

Program EVALPLOT
(Version 2015-2)

by

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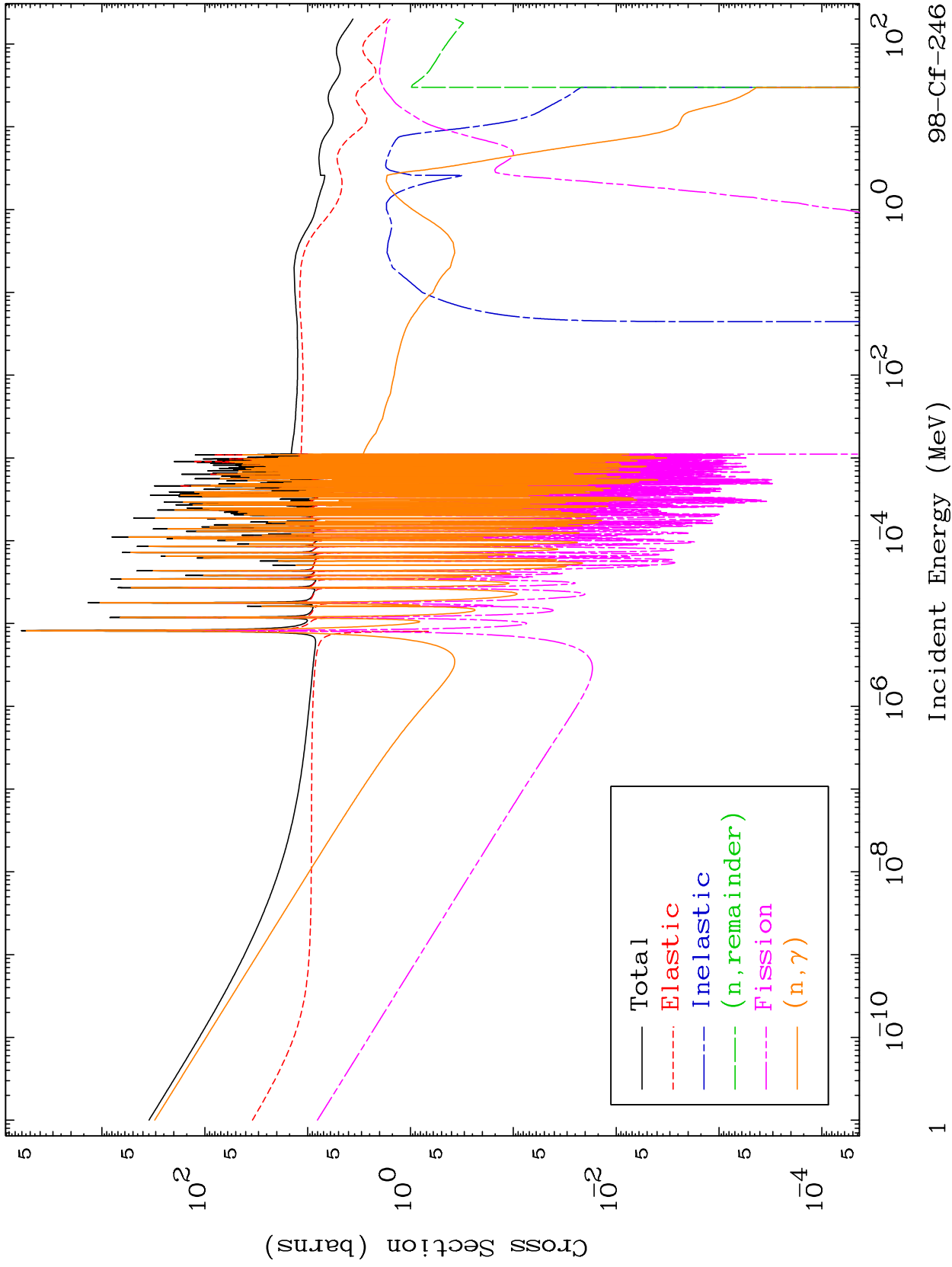
Press Mouse Button to Start

MAT 9843

Major

293 Kelvin Cross Sections

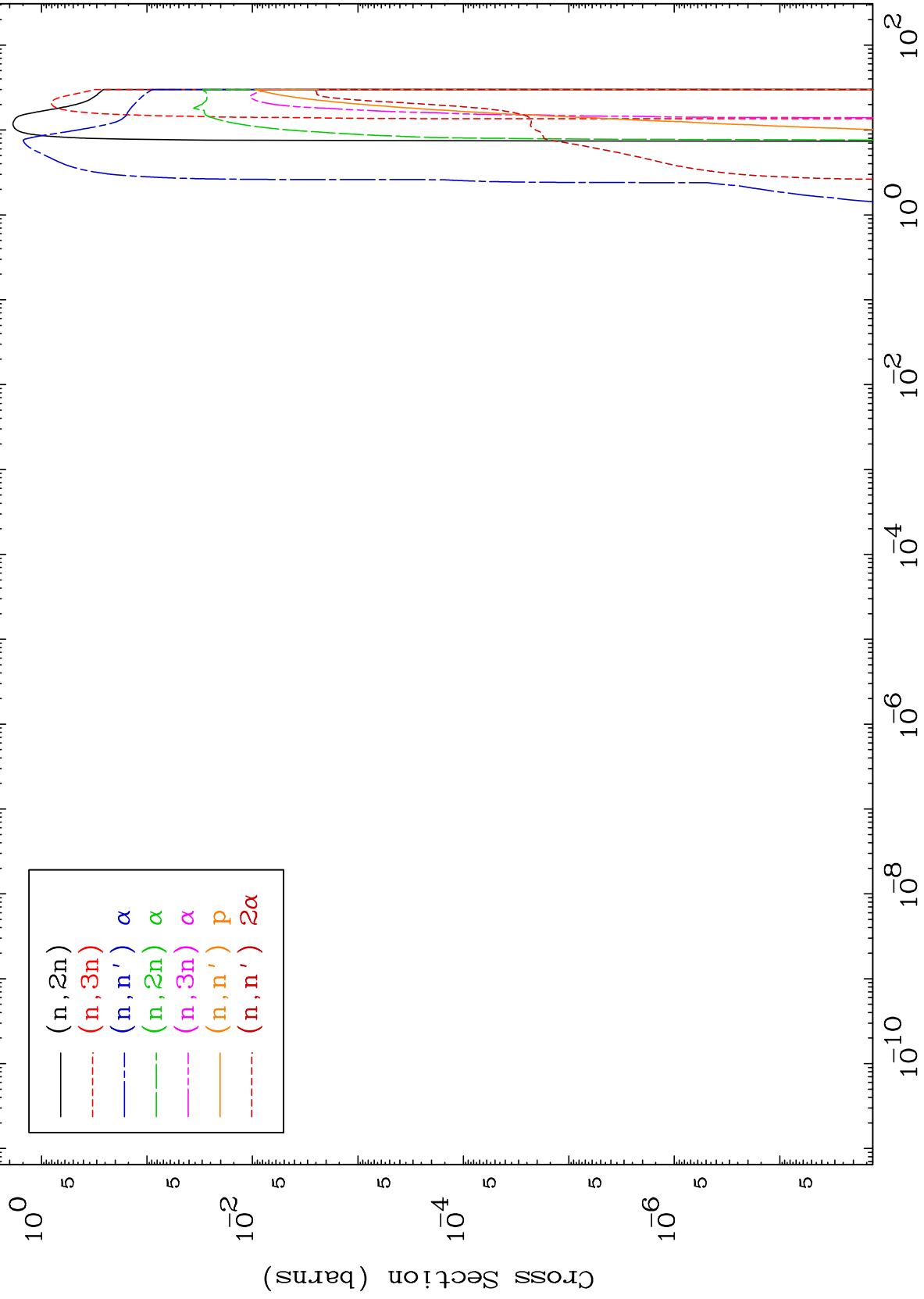
98-Cf-246



MAT 9843

Neutron Production
293 Kelvin Cross Sections

98-Cf-246



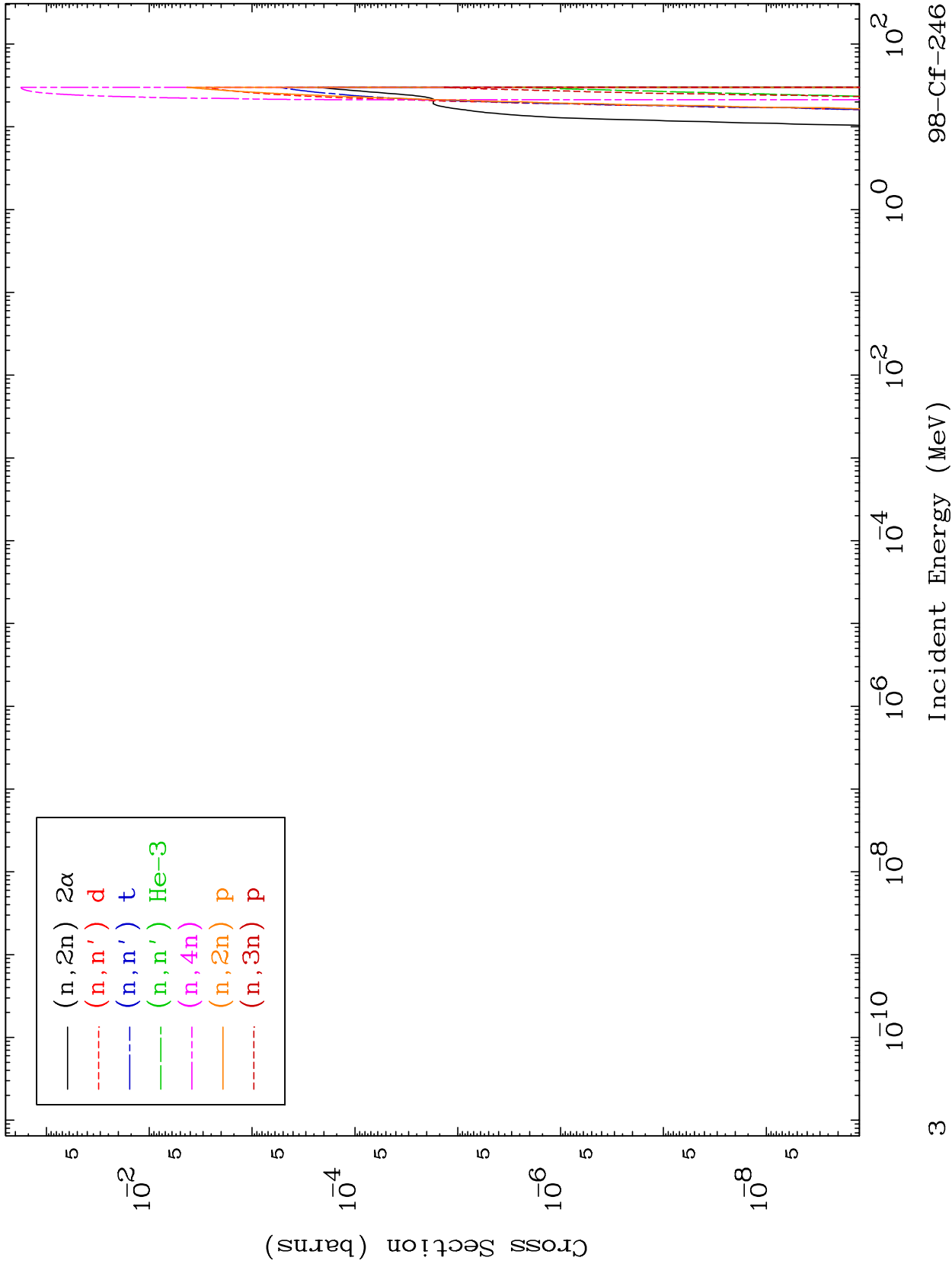
98-Cf-246

Incident Energy (MeV)

