

Program EVALPLOT  
(Version 2015-2)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

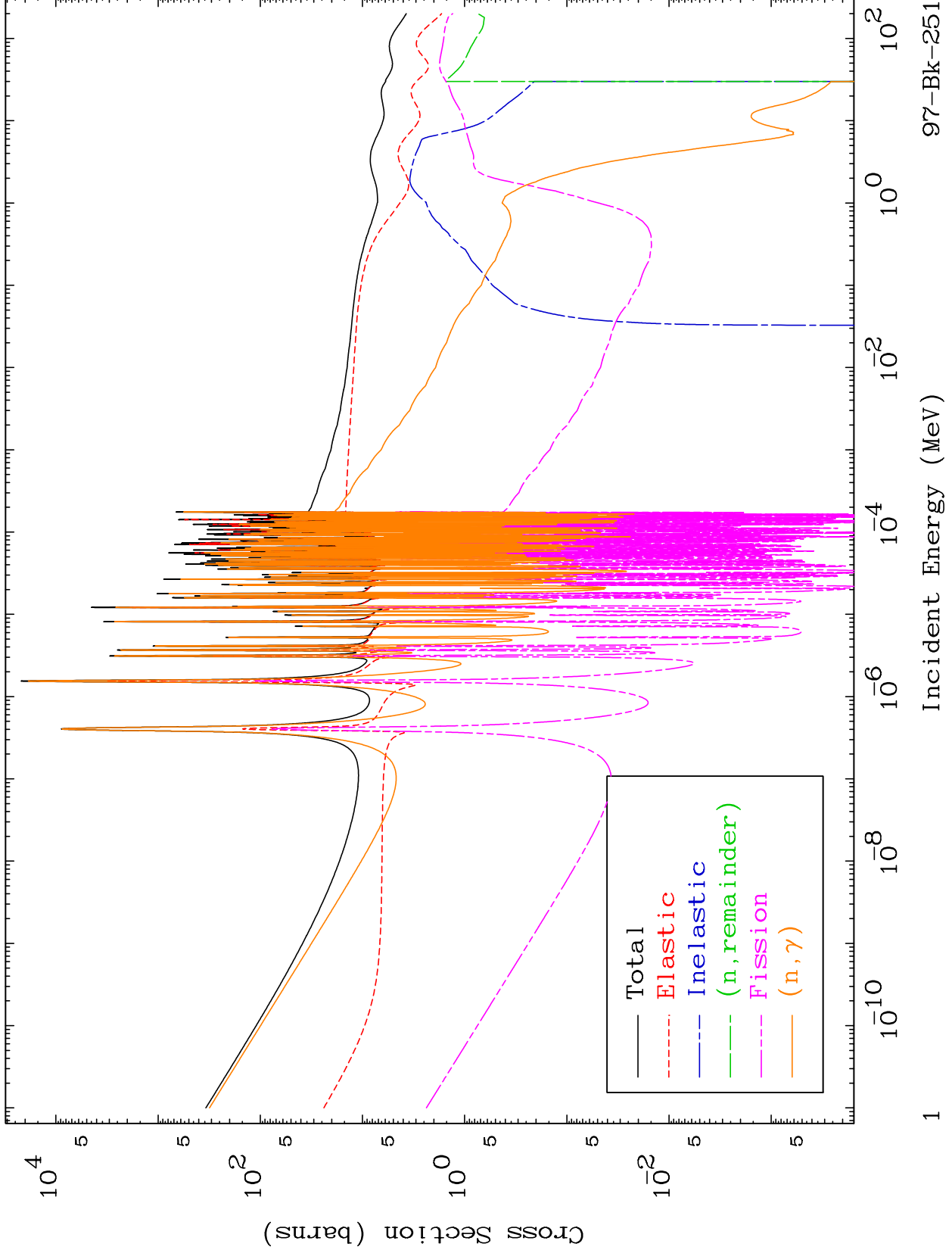
Press Mouse Button to Start

MAT 9758

Major

293 Kelvin Cross Sections

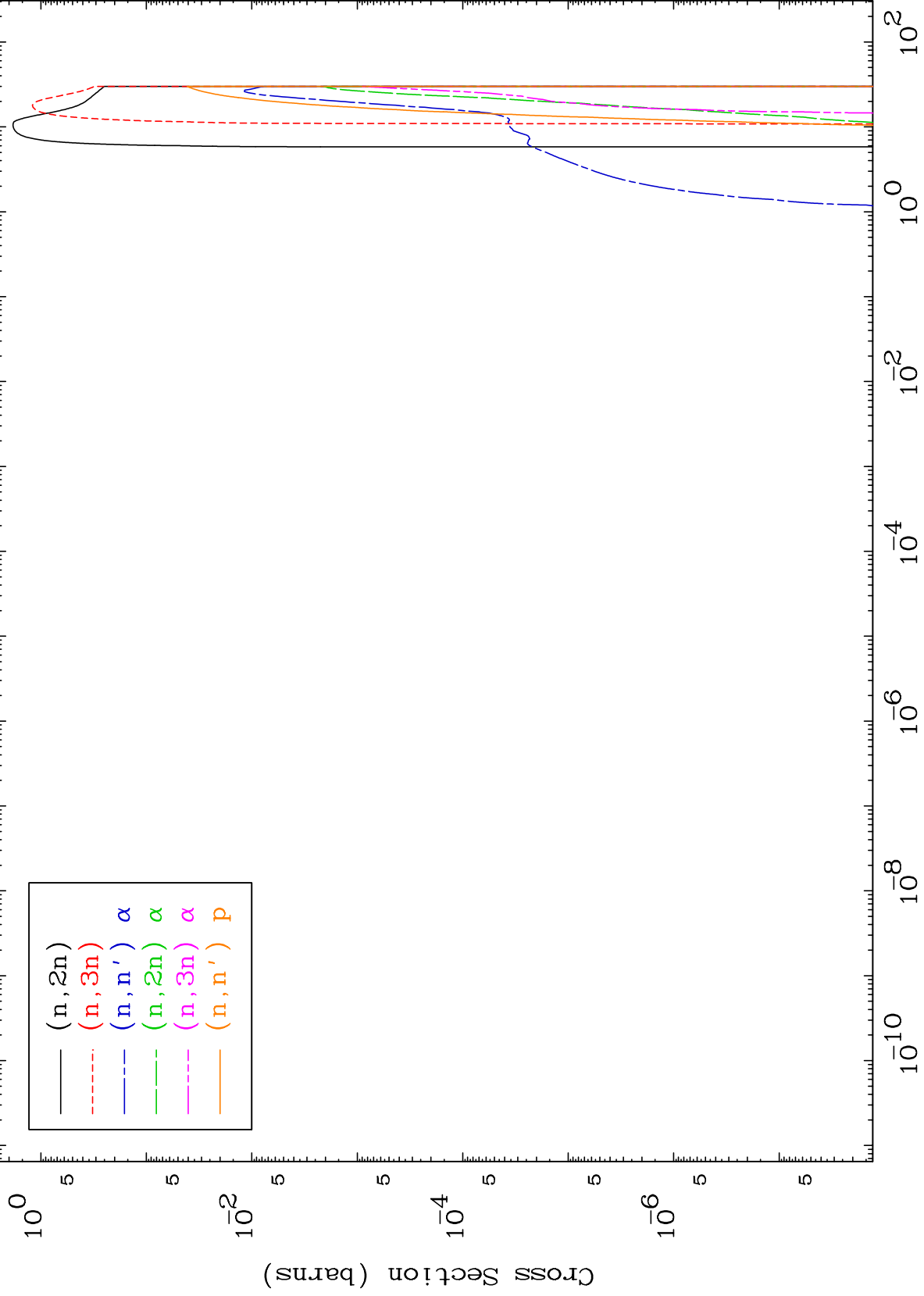
97-Bk-251



MAT 9758

Neutron Production  
293 Kelvin Cross Sections

97-Bk-251



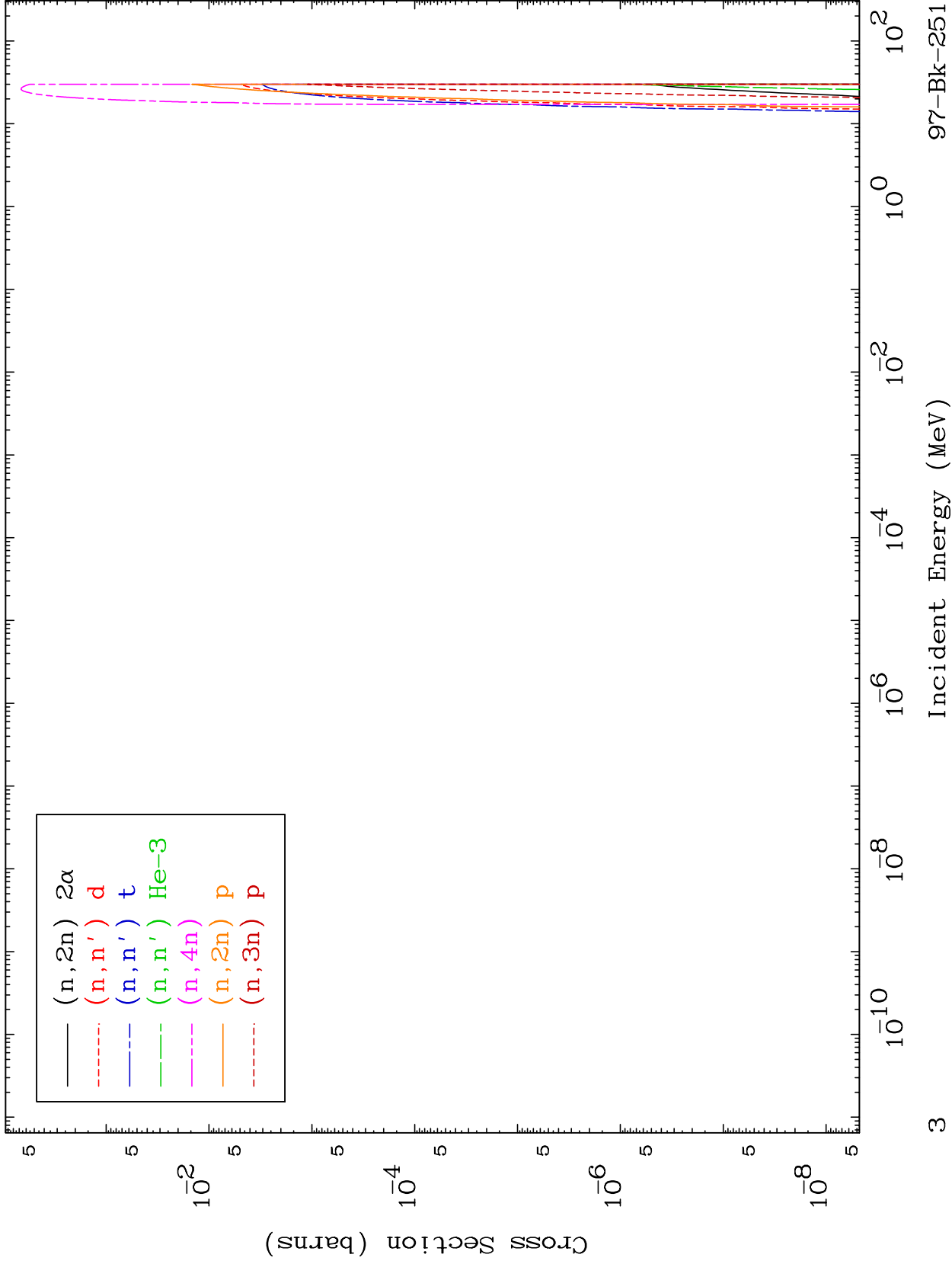
97-Bk-251

Incident Energy (MeV)

