

Program EVALPLOT
(Version 2015-2)

by

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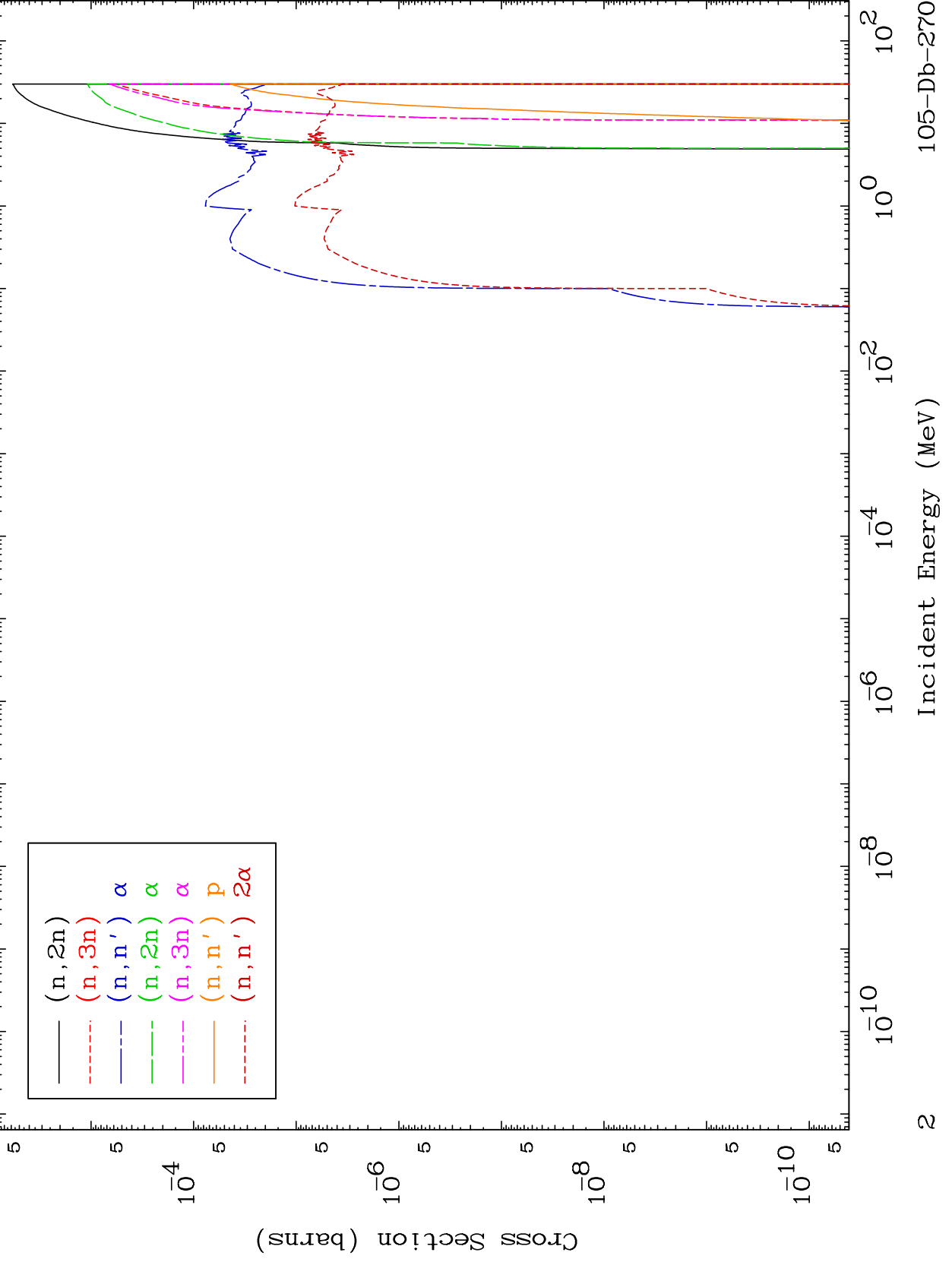
E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