MAT 9843

Neutron Production
293 Kelvin Cross Sections

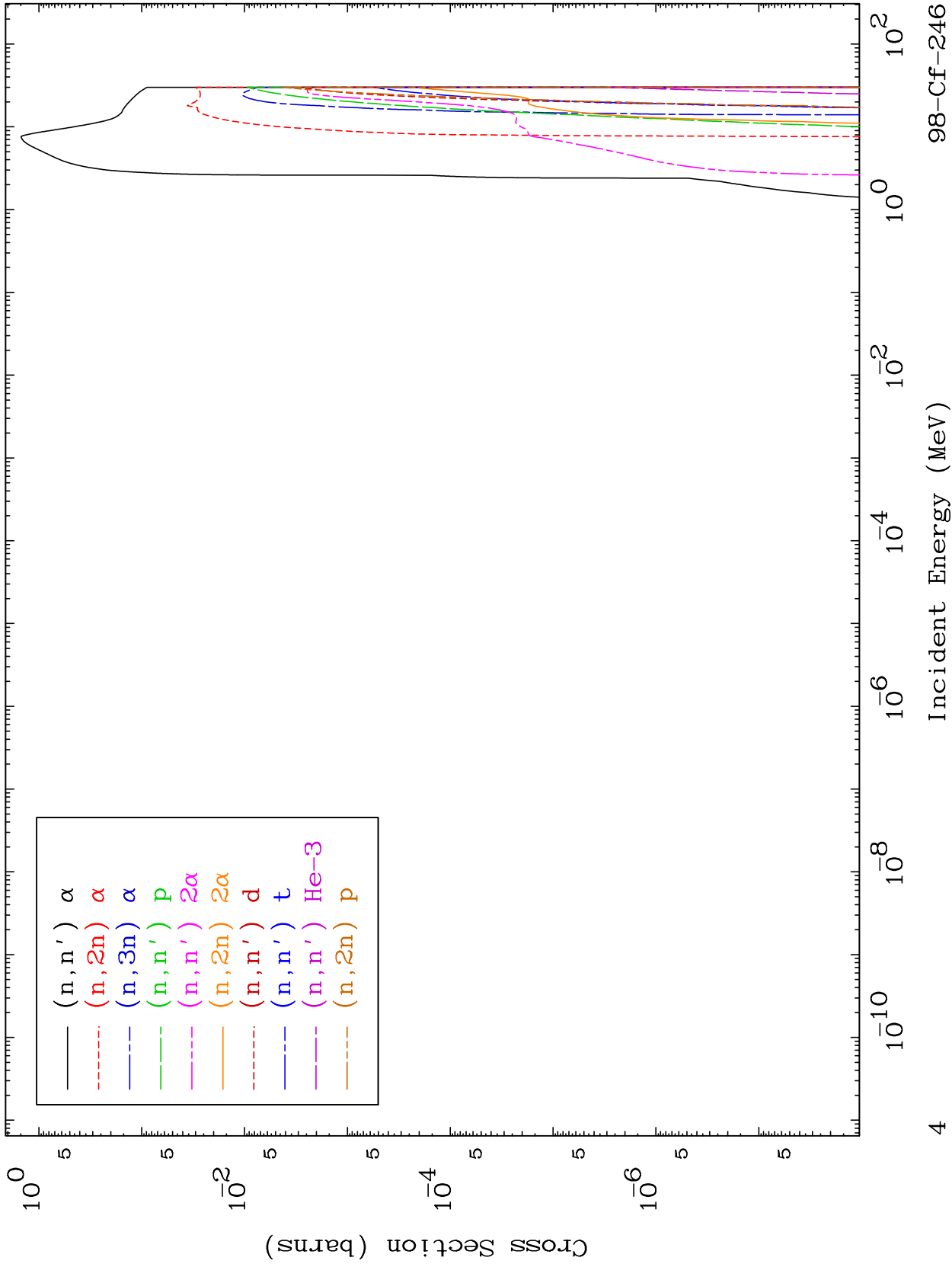
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MAT 9843

Charged Particle
293 Kelvin Cross Sections

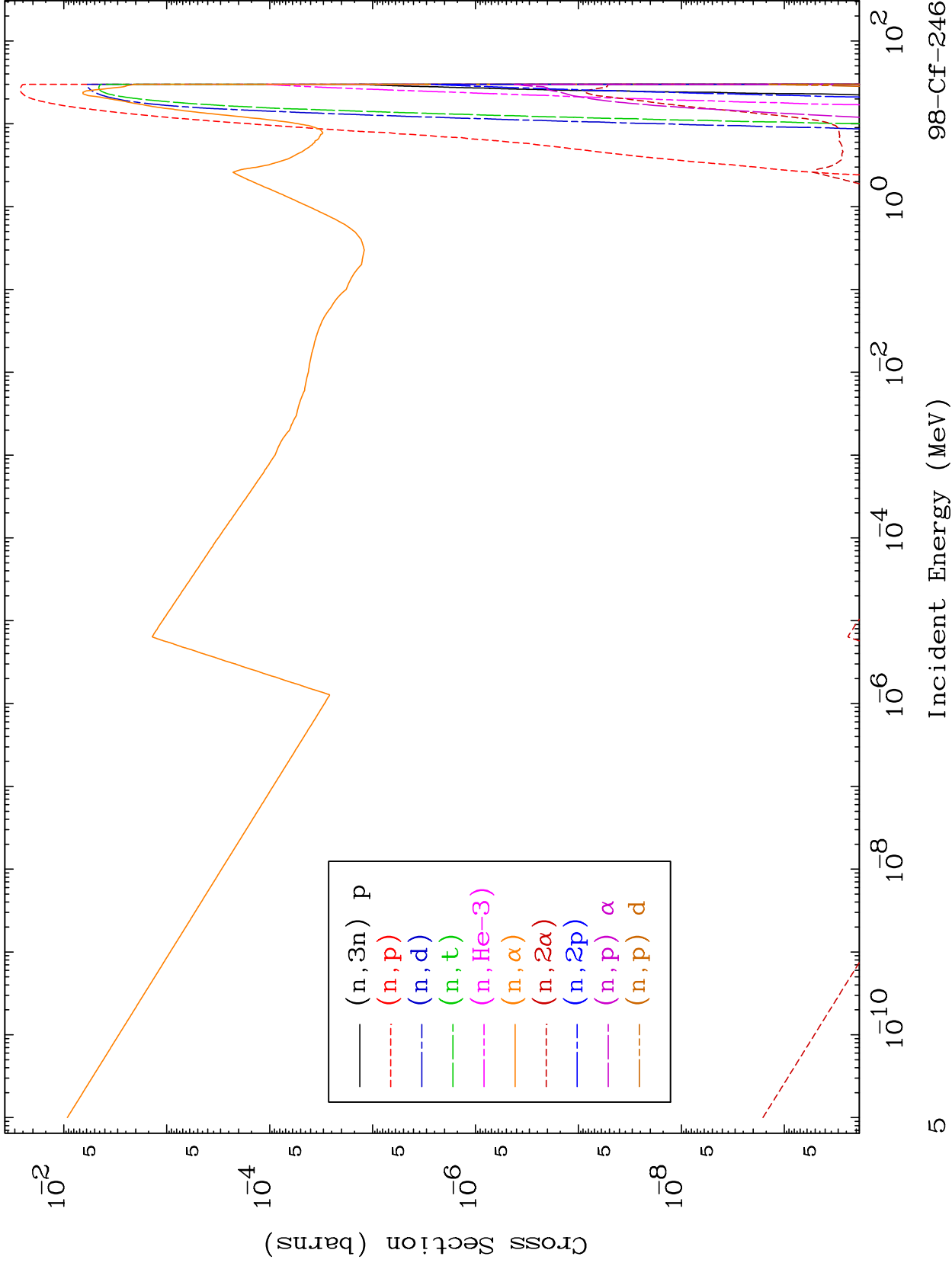
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MAT 9843

Charged Particle
293 Kelvin Cross Sections

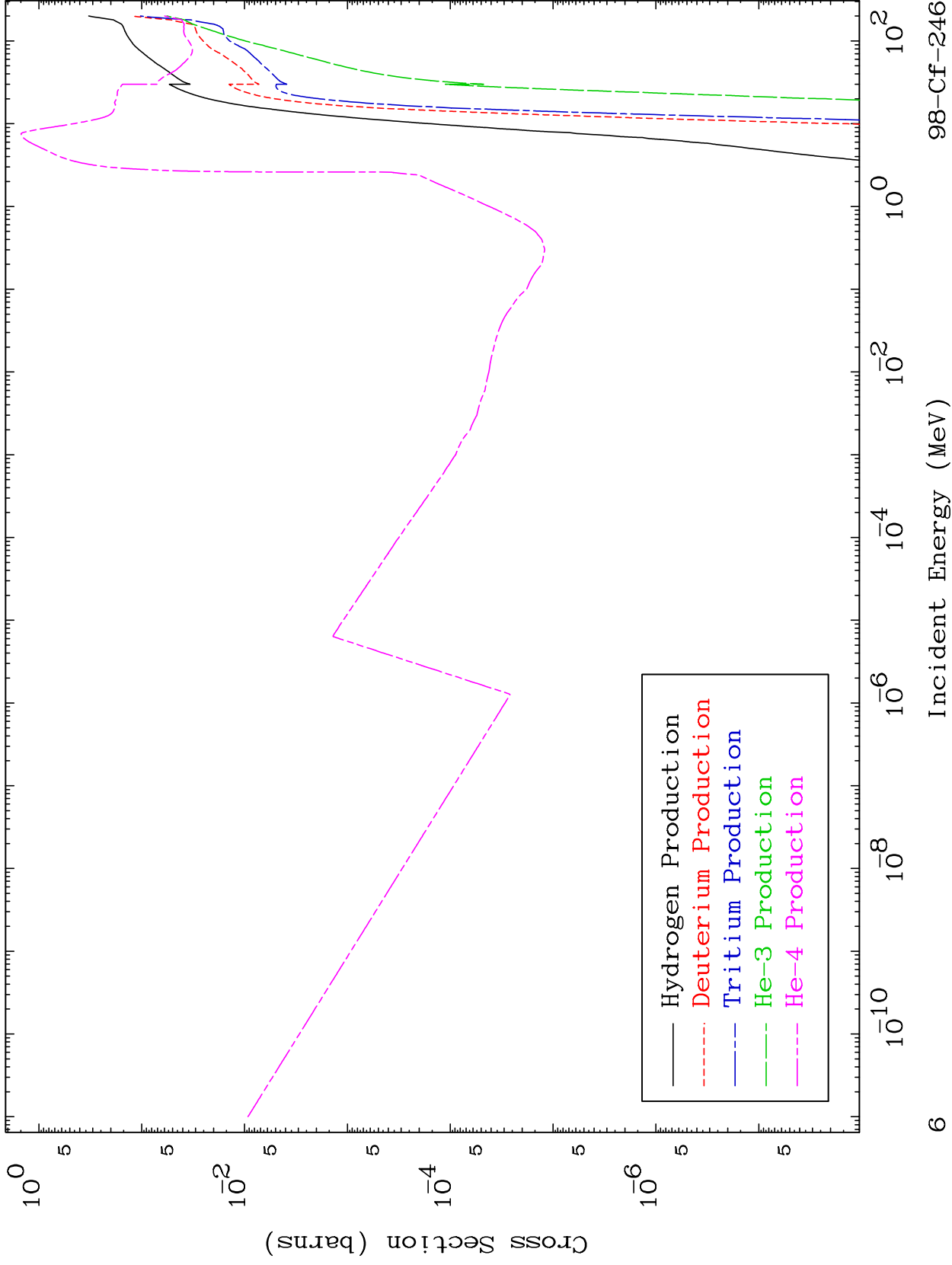
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MAT 9843