MAT 9758

Neutron Production  
293 Kelvin Cross Sections

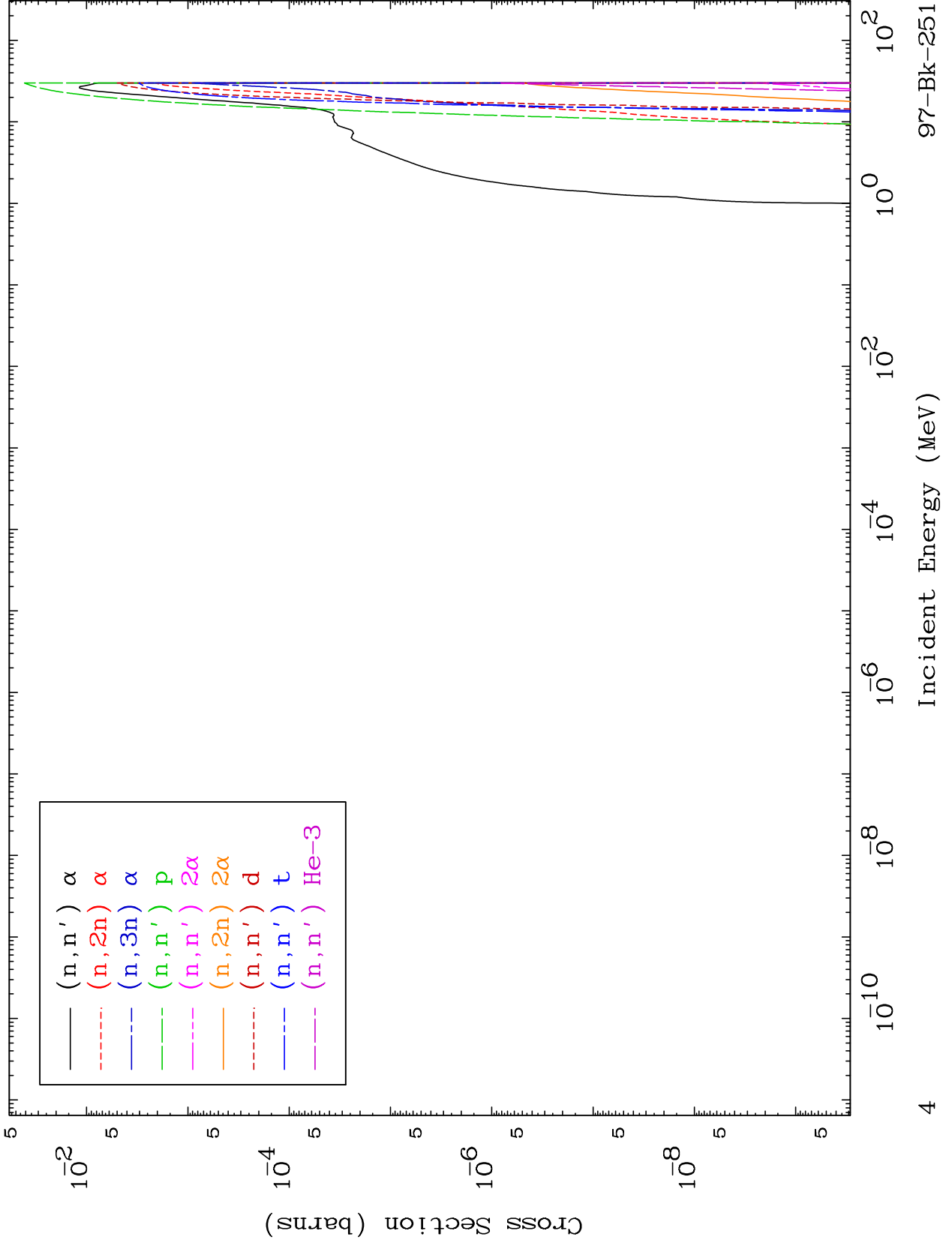
97-Bk-251



MAT 9758

Charged Particle  
293 Kelvin Cross Sections

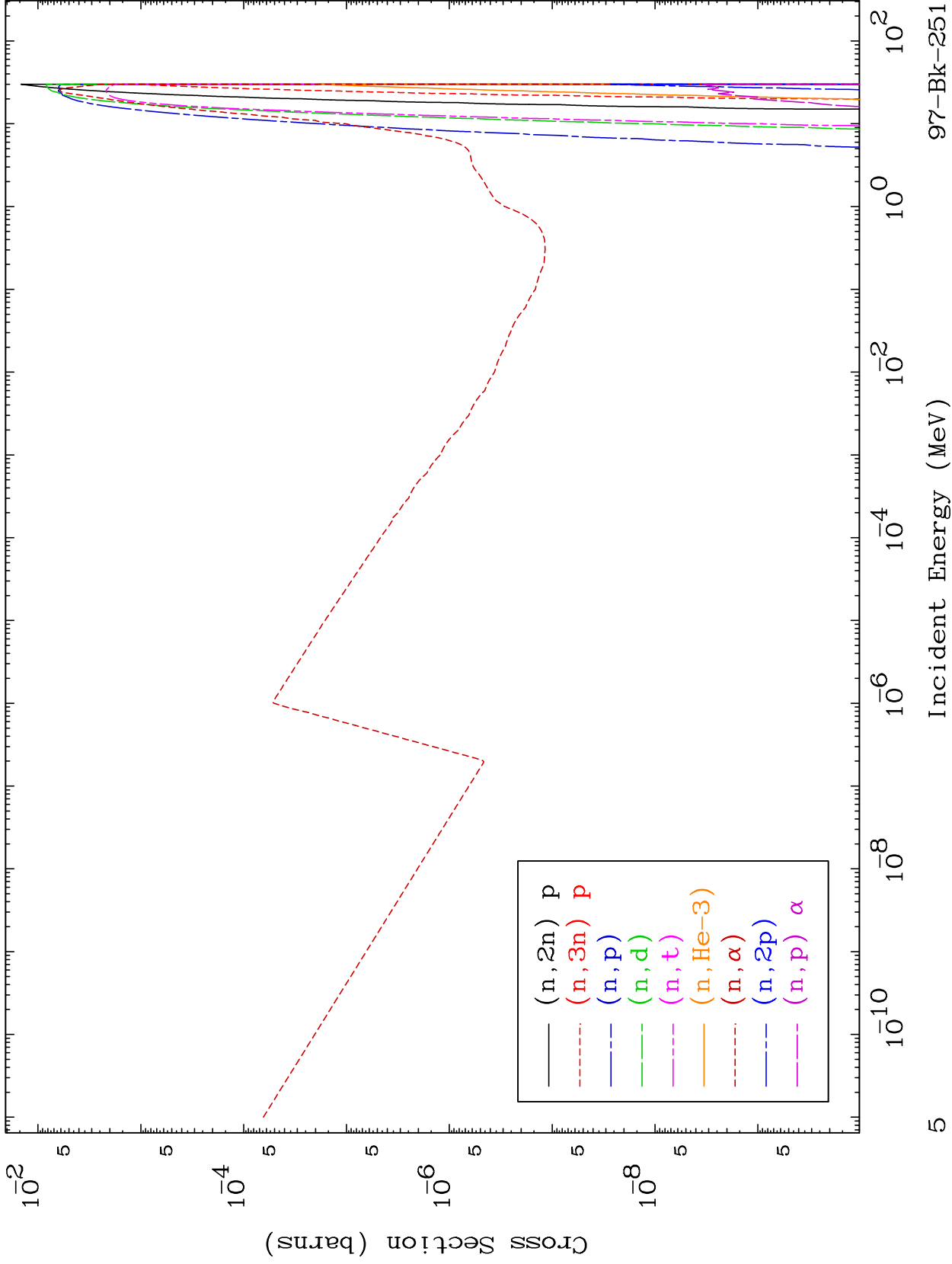
97-Bk-251



MAT 9758

Charged Particle  
293 Kelvin Cross Sections

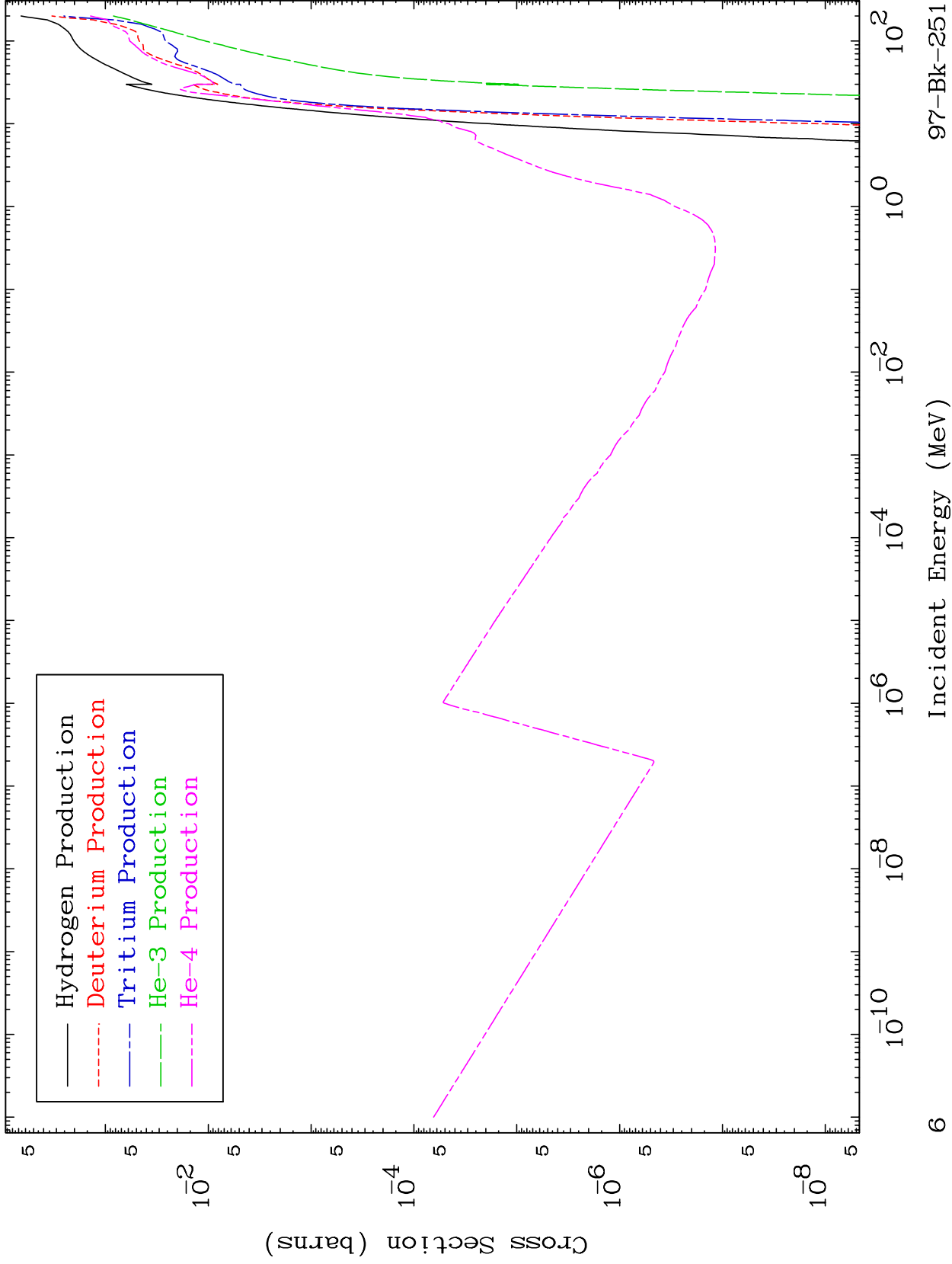
97-Bk-251



MAT 9758

Particle Production  
