

Program EVALPLOT
(Version 2015-2)

by

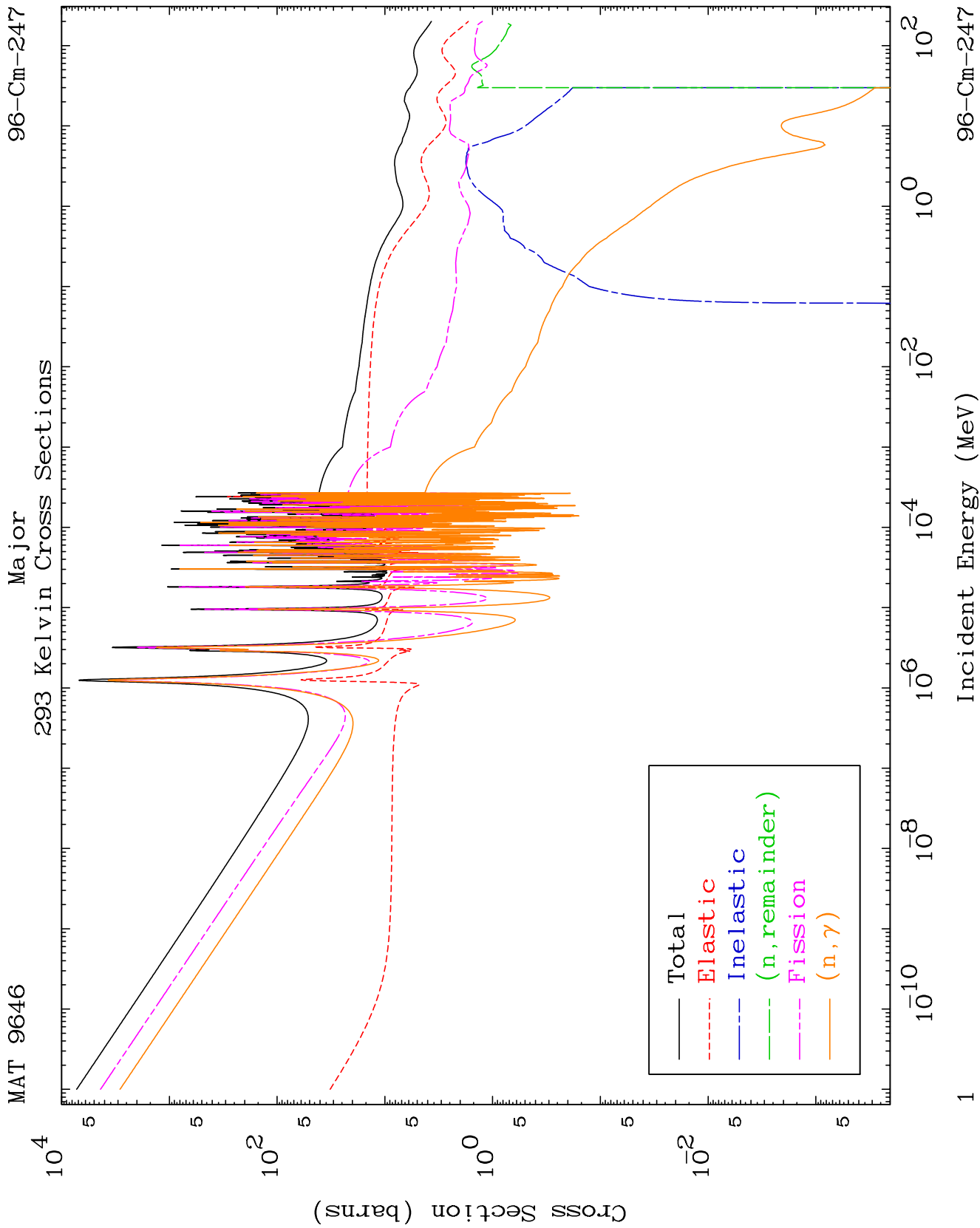
Dermott E. Cullen
(Present Contact Information)

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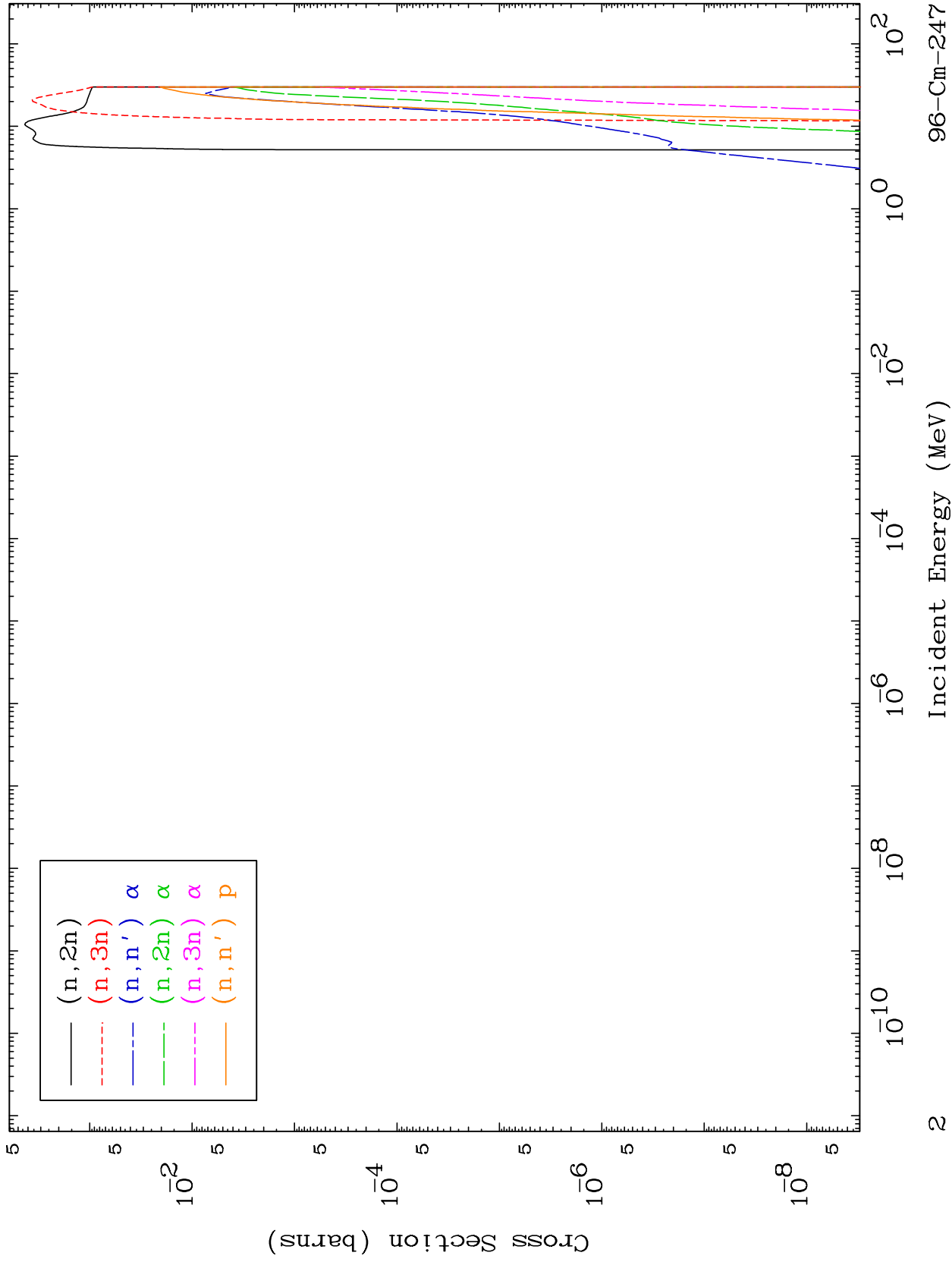
Press Mouse Button to Start



MAT 9646

Neutron Production
293 Kelvin Cross Sections

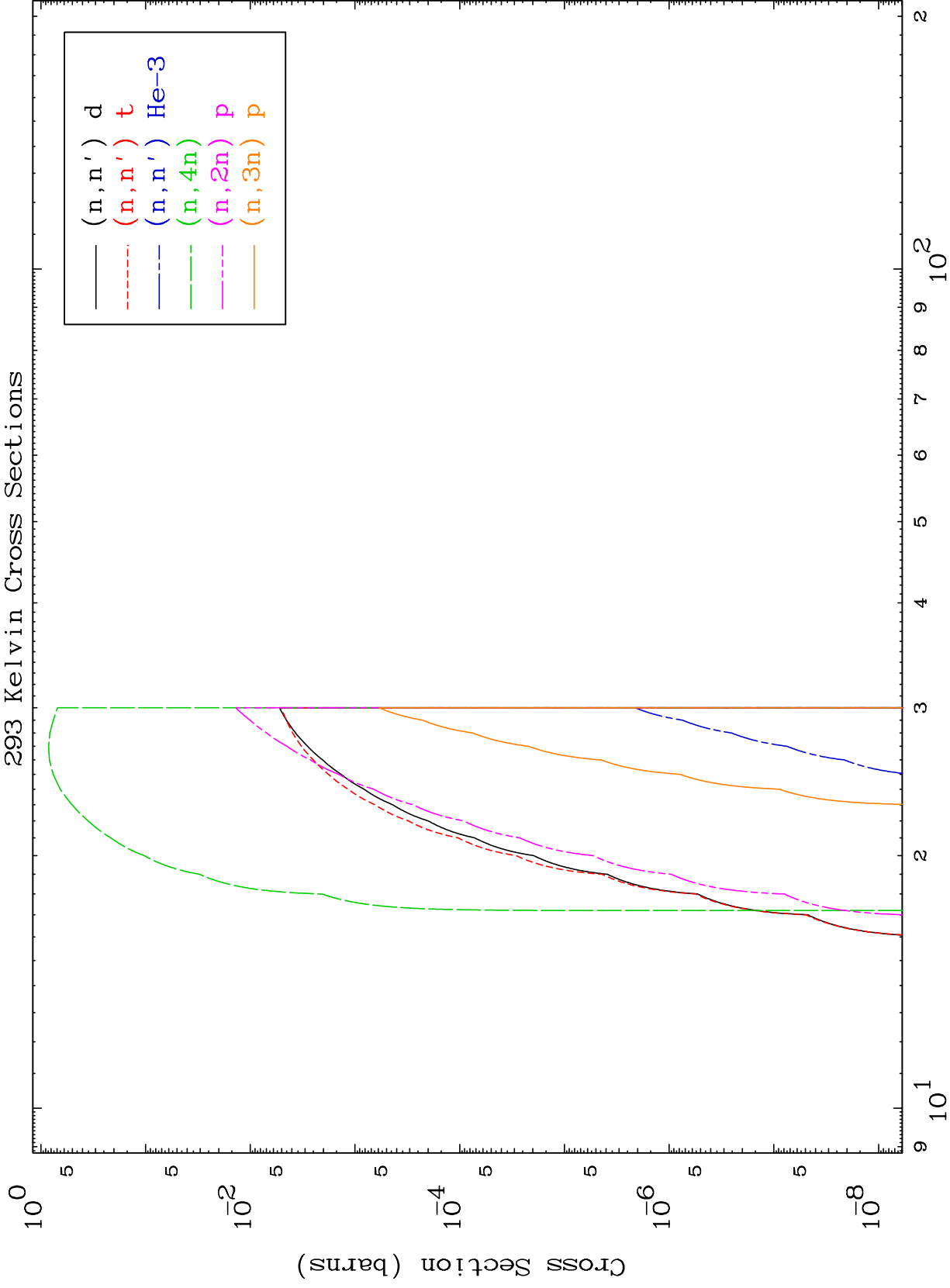
96-Cm-247



MAT 9646

Neutron Production
293 Kelvin Cross Sections

96-Cm-247



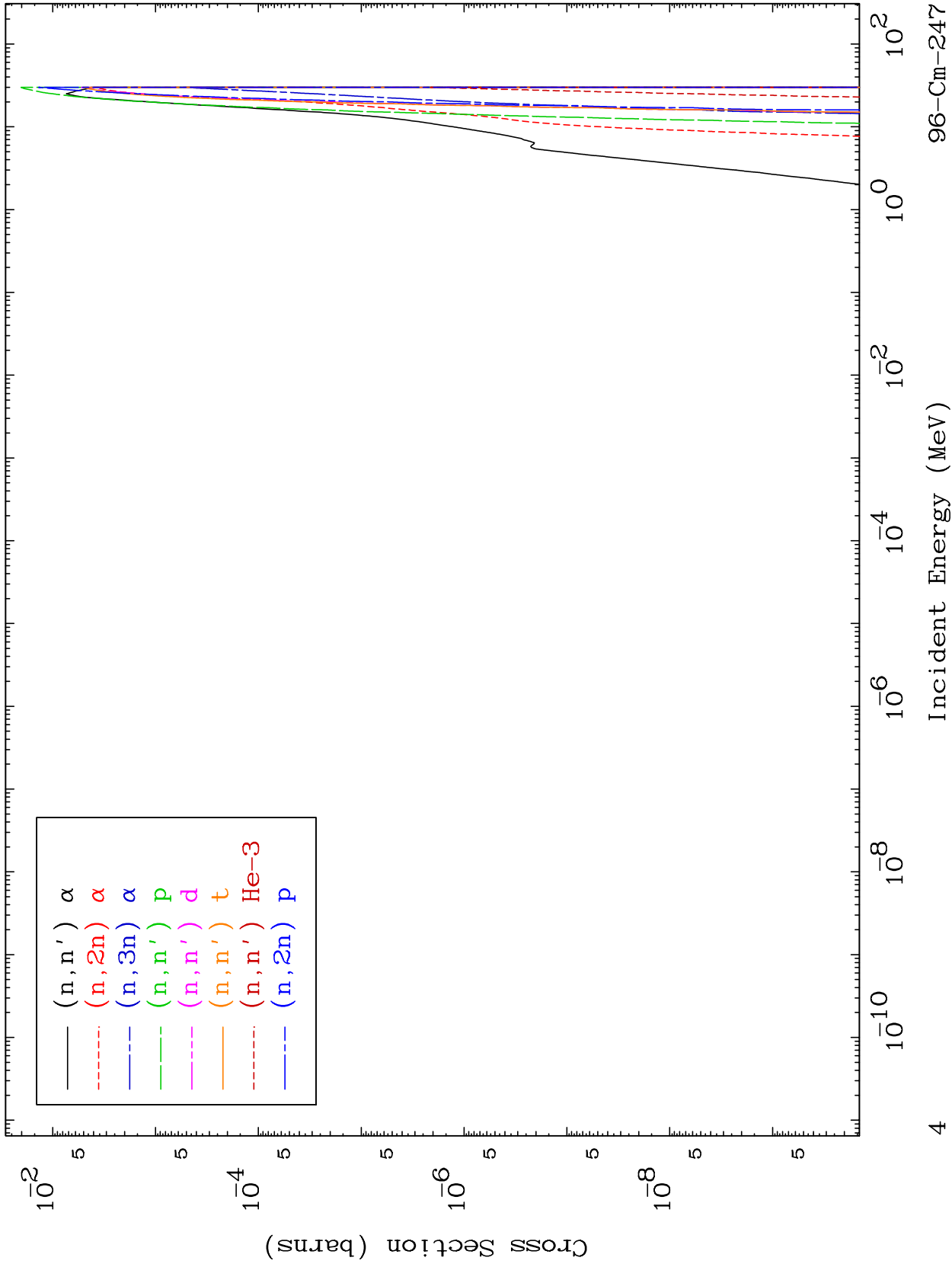
Incident Energy (MeV)

