

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

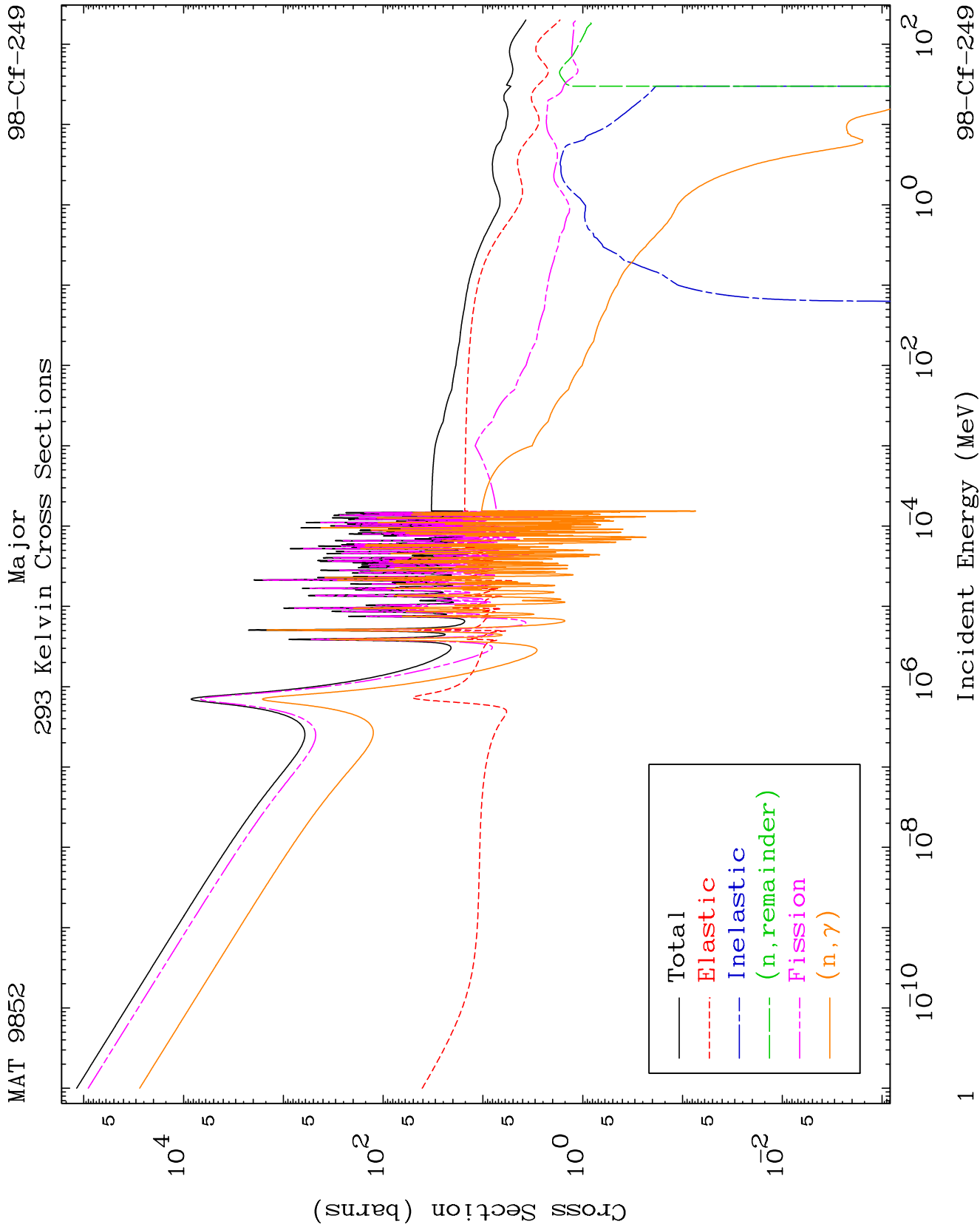
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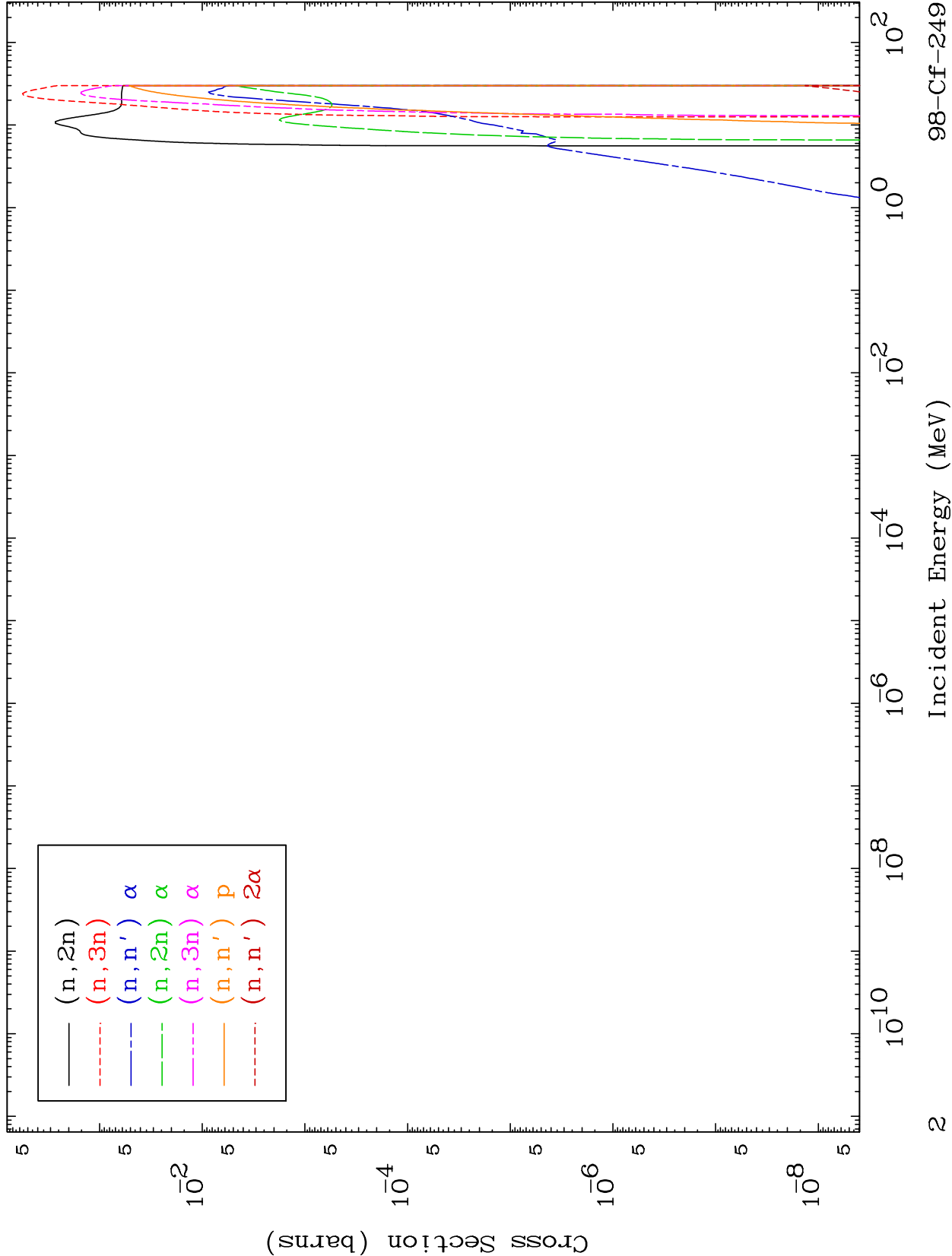
Press Mouse Button to Start



MAT 9852

Neutron Production
293 Kelvin Cross Sections

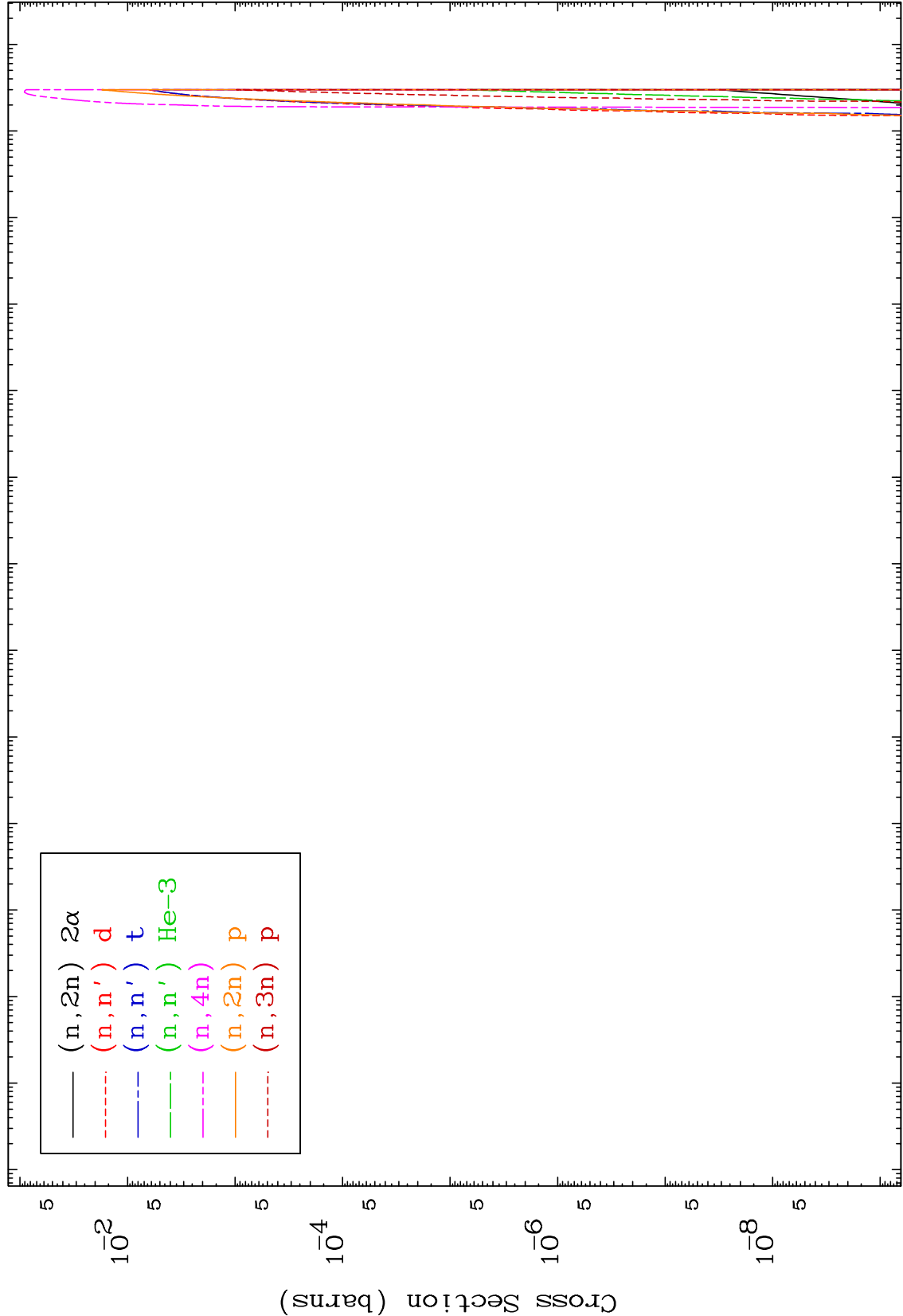
98-Cf-249



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Neutron Production
293 Kelvin Cross Sections