MAT 570

Neutron Production
293 Kelvin Cross Sections

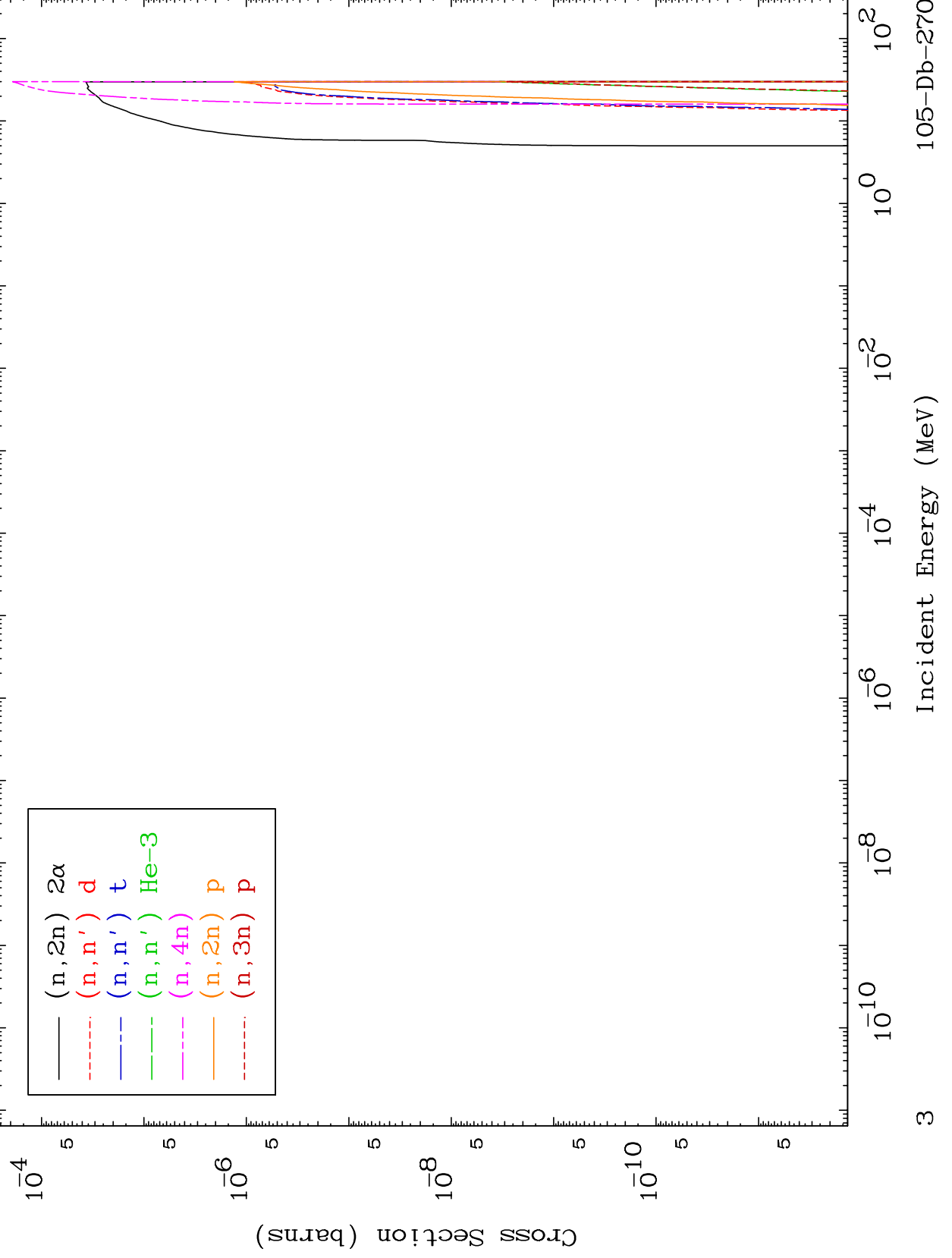
105-Db-270



MAT 570

Neutron Production
293 Kelvin Cross Sections

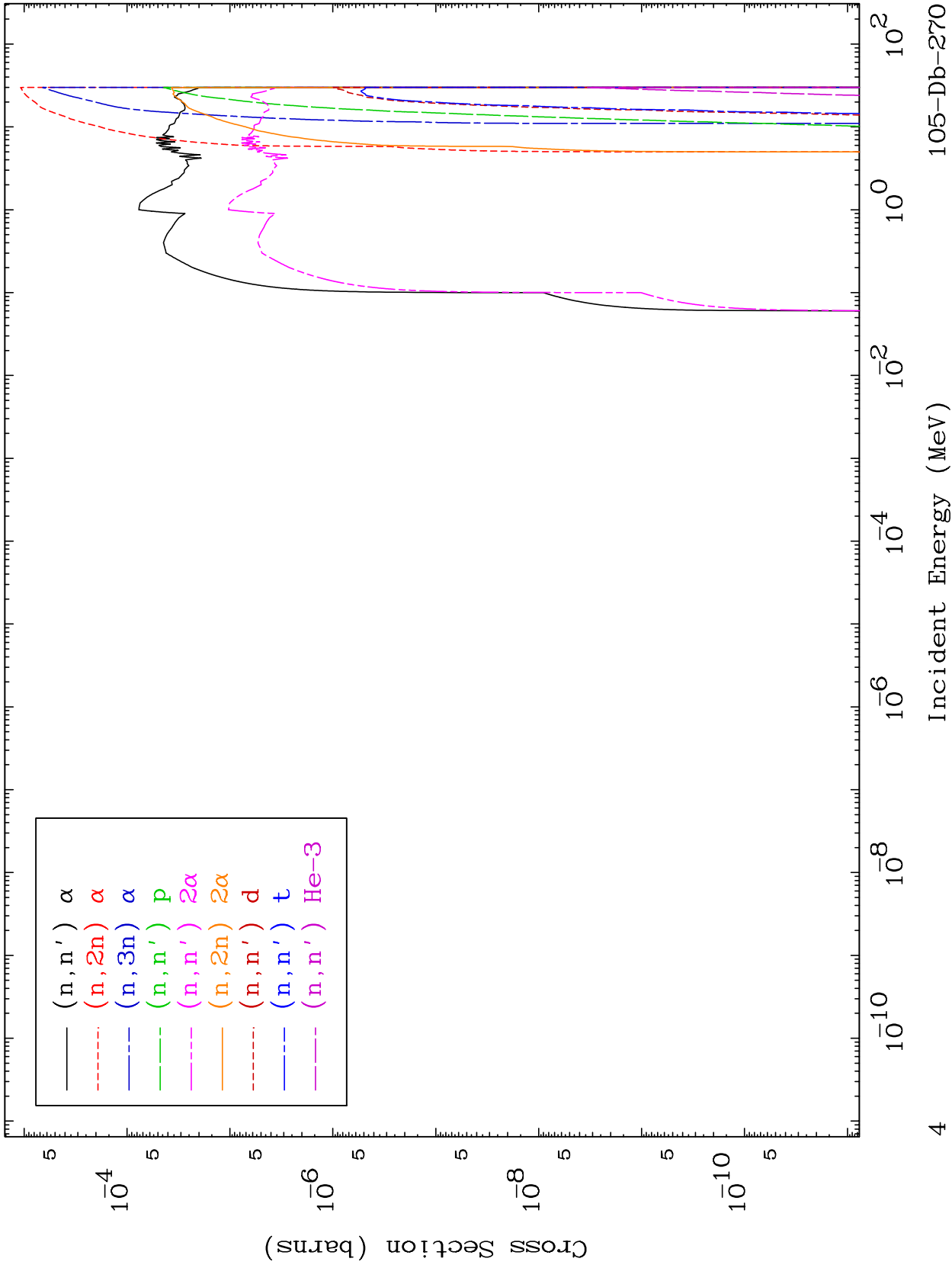
105-Db-270



MAT 570

Charged Particle
293 Kelvin Cross Sections

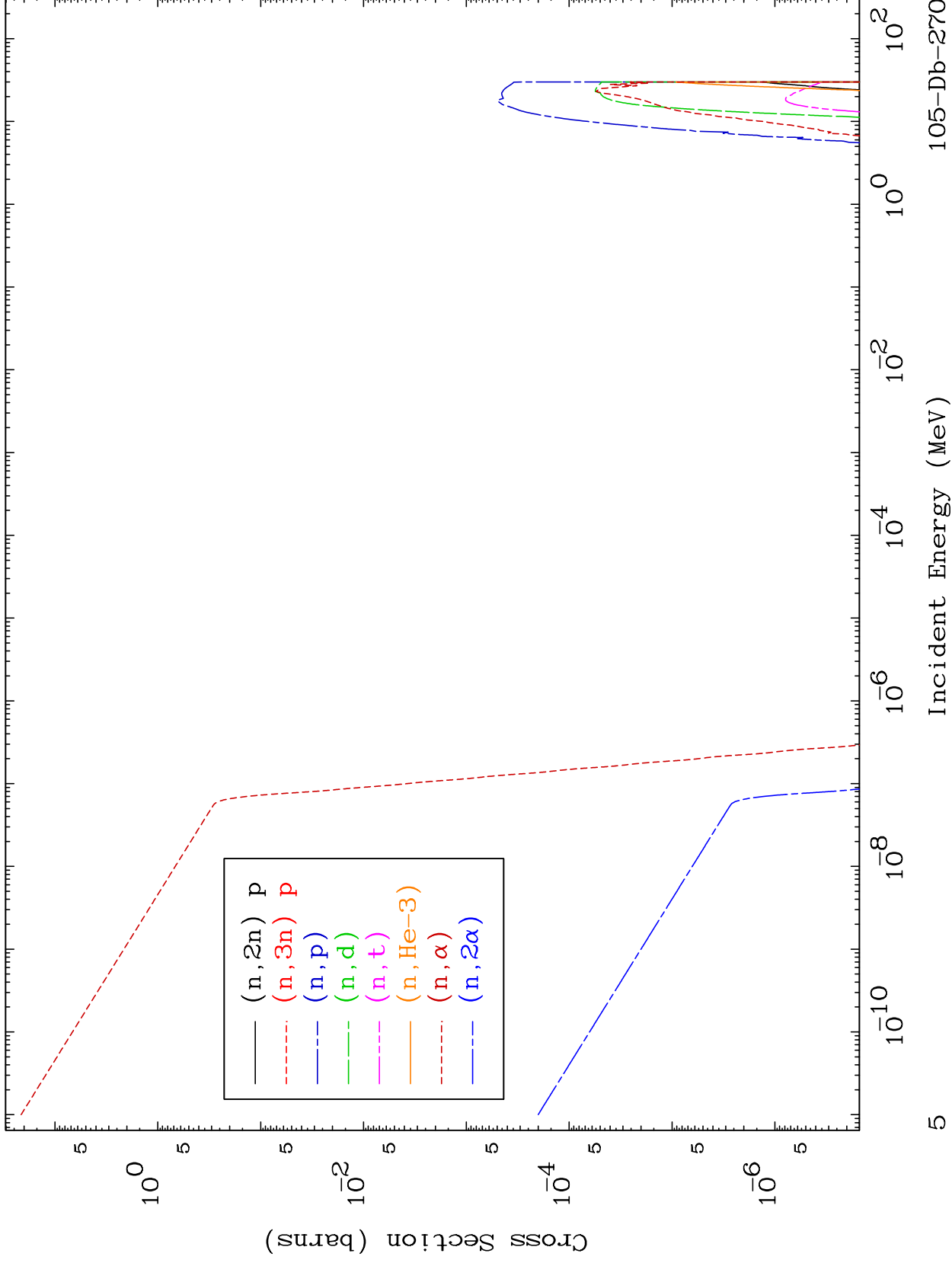
105-Db-270



MAT 570

Charged Particle