96-Cm-247

MAT 9646

Charged Particle
293 Kelvin Cross Sections

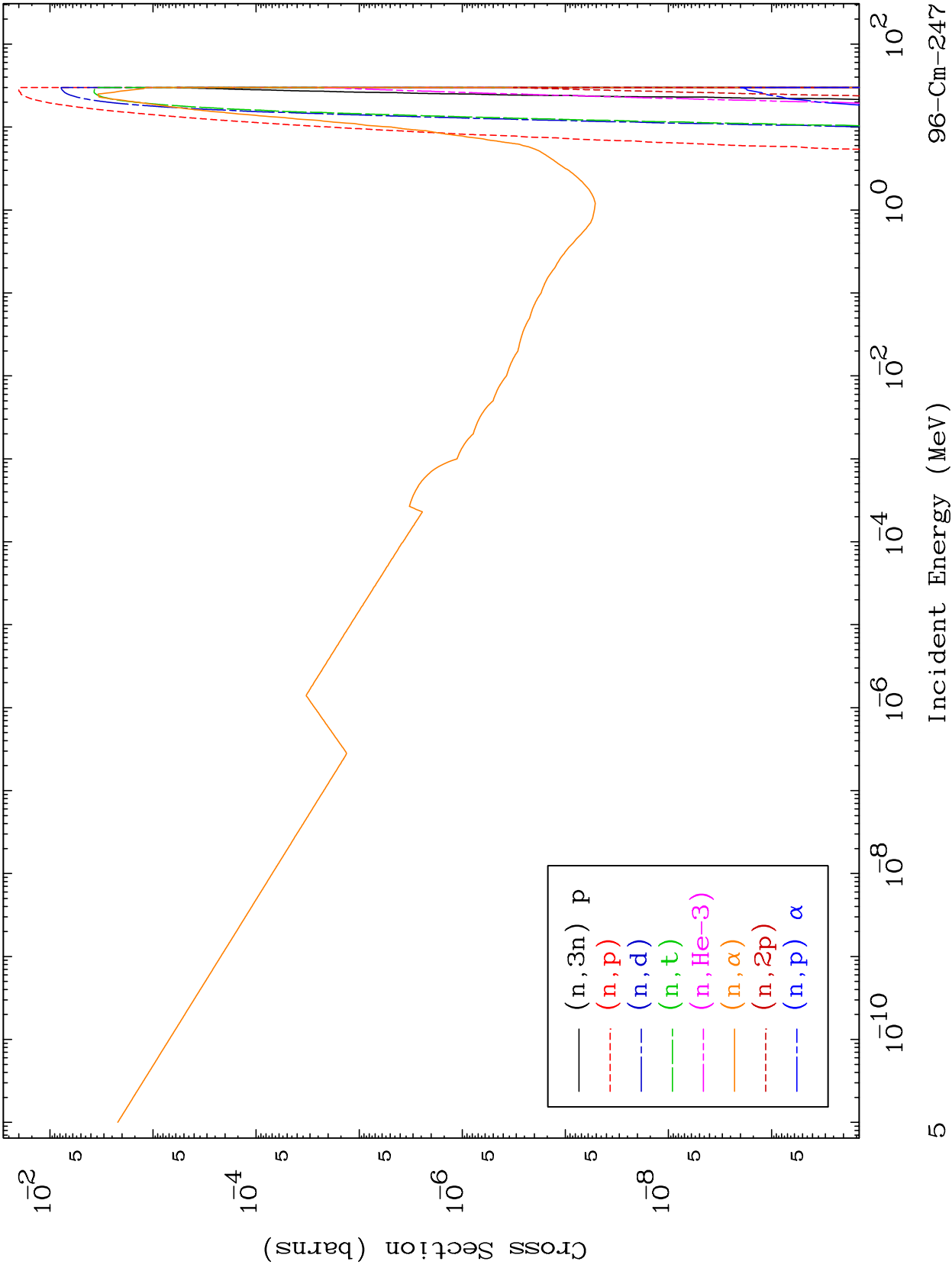
96-Cm-247



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Charged Particle
293 Kelvin Cross Sections

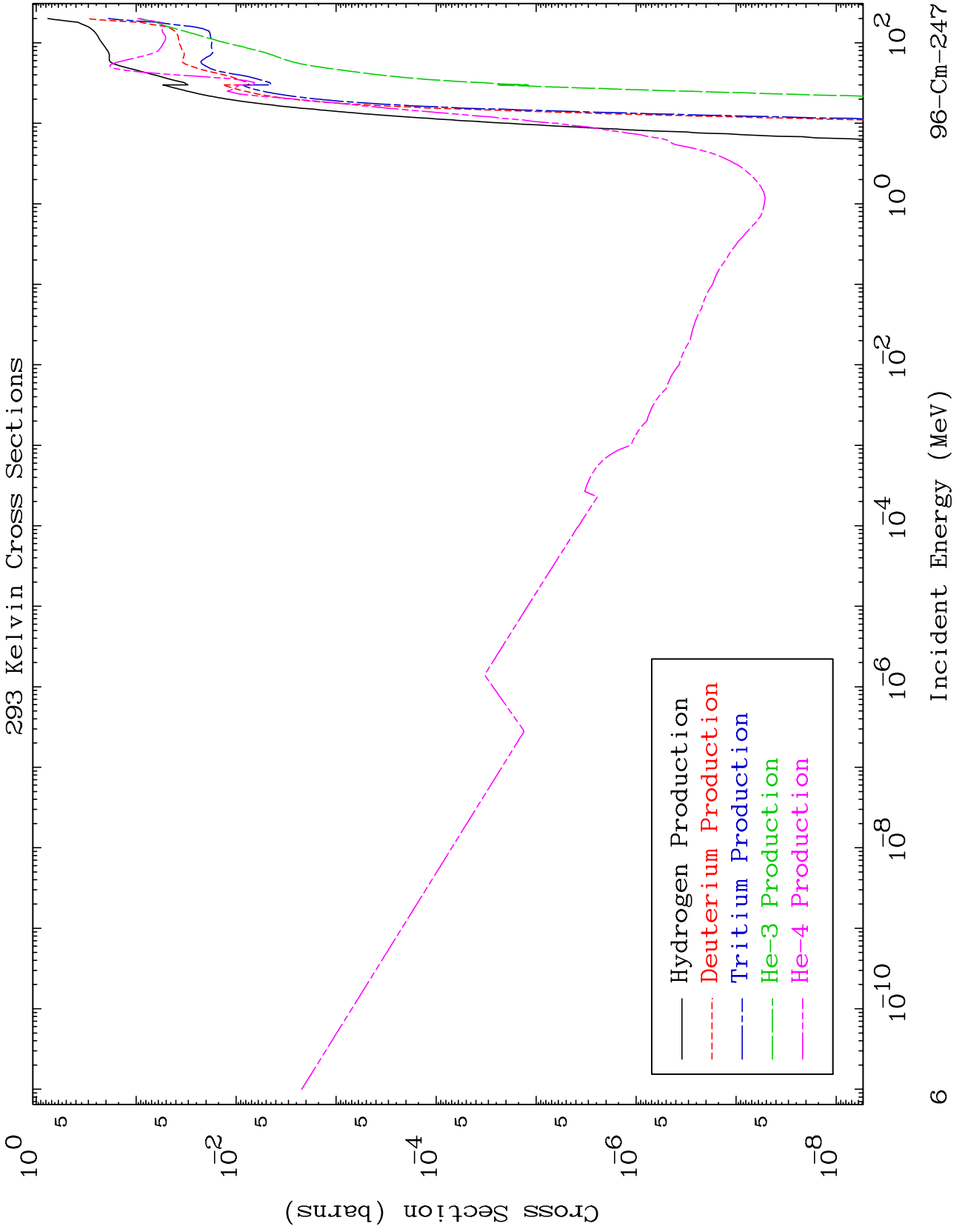
96-Cm-247



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Particle Production
293 Kelvin Cross Sections