293 Kelvin Cross Sections

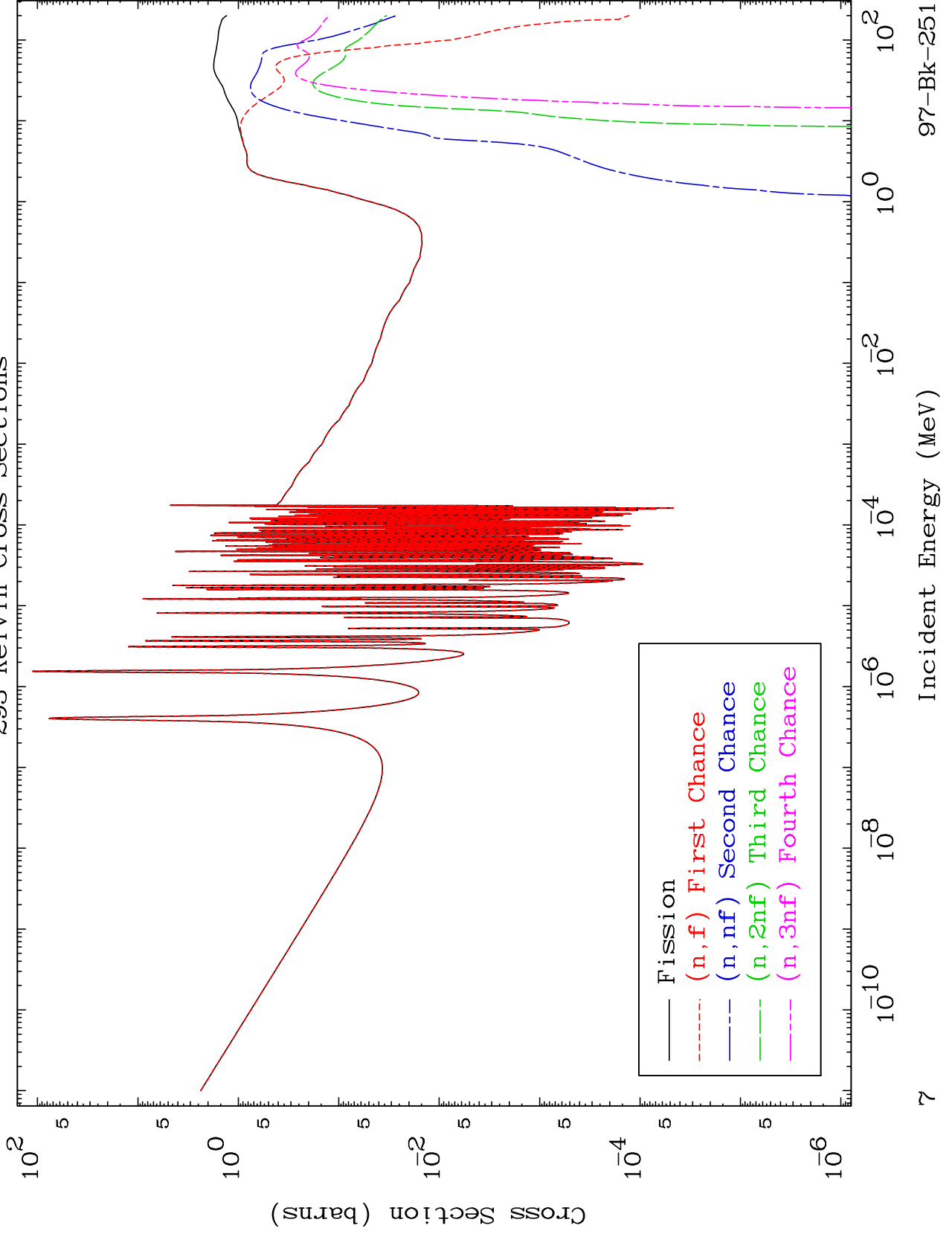
97-Bk-251



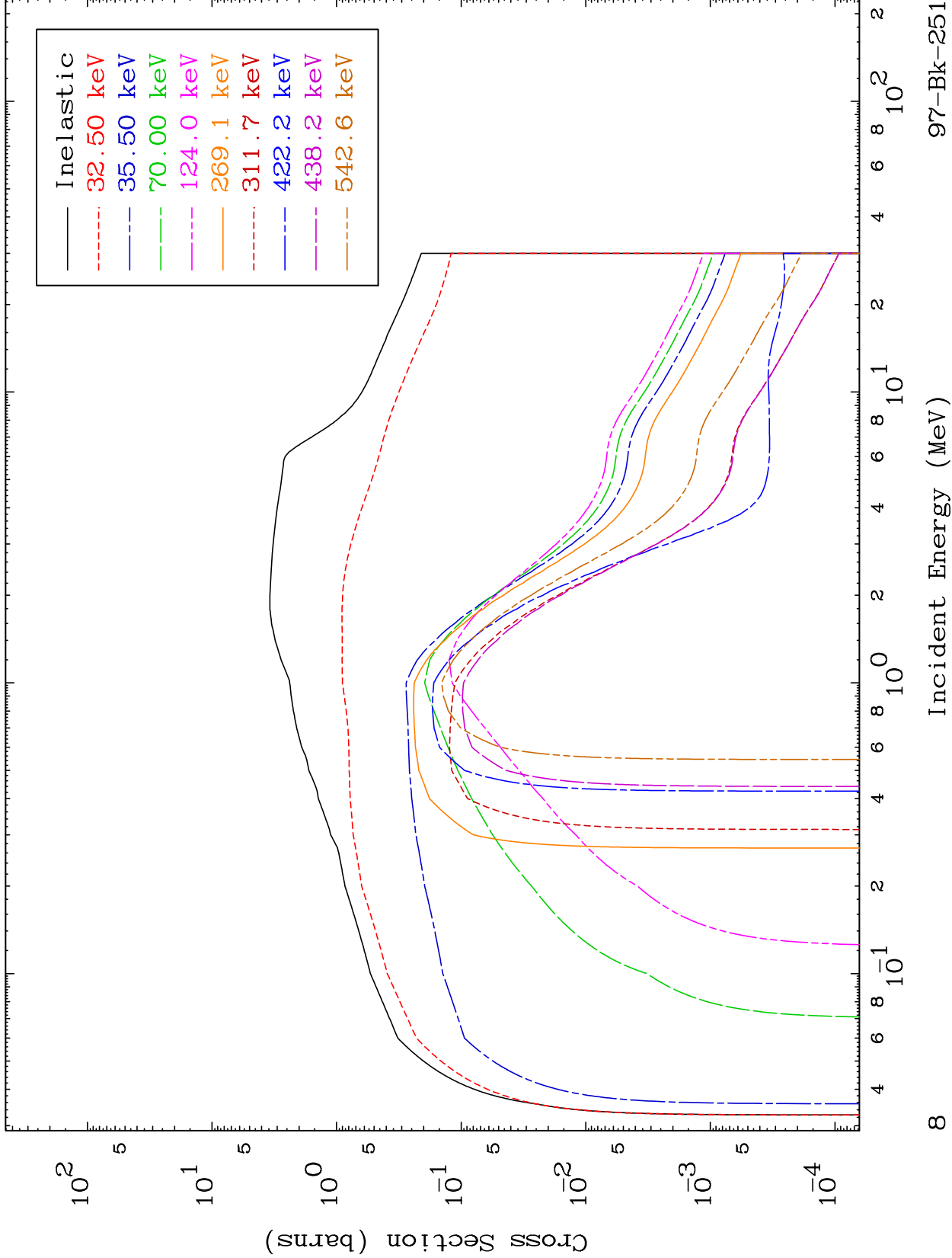
MAT 9758

Fission  
293 Kelvin Cross Sections

97-Bk-251



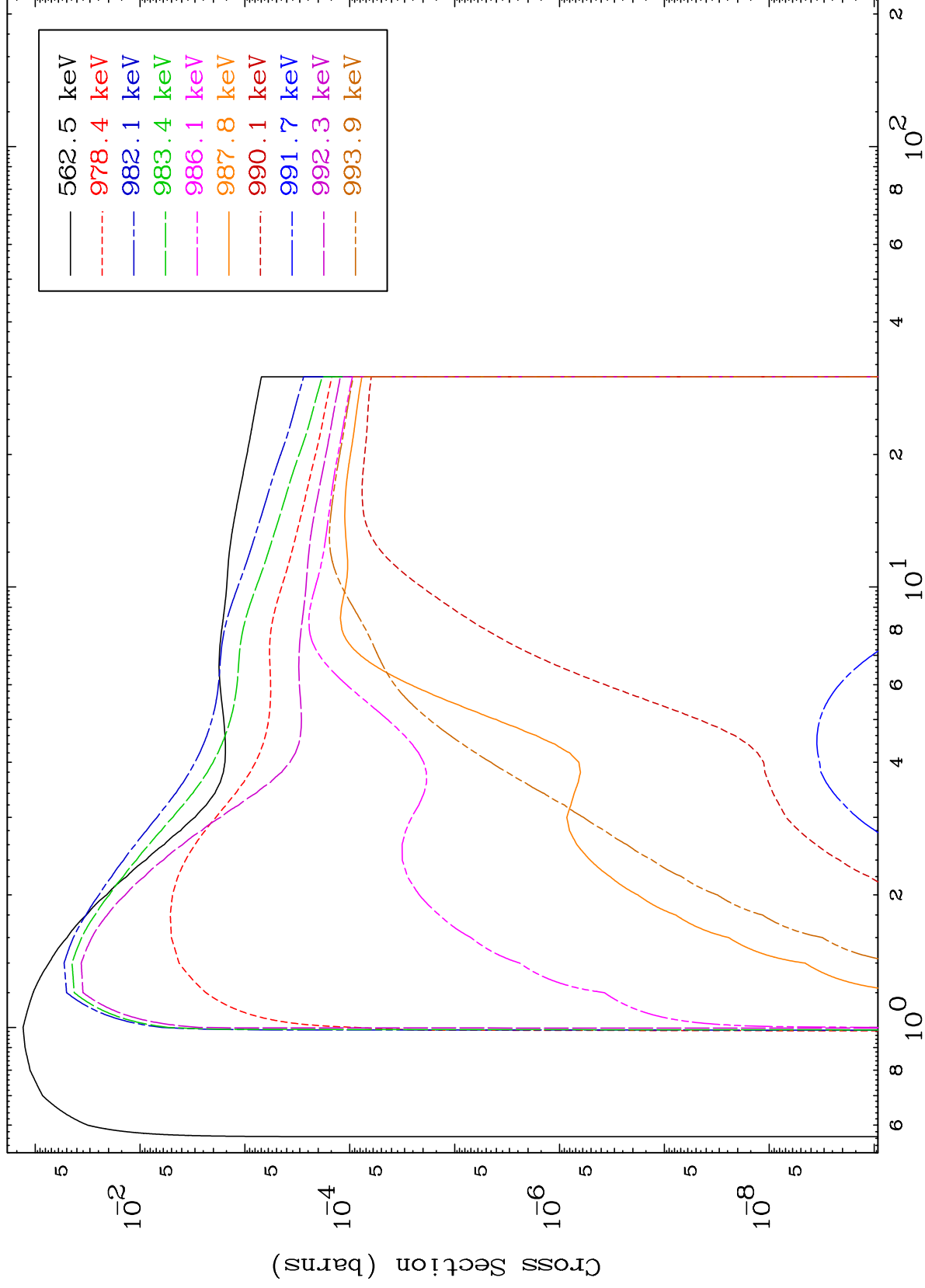




MAT 9758

(n,n') Level  
293 Kelvin Cross Sections

97-Bk-251



9

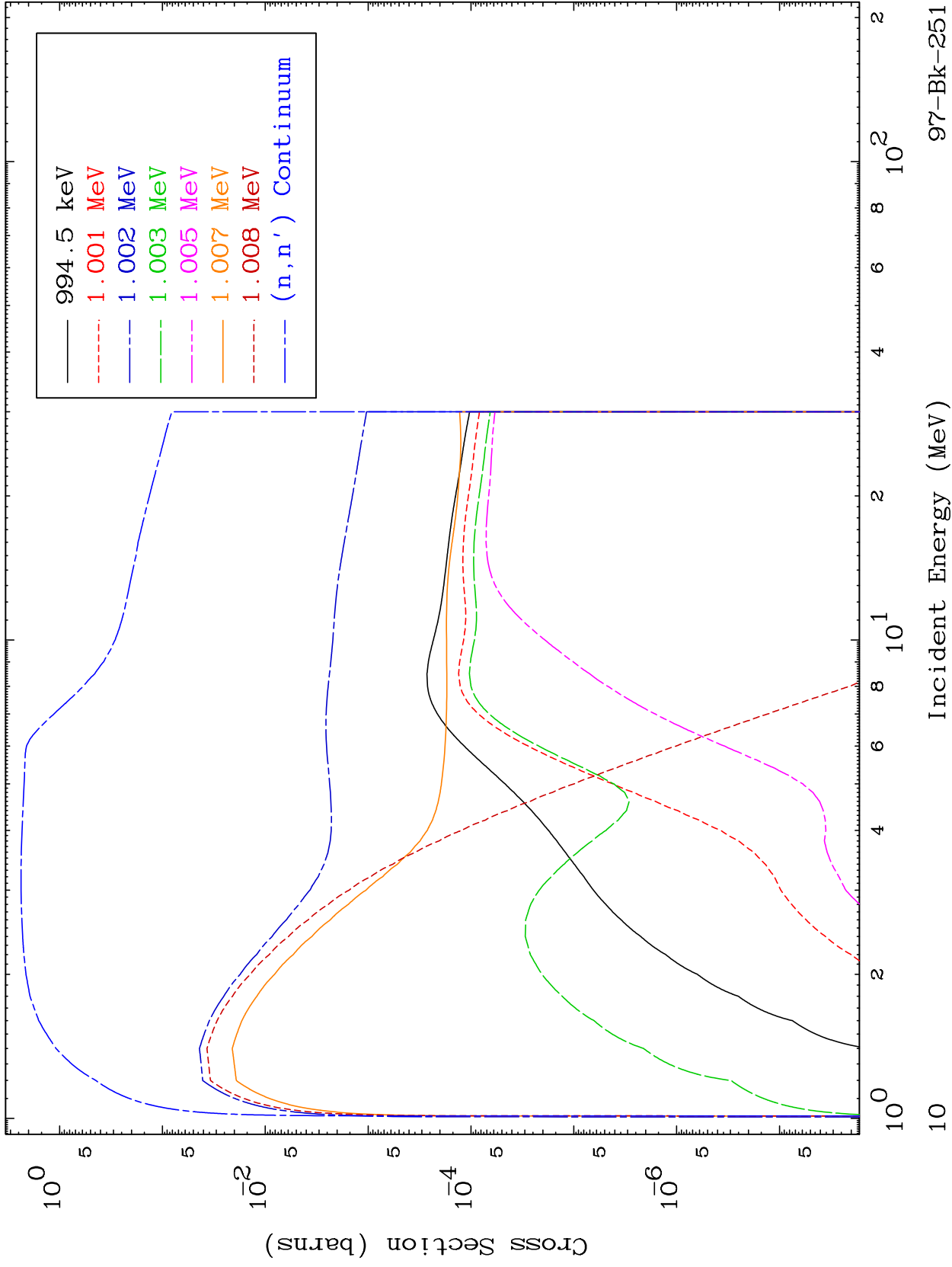
Incident Energy (MeV)