293 Kelvin Cross Sections

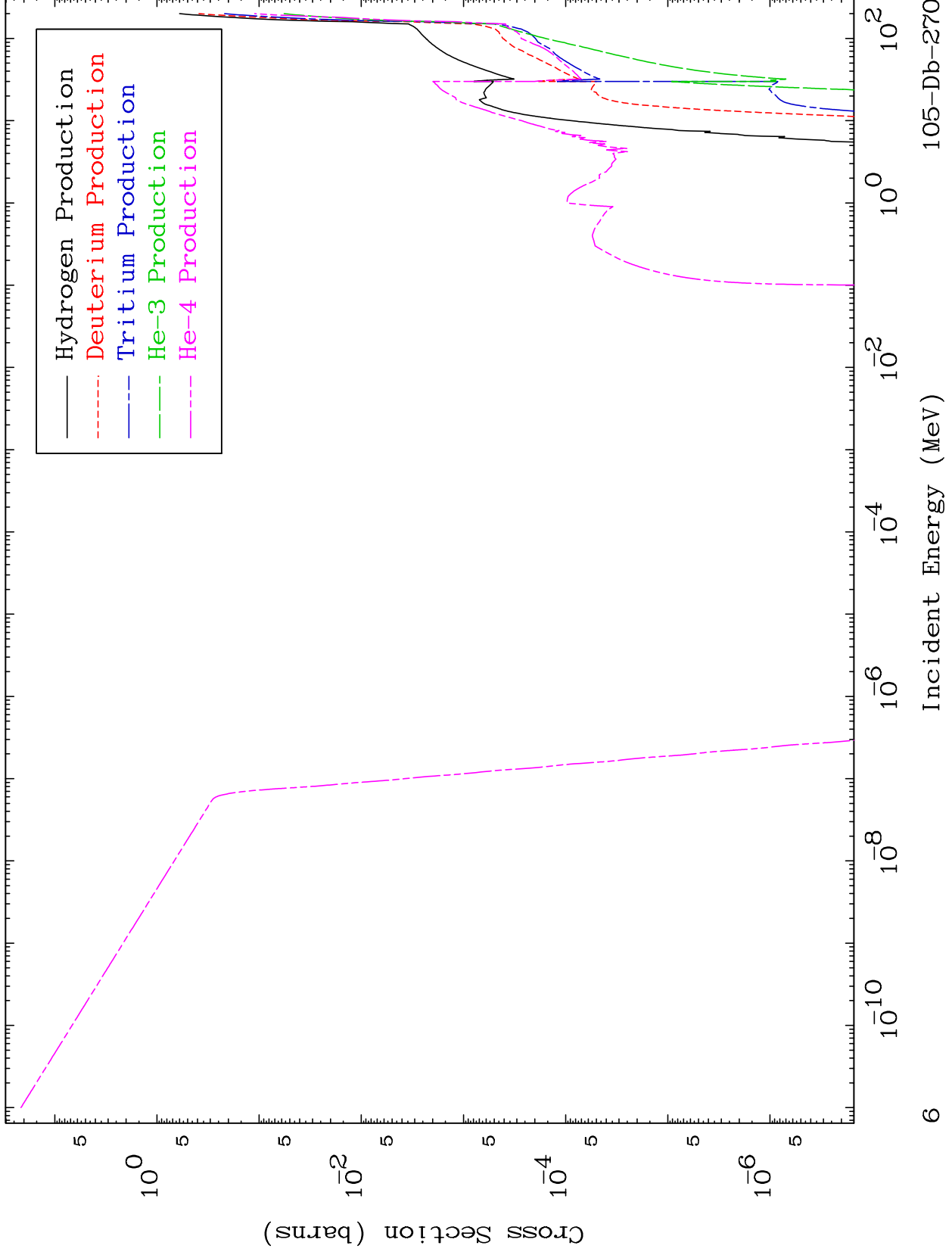
105-Db-270



MAT 570

Particle Production
293 Kelvin Cross Sections

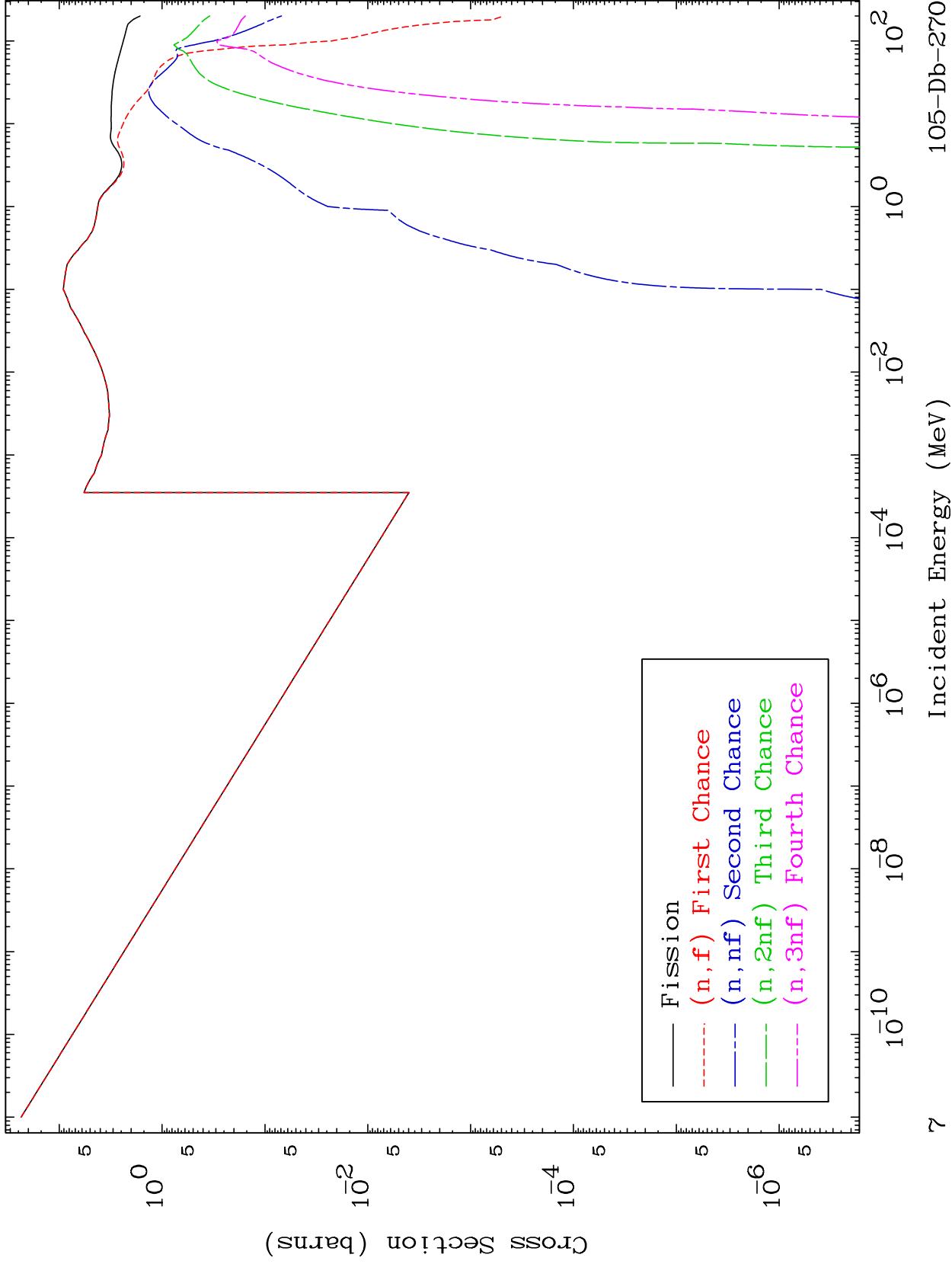
105-Db-270



MAT 570

Fission
293 Kelvin Cross Sections

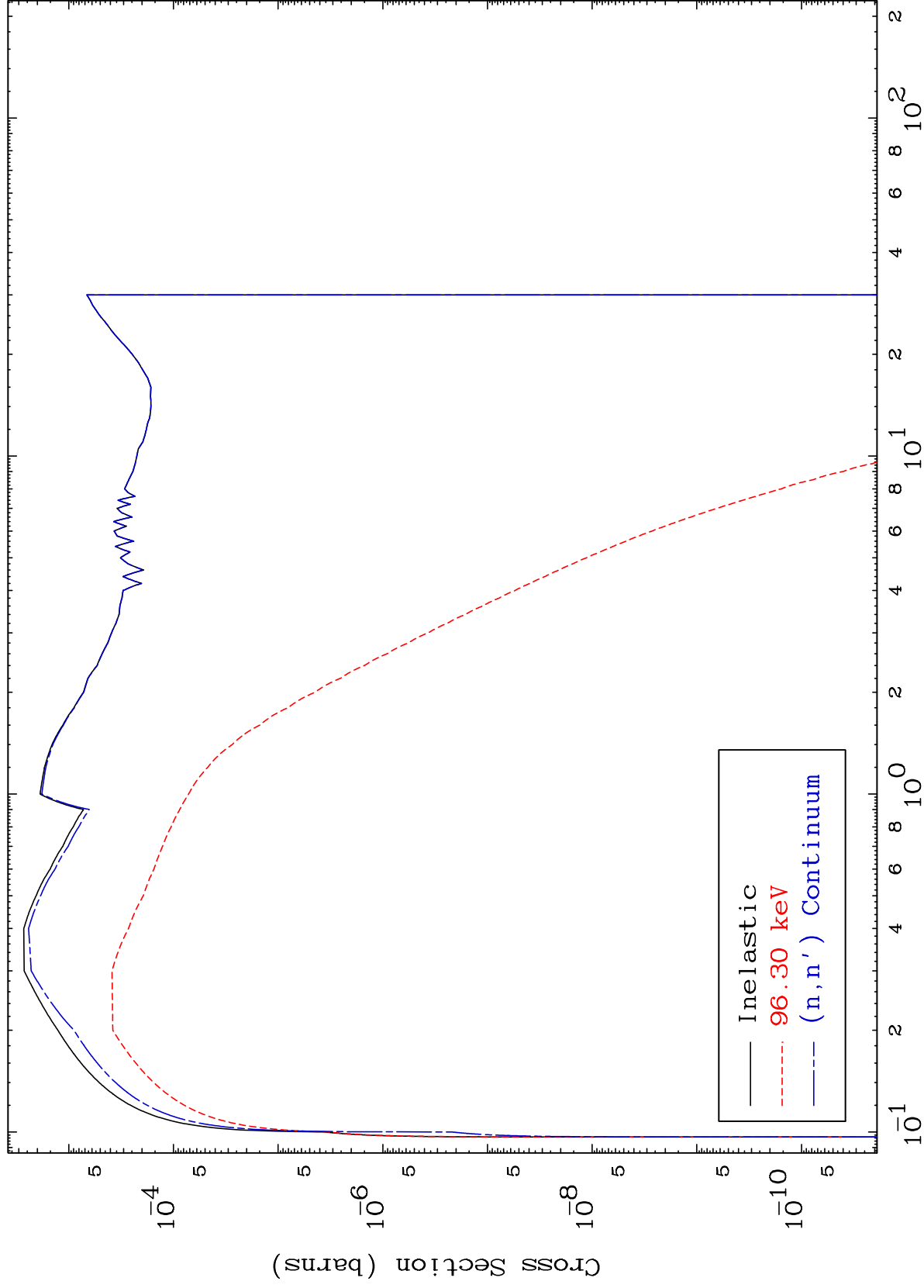
105-Db-270



MAT 570

105-Db-270

(n,n') Level
293 Kelvin Cross Sections



Incident Energy (MeV)