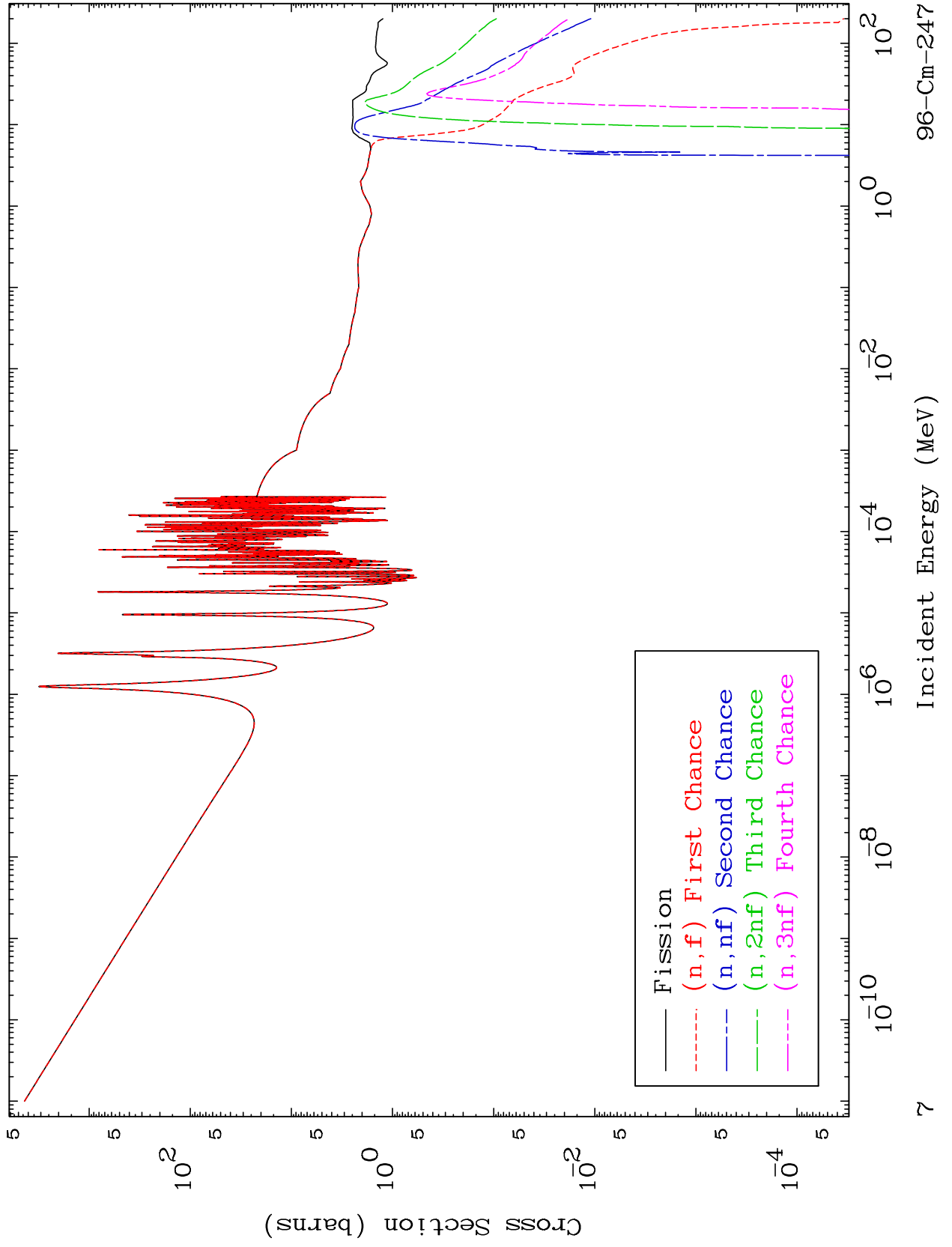
96-Cm-247



MAT 9646

Fission
293 Kelvin Cross Sections

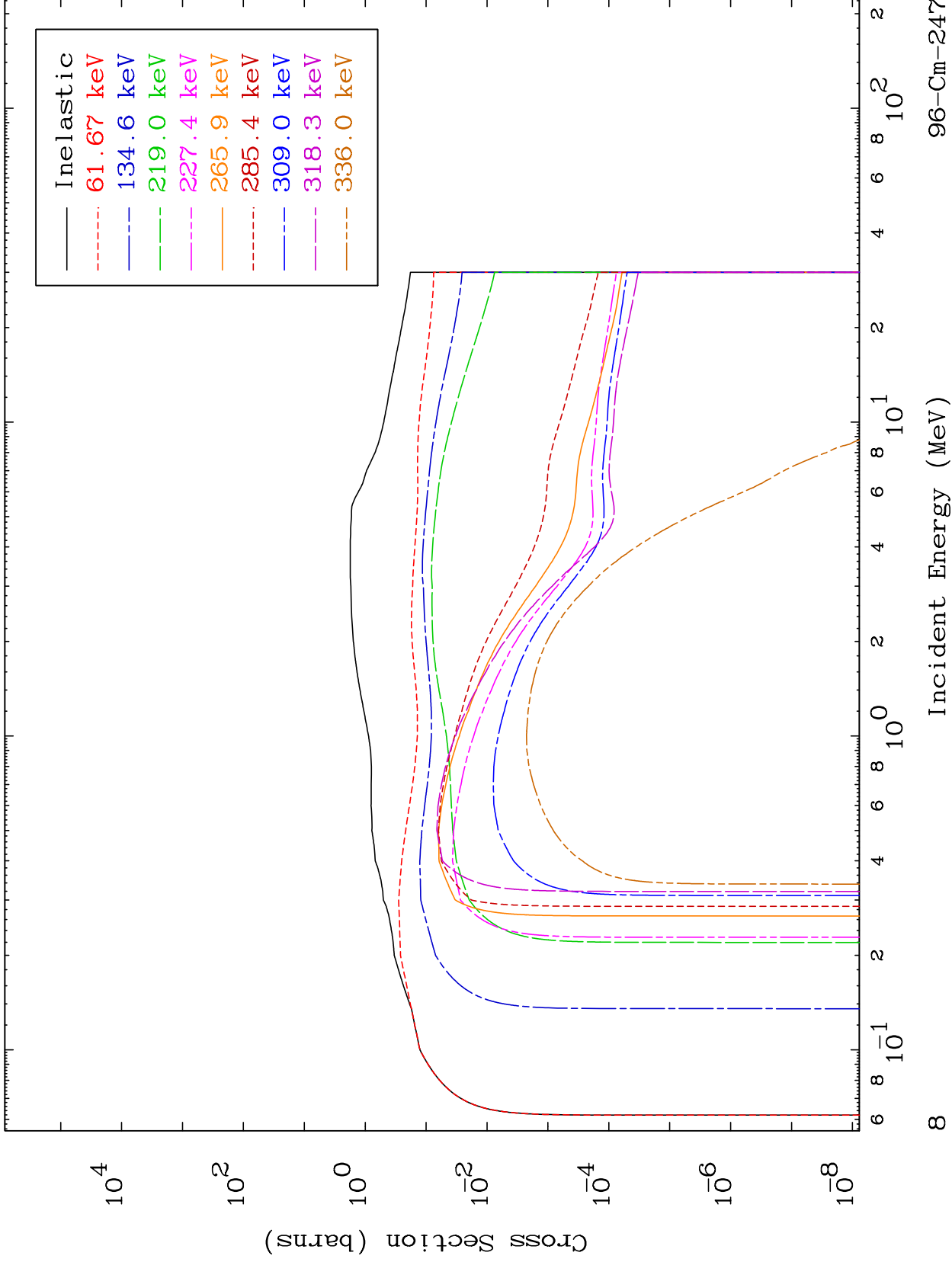
96-Cm-247



MAT 9646

(n,n') Level
293 Kelvin Cross Sections

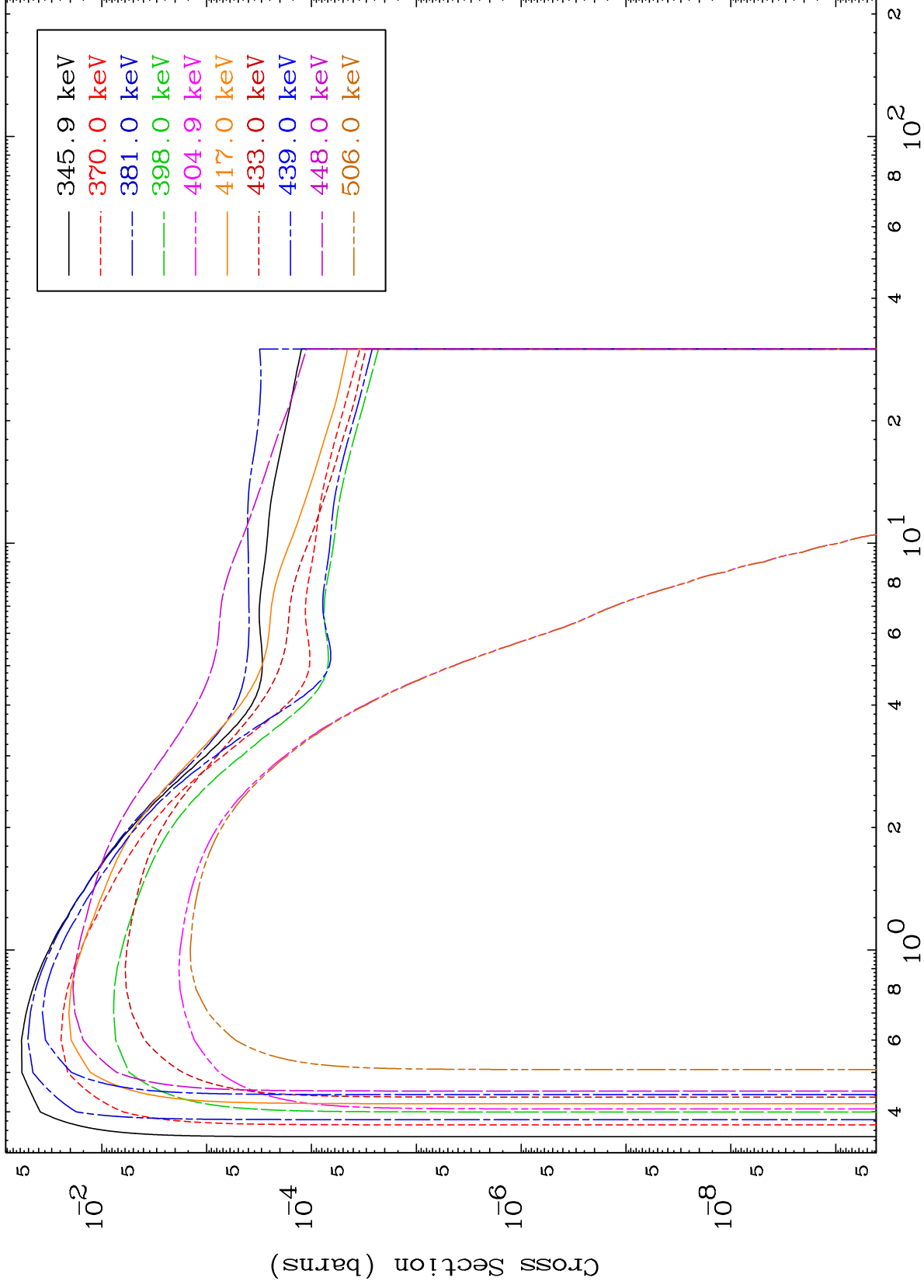
96-Cm-247



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(n,n') Level
293 Kelvin Cross Sections

96-Cm-247



9

Incident Energy (MeV)

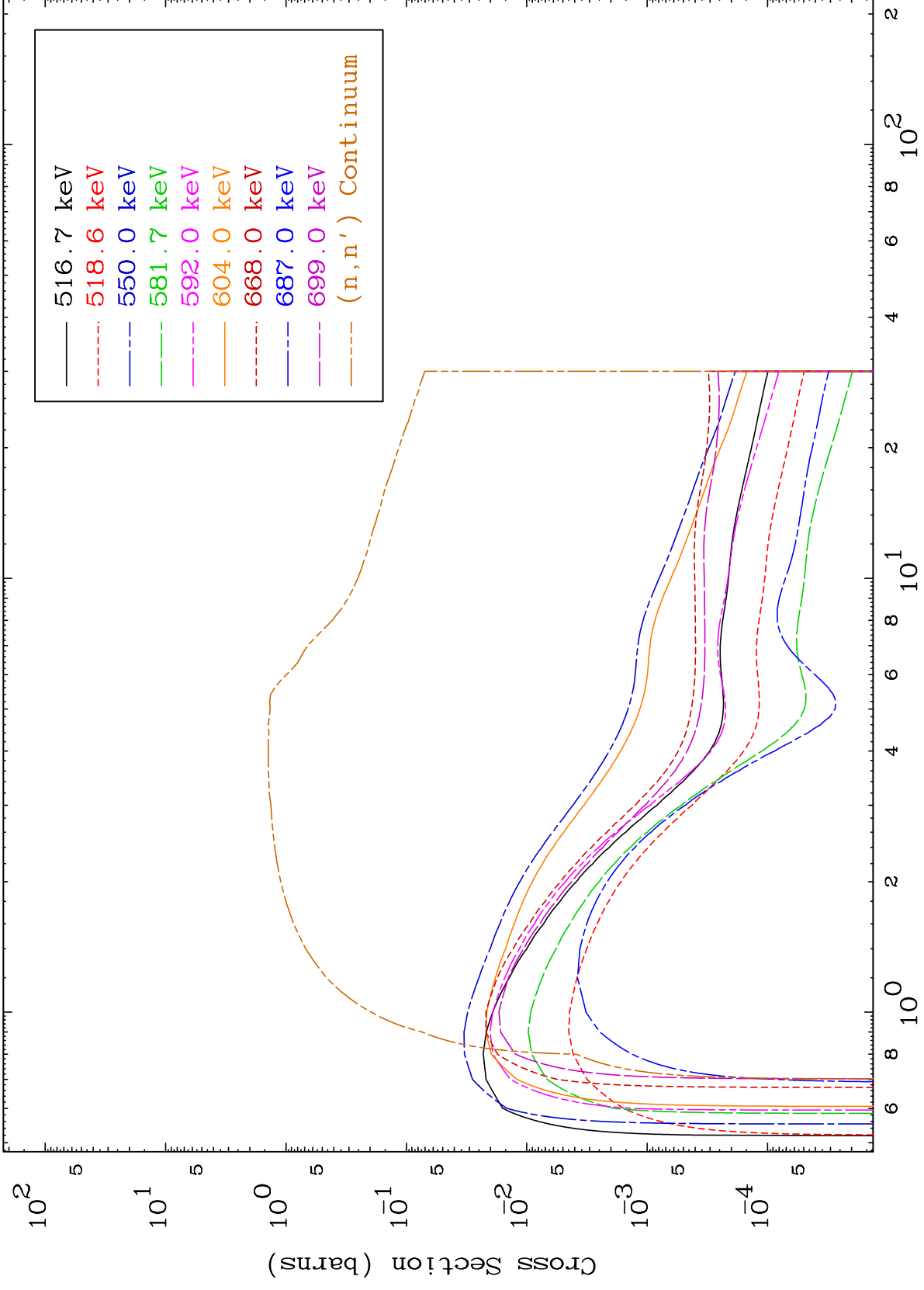
96-Cm-247

MAT 9646

(n,n') Level

96-Cm-247

293 Kelvin Cross Sections



10

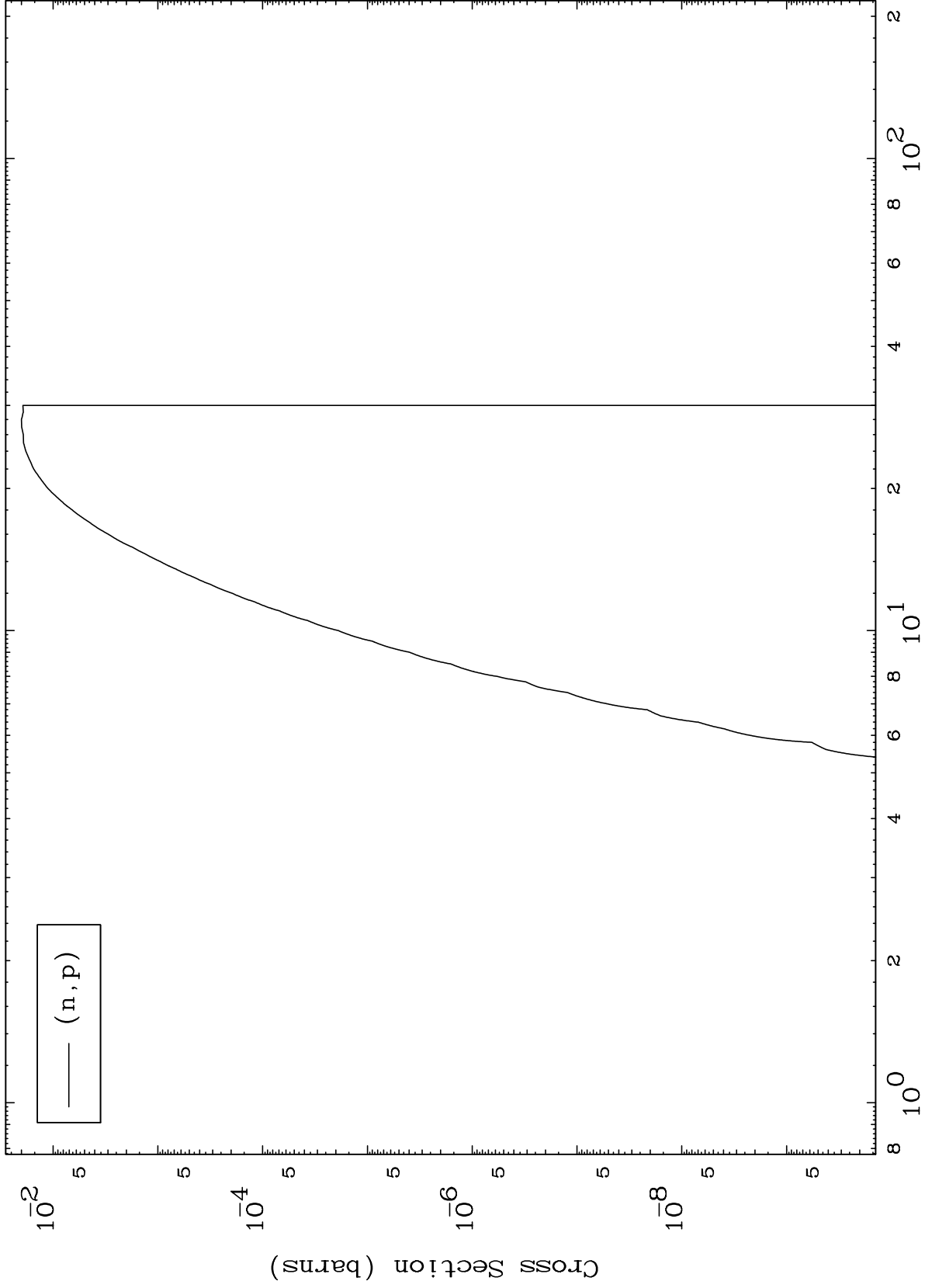
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,p) Levels
293 Kelvin Cross Sections

96-Cm-247



(n,p)

11

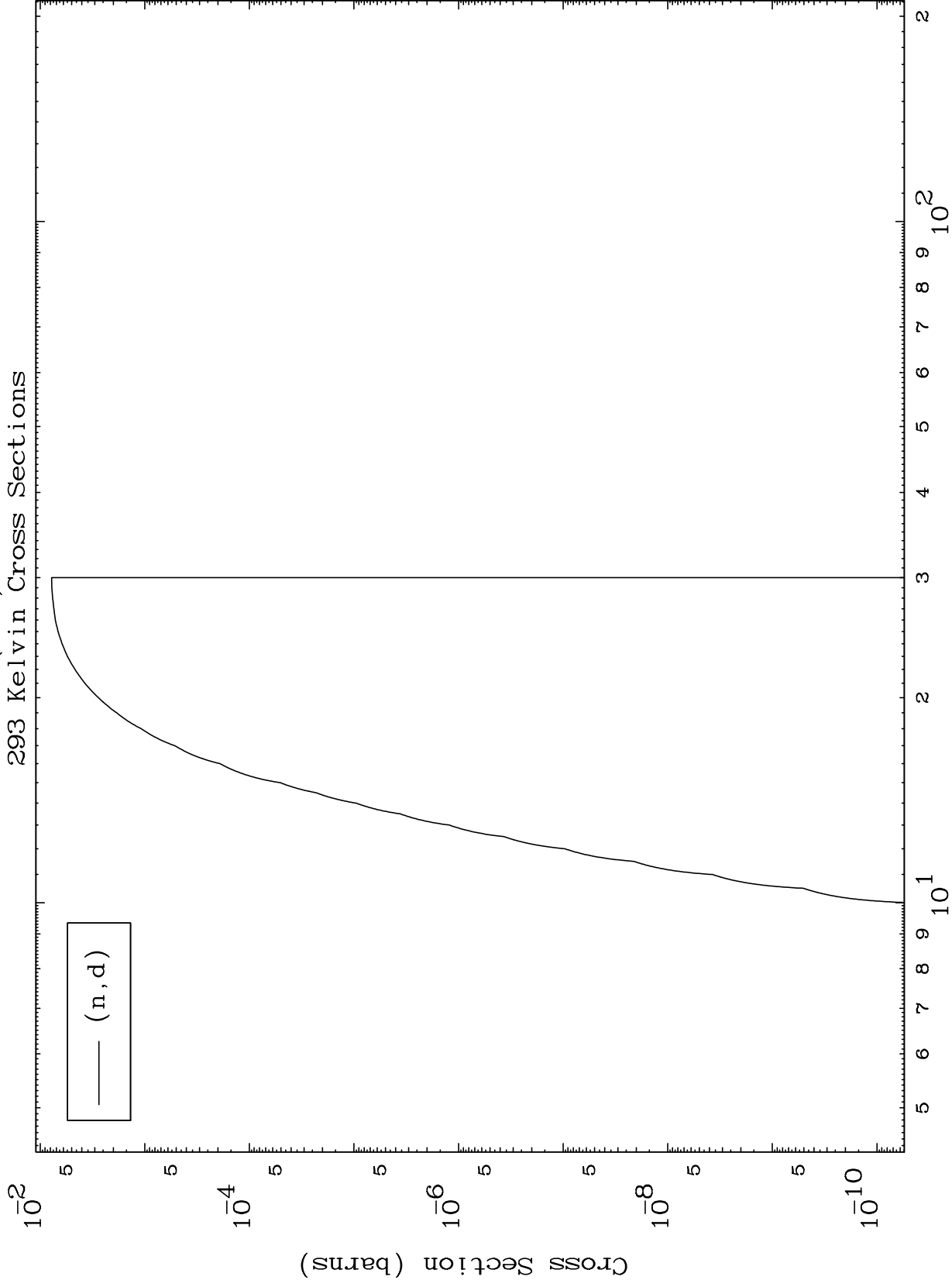
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,d) Levels
293 Kelvin Cross Sections

