

Program EVALPLOT  
(Version 2015-2)

by

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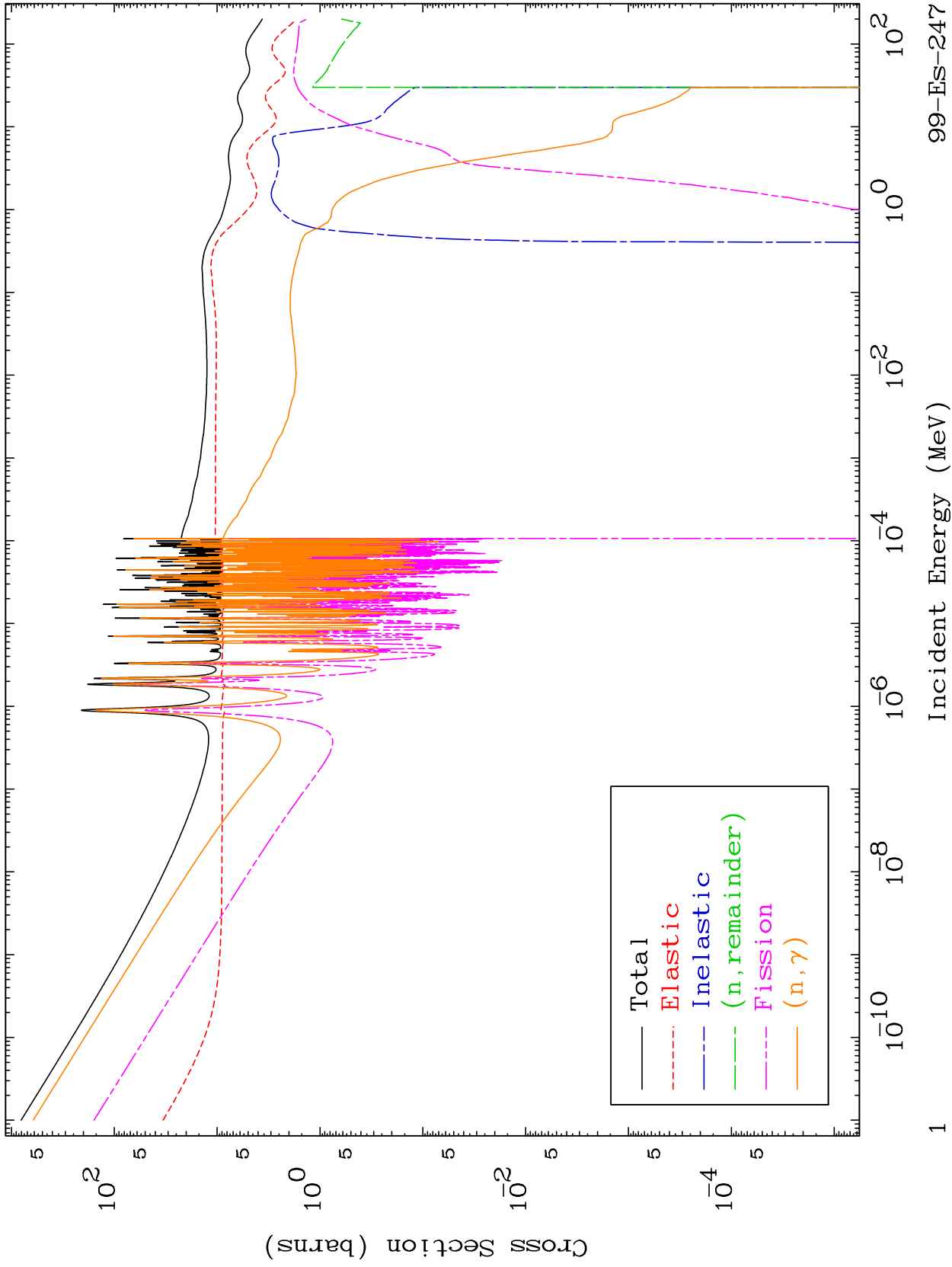
Press Mouse Button to Start

