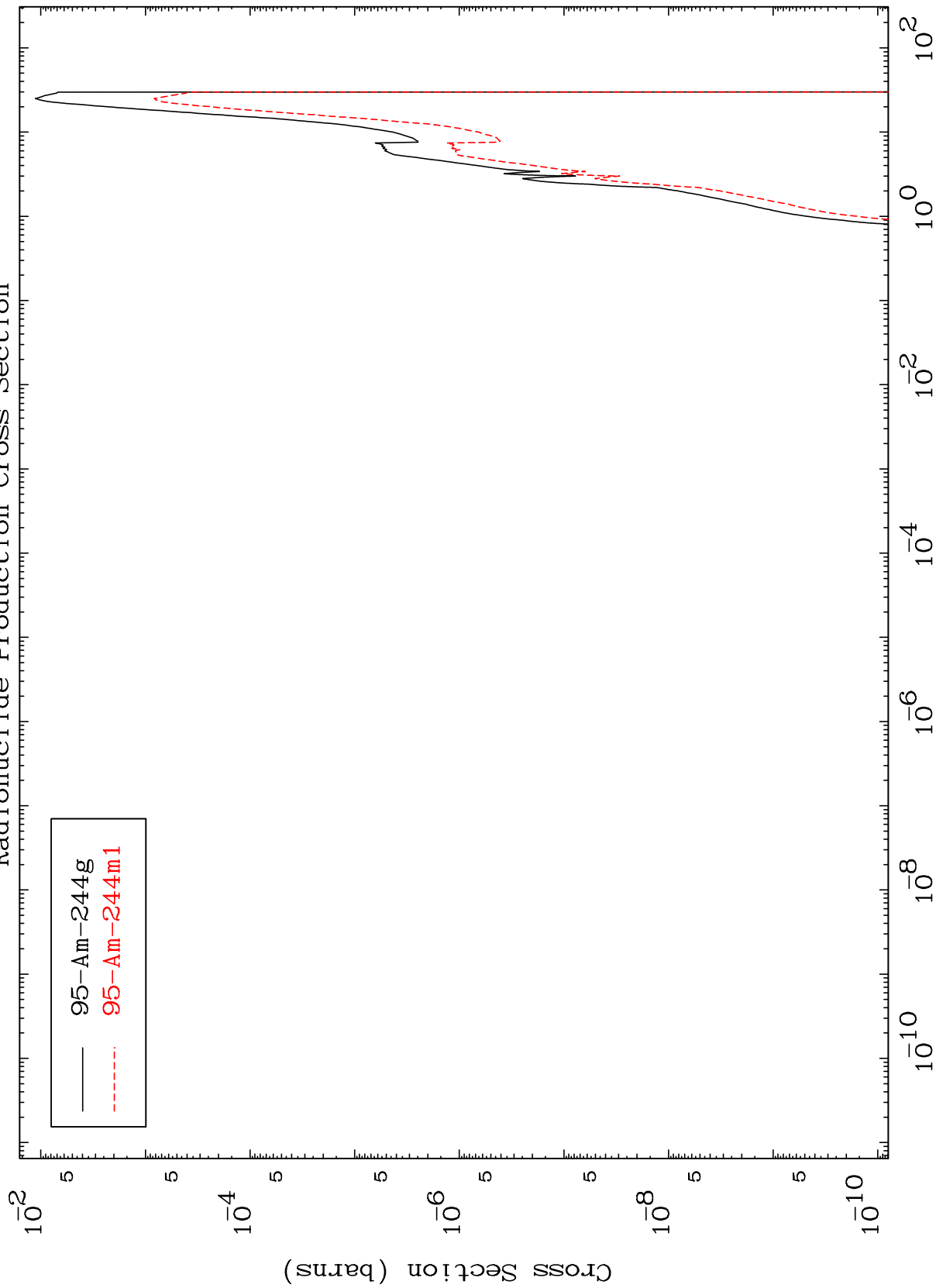
26

MAT 9750

(n,n') α

97-Bk-248

Radionuclide Production Cross Section



— 95-Am-244g
- - - 95-Am-244m1

Incident Energy (MeV)