96-Cm-247



12

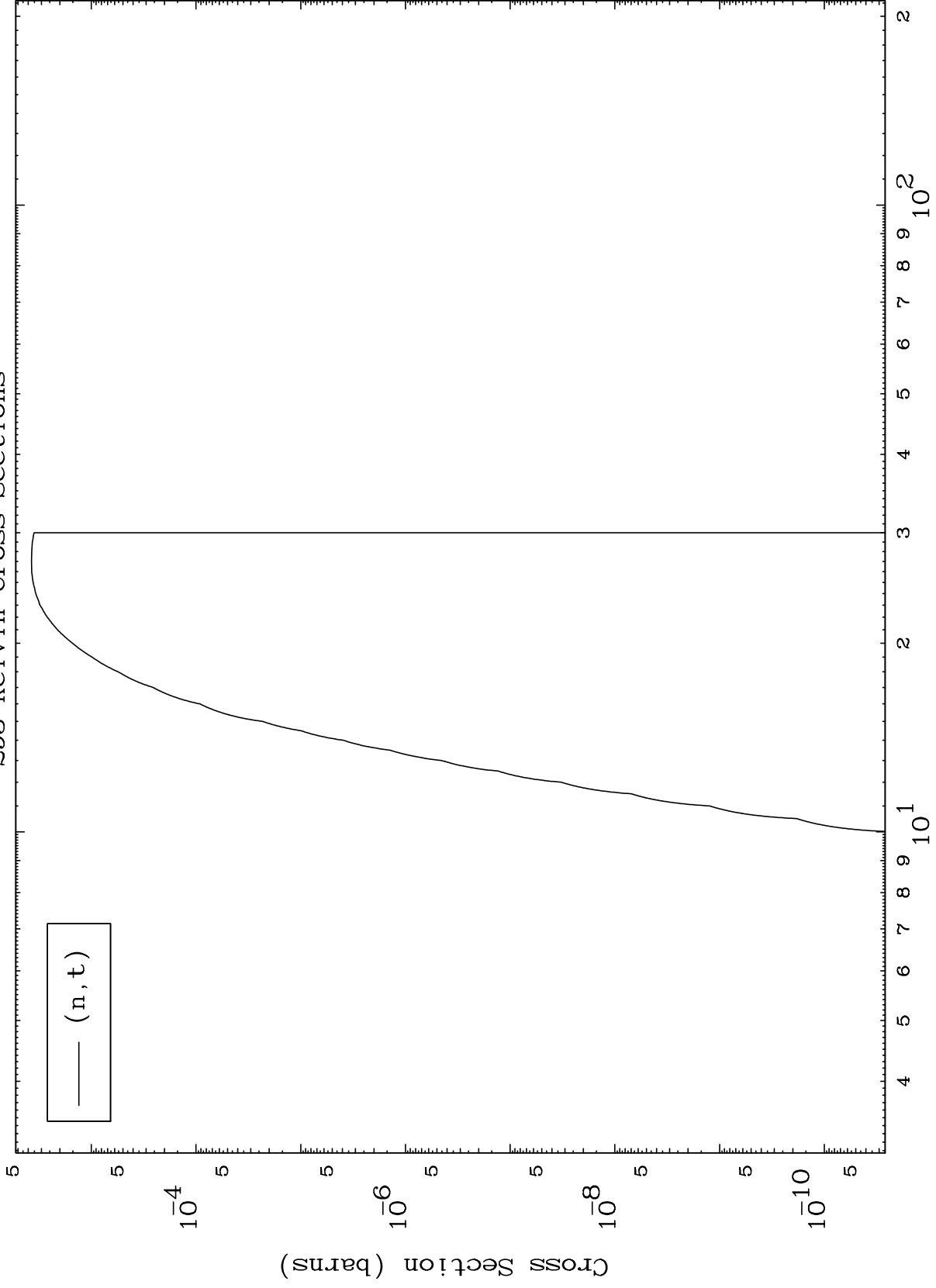
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,t) Levels
293 Kelvin Cross Sections

96-Cm-247



13

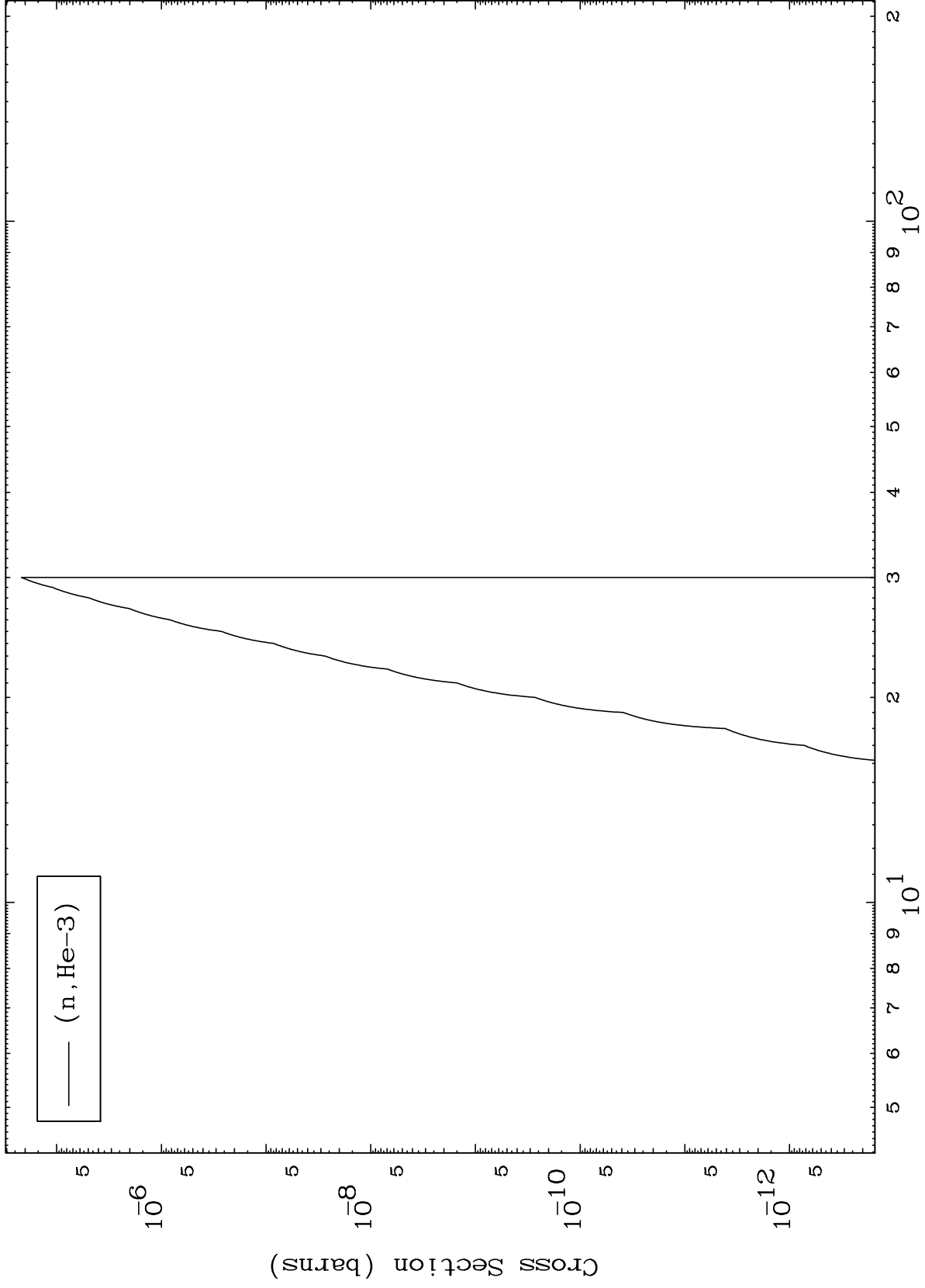
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,He3) Levels
293 Kelvin Cross Sections

96-Cm-247



14

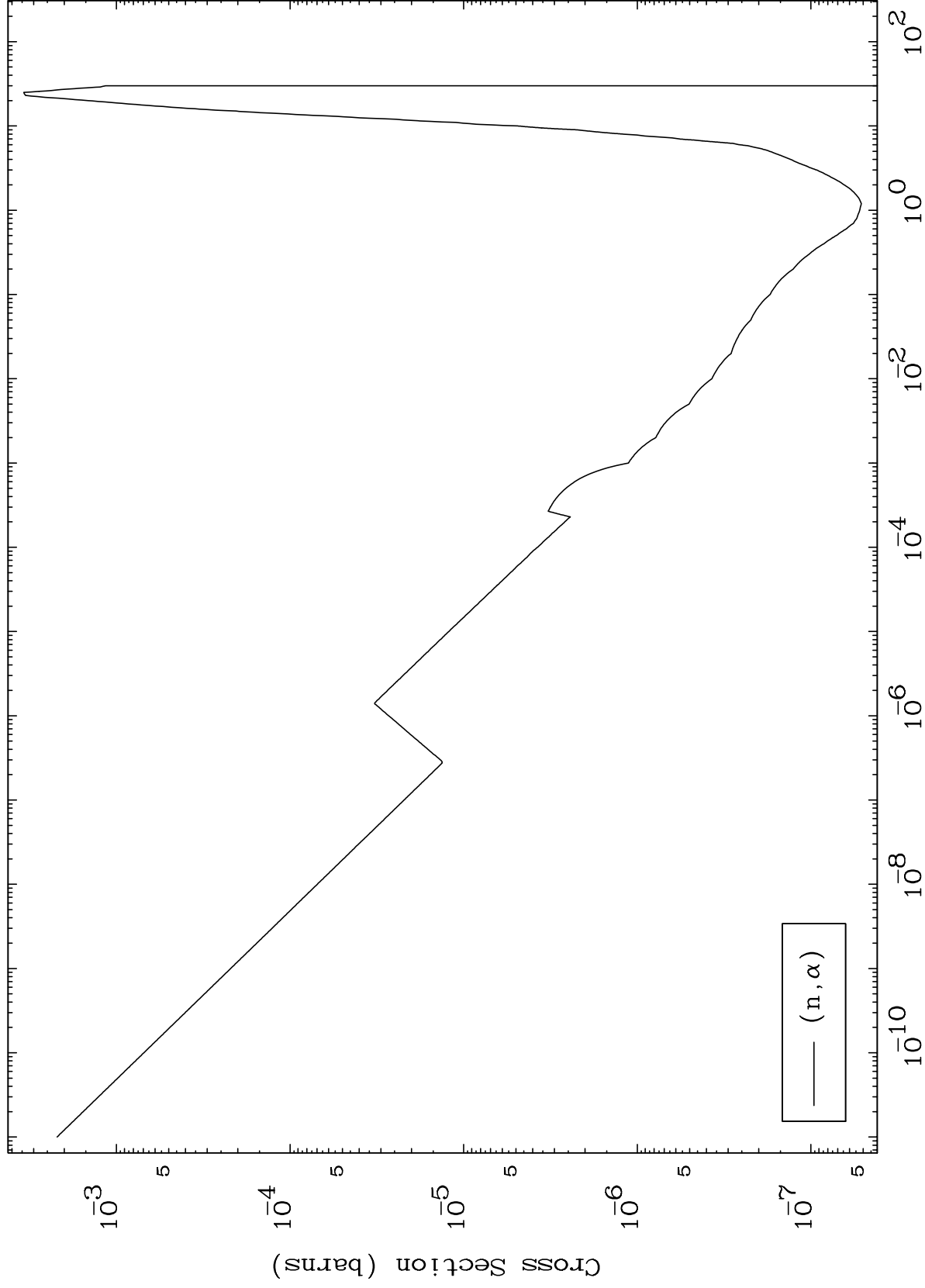
Incident Energy (MeV)