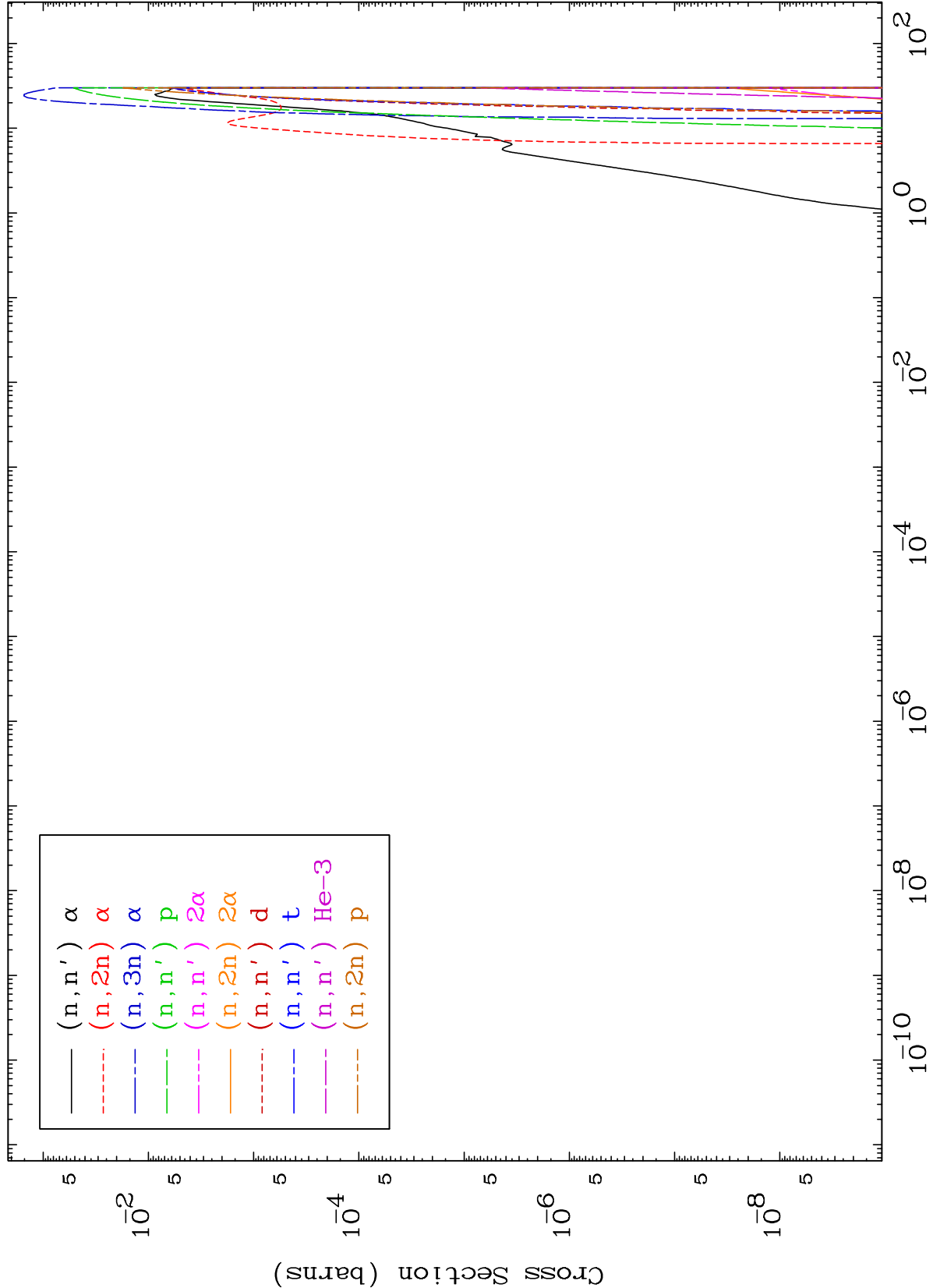
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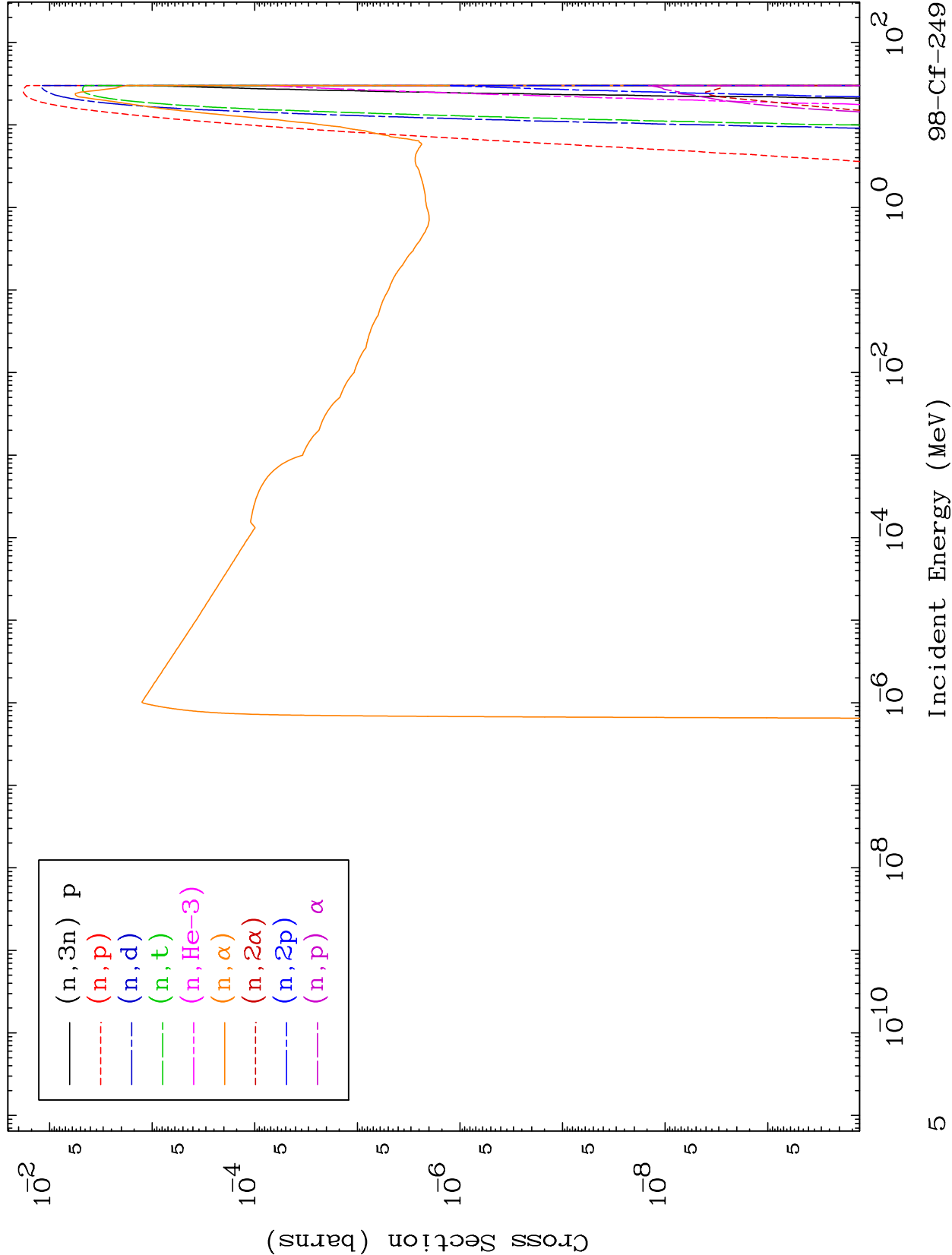


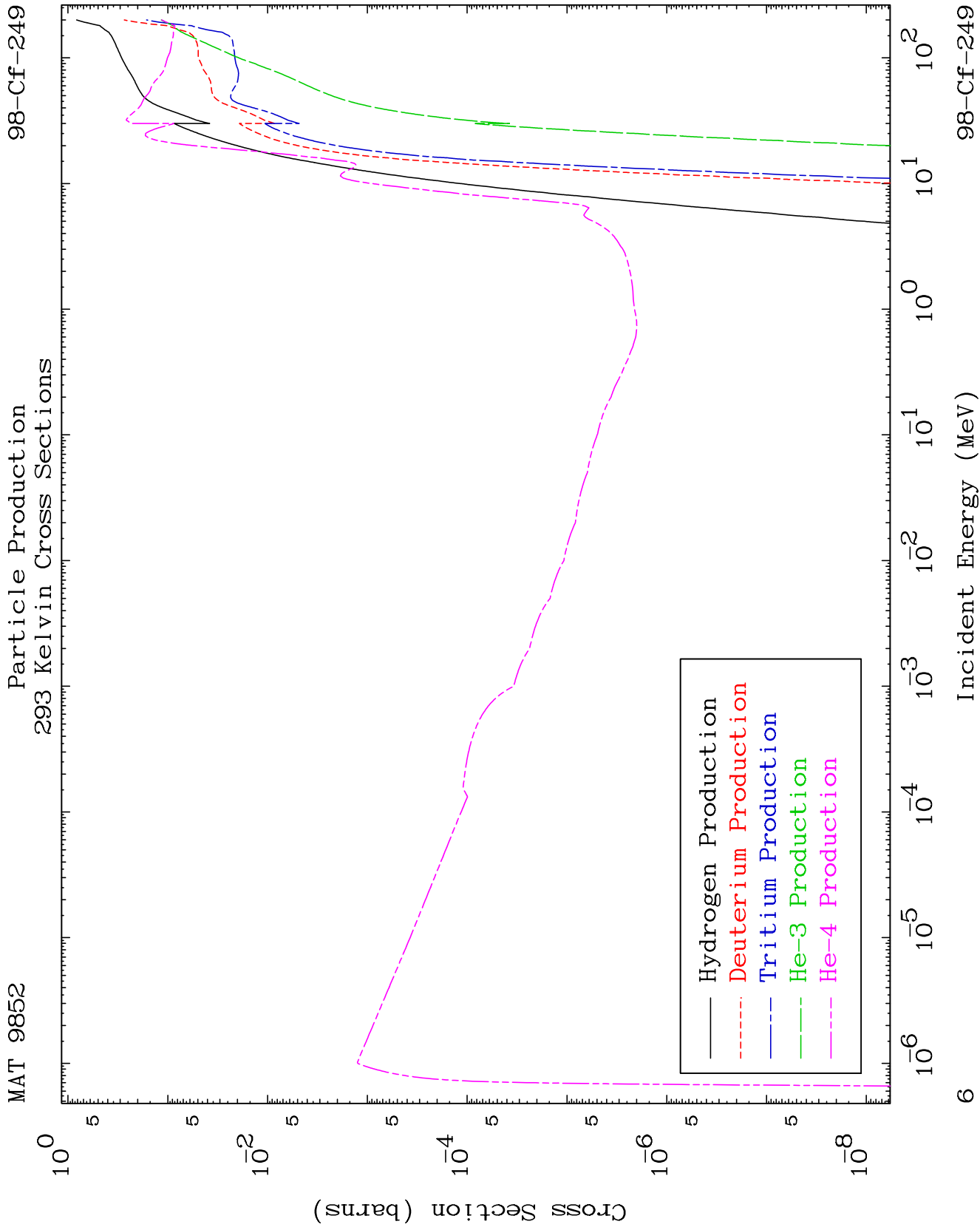
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Charged Particle
293 Kelvin Cross Sections

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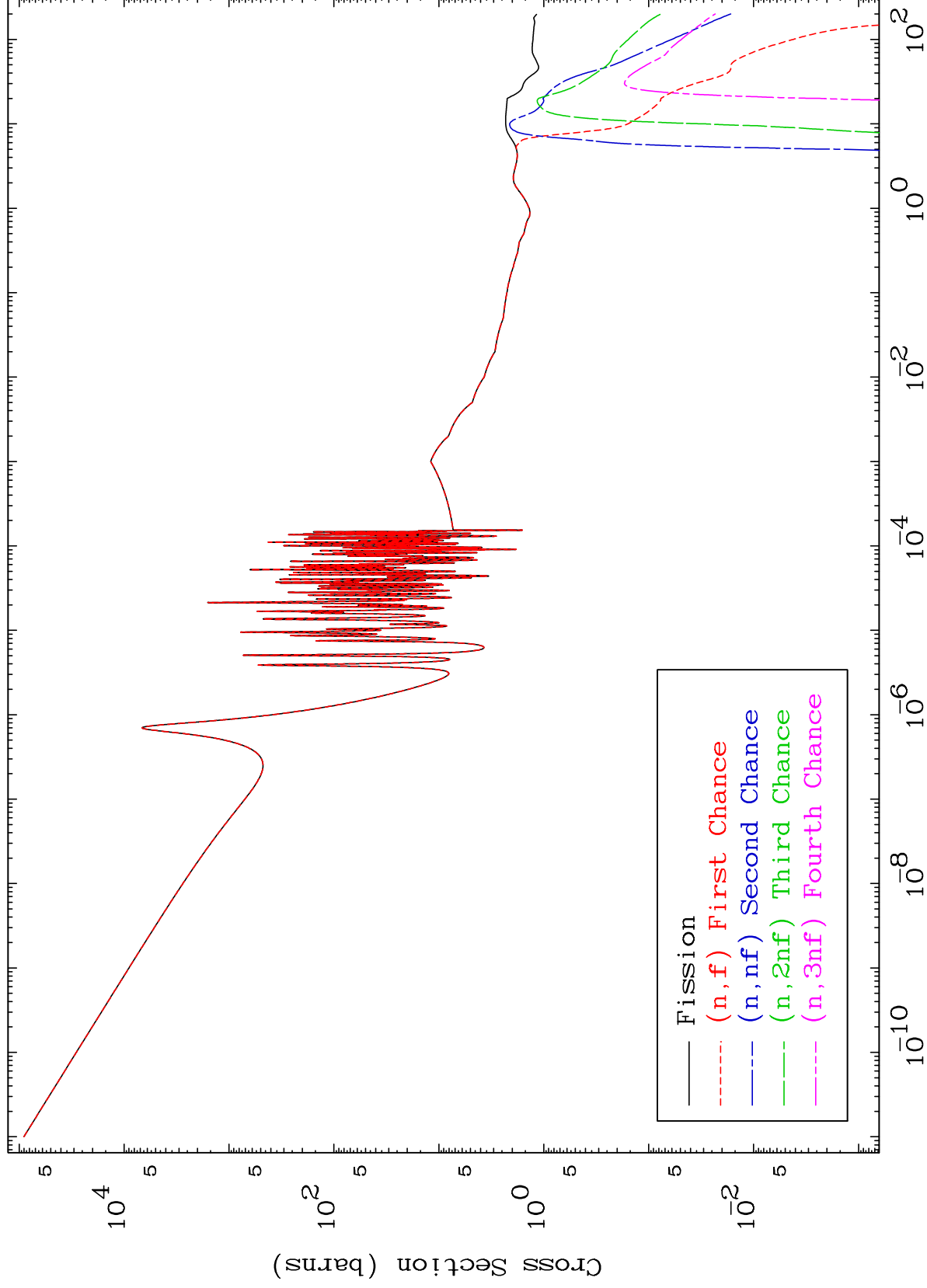


