

Program EVALPLOT  
(Version 2015-2)

by

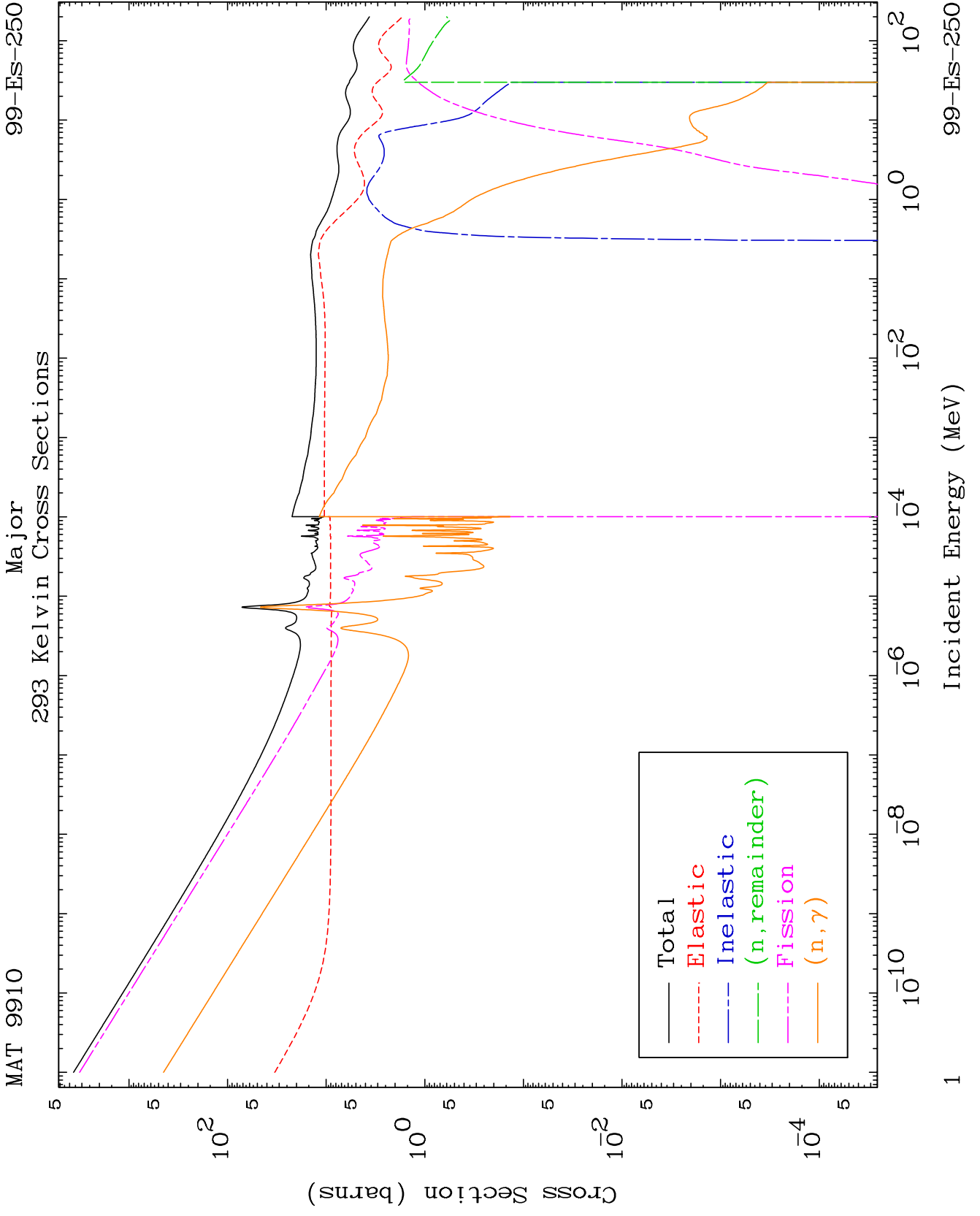
Dermott E. Cullen  
(Present Contact Information)

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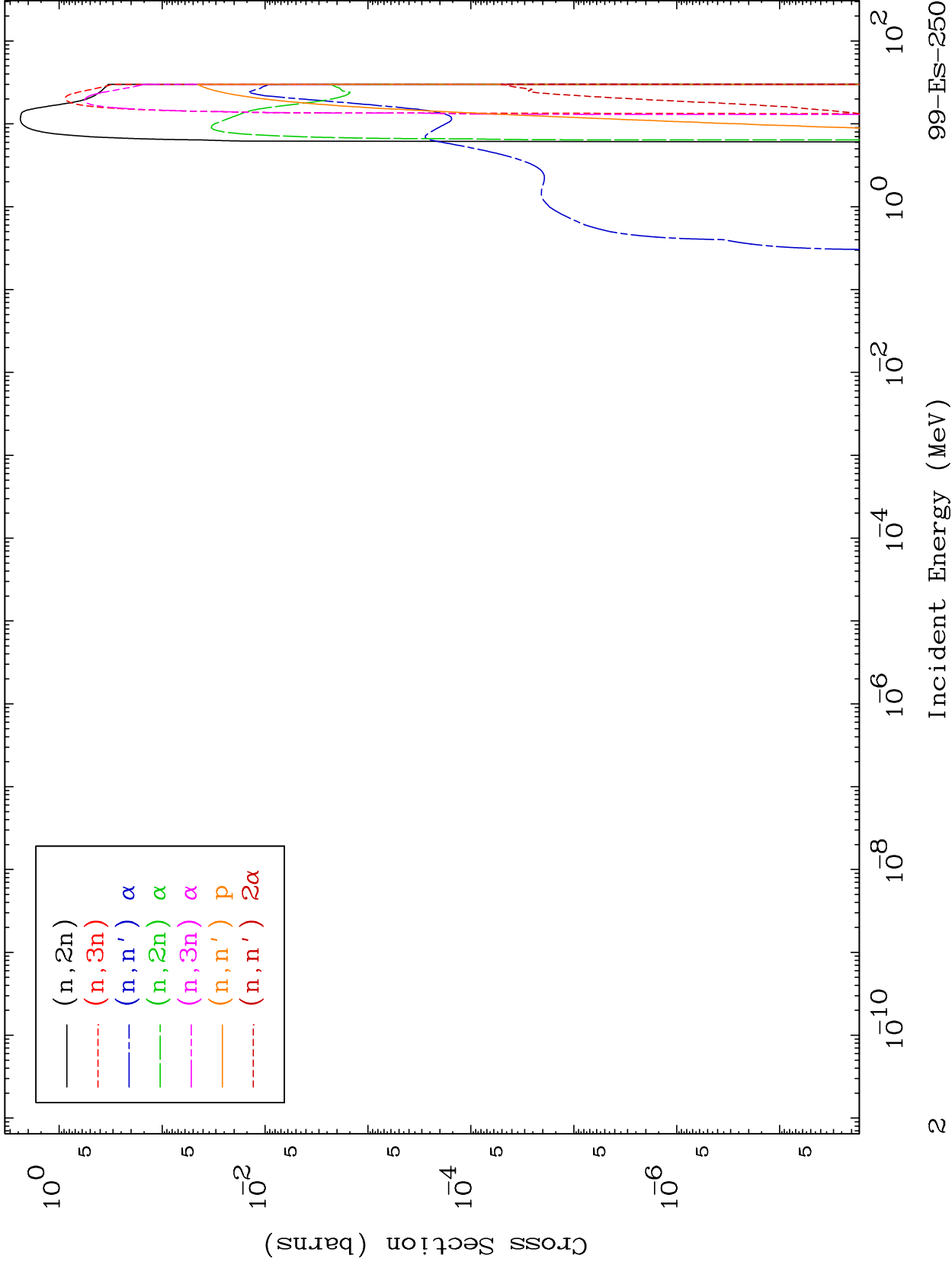
Press Mouse Button to Start