97-Bk-251

MAT 9758

(n,n') Level  
293 Kelvin Cross Sections

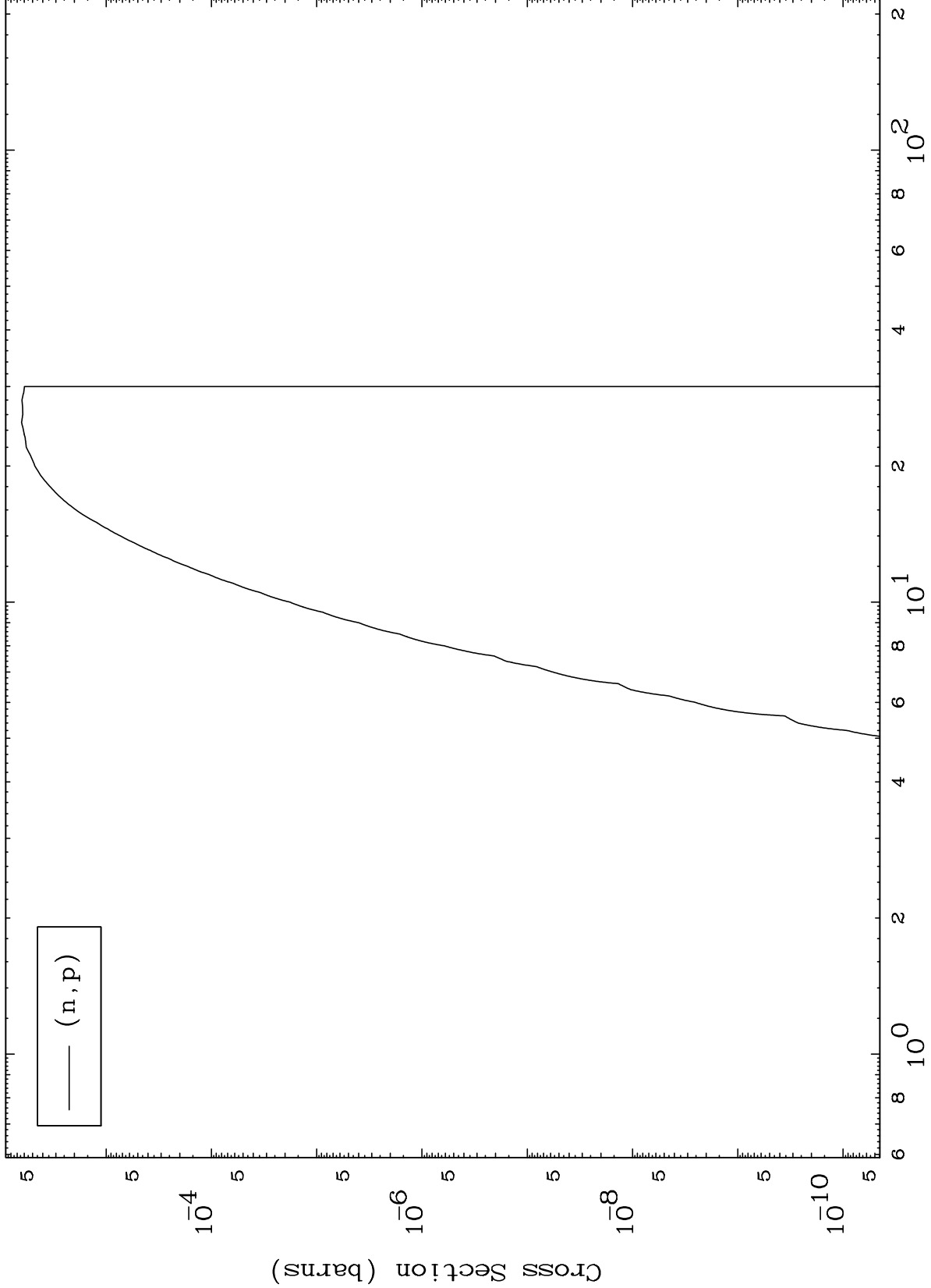
97-Bk-251



MAT 9758

(n,p) Levels  
293 Kelvin Cross Sections

97-Bk-251



11

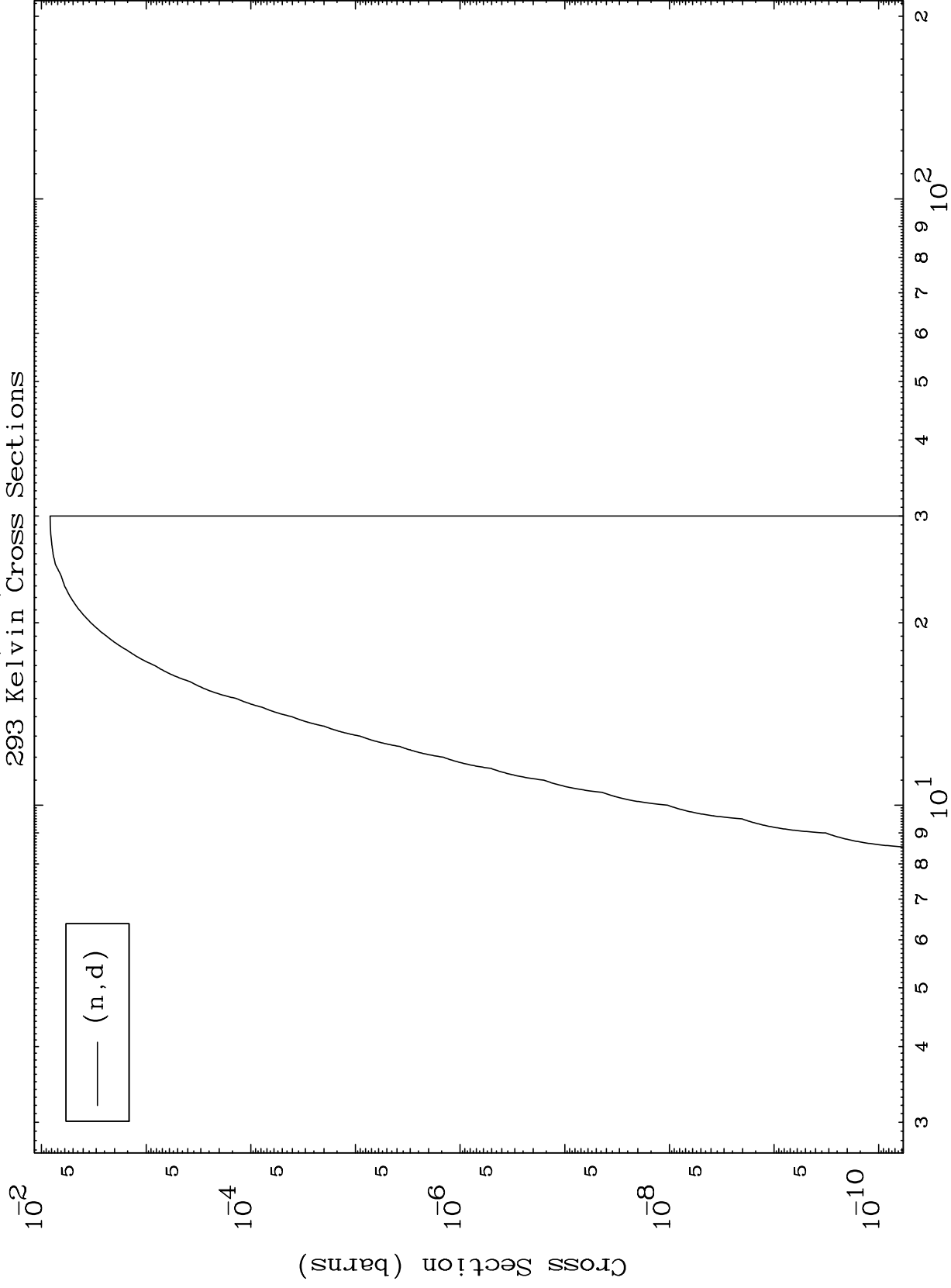
Incident Energy (MeV)

97-Bk-251

MAT 9758

(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-251



12

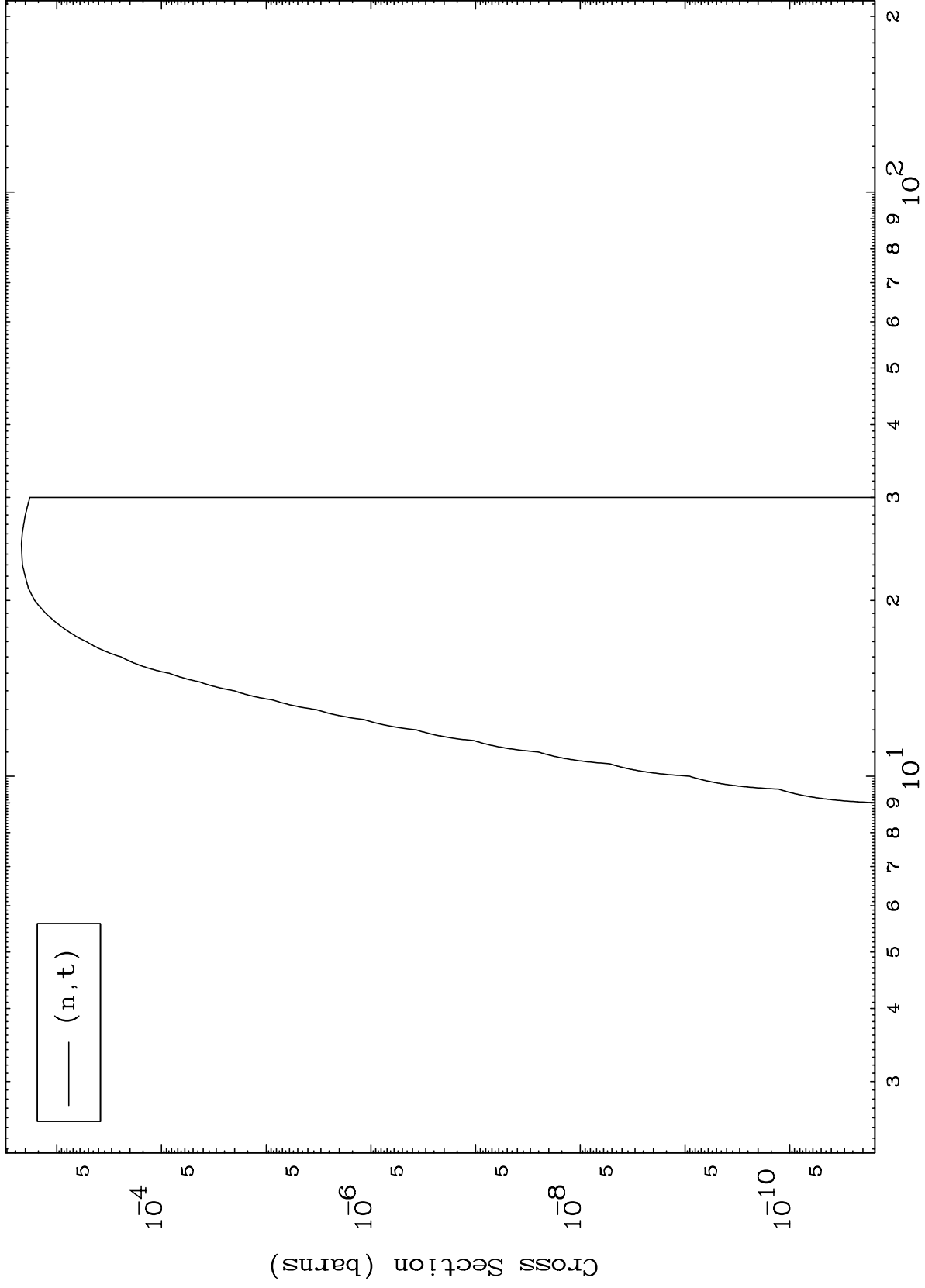
Incident Energy (MeV)