Particle Production
293 Kelvin Cross Sections

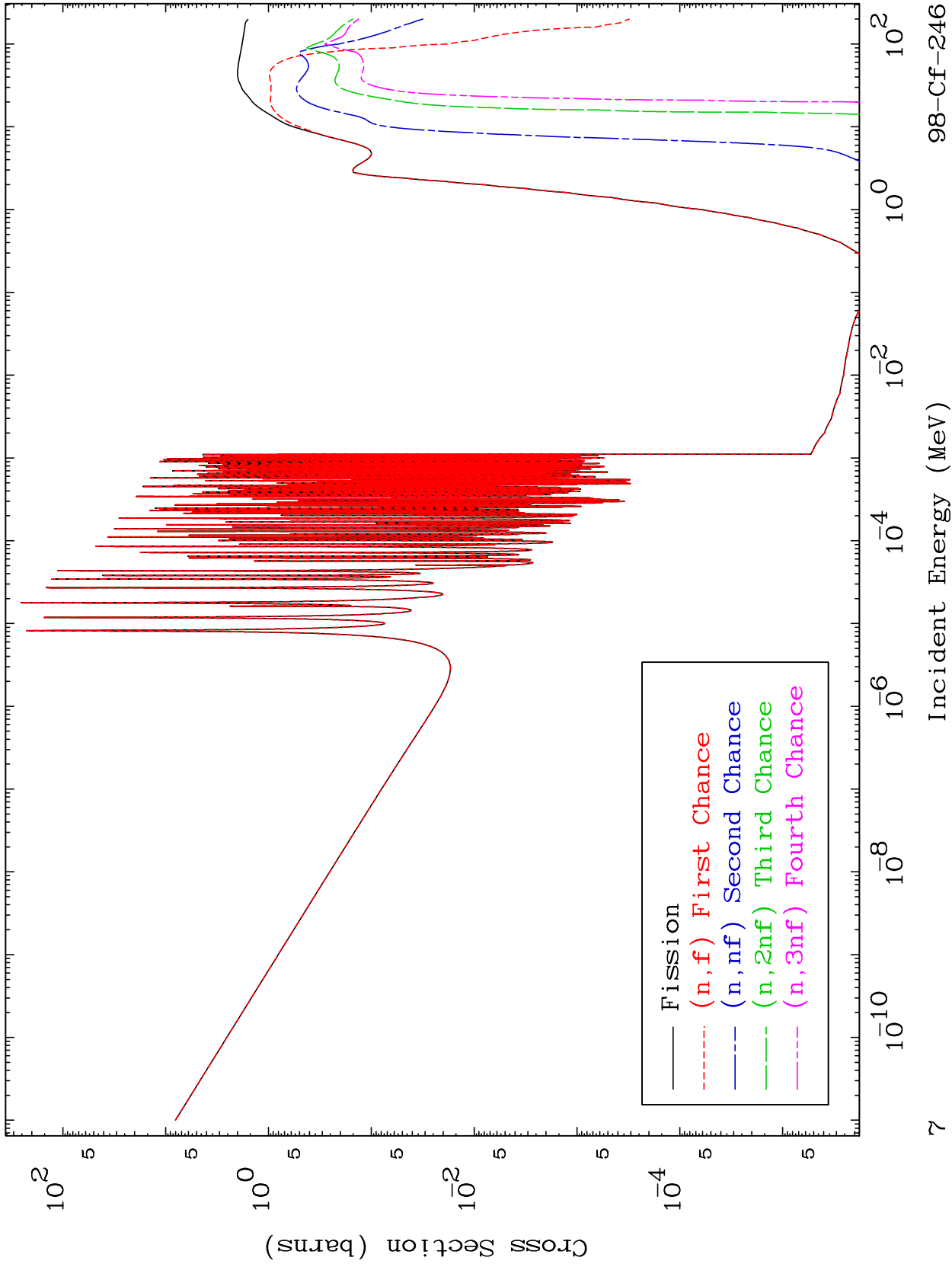
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MAT 9843

Fission
293 Kelvin Cross Sections

98-Cf-246

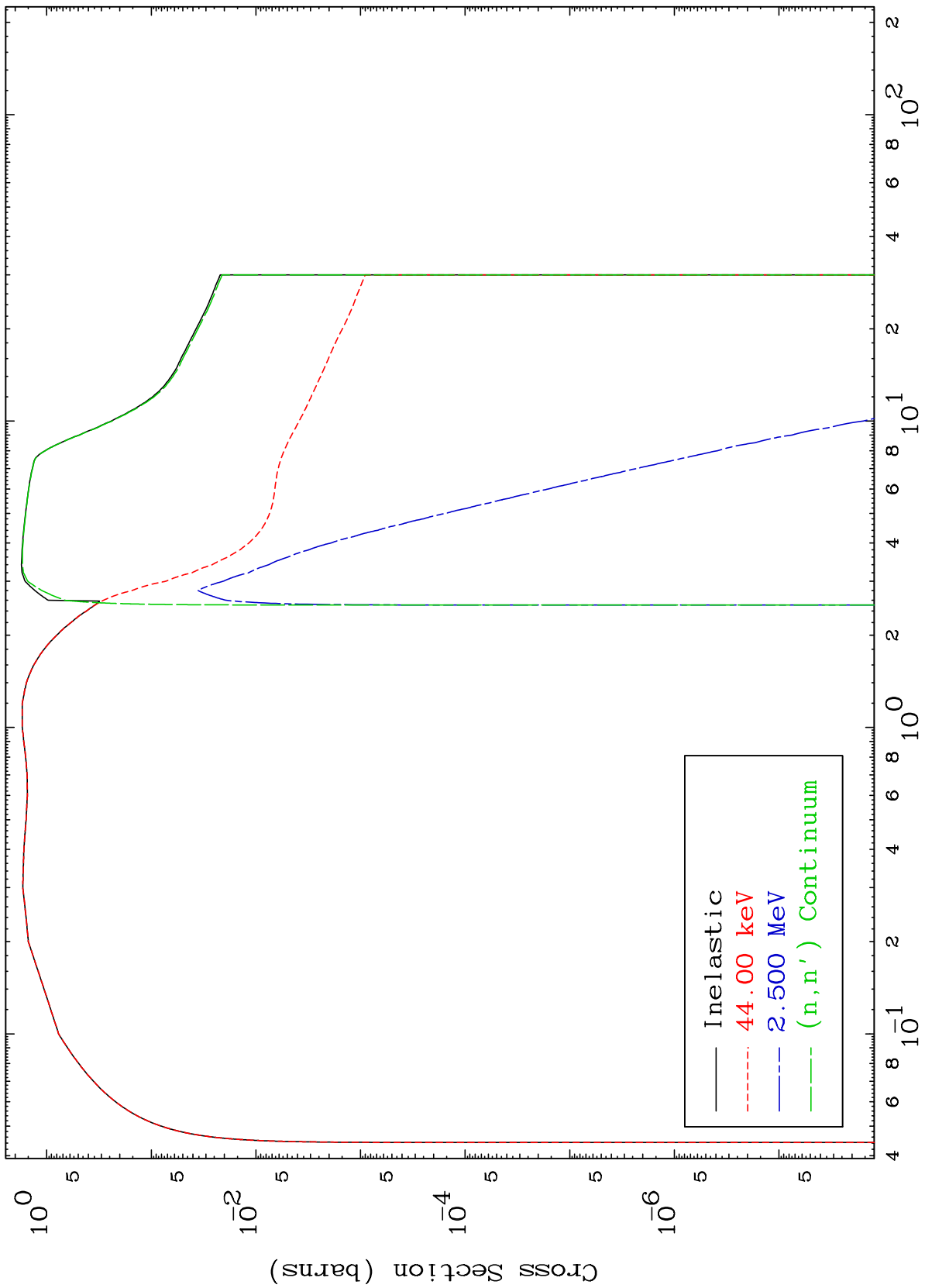


MAT 9843

(n,n') Level

98-Cf-246

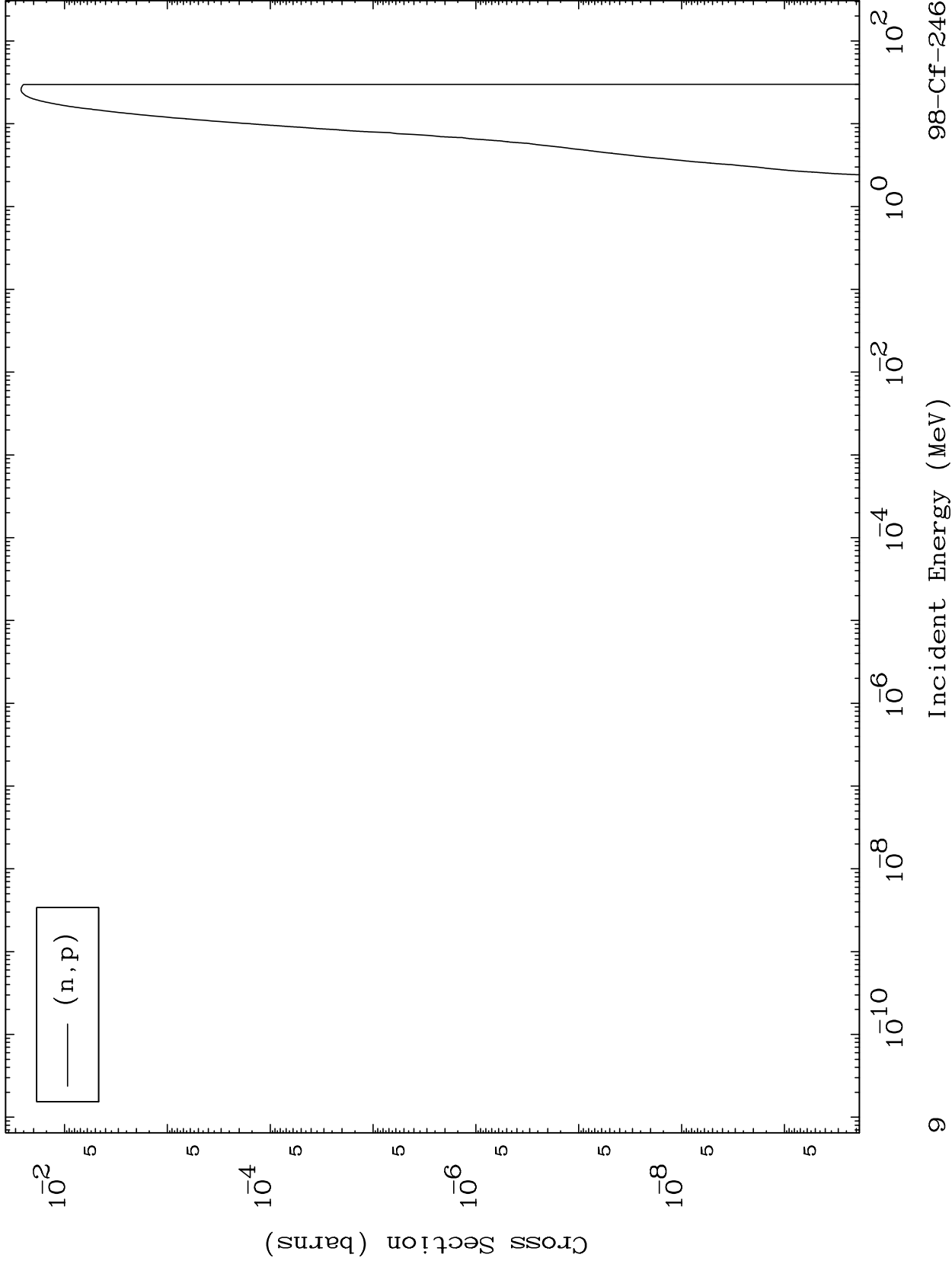
293 Kelvin Cross Sections



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(n,p) Levels
293 Kelvin Cross Sections