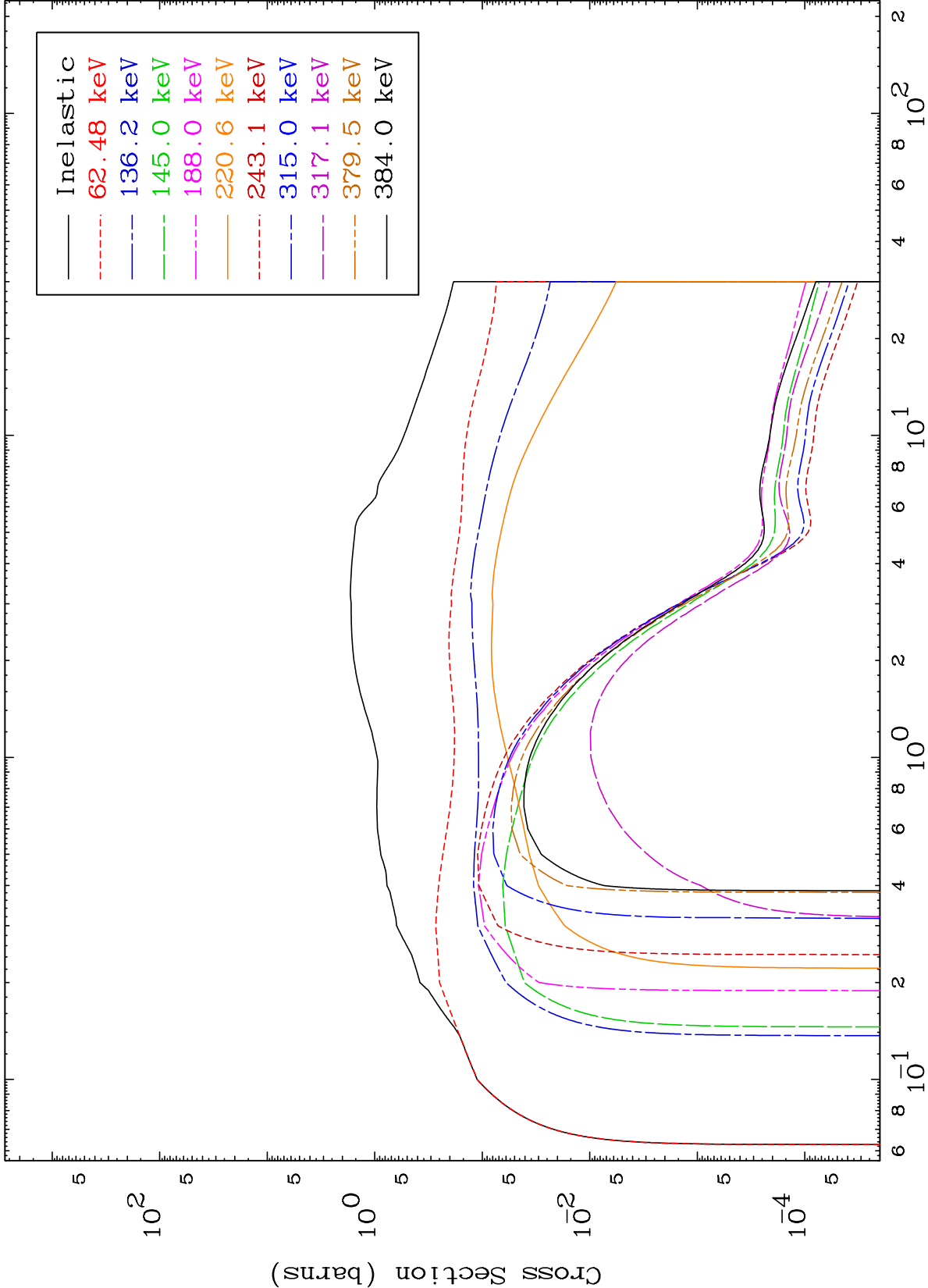
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Fission

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293 Kelvin Cross Sections

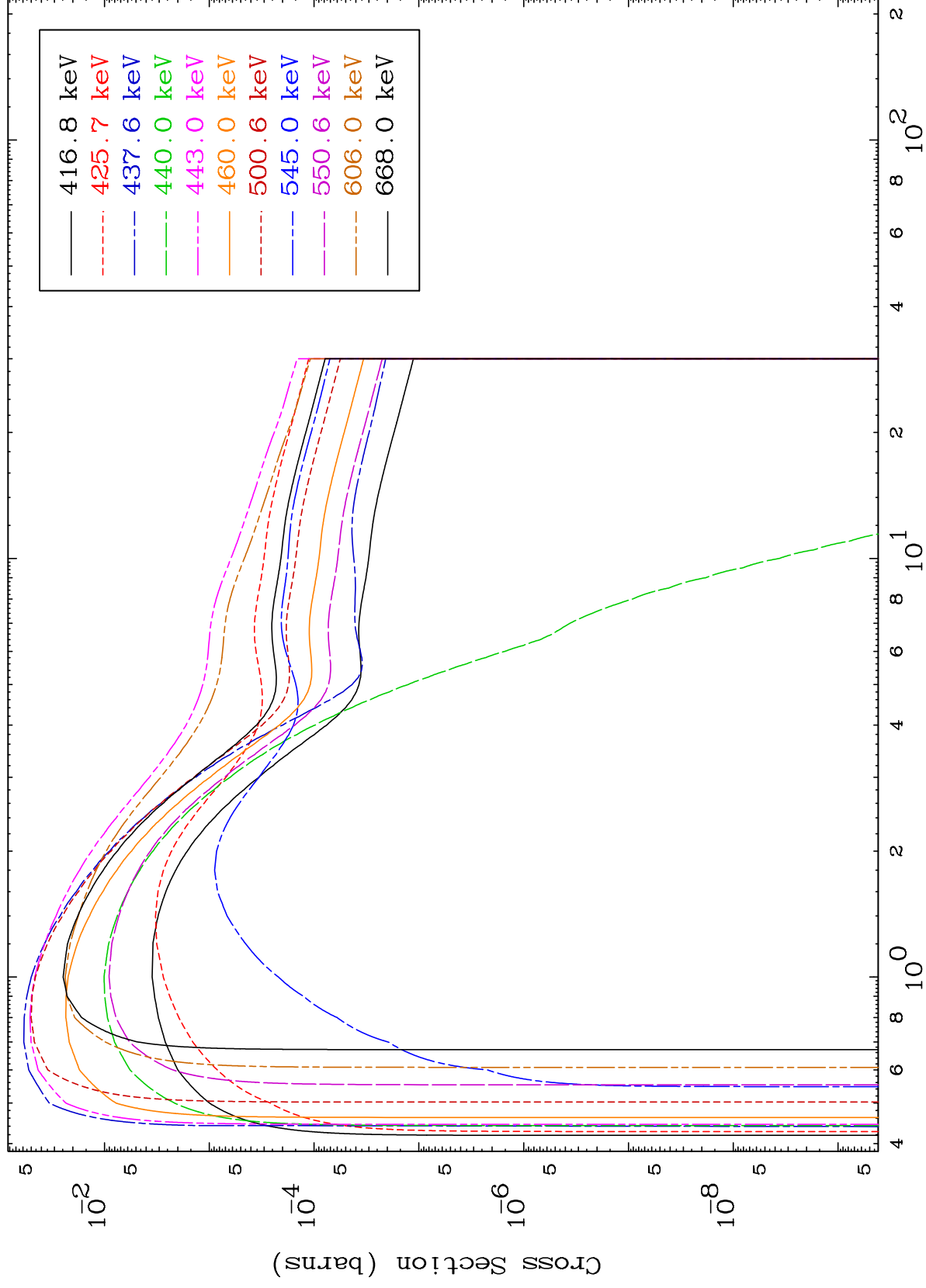




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(n,n') Level
293 Kelvin Cross Sections

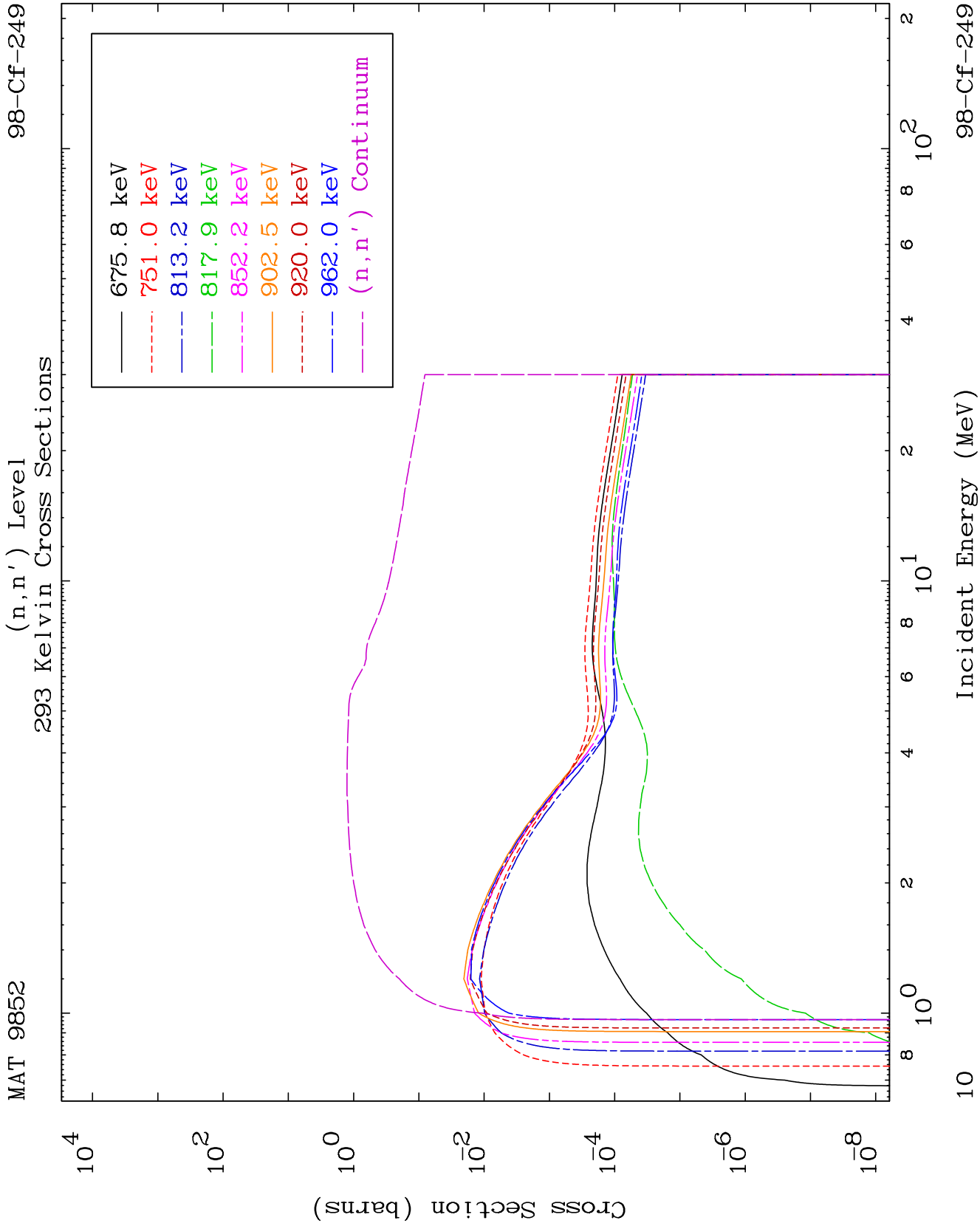
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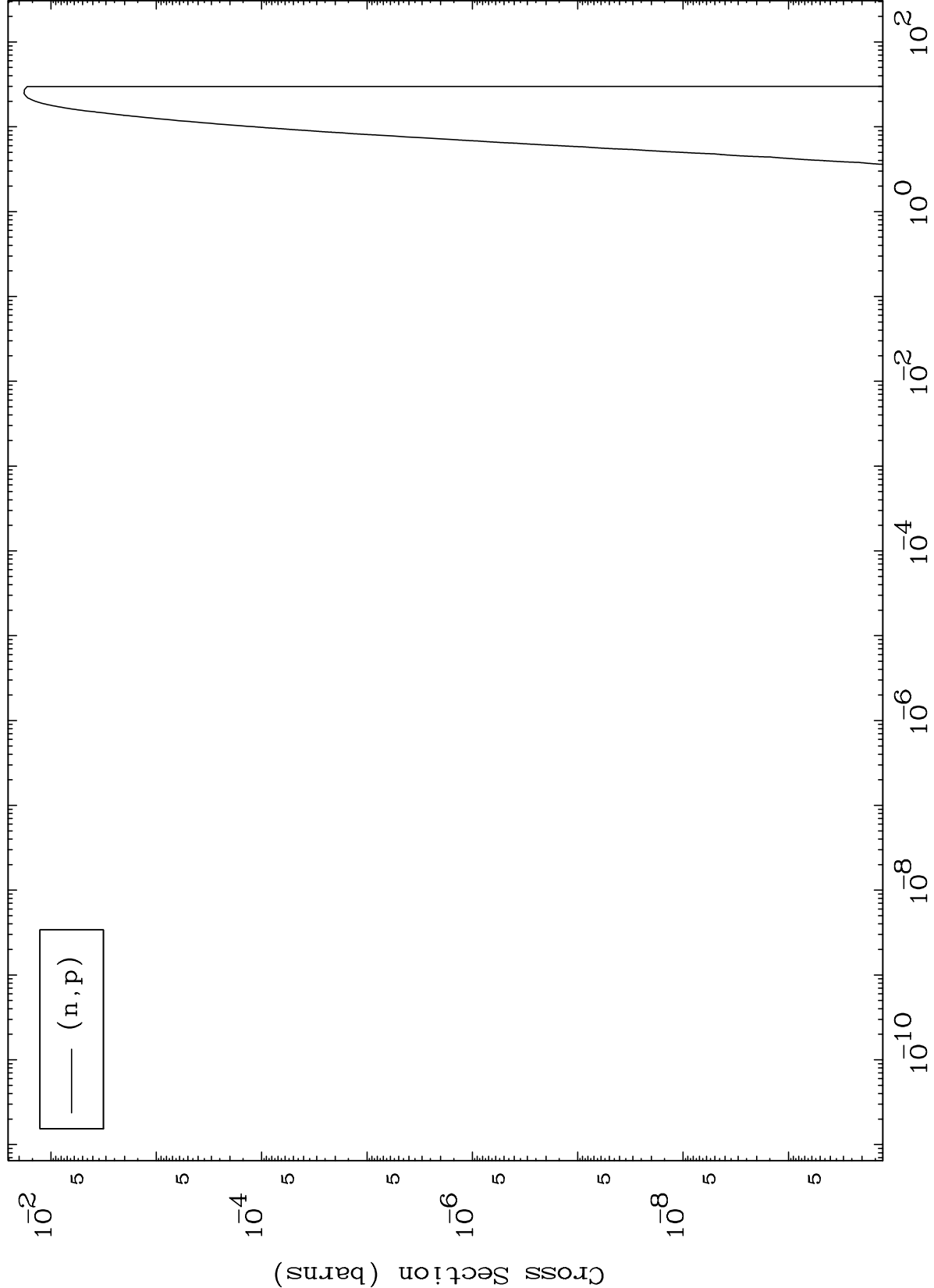