96-Cm-247

MAT 9646

(n,α) Levels
293 Kelvin Cross Sections

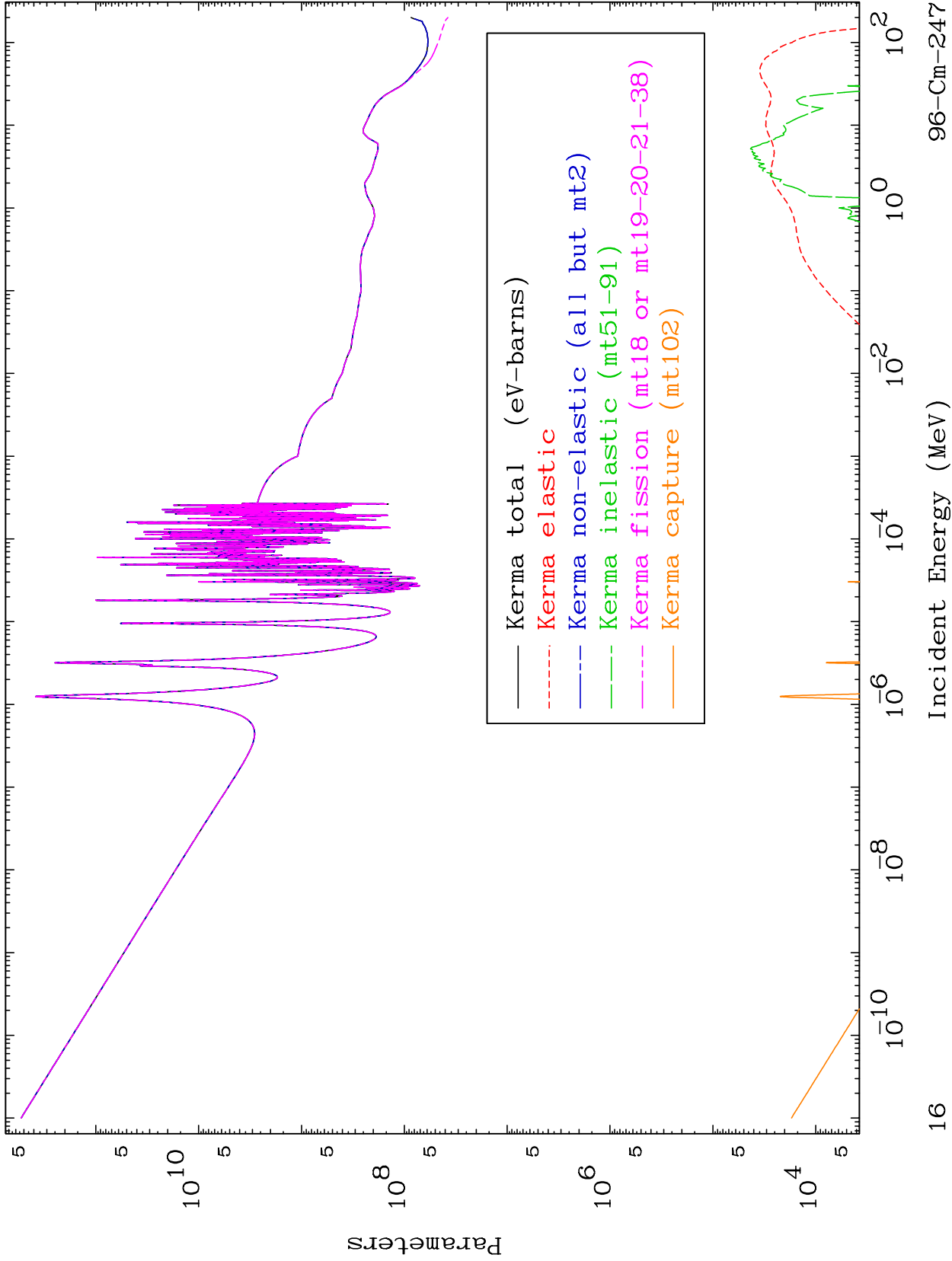
96-Cm-247

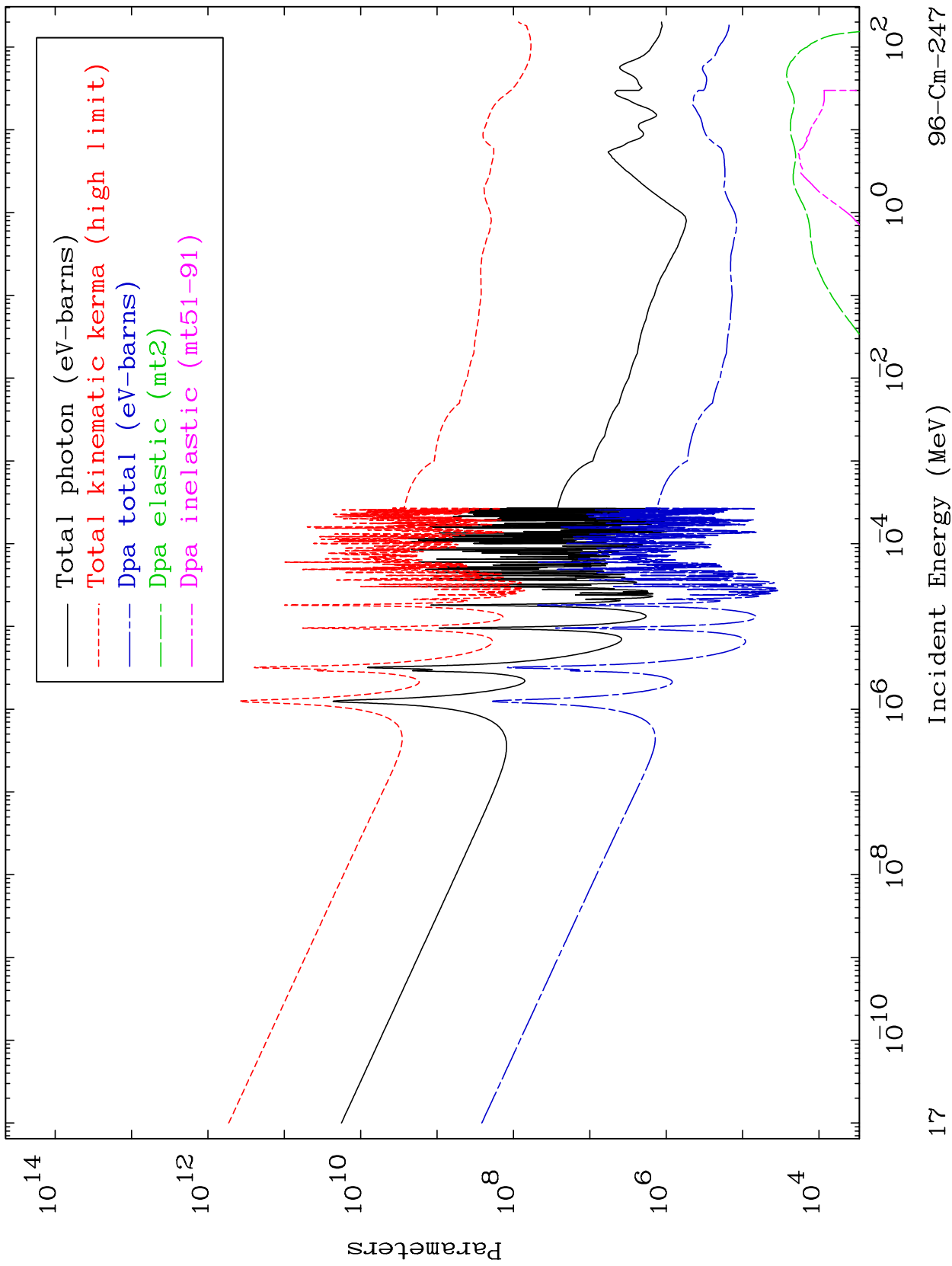


15

Incident Energy (MeV)

96-Cm-247

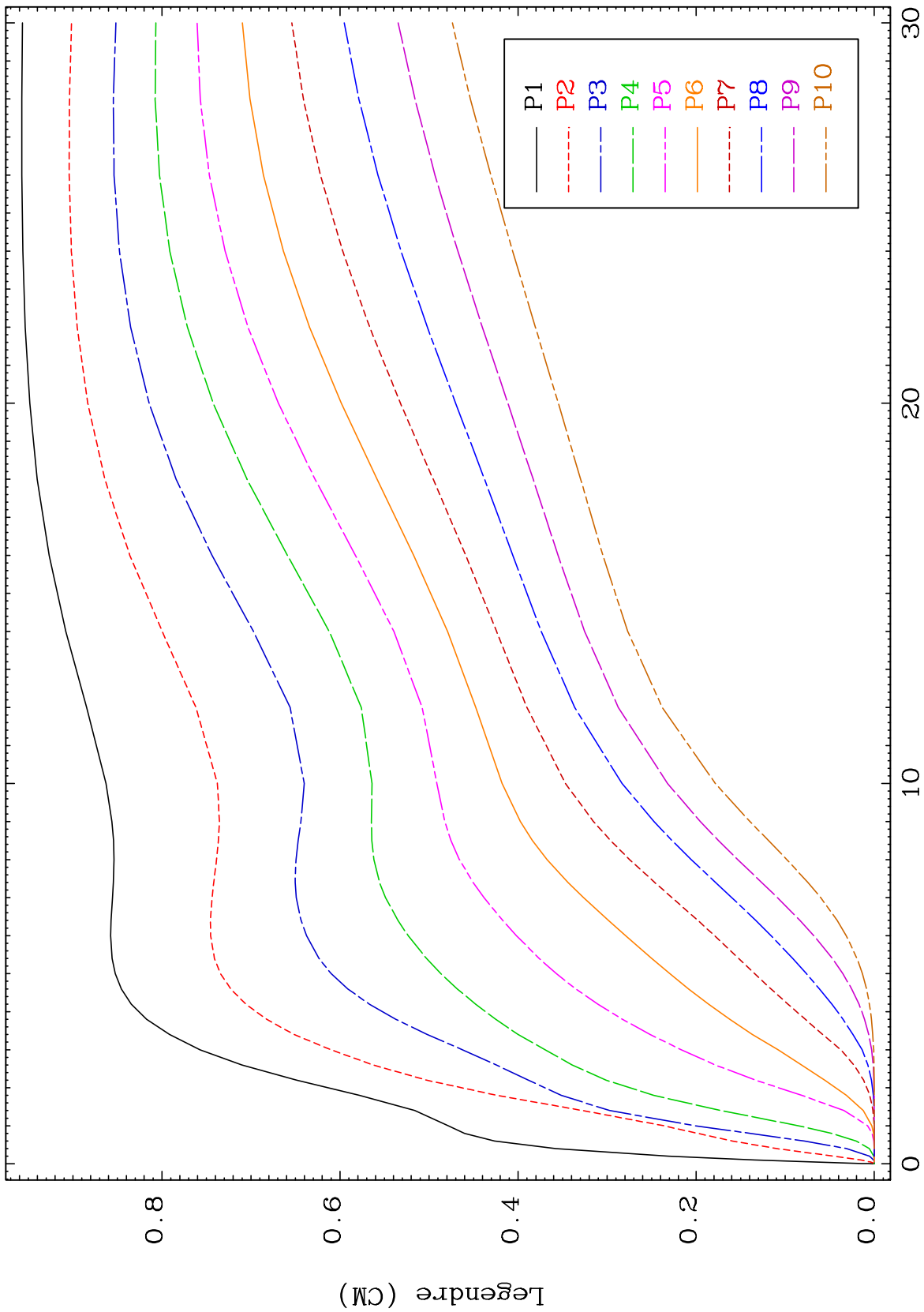




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96-Cm-247

Elastic Legendre Coefficients



96-Cm-247

Incident Energy (MeV)

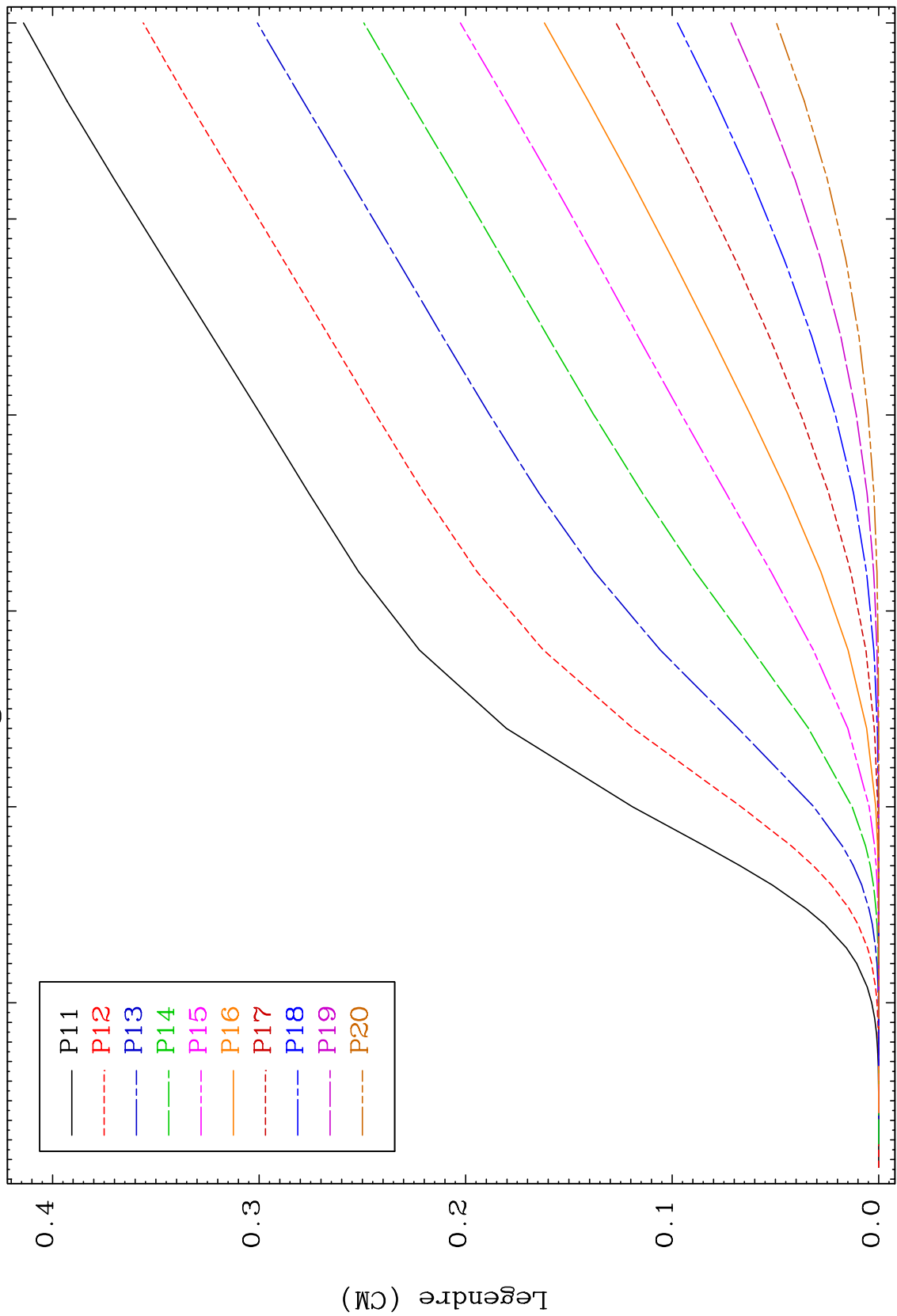
18

30

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Elastic Legendre Coefficients

96-Cm-247



19

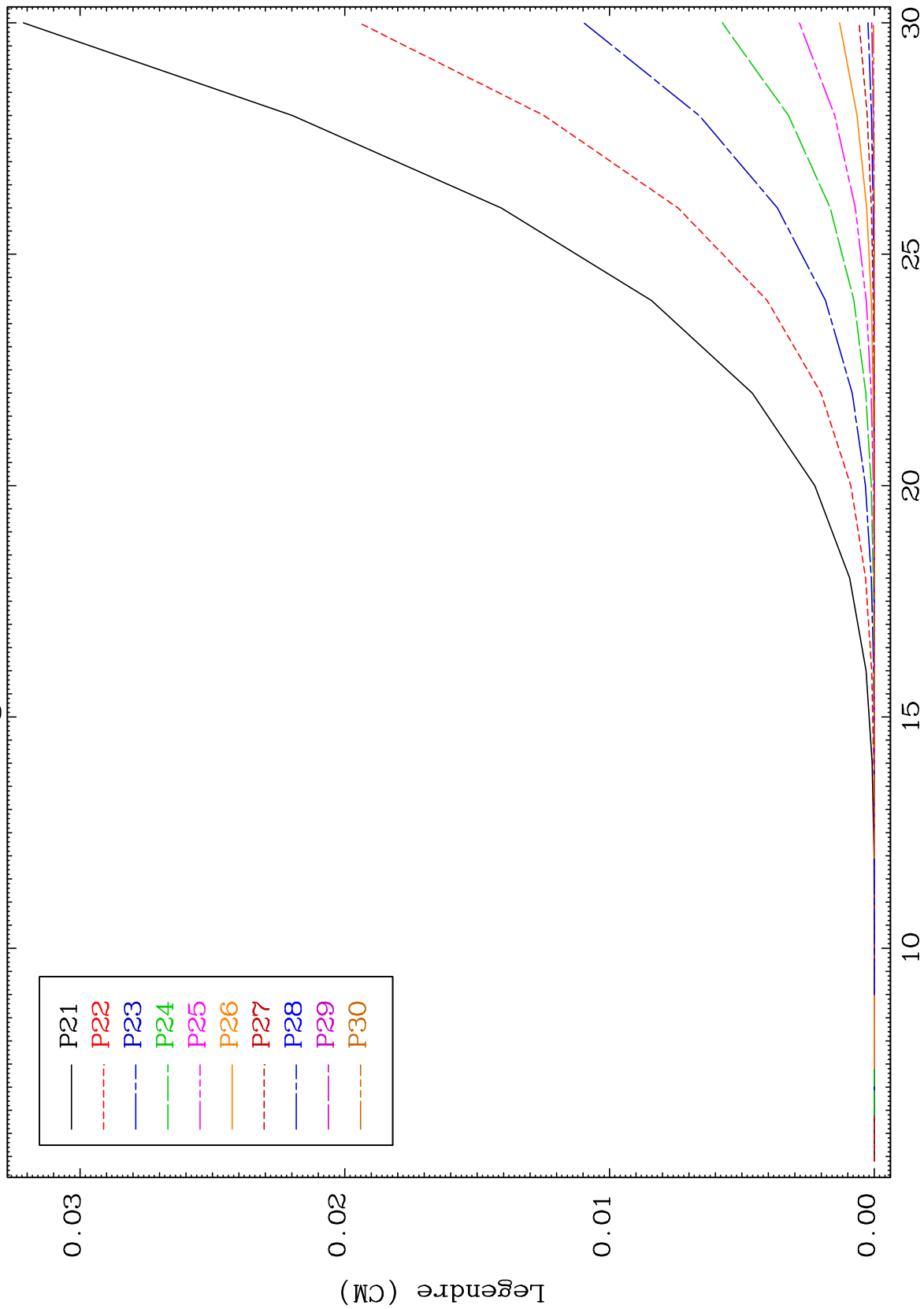
Incident Energy (MeV)

96-Cm-247

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Elastic Legendre Coefficients