97-Bk-248

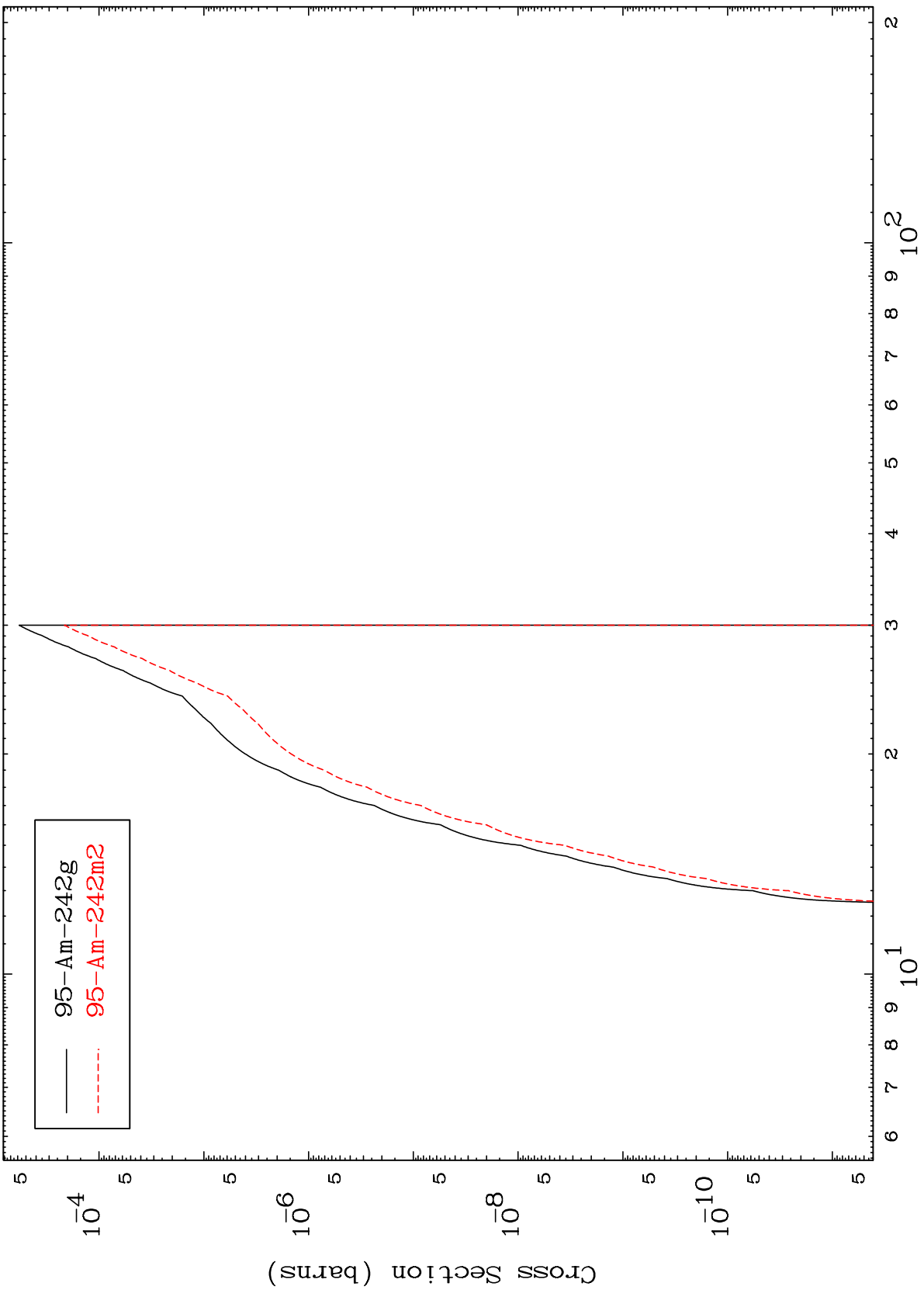
27

MAT 9750

(n,3n) α

97-Bk-248

Radionuclide Production Cross Section



28

Incident Energy (MeV)