MAT 9910

Neutron Production  
293 Kelvin Cross Sections

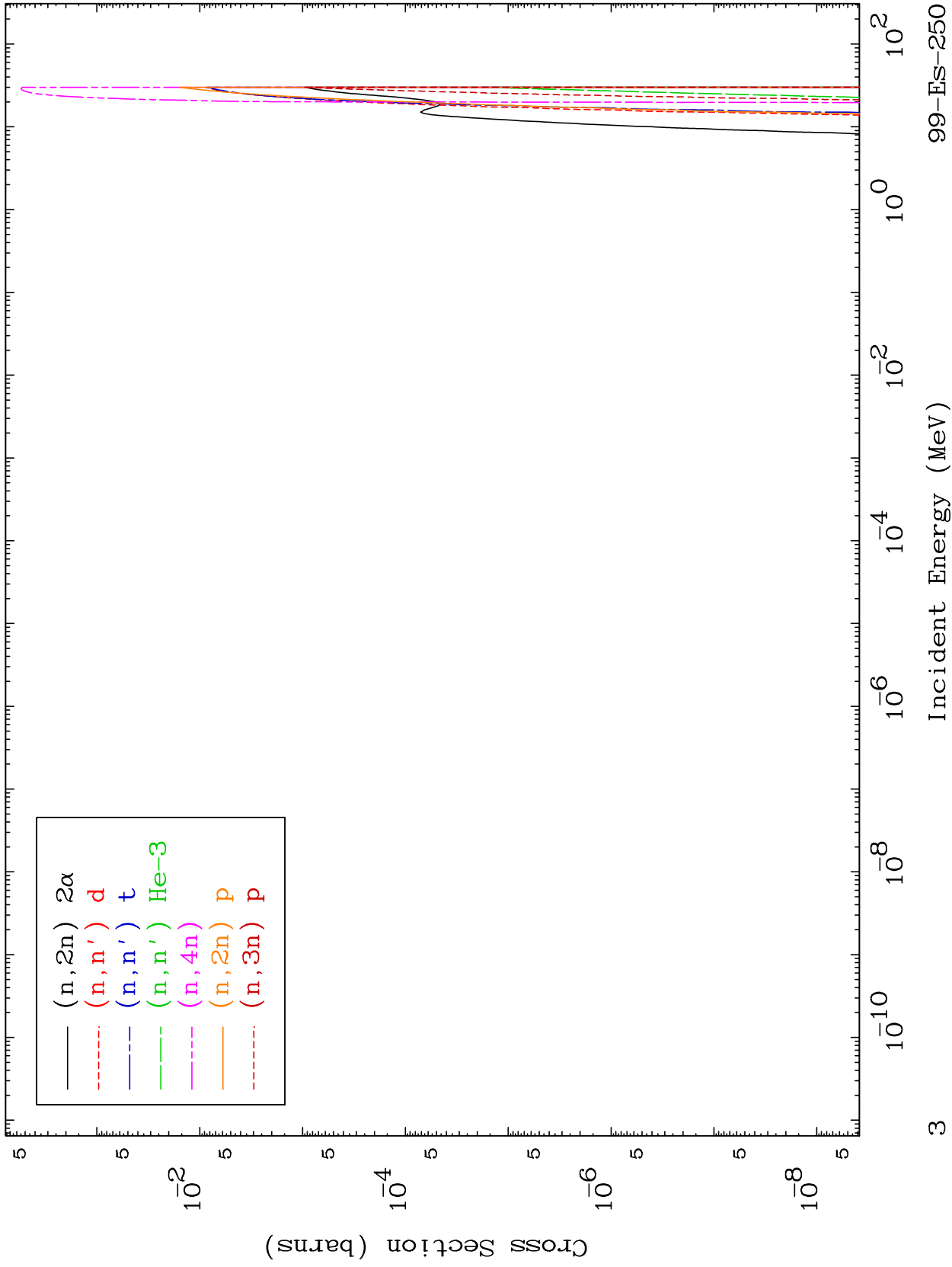
99-Es-250



MAT 9910

Neutron Production  
293 Kelvin Cross Sections

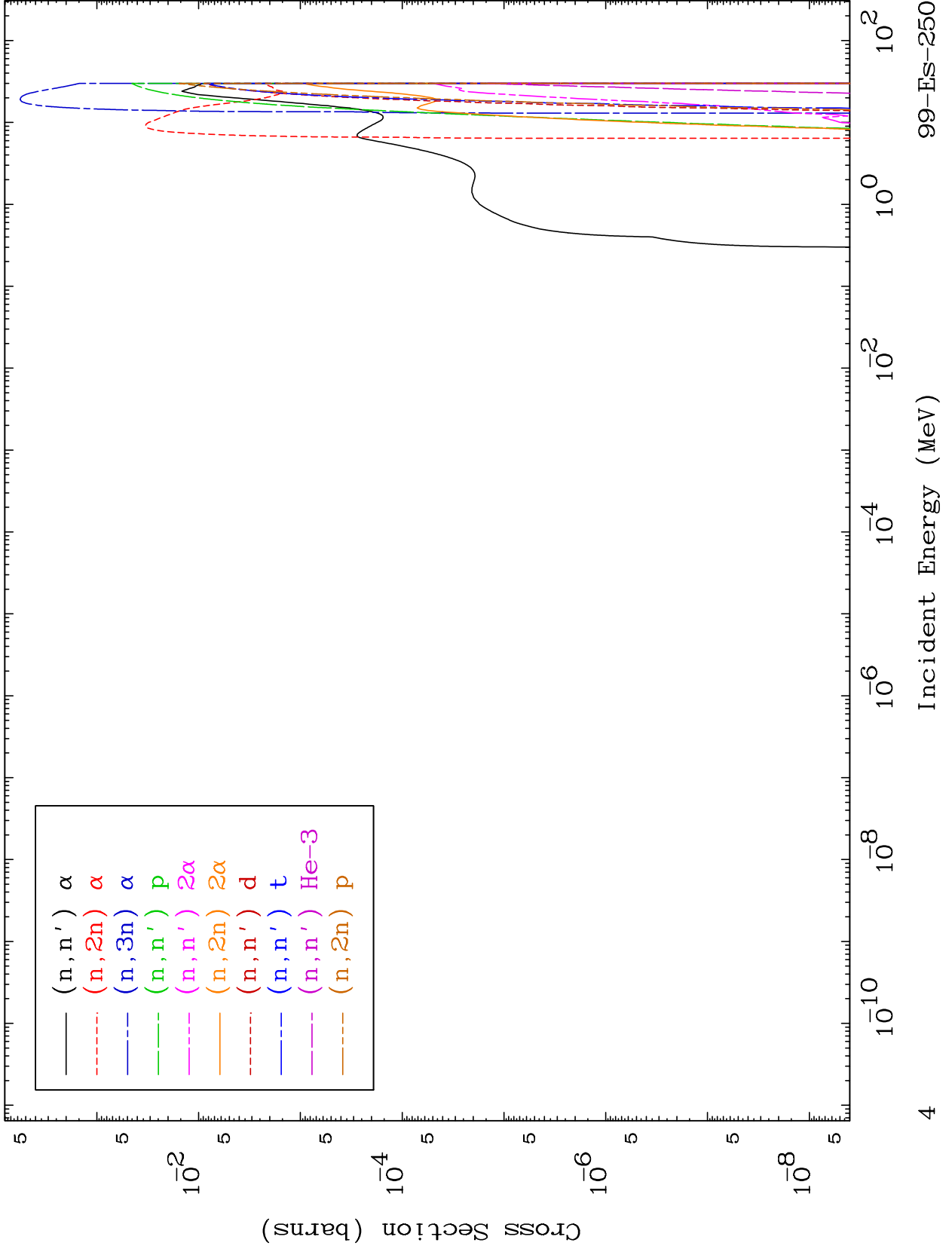
99-Es-250



MAT 9910

Charged Particle  
293 Kelvin Cross Sections

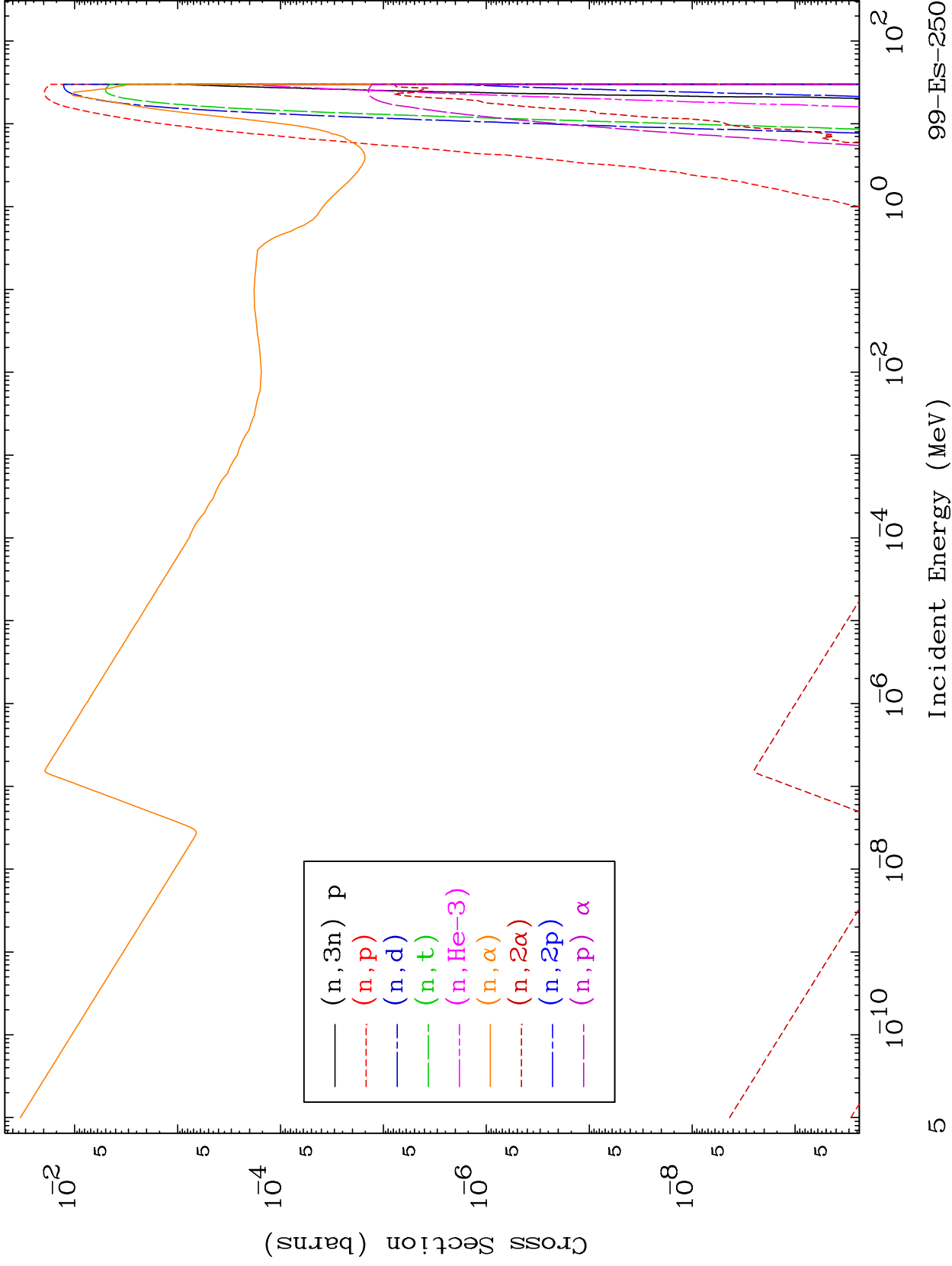
99-Es-250



MAT 9910

Charged Particle  
293 Kelvin Cross Sections

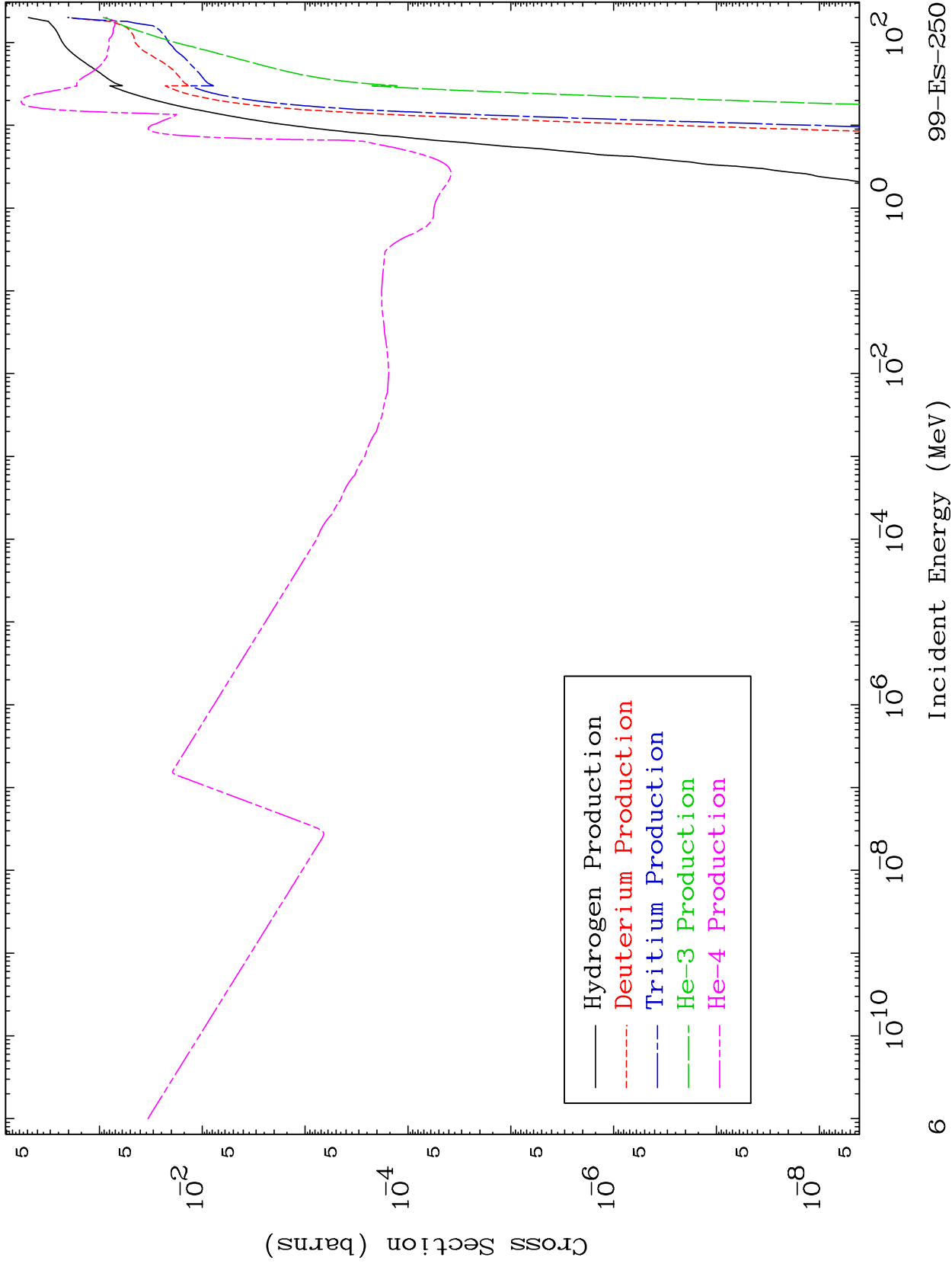
99-Es-250