96-Cm-247



20

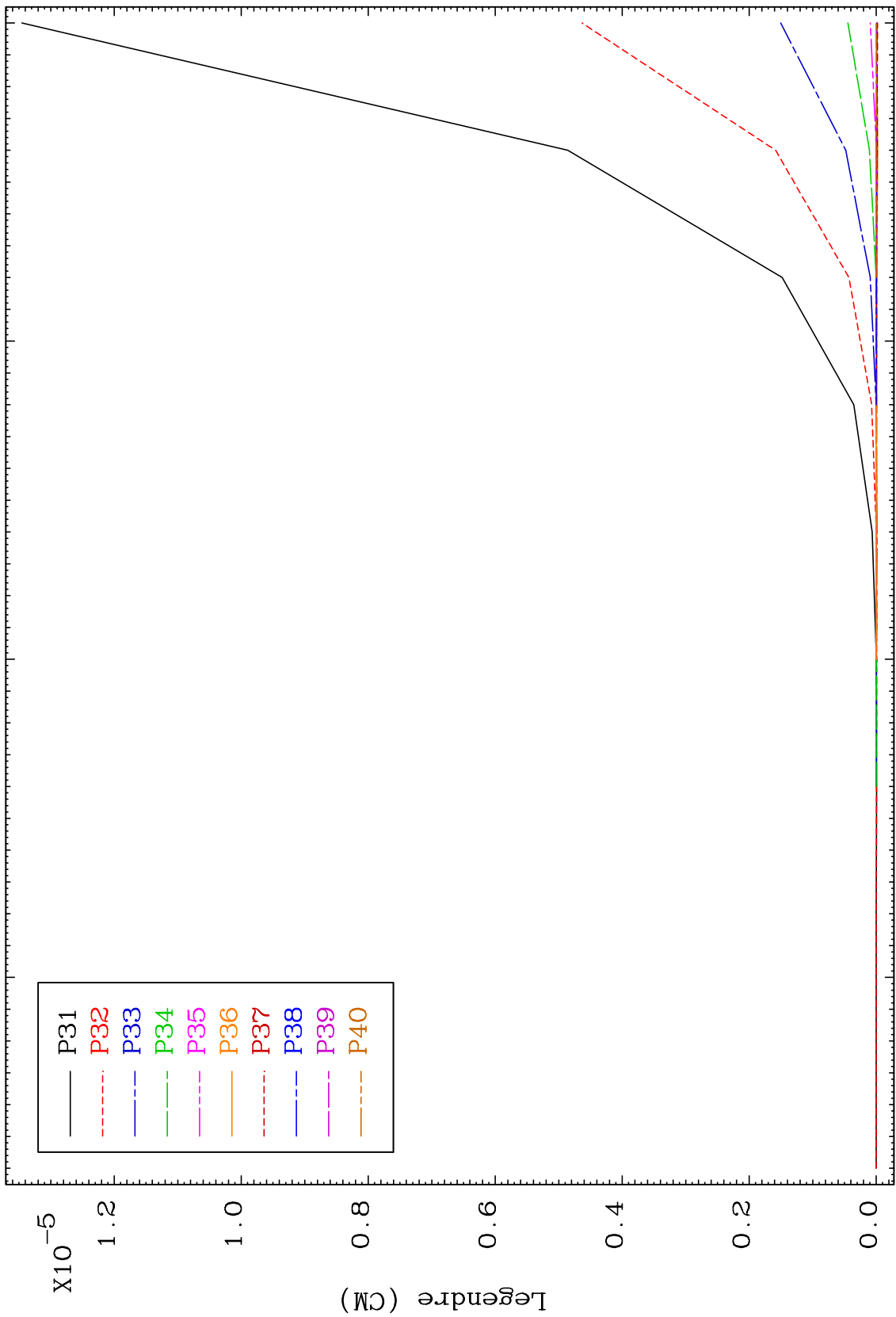
Incident Energy (MeV)

96-Cm-247

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Elastic Legendre Coefficients

96-Cm-247

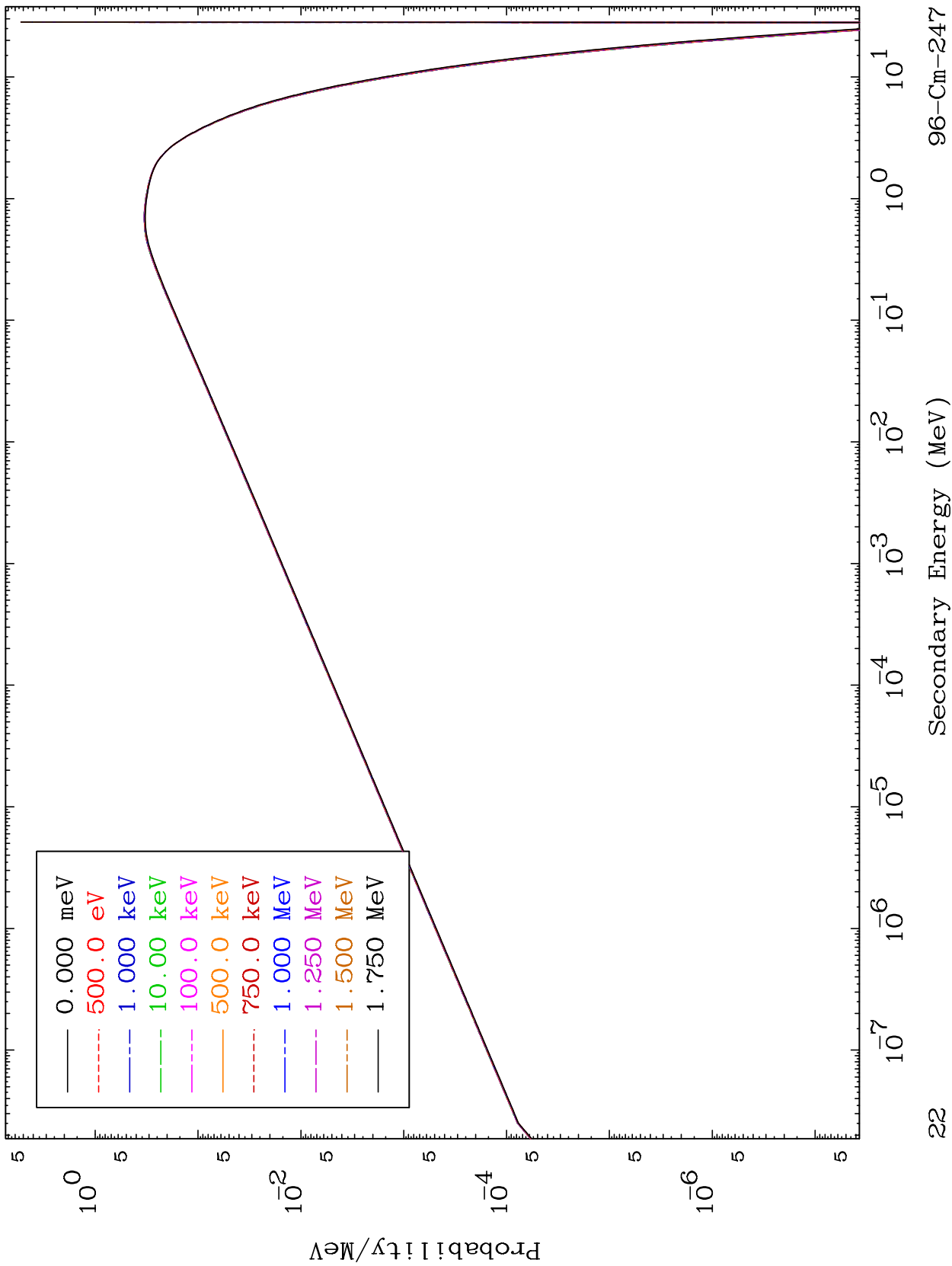


21

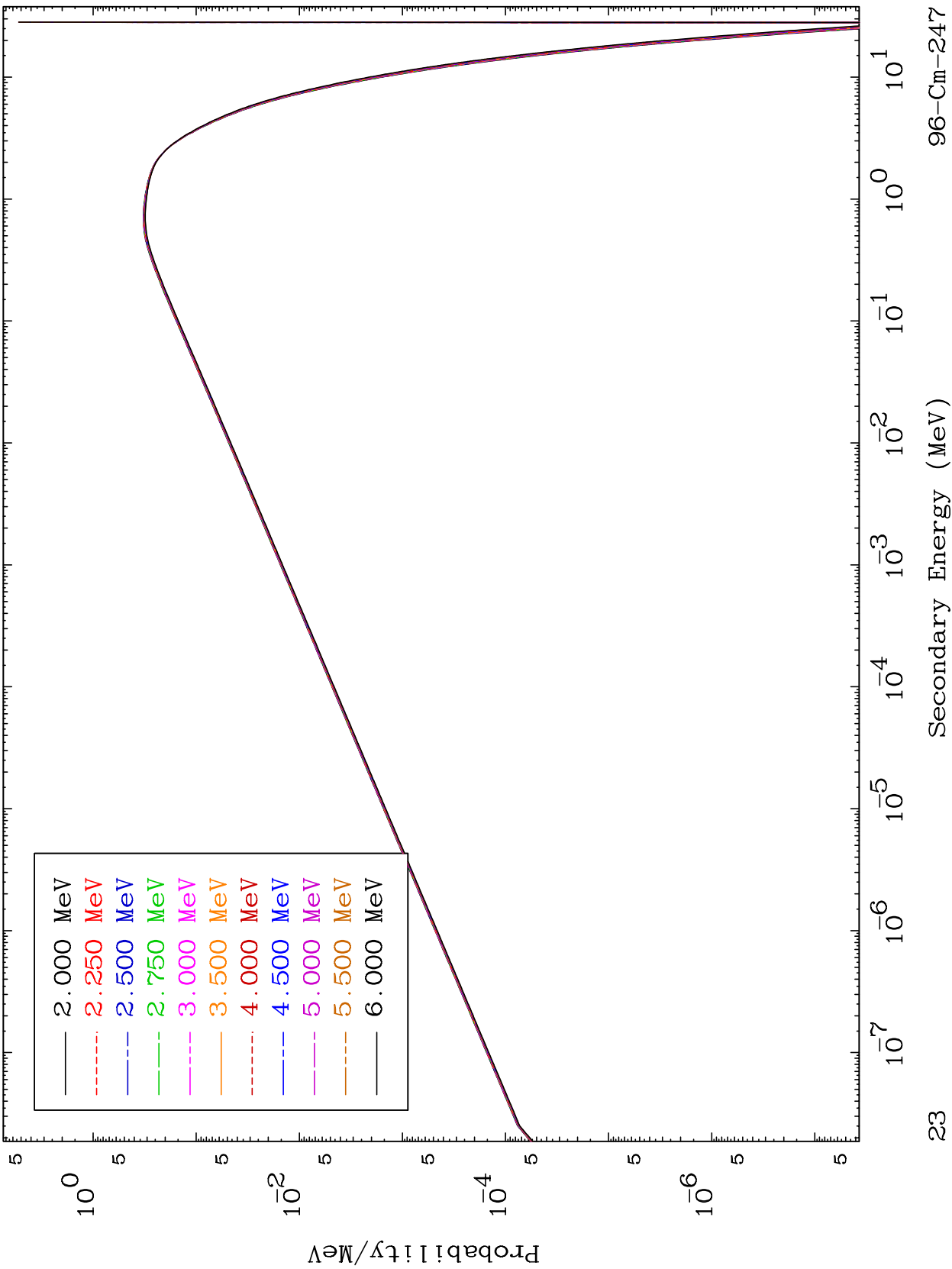
Incident Energy (MeV)