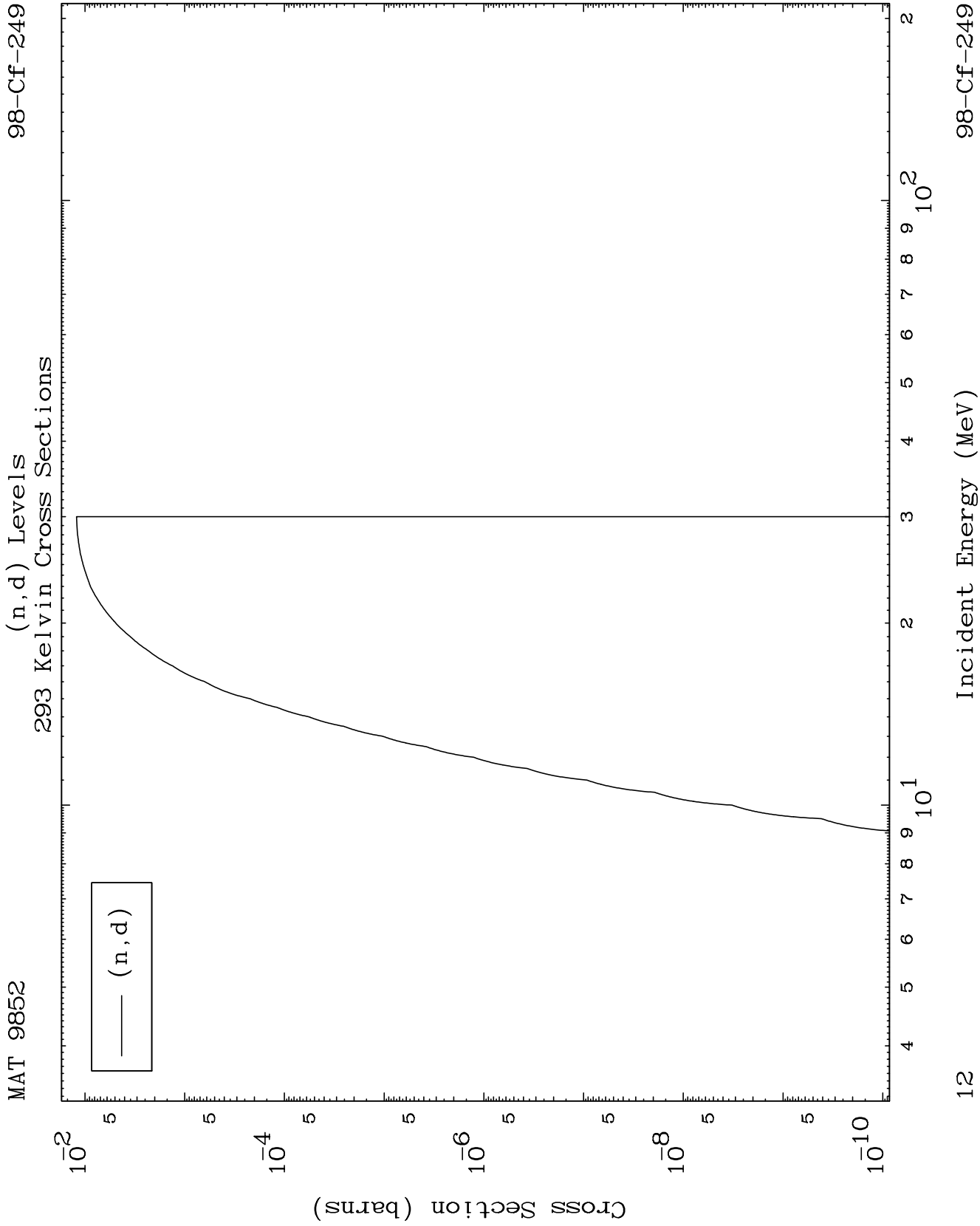
9

Incident Energy (MeV)

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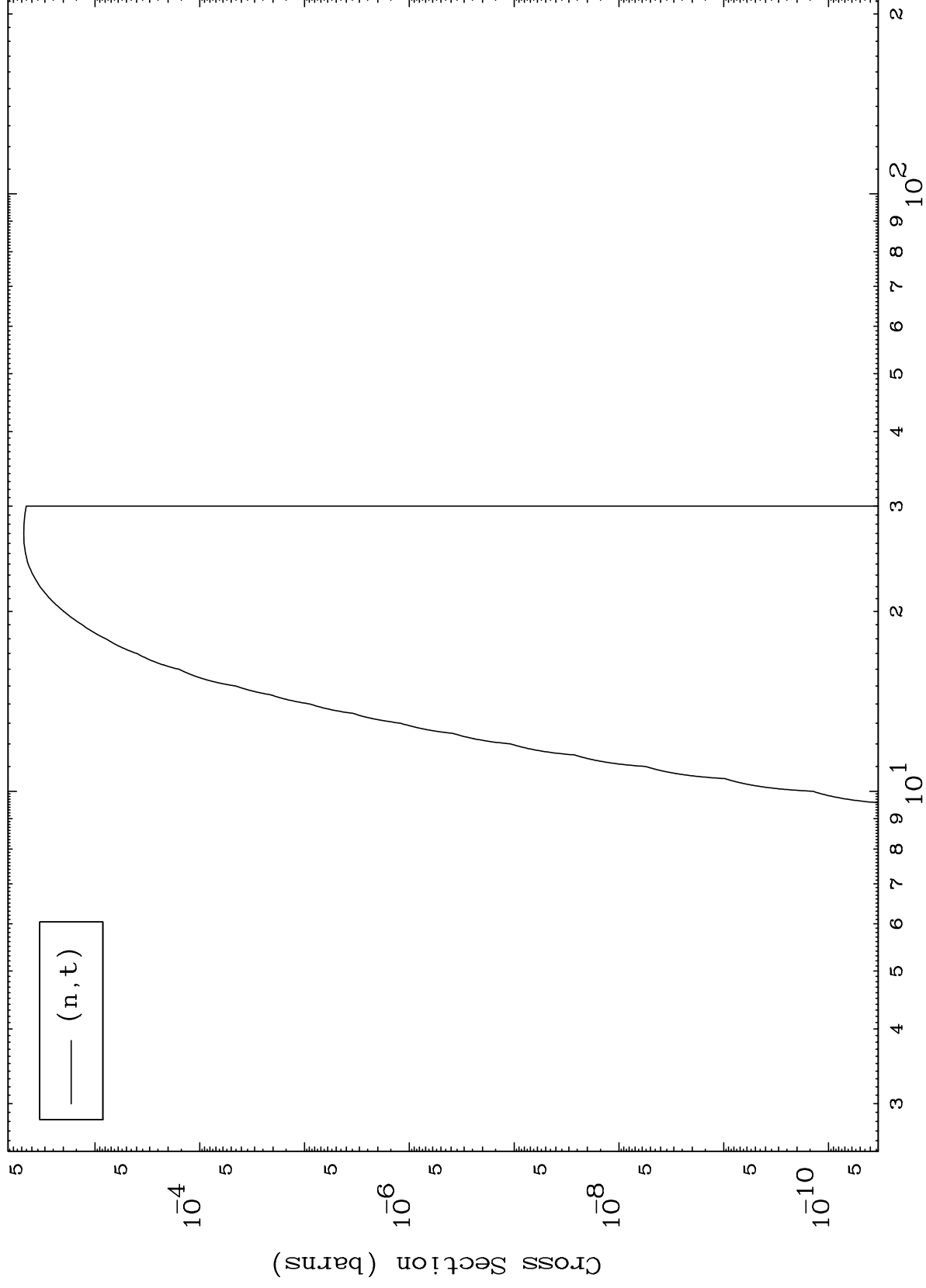




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(n,t) Levels
293 Kelvin Cross Sections



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Incident Energy (MeV)

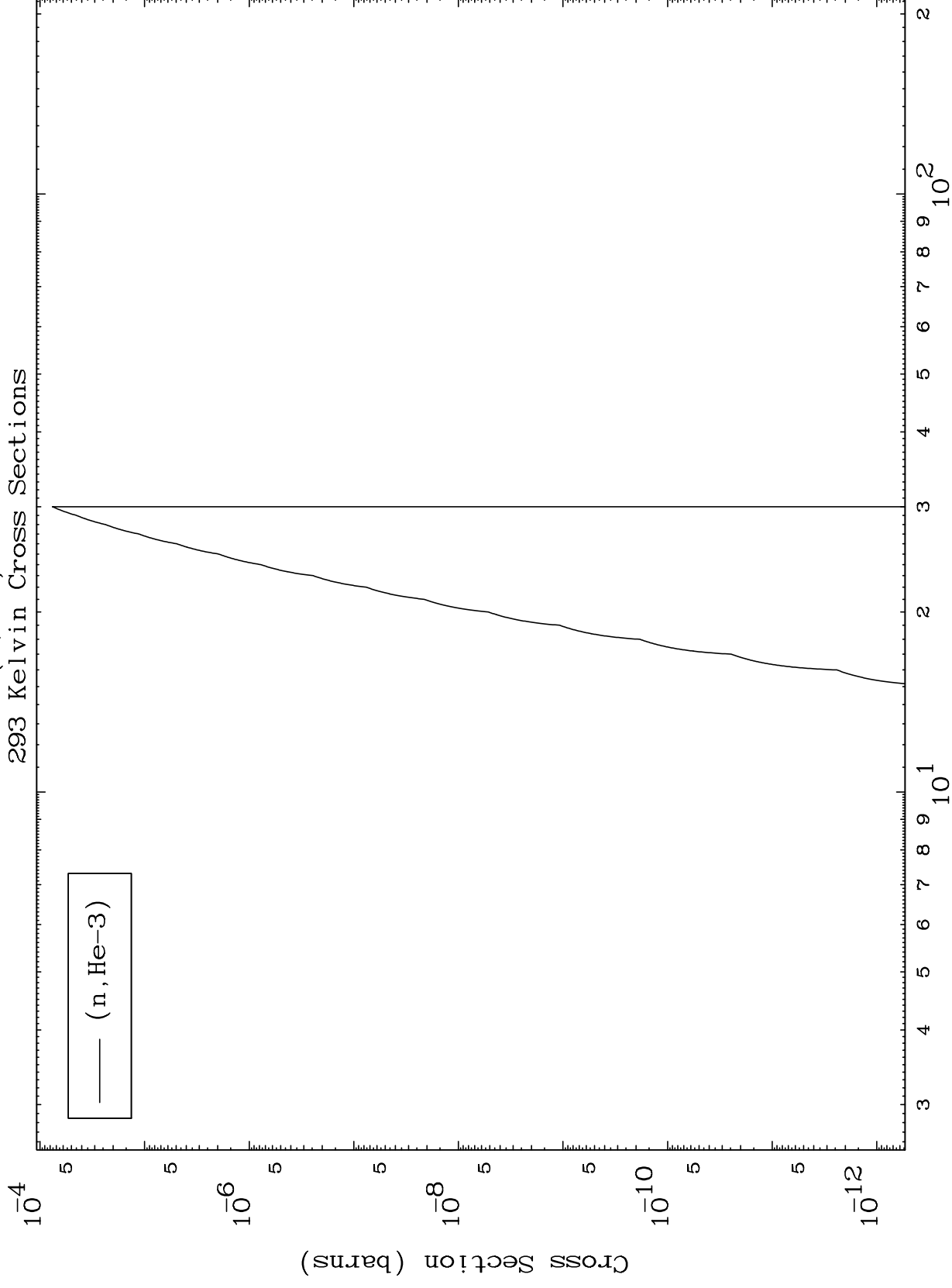
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(n, He3) Levels

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293 Kelvin Cross Sections



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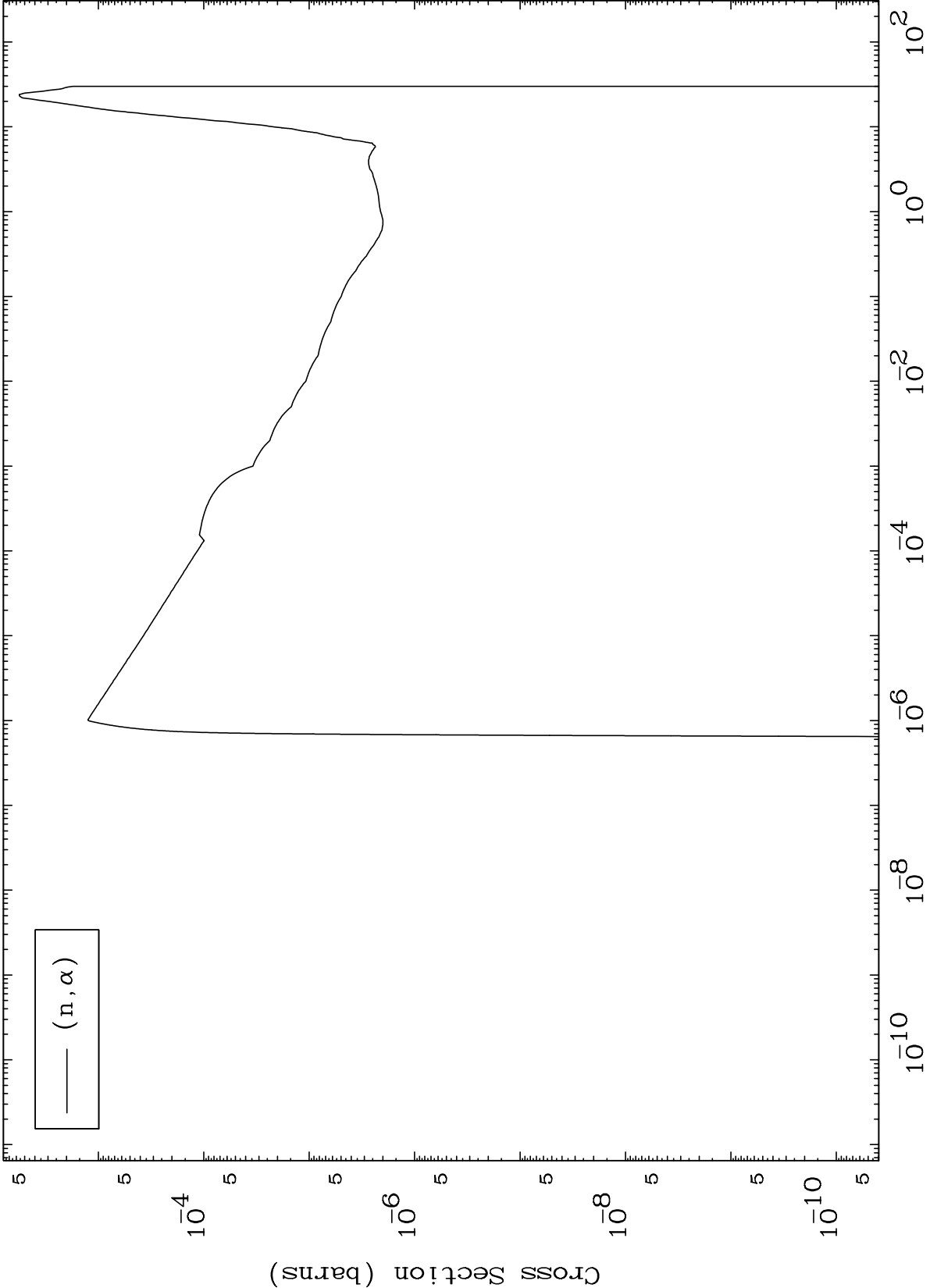
Incident Energy (MeV)