MAT 9910

Particle Production  
293 Kelvin Cross Sections

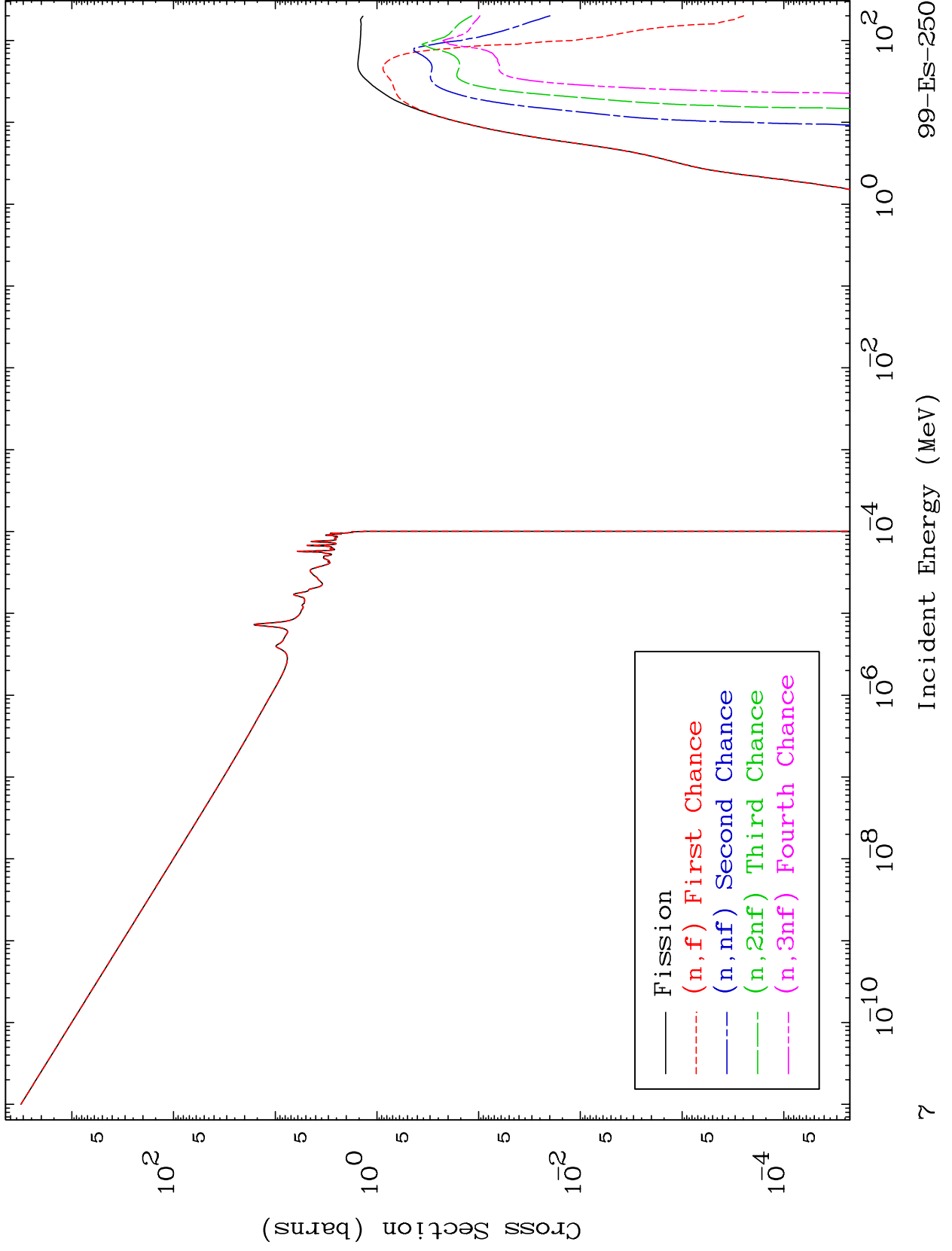
99-Es-250



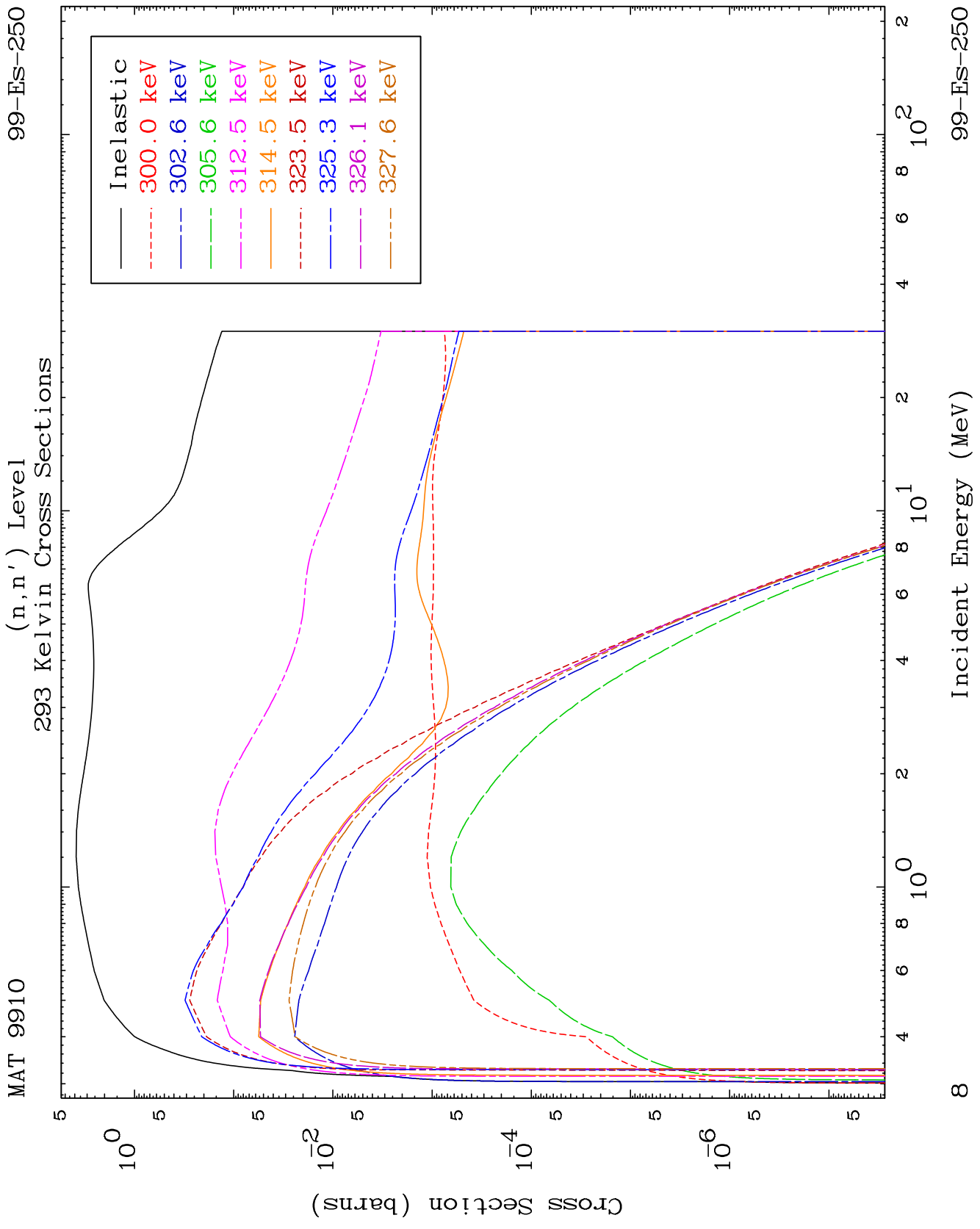
MAT 9910

Fission  
293 Kelvin Cross Sections

99-Es-250



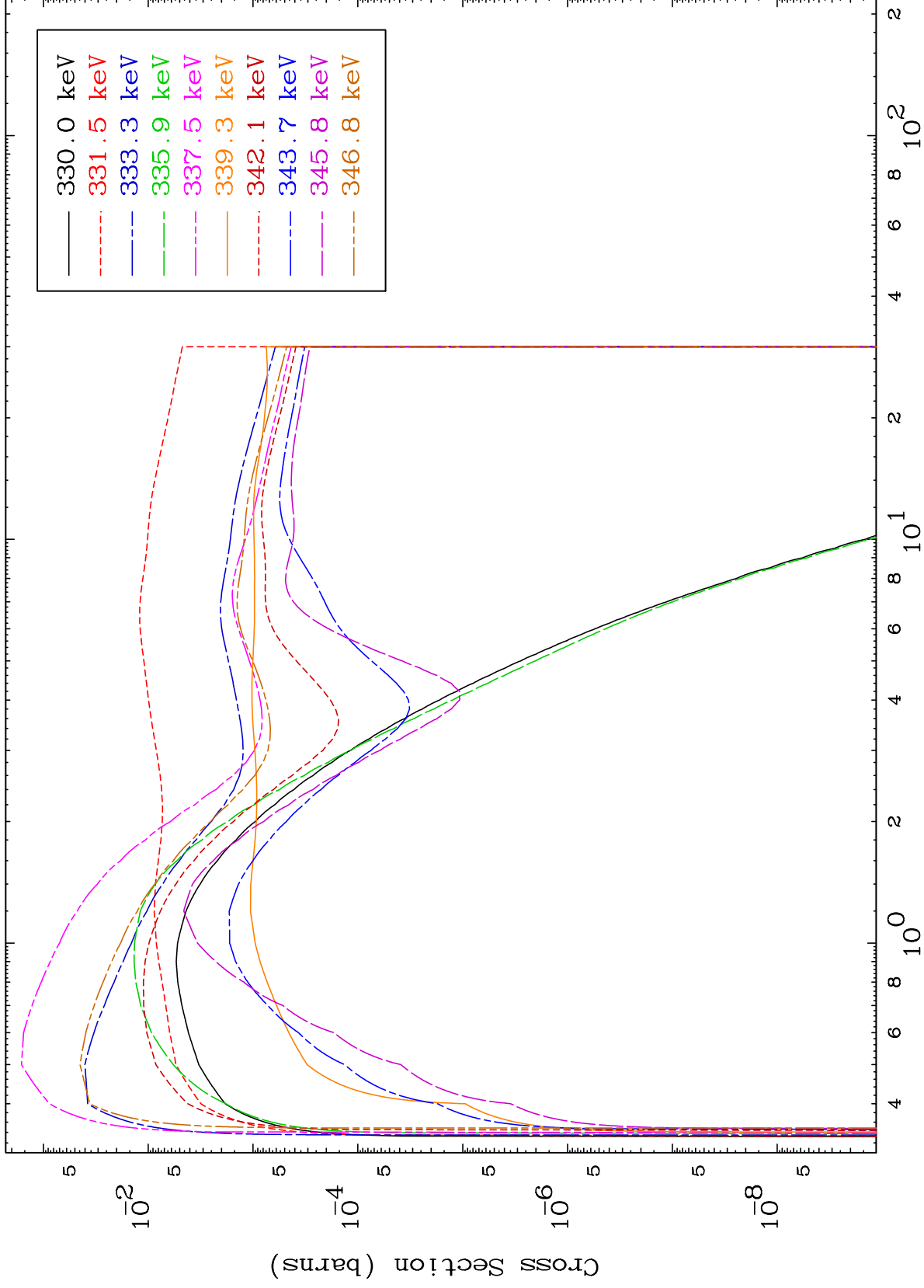




MAT 9910

(n,n') Level  
293 Kelvin Cross Sections

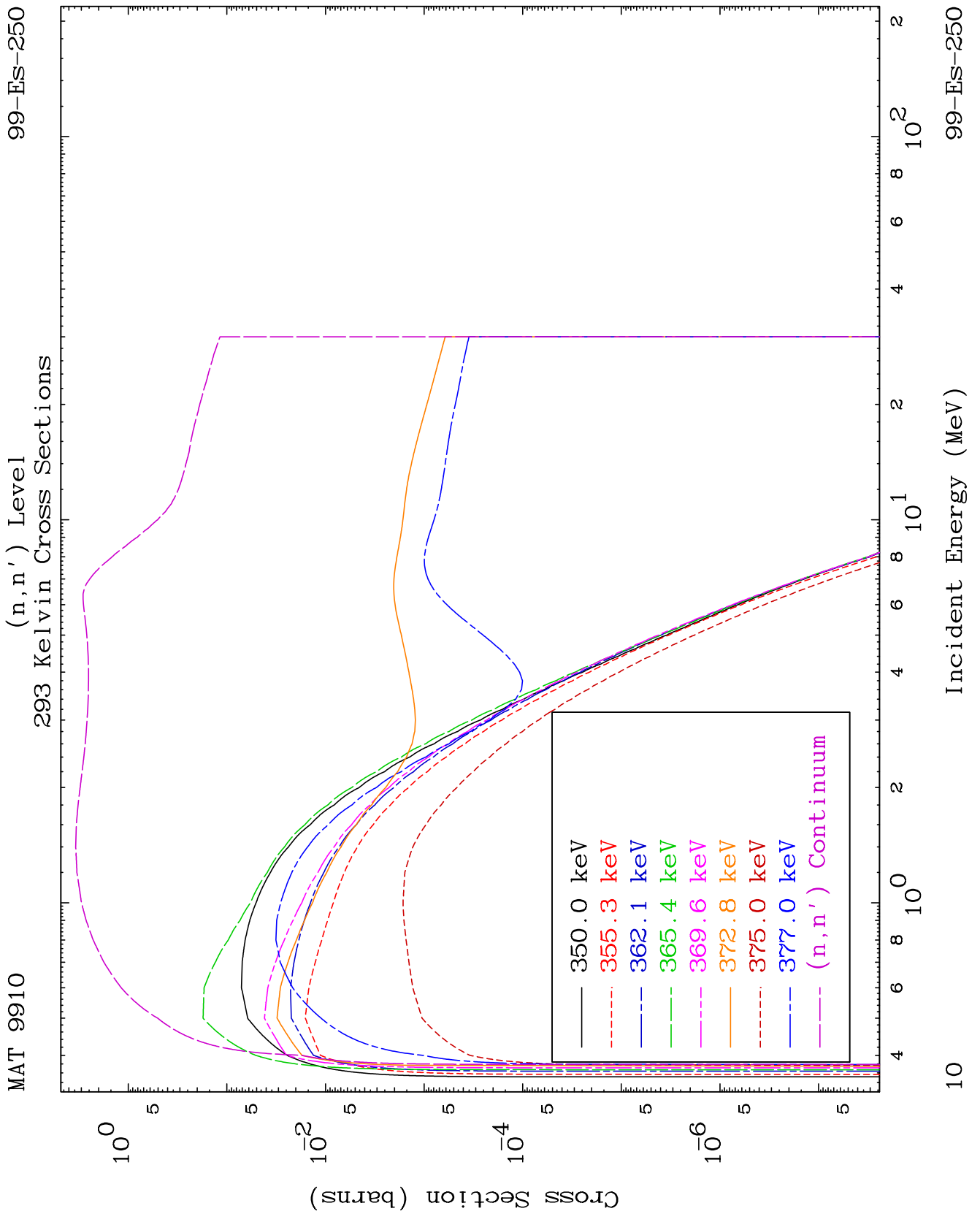
99-Es-250



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Incident Energy (MeV)