MAT 9907

Major

293 Kelvin Cross Sections

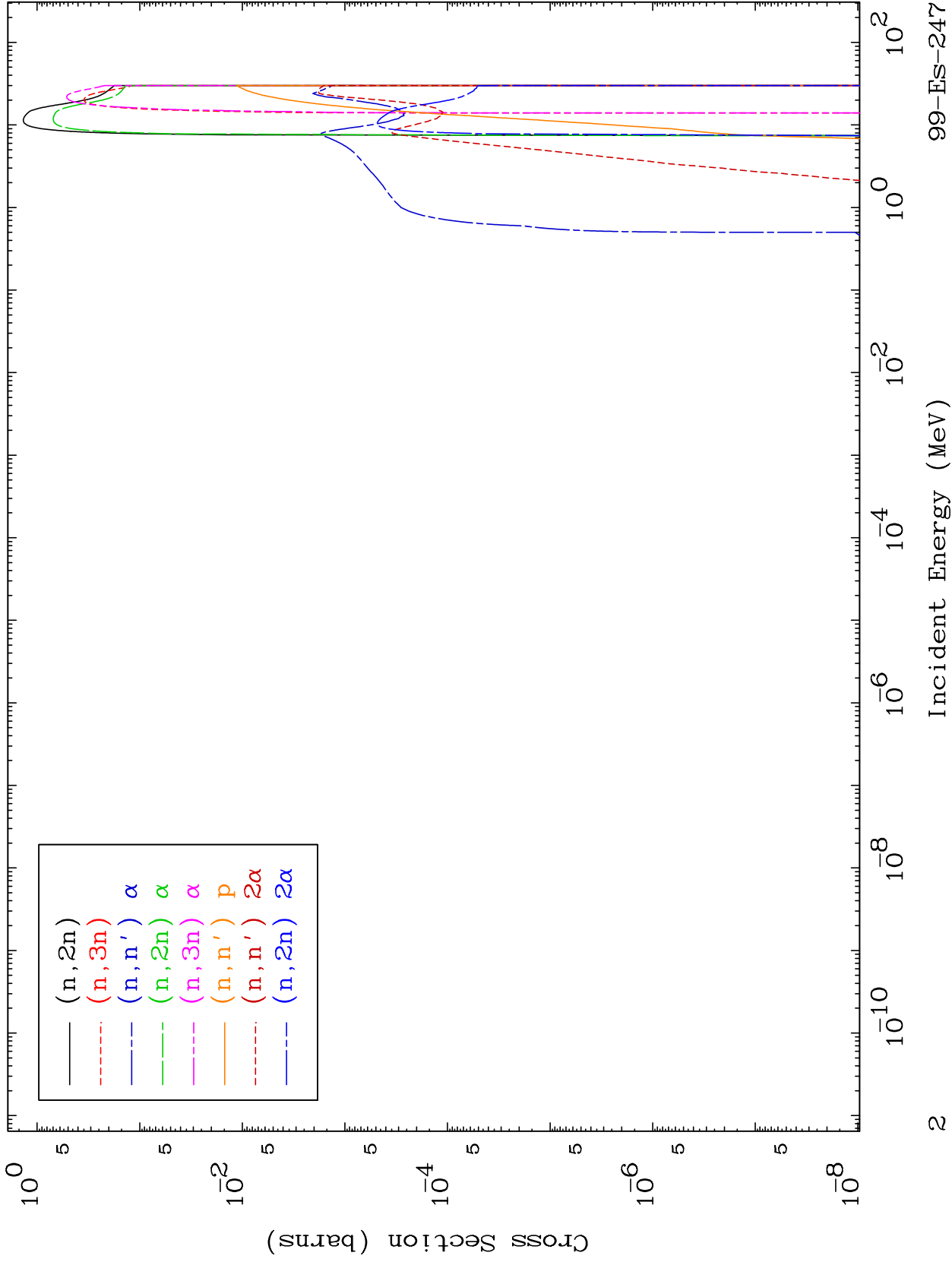
99-Es-247



MAT 9907

Neutron Production  
293 Kelvin Cross Sections

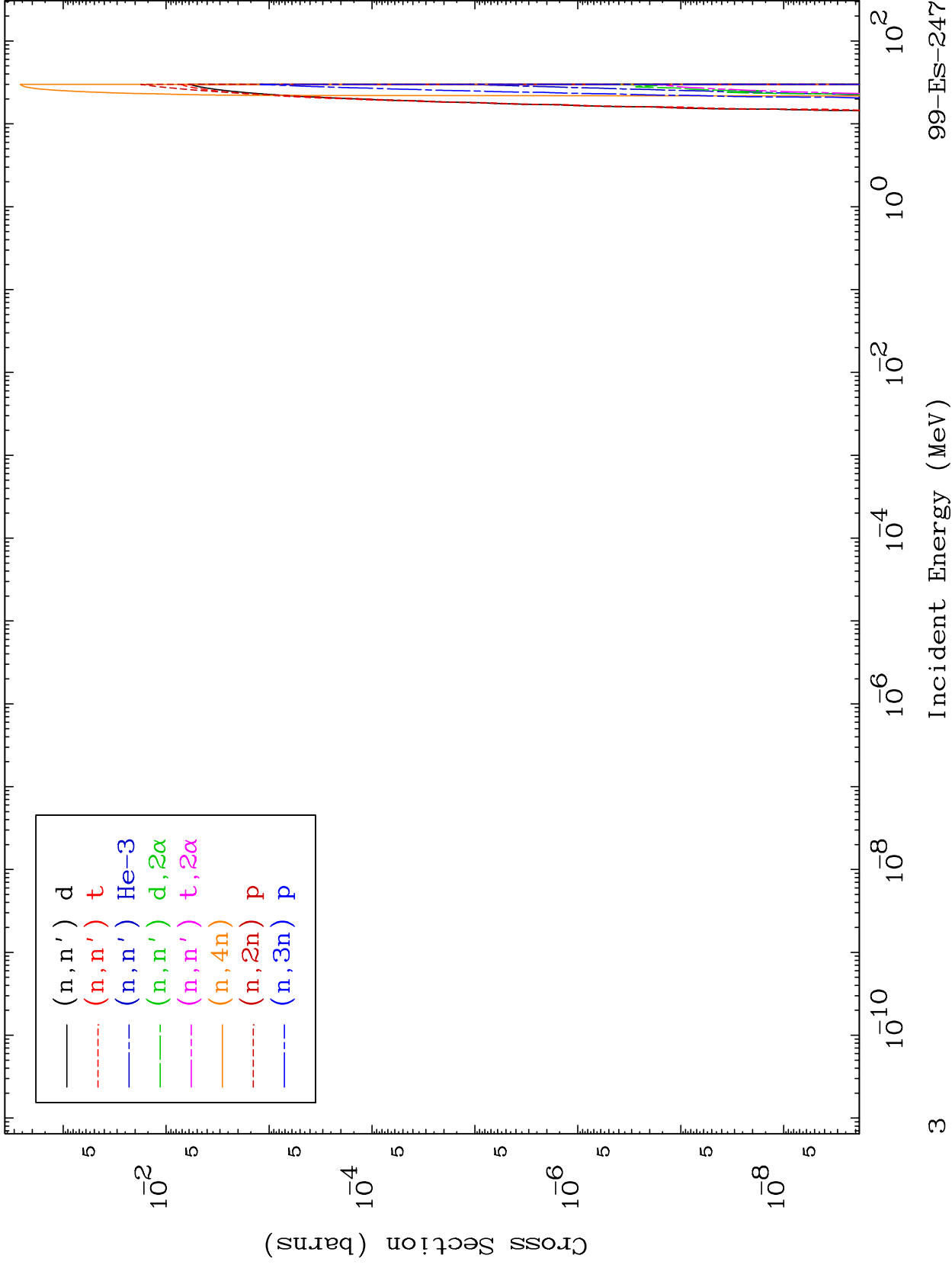
99-Es-247



MAT 9907

Neutron Production  
293 Kelvin Cross Sections

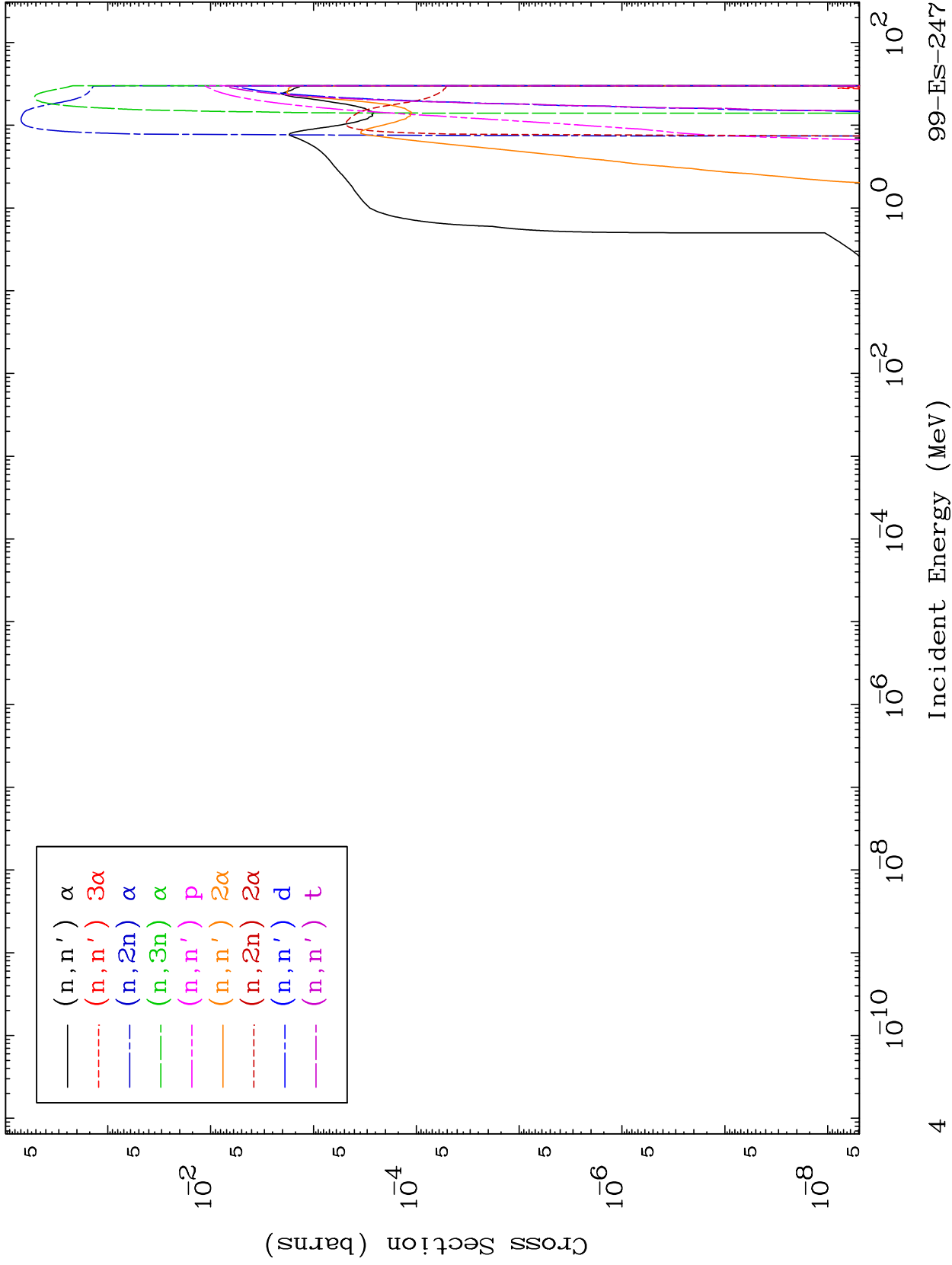
99-Es-247



MAT 9907

Charged Particle  
293 Kelvin Cross Sections

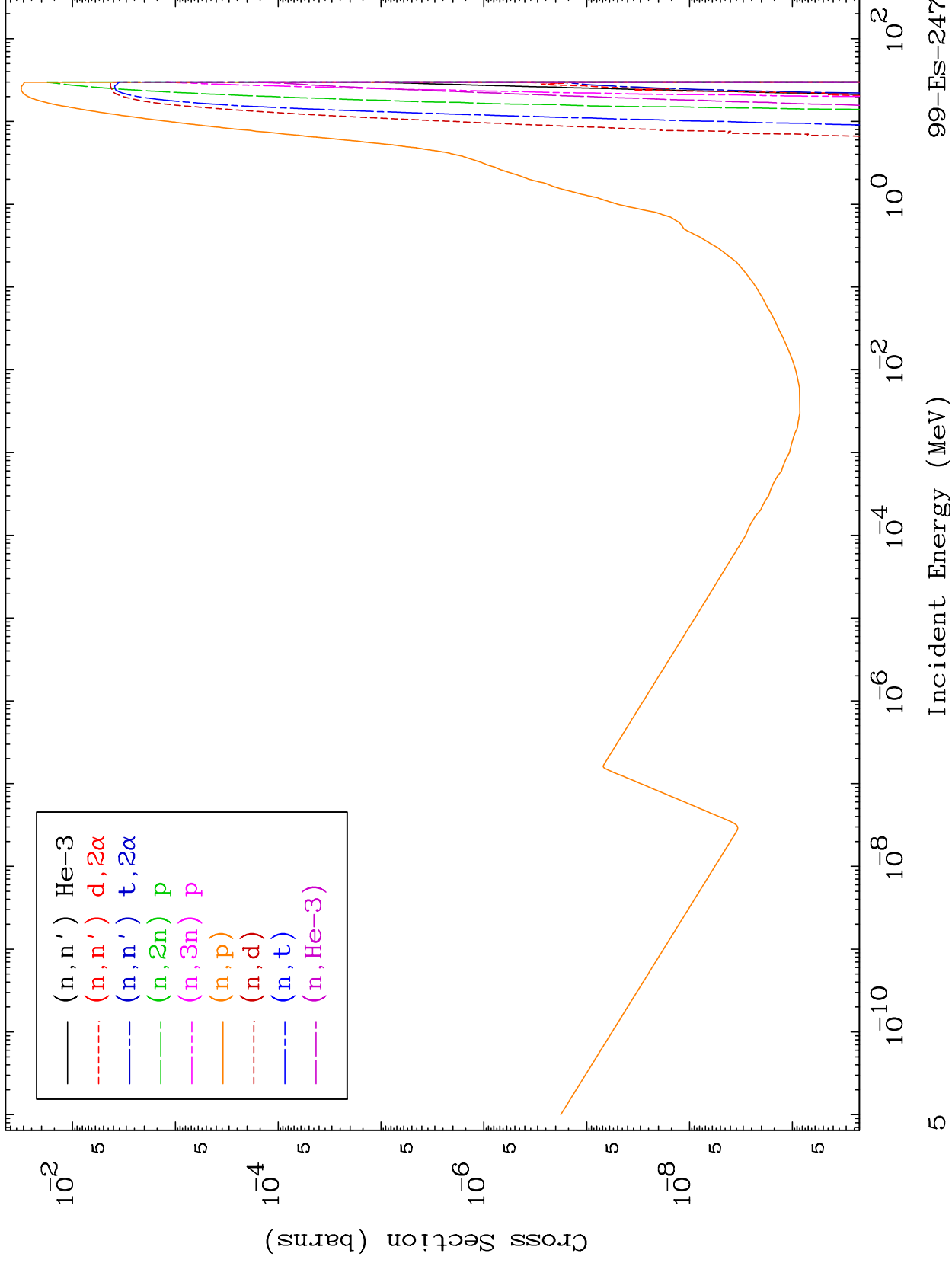
99-Es-247



MAT 9907

Charged Particle  
293 Kelvin Cross Sections

99-Es-247



99-Es-247

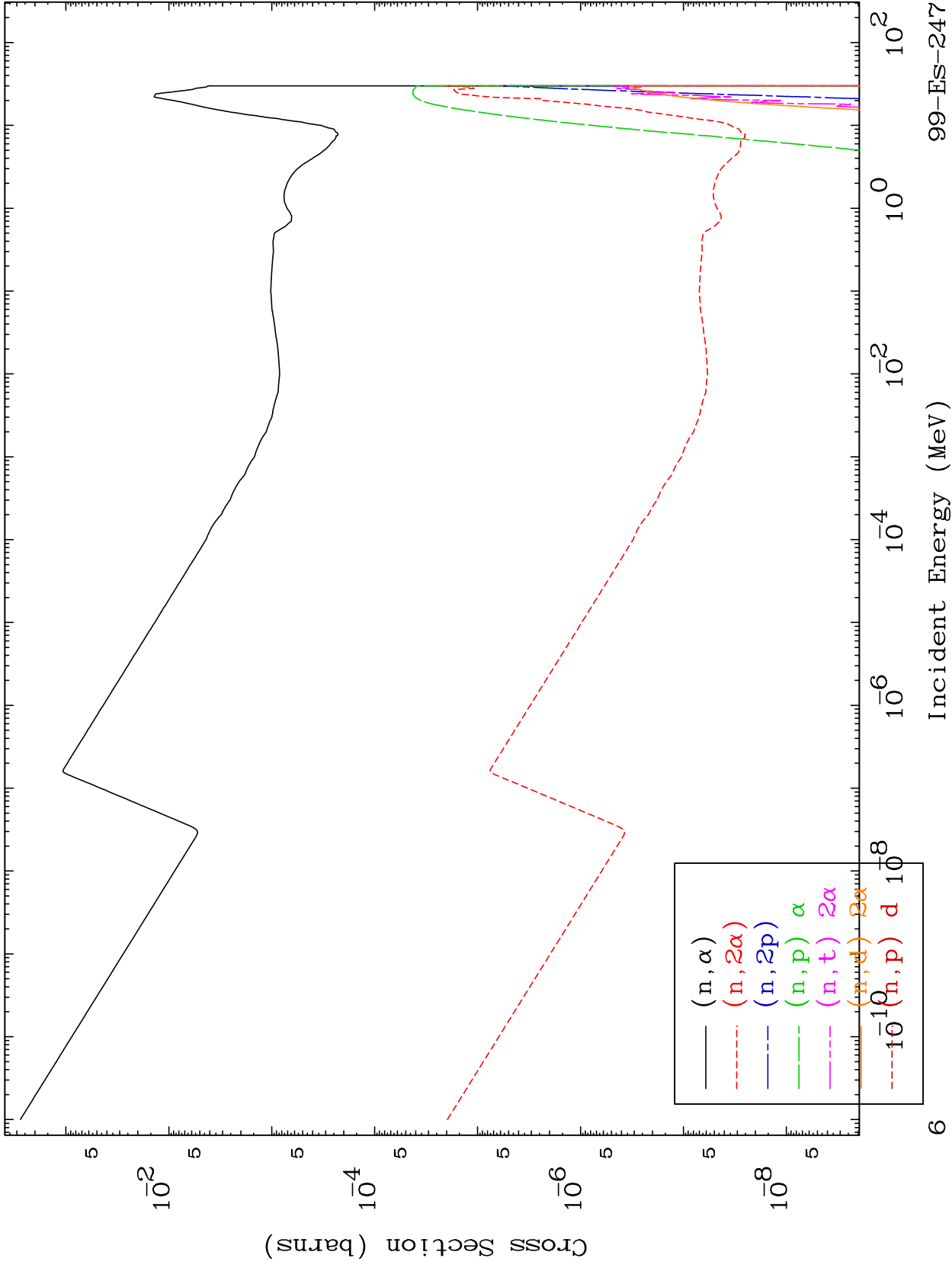
Incident Energy (MeV)

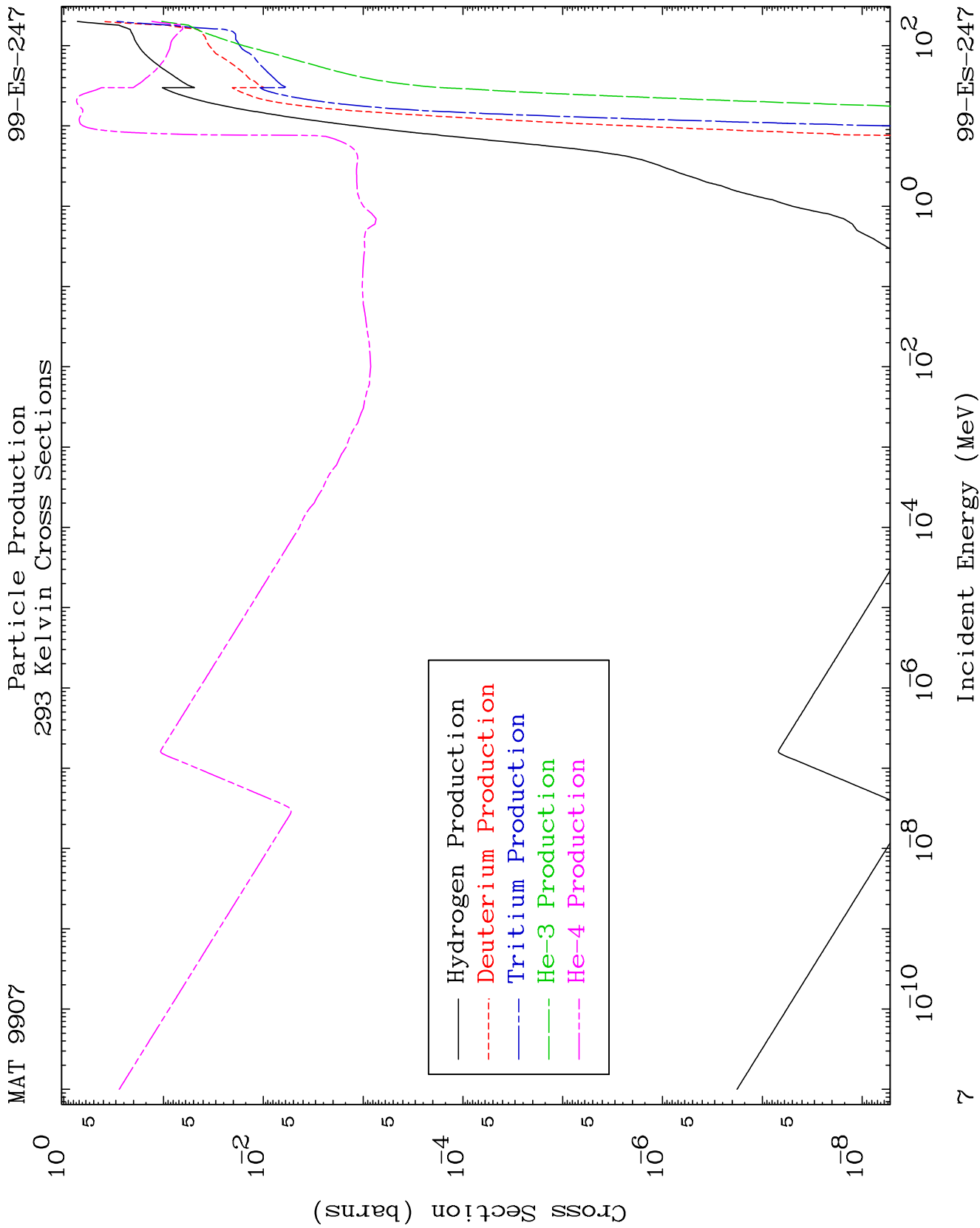
5

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Charged Particle  
293 Kelvin Cross Sections