98-Cf-249

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(n, α) Levels
293 Kelvin Cross Sections

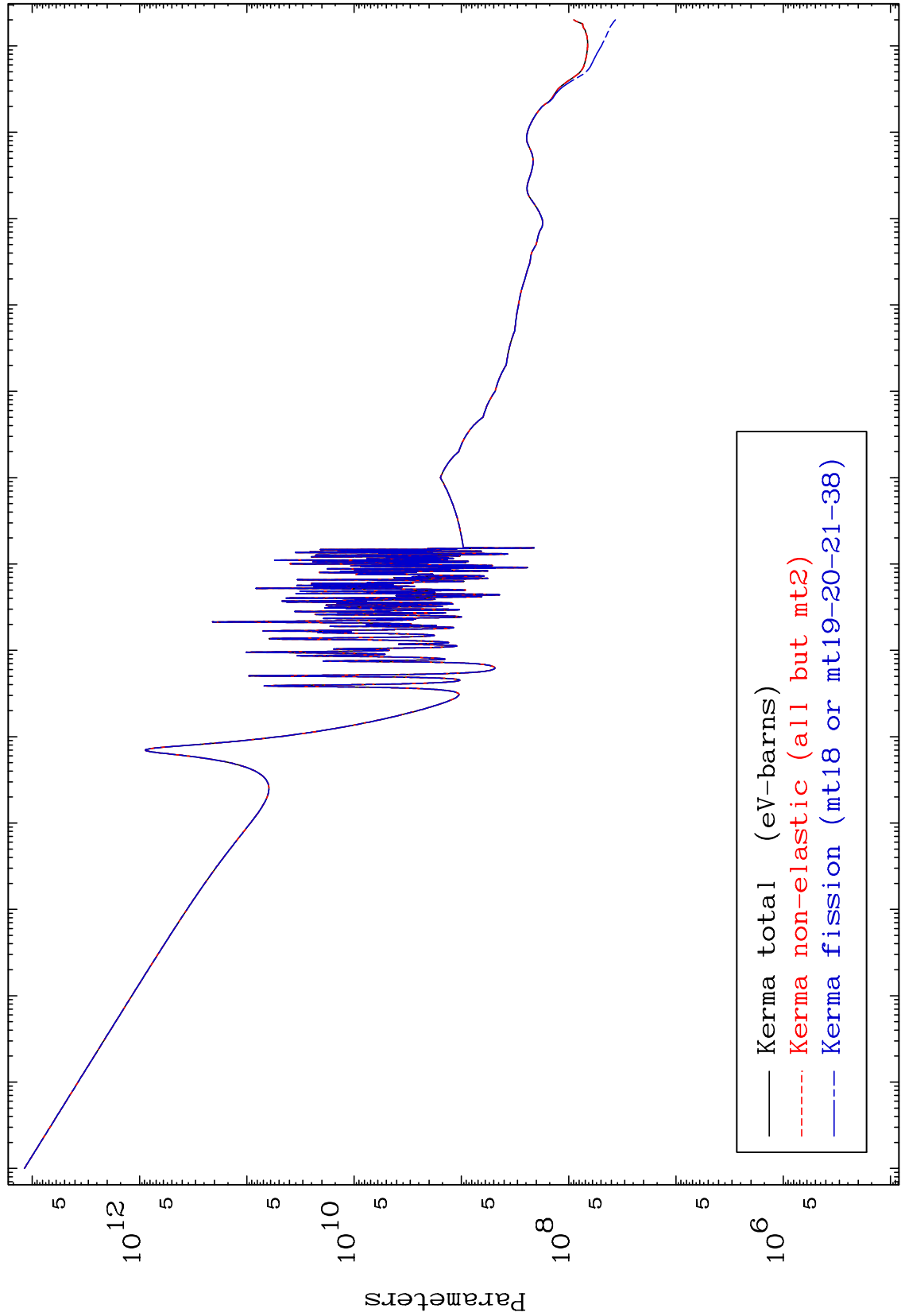
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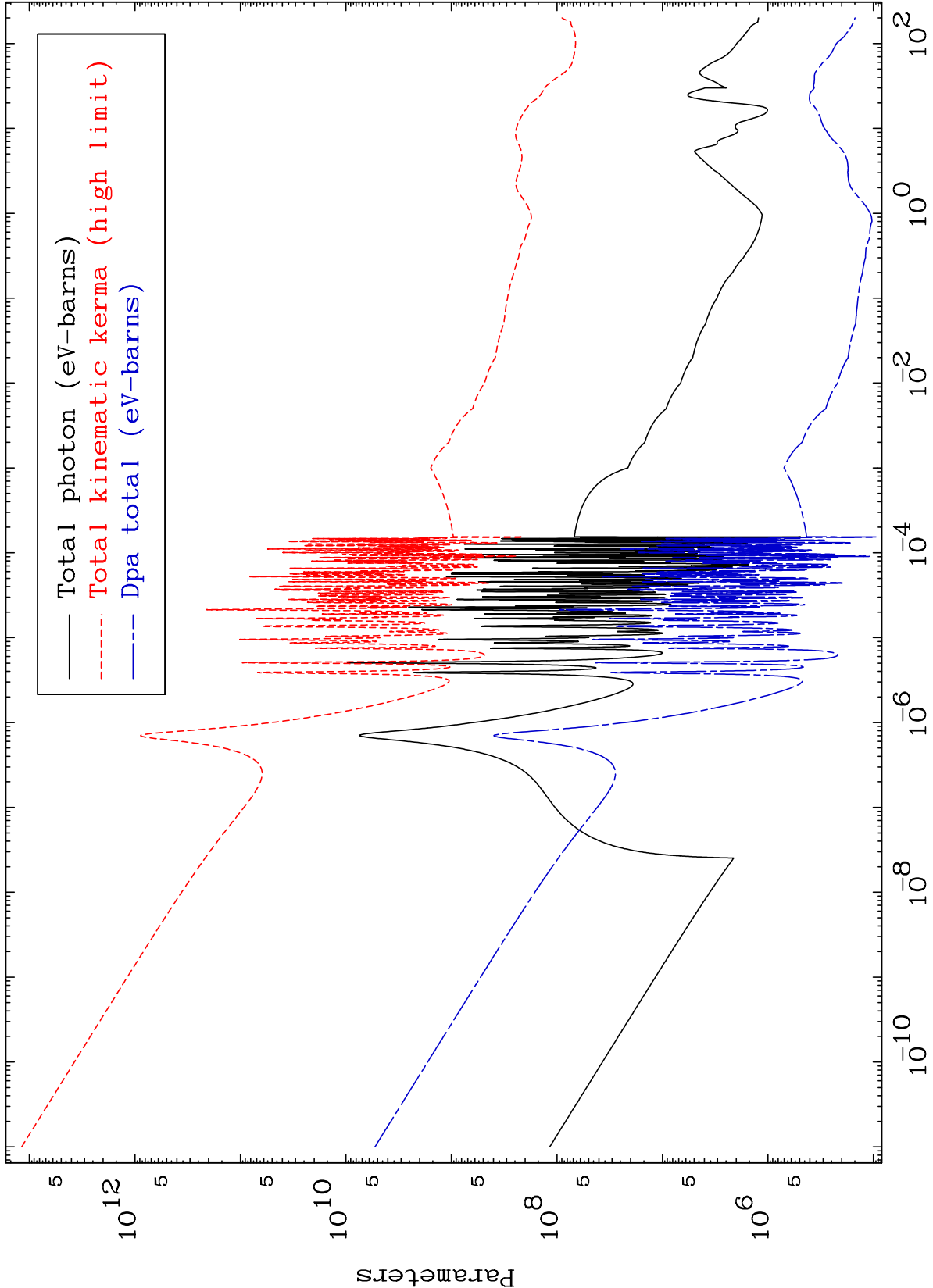


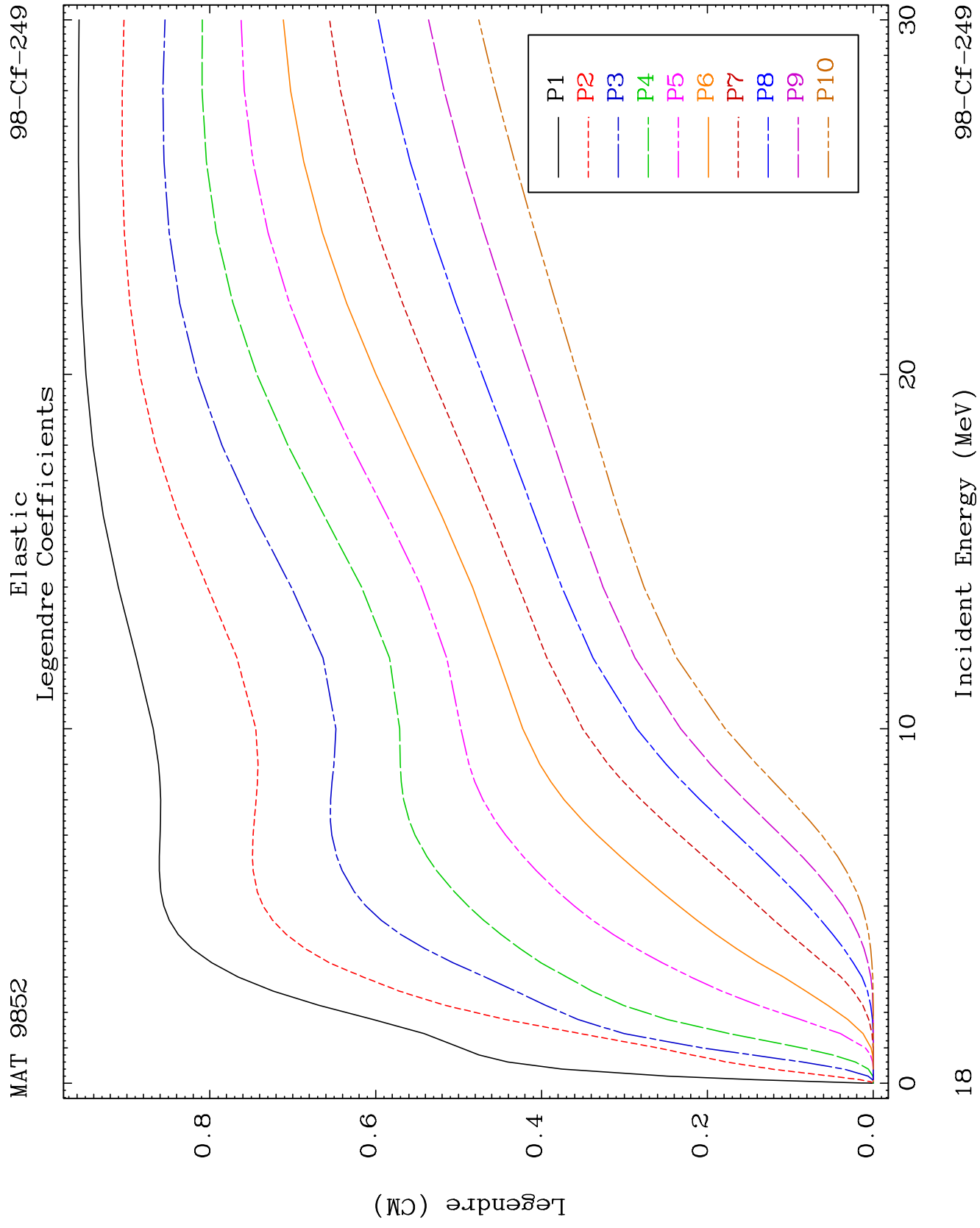
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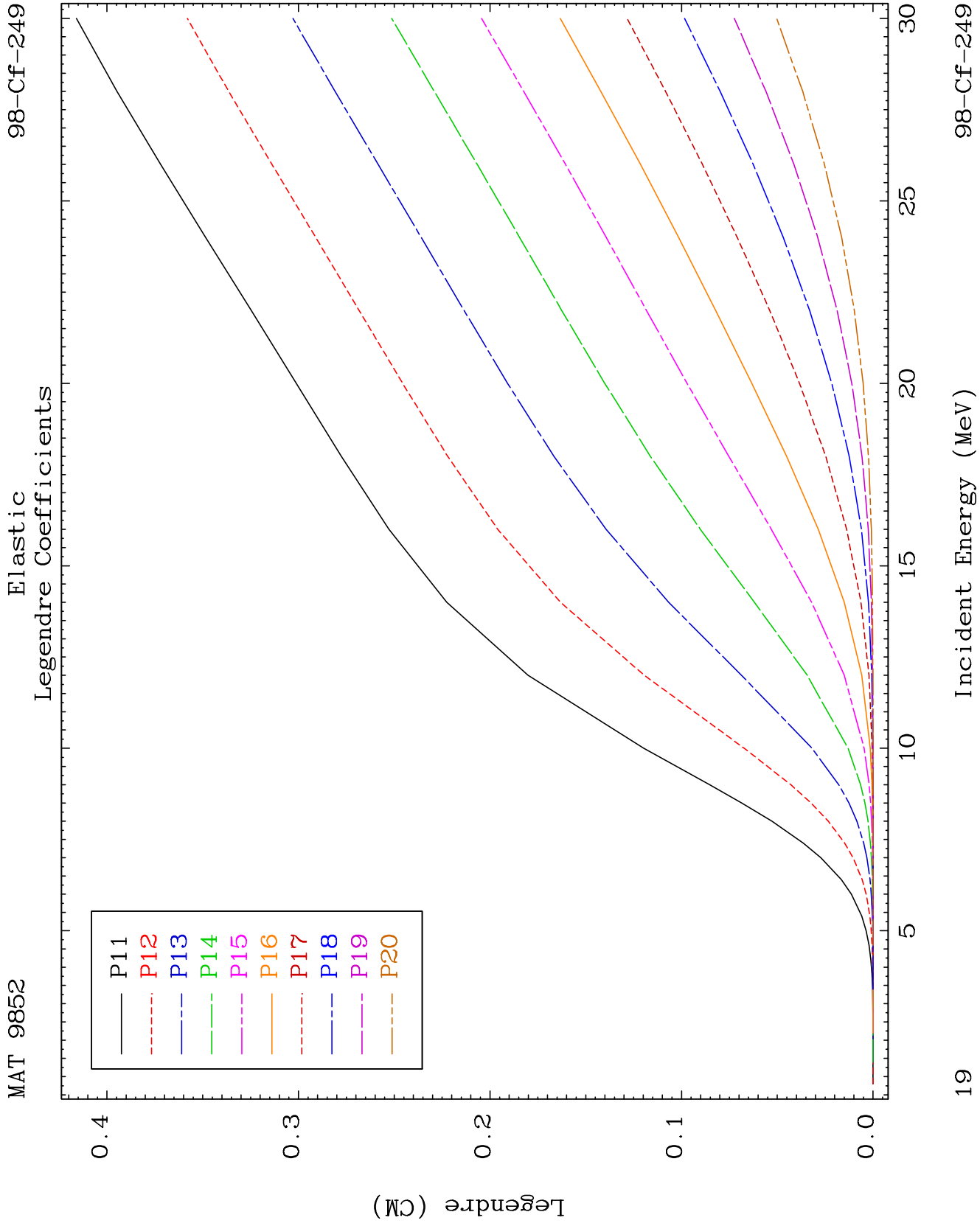
Incident Energy (MeV)