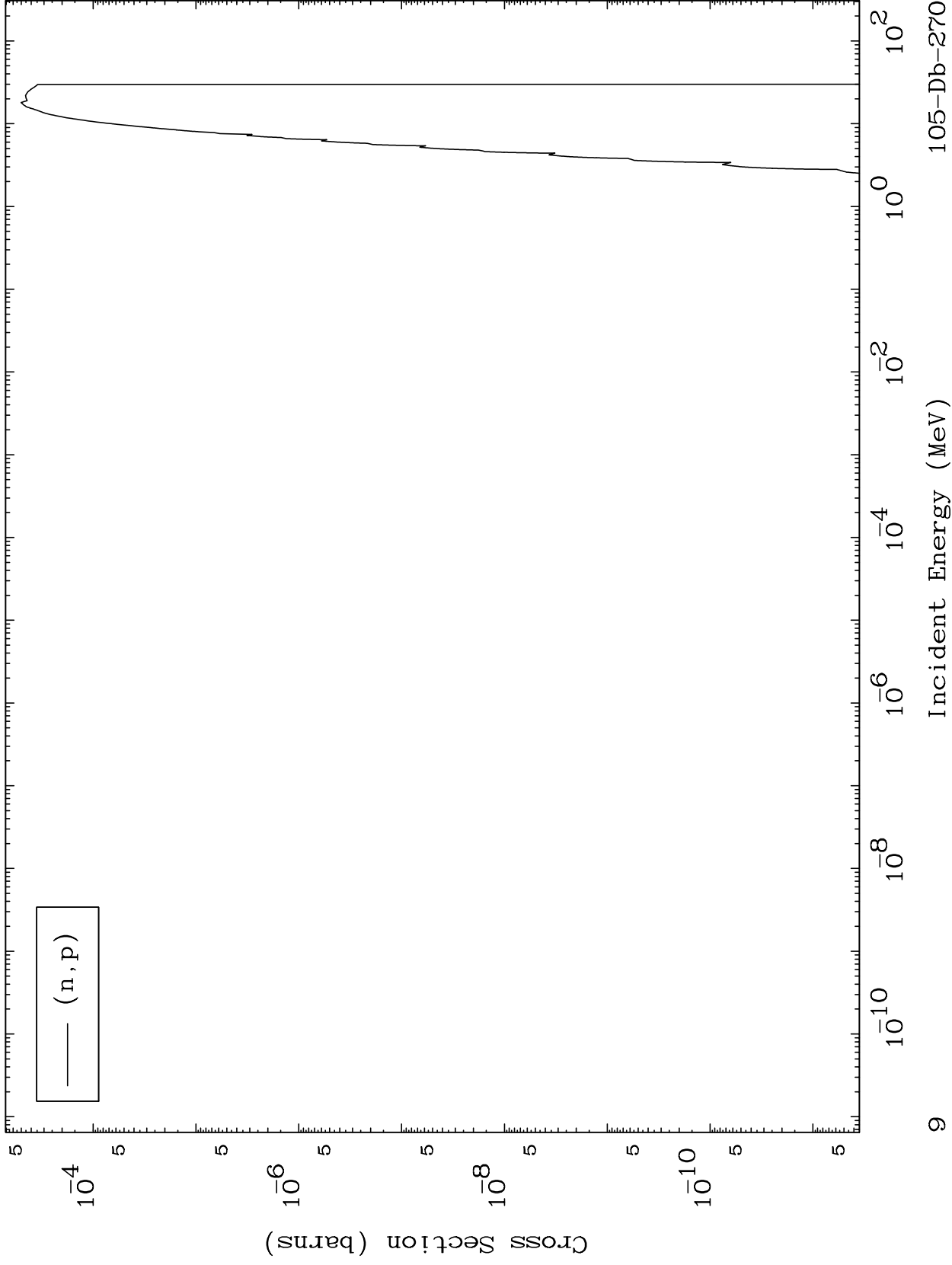
105-Db-270

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MAT 570

(n,p) Levels
293 Kelvin Cross Sections

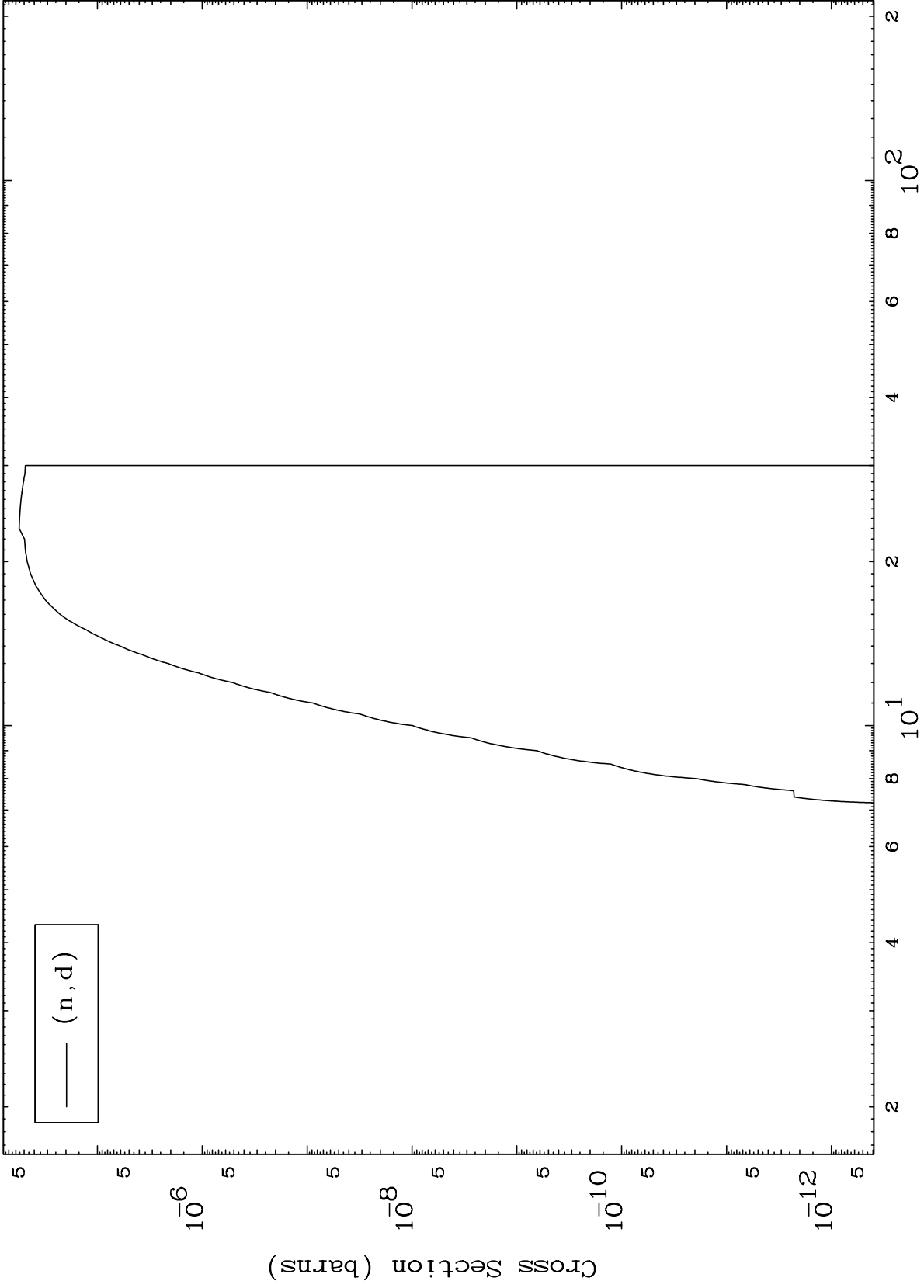
105-Db-270



MAT 570

(n,d) Levels
293 Kelvin Cross Sections

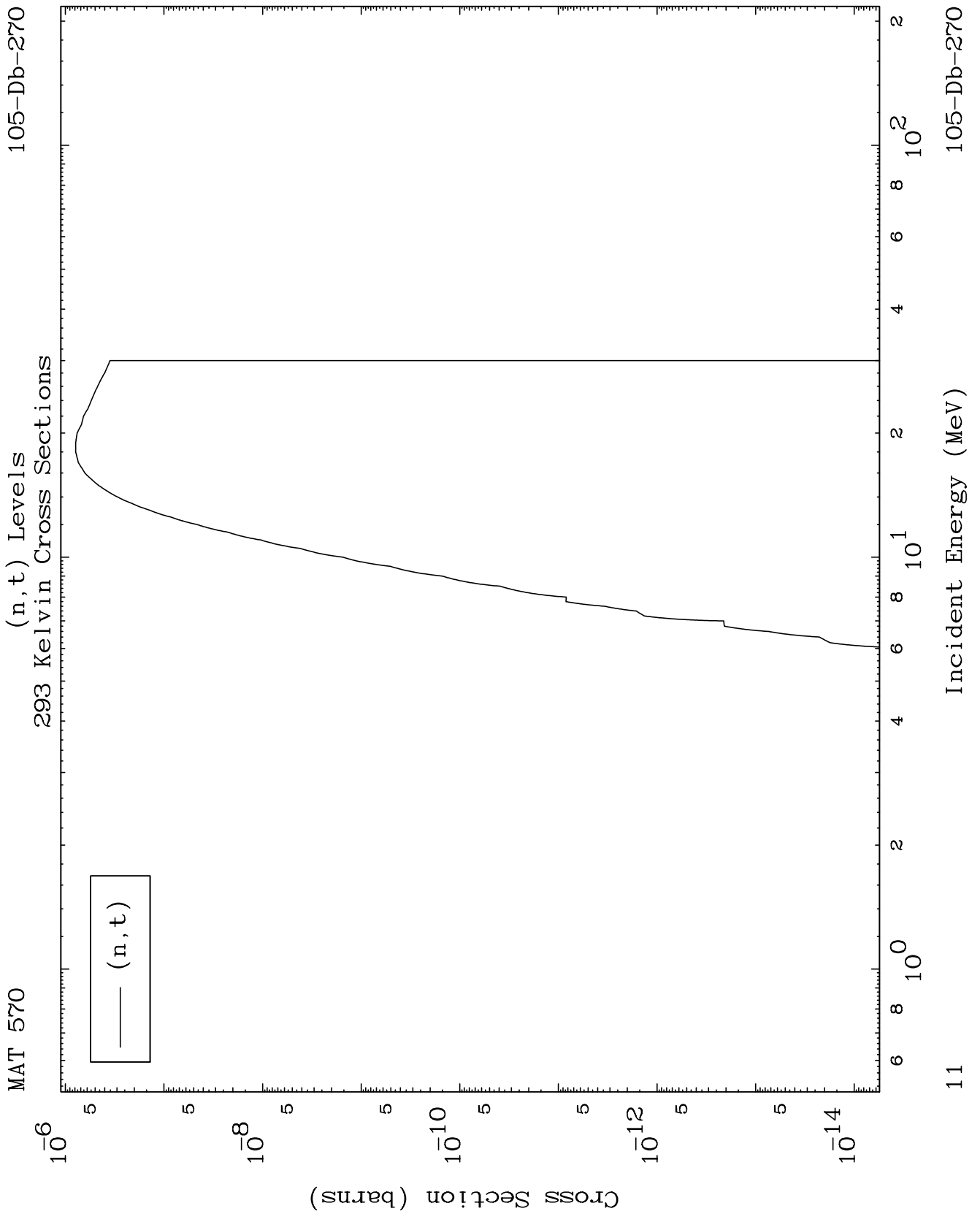
105-Db-270



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Incident Energy (MeV)