97-Bk-251

MAT 9758

(n,t) Levels  
293 Kelvin Cross Sections

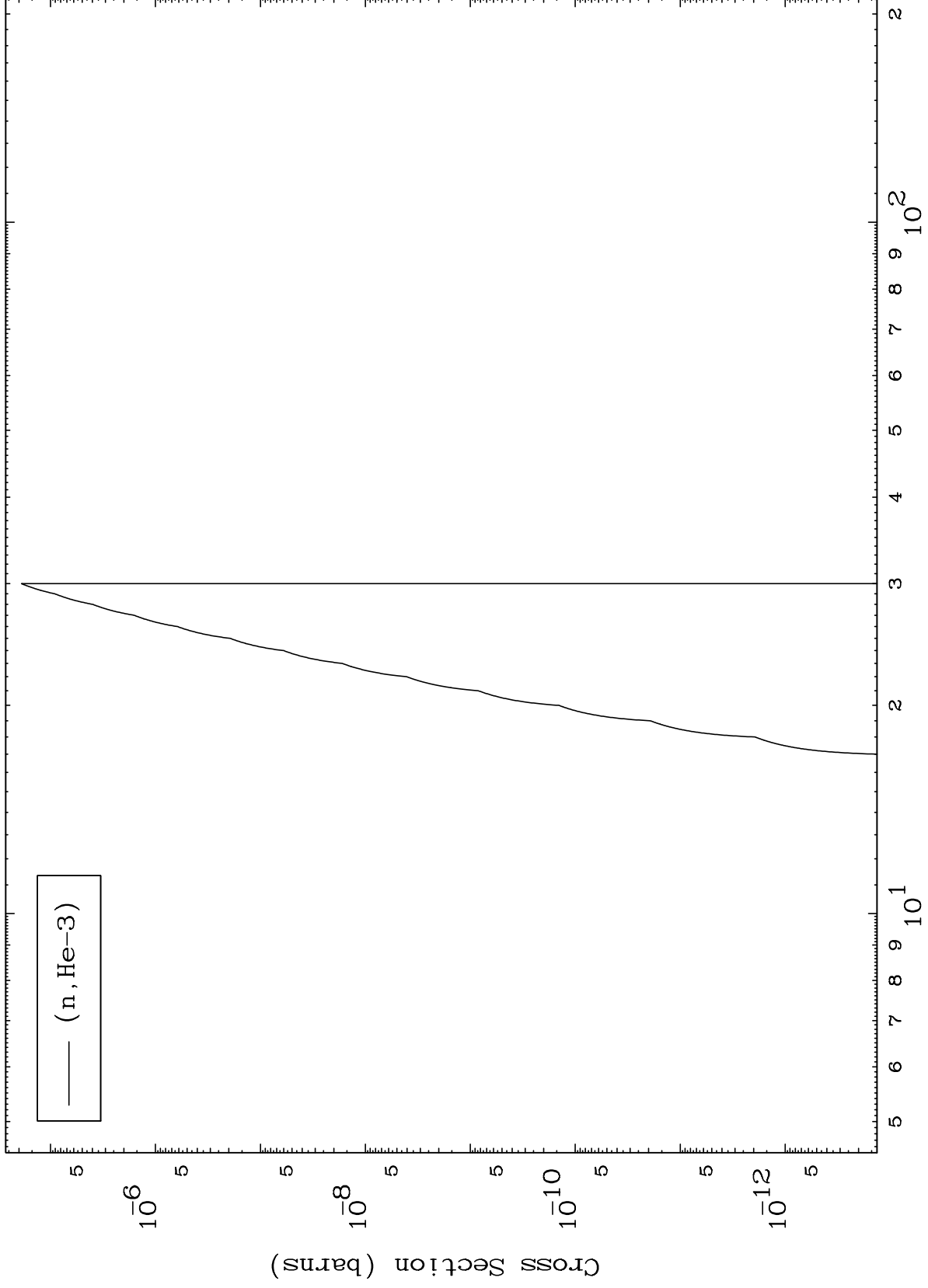
97-Bk-251



MAT 9758

(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-251



14

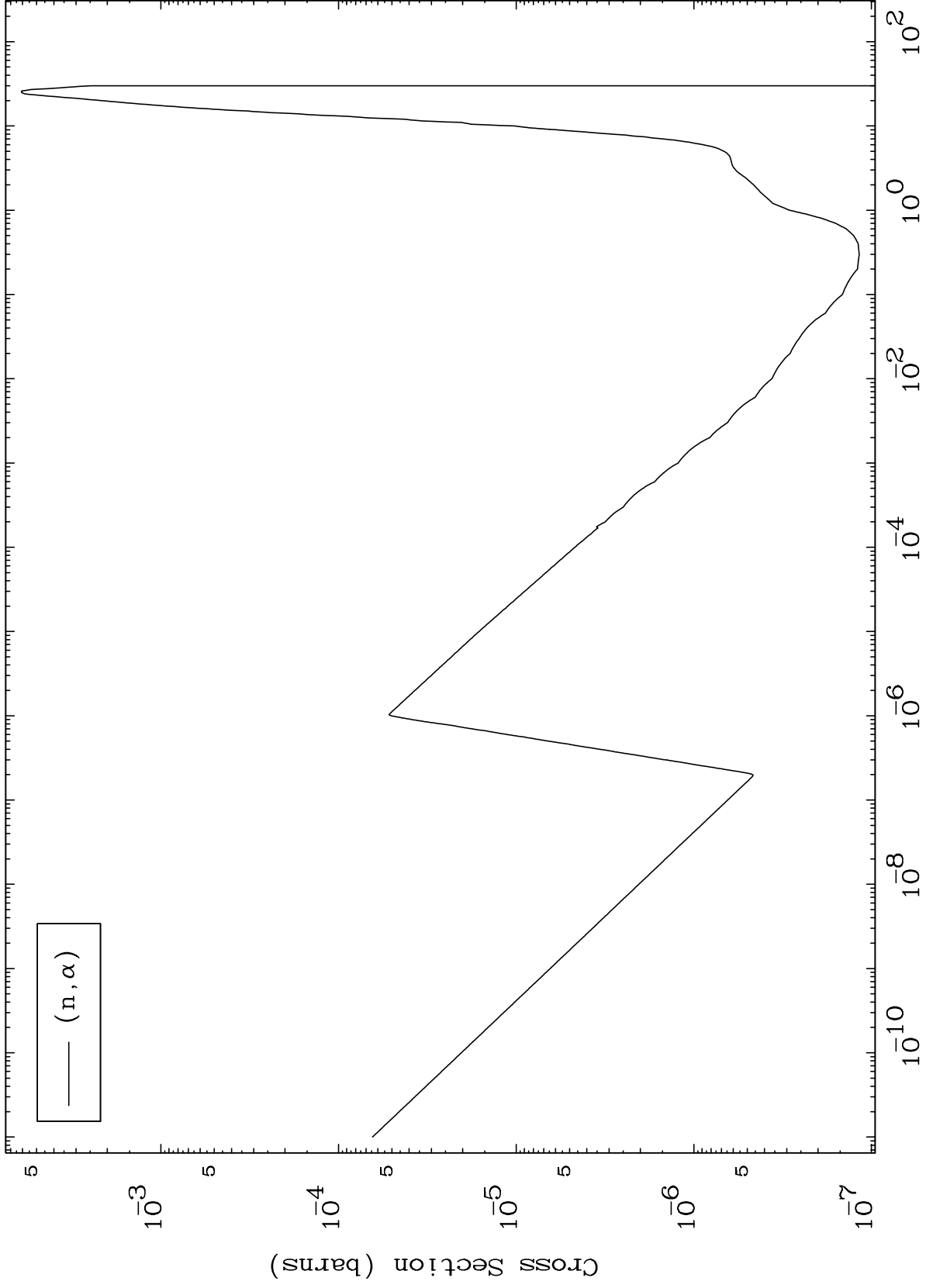
Incident Energy (MeV)

97-Bk-251

MAT 9758

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

97-Bk-251

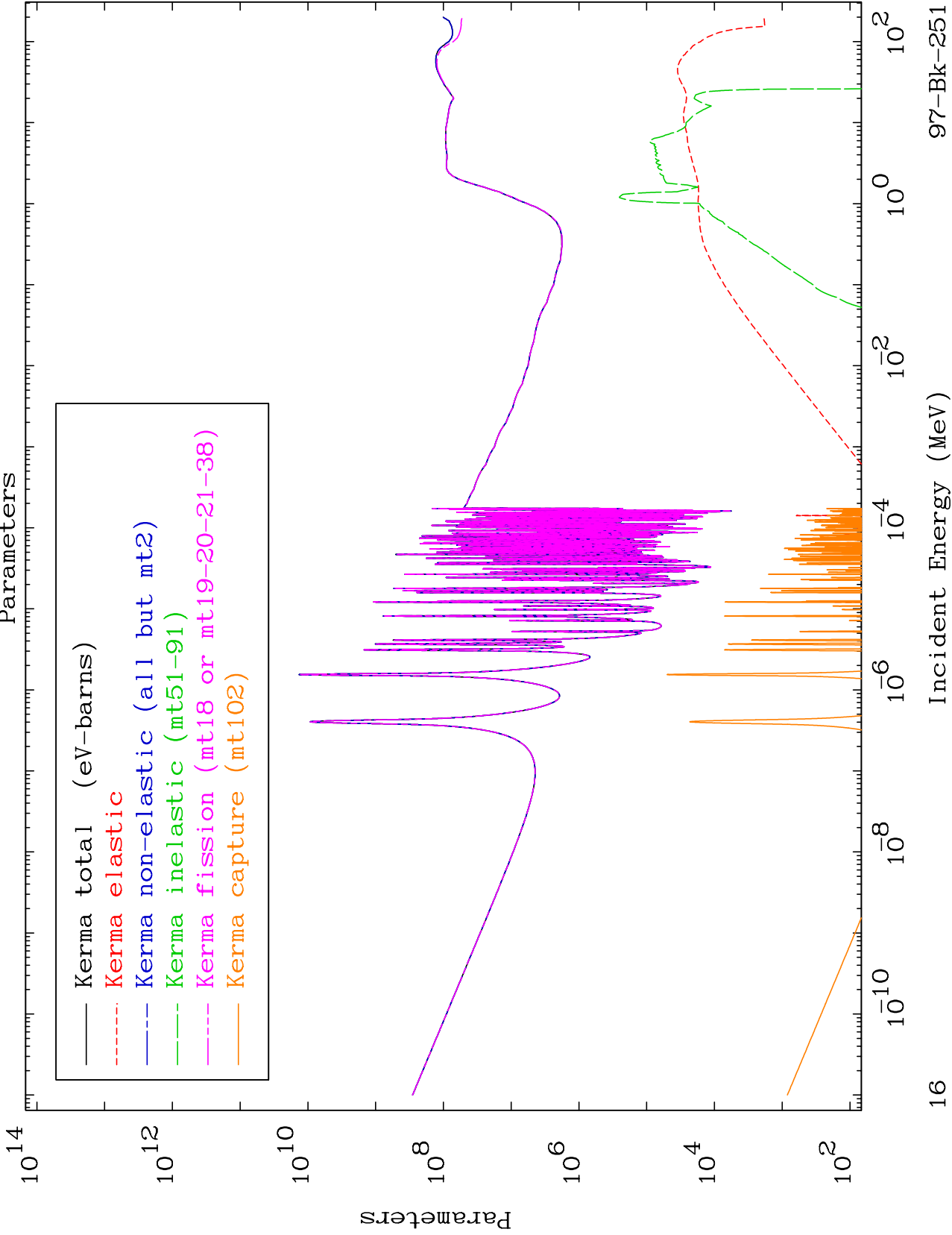


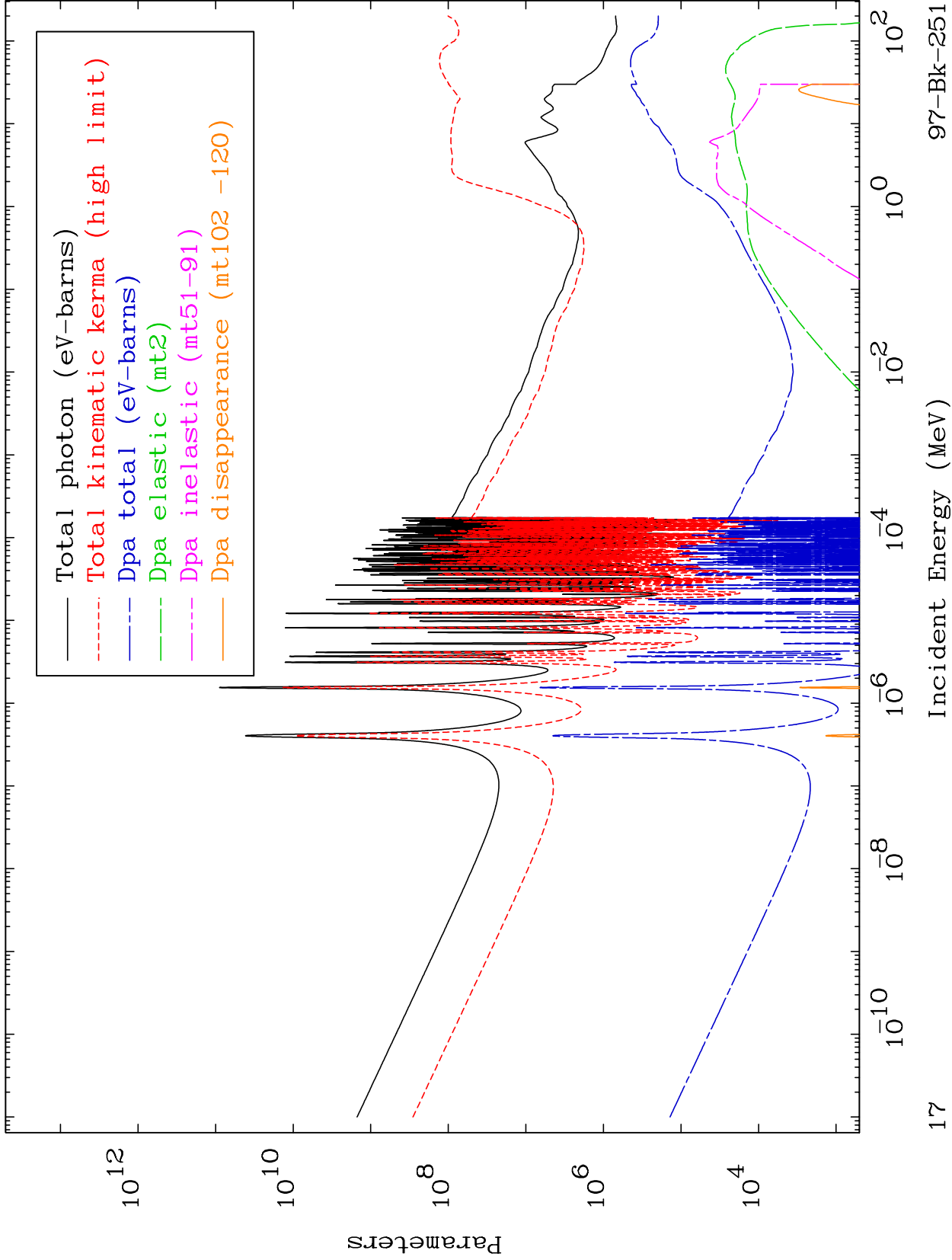
15

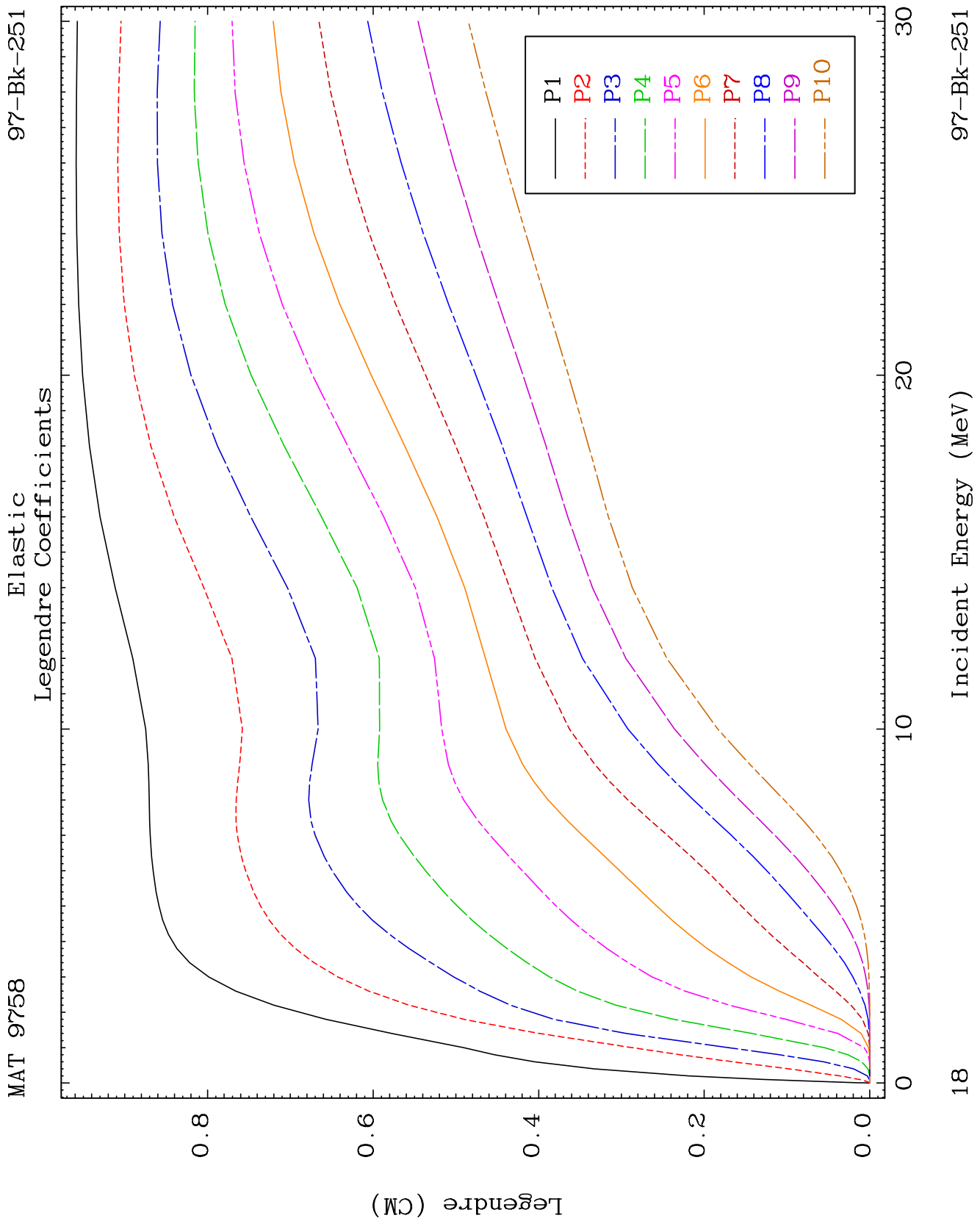
Incident Energy (MeV)