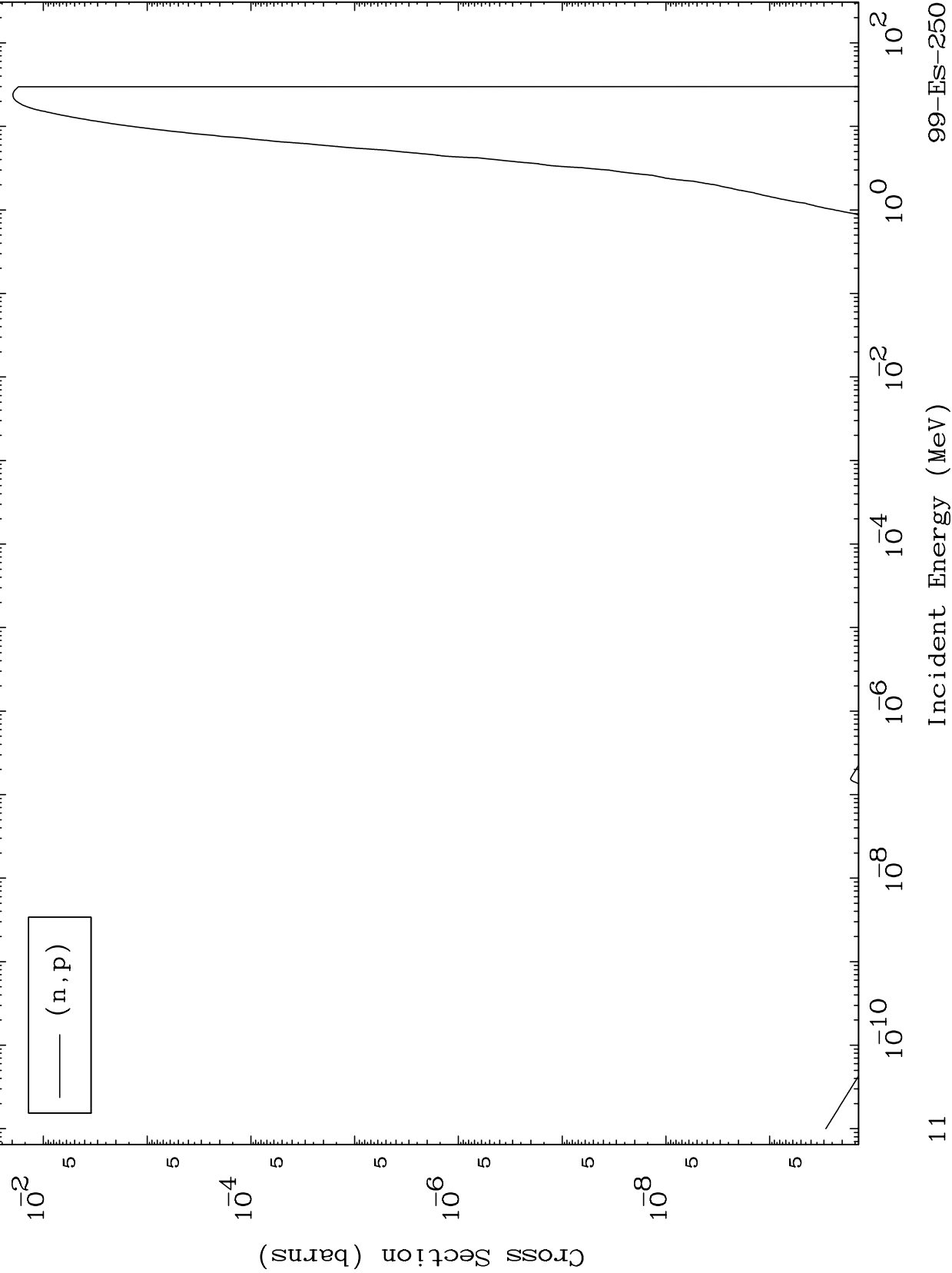
99-Es-250



MAT 9910

(n,p) Levels  
293 Kelvin Cross Sections

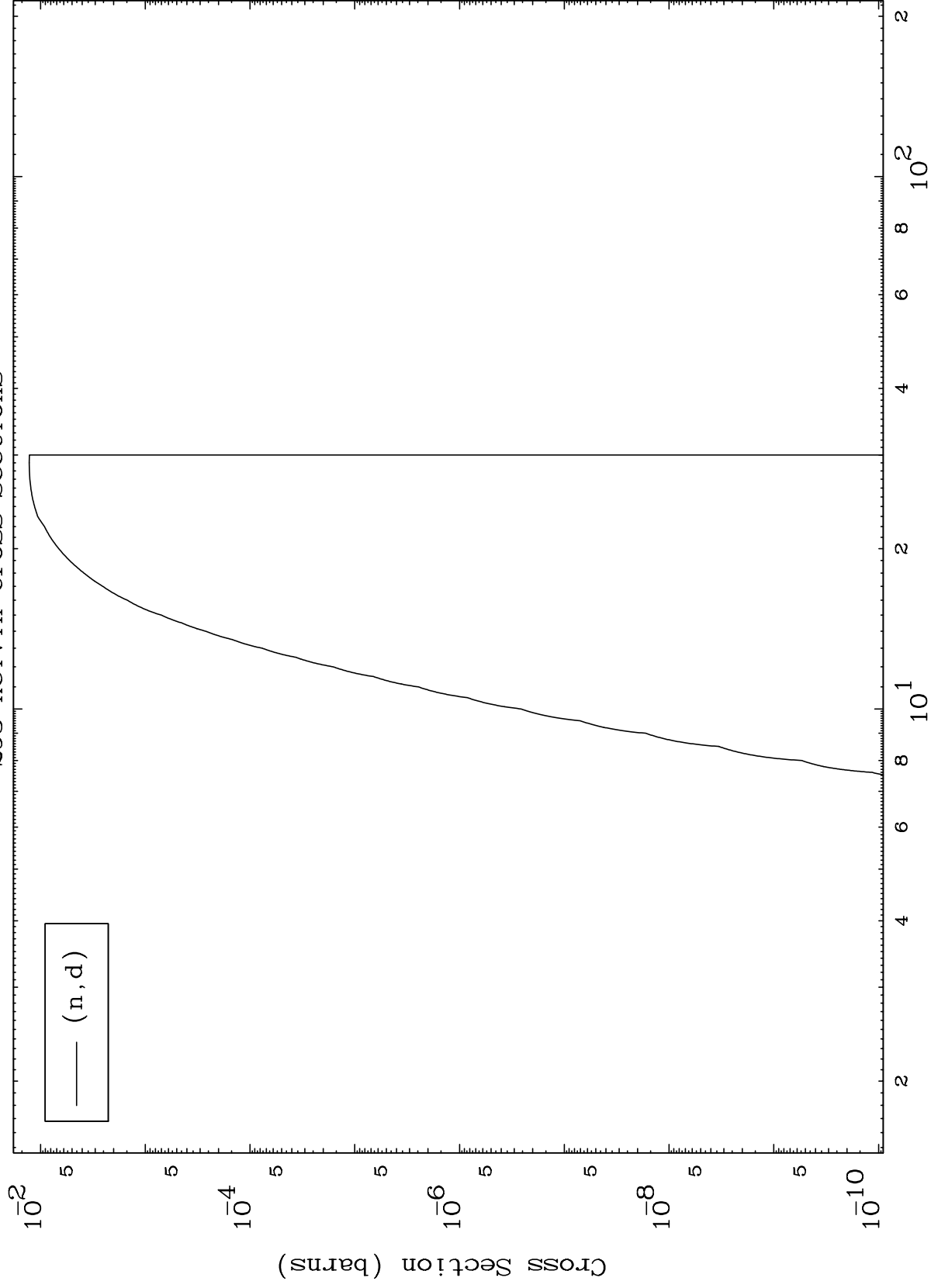
99-Es-250



MAT 9910

(n,d) Levels  
293 Kelvin Cross Sections

99-Es-250



12

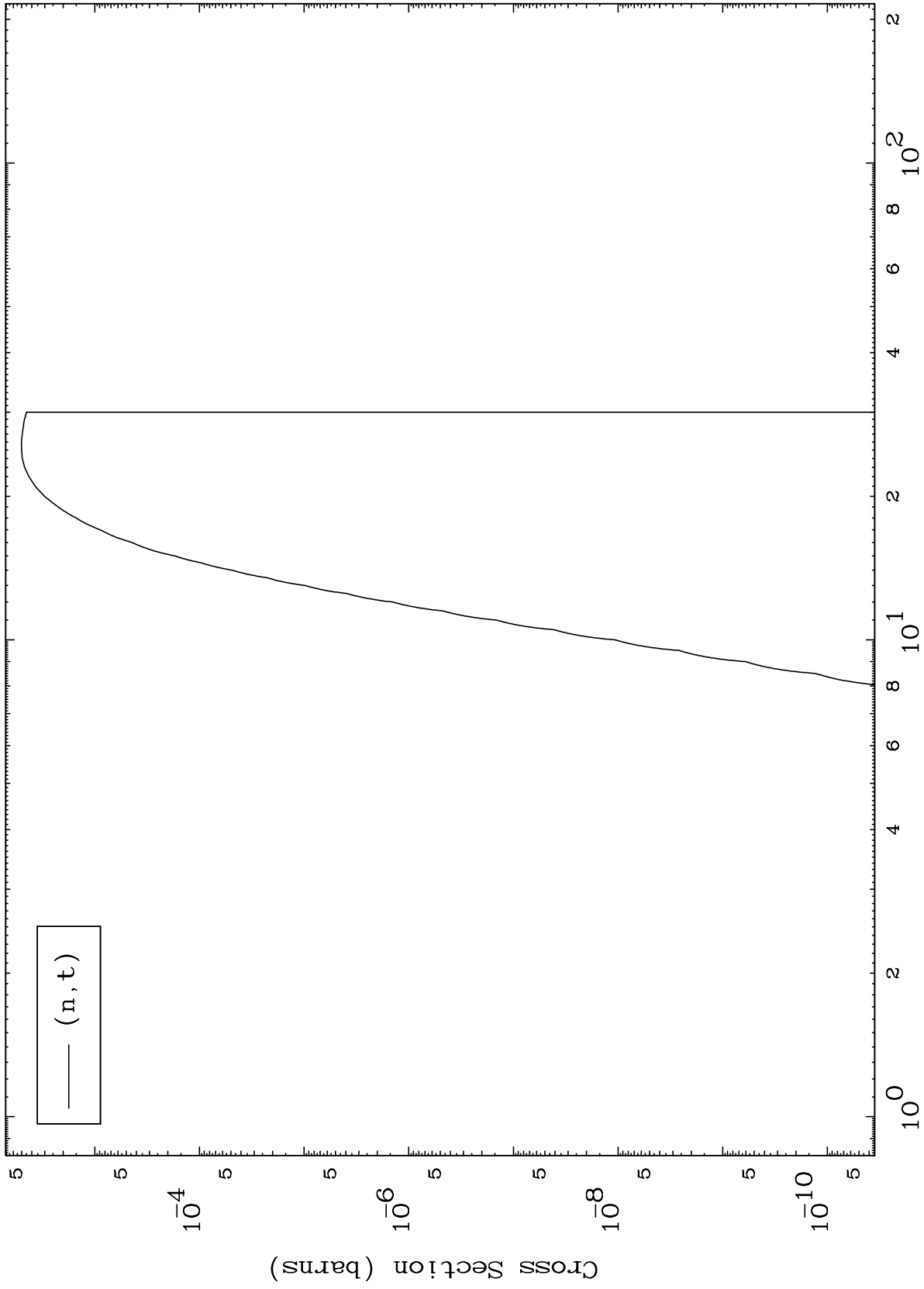
Incident Energy (MeV)

99-Es-250

MAT 9910

(n,t) Levels  
293 Kelvin Cross Sections

99-Es-250



(n,t)

Incident Energy (MeV)

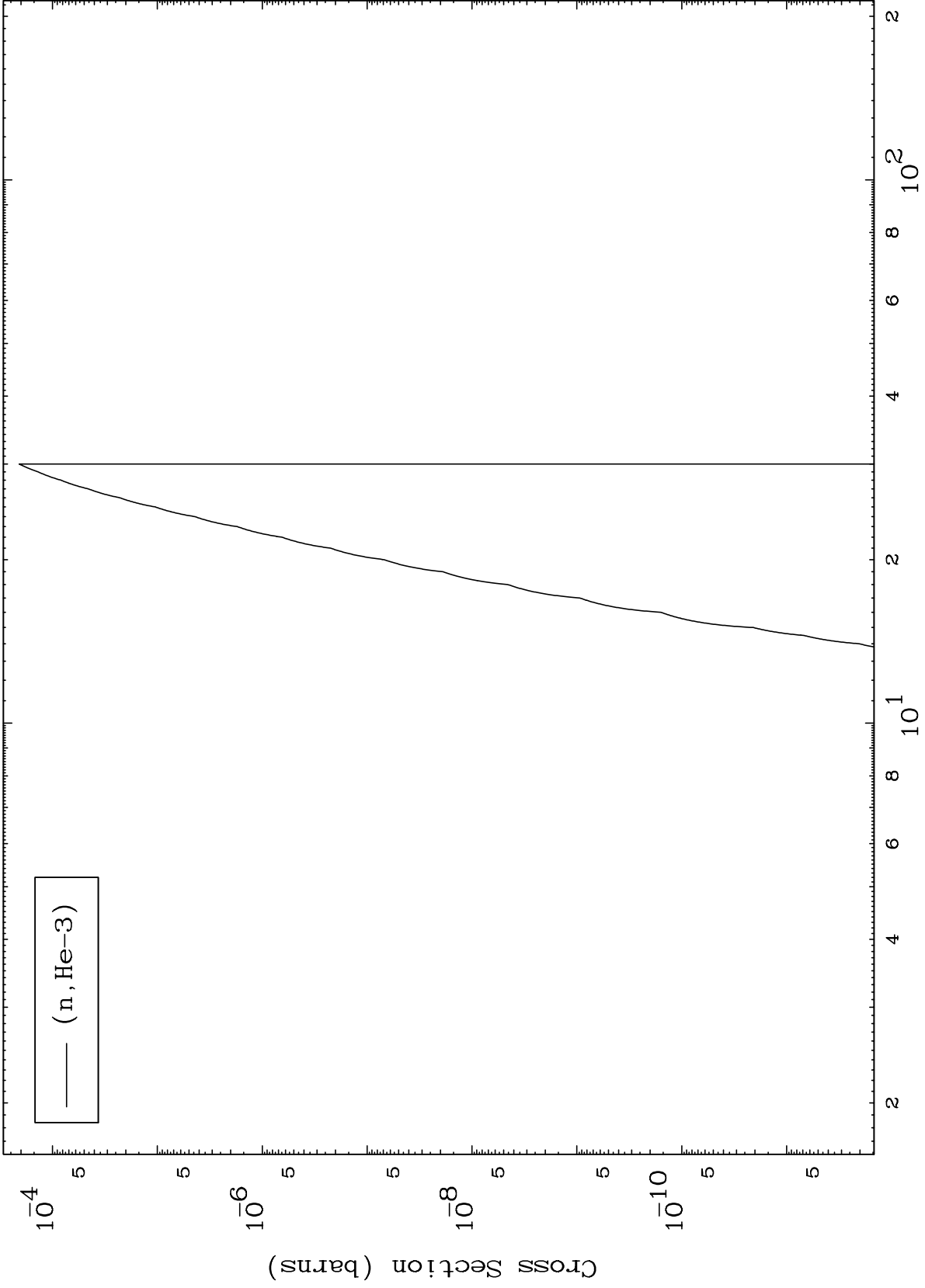
99-Es-250

13

MAT 9910

(n,He3) Levels  
293 Kelvin Cross Sections

99-Es-250



14

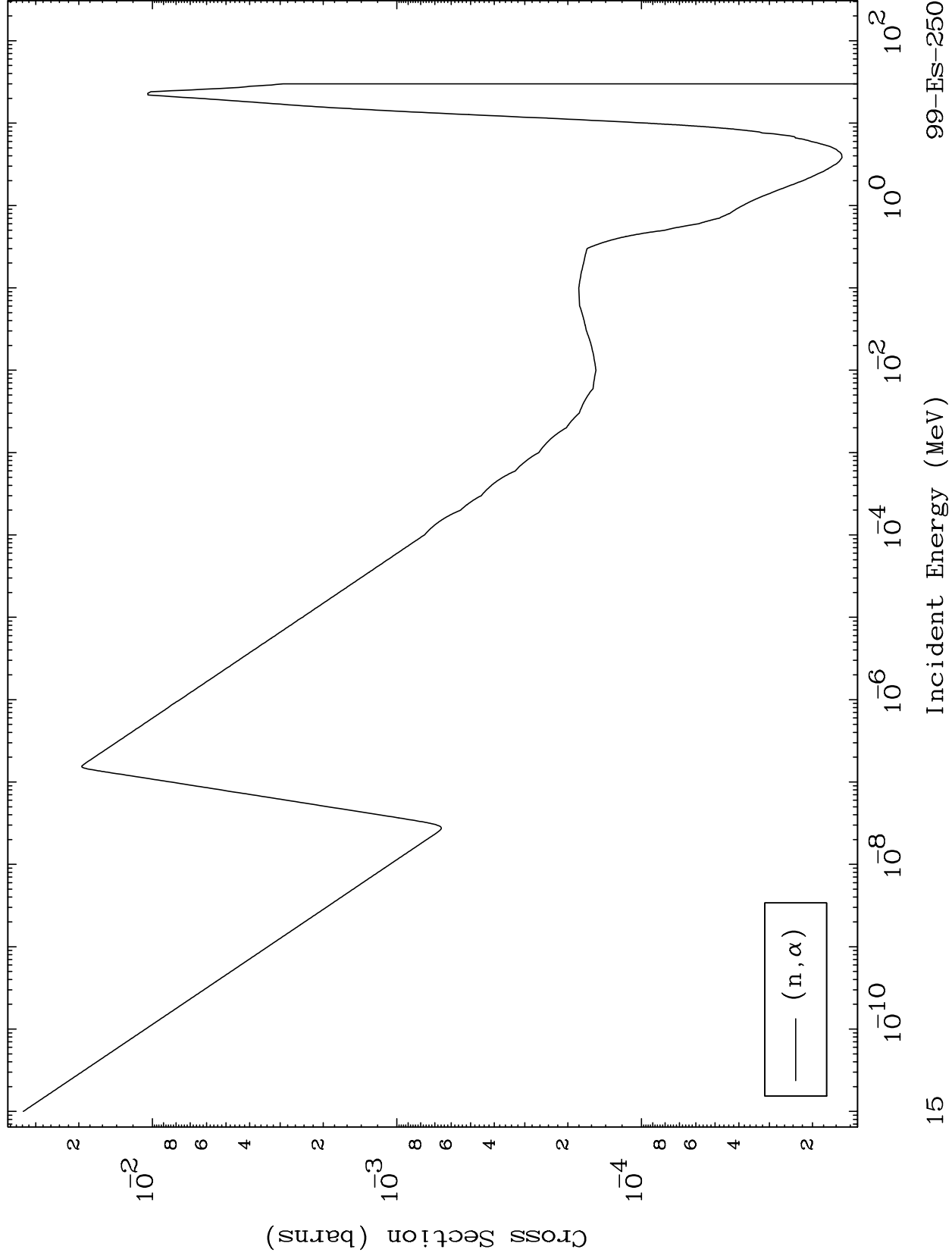
Incident Energy (MeV)

