

Program EVALPLOT
(Version 2015-2)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

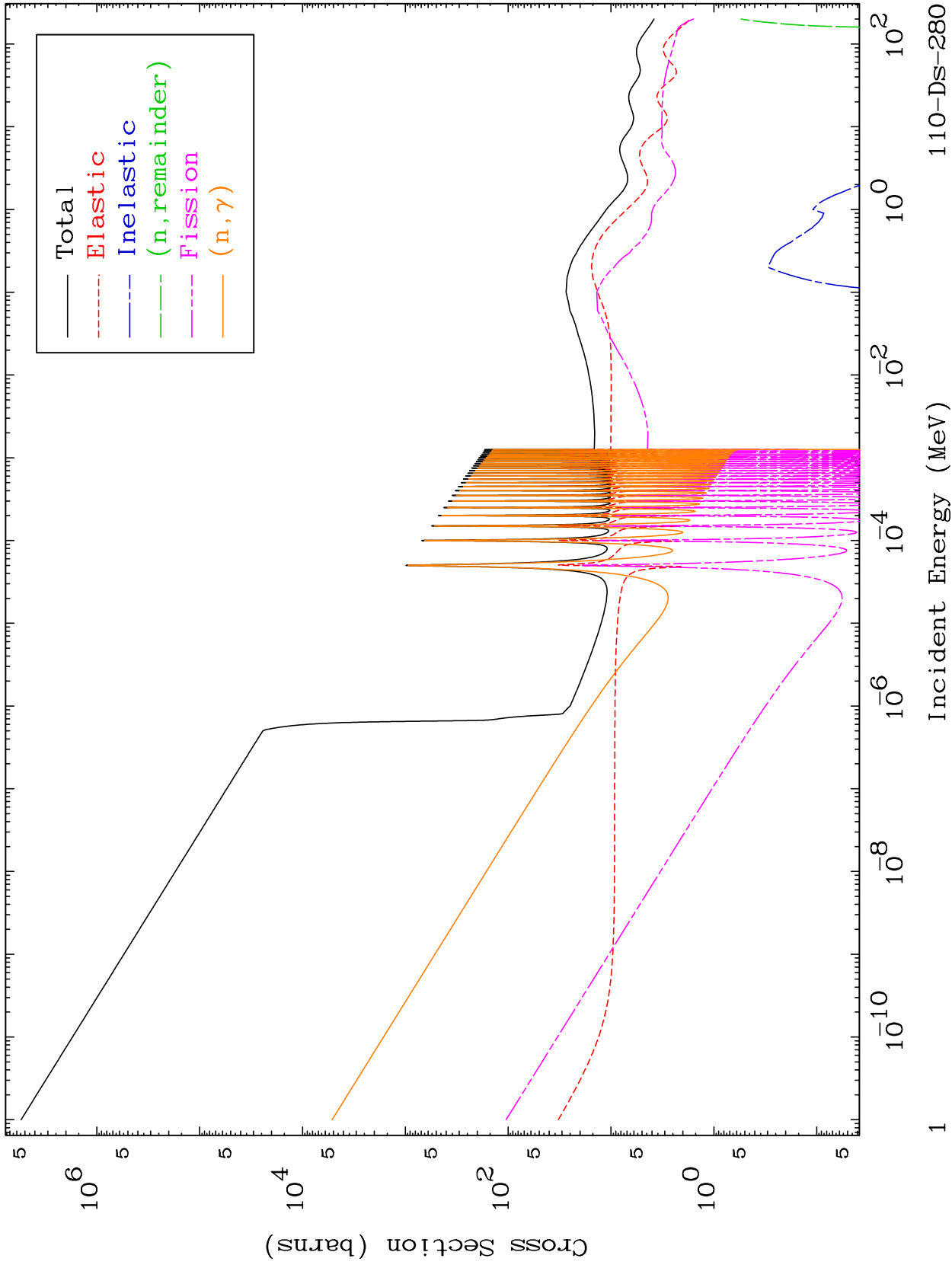
E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