97-Bk-251





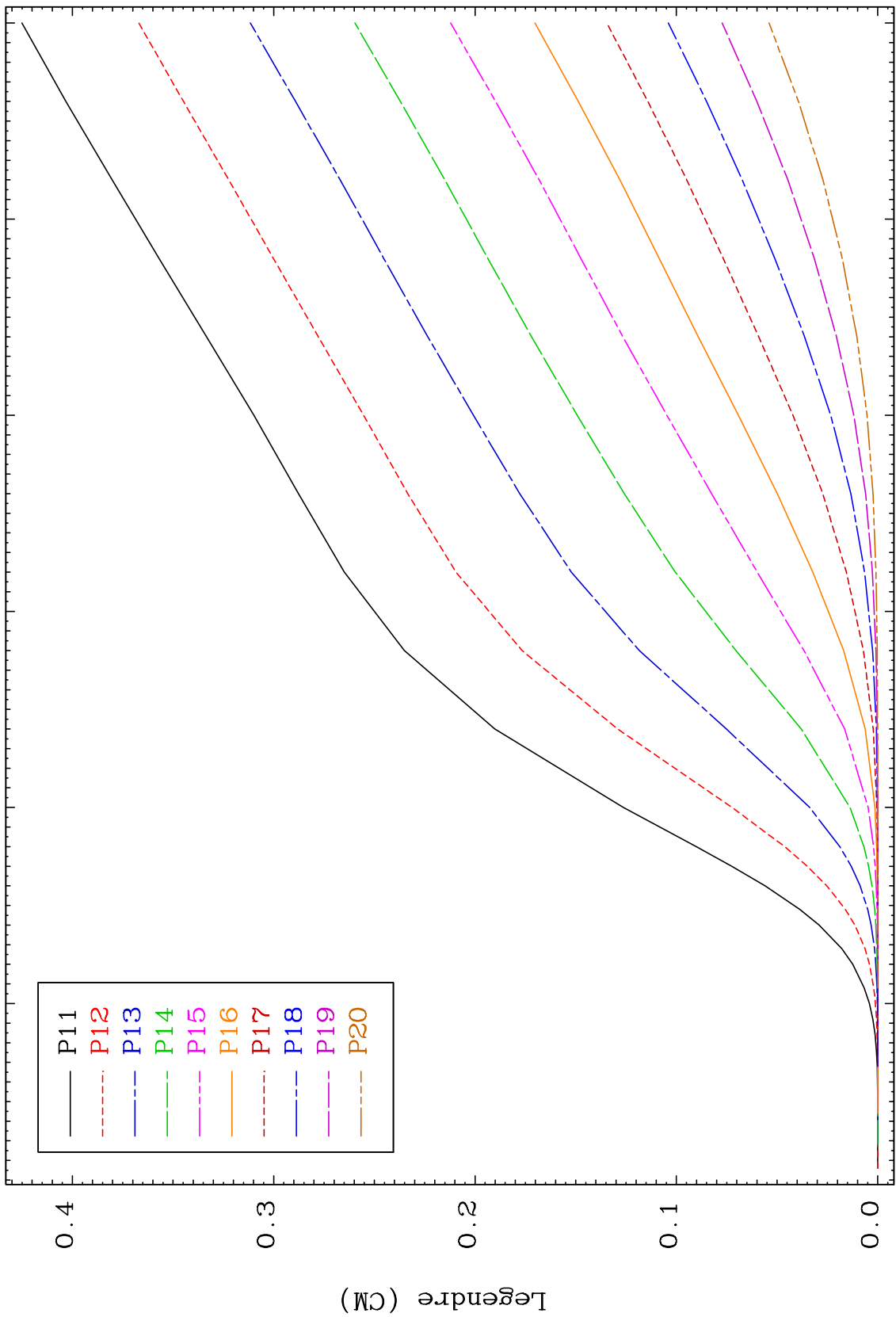




MAT 9758

Elastic Legendre Coefficients

97-Bk-251



19

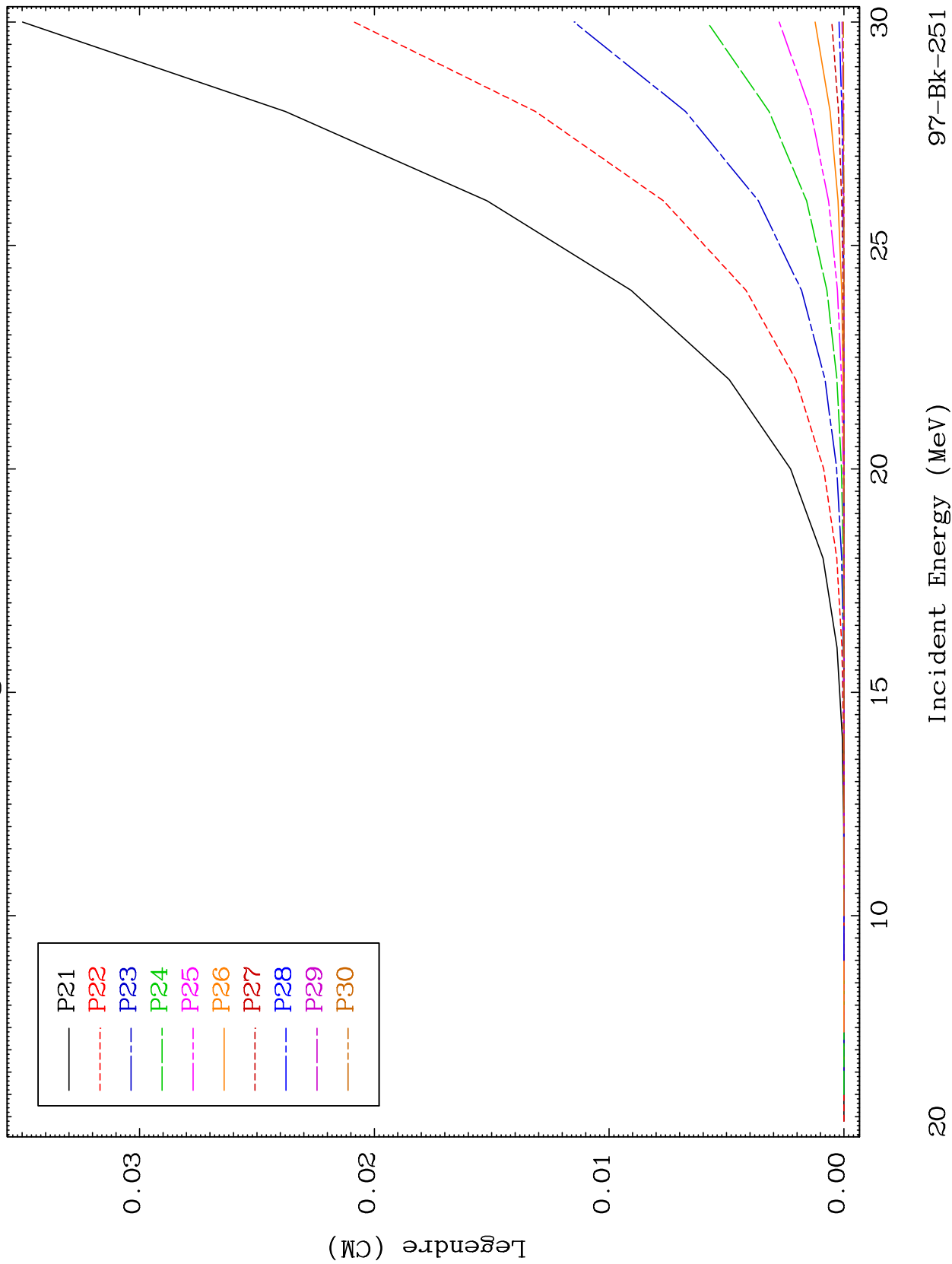
97-Bk-251

Incident Energy (MeV)

MAT 9758

### Elastic Legendre Coefficients

97-Bk-251



97-Bk-251

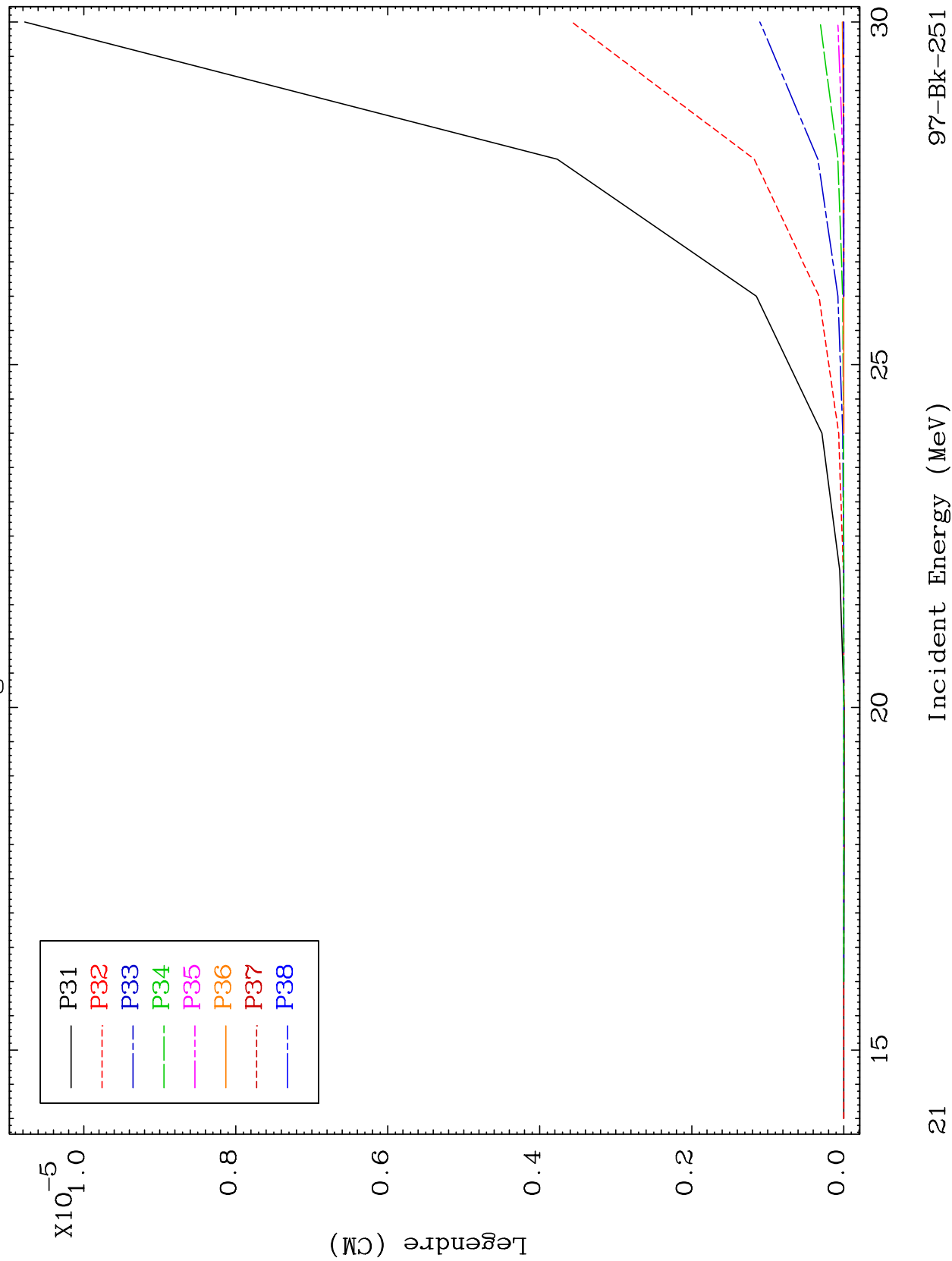
Incident Energy (MeV)