99-Es-250

MAT 9910

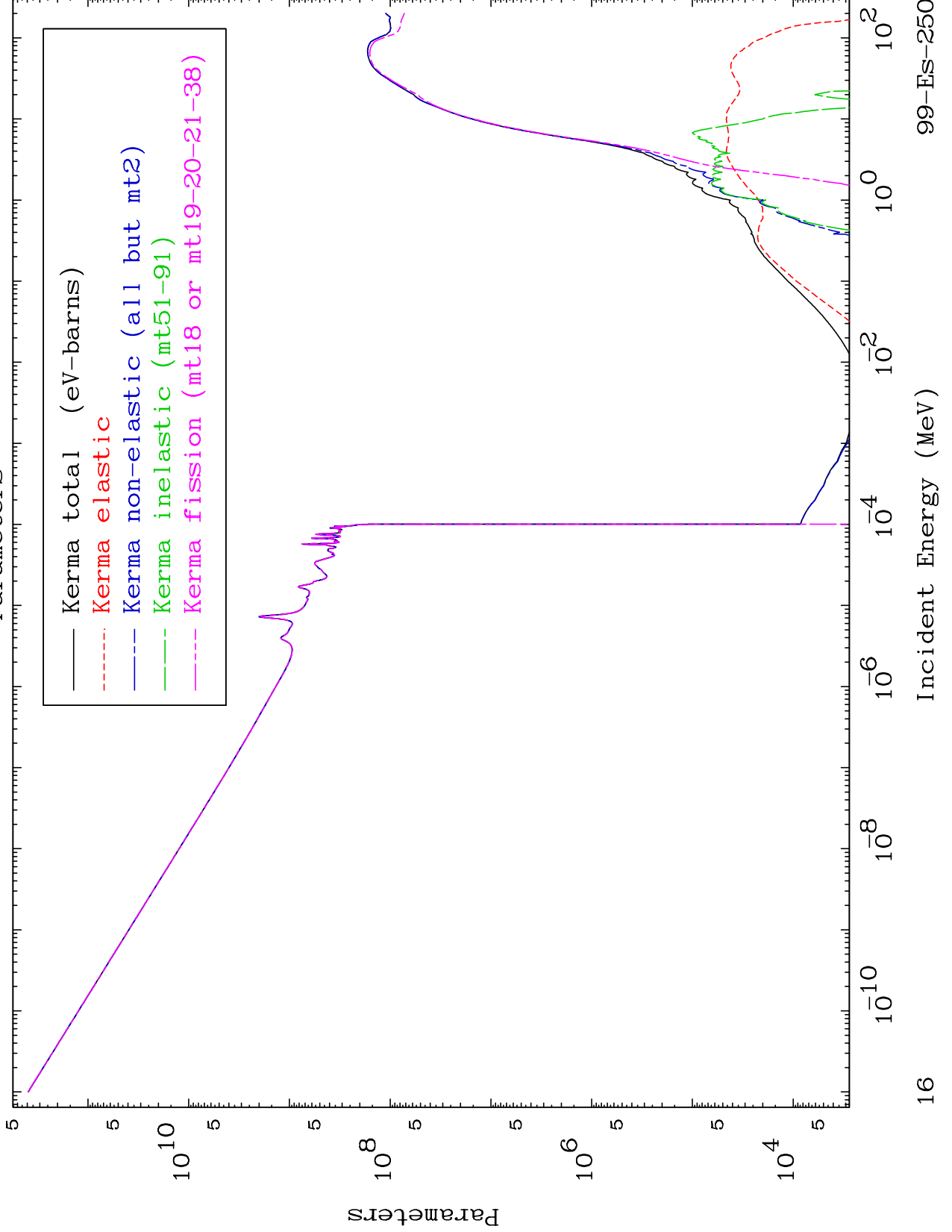
(n,α) Levels  
293 Kelvin Cross Sections