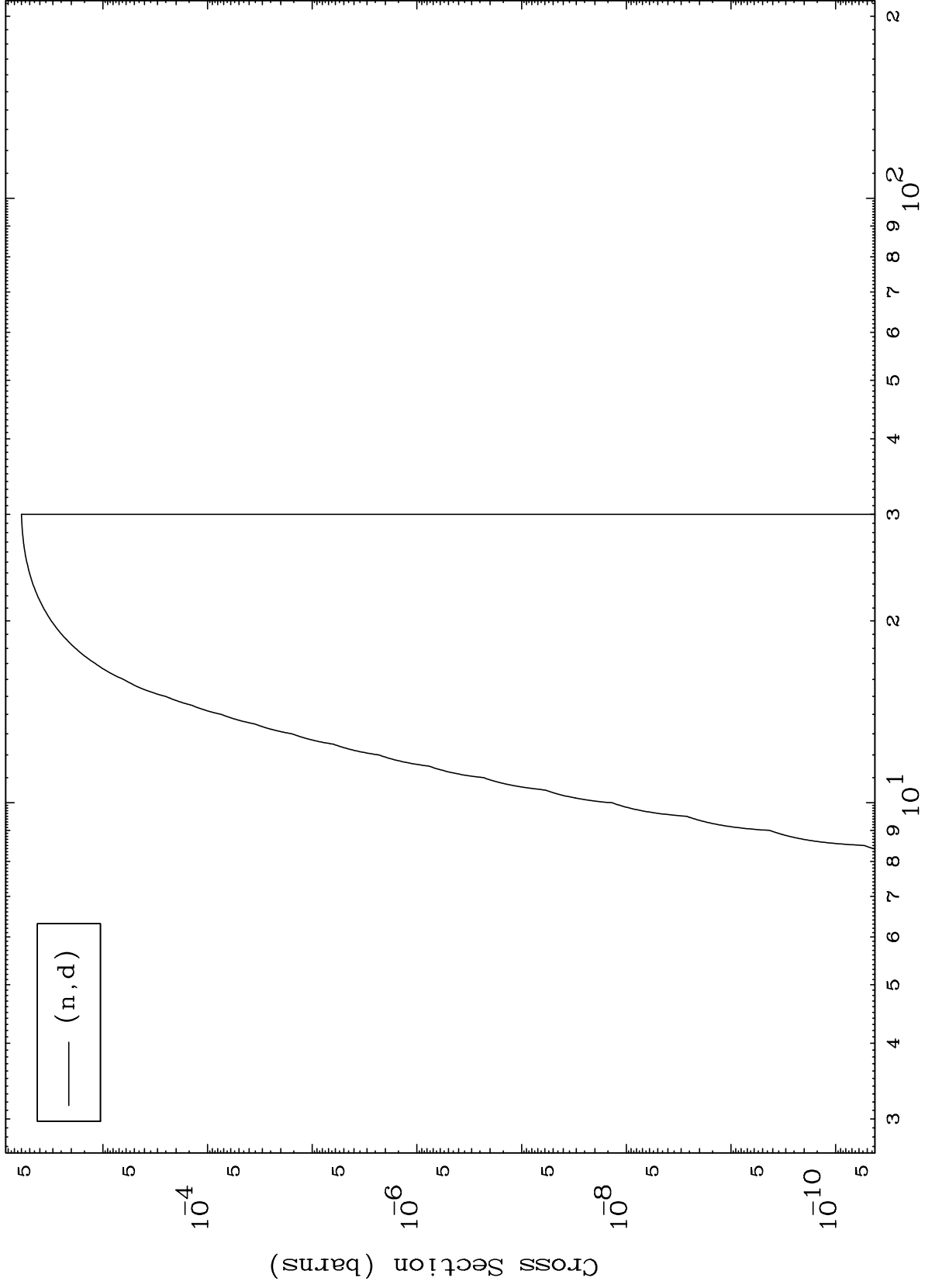
98-Cf-246



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(n,d) Levels
293 Kelvin Cross Sections

98-Cf-246



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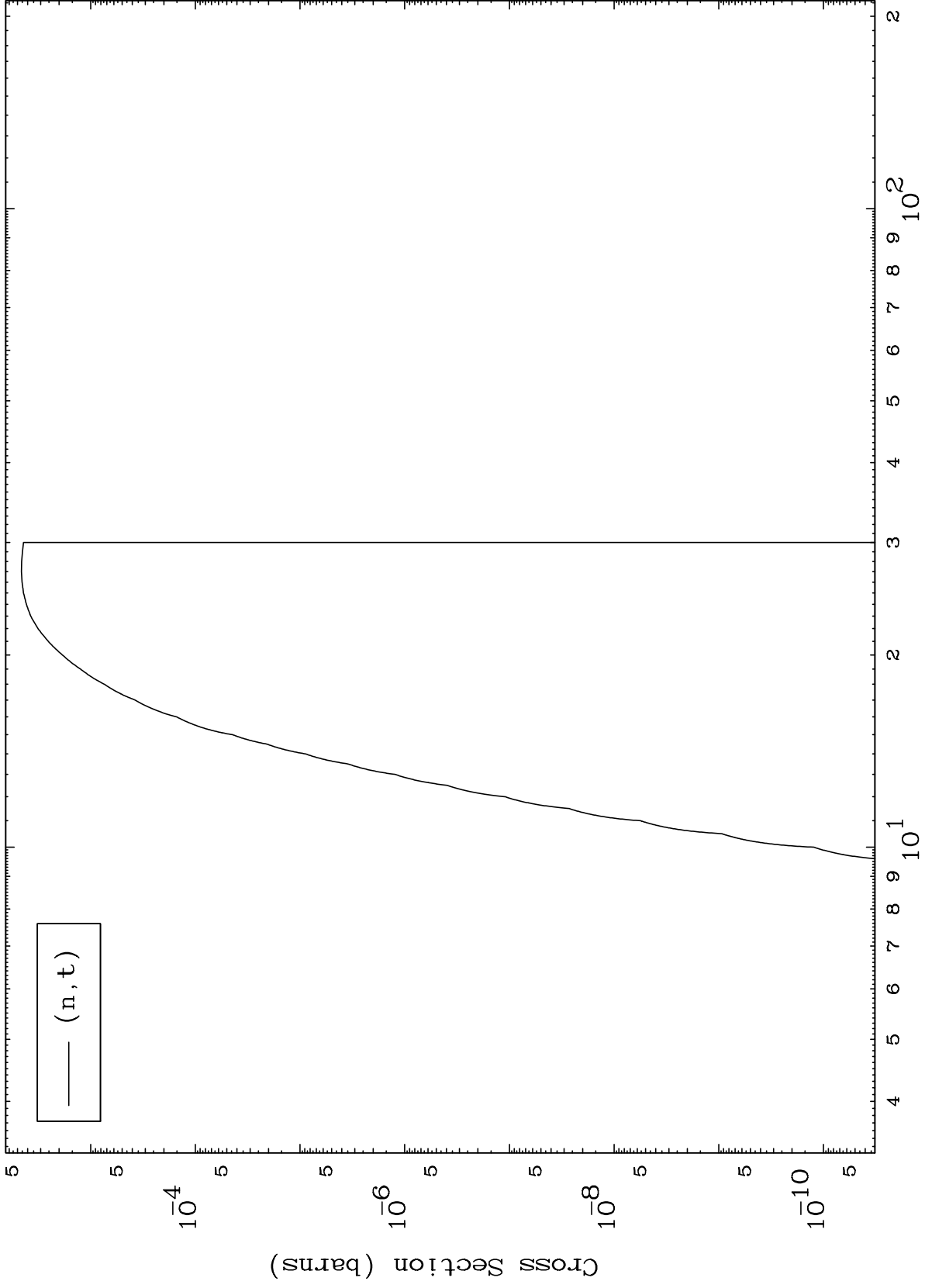
Incident Energy (MeV)

98-Cf-246

MAT 9843

(n,t) Levels
293 Kelvin Cross Sections

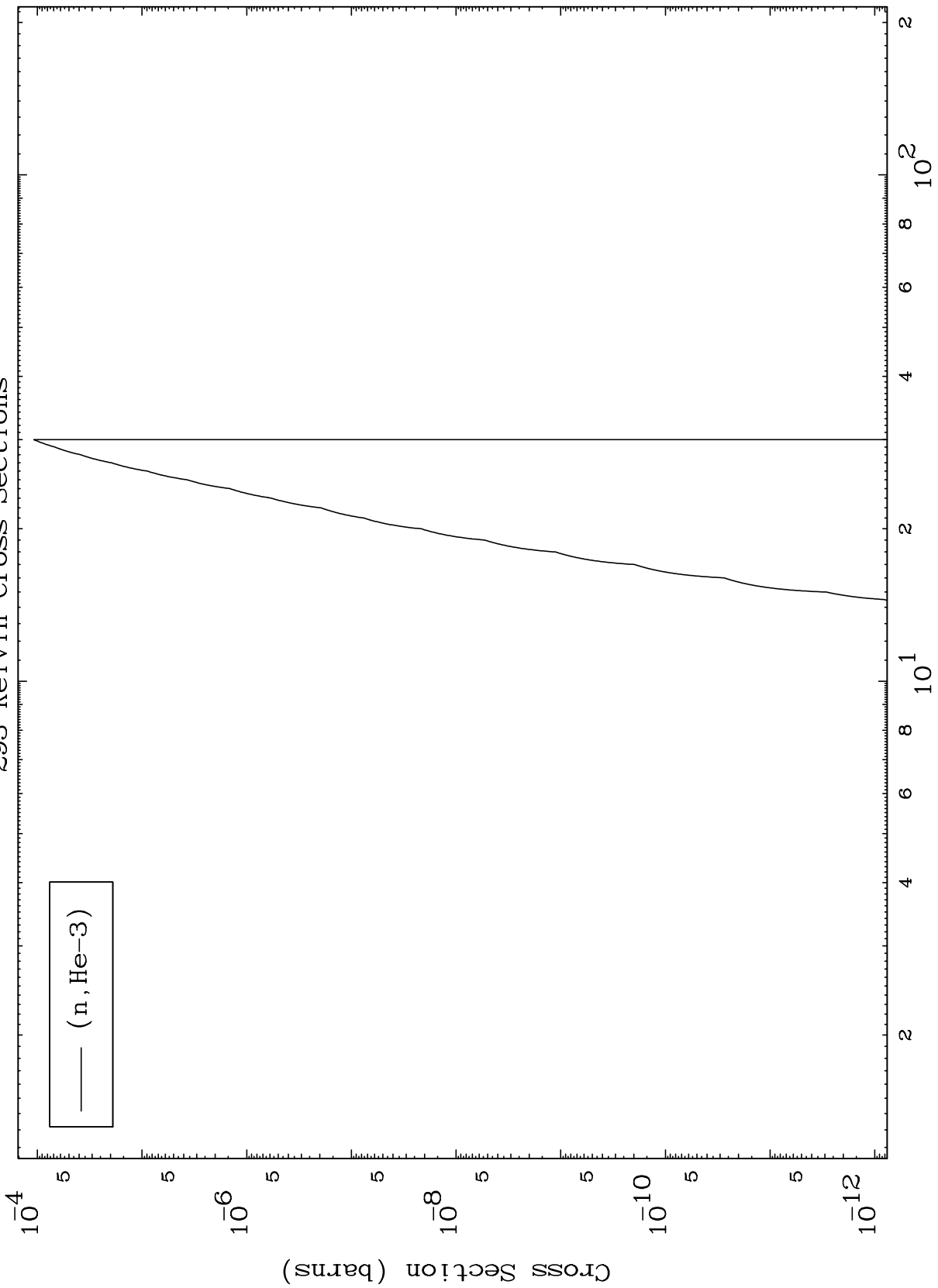
98-Cf-246



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(n,He3) Levels
293 Kelvin Cross Sections