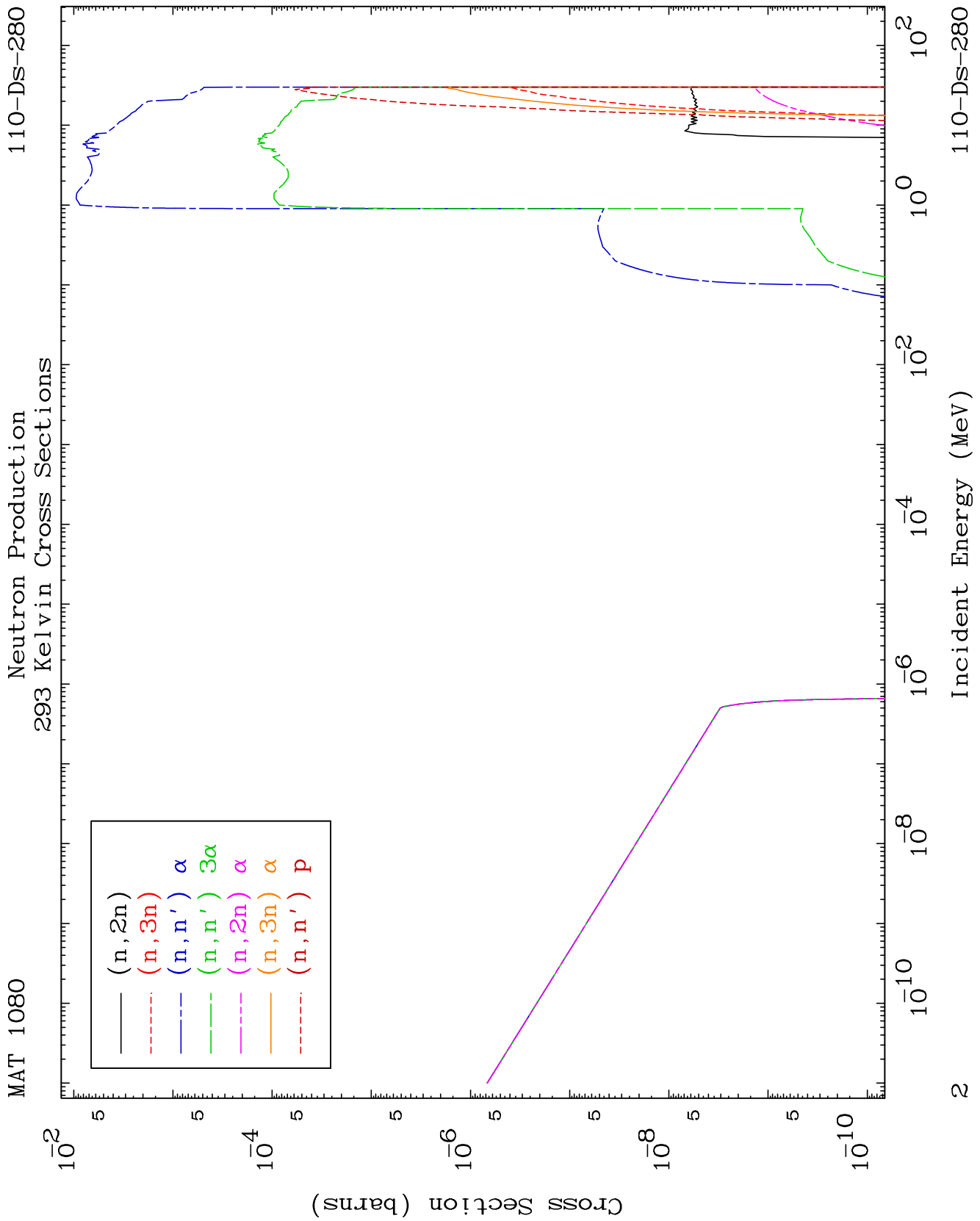
Press Mouse Button to Start

MAT 1080

Major
293 Kelvin Cross Sections

110-Ds-280

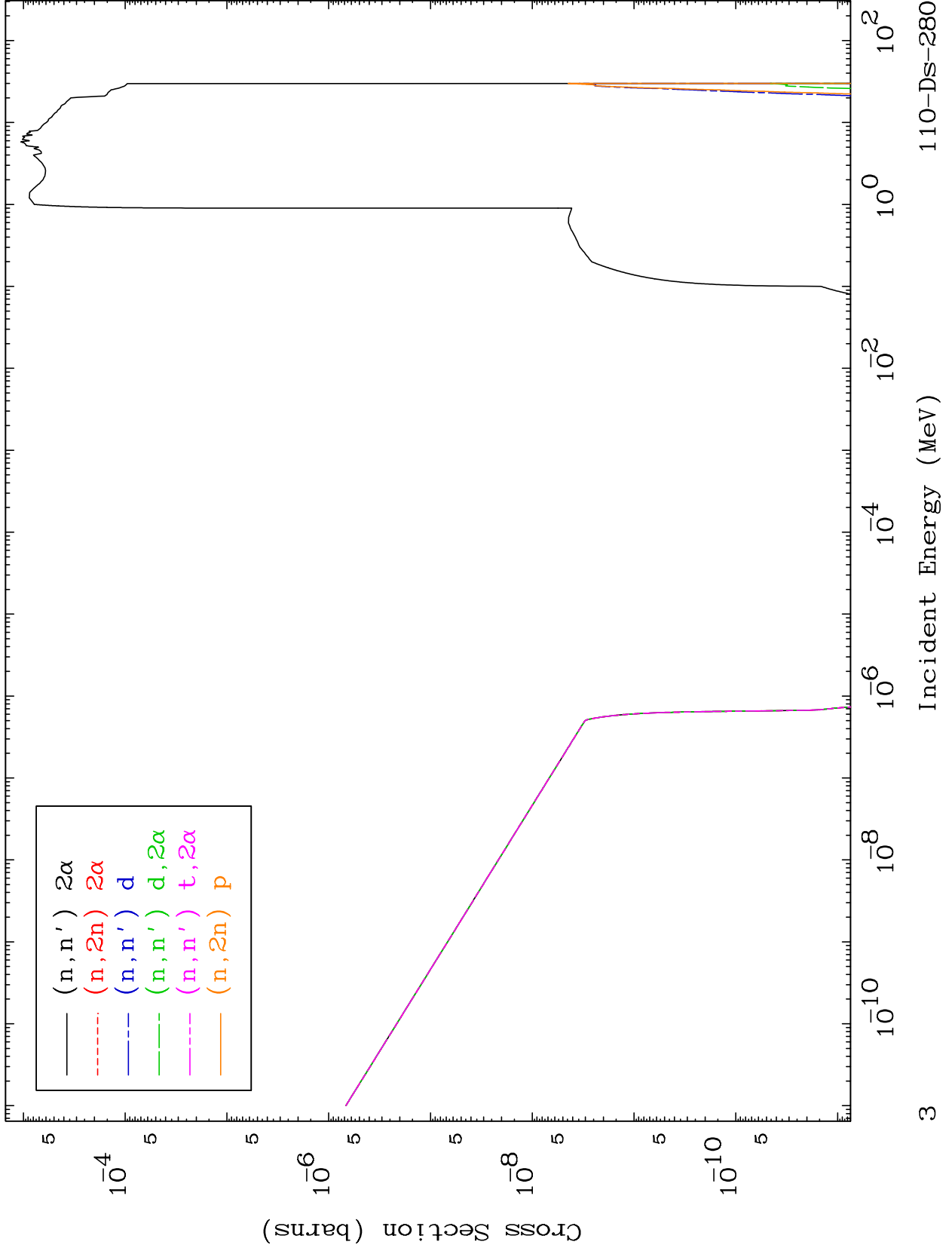


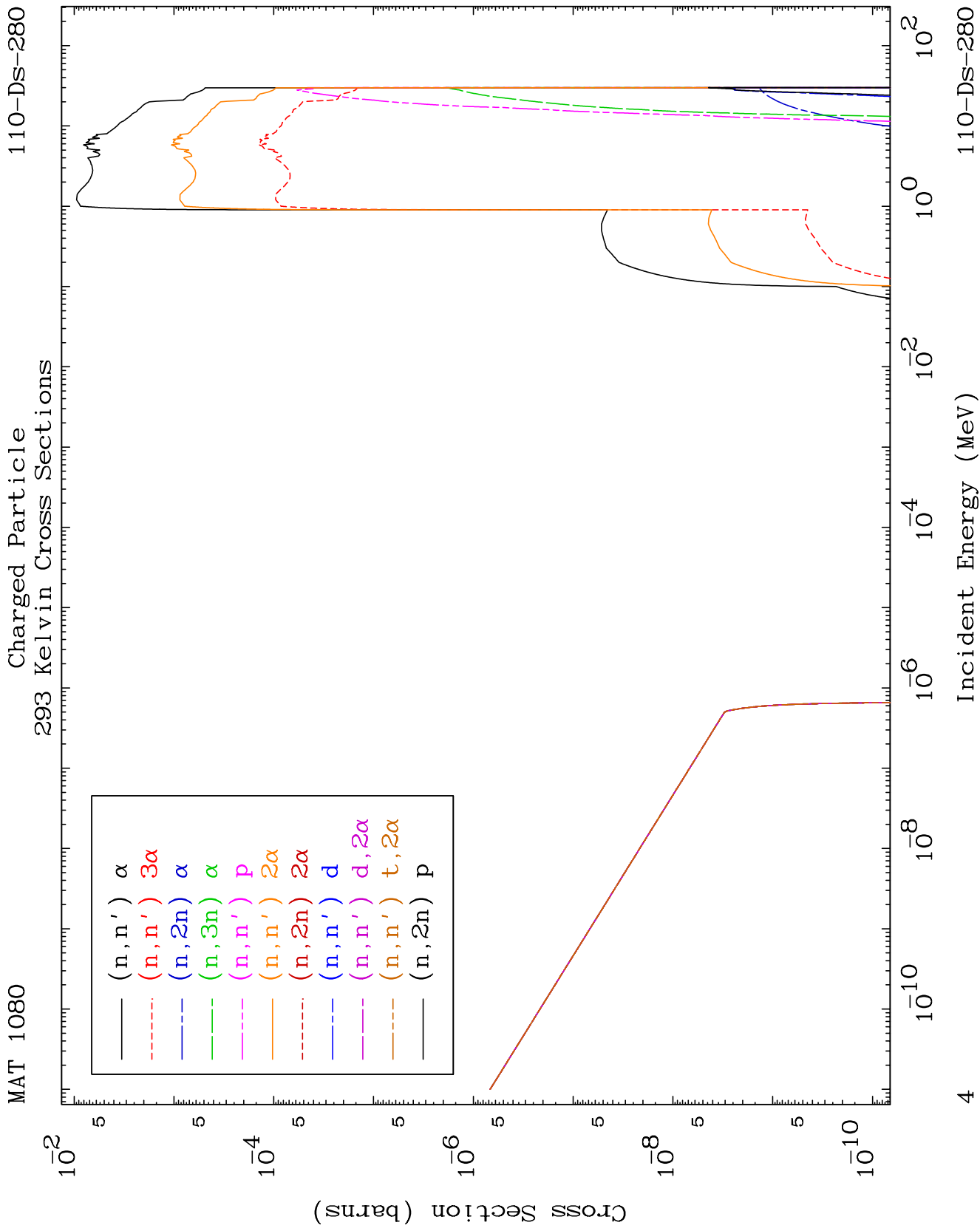


MAT 1080

Neutron Production
293 Kelvin Cross Sections

110-Ds-280