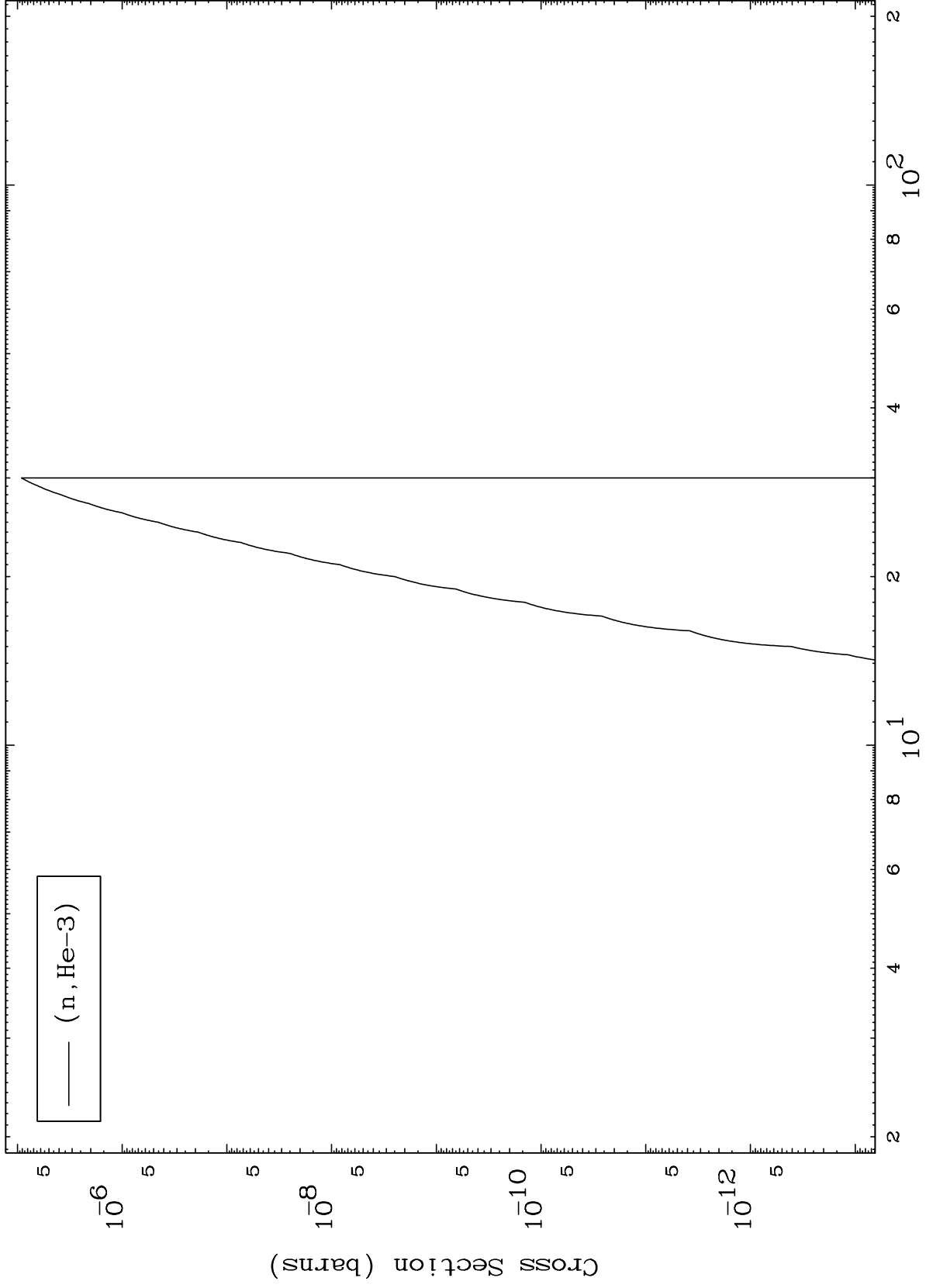
105-Db-270



MAT 570

(n,He3) Levels
293 Kelvin Cross Sections

105-Db-270



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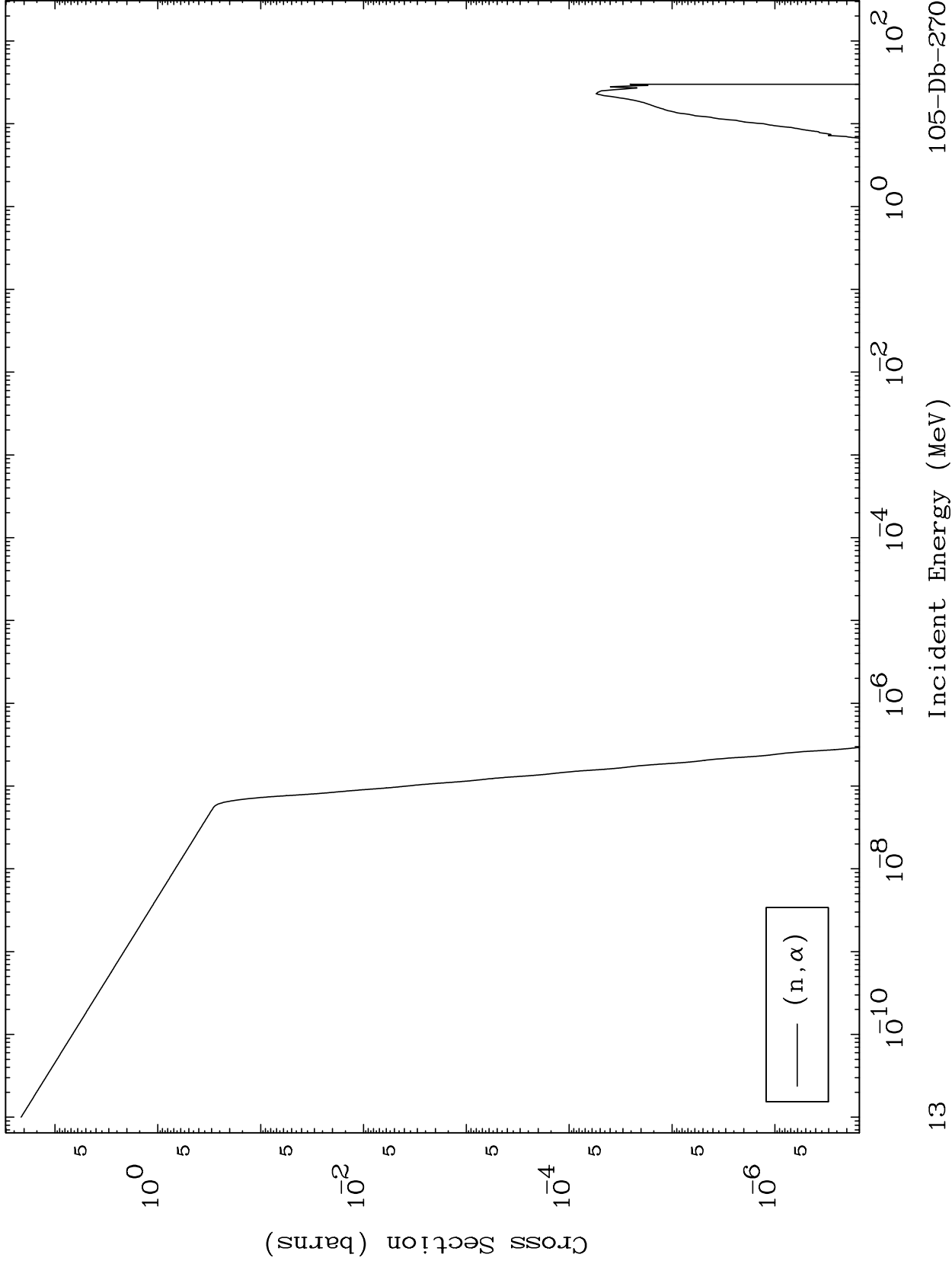
Incident Energy (MeV)