99-Es-247



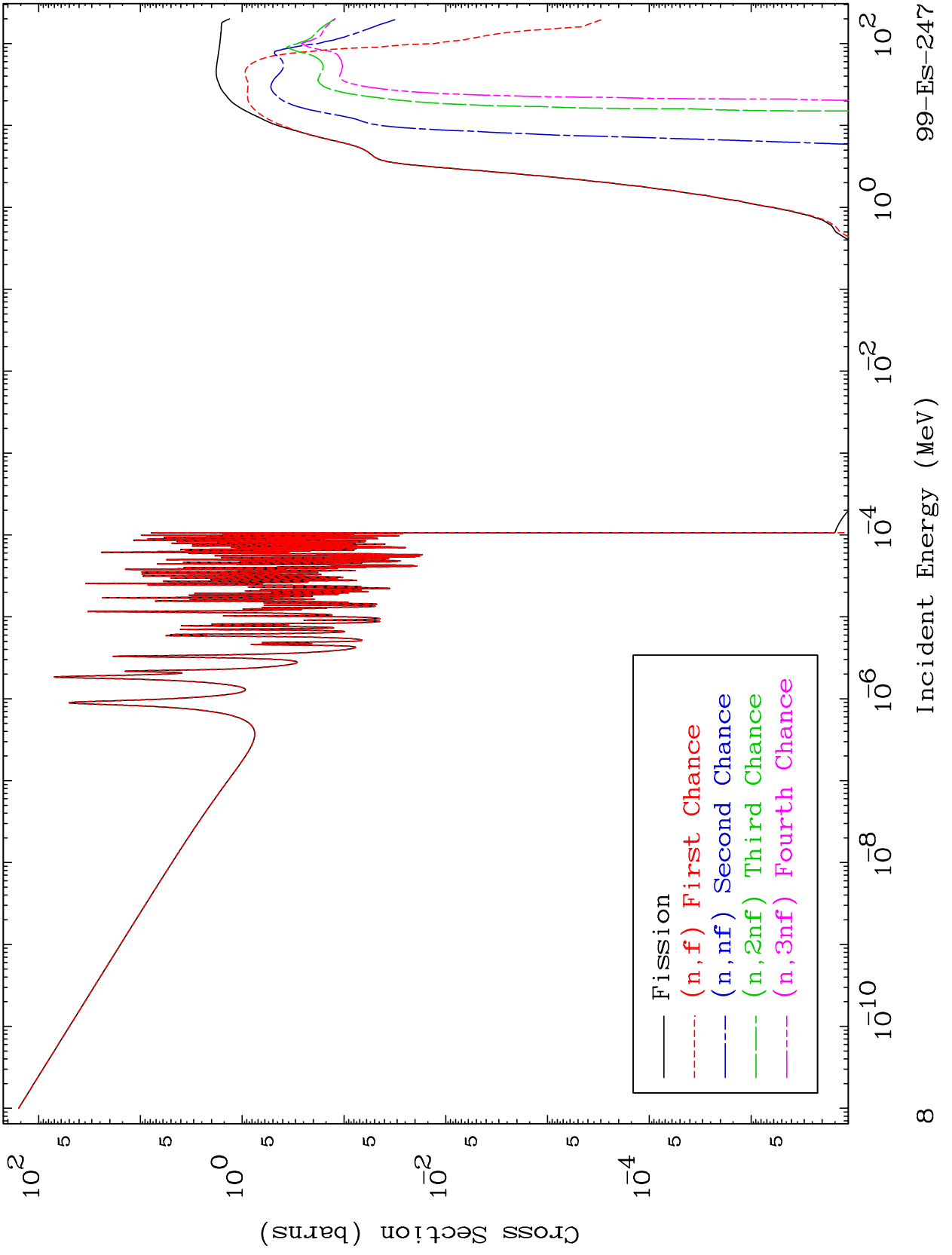




MAT 9907

Fission  
293 Kelvin Cross Sections

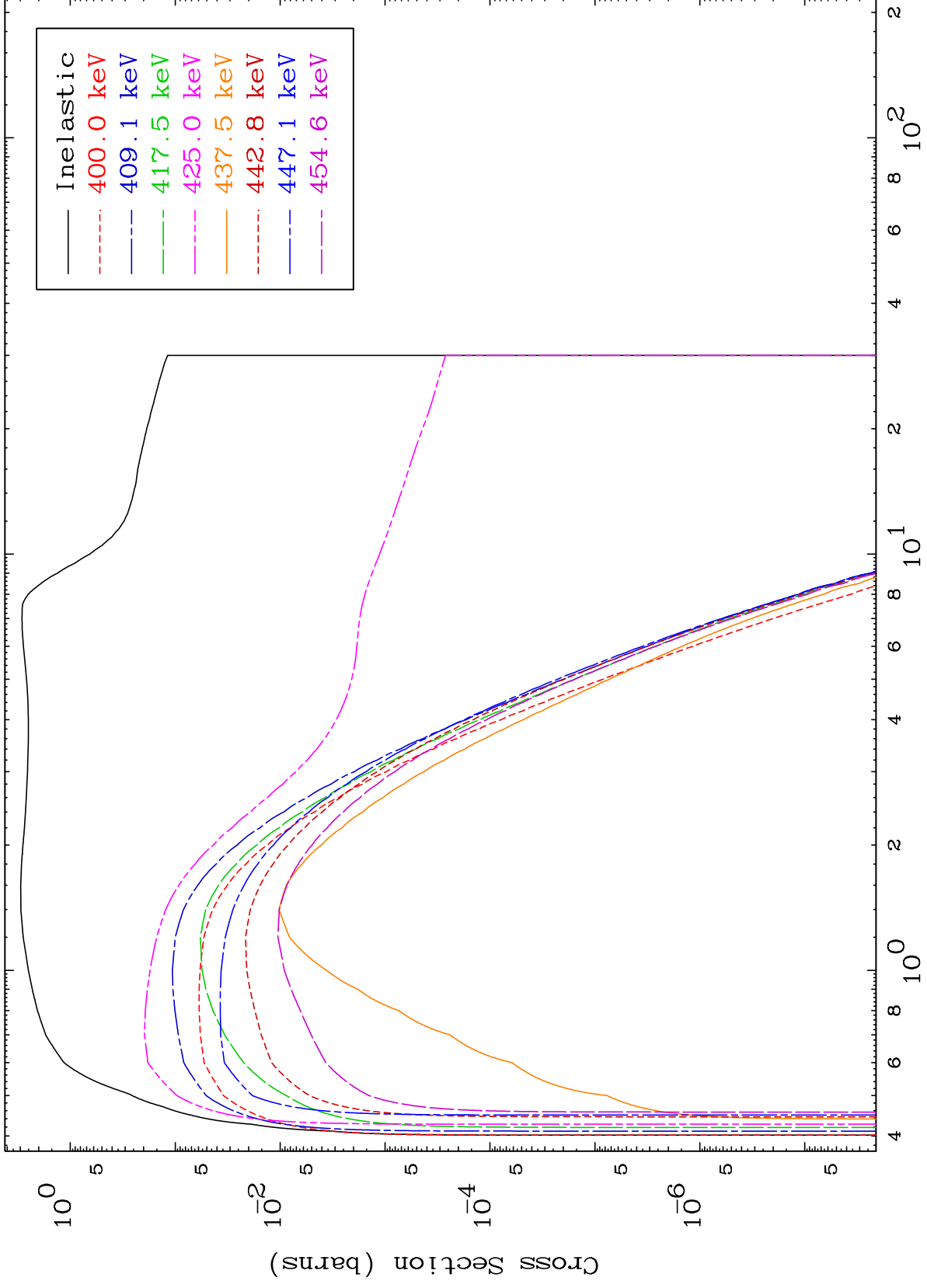
99-Es-247



MAT 9907

(n,n') Level  
293 Kelvin Cross Sections

99-Es-247



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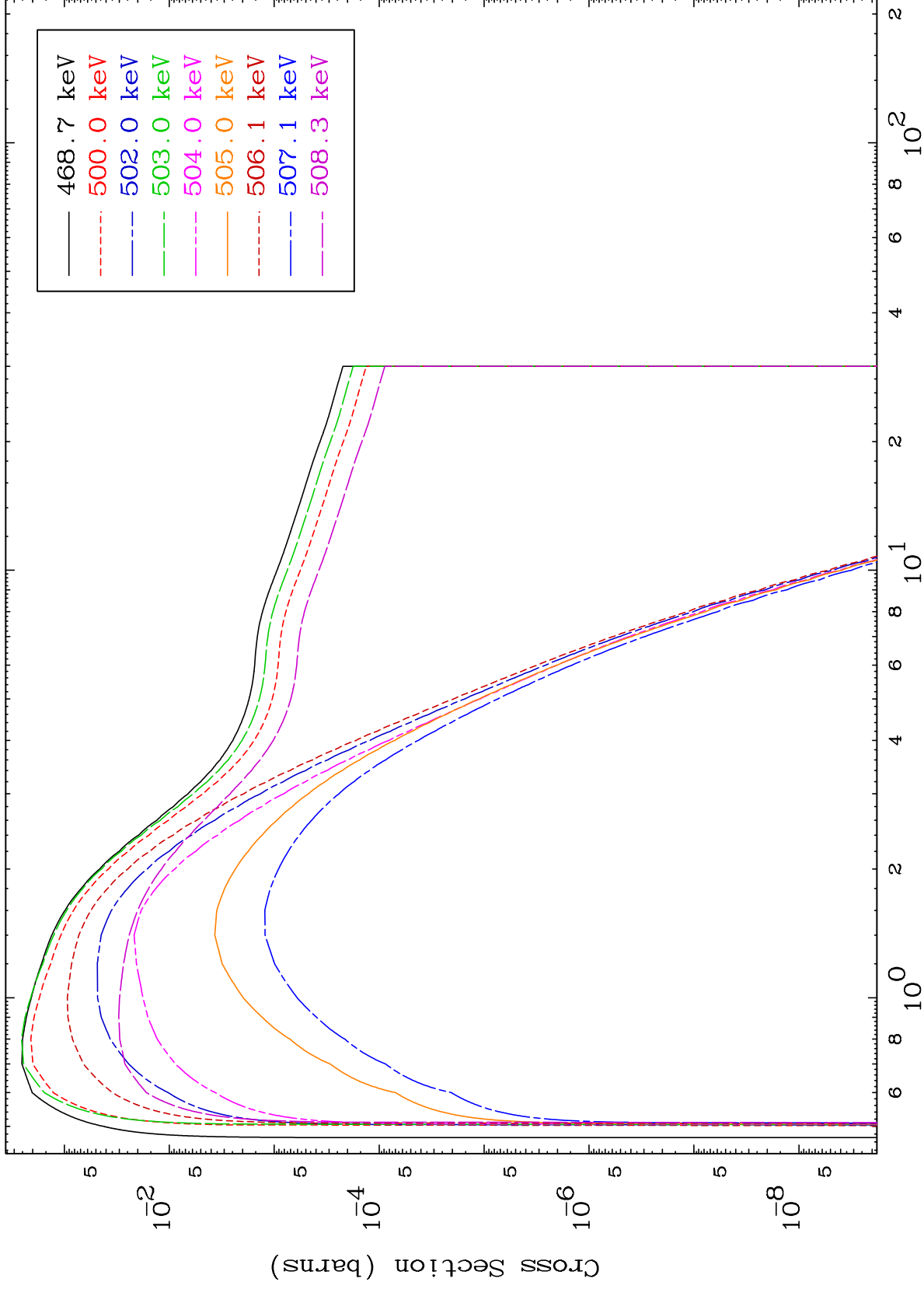
Incident Energy (MeV)

99-Es-247

MAT 9907

(n,n') Level  
293 Kelvin Cross Sections

99-Es-247



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Incident Energy (MeV)