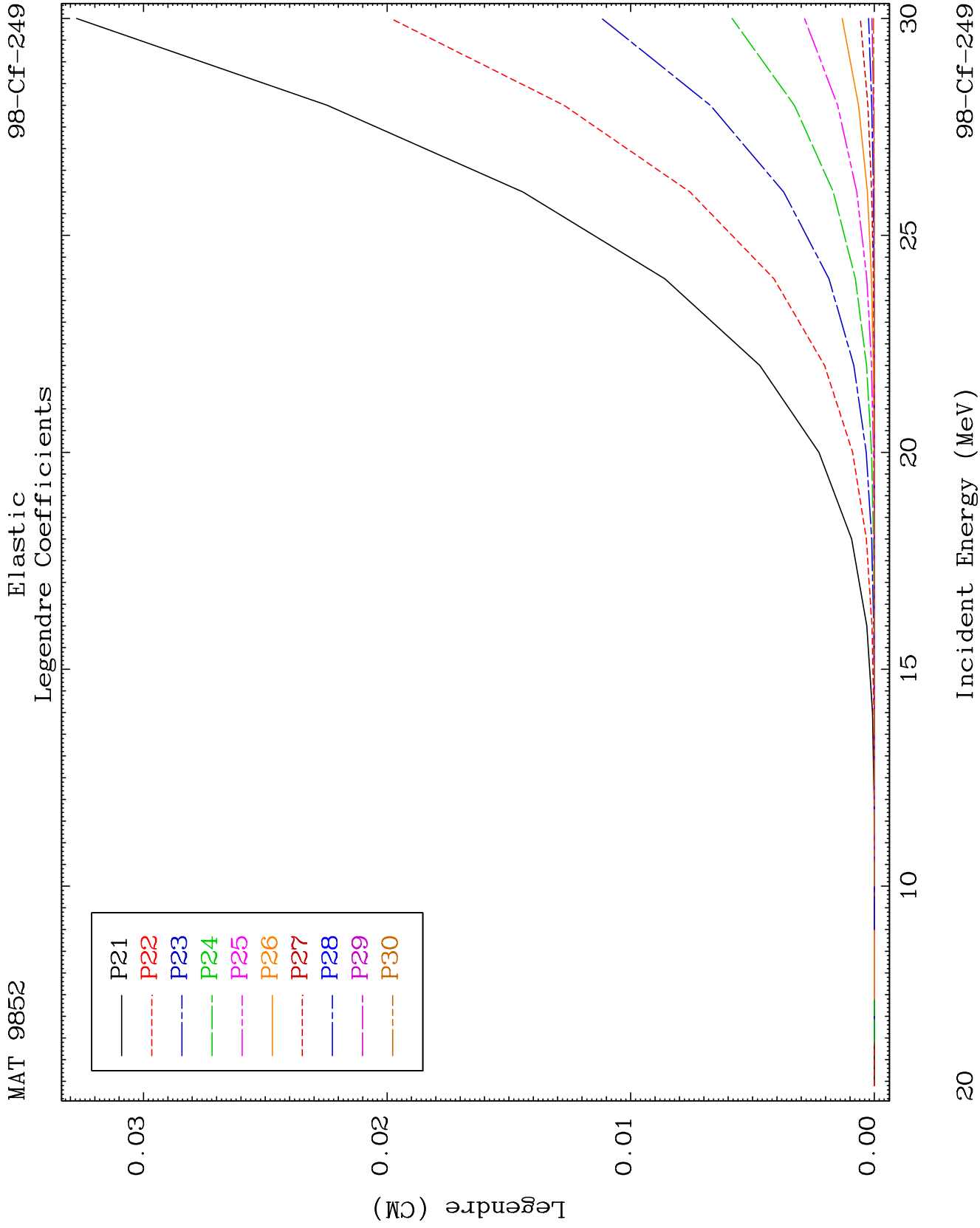
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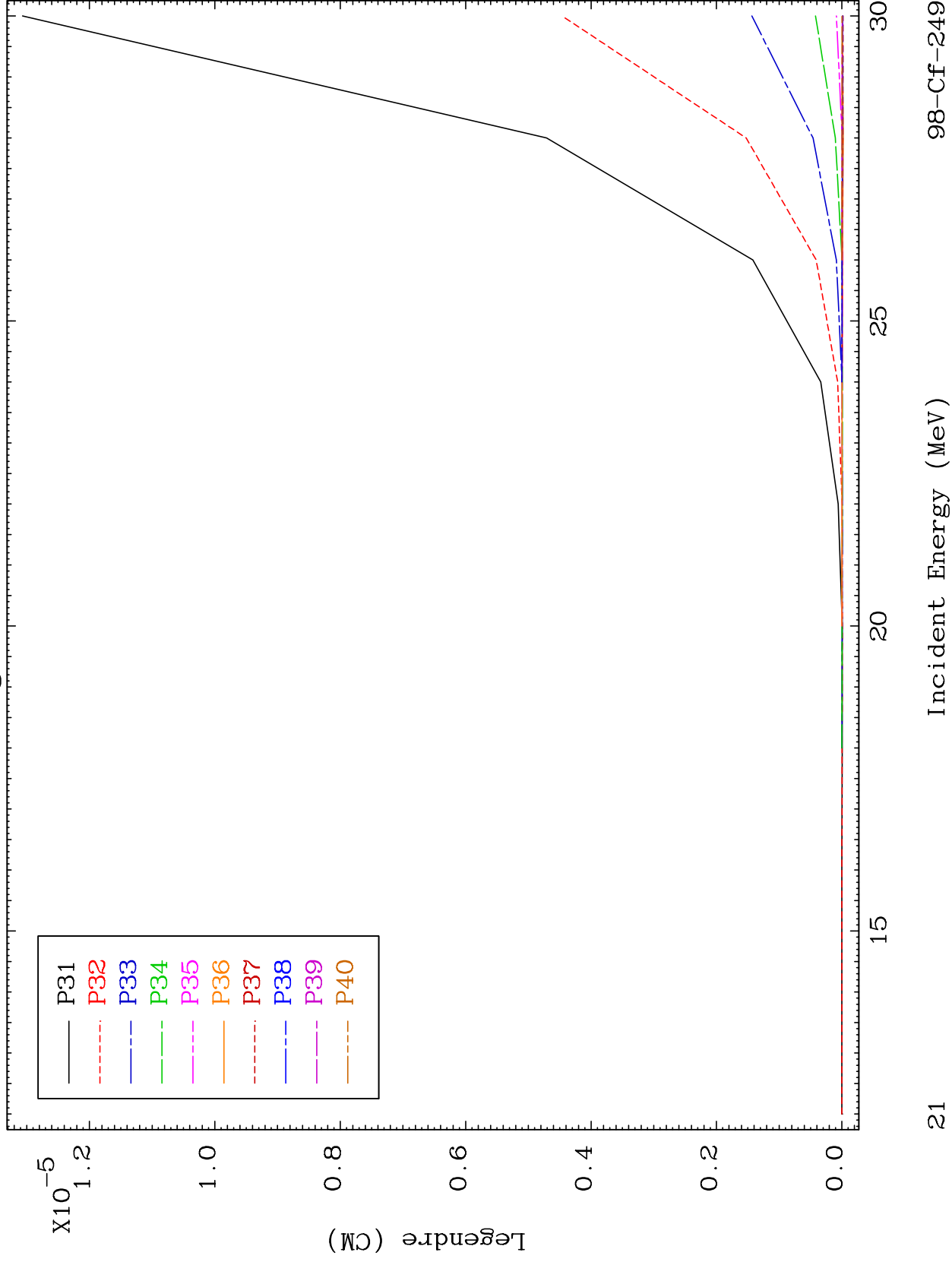




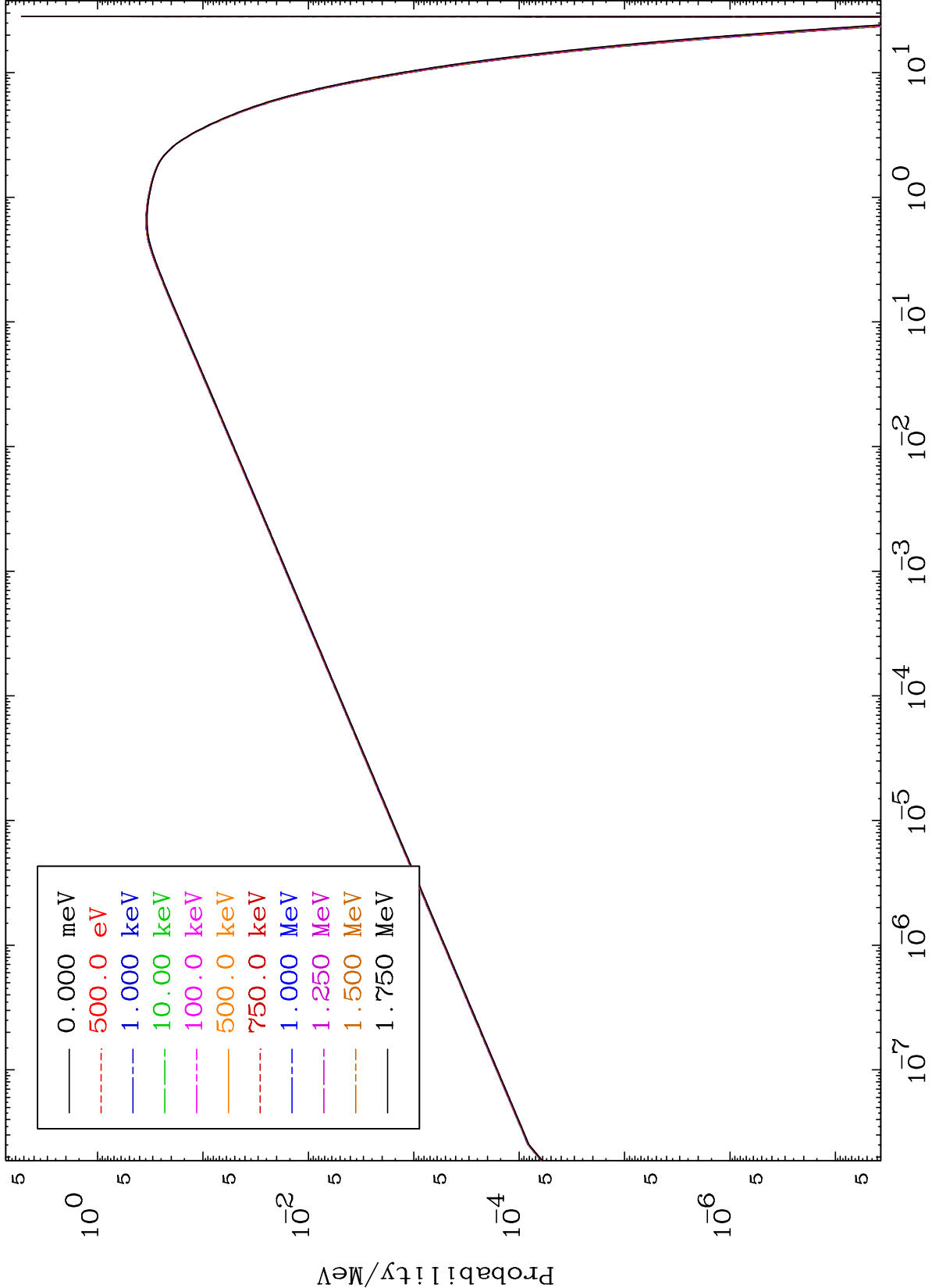
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Elastic Legendre Coefficients