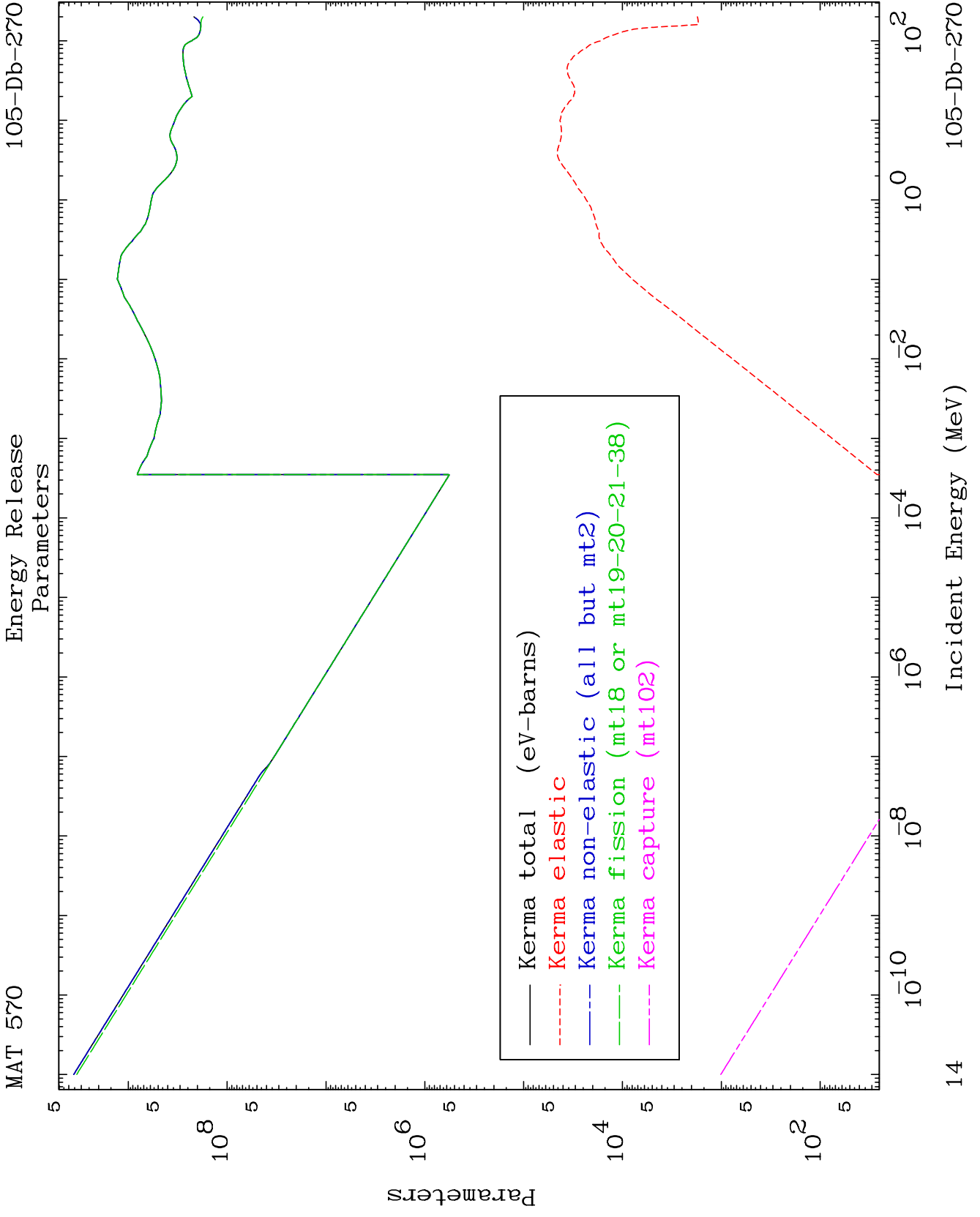
105-Db-270

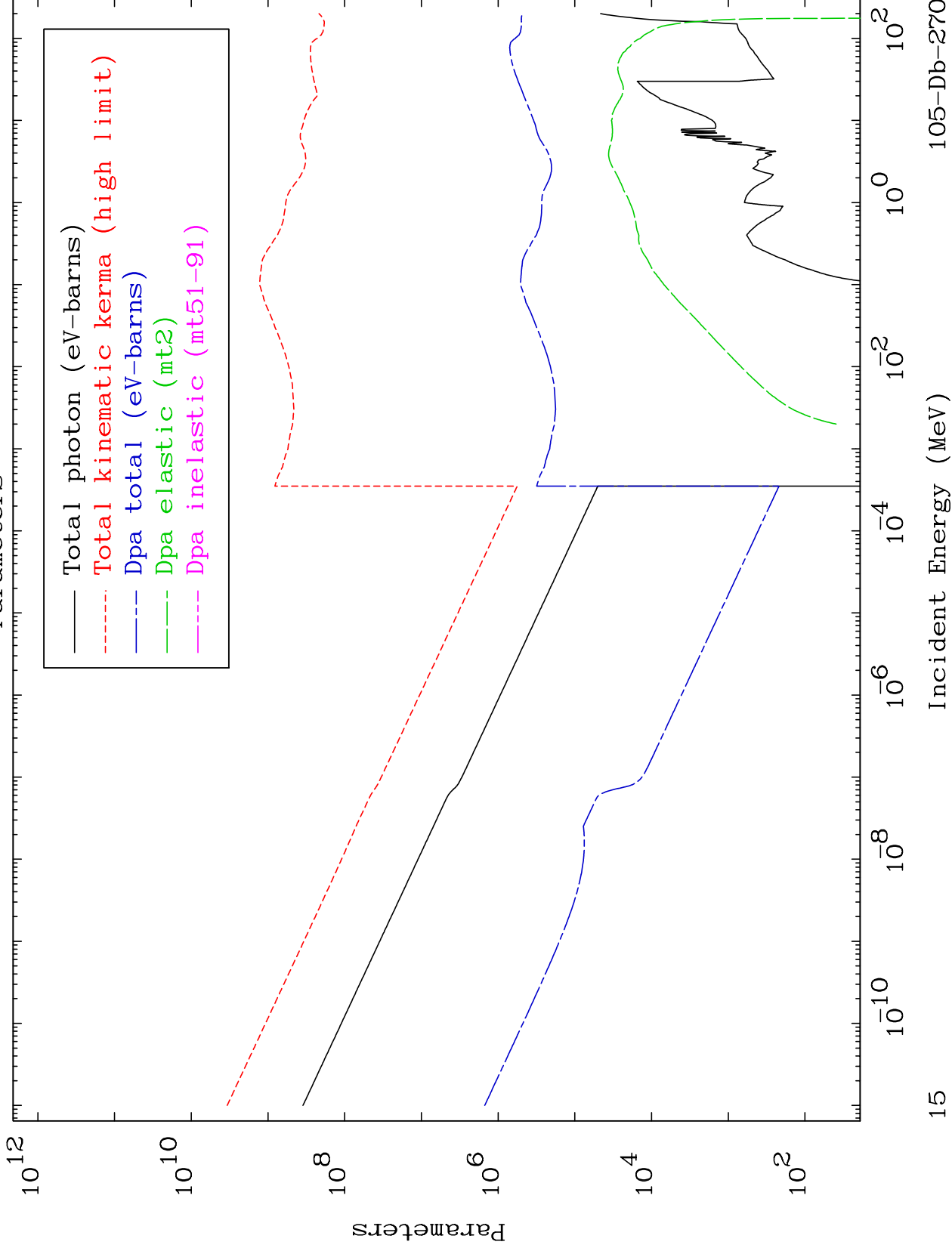
MAT 570

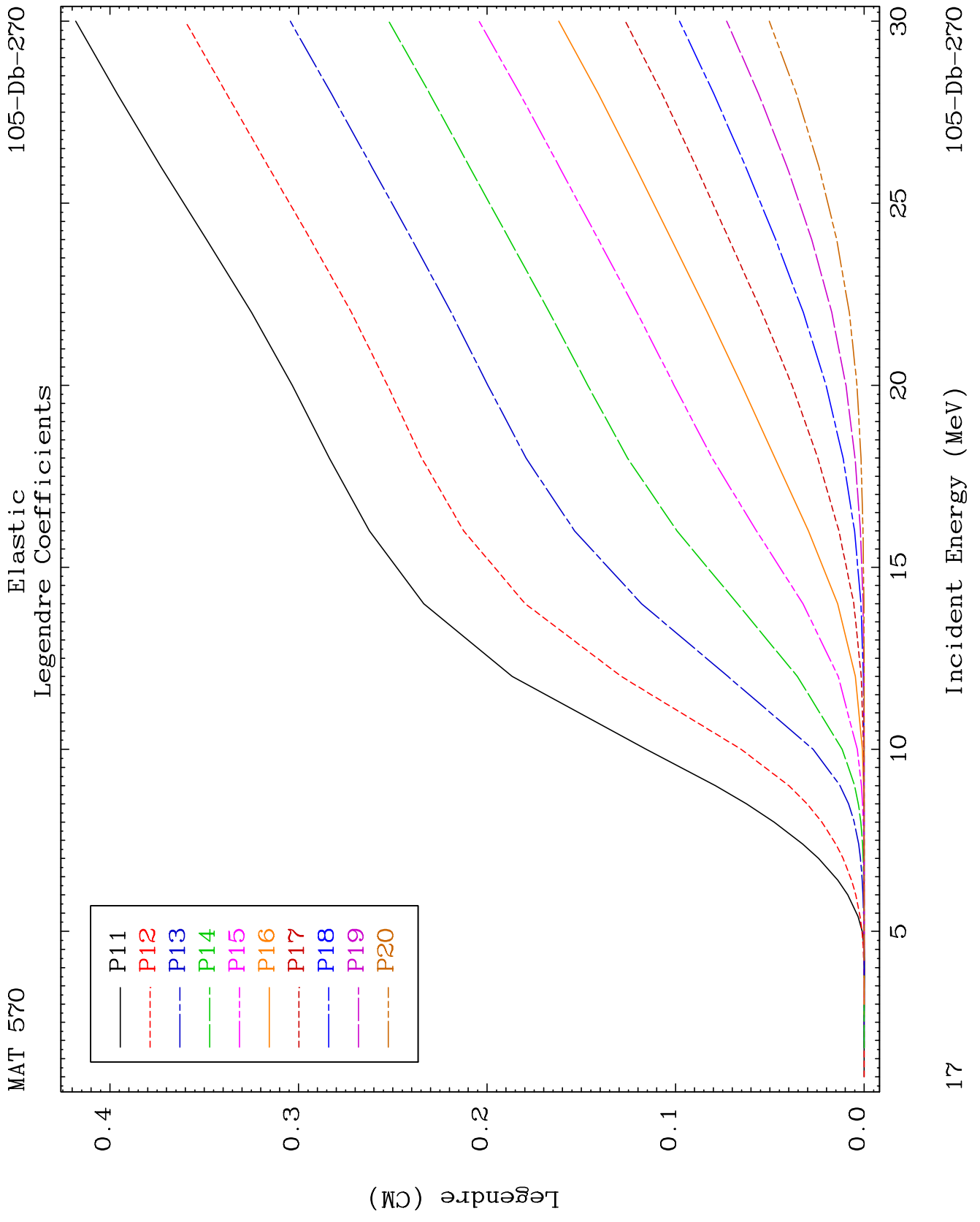
(n, α) Levels
293 Kelvin Cross Sections

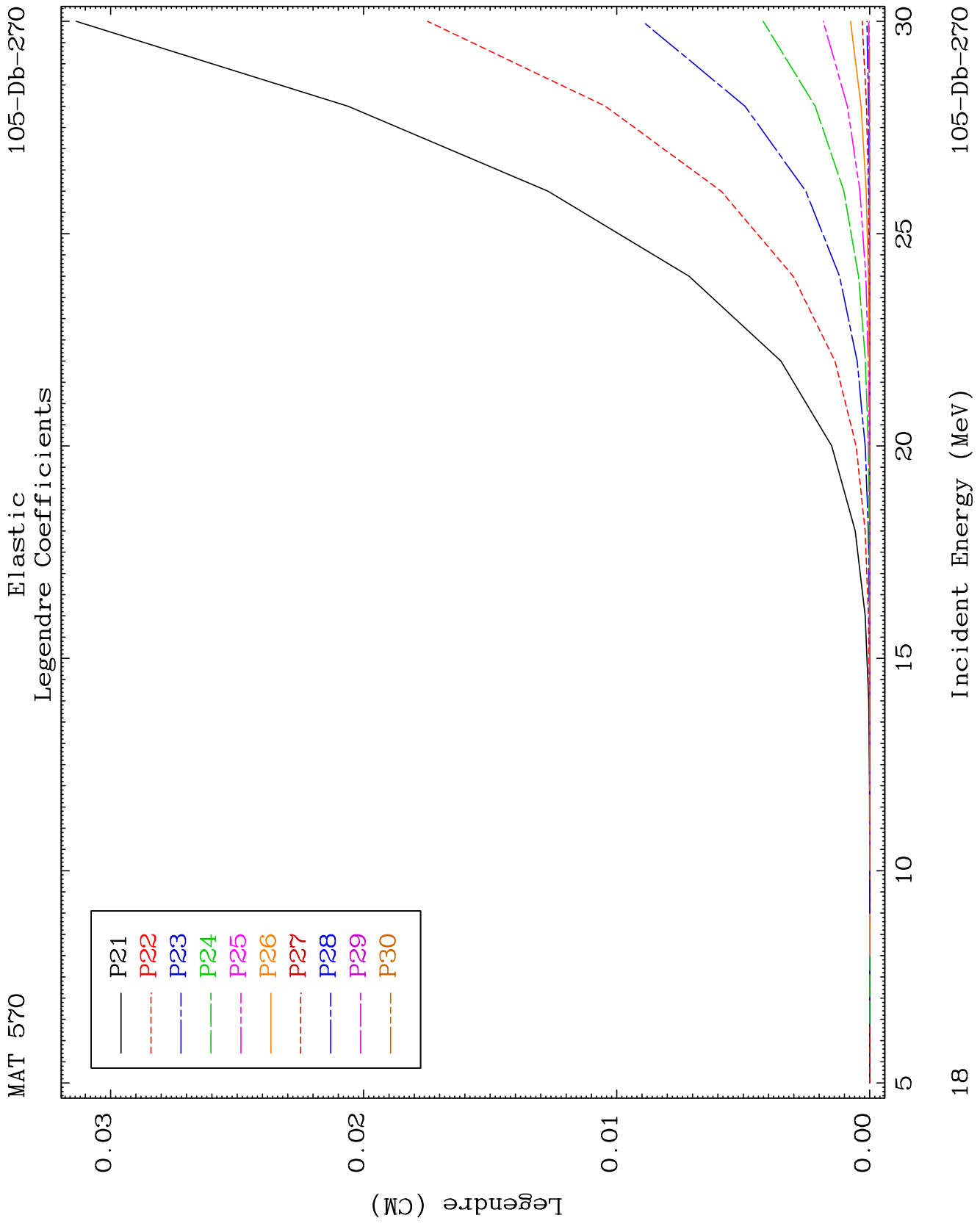
105-Db-270











MAT 570