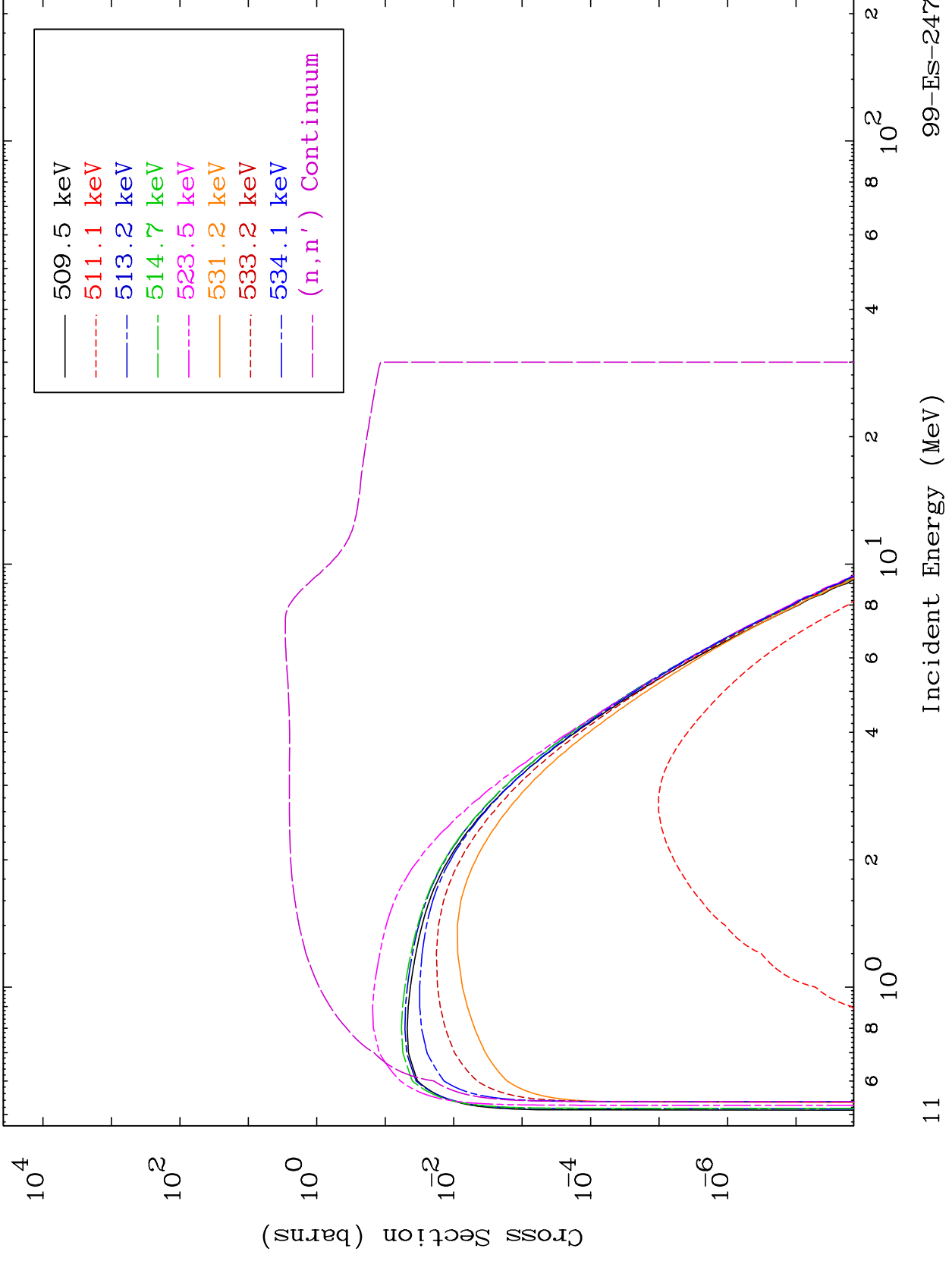
99-Es-247

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(n,n') Level

99-Es-247

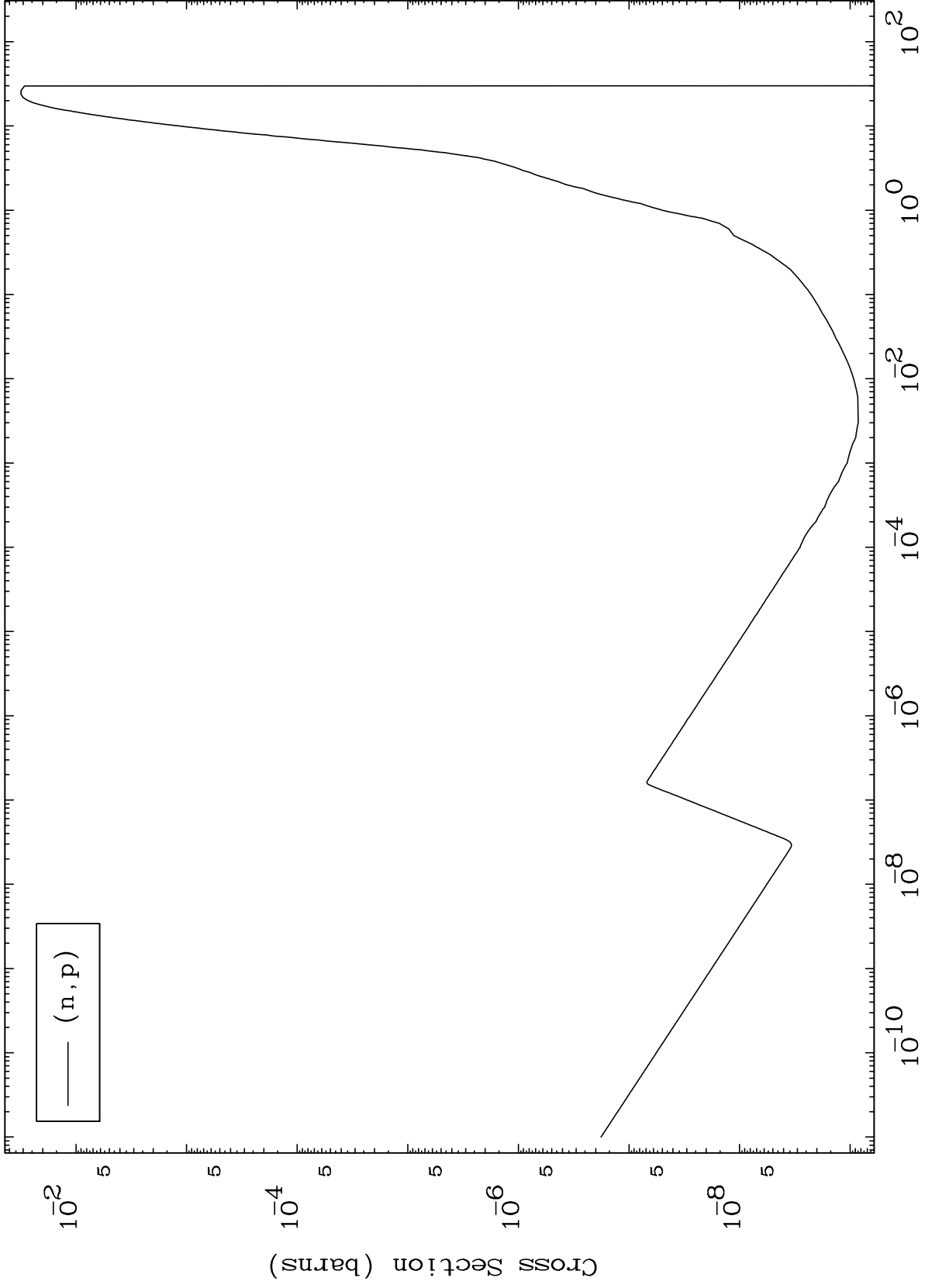
293 Kelvin Cross Sections



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(n,p) Levels  
293 Kelvin Cross Sections

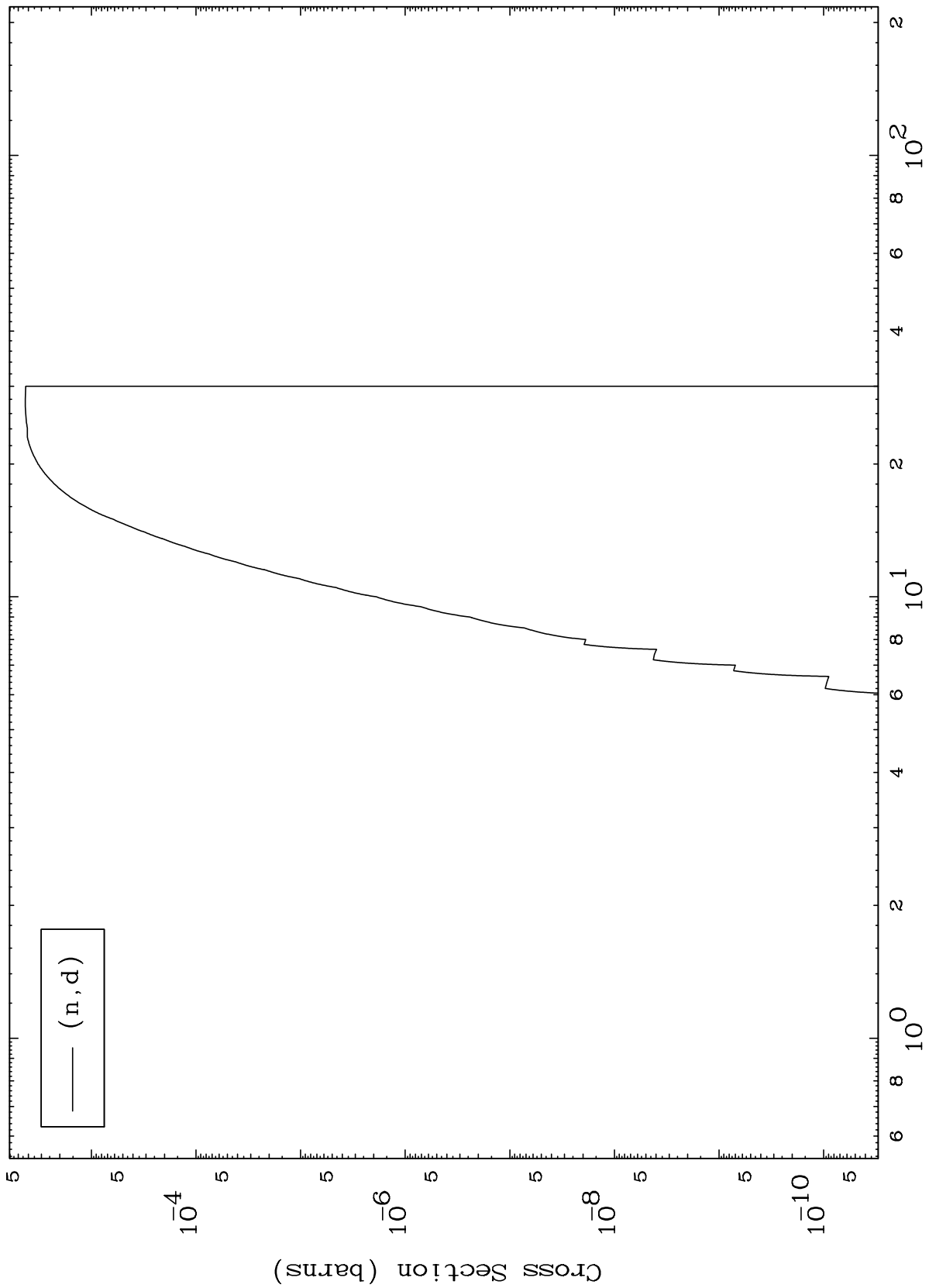
99-Es-247



MAT 9907

(n,d) Levels  
293 Kelvin Cross Sections

99-Es-247



13

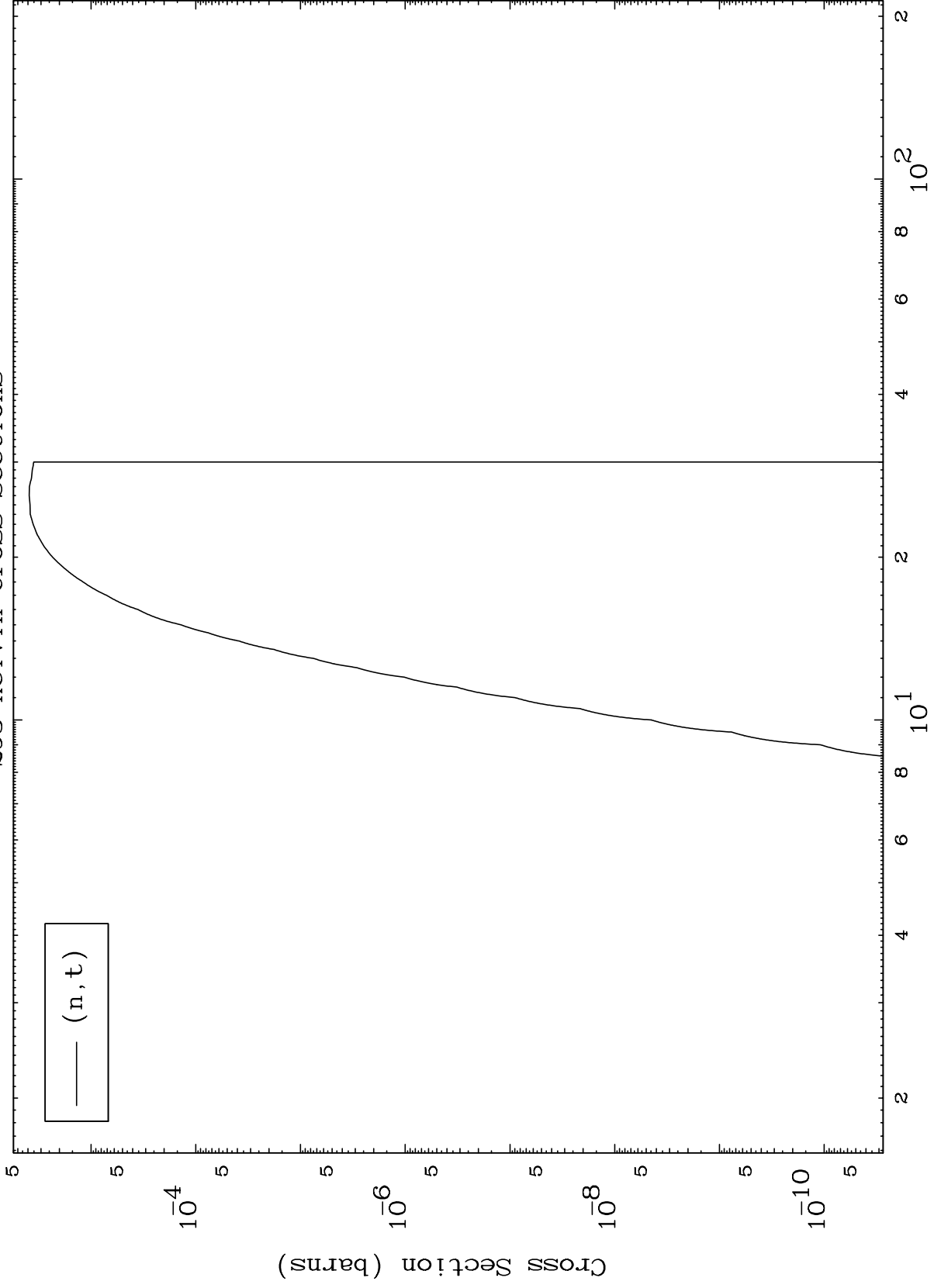
Incident Energy (MeV)