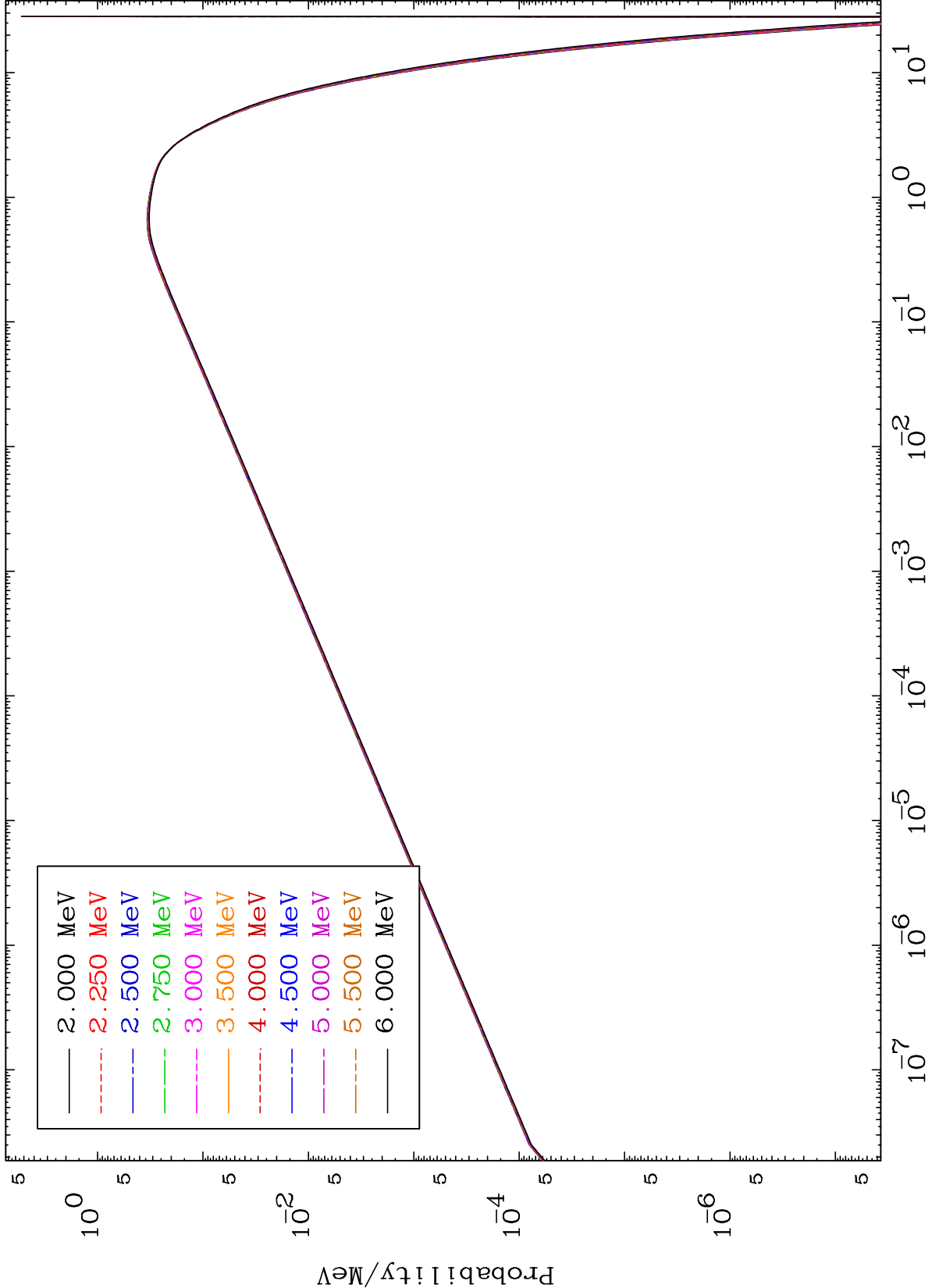
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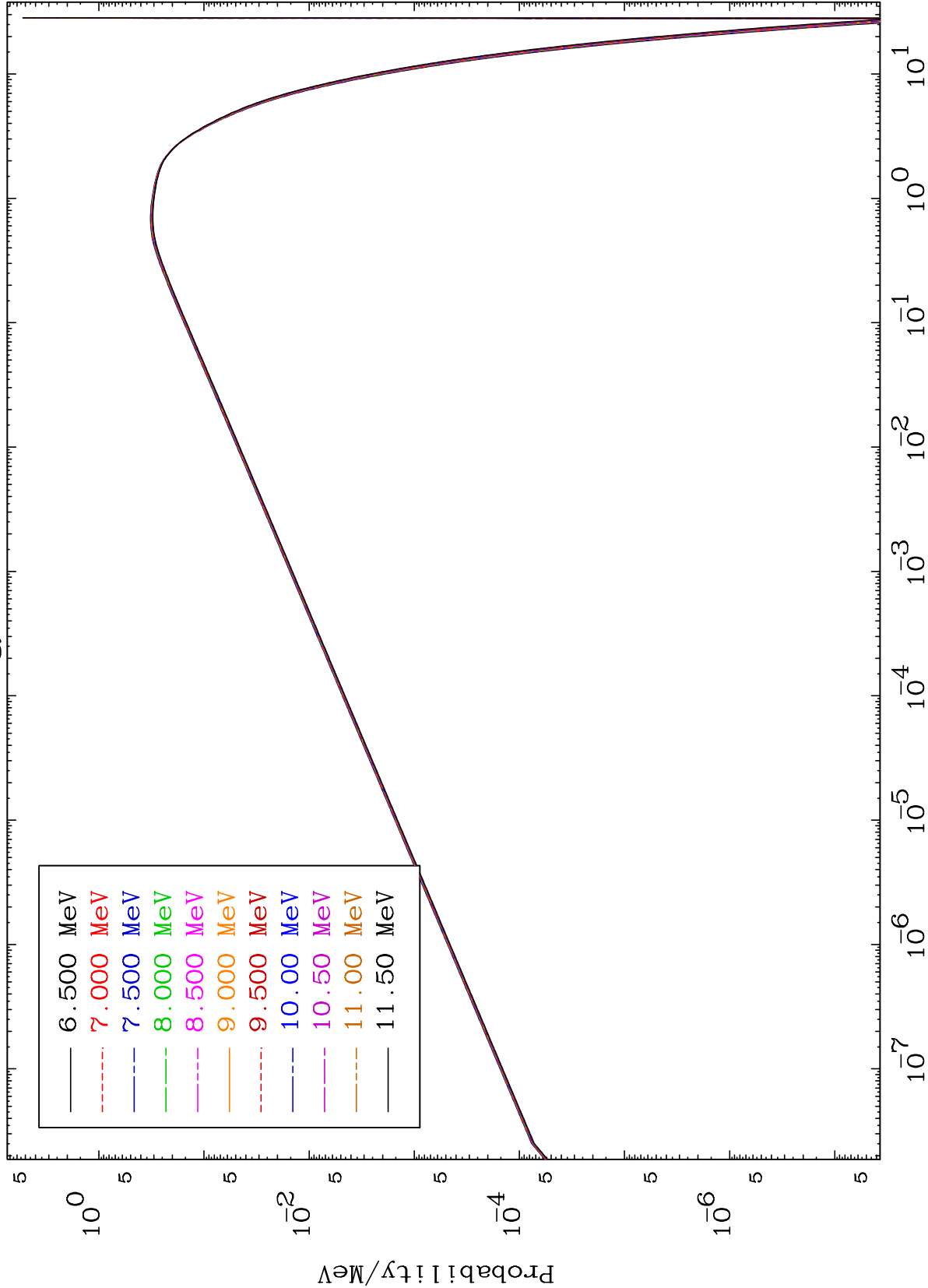
Fission
Energy Distribution