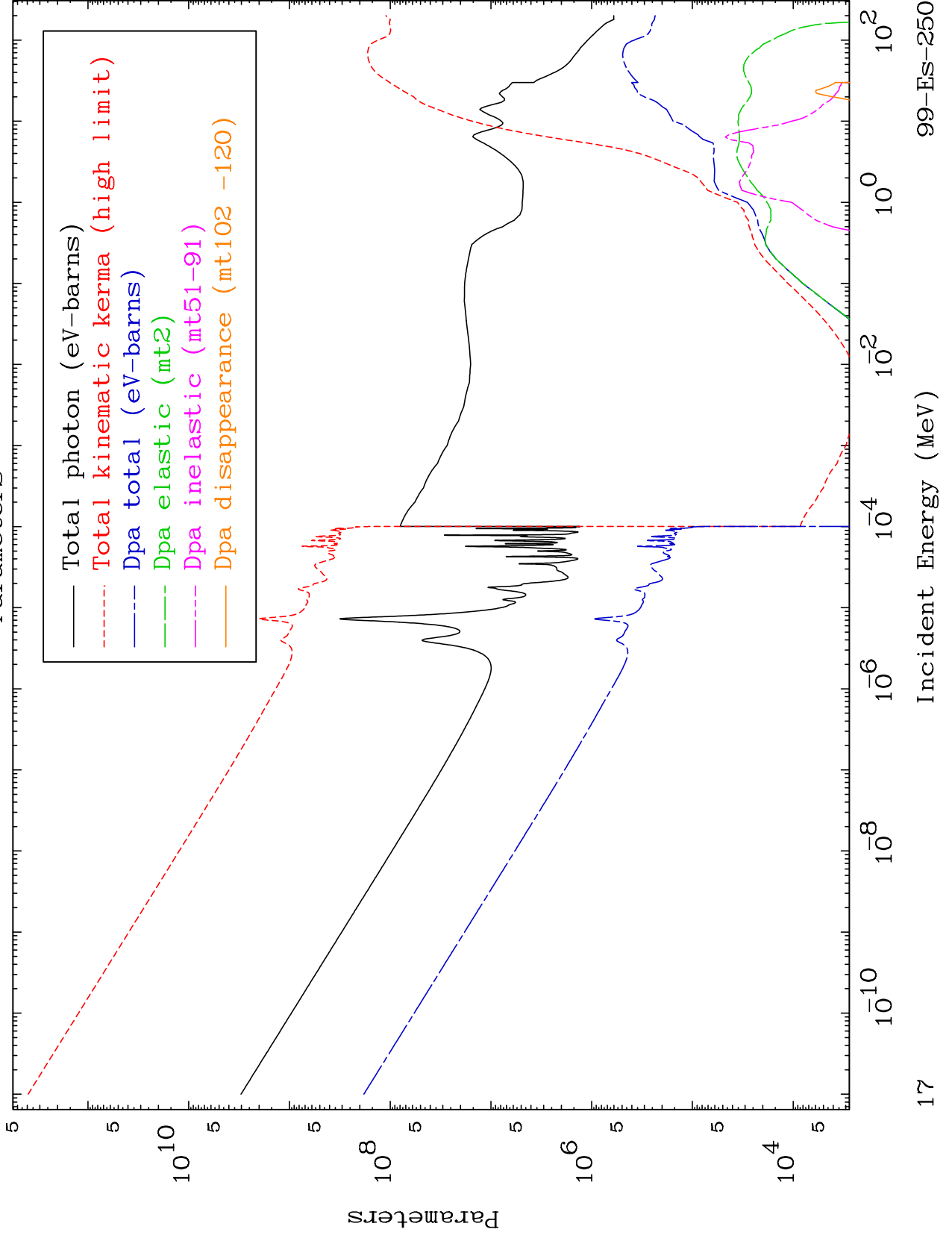
99-Es-250

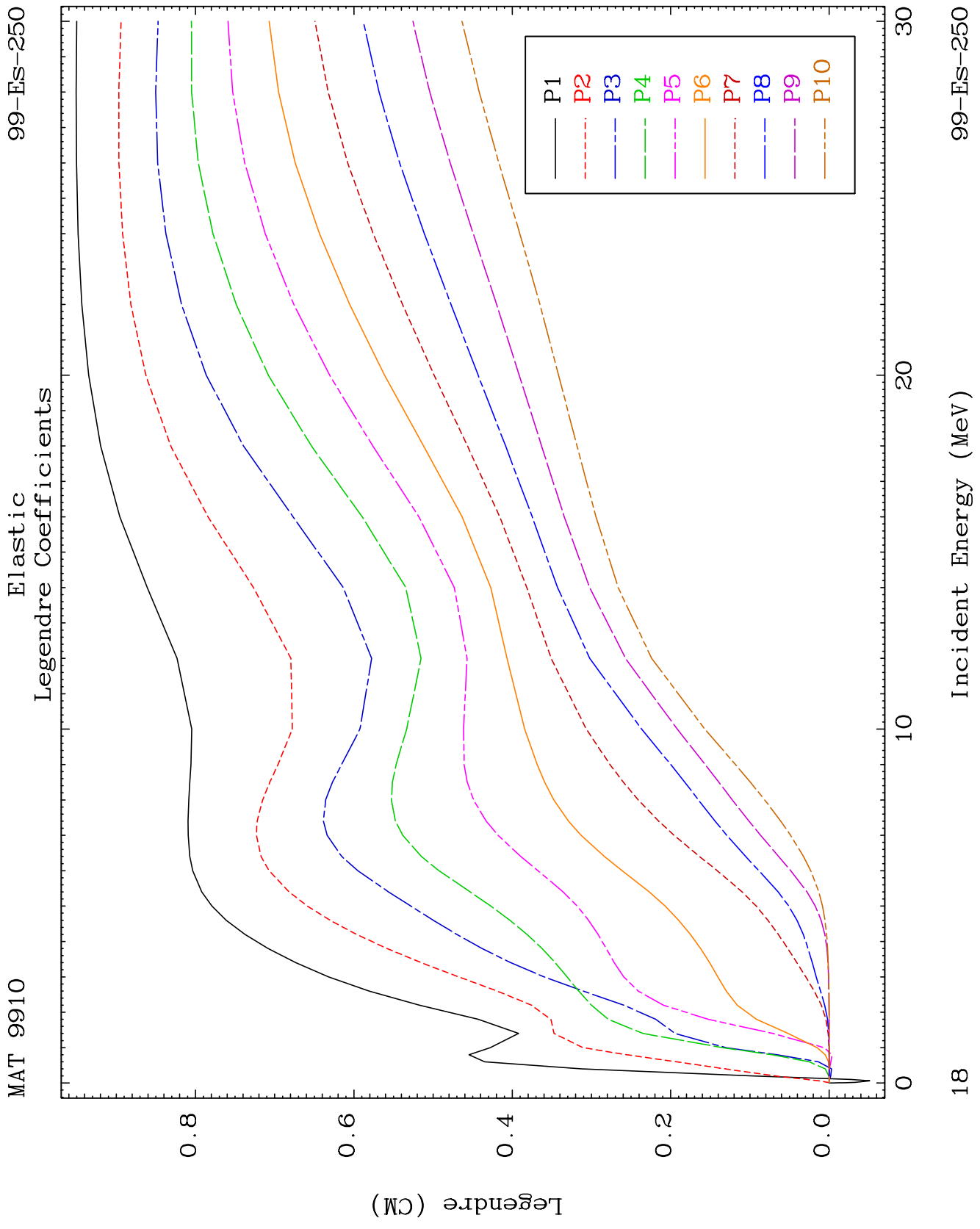


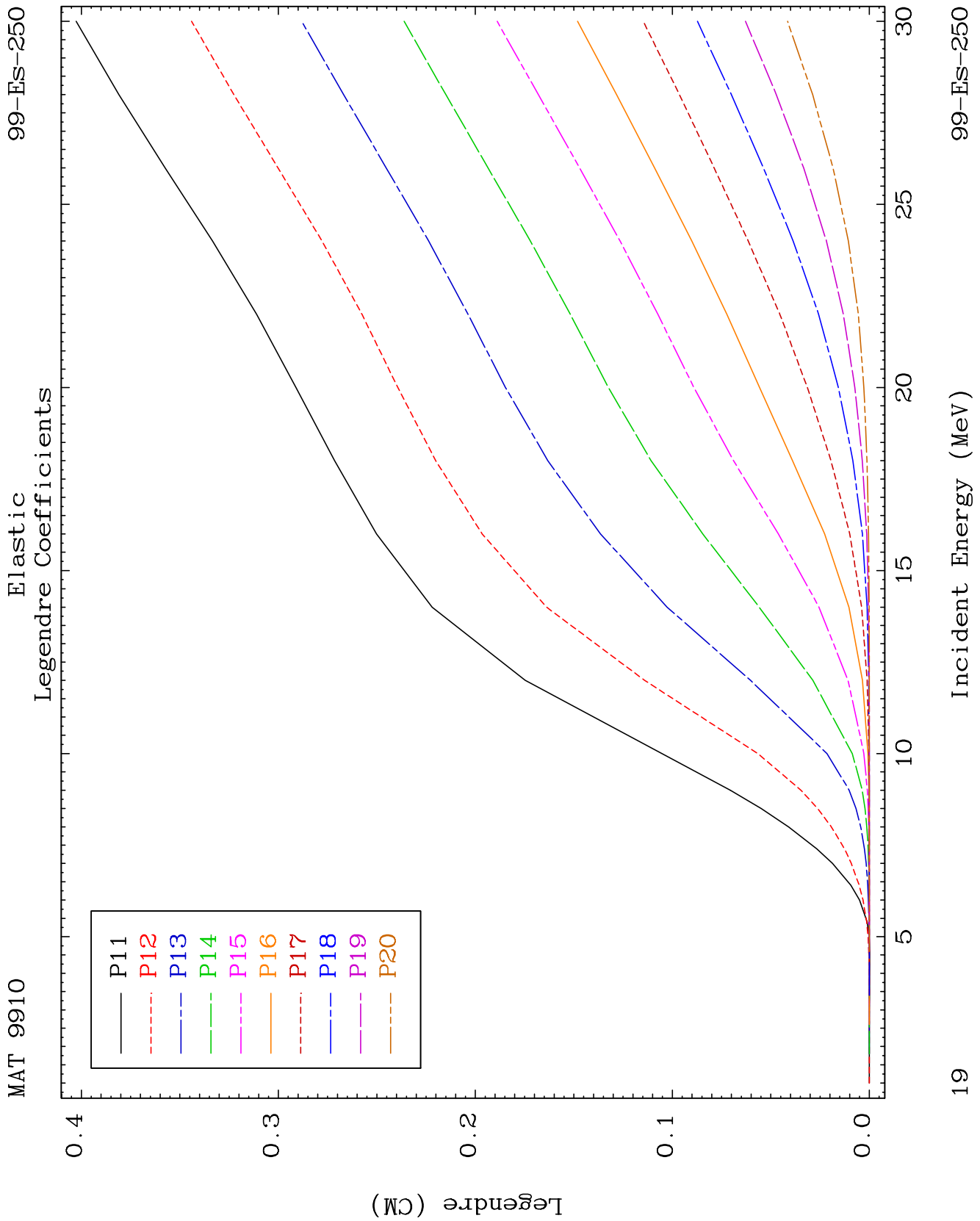
15

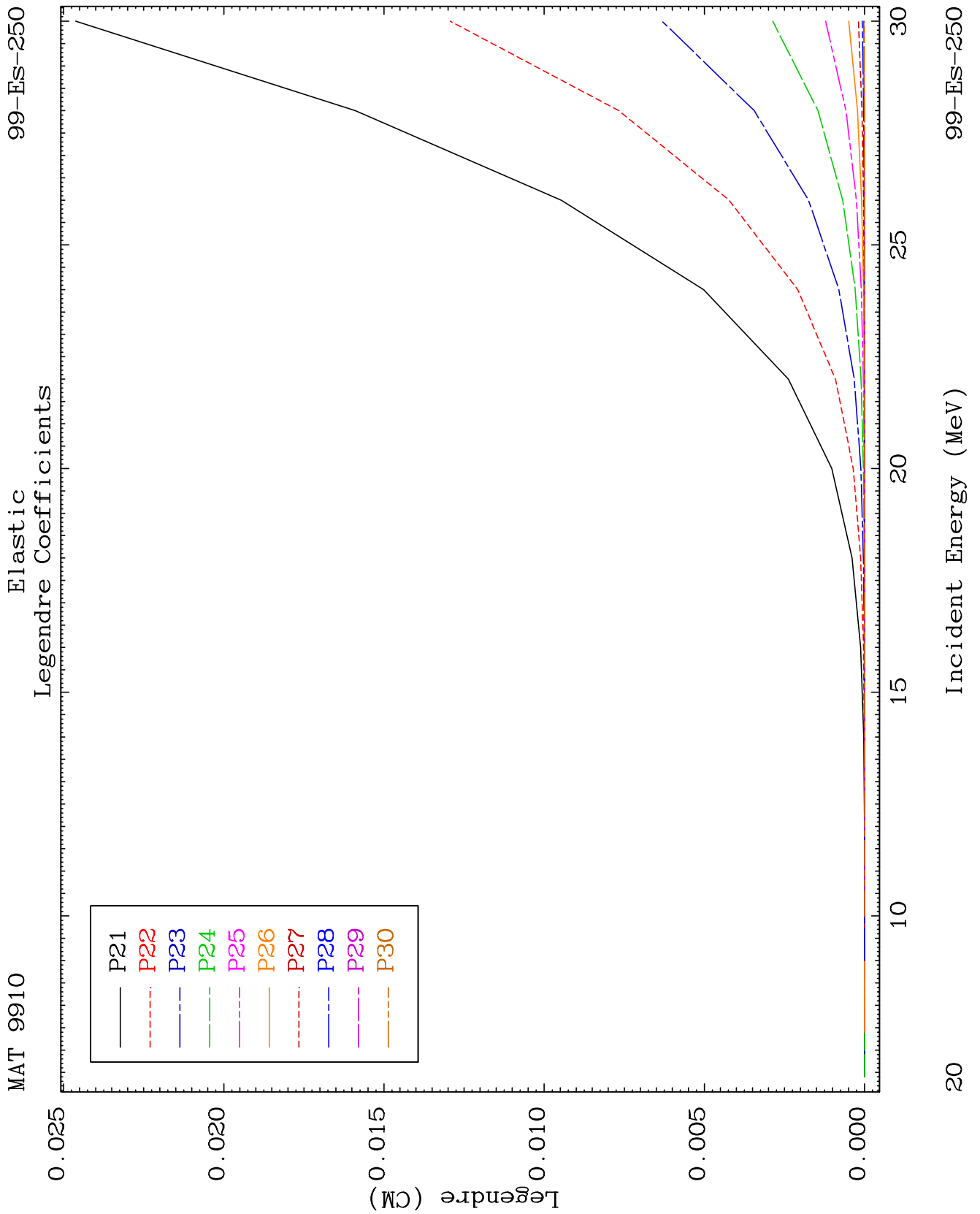


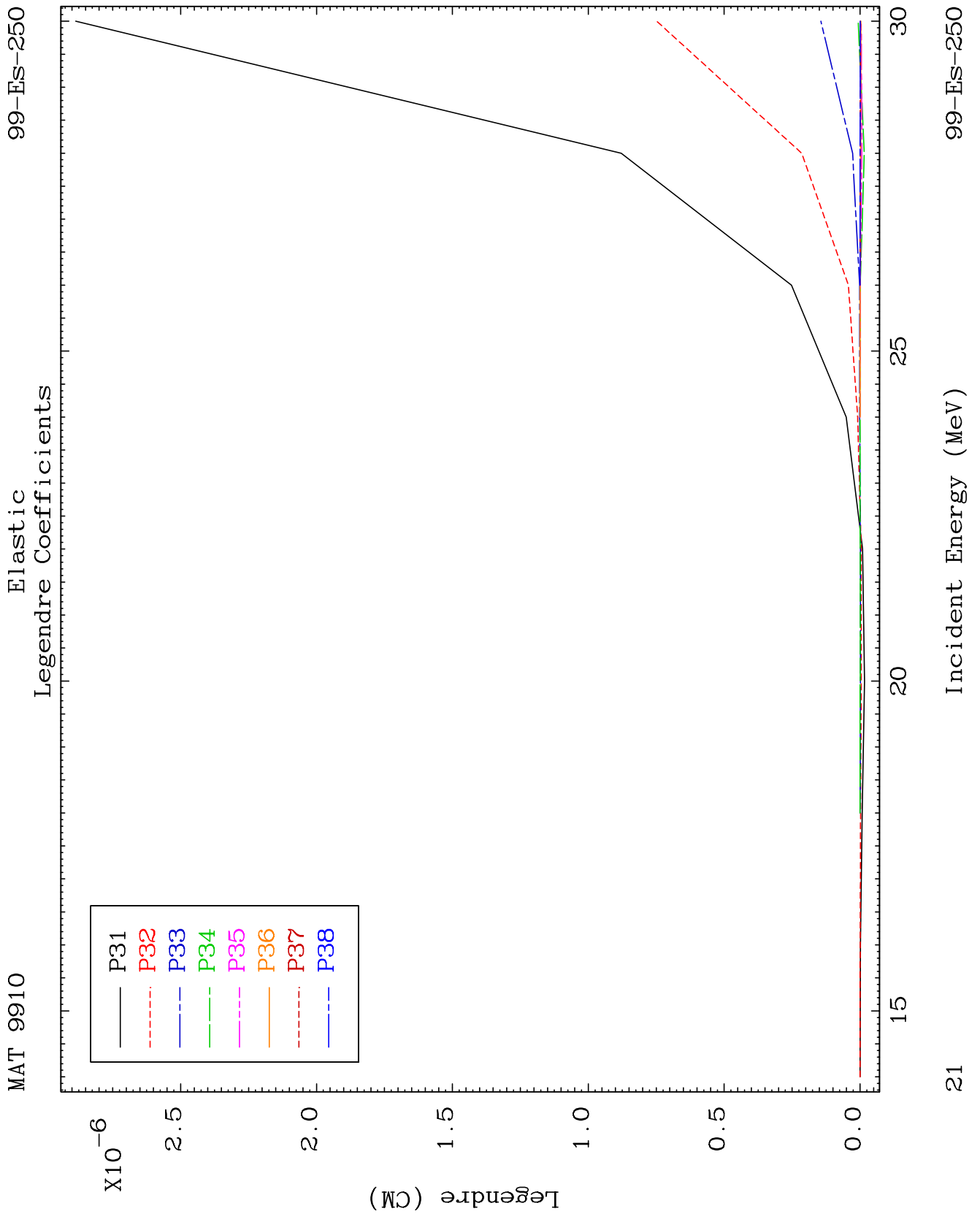




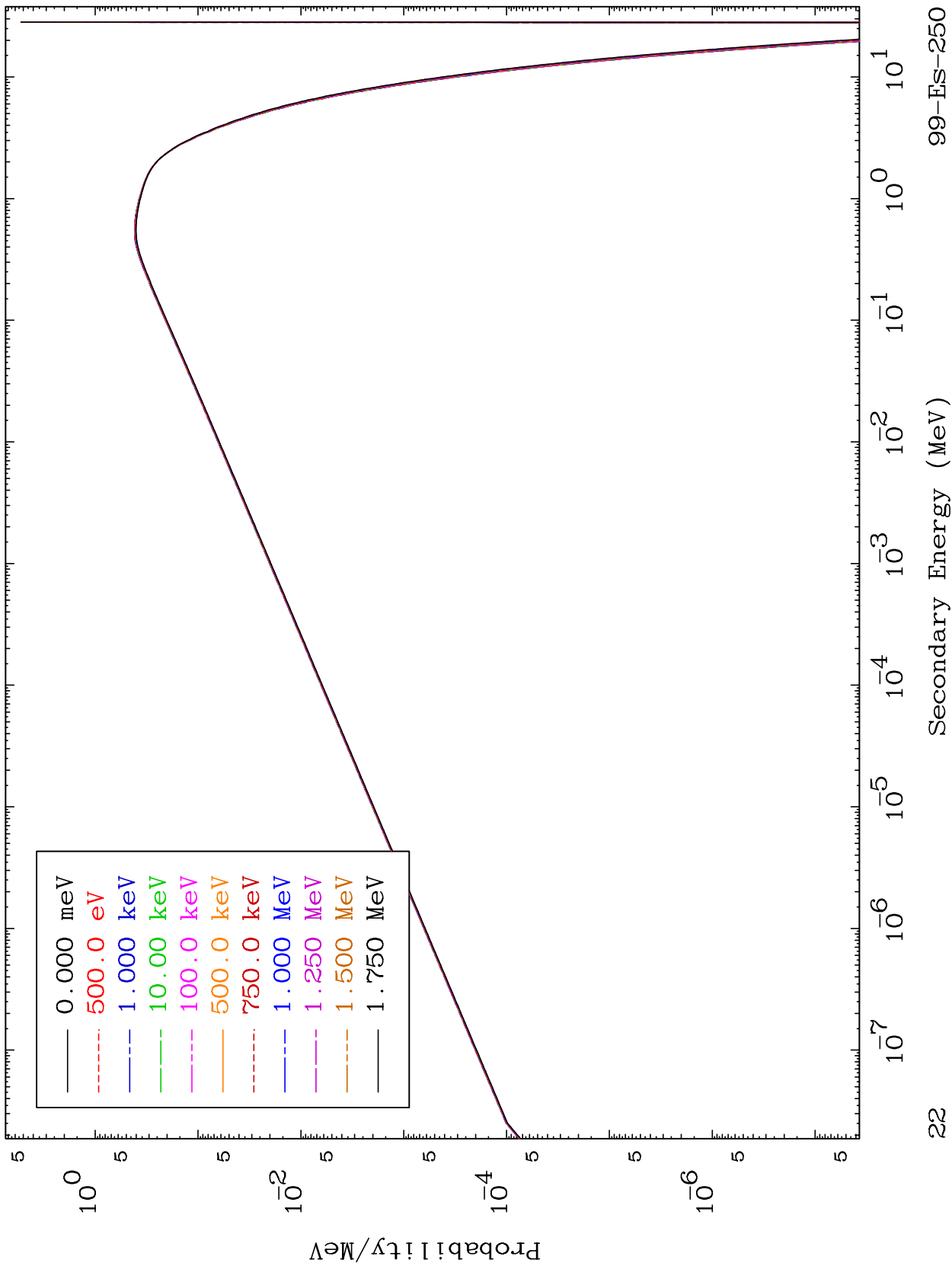




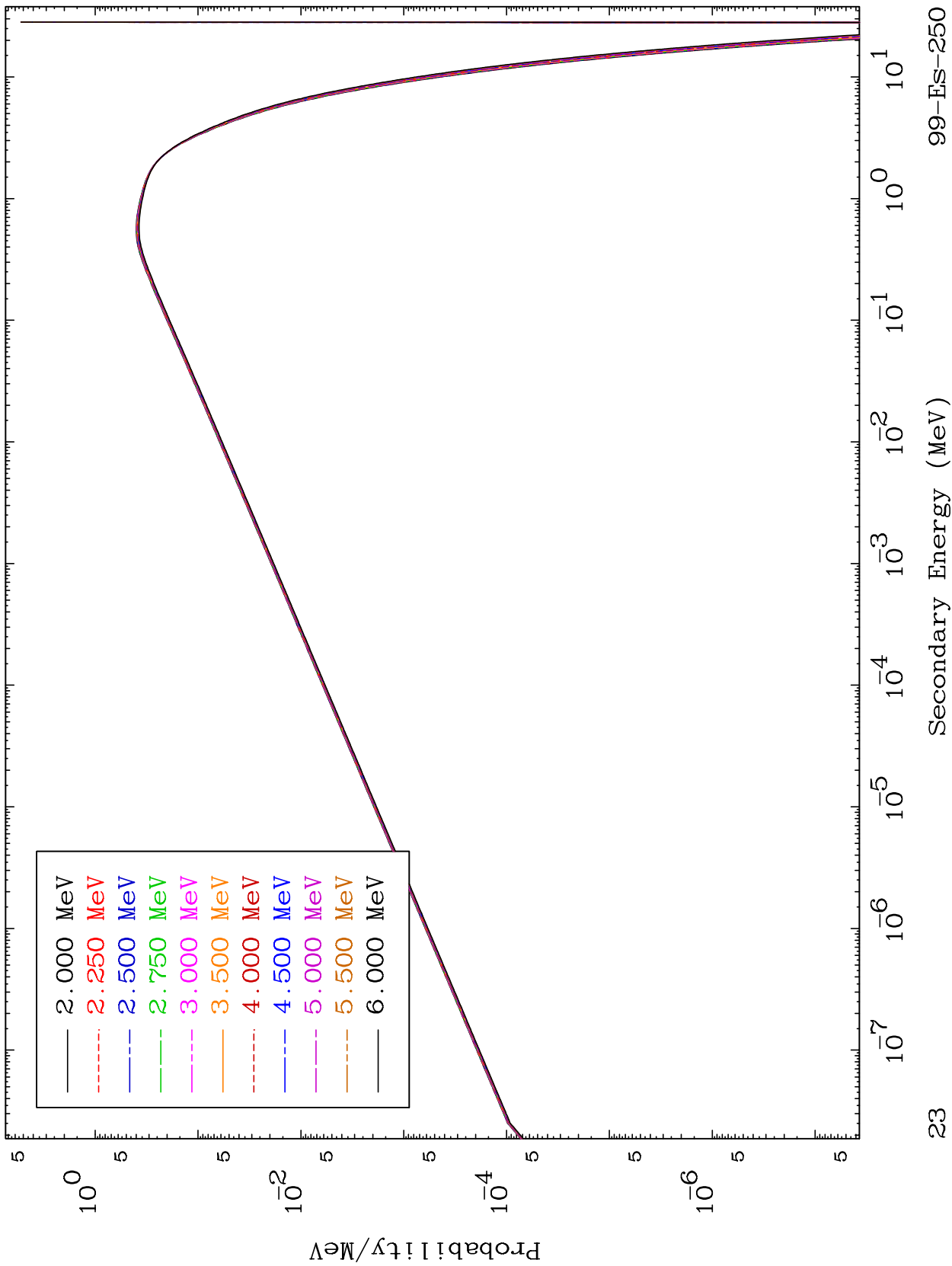




Fission Energy Distribution