99-Es-247

MAT 9907

(n,t) Levels  
293 Kelvin Cross Sections

99-Es-247



(n,t)

14

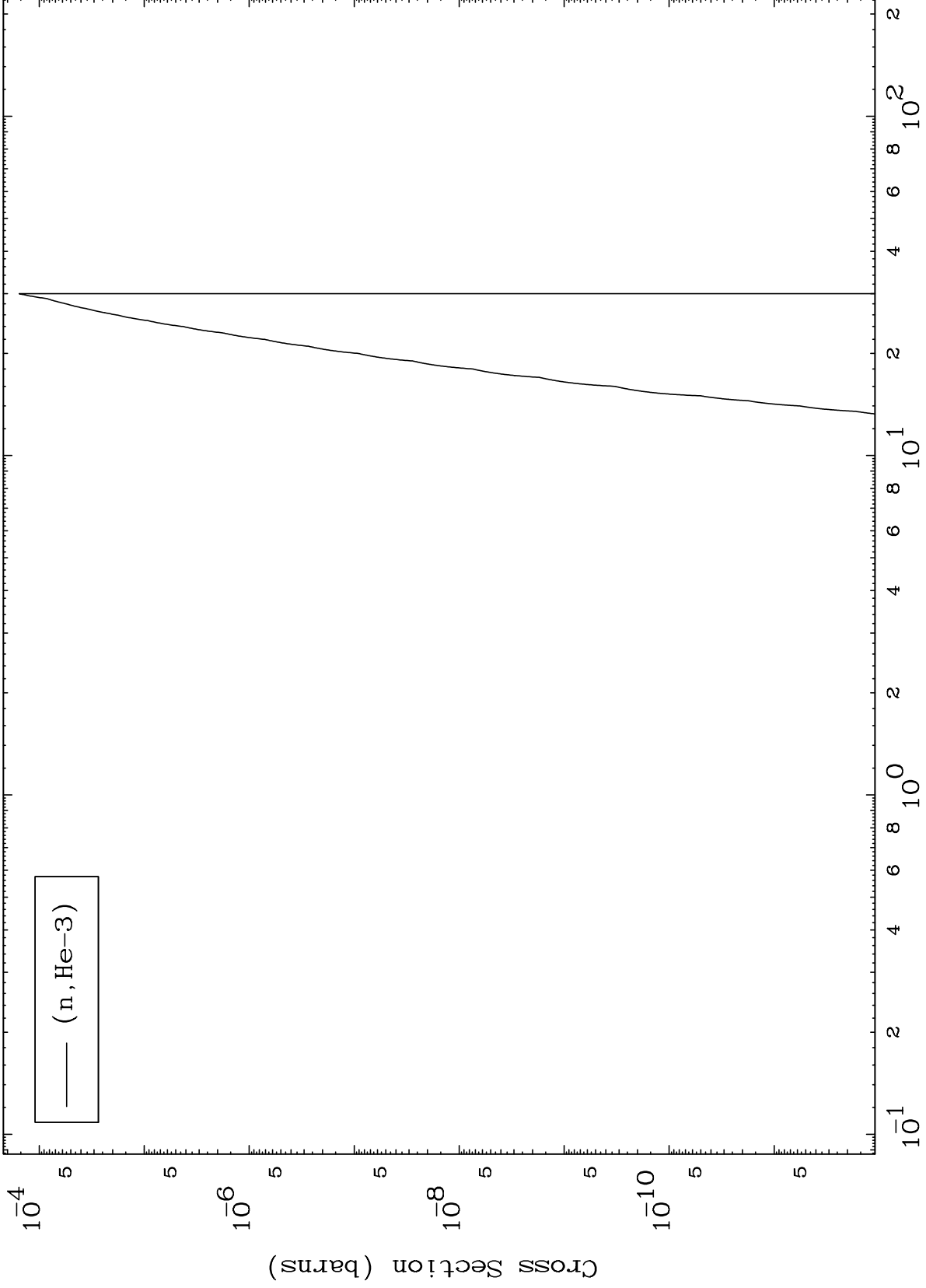
Incident Energy (MeV)

99-Es-247

MAT 9907

(n,He3) Levels  
293 Kelvin Cross Sections

99-Es-247



Incident Energy (MeV)

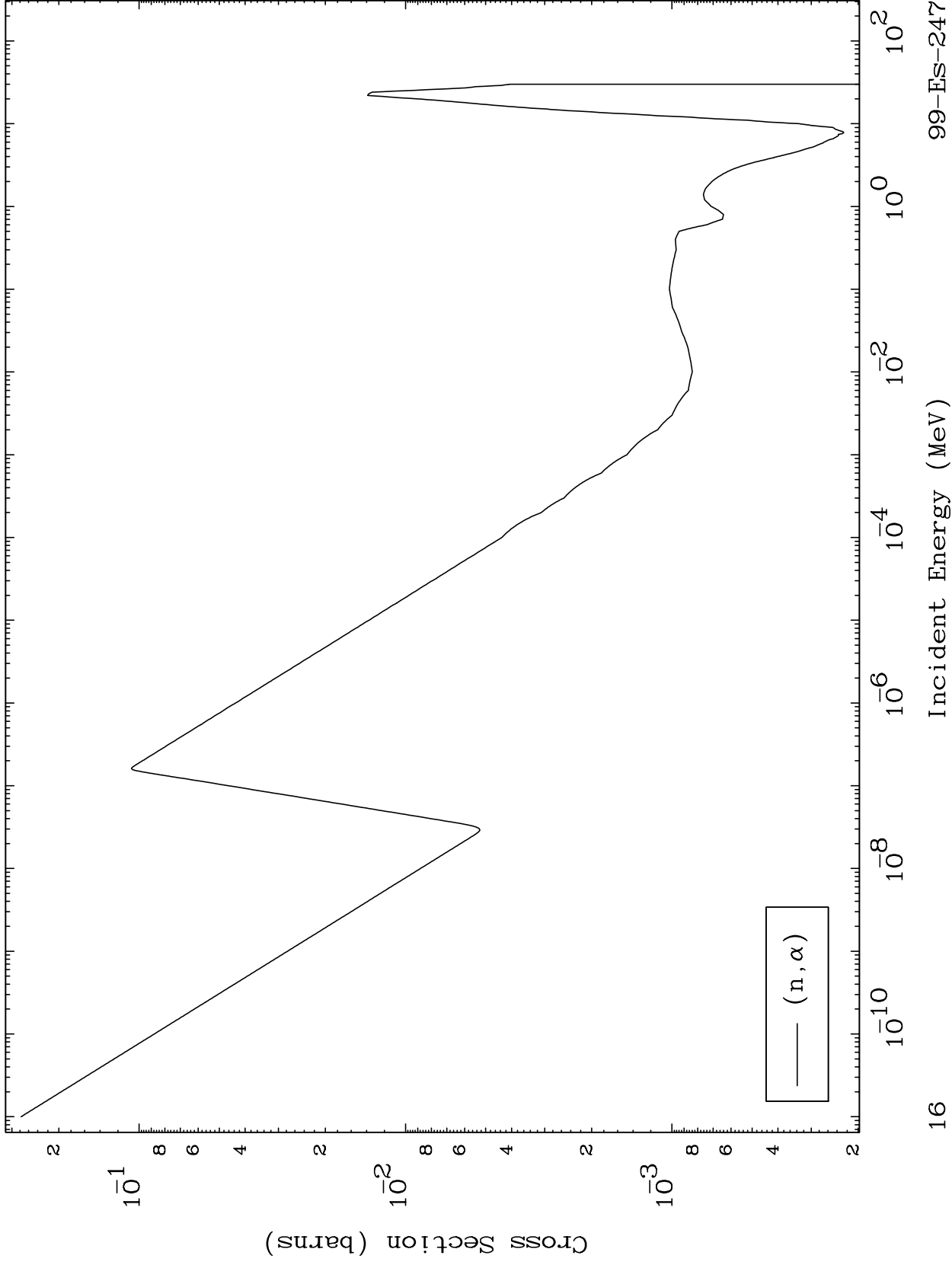
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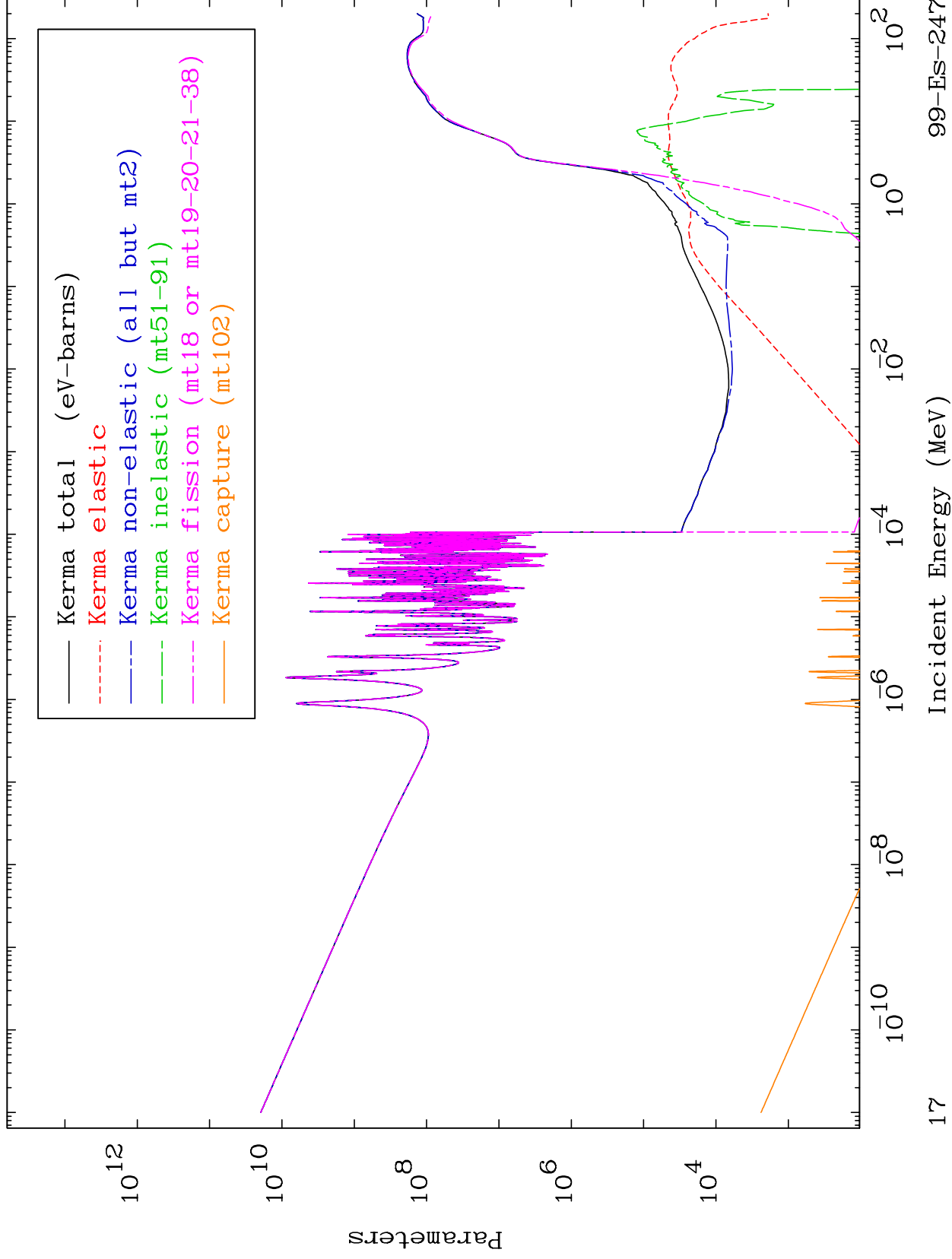