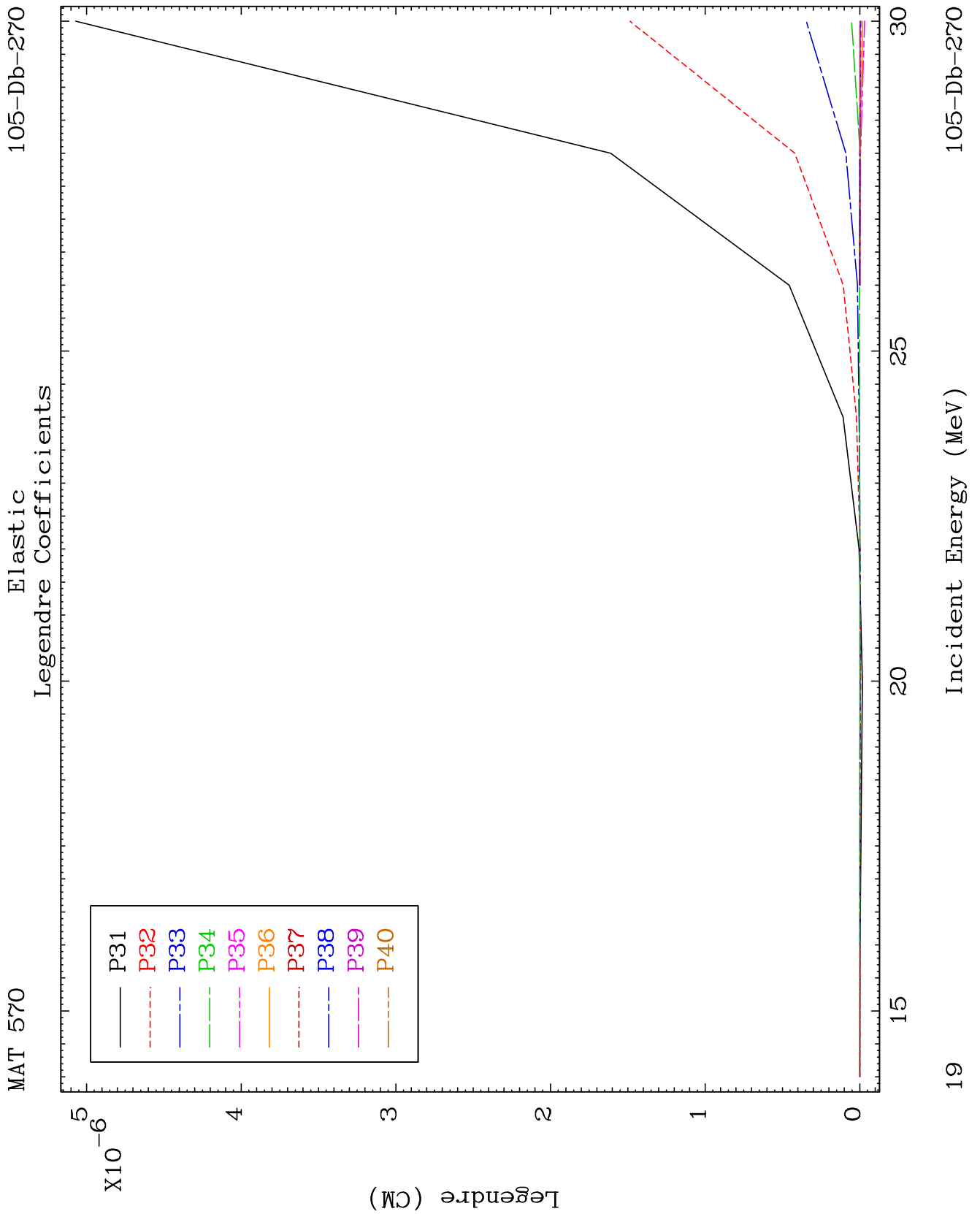
Elastic Legendre Coefficients

105-Db-270

18

Incident Energy (MeV)

105-Db-270



MAT 570

Elastic Legendre Coefficients

105-Db-270

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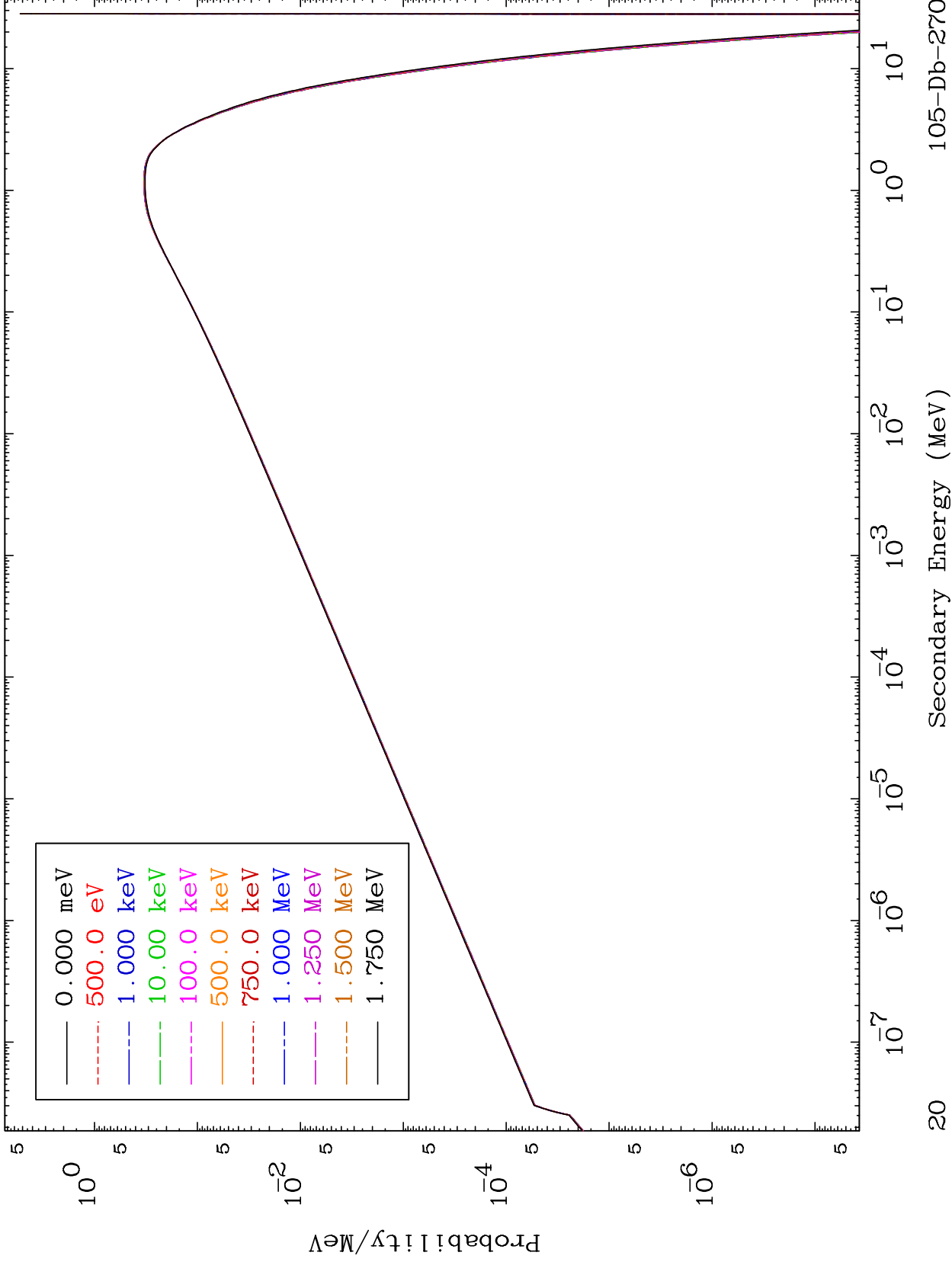
Incident Energy (MeV)