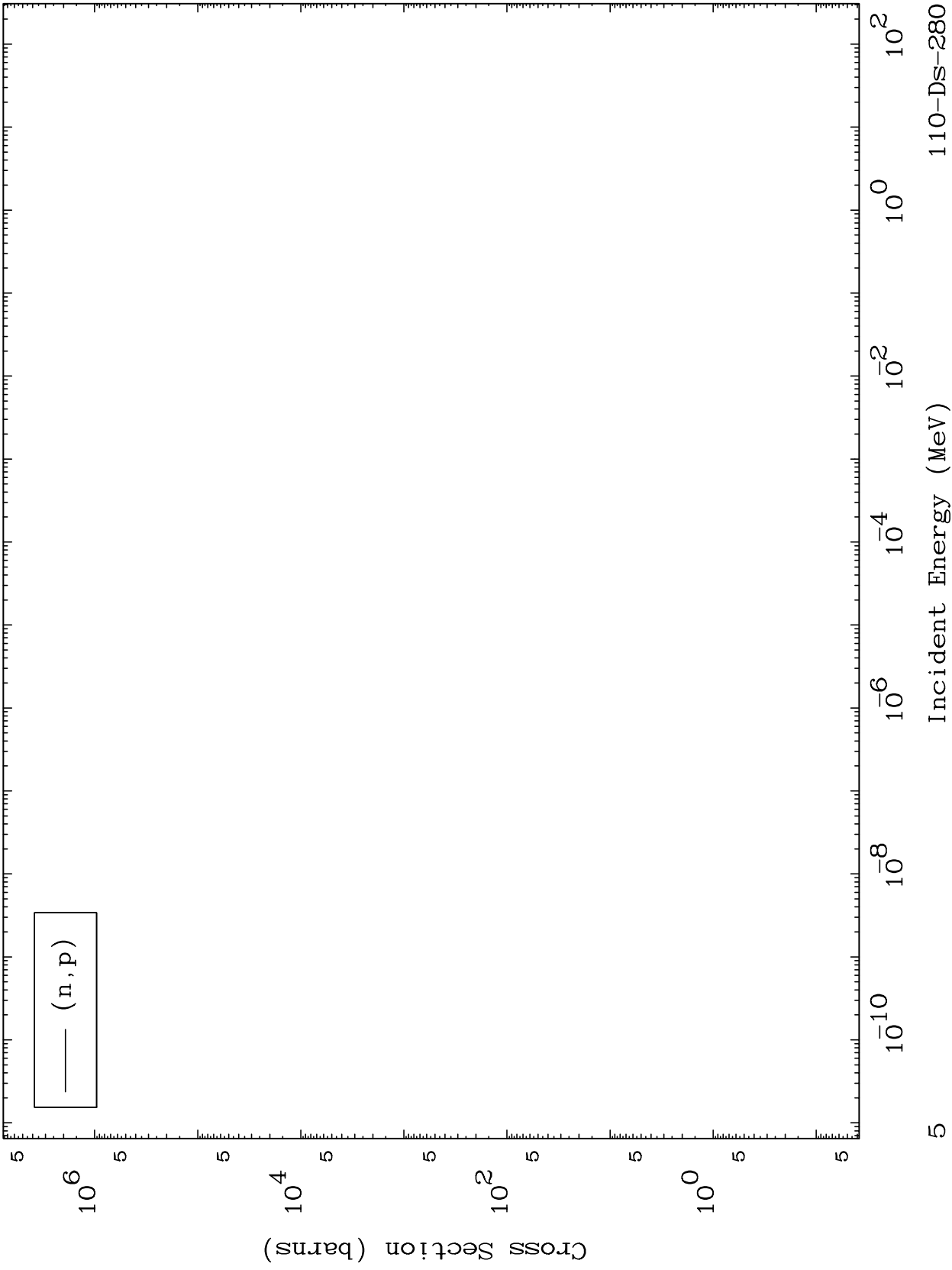




MAT 1080

Charged Particle
293 Kelvin Cross Sections

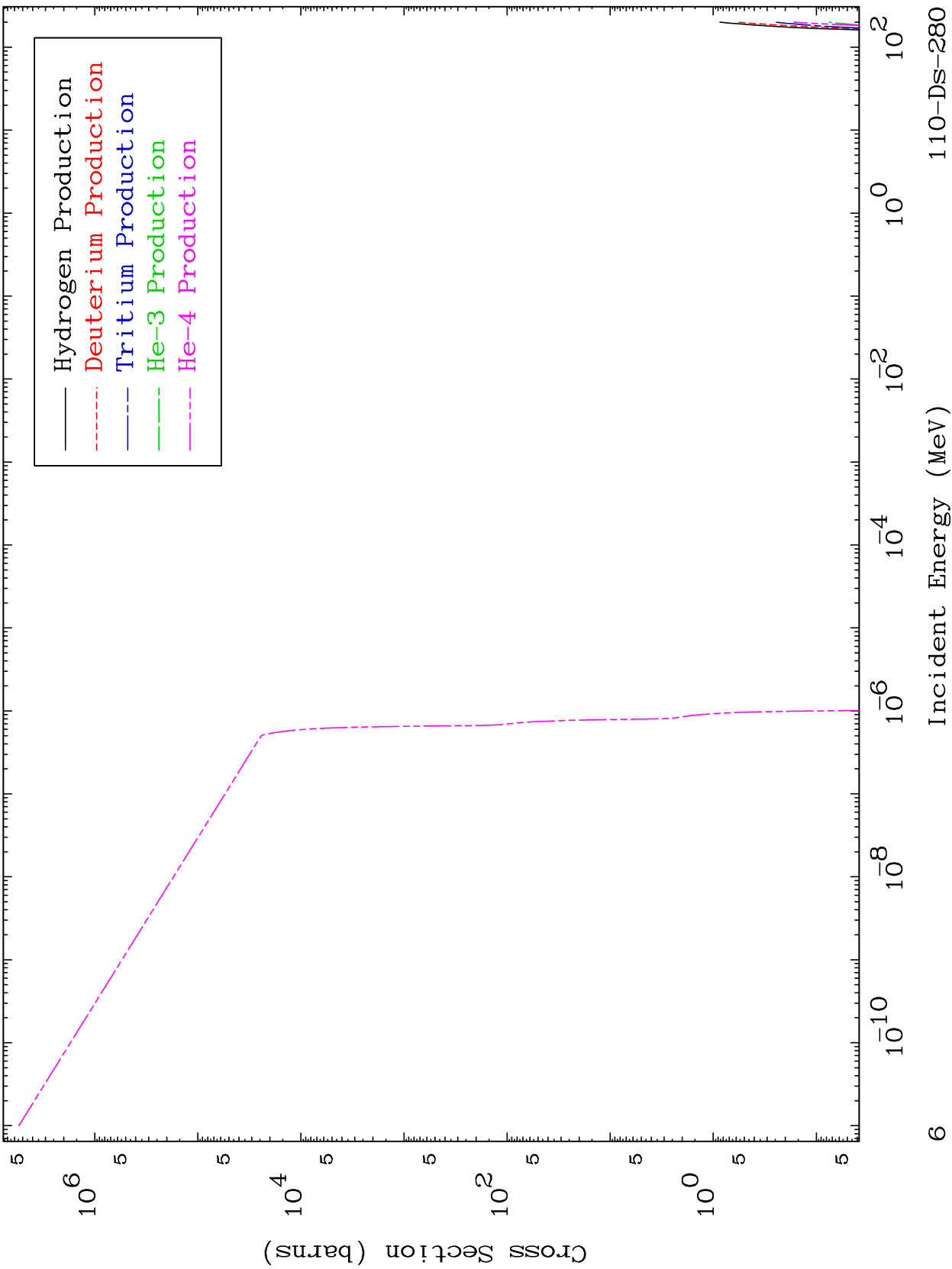
110-Ds-280

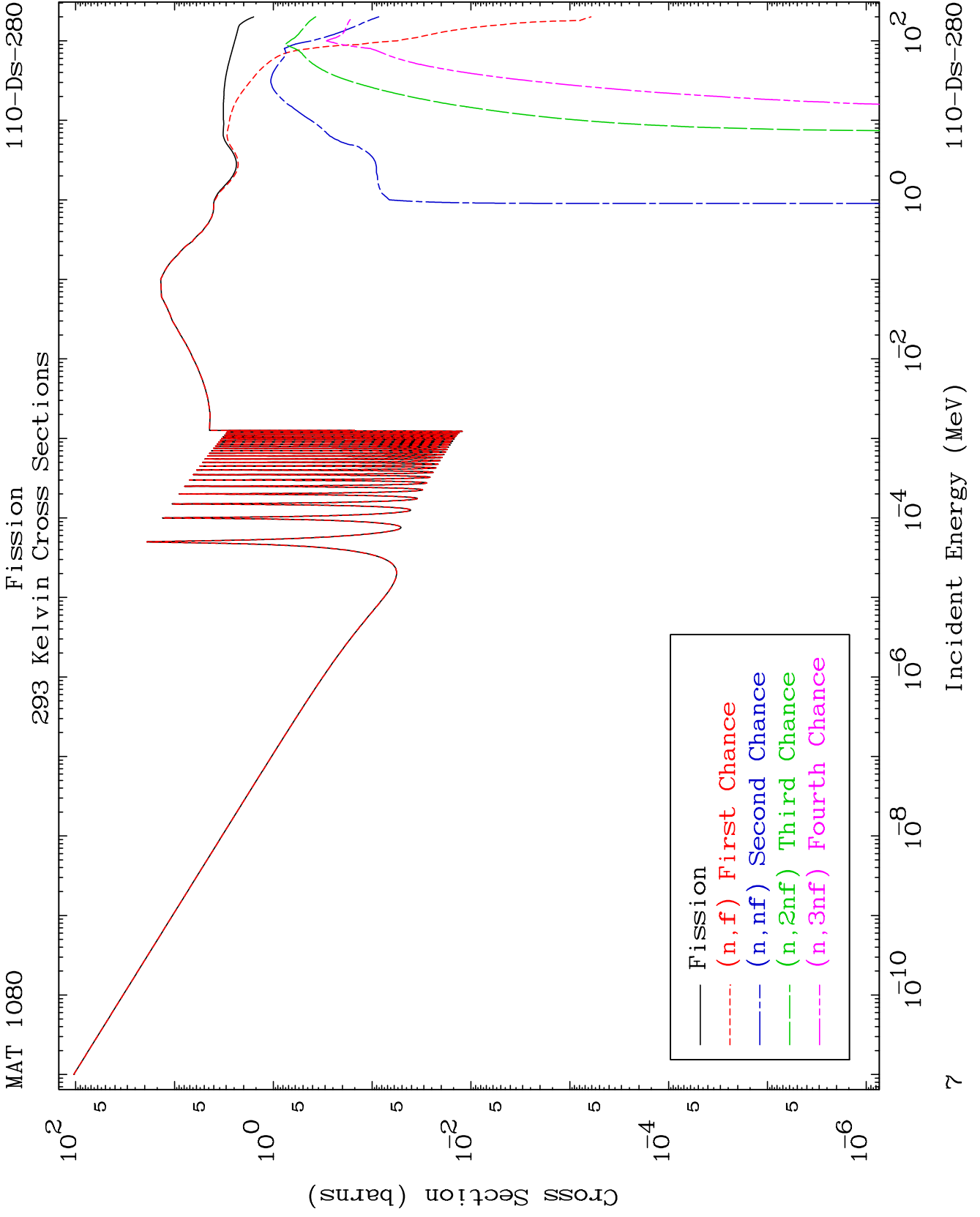


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Particle Production
293 Kelvin Cross Sections