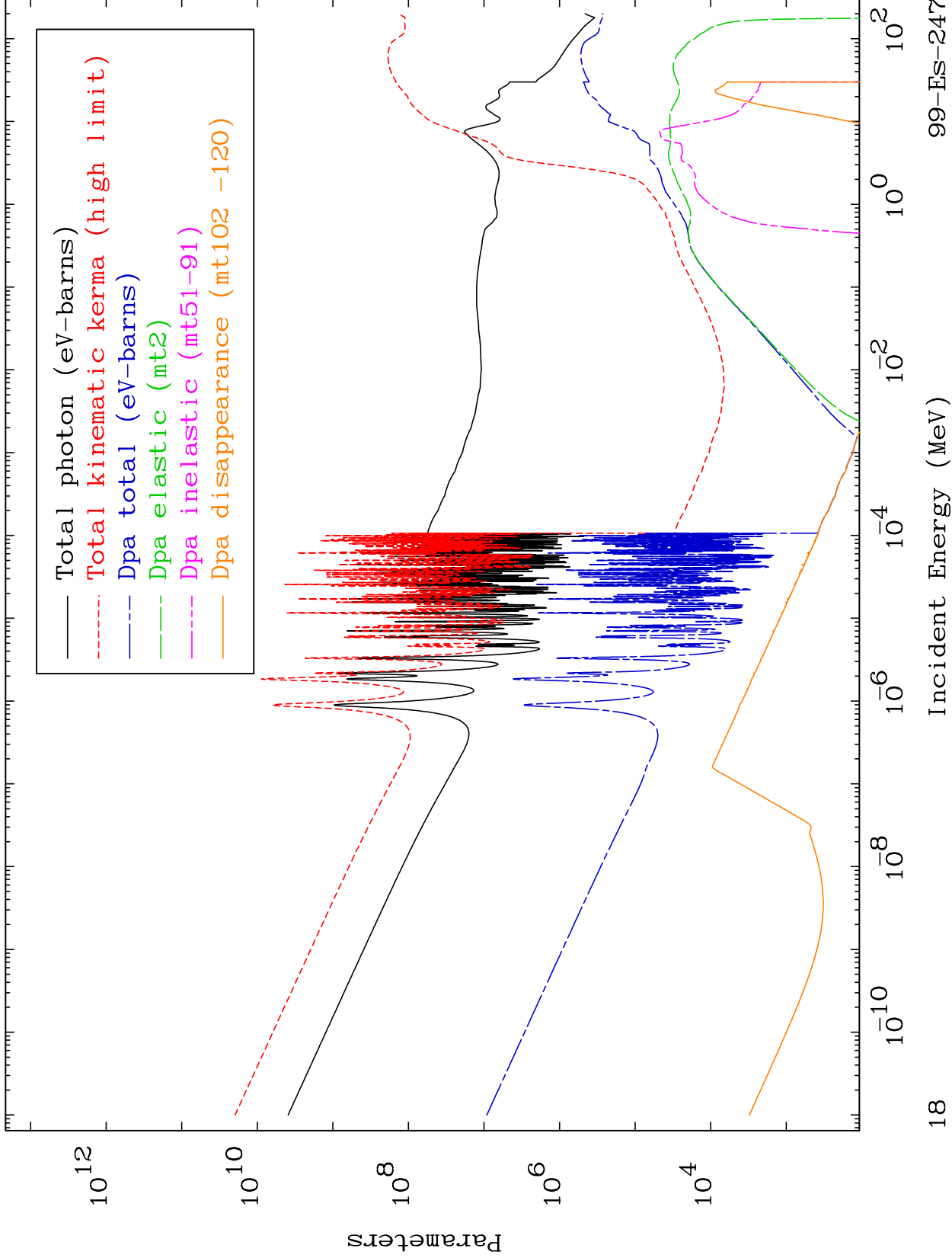
MAT 9907

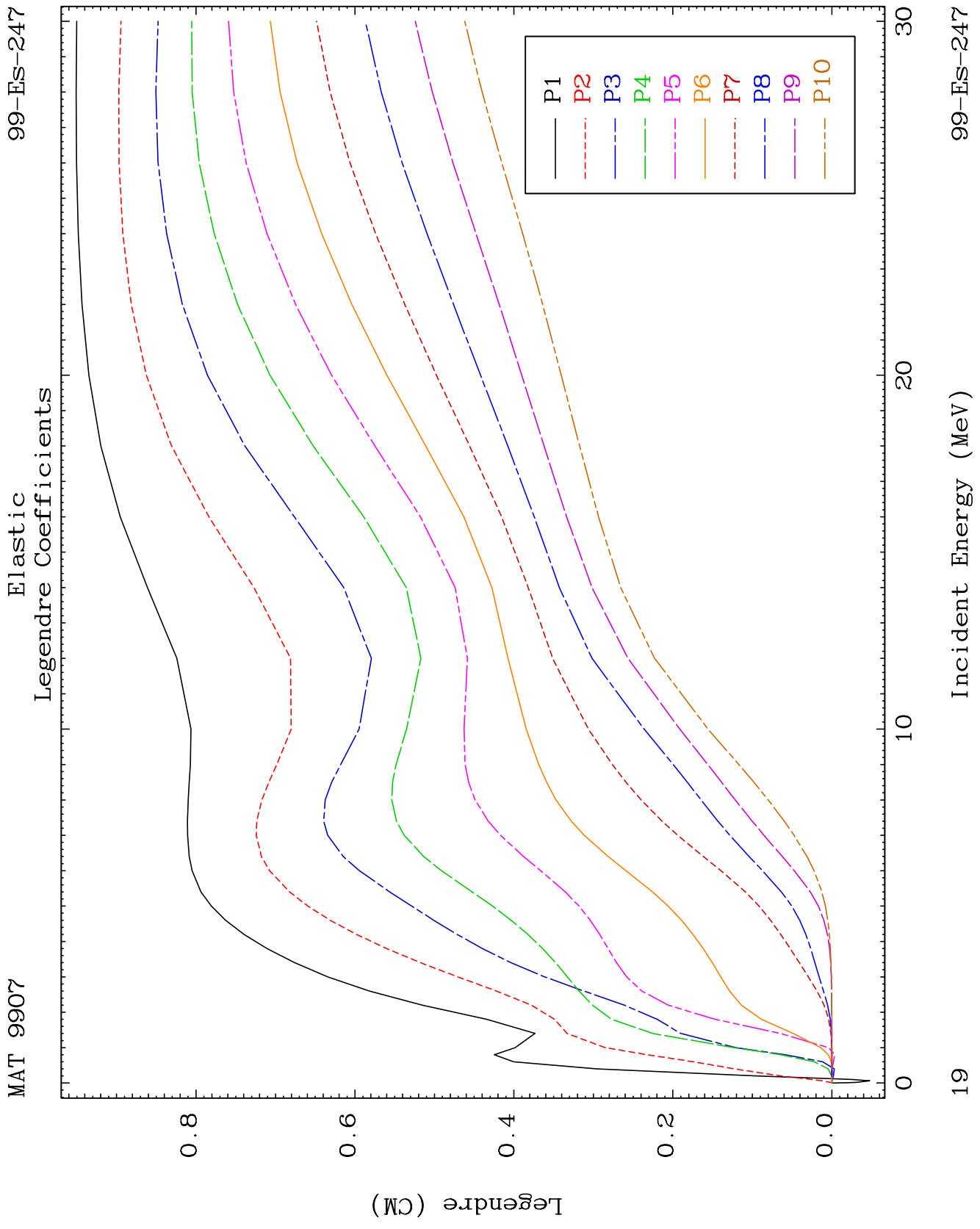
(n,α) Levels  
293 Kelvin Cross Sections

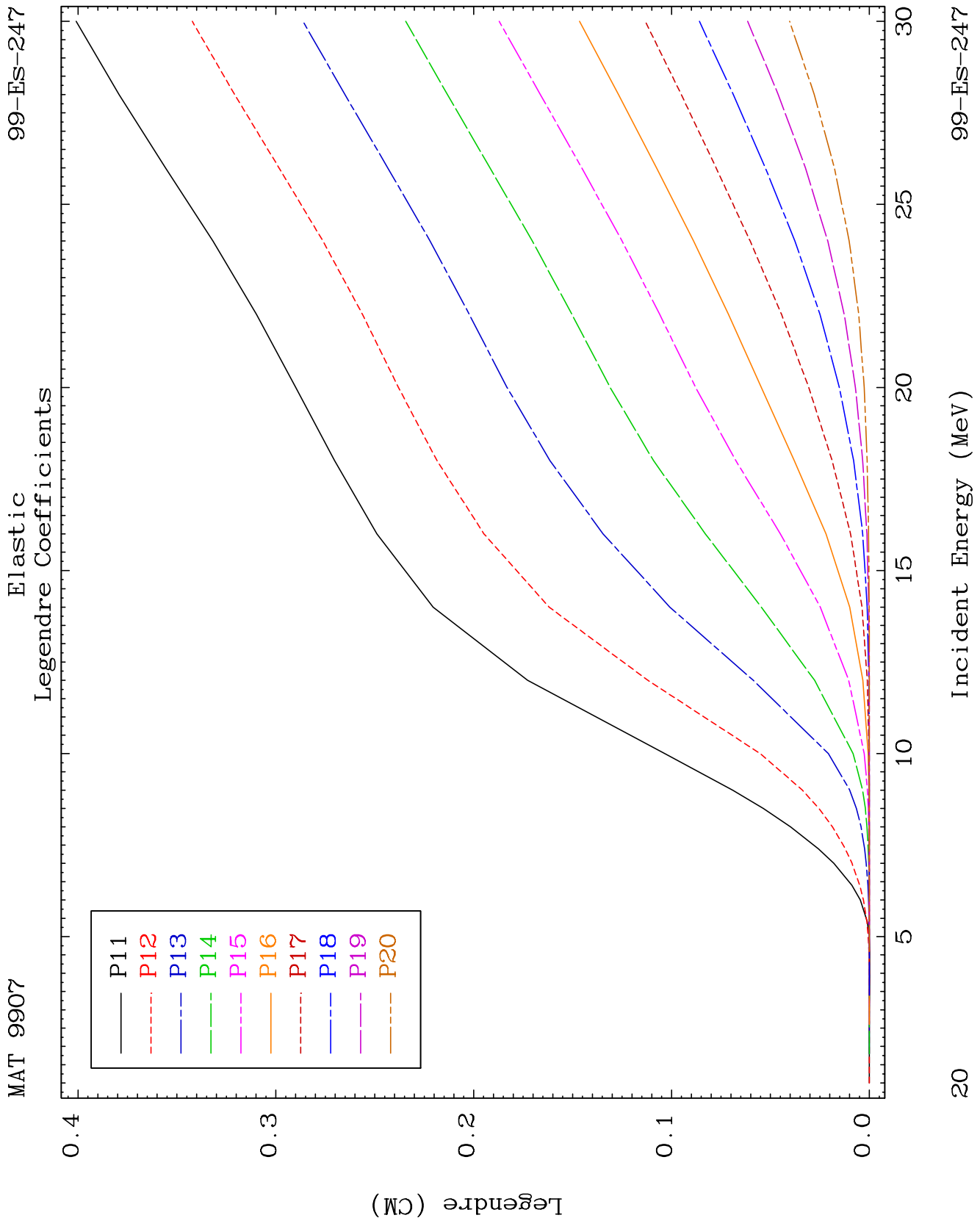
99-Es-247







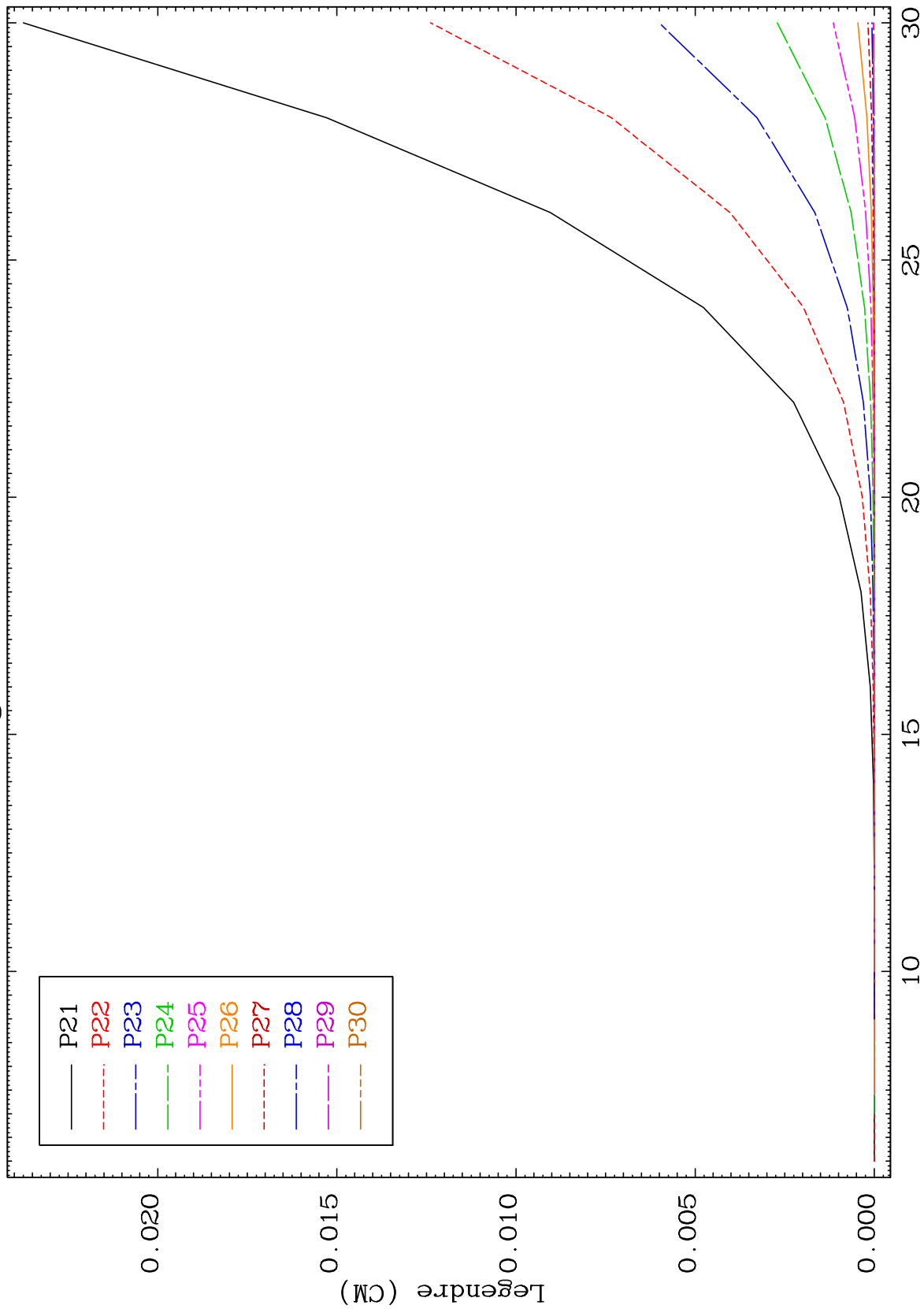




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Elastic Legendre Coefficients