96-Cm-247

Fission Energy Distribution



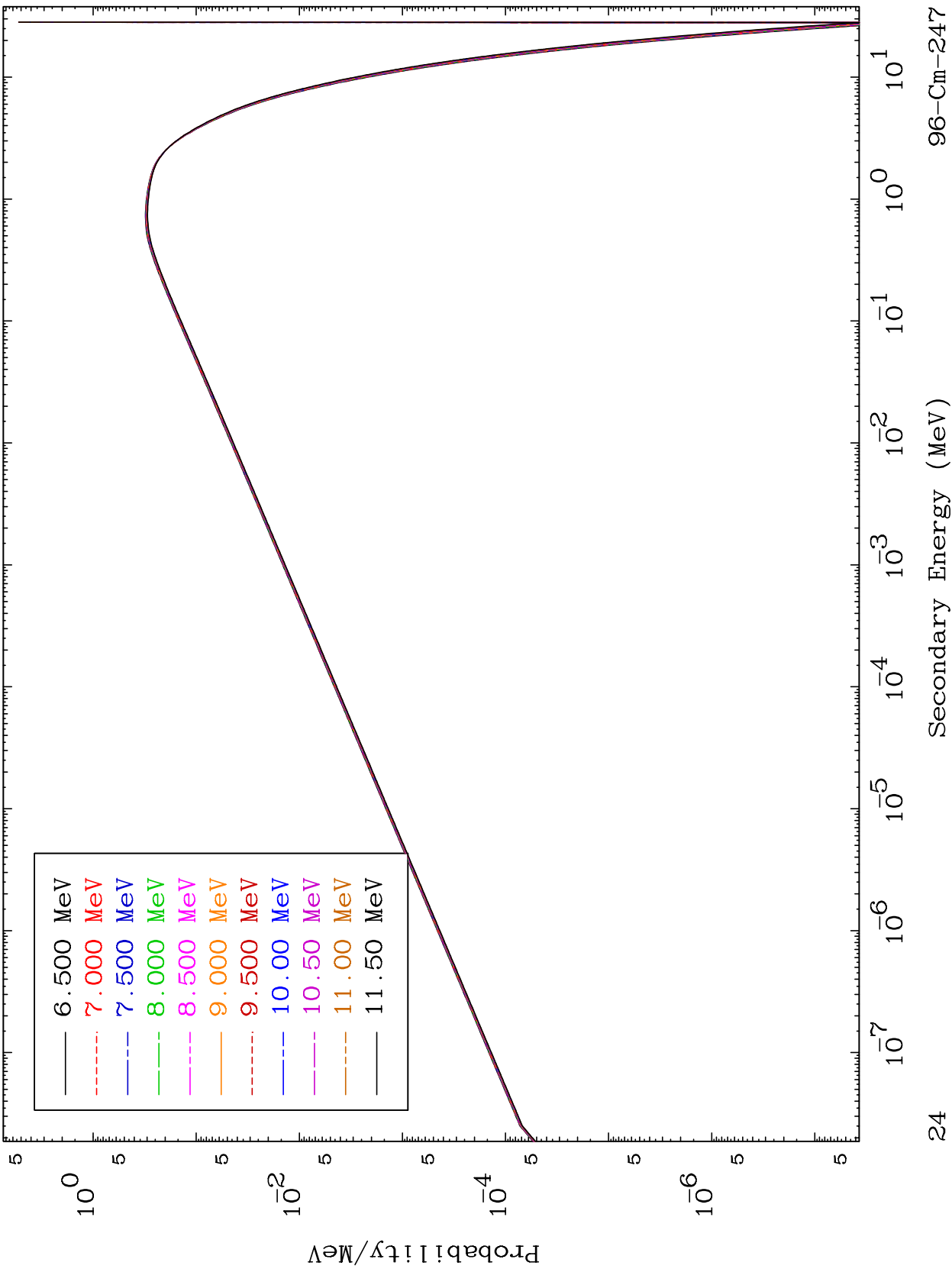
Fission Energy Distribution



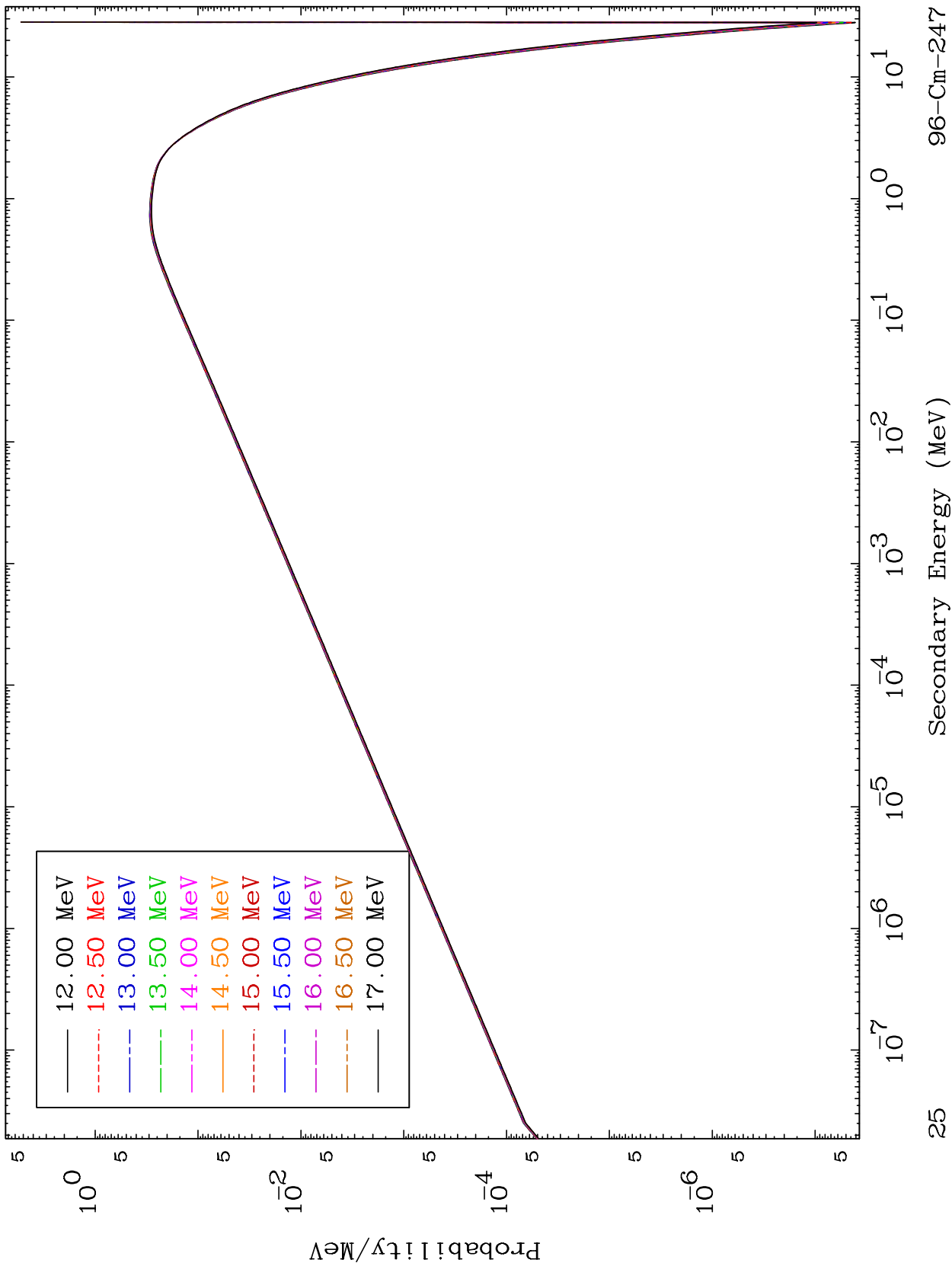
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Fission Energy Distribution

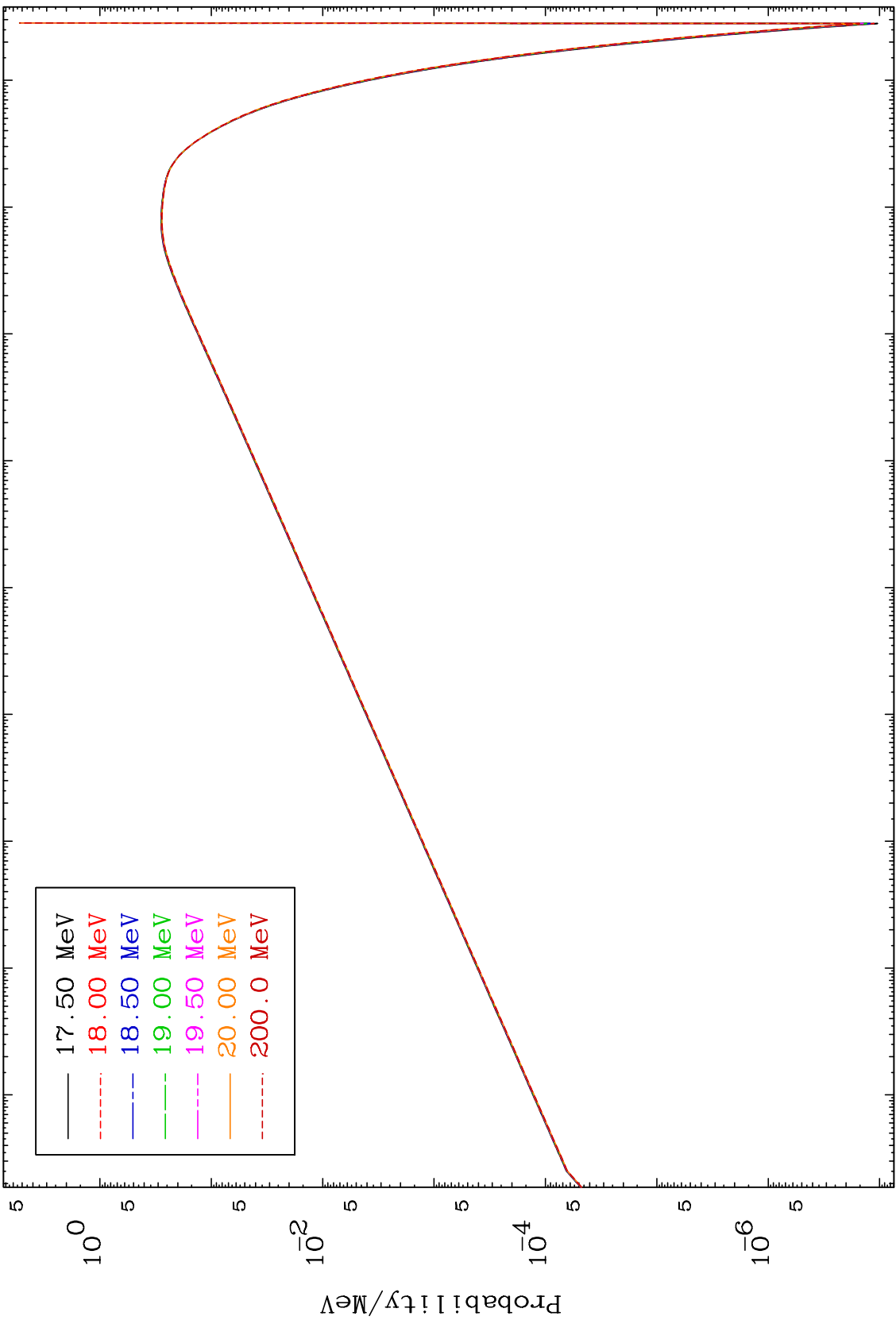
96-Cm-247



Fission Energy Distribution



Fission
Energy Distribution

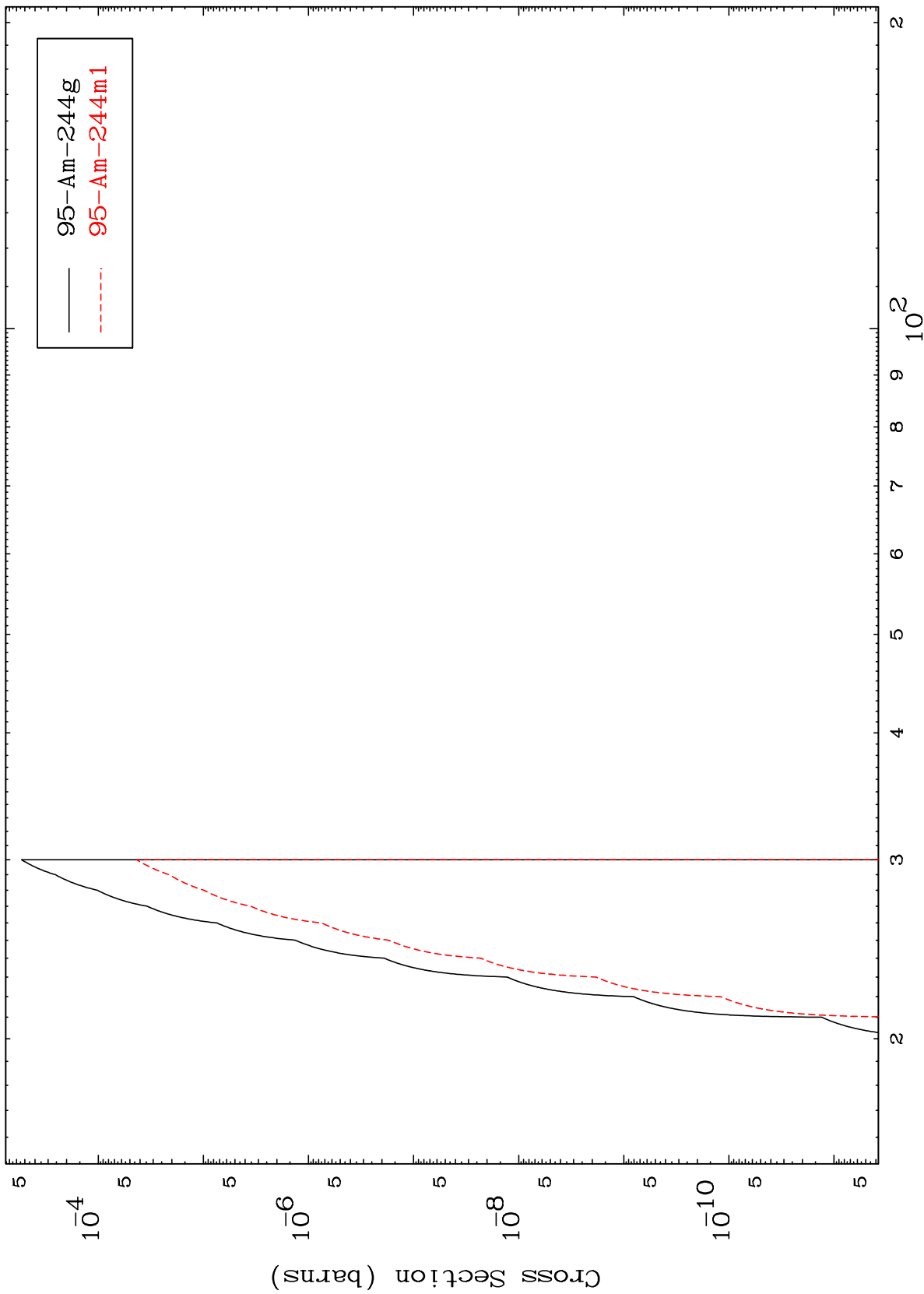


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(n,2n) d

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Radionuclide Production Cross Section



27

Incident Energy (MeV)

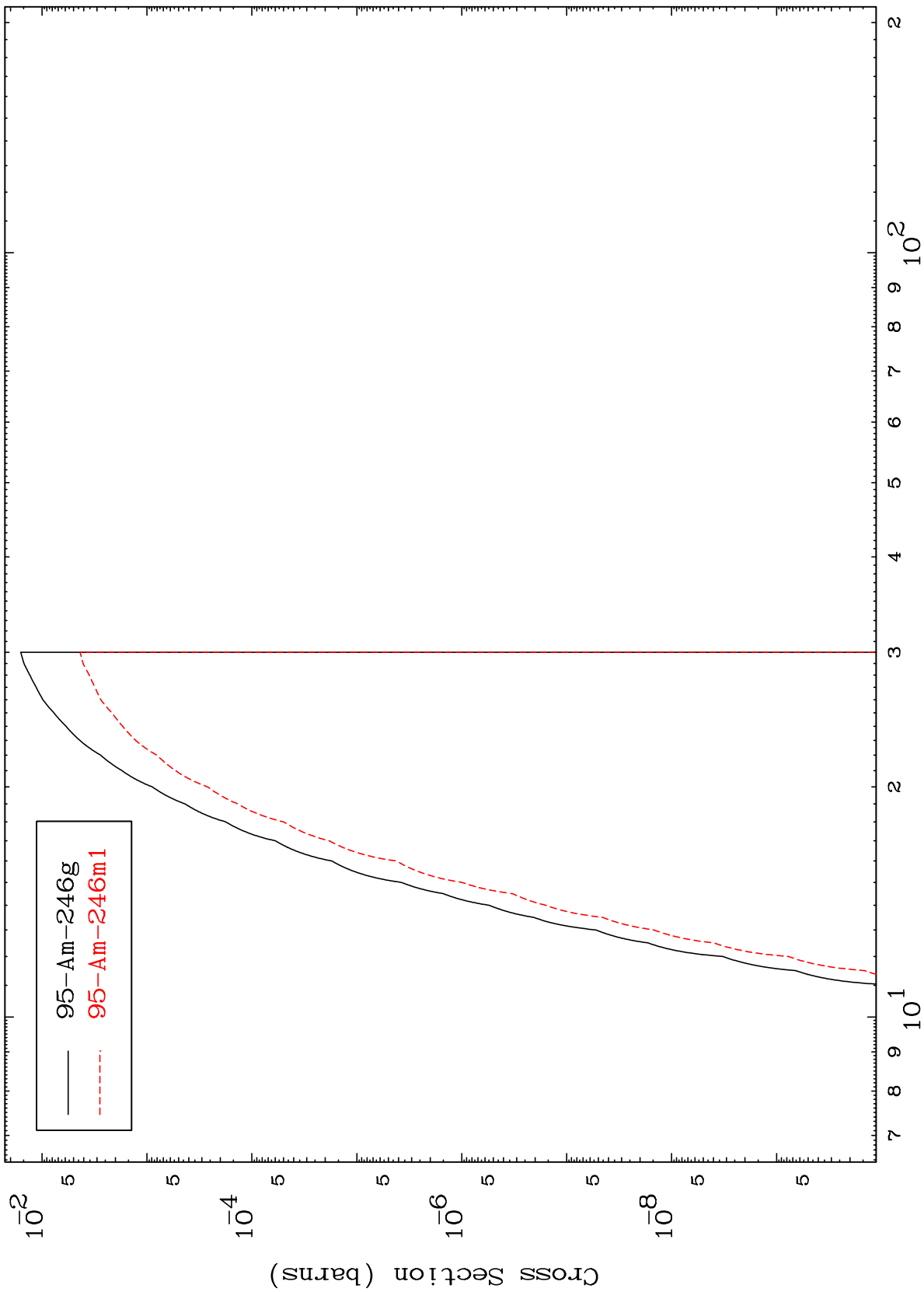
96-Cm-247

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(n,n') p

96-Cm-247

Radionuclide Production Cross Section



28

Incident Energy (MeV)