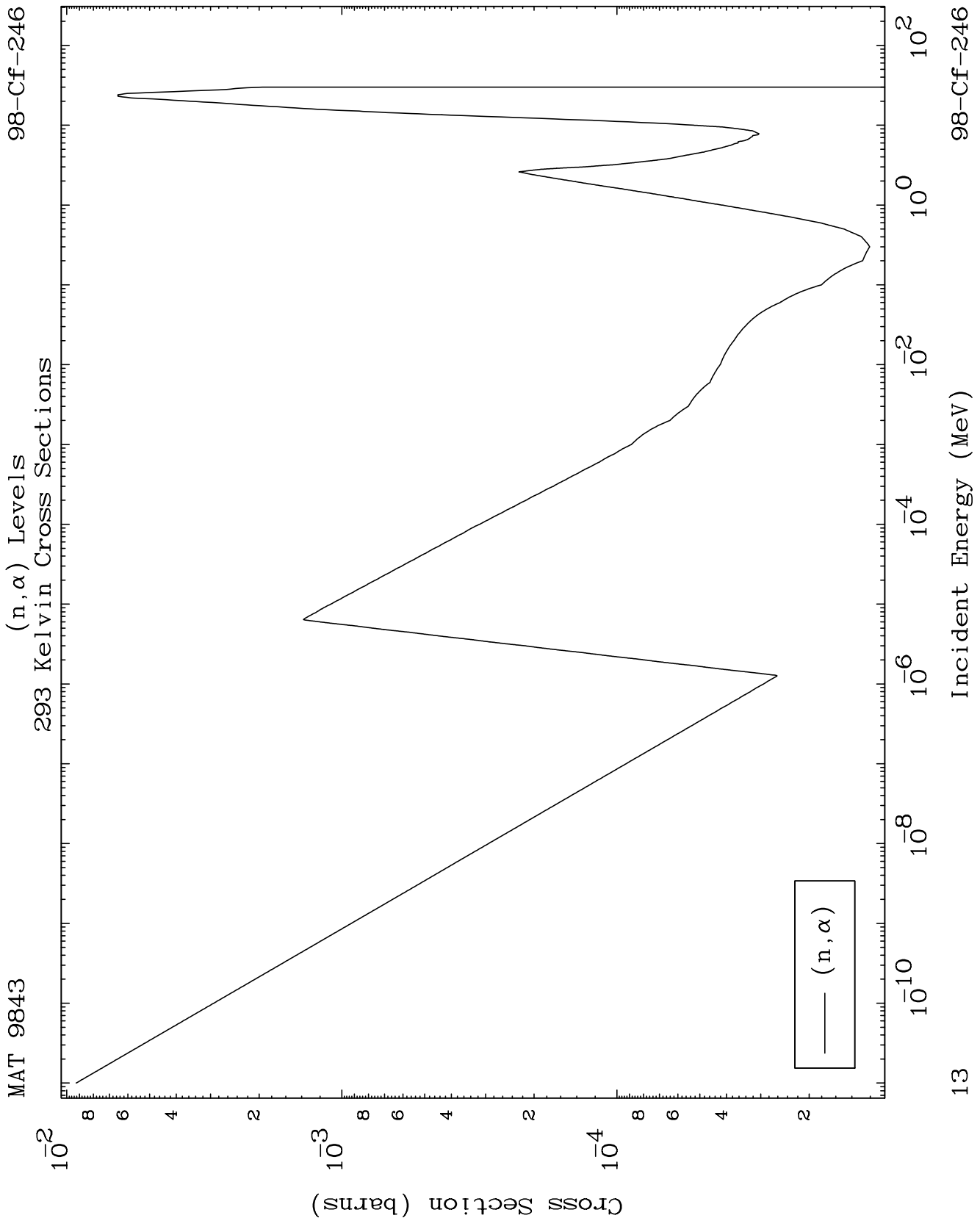
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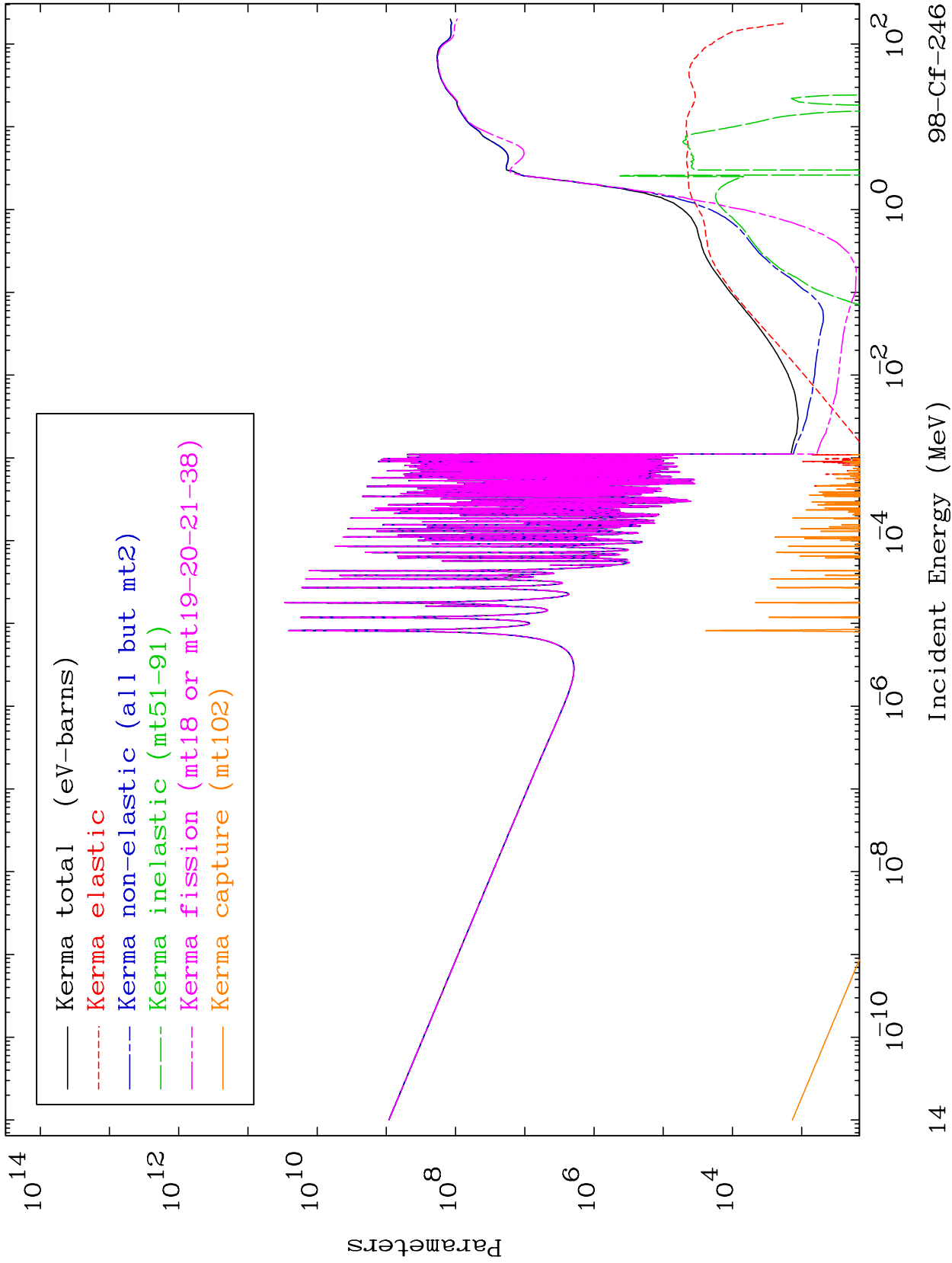


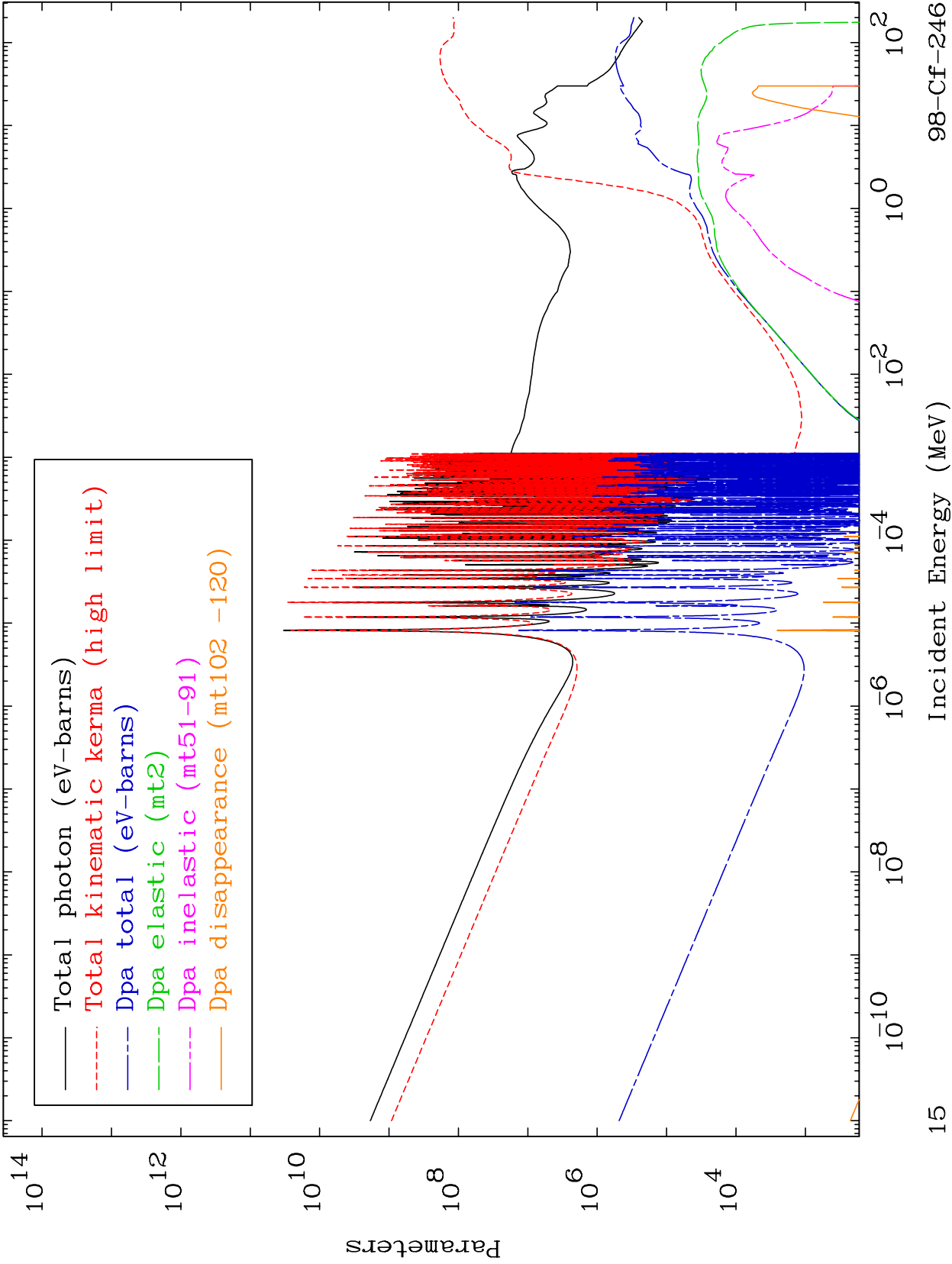
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Incident Energy (MeV)

98-Cf-246



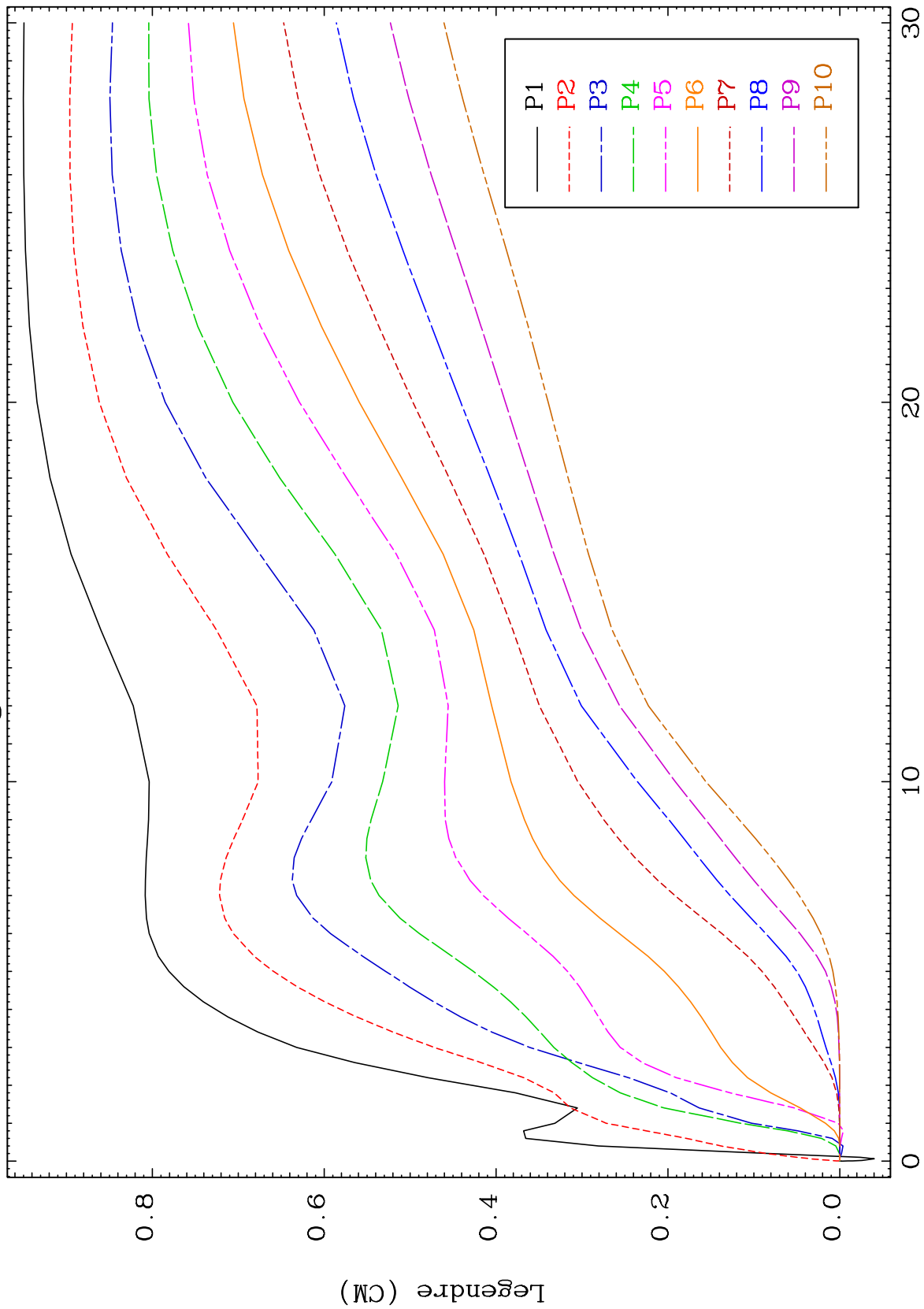




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Elastic Legendre Coefficients