99-Es-247



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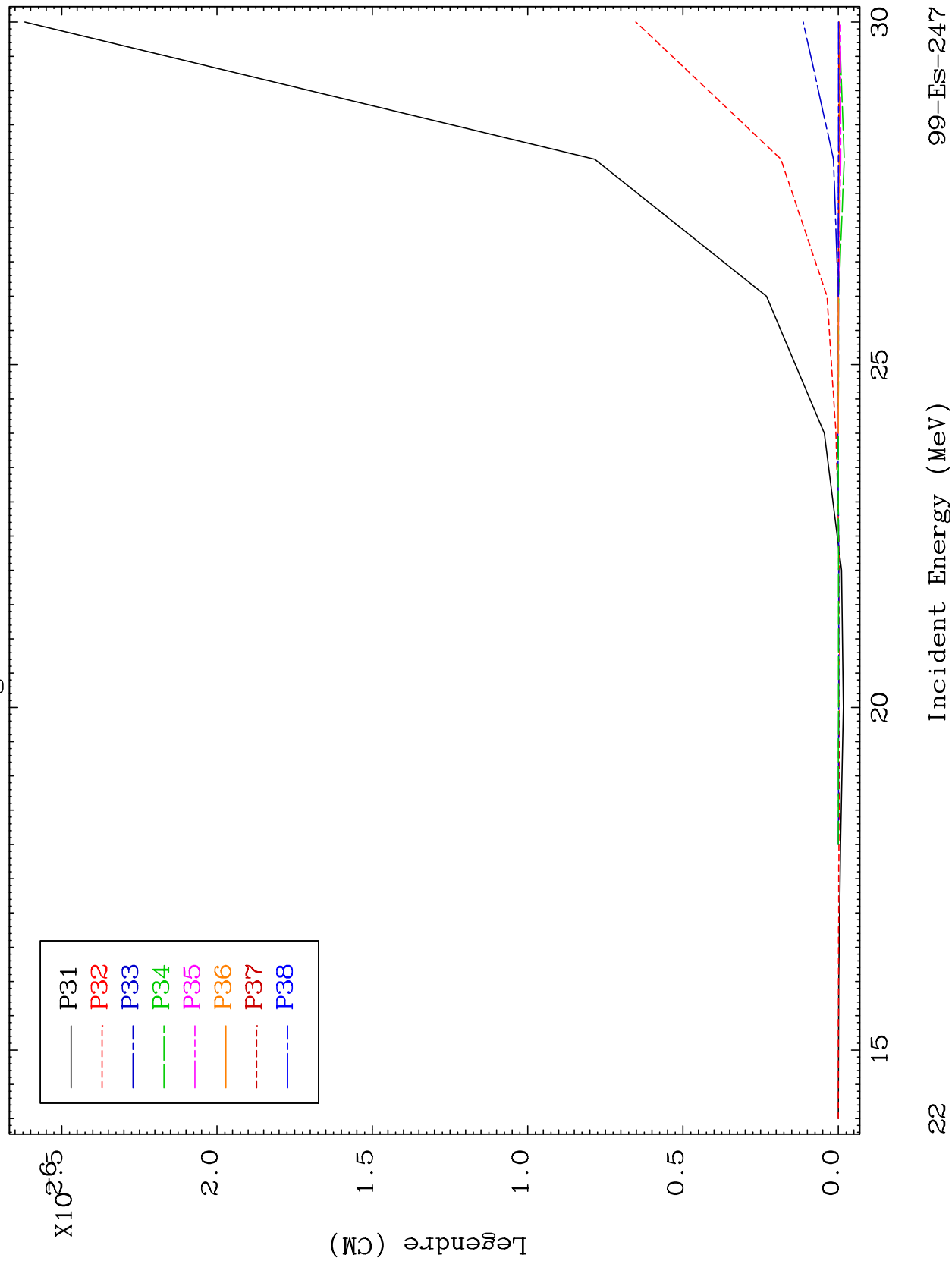
Incident Energy (MeV)

99-Es-247

MAT 9907

Elastic Legendre Coefficients

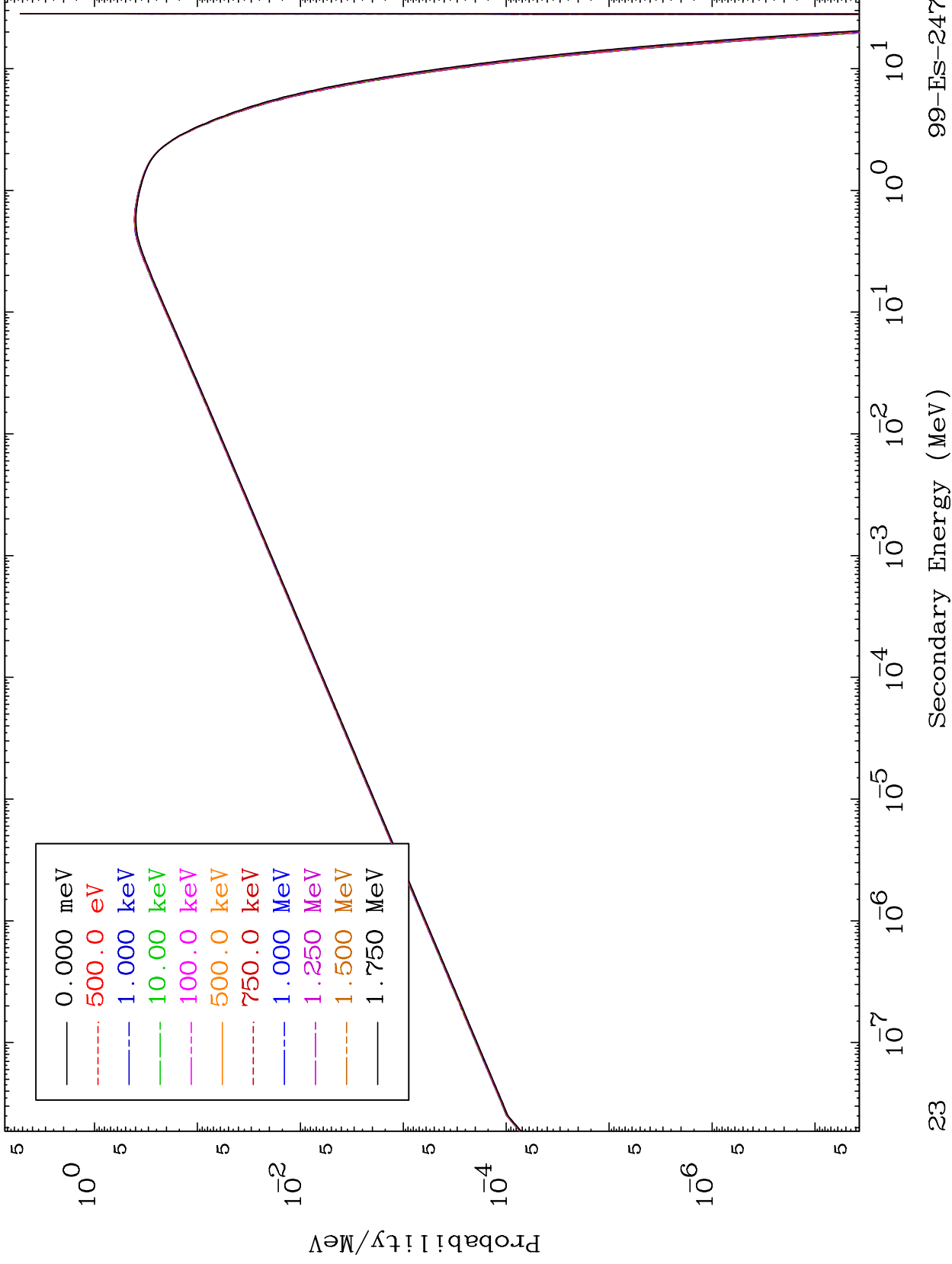
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22