110-Ds-280



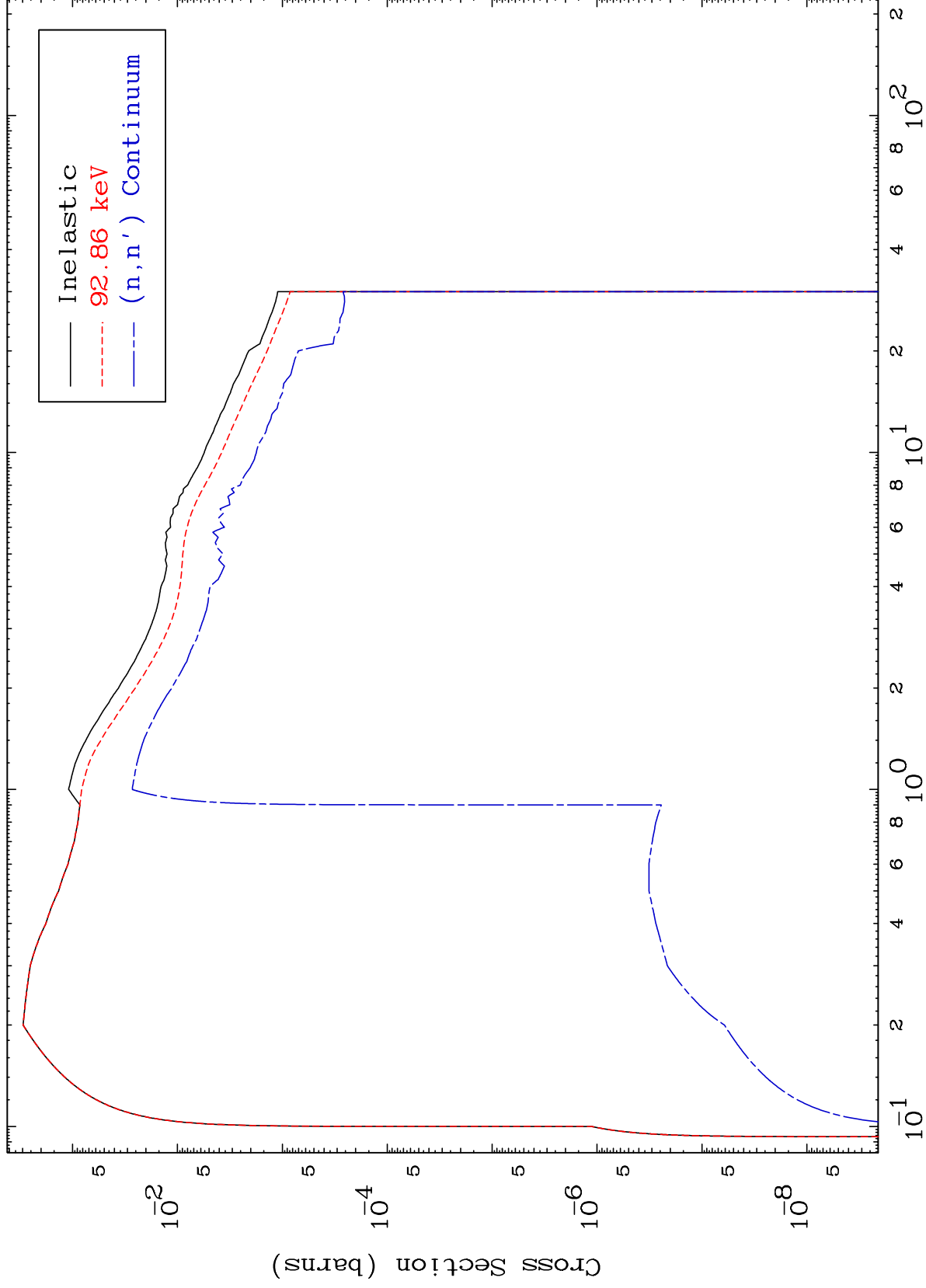


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(n,n') Level

110-Ds-280

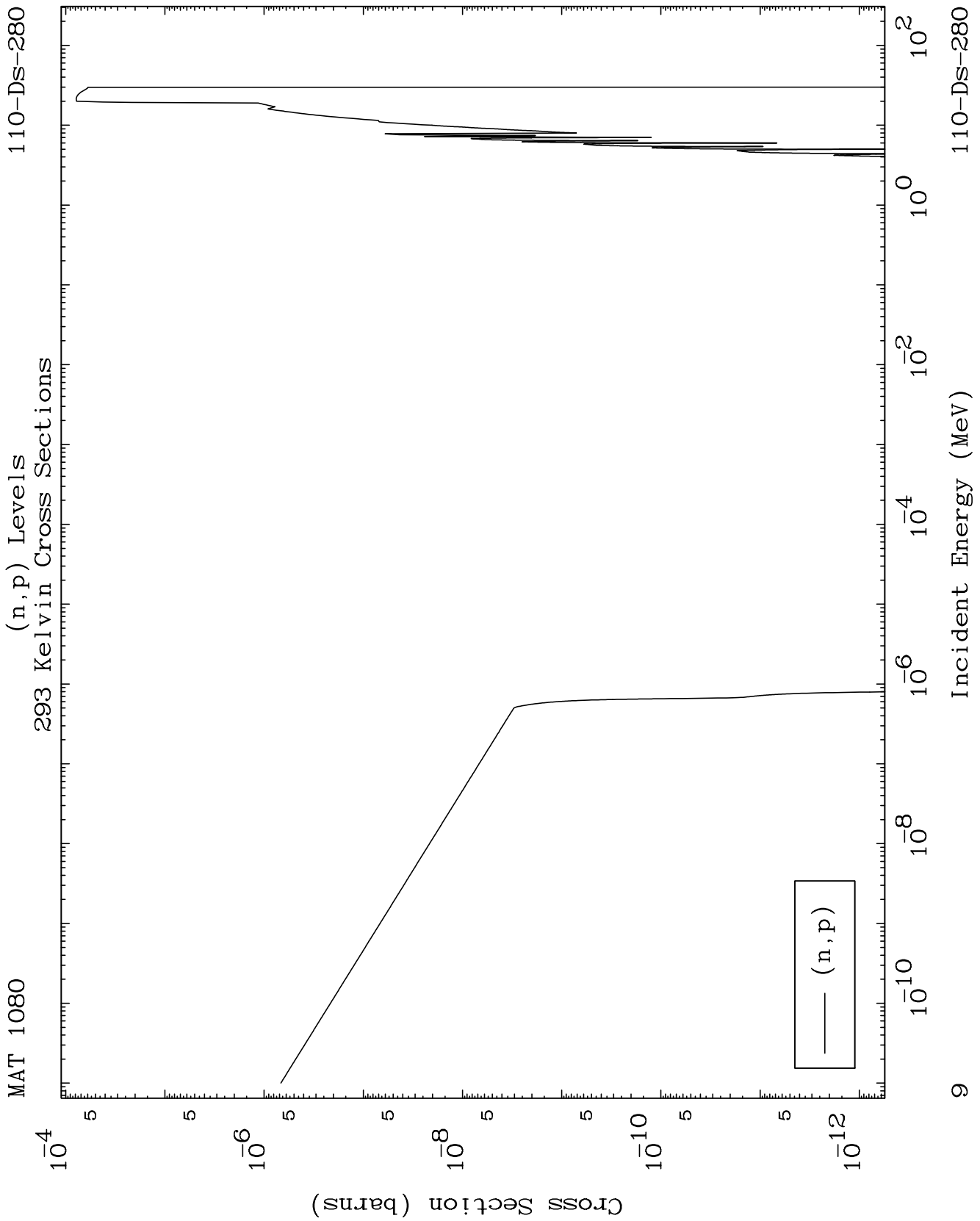
293 Kelvin Cross Sections



Incident Energy (MeV)

110-Ds-280

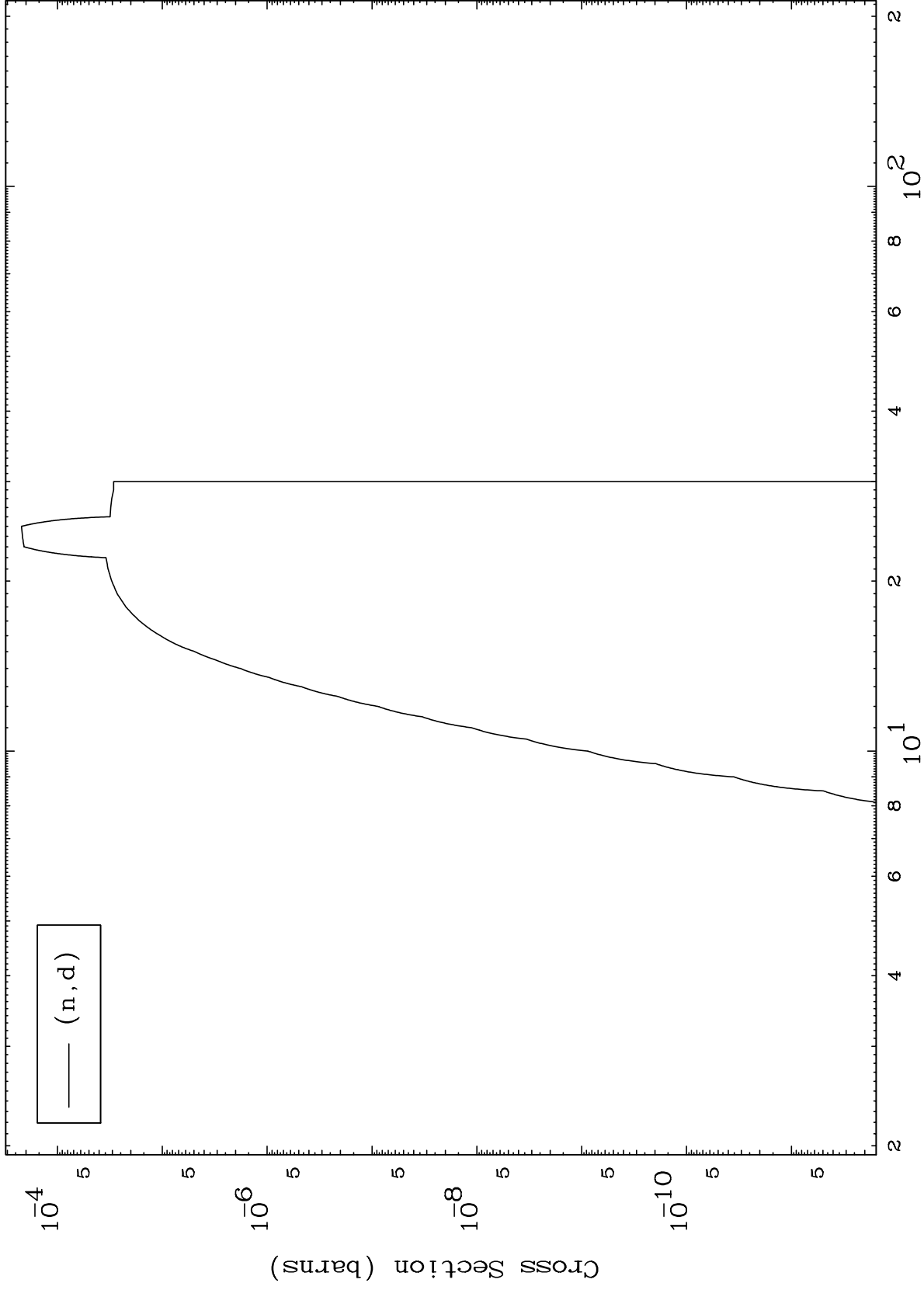
8



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(n,d) Levels
293 Kelvin Cross Sections

110-Ds-280



10

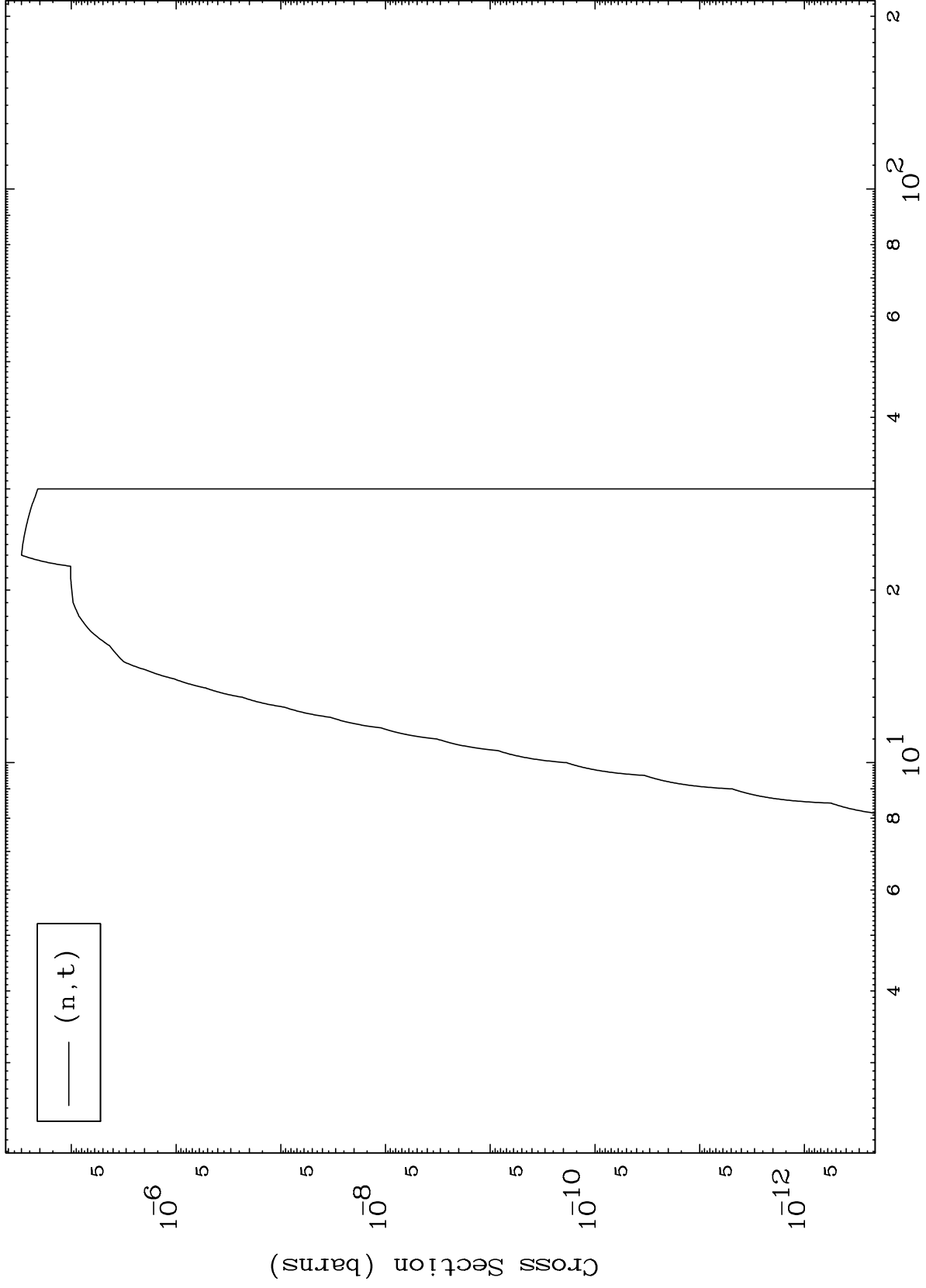
Incident Energy (MeV)