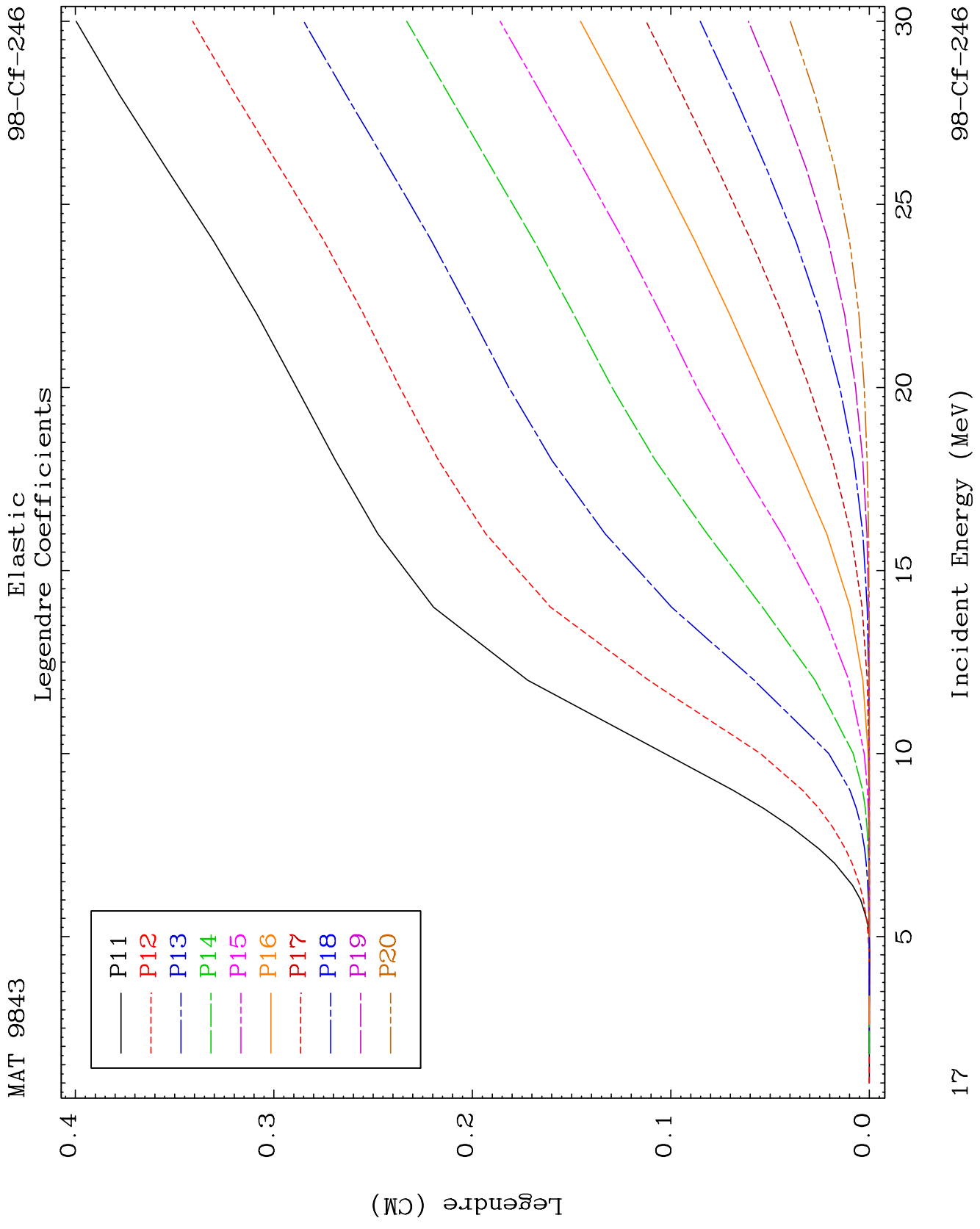
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Incident Energy (MeV)

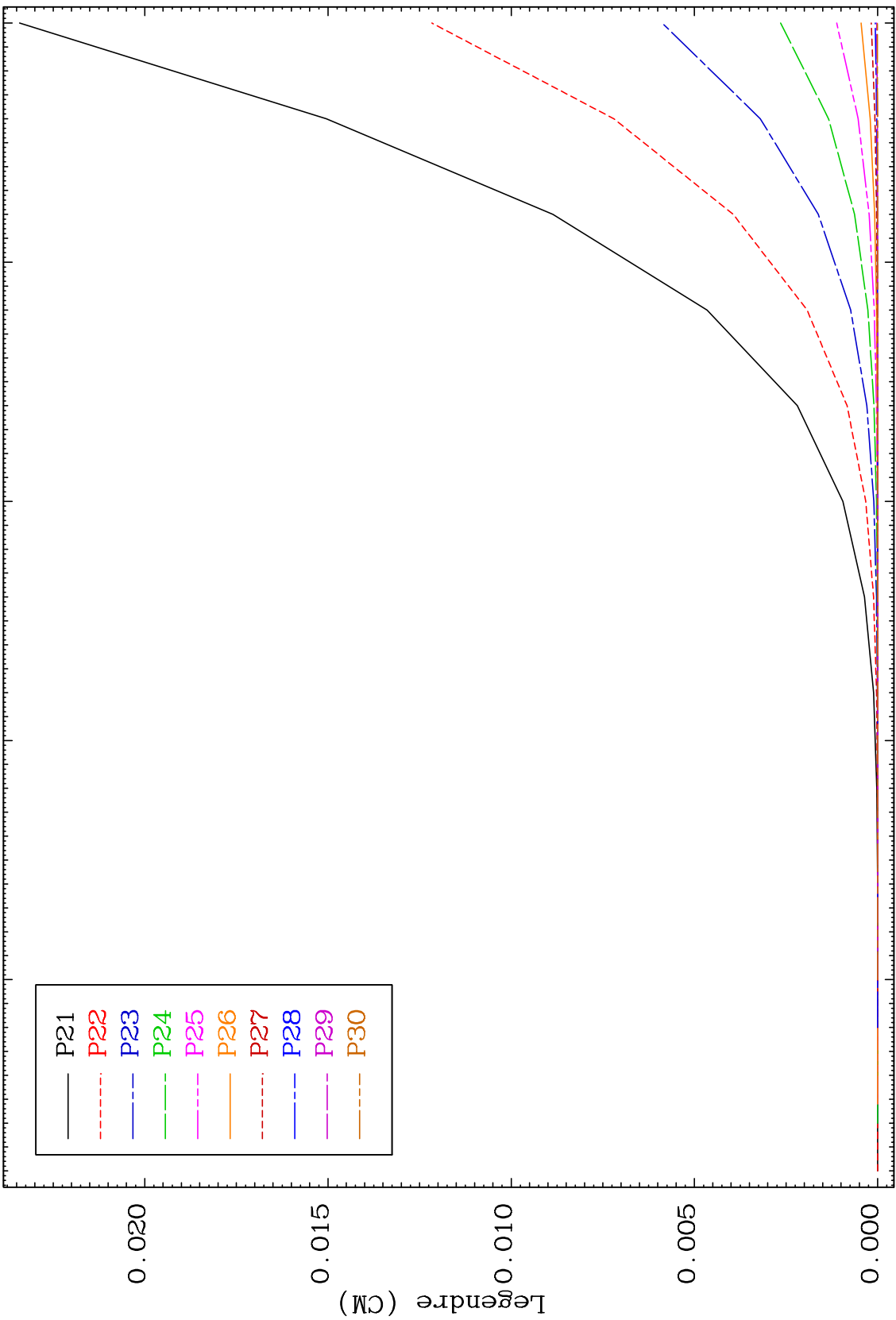
98-Cf-246



MAT 9843

Elastic Legendre Coefficients

98-Cf-246



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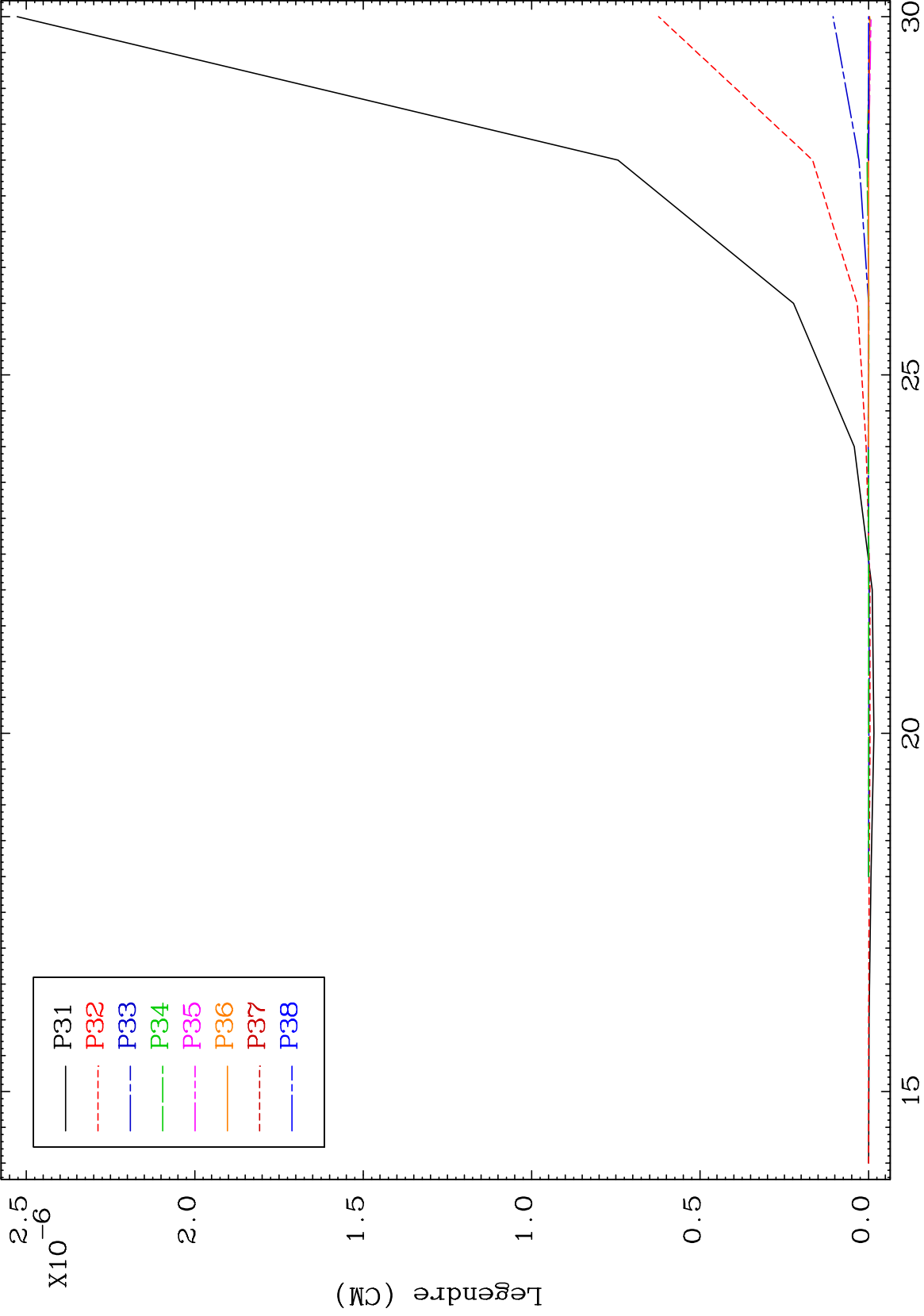
Incident Energy (MeV)