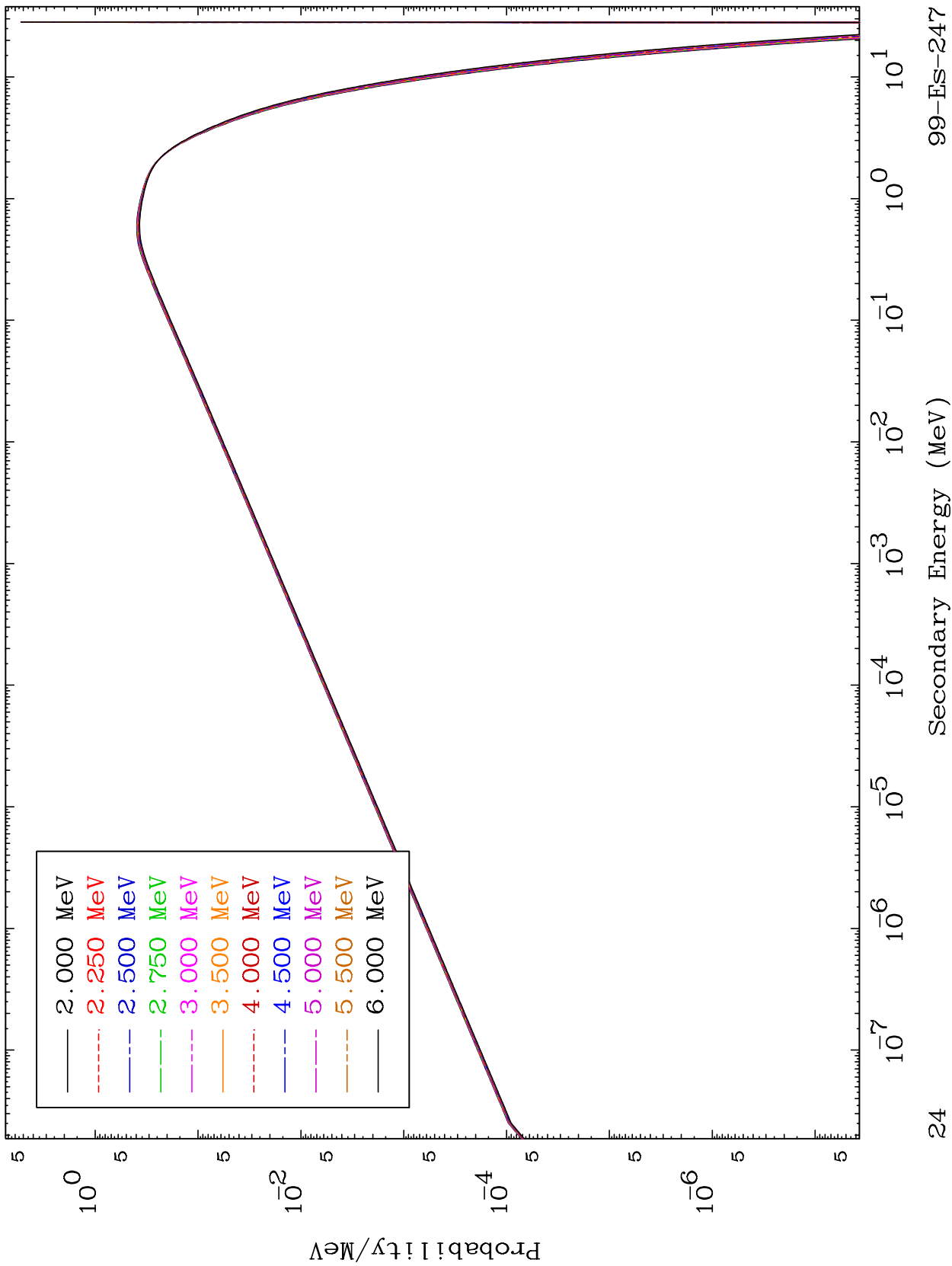
99-Es-247

Fission  
Energy Distribution

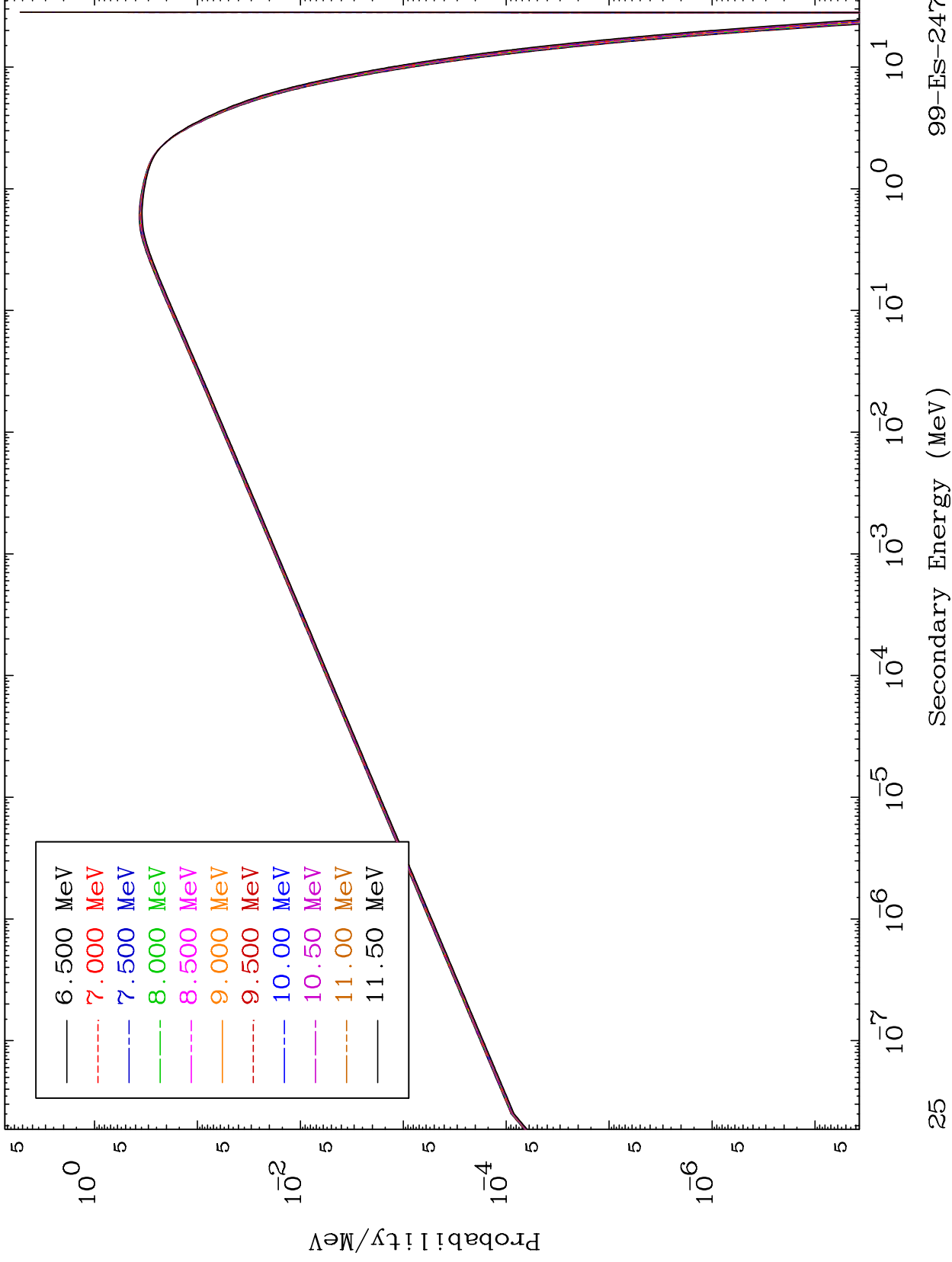




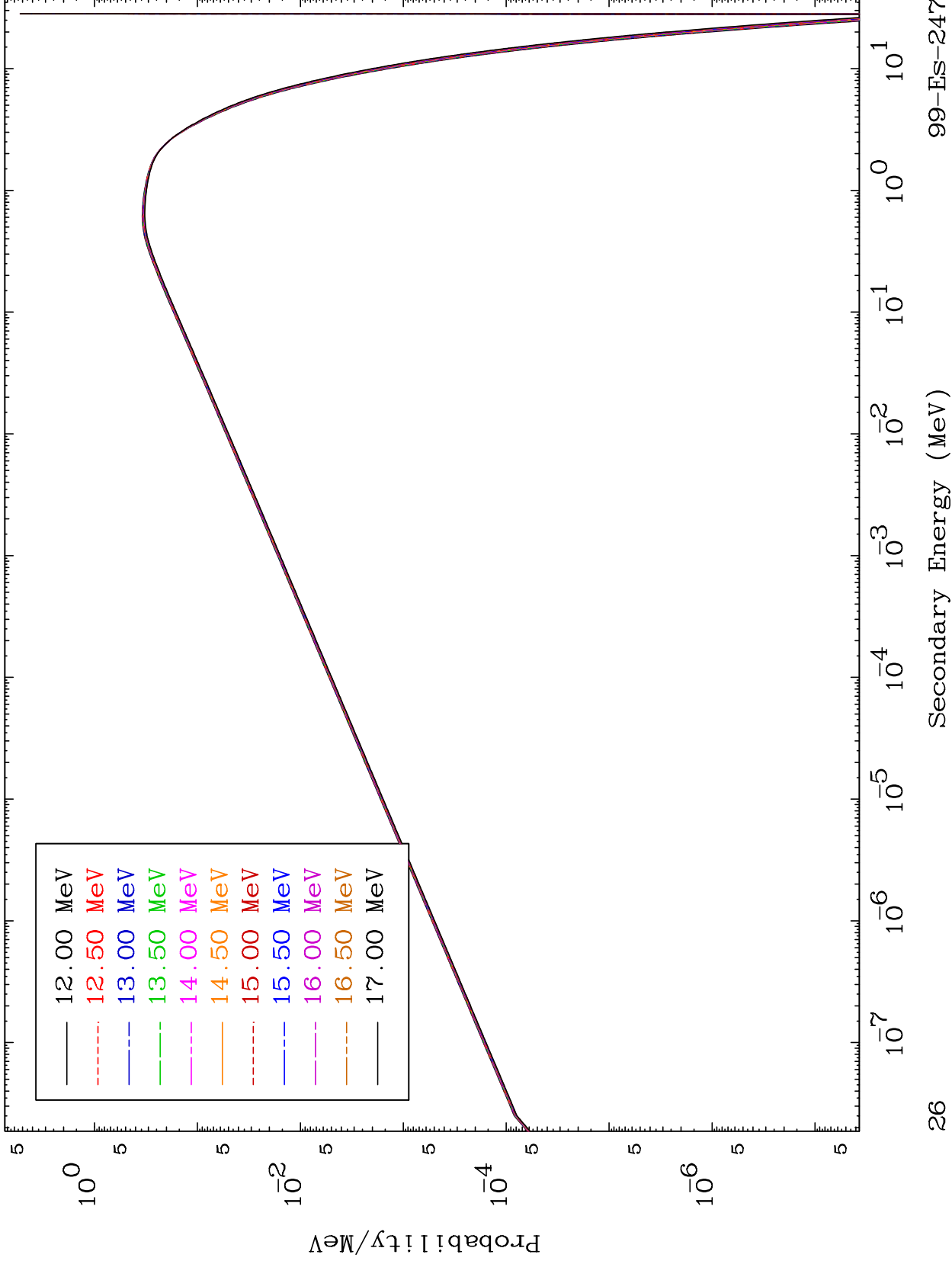
Fission  
Energy Distribution



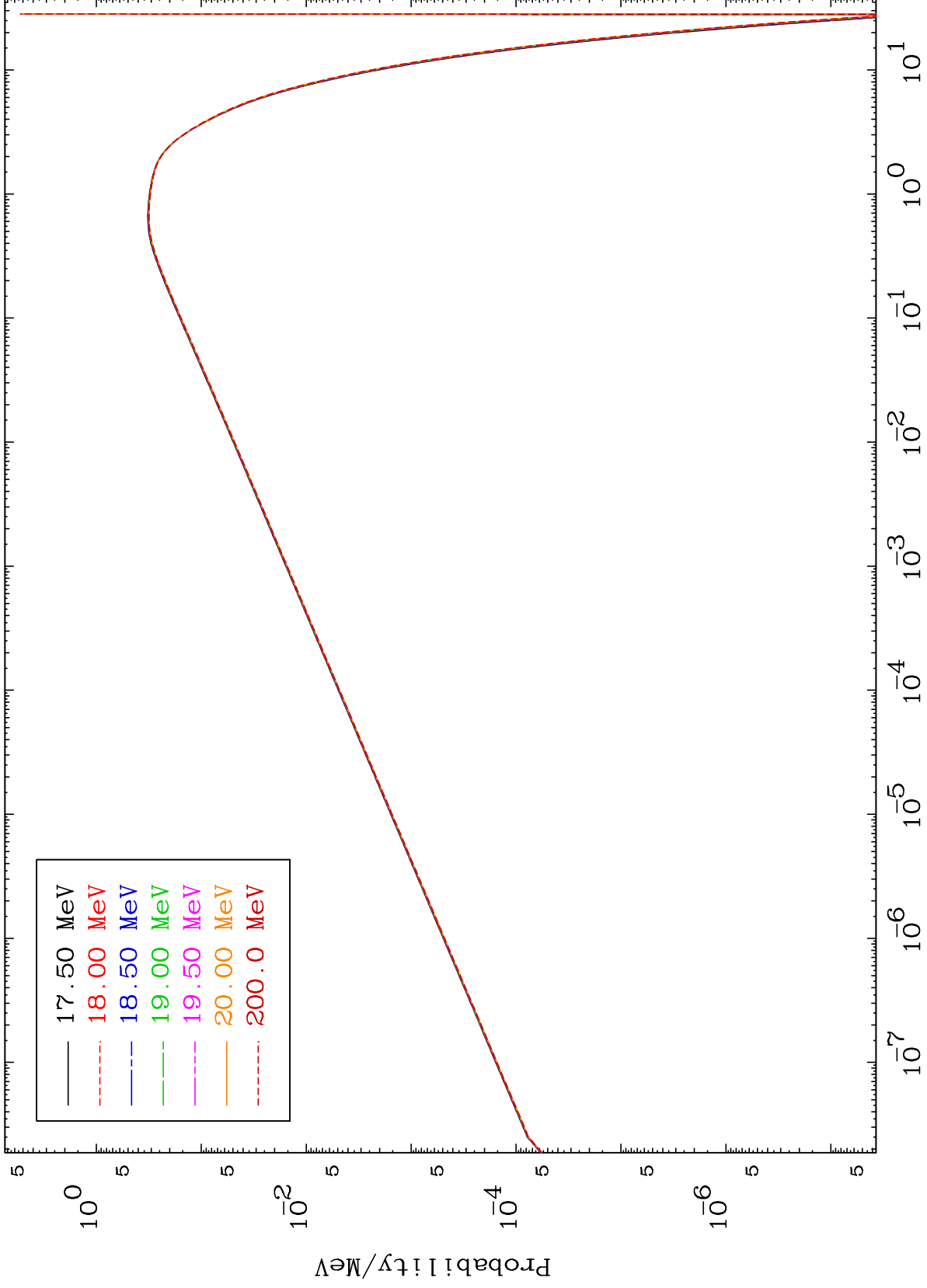
Fission  
Energy Distribution



Fission  
Energy Distribution



Fission  
Energy Distribution

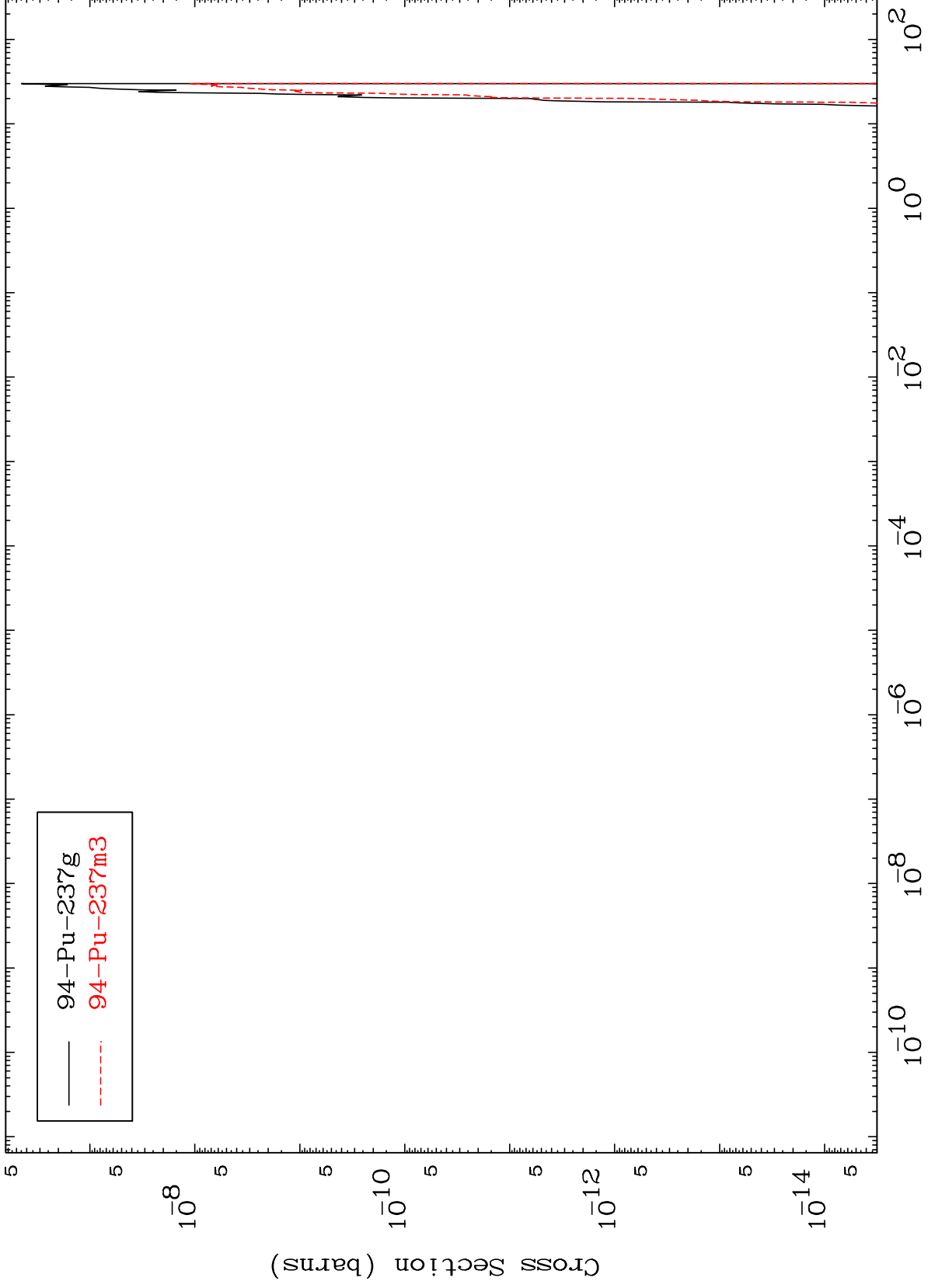


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(n,n') d,2α

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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MAT 9907

(n,t) 2 $\alpha$

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Radionuclide Production Cross Section