Fission Energy Distribution

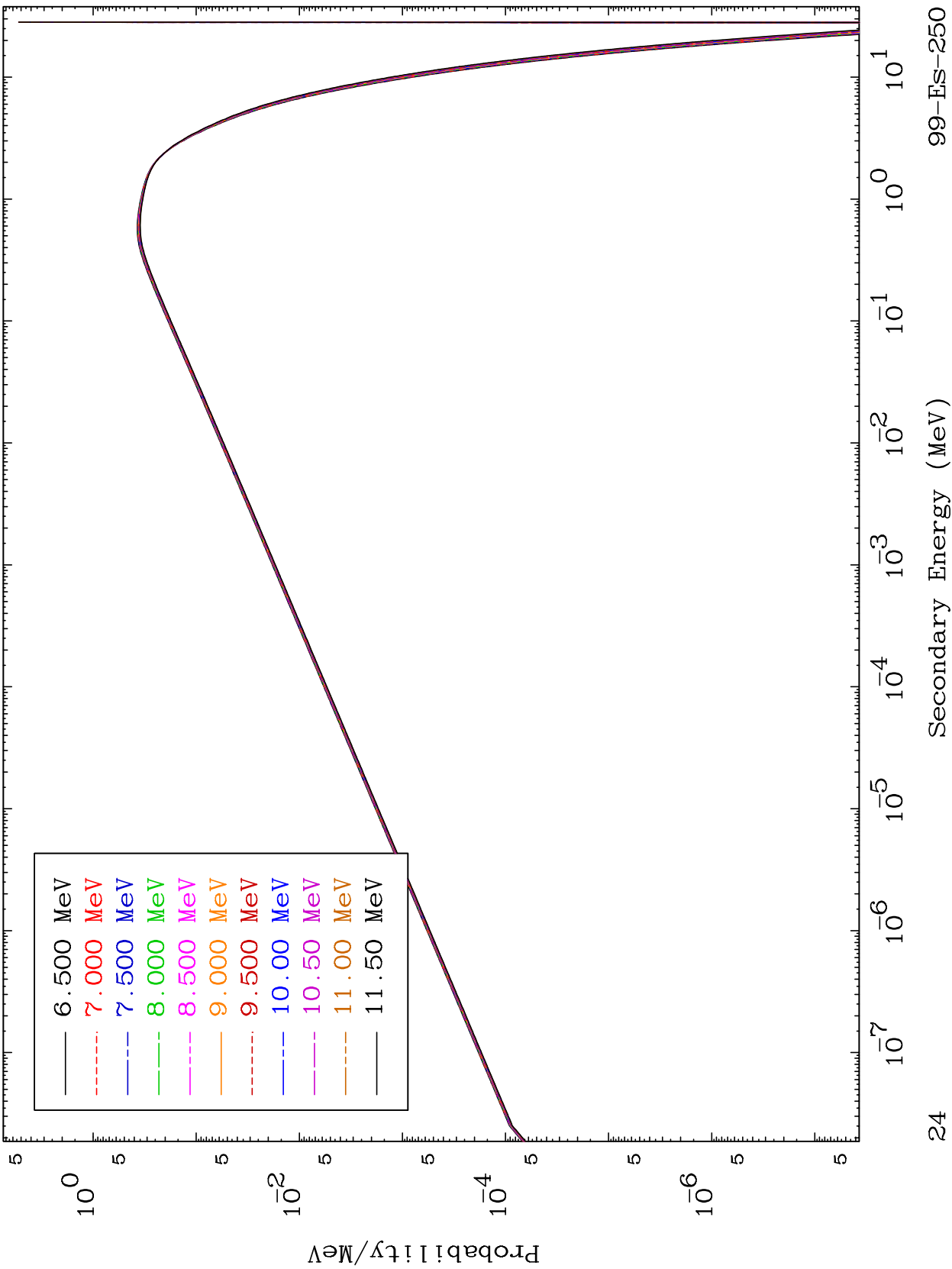




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### Fission Energy Distribution

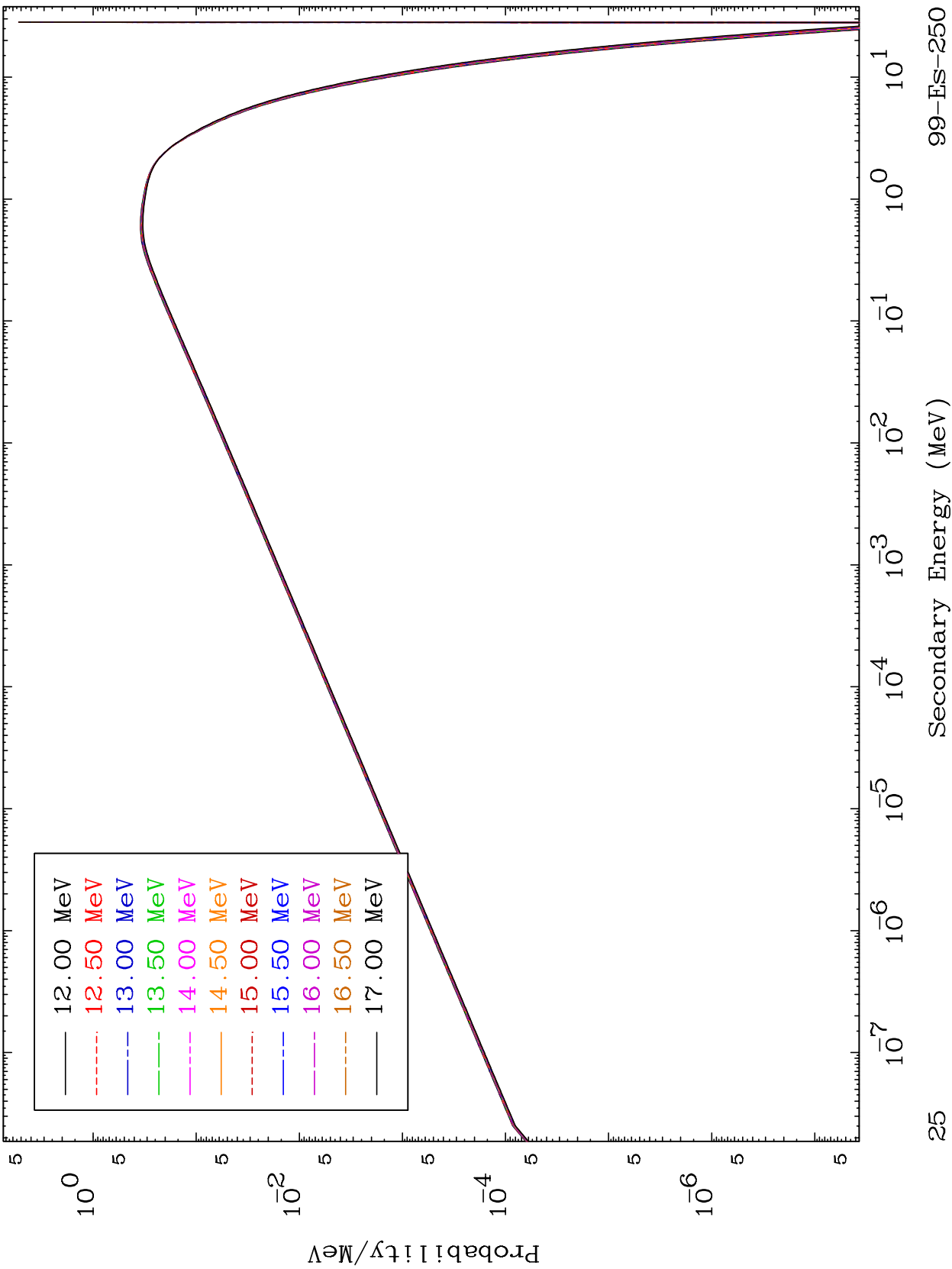
99-Es-250



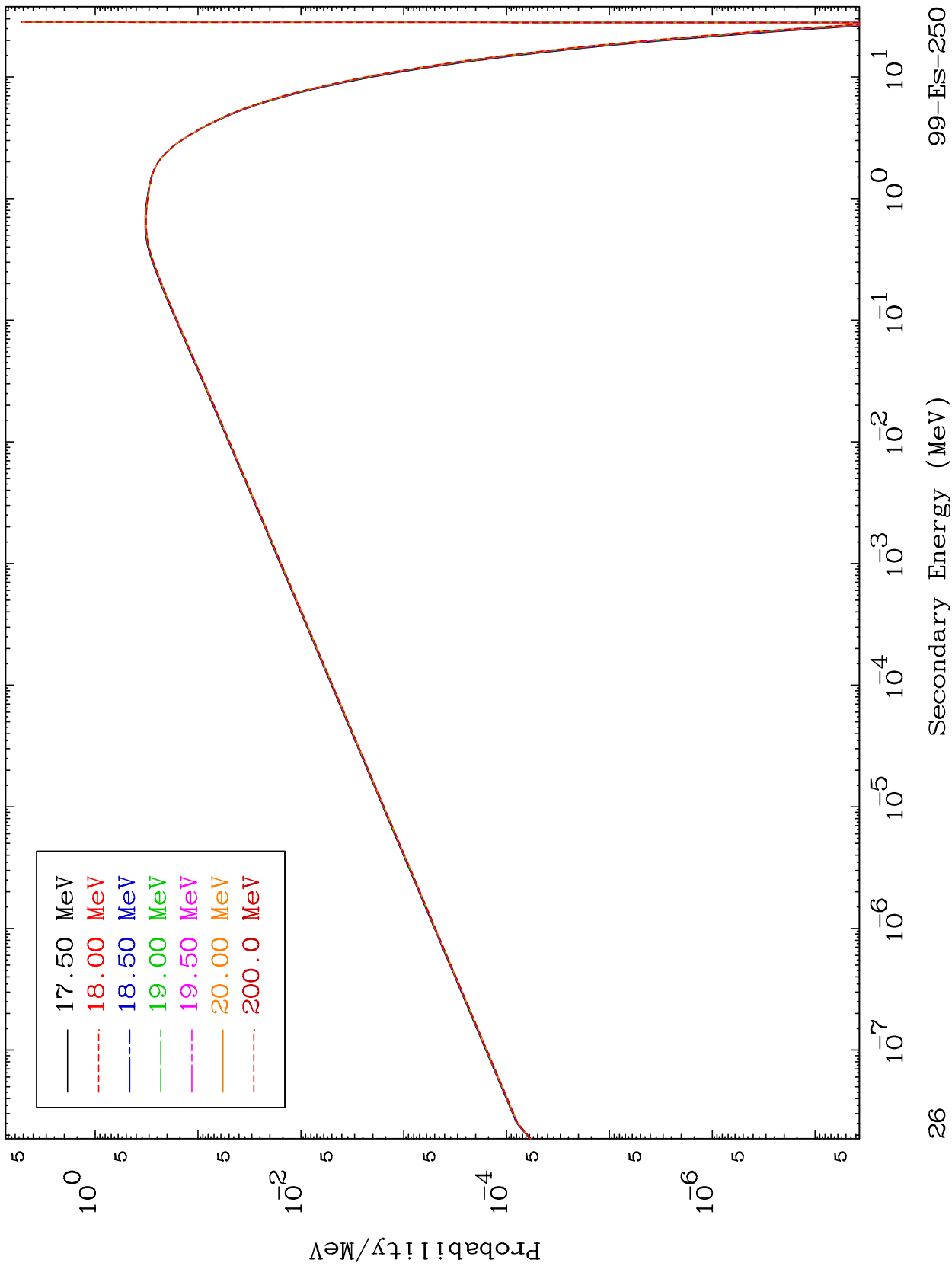
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### Fission Energy Distribution