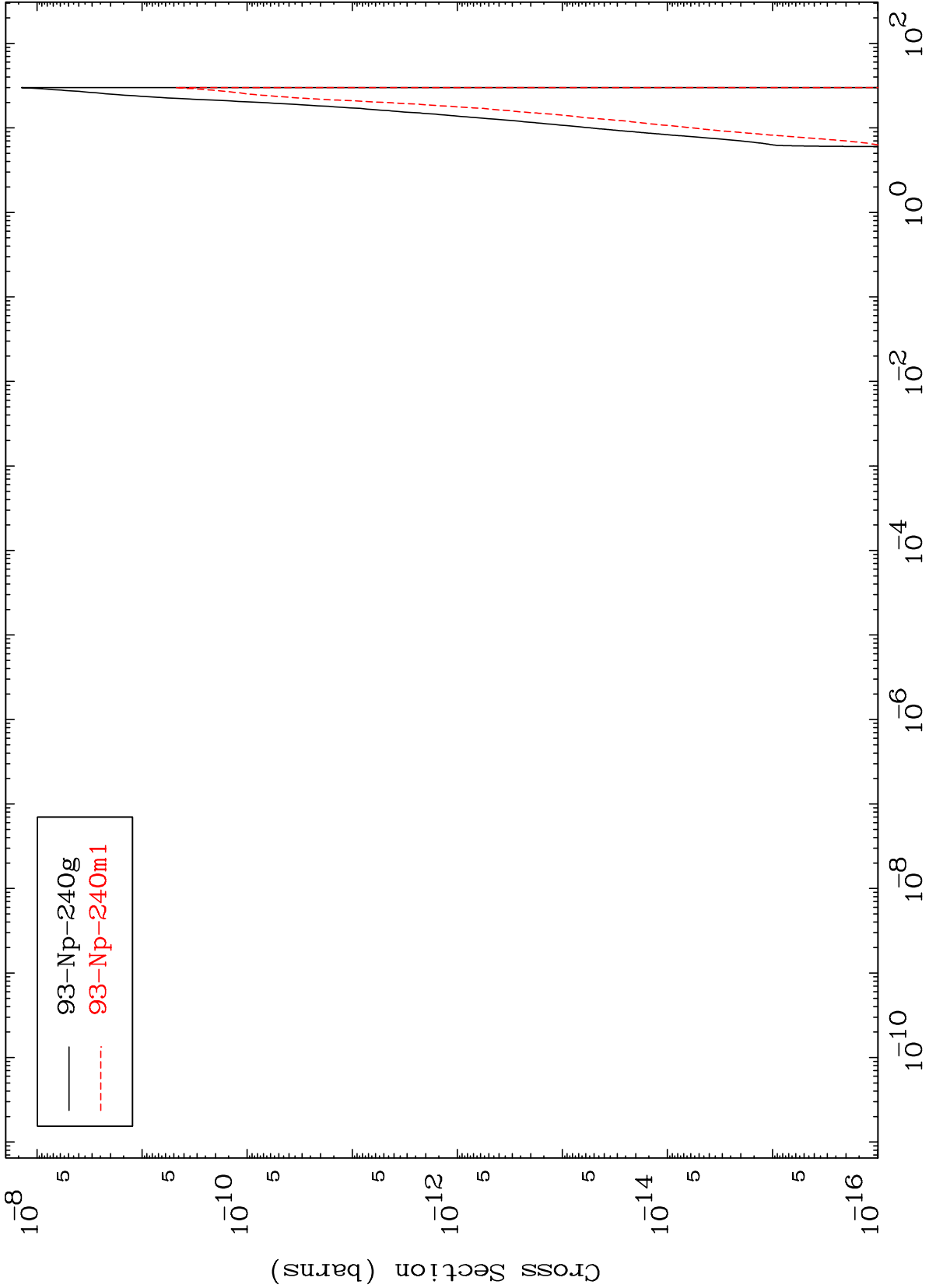
97-Bk-248

MAT 9750

(n,n') 2α

97-Bk-248

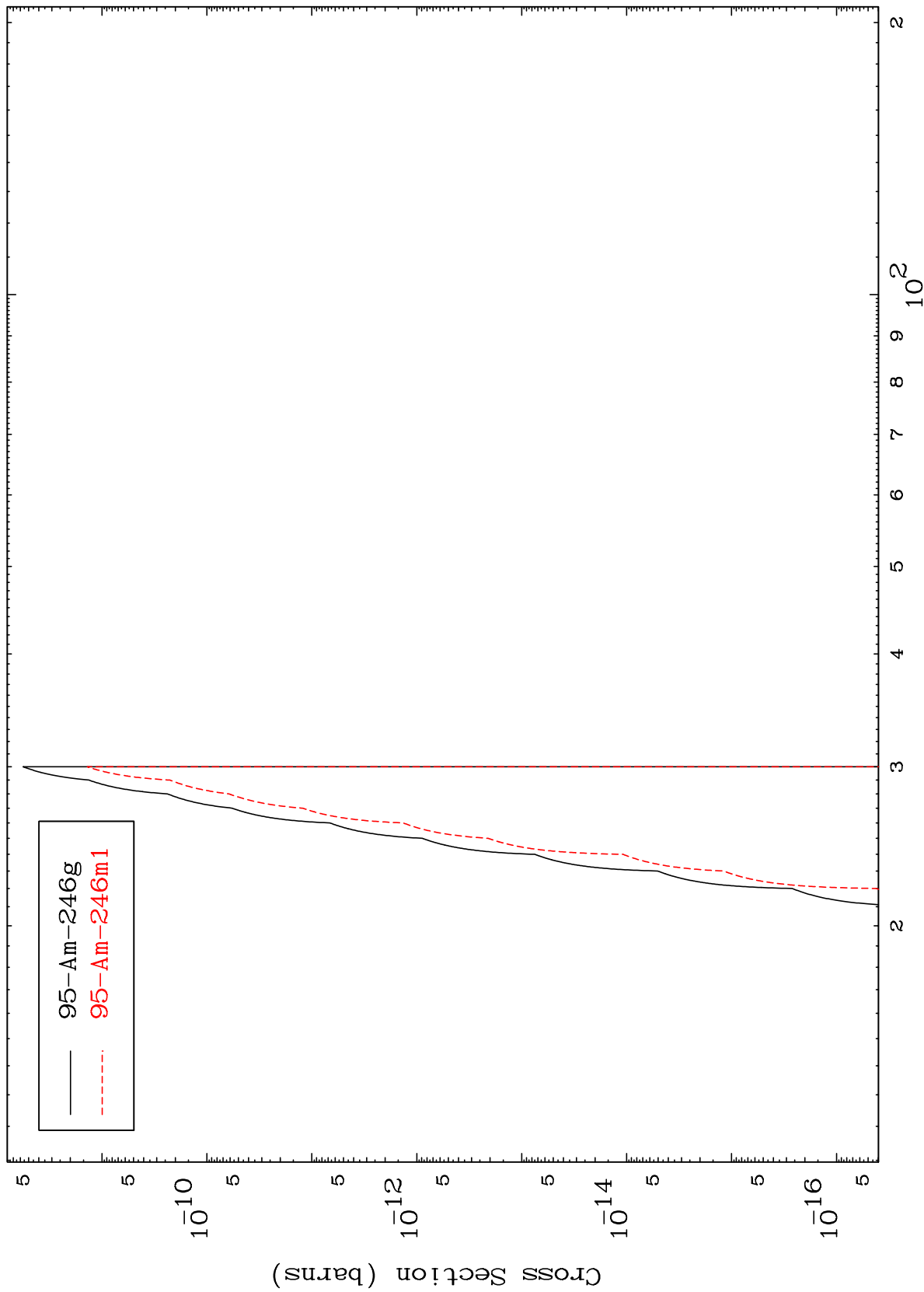
Radionuclide Production Cross Section



MAT 9750

97-Bk-248

(n,2n) p
Radionuclide Production Cross Section



Incident Energy (MeV)