20

MAT 9758

Elastic Legendre Coefficients

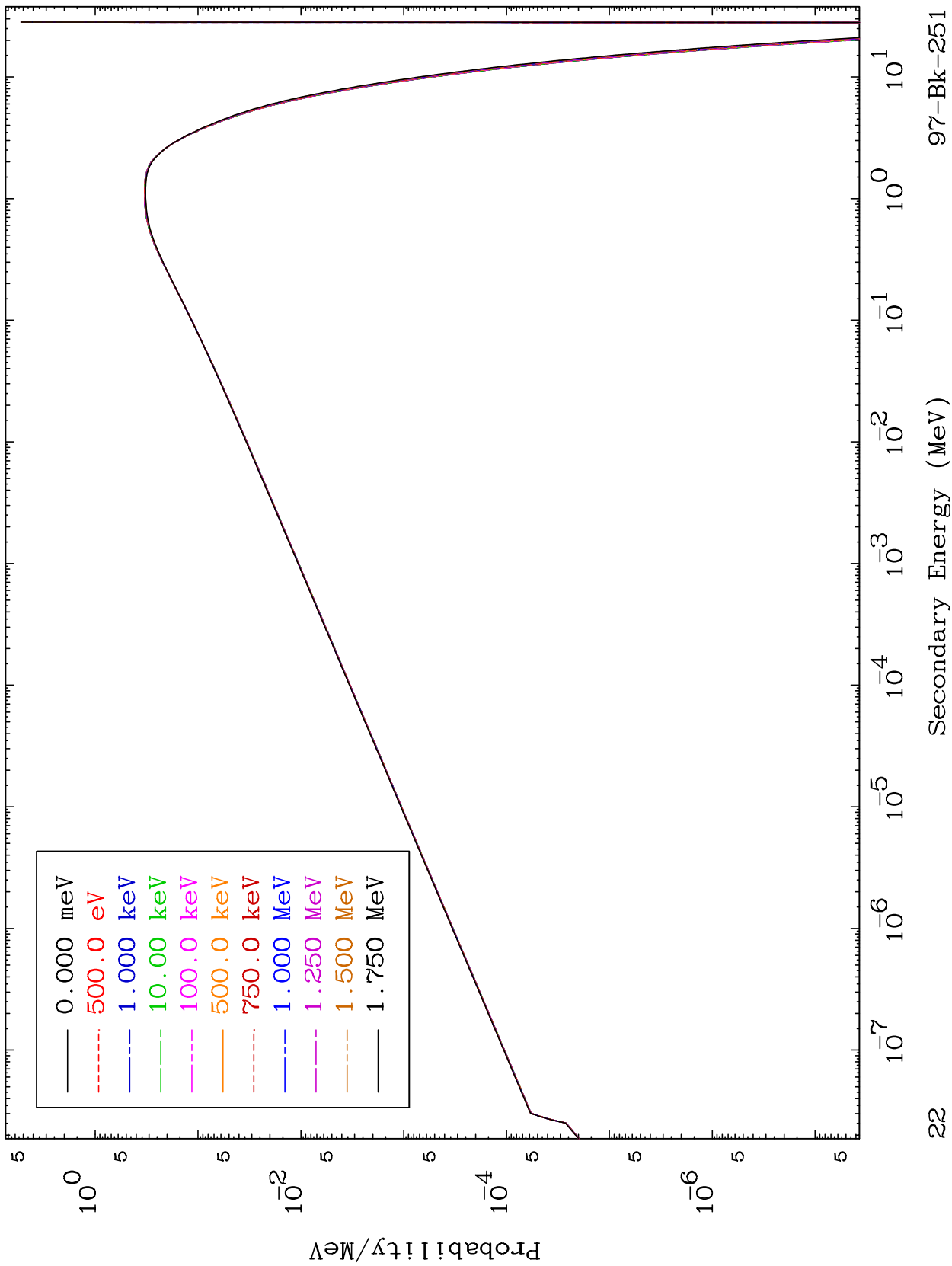
97-Bk-251

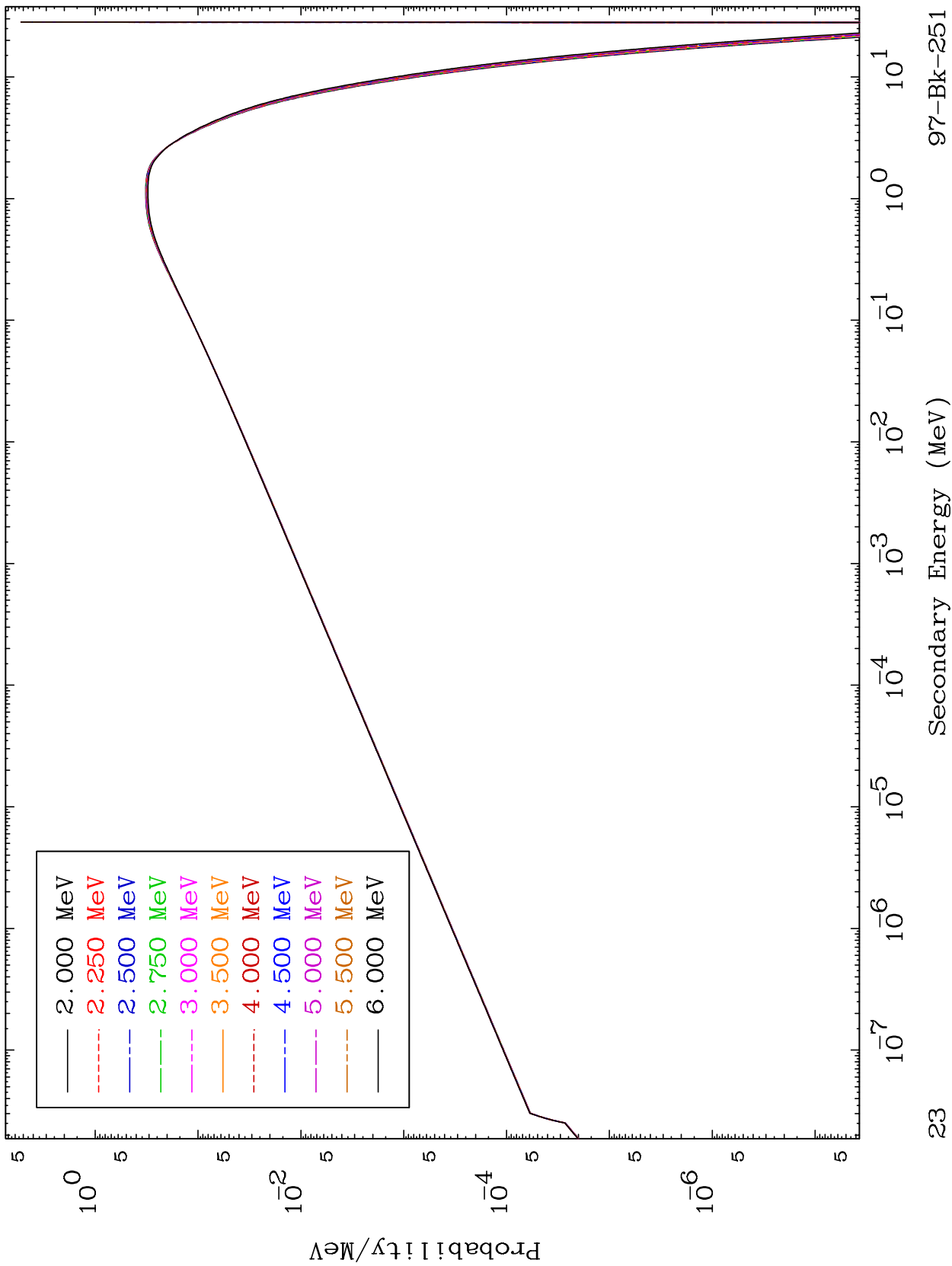


21

97-Bk-251

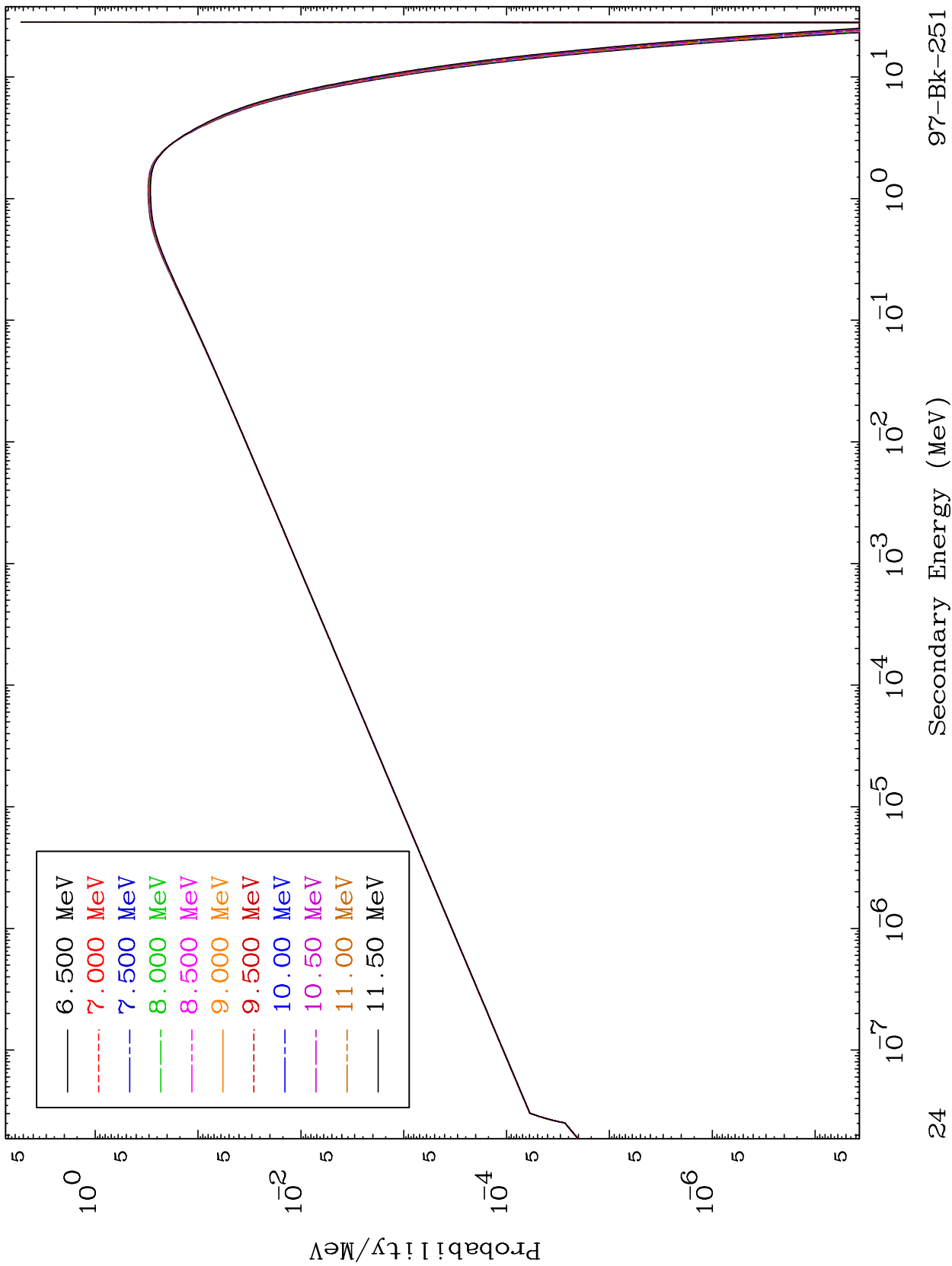
Incident Energy (MeV)