99-Es-250



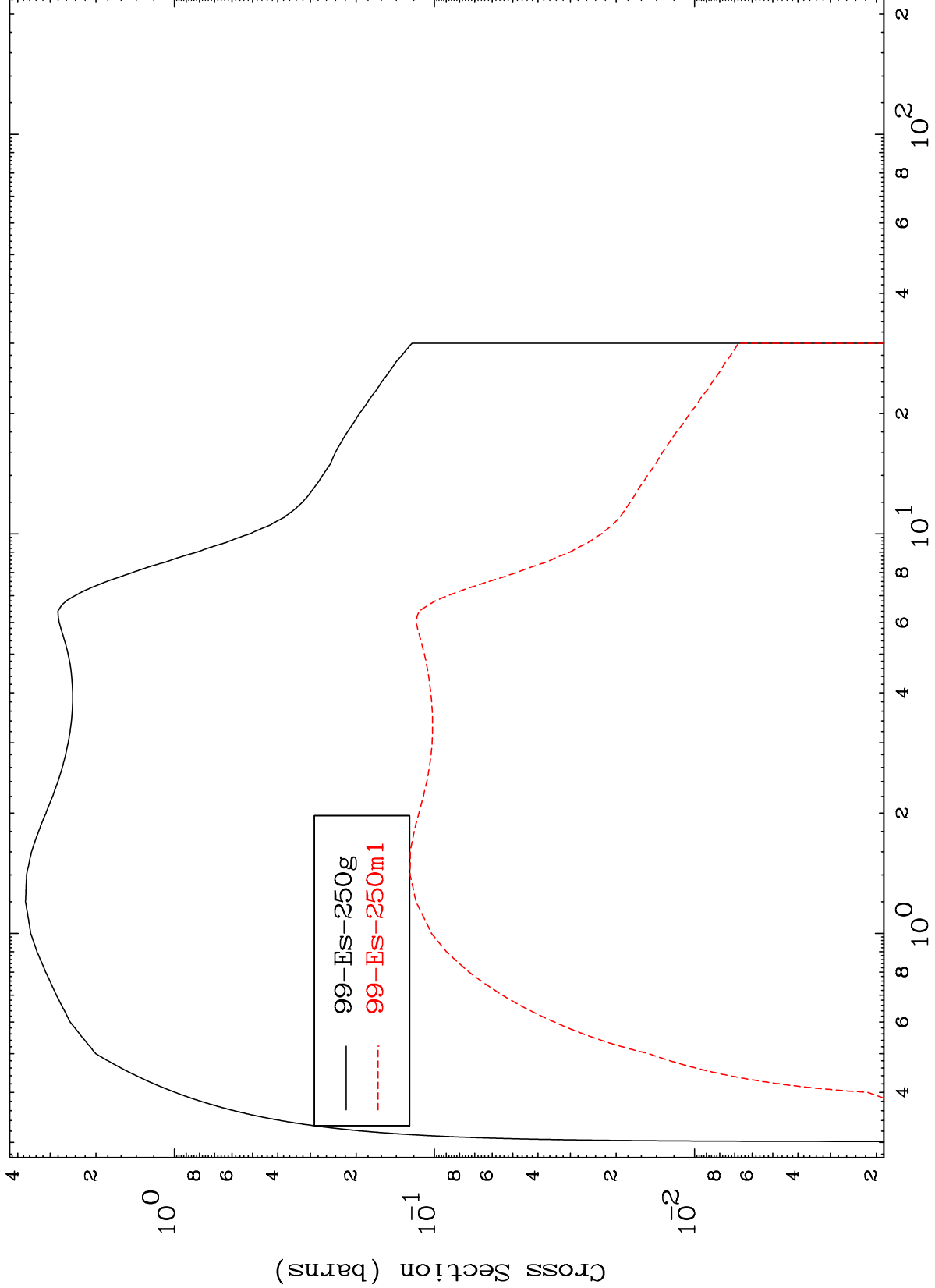
Fission  
Energy Distribution



MAT 9910

Inelastic  
Radionuclide Production Cross Section

99-Es-250



27

Incident Energy (MeV)

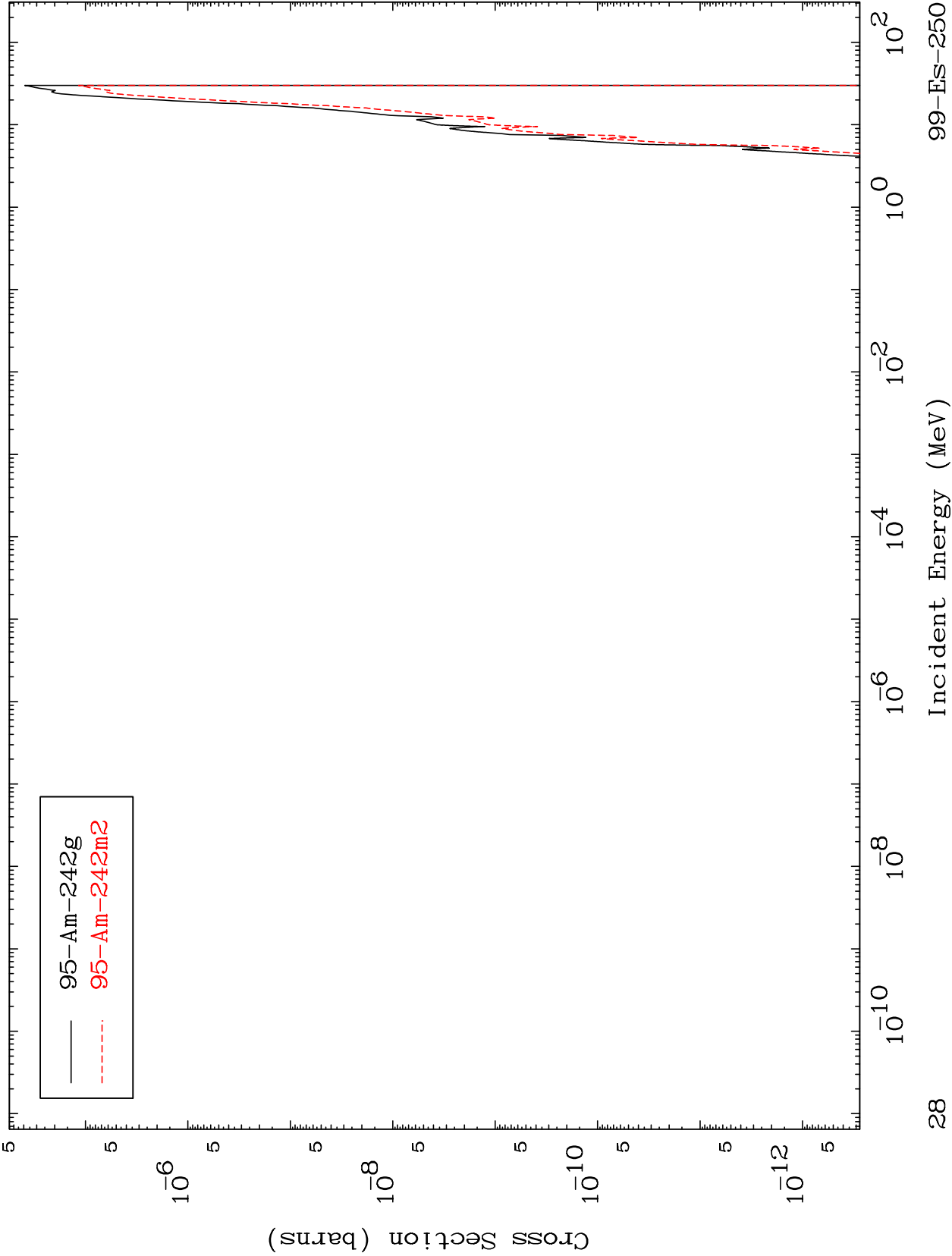
99-Es-250

MAT 9910

(n,n') 2α

99-Es-250

Radionuclide Production Cross Section

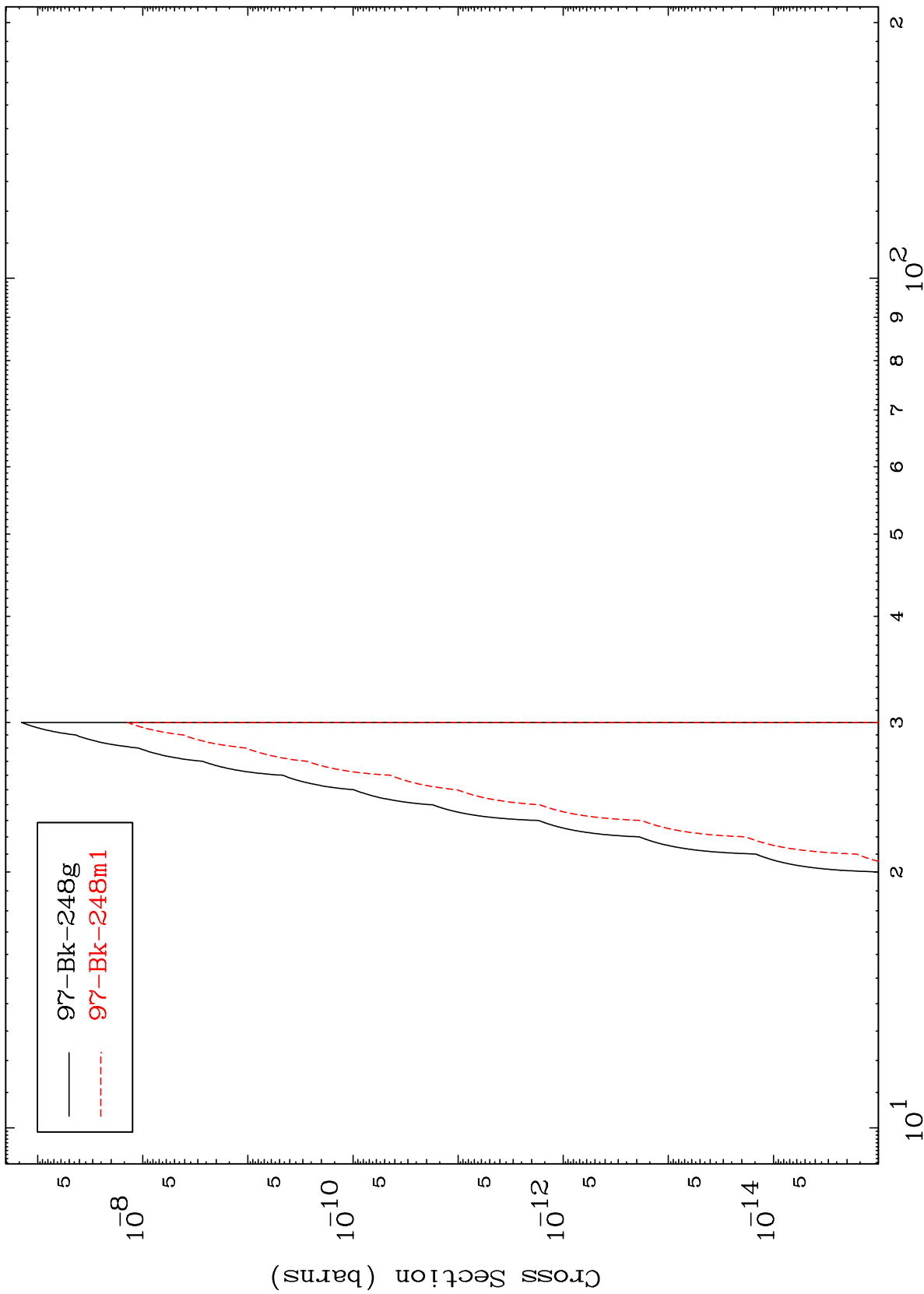


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(n,2n) p

99-Es-250

Radionuclide Production Cross Section



97-Bk-248g  
97-Bk-248m1

29

Incident Energy (MeV)

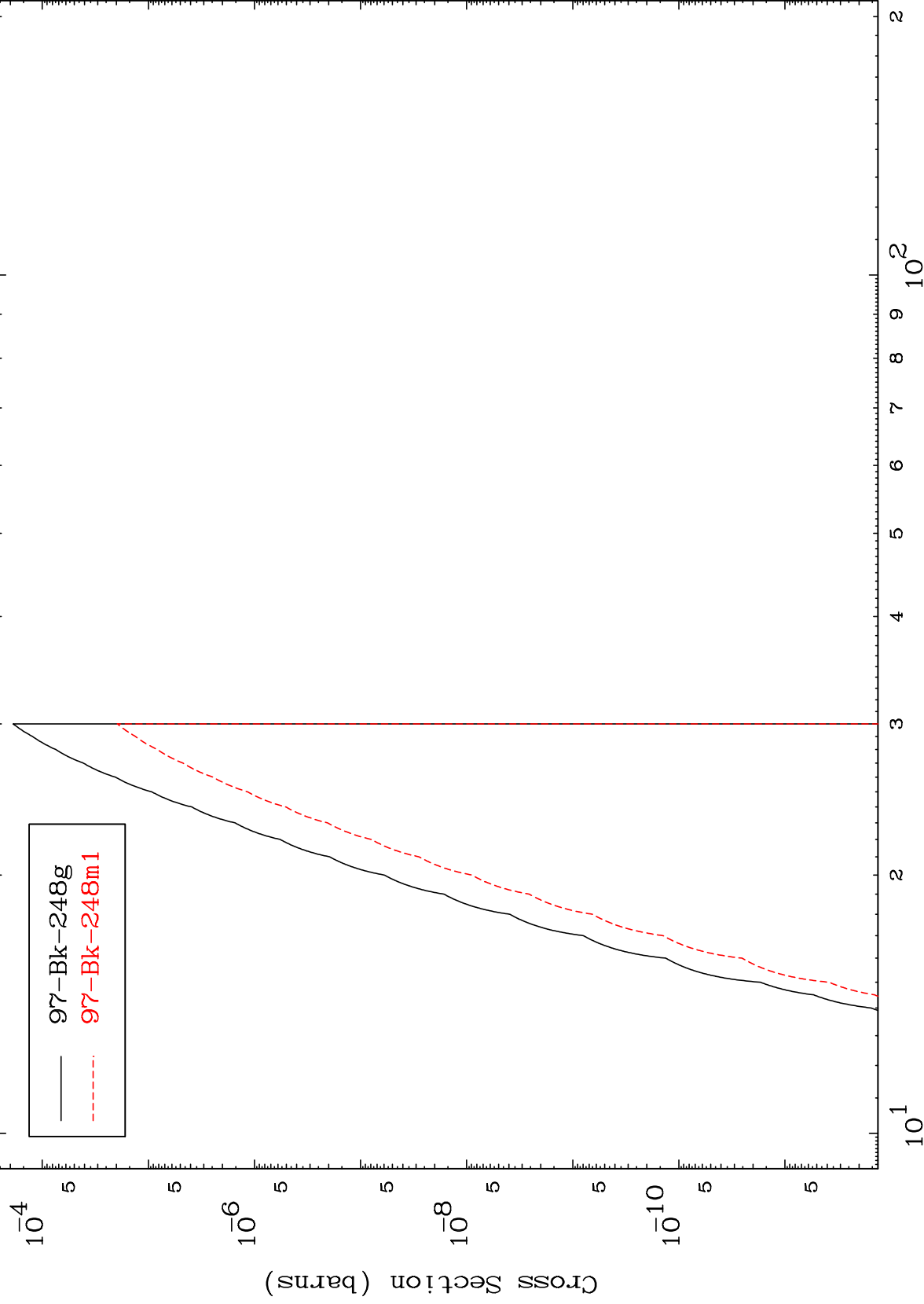
99-Es-250

MAT 9910

(n,He-3)

99-Es-250

Radionuclide Production Cross Section



Incident Energy (MeV)

99-Es-250

30