98-Cf-246

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Elastic
Legendre Coefficients

98-Cf-246

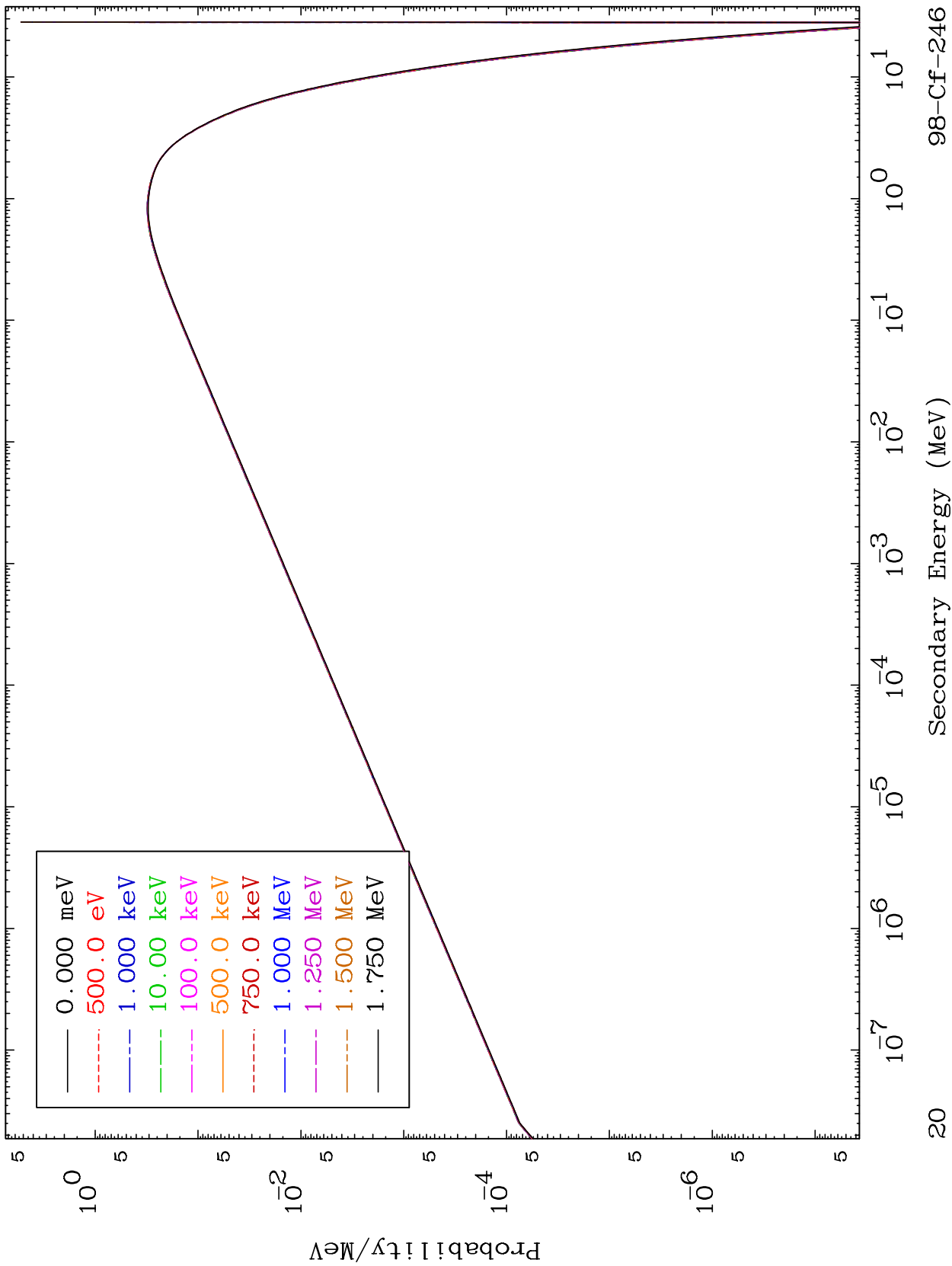


19

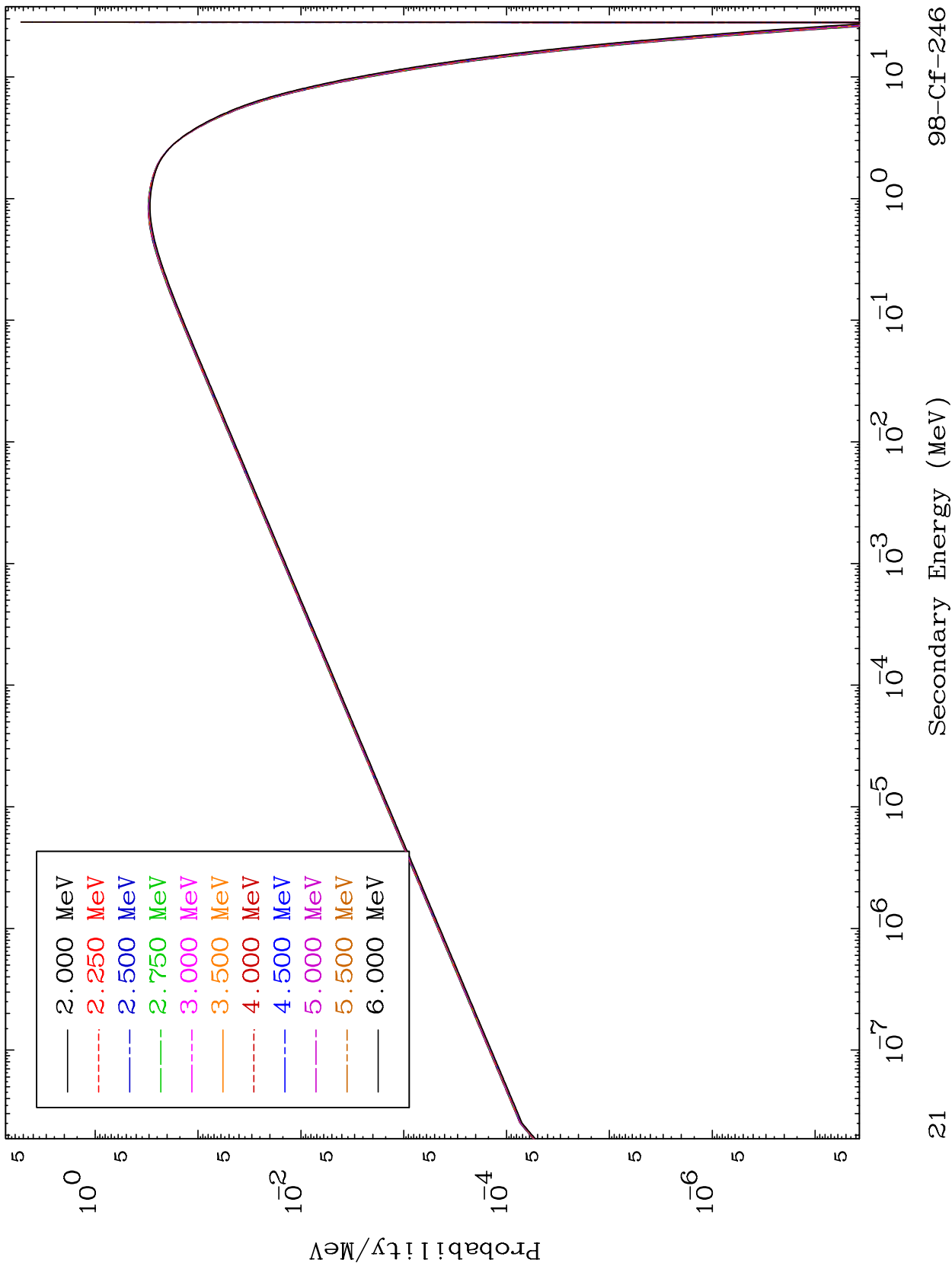
Incident Energy (MeV)