105-Db-270

MAT 570

Fission Energy Distribution

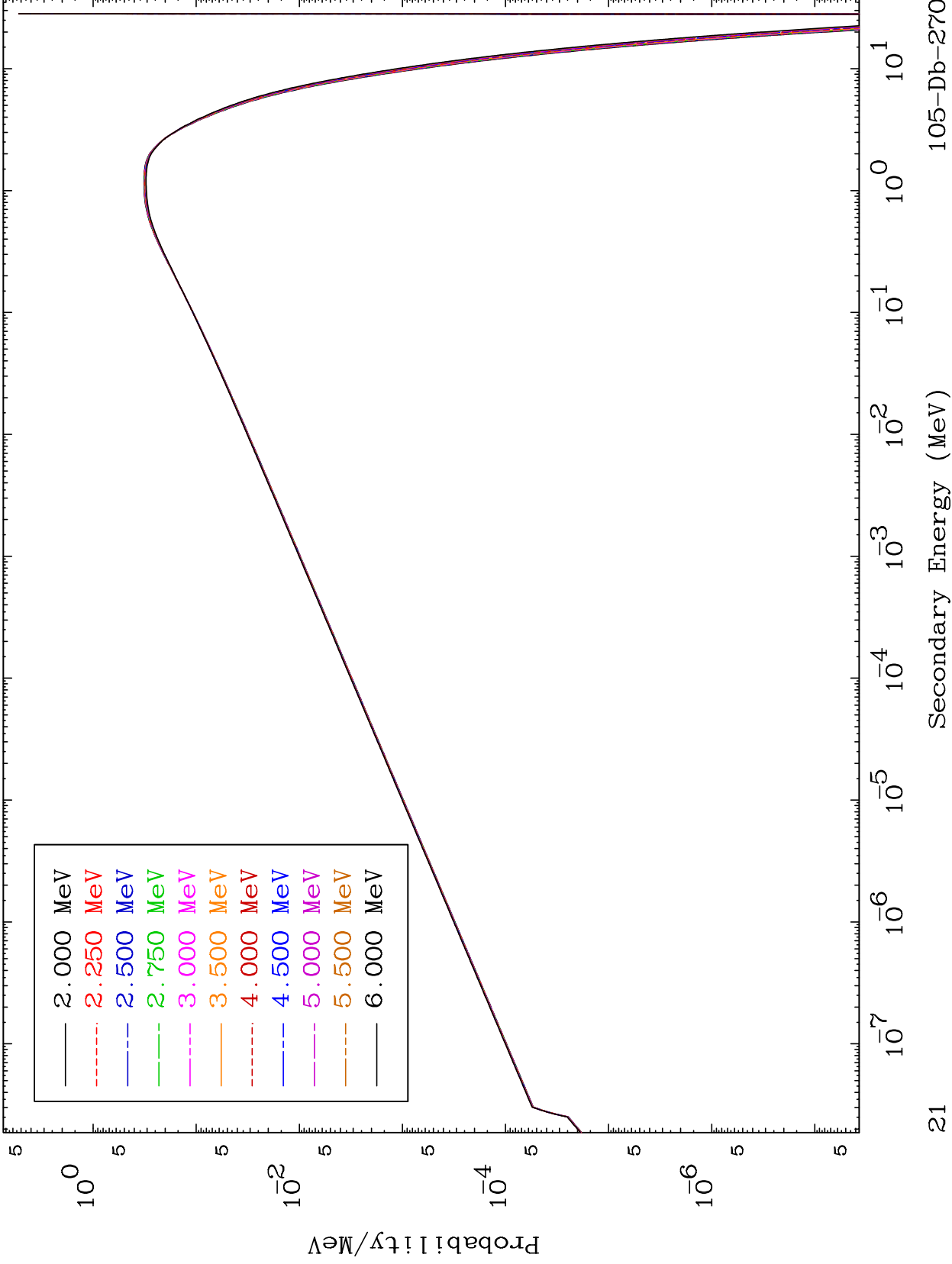
105-Db-270



MAT 570

Fission Energy Distribution

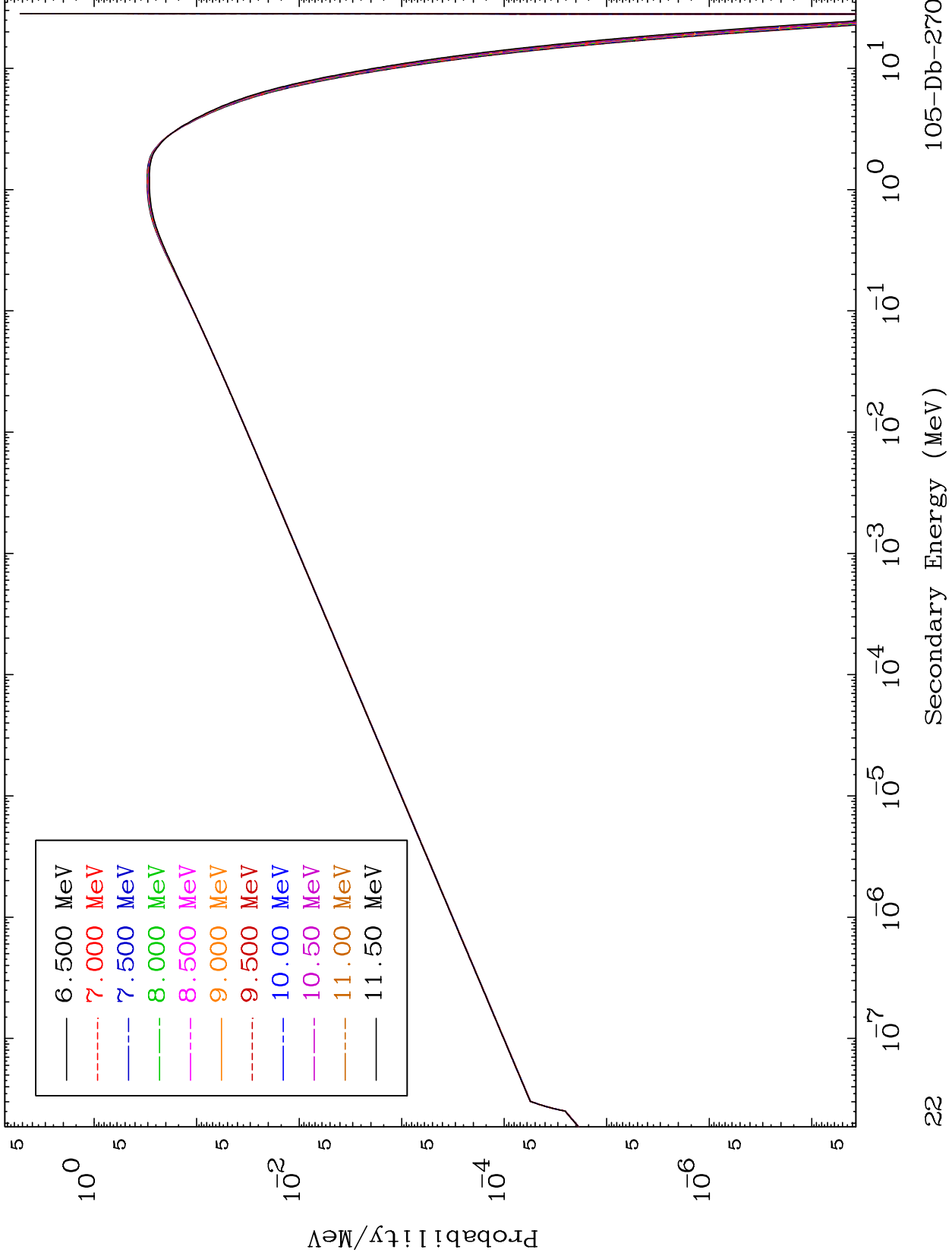
105-Db-270



MAT 570

Fission Energy Distribution

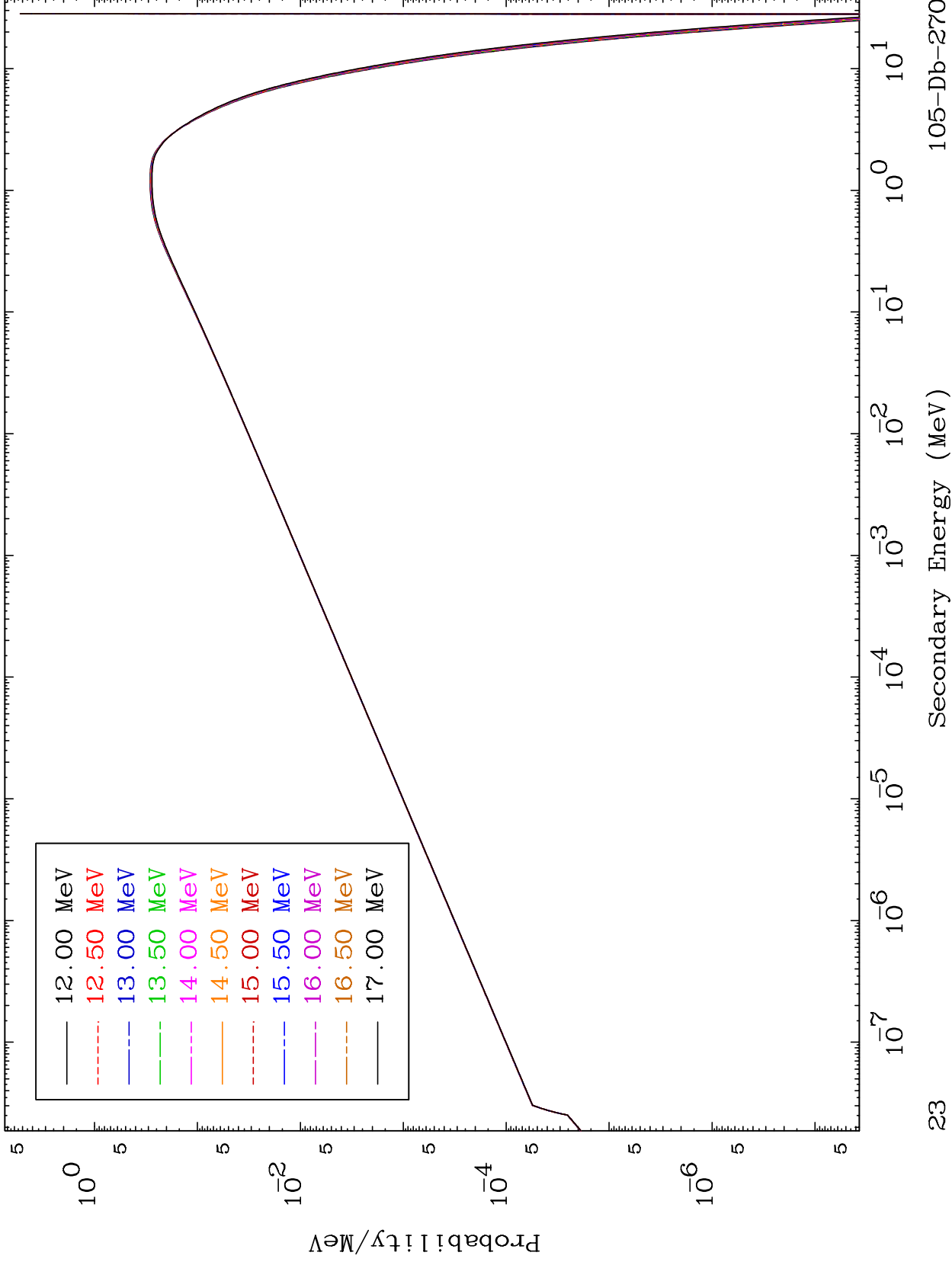
105-Db-270



MAT 570

Fission Energy Distribution

105-Db-270



MAT 570

Fission Energy Distribution

105-Db-270