97-Bk-248

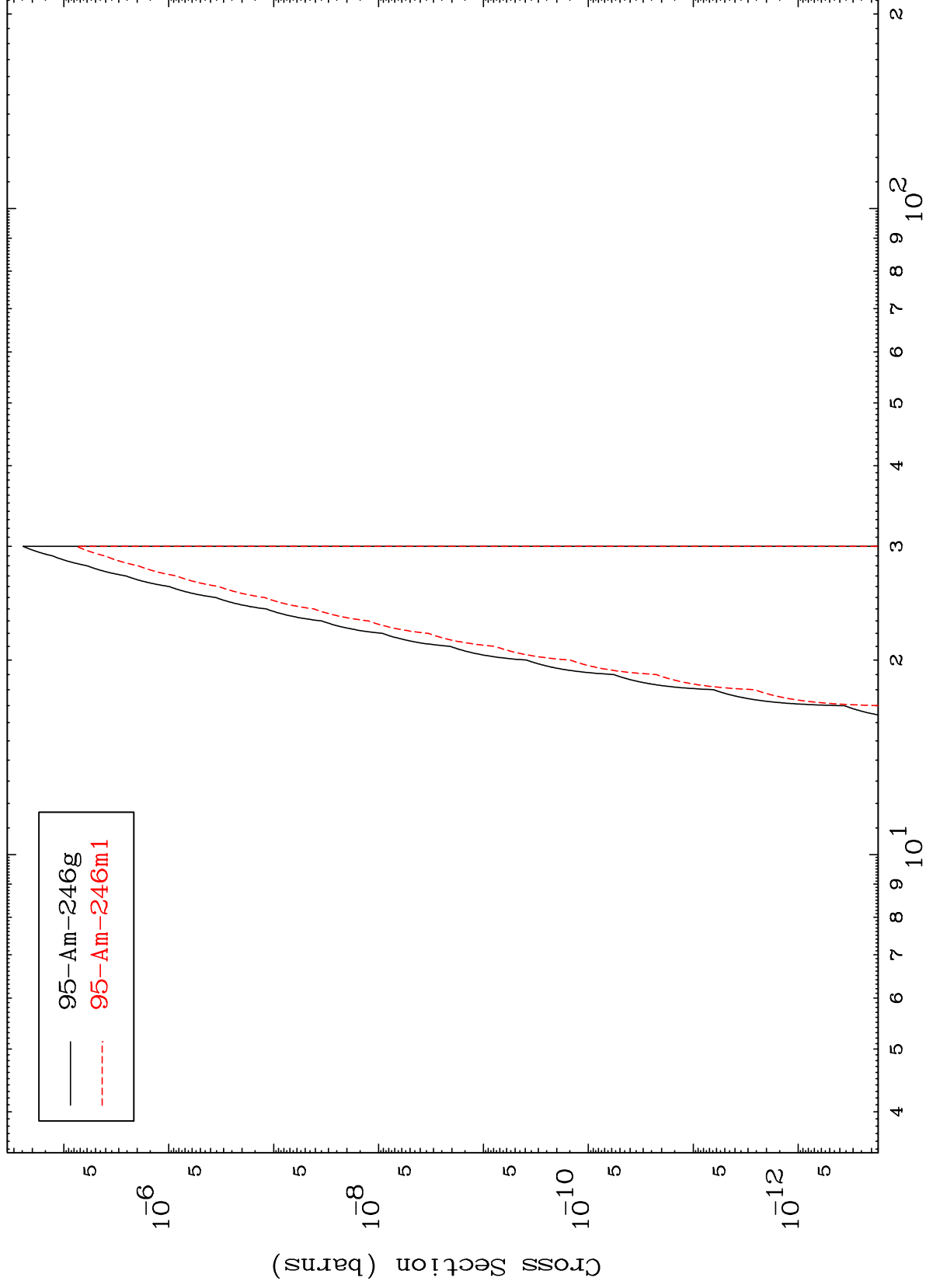
30

MAT 9750

(n,He-3)

97-Bk-248

Radionuclide Production Cross Section



31

Incident Energy (MeV)

97-Bk-248