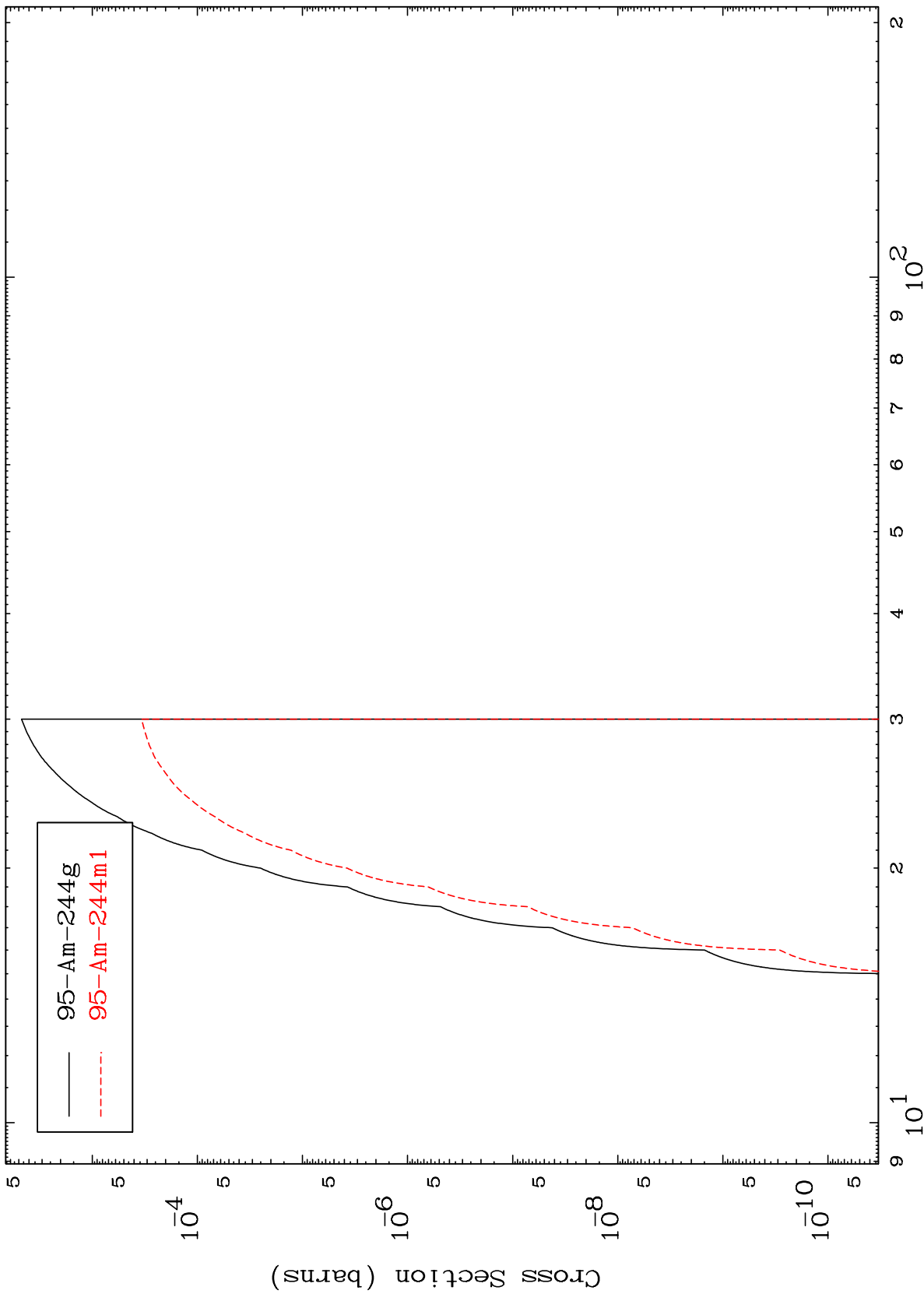
96-Cm-247

MAT 9646

(n,n') t

96-Cm-247

Radionuclide Production Cross Section



Incident Energy (MeV)

96-Cm-247

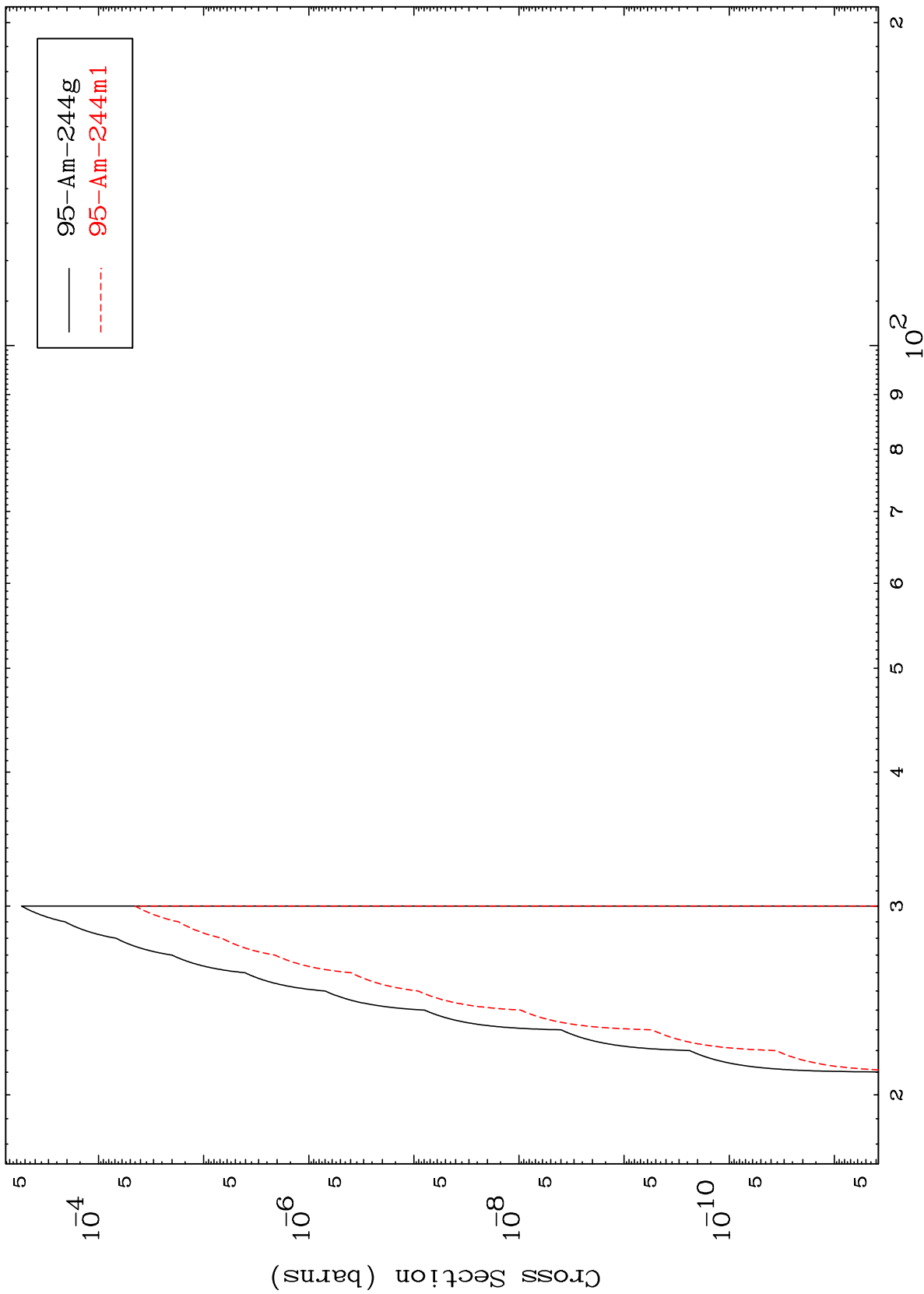
29

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(n,3n) p

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Radionuclide Production Cross Section



30

Incident Energy (MeV)

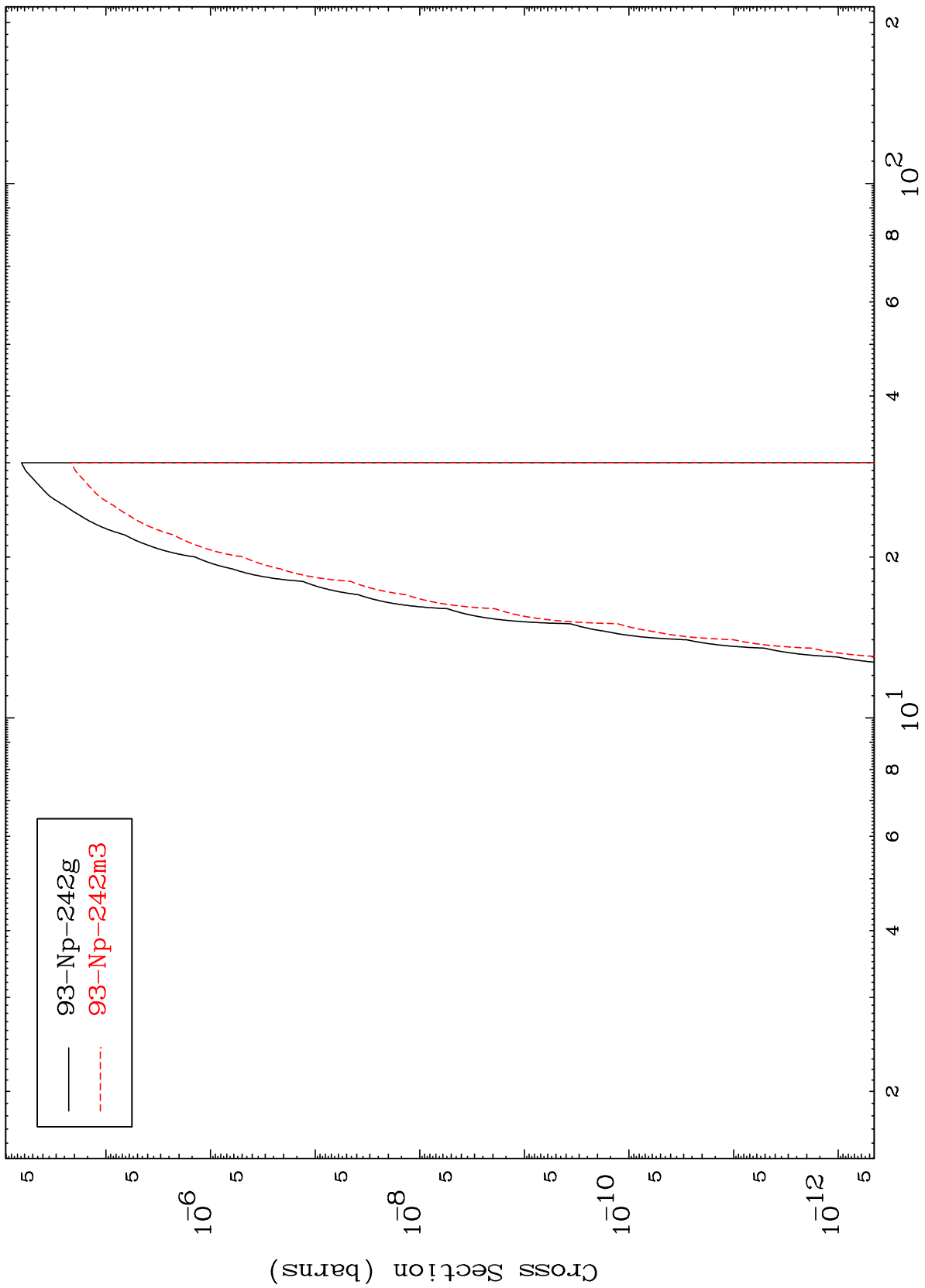
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(n,n') p α

96-Cm-247

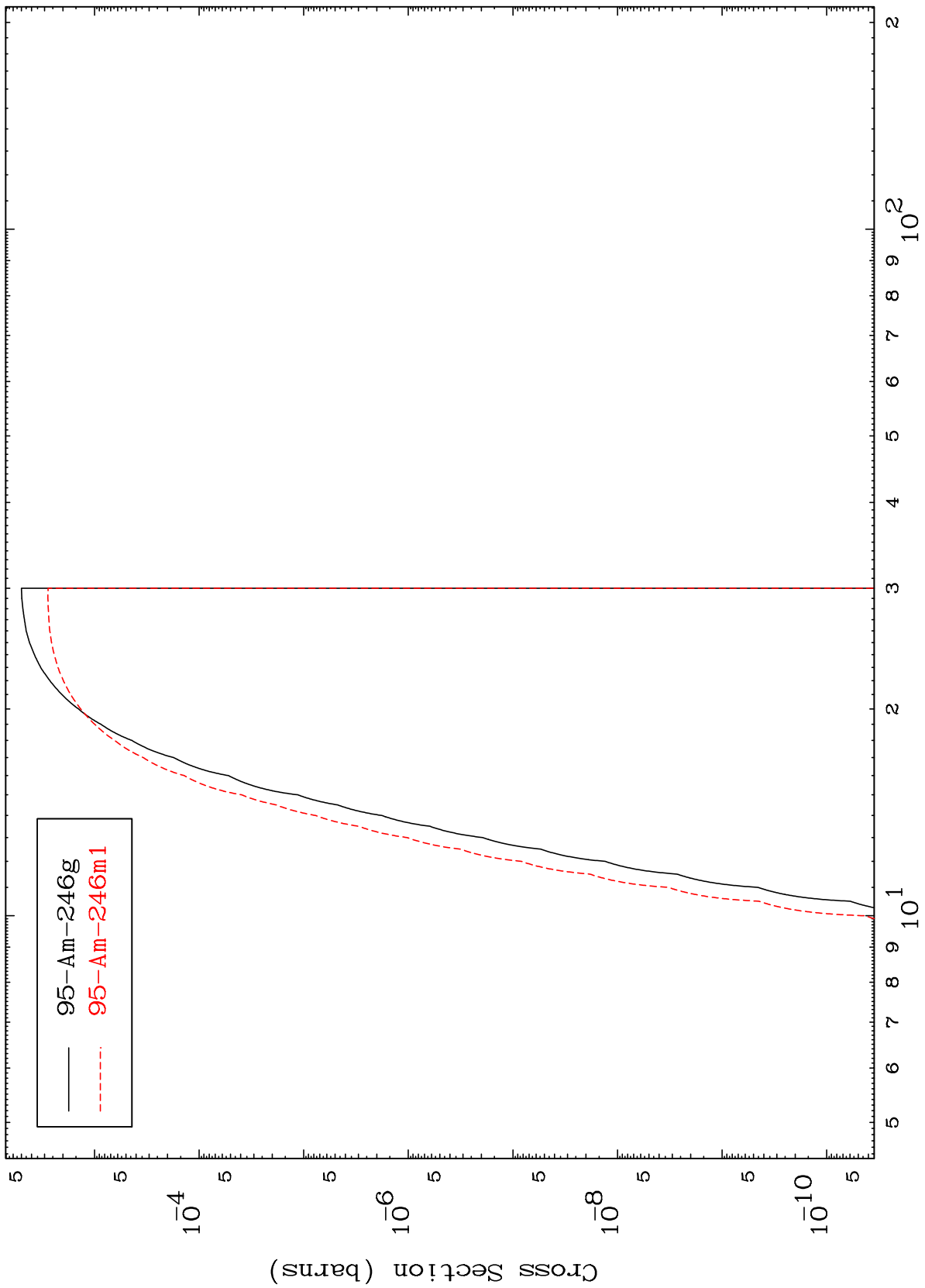
Radionuclide Production Cross Section



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(n,d)
Radionuclide Production Cross Section



Incident Energy (MeV)

96-Cm-247

32

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(n,d) α

96-Cm-247

Radionuclide Production Cross Section