110-Ds-280

MAT 1080

(n,t) Levels
293 Kelvin Cross Sections

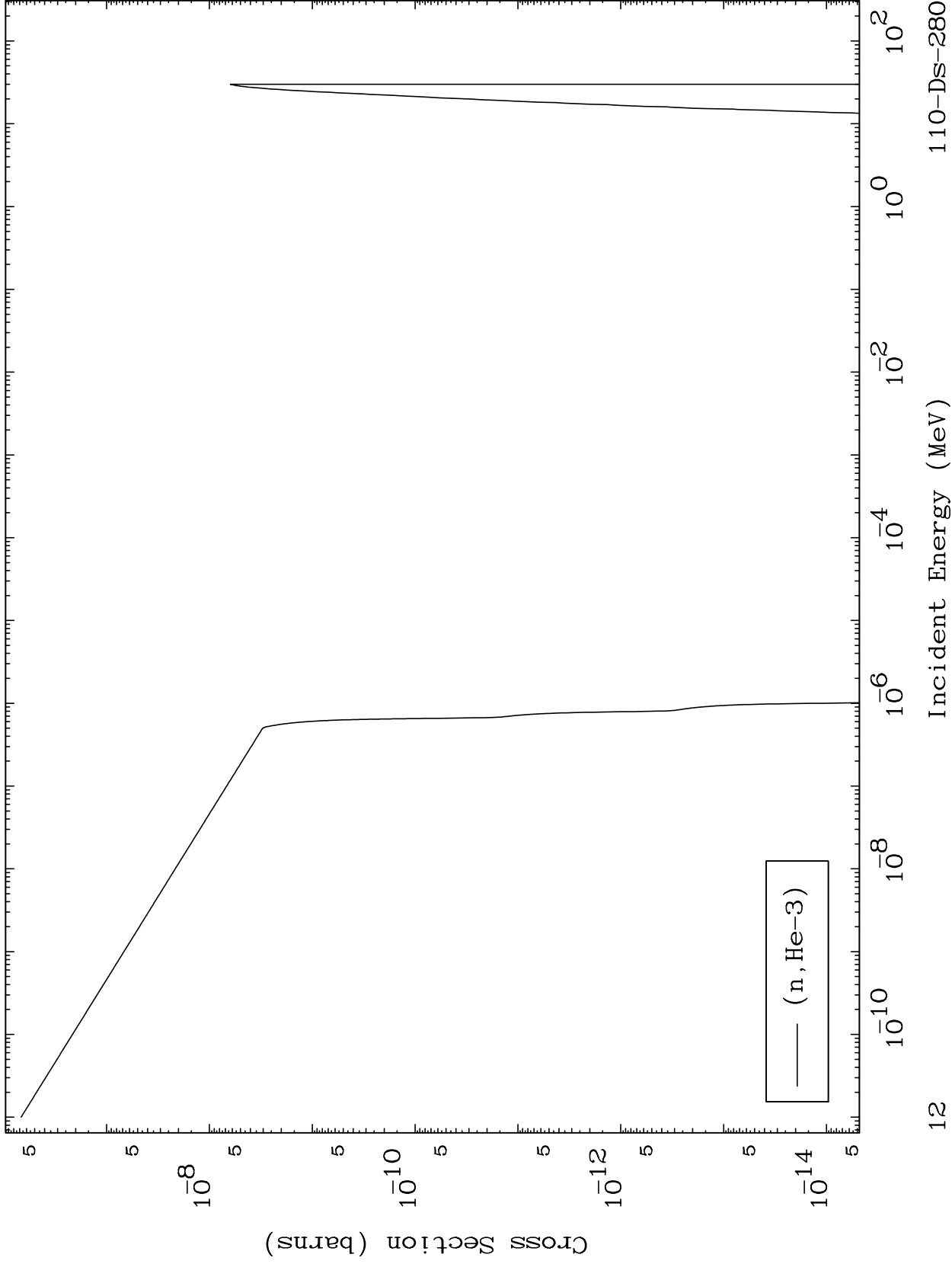
110-Ds-280



MAT 1080

(n,He3) Levels
293 Kelvin Cross Sections

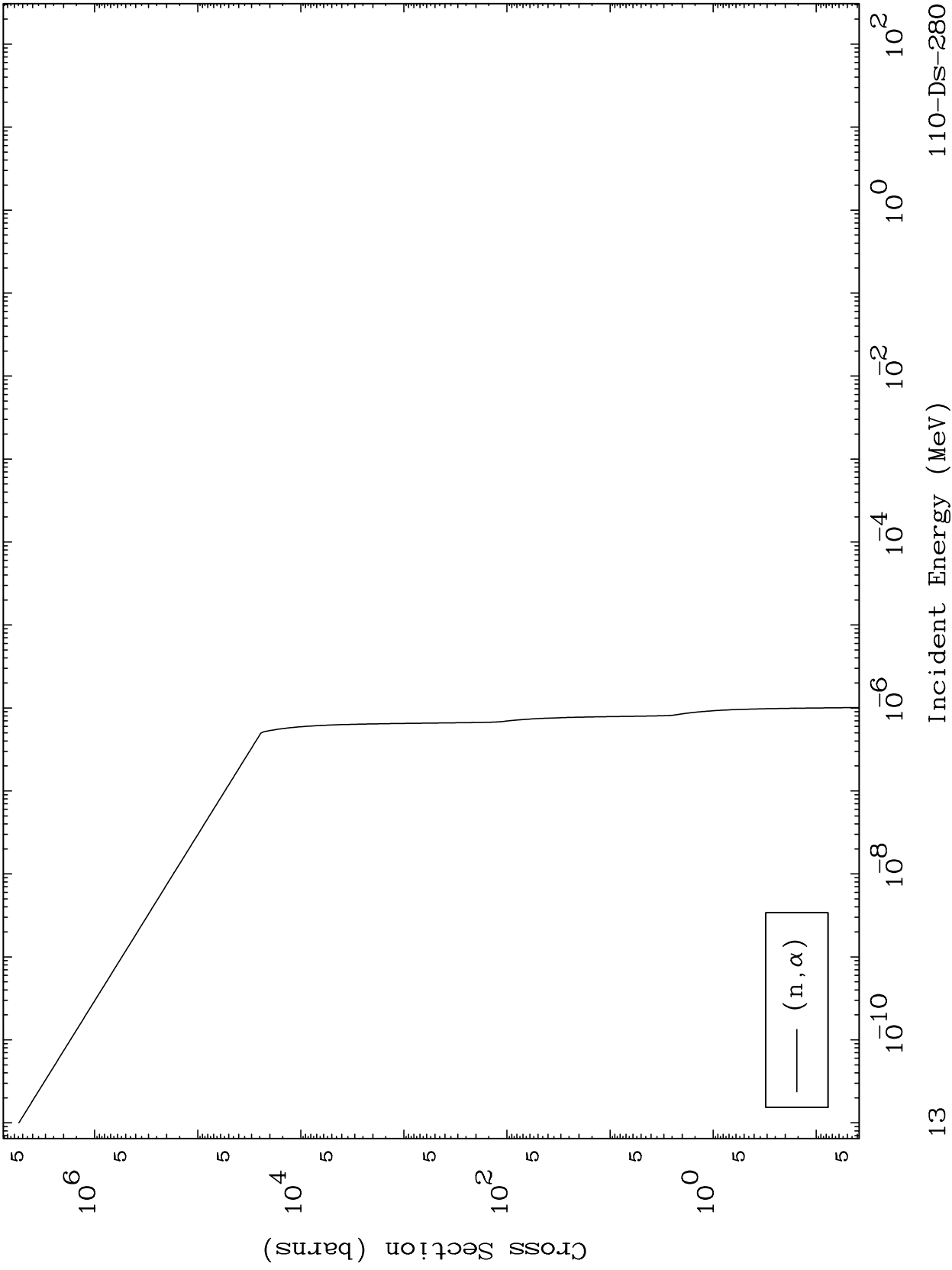
110-Ds-280