98-Cf-246

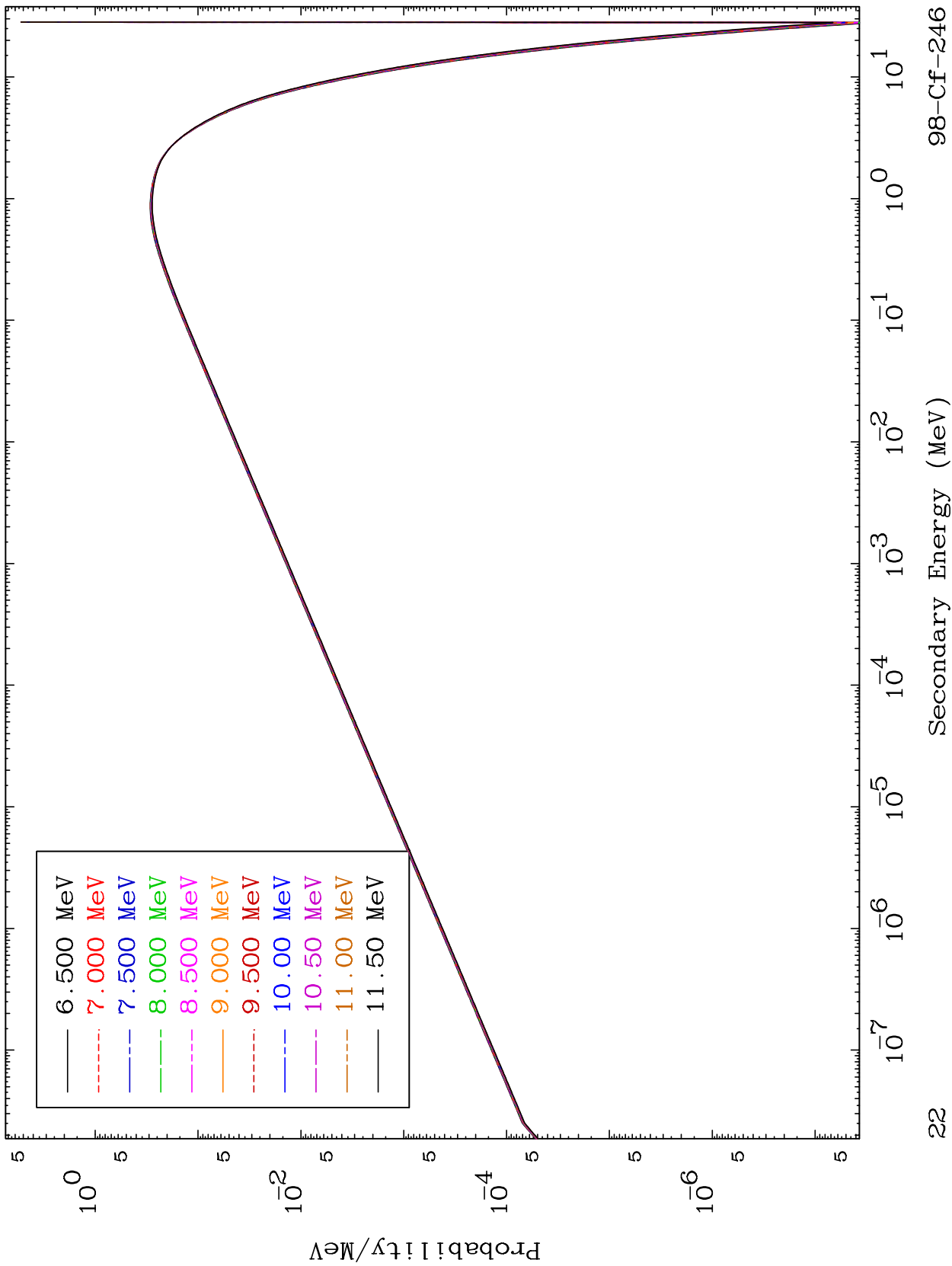
Fission
Energy Distribution

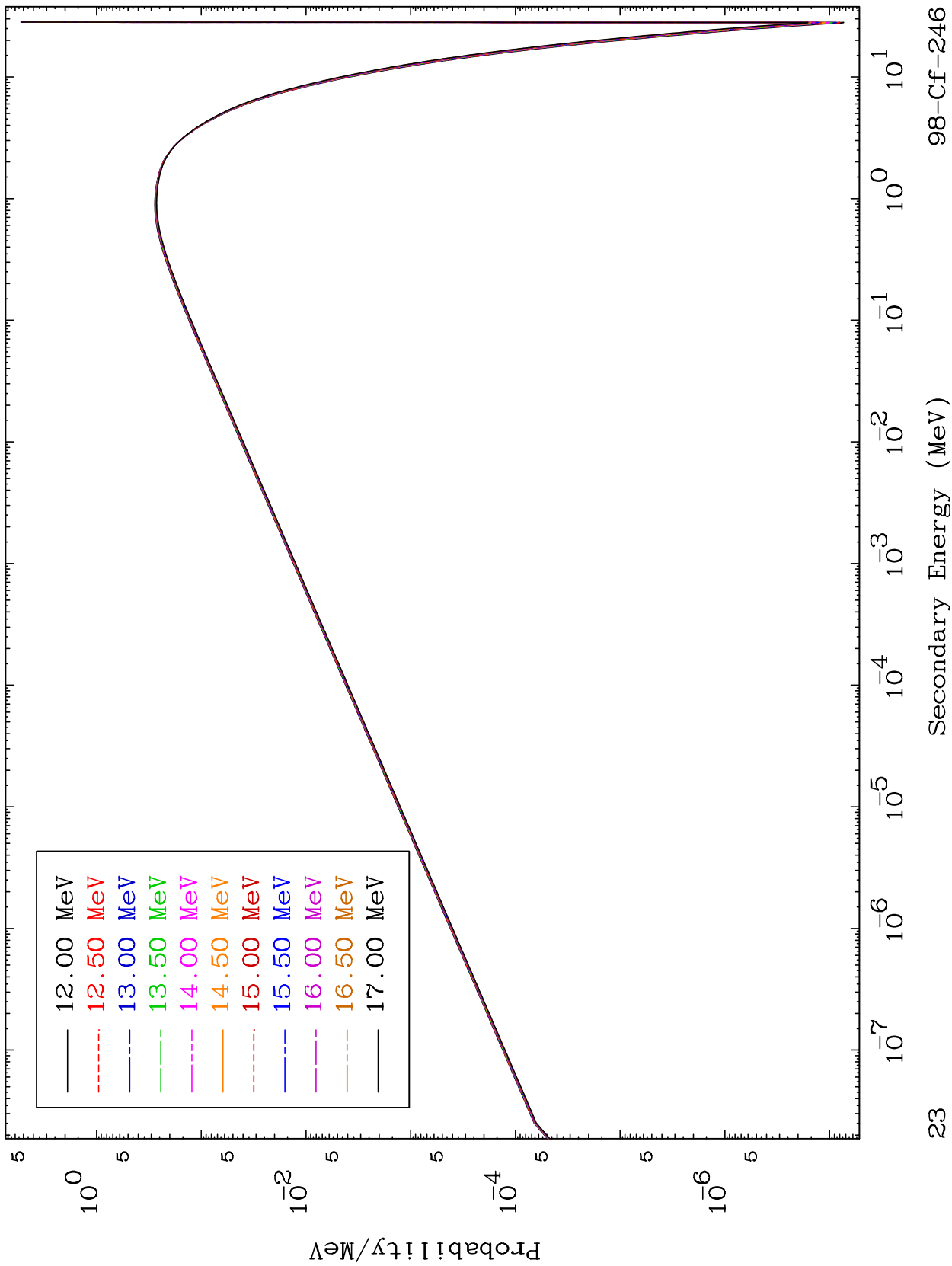


Fission Energy Distribution

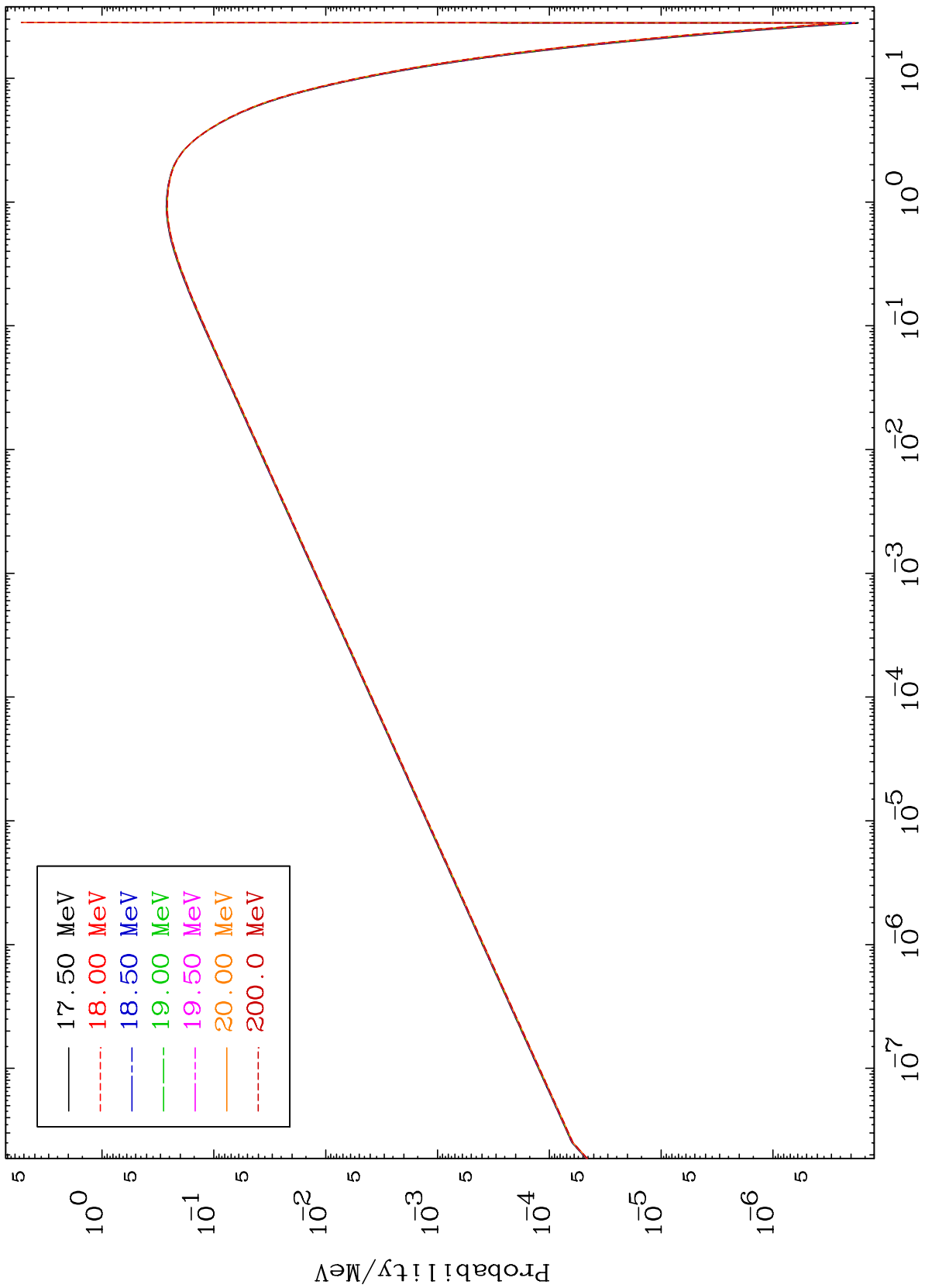


Fission
Energy Distribution





Fission
Energy Distribution

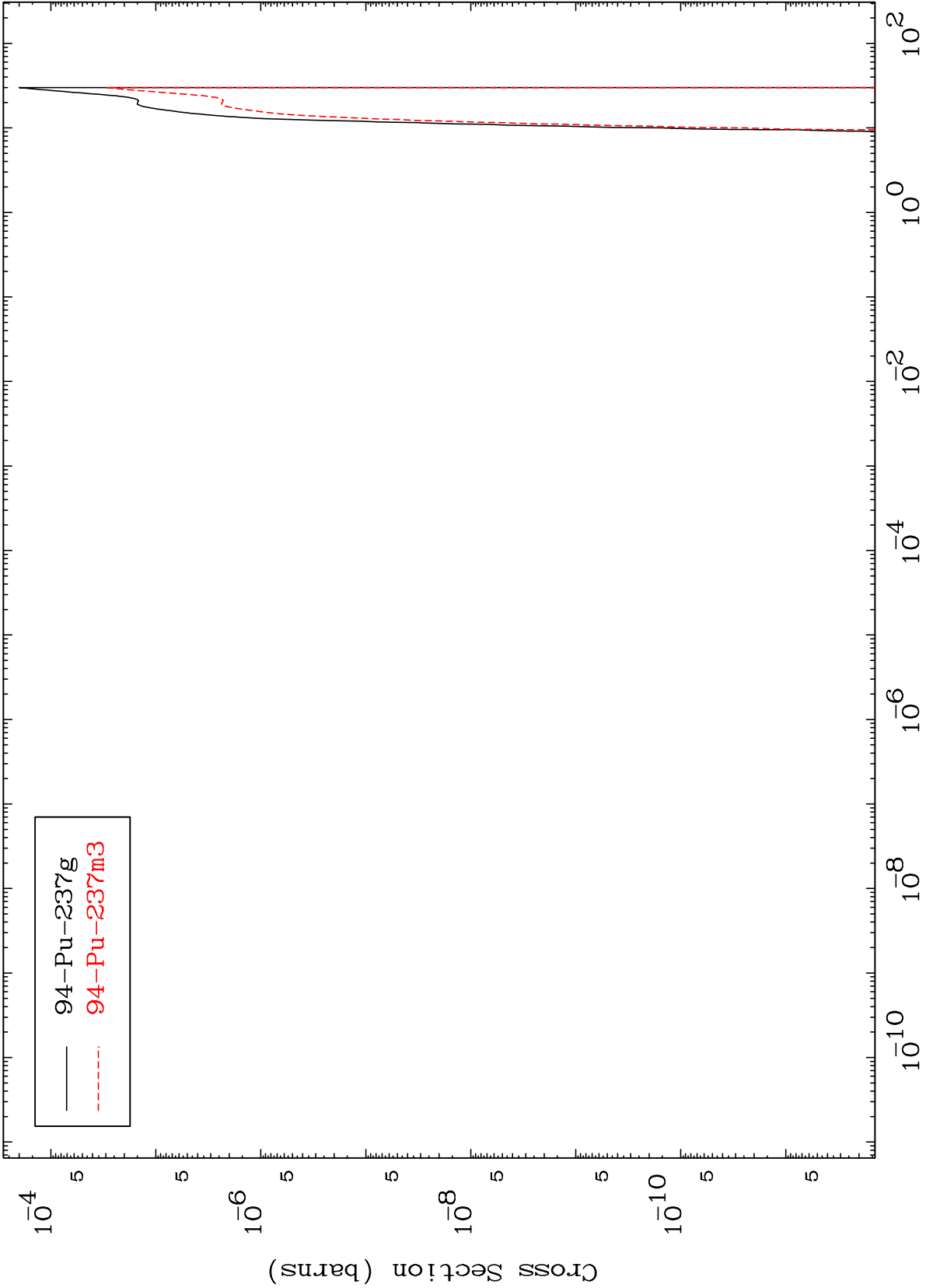


MAT 9843

(n,2n) 2 α

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Radionuclide Production Cross Section



MAT 9843

(n,p) α

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Radionuclide Production Cross Section