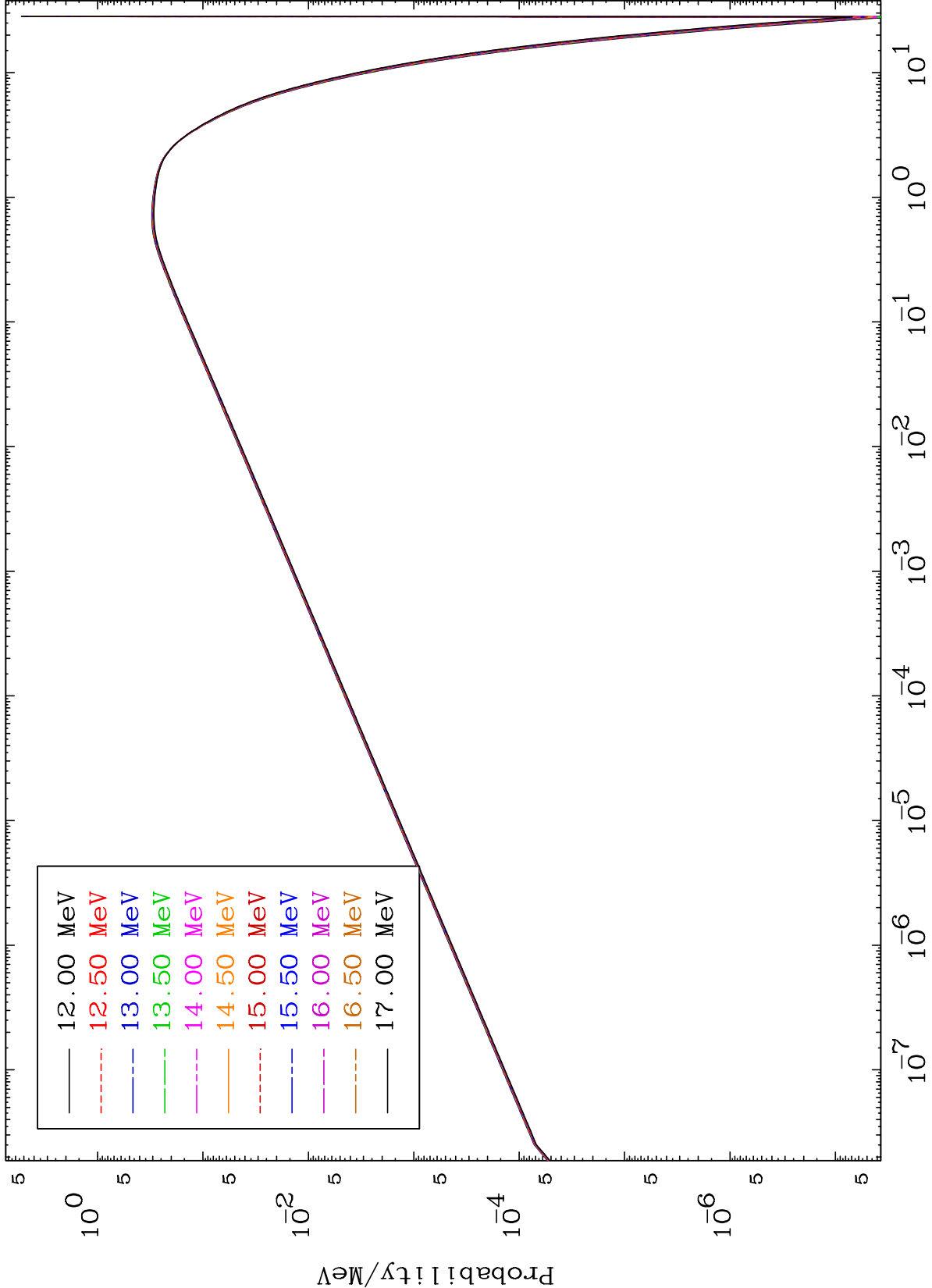
Fission
Energy Distribution



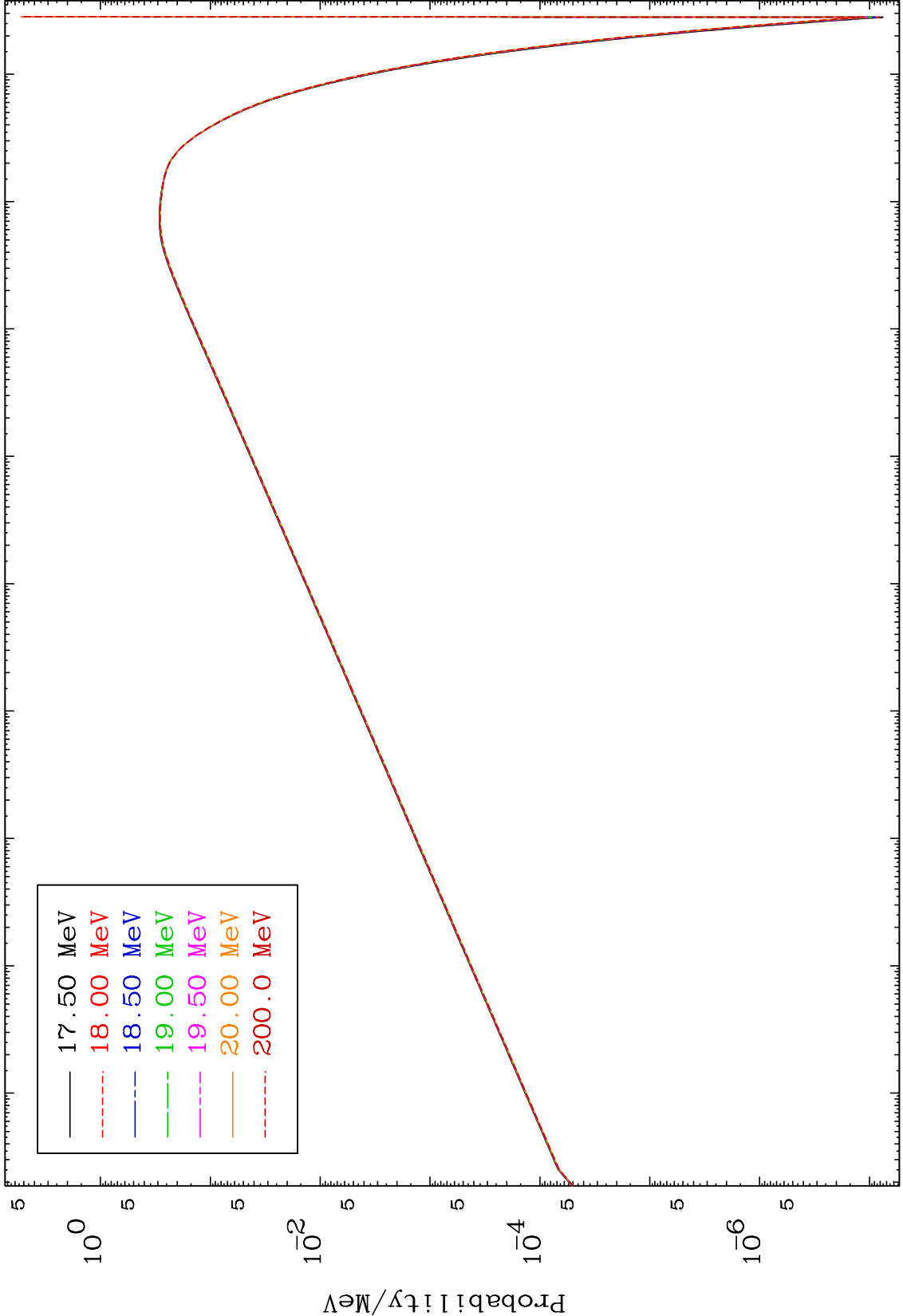
Fission
Energy Distribution



Fission
Energy Distribution



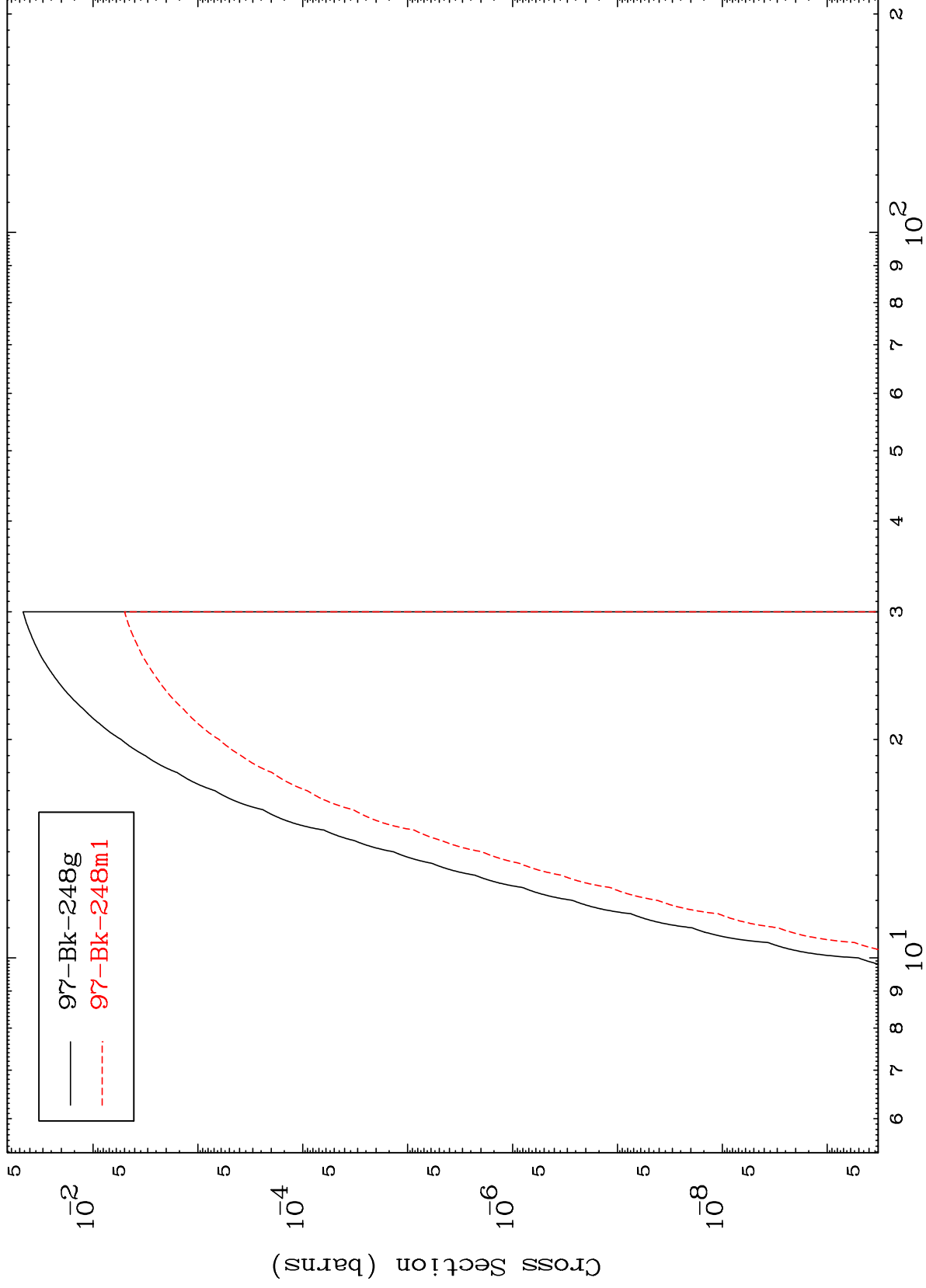
Energy Distribution



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(n,n') p
Radionuclide Production Cross Section

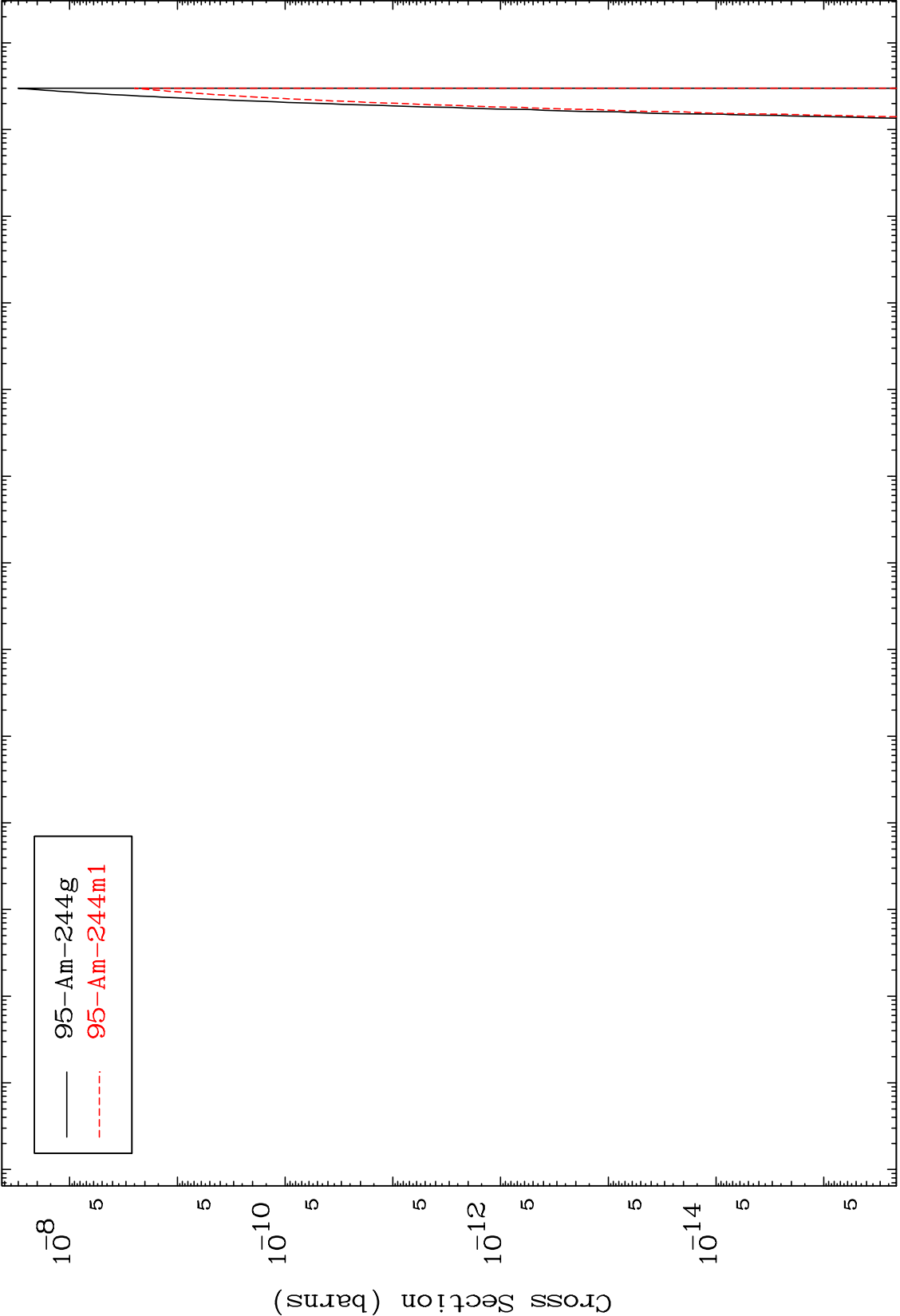


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Incident Energy (MeV)

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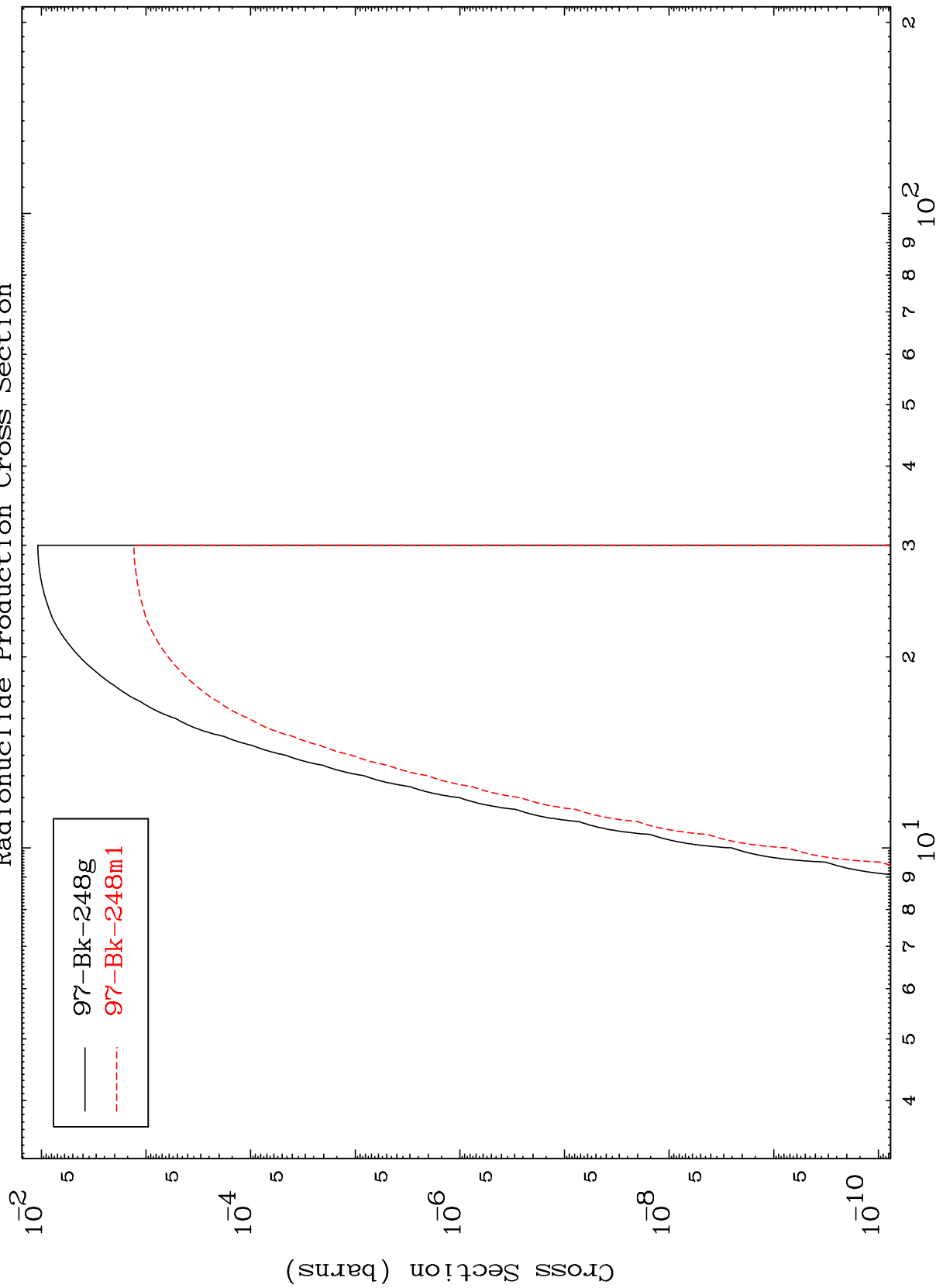
Radionuclide Production Cross Section



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Radionuclide Production Cross Section
(n,d)



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Incident Energy (MeV)

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