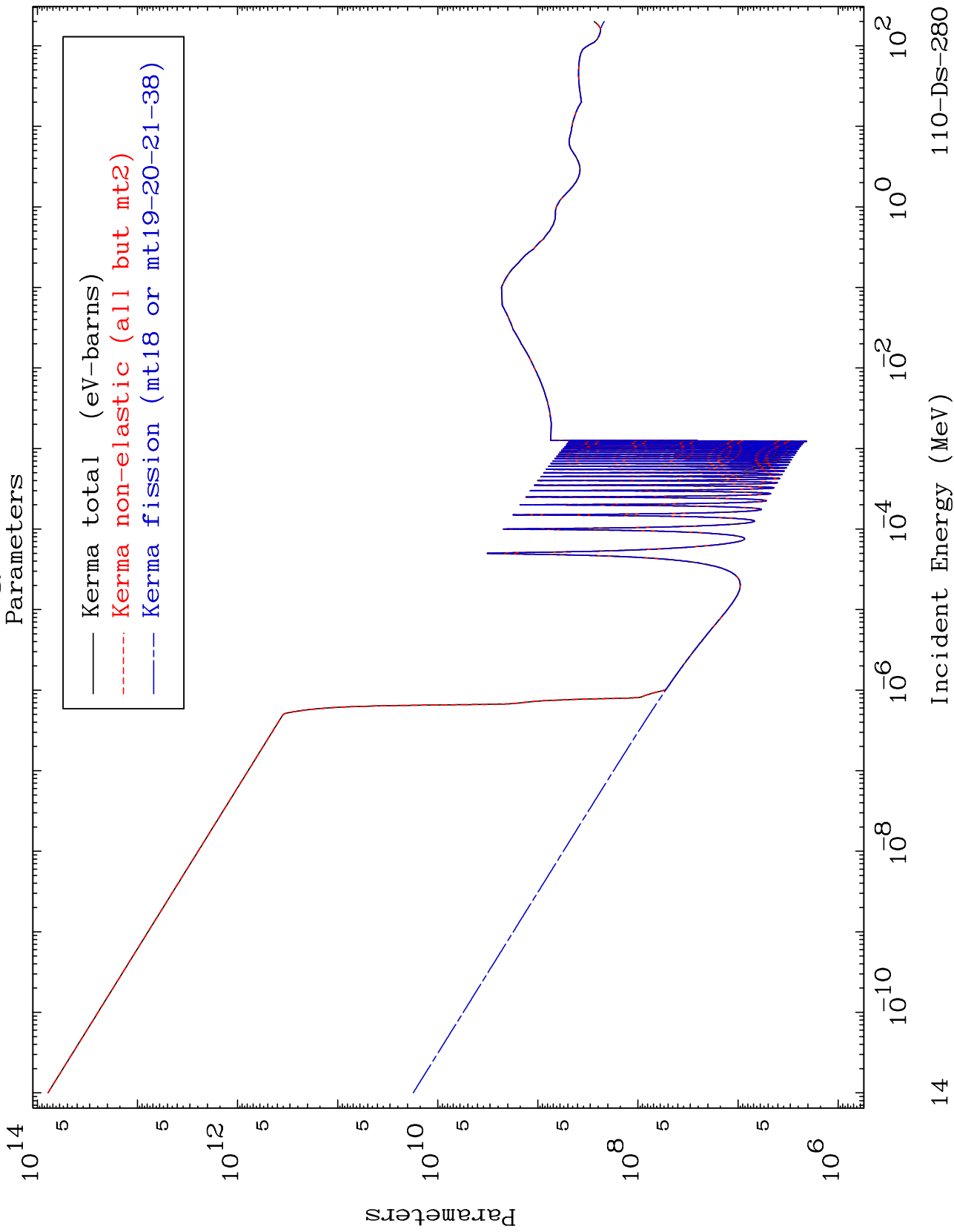


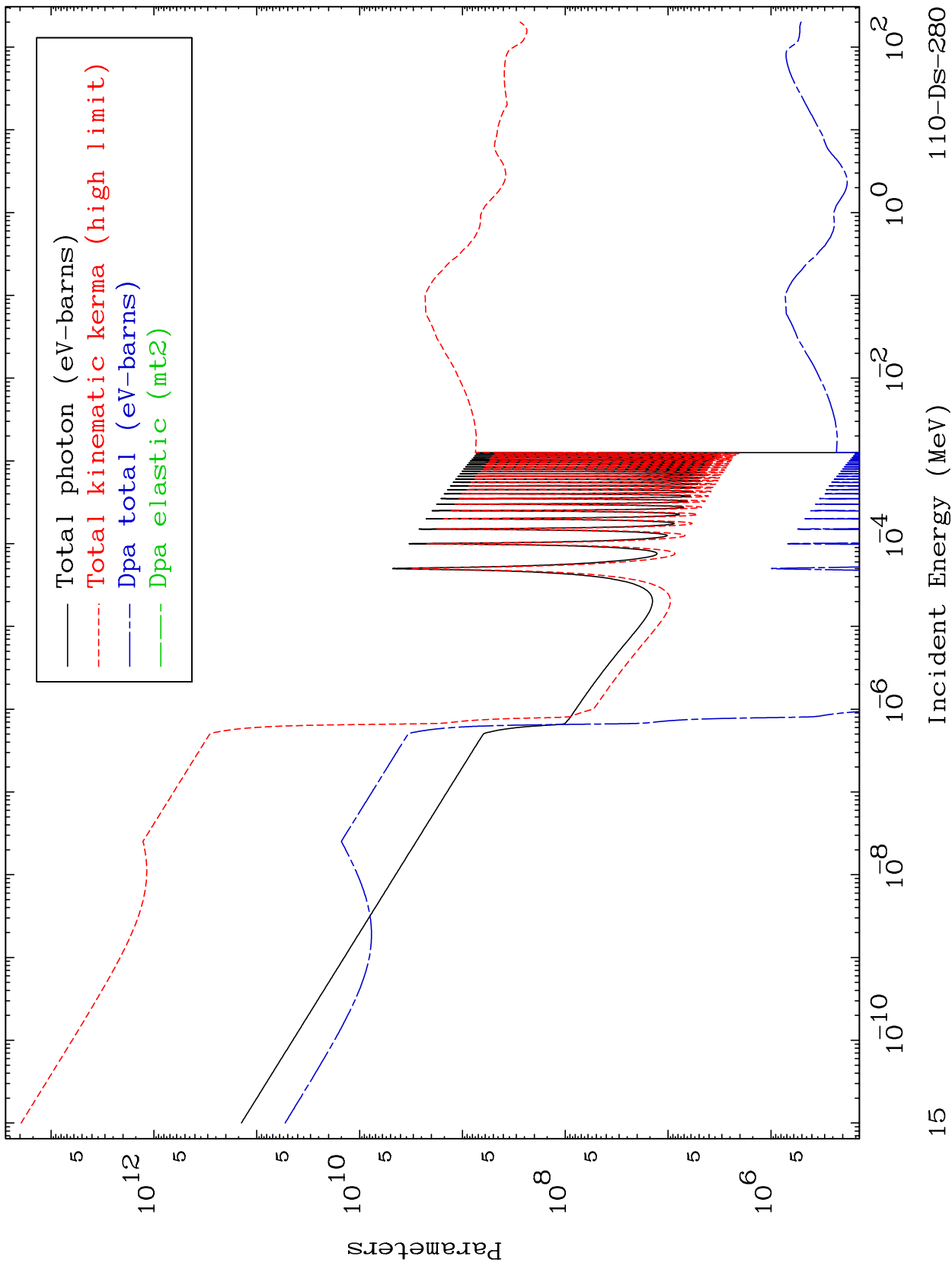
MAT 1080

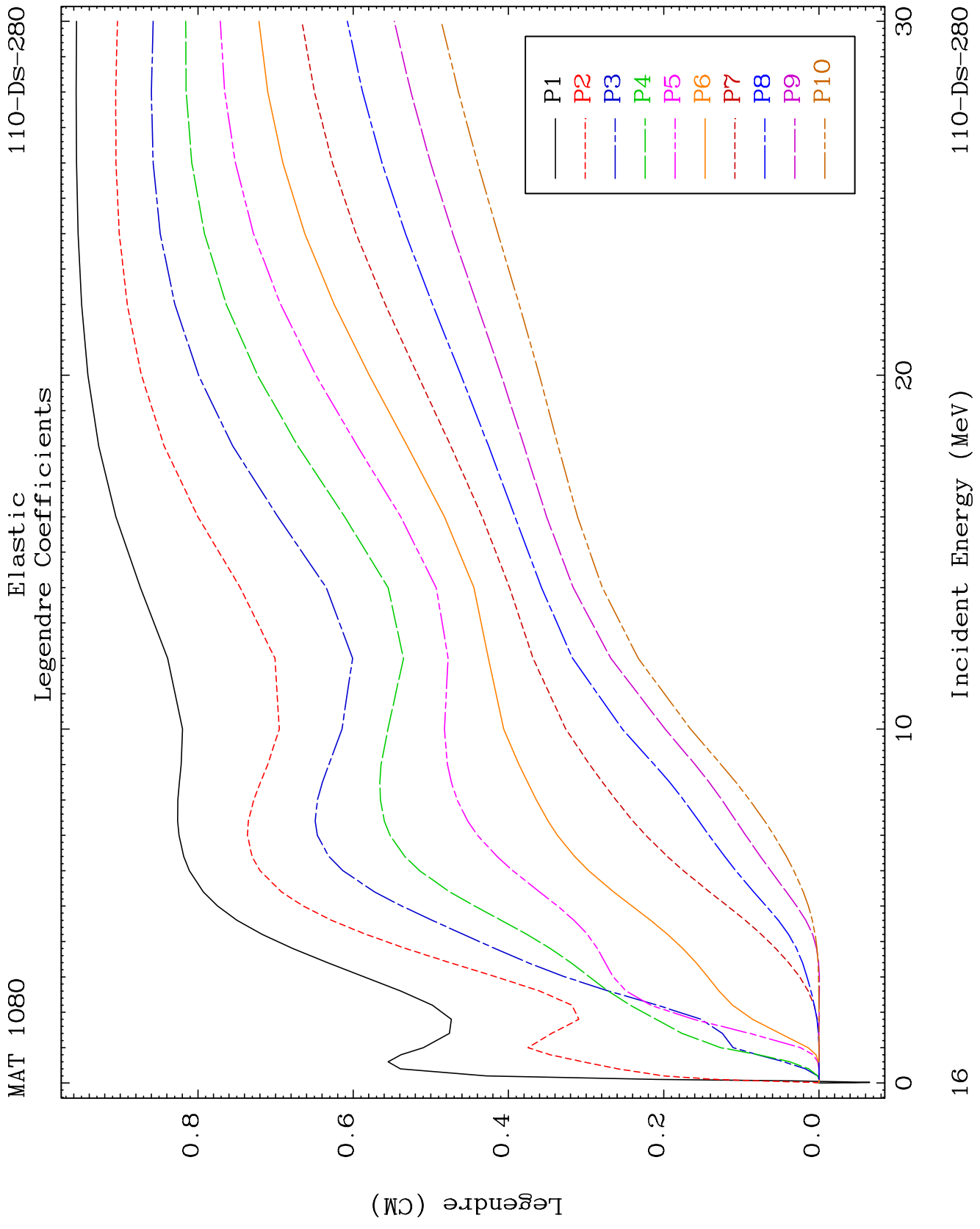
(n, α) Levels
293 Kelvin Cross Sections

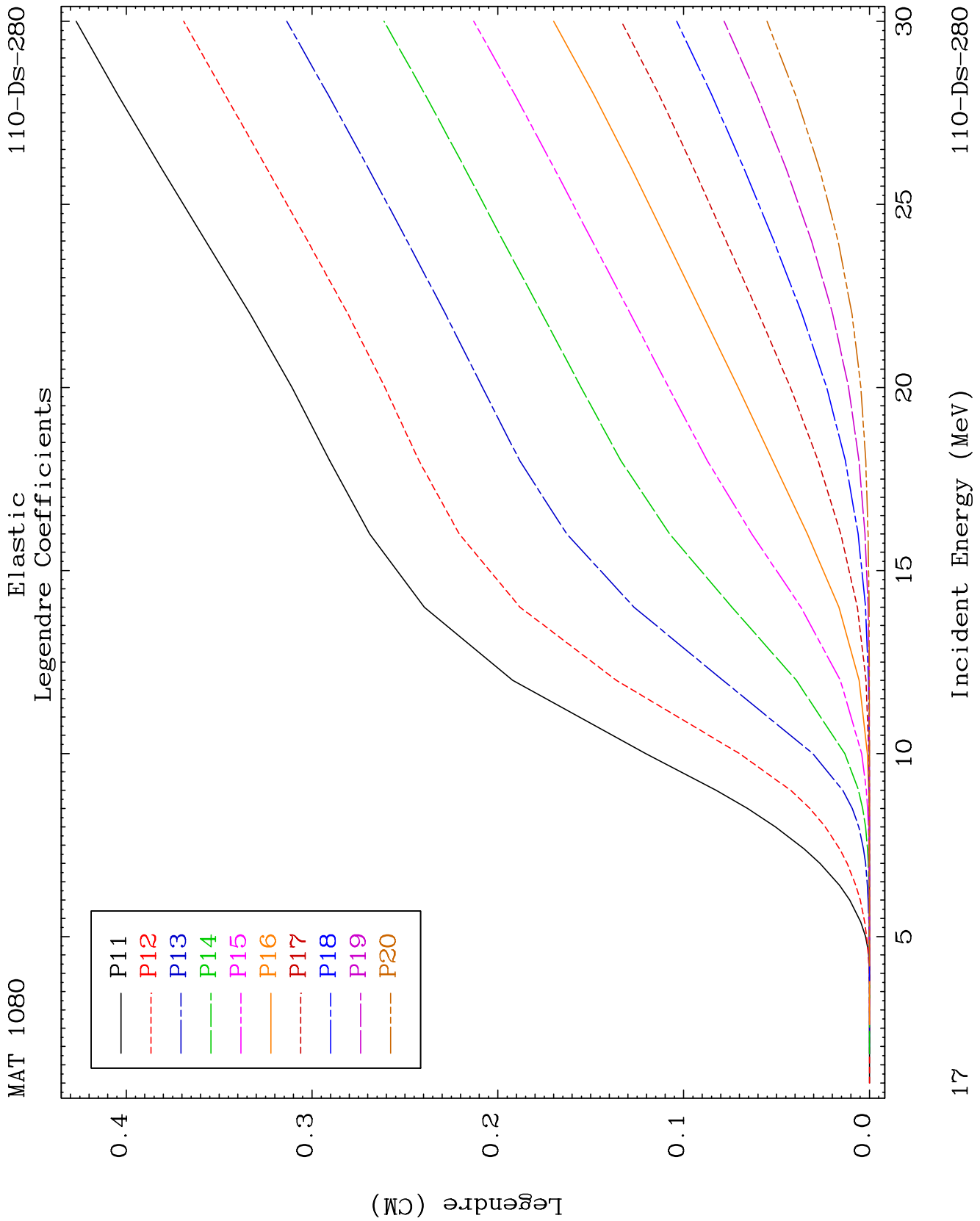
110-Ds-280