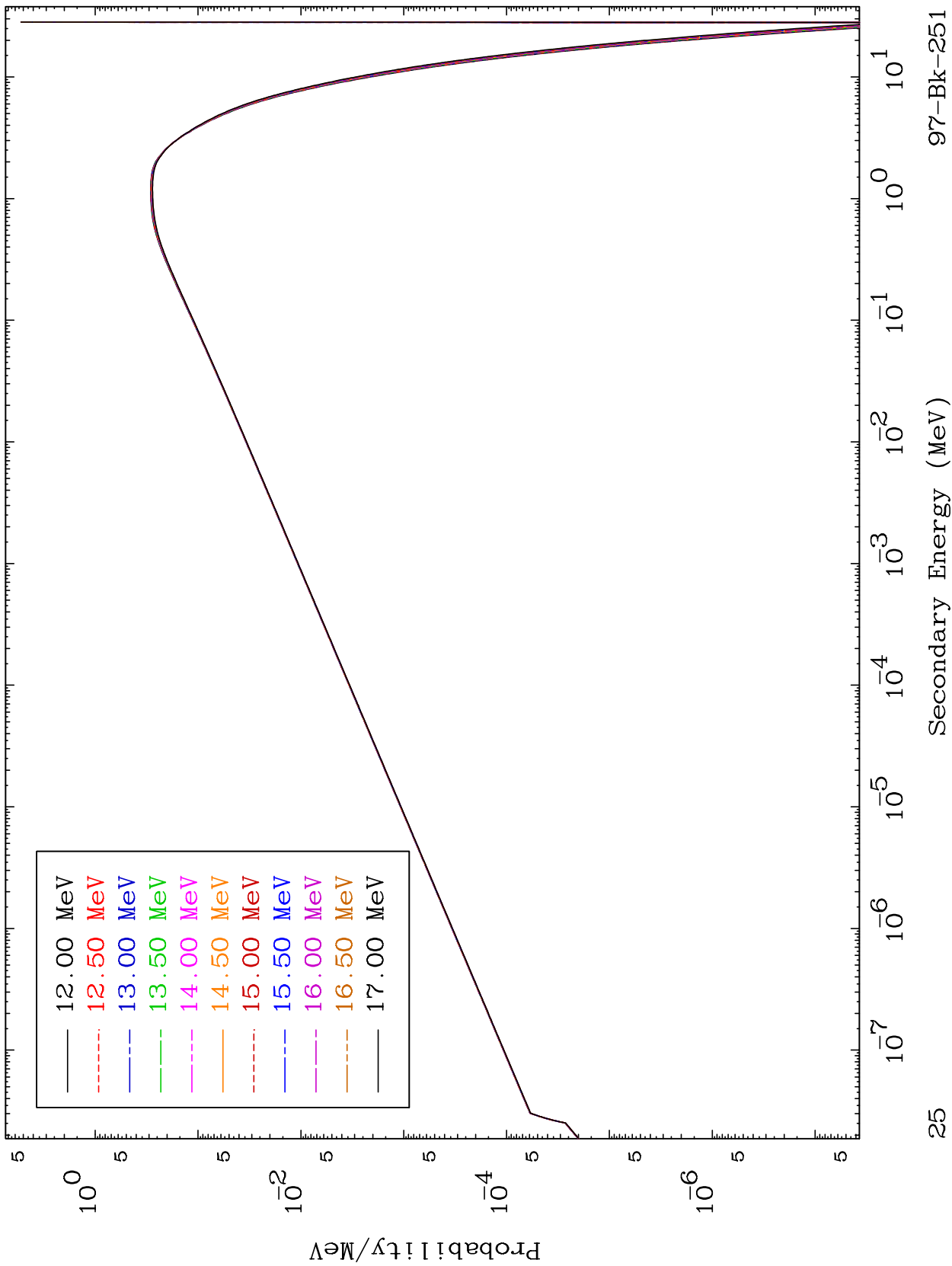




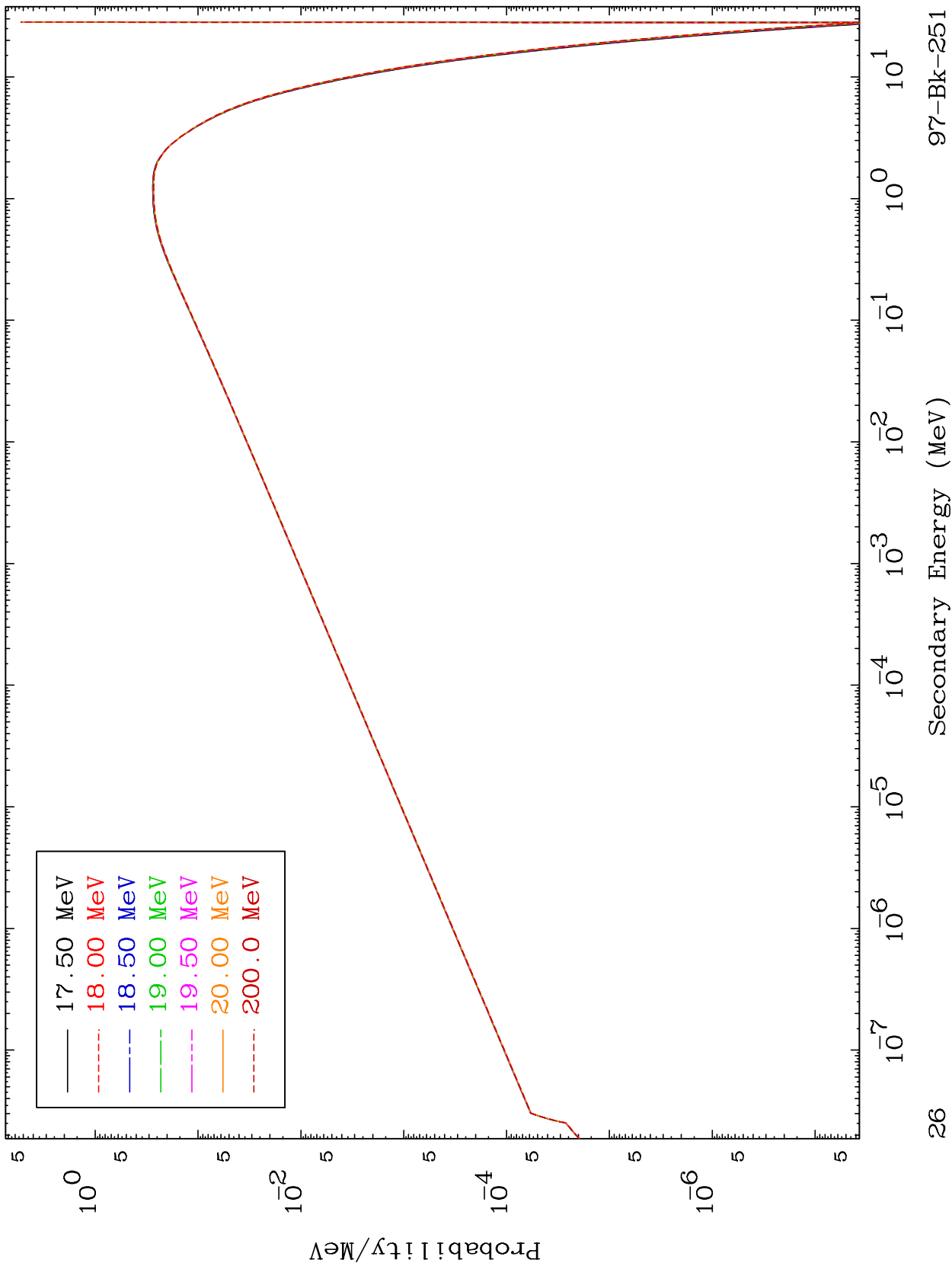


Fission  
Energy Distribution





Fission  
Energy Distribution

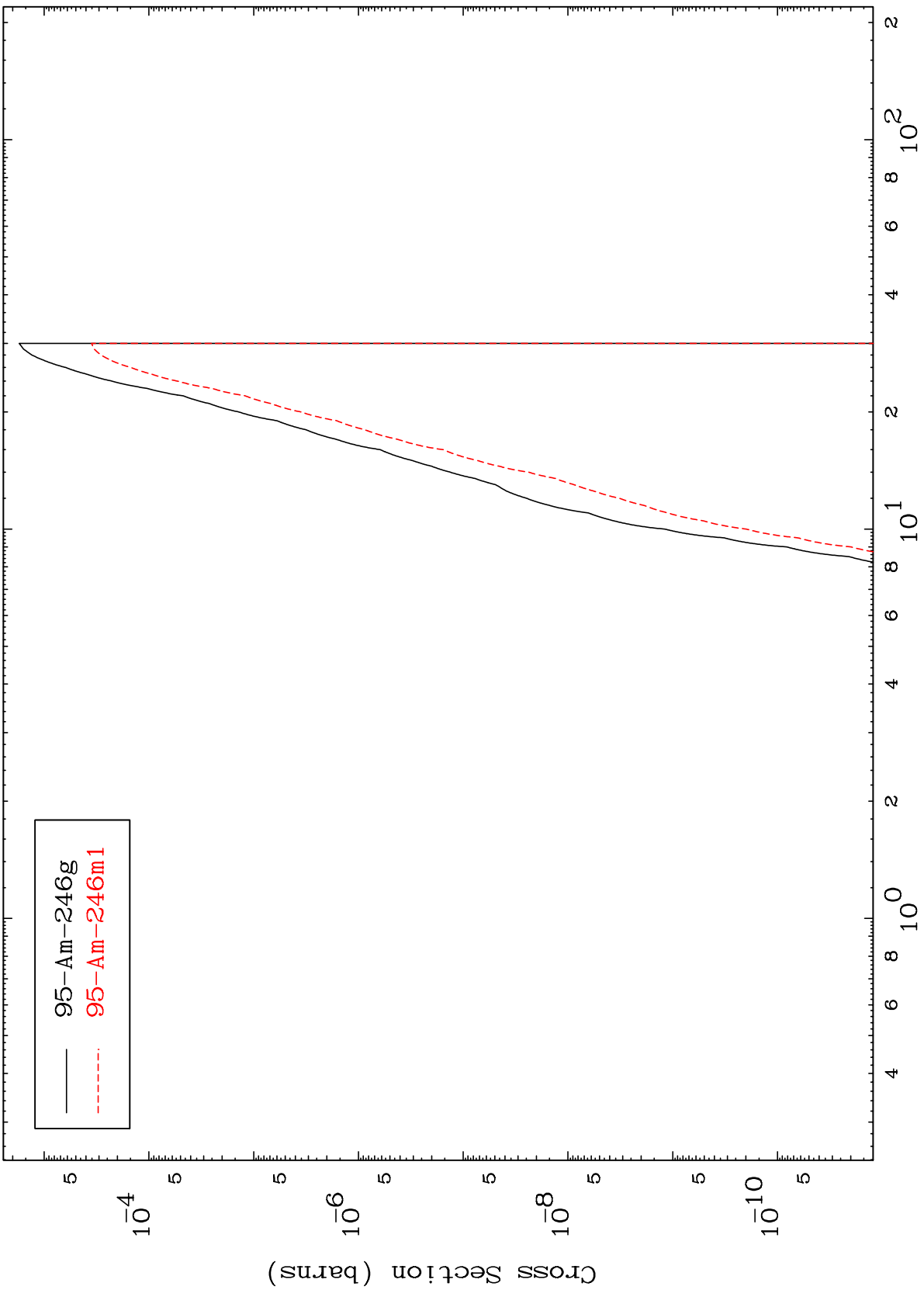


MAT 9758

(n,2n)  $\alpha$

97-Bk-251

Radionuclide Production Cross Section



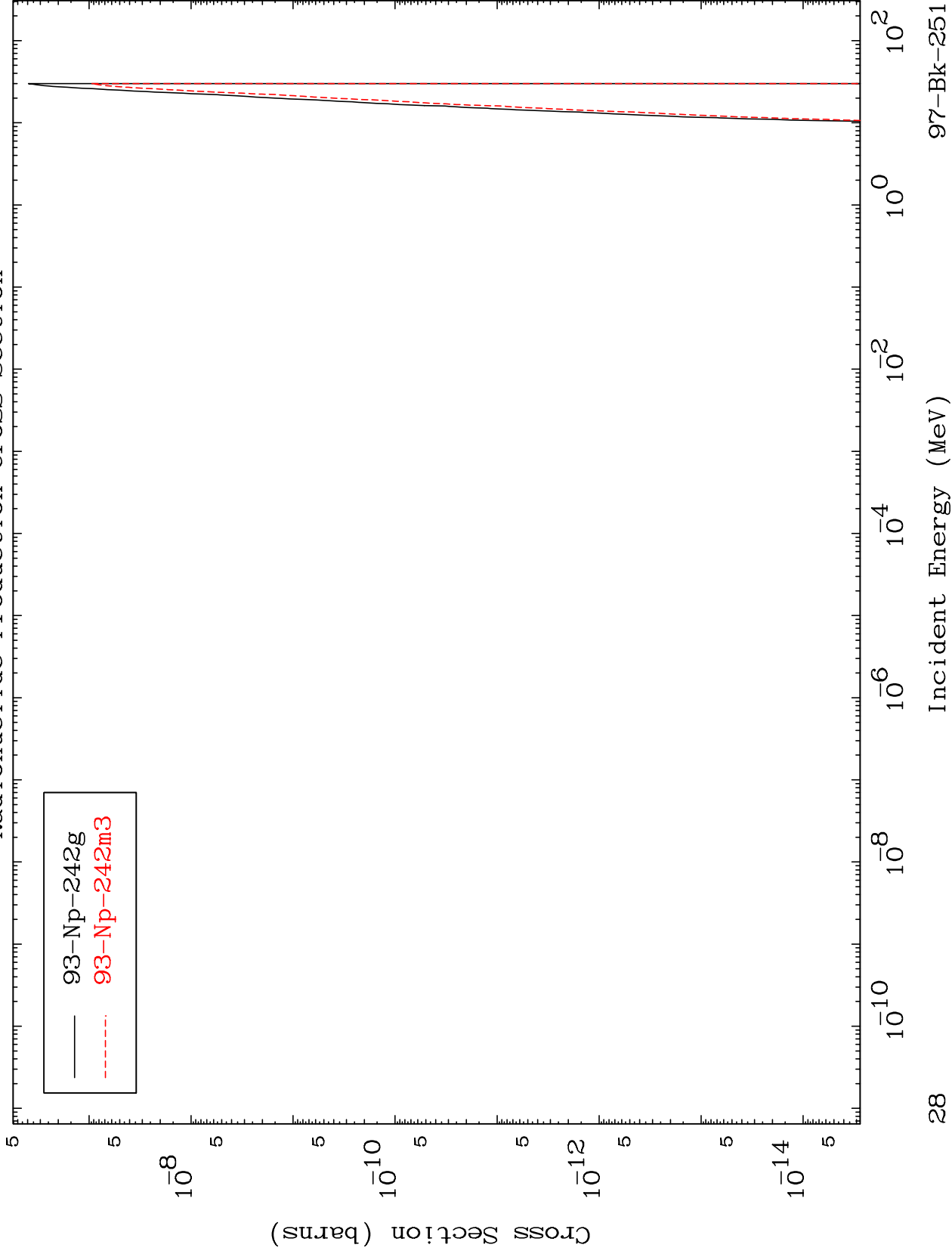
95-Am-246g  
95-Am-246m1

MAT 9758

(n,2n) 2 $\alpha$

97-Bk-251

Radionuclide Production Cross Section

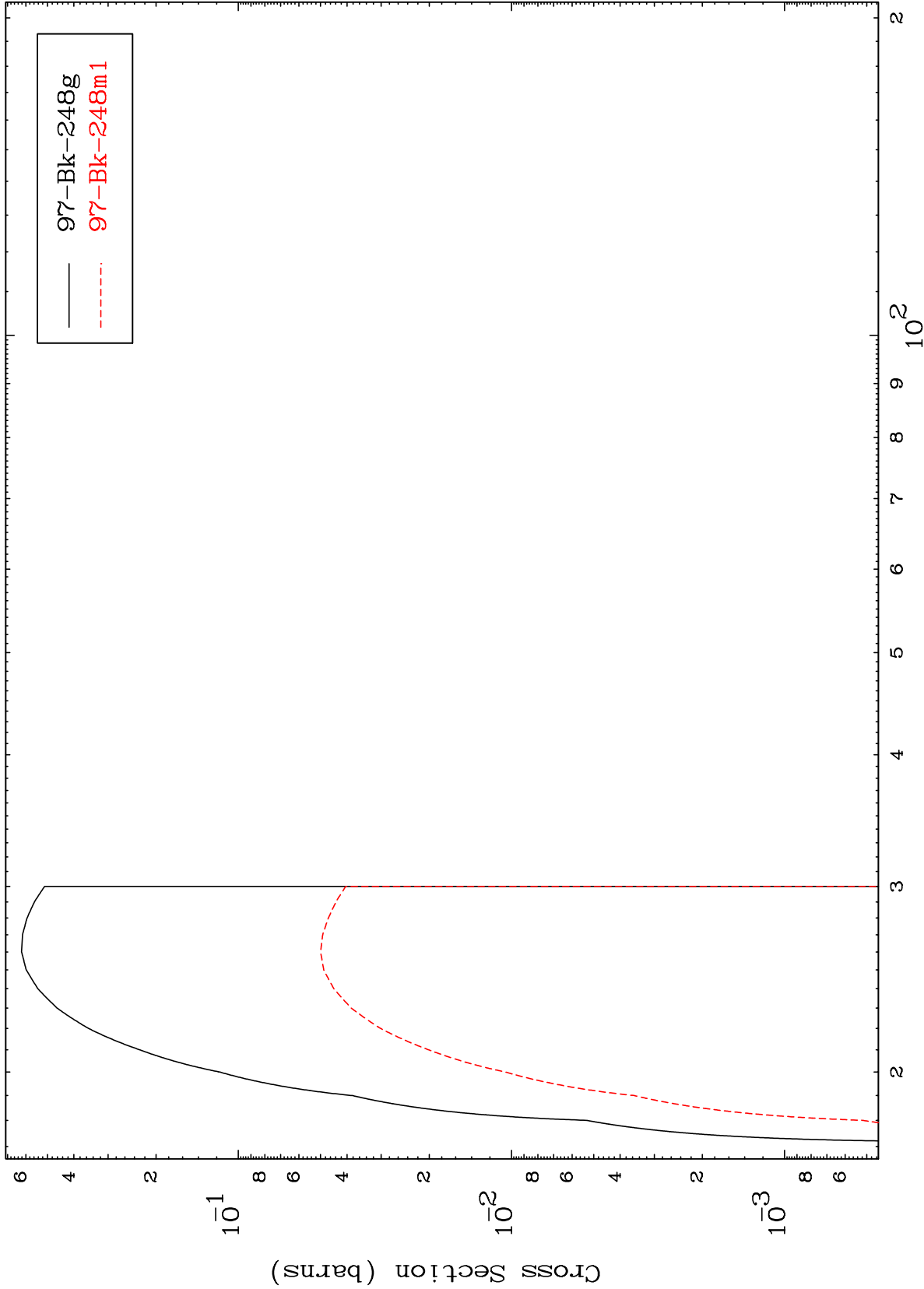


MAT 9758

(n,4n)

97-Bk-251

Radionuclide Production Cross Section



29

Incident Energy (MeV)

97-Bk-251