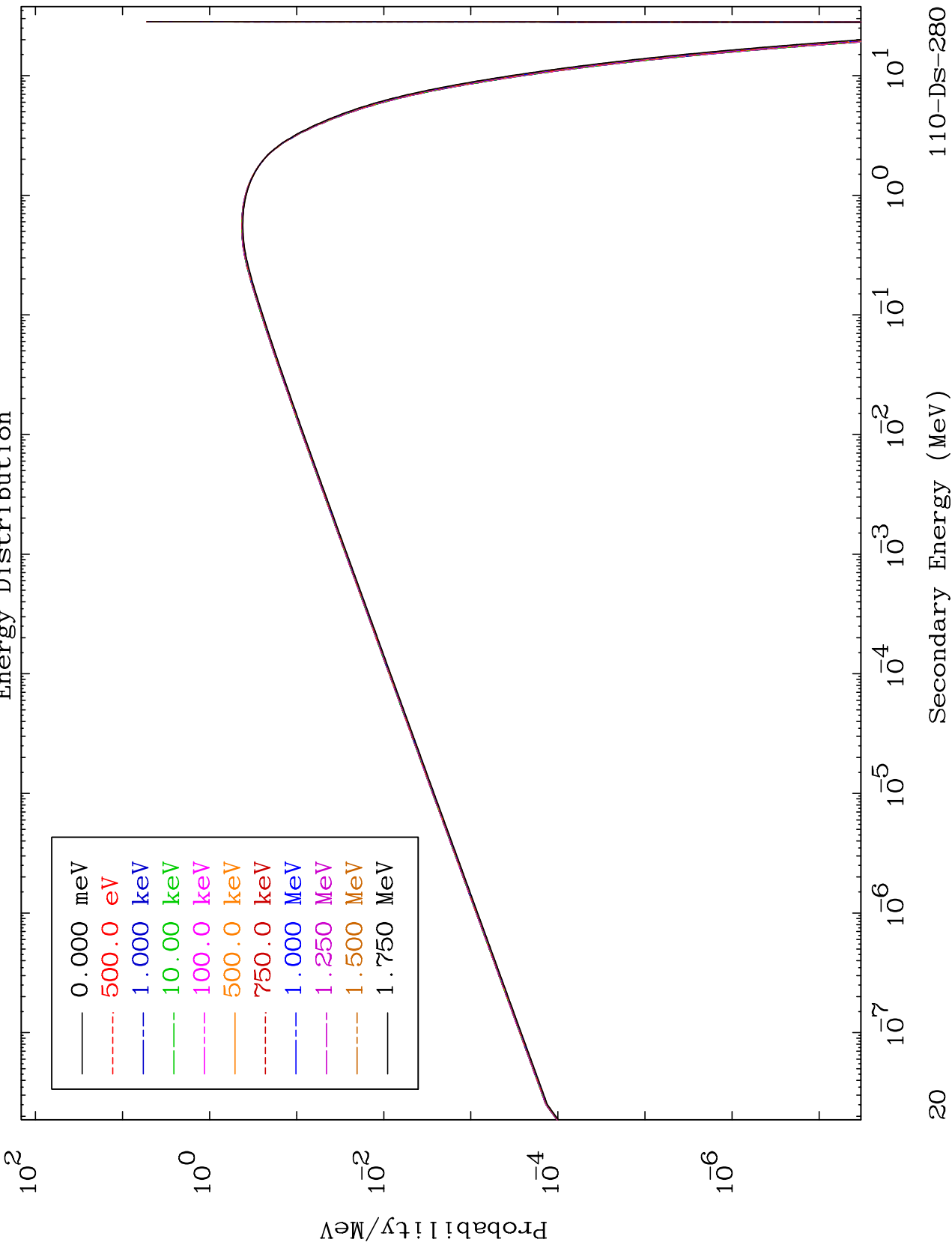




MAT 1080

Fission Energy Distribution

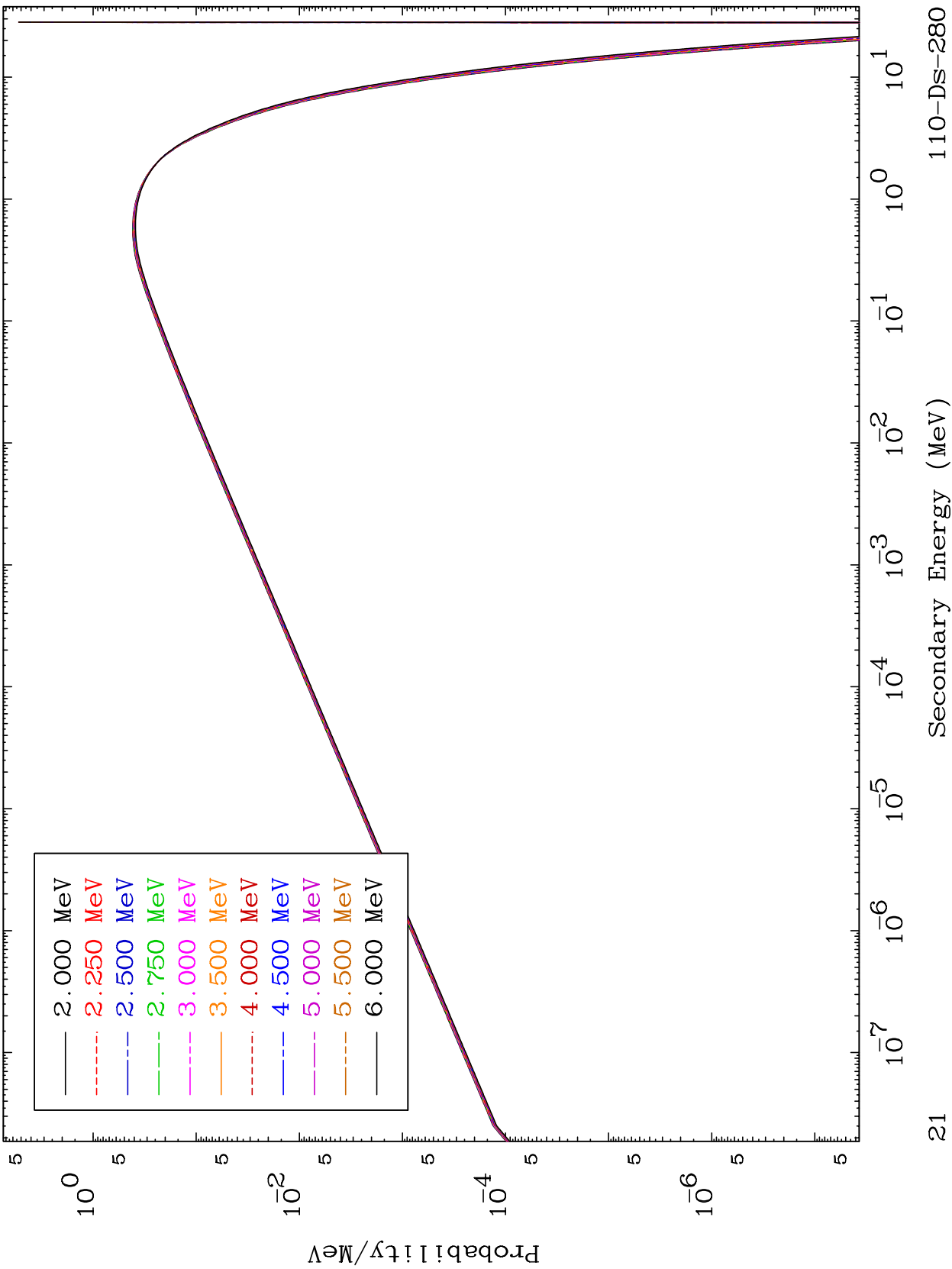
110-Ds-280



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Fission Energy Distribution

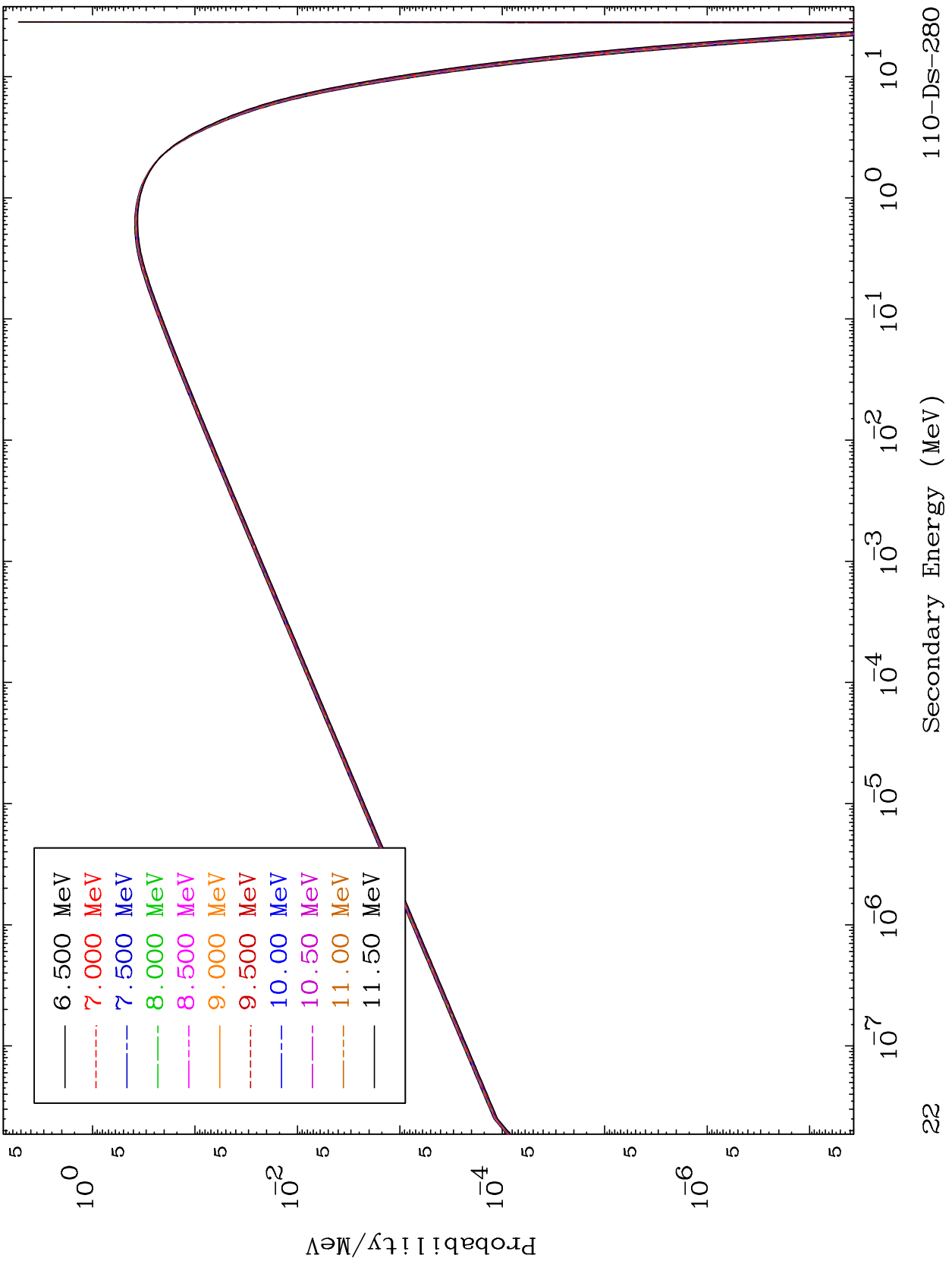
110-Ds-280



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Fission
Energy Distribution

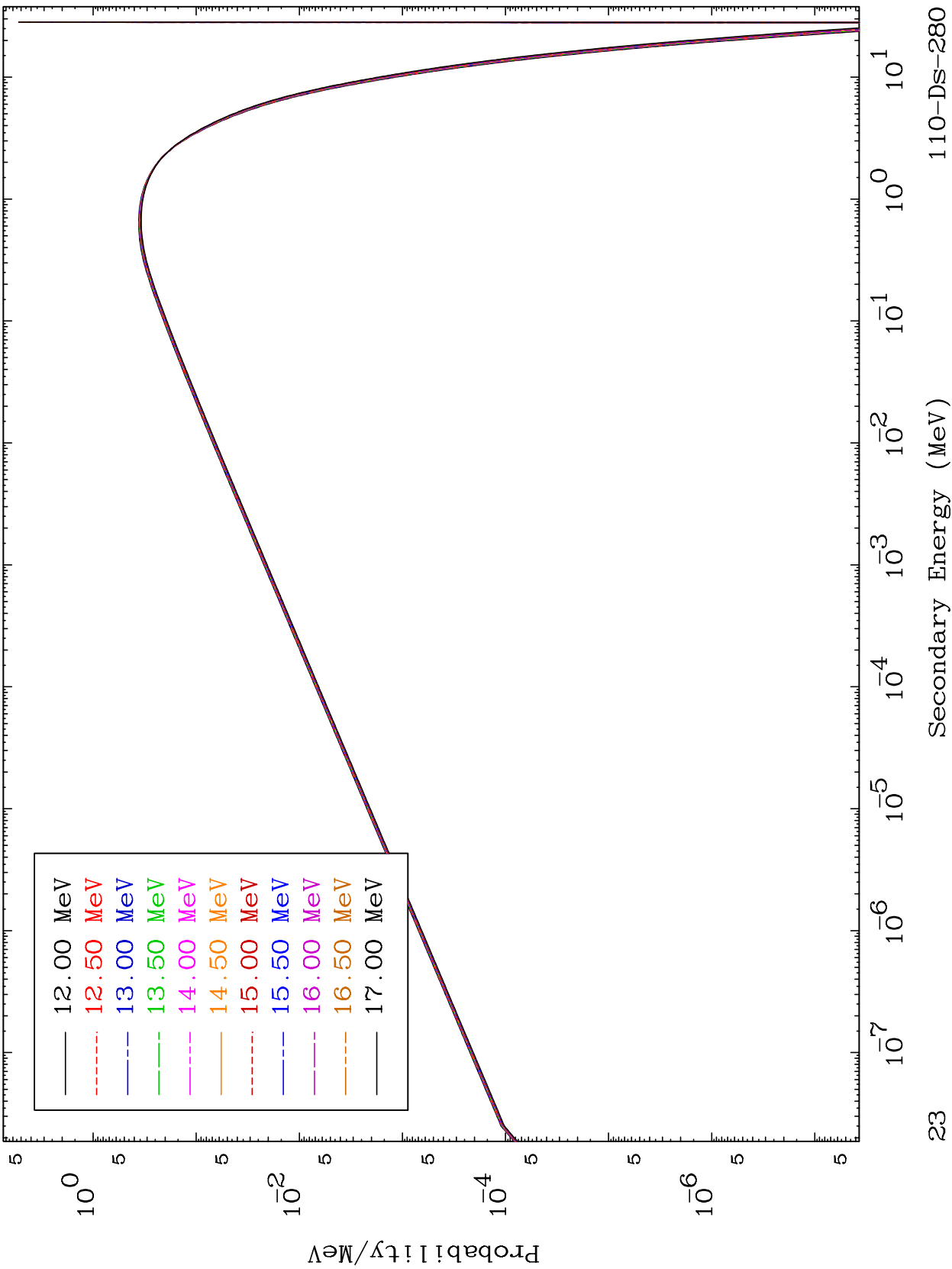
110-Ds-280



MAT 1080

Fission Energy Distribution

110-Ds-280



Fission
Energy Distribution

