

Program EVALPLOT  
(Version 2015-2)

by

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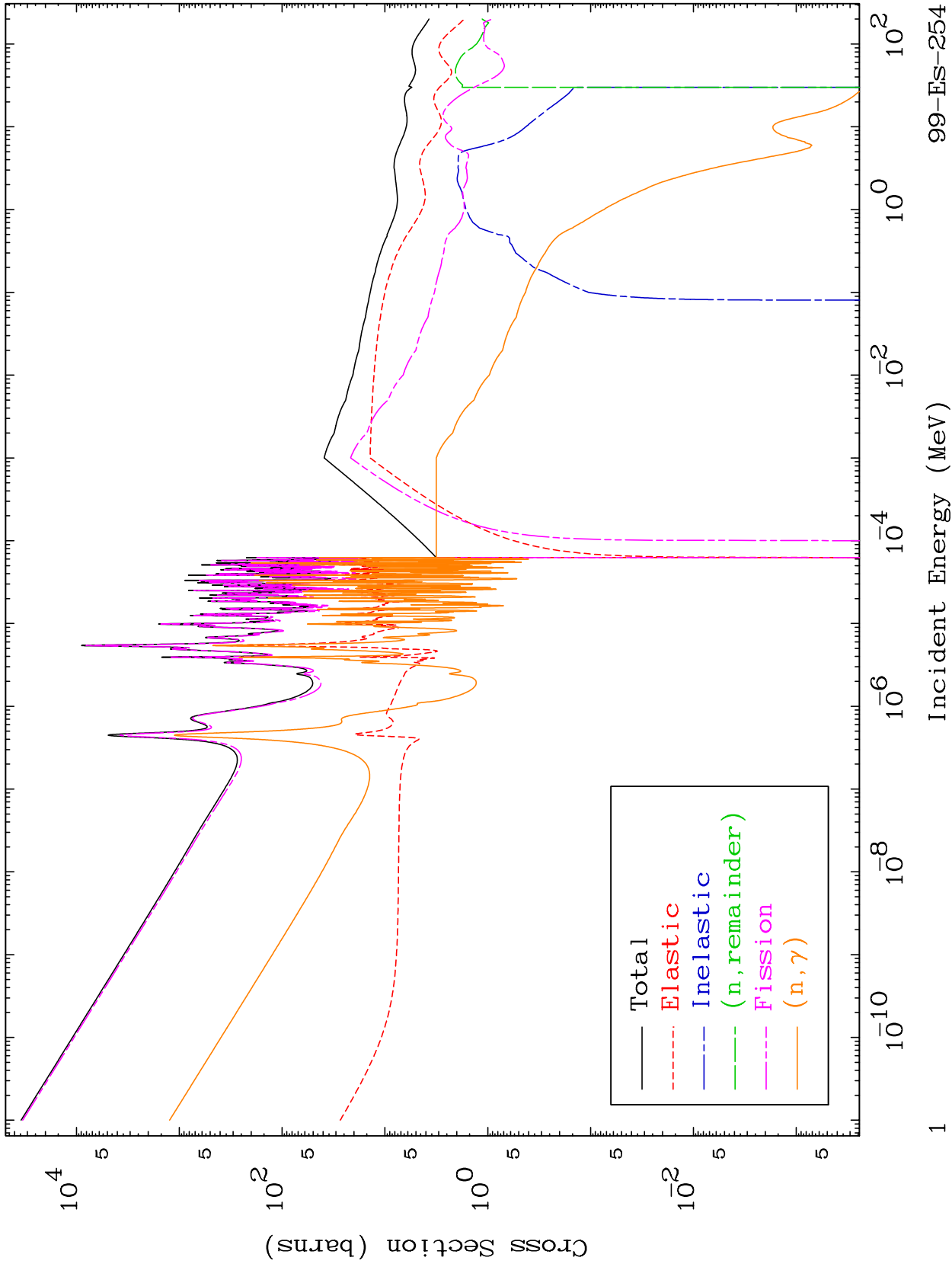
Press Mouse Button to Start

MAT 9914

Major

293 Kelvin Cross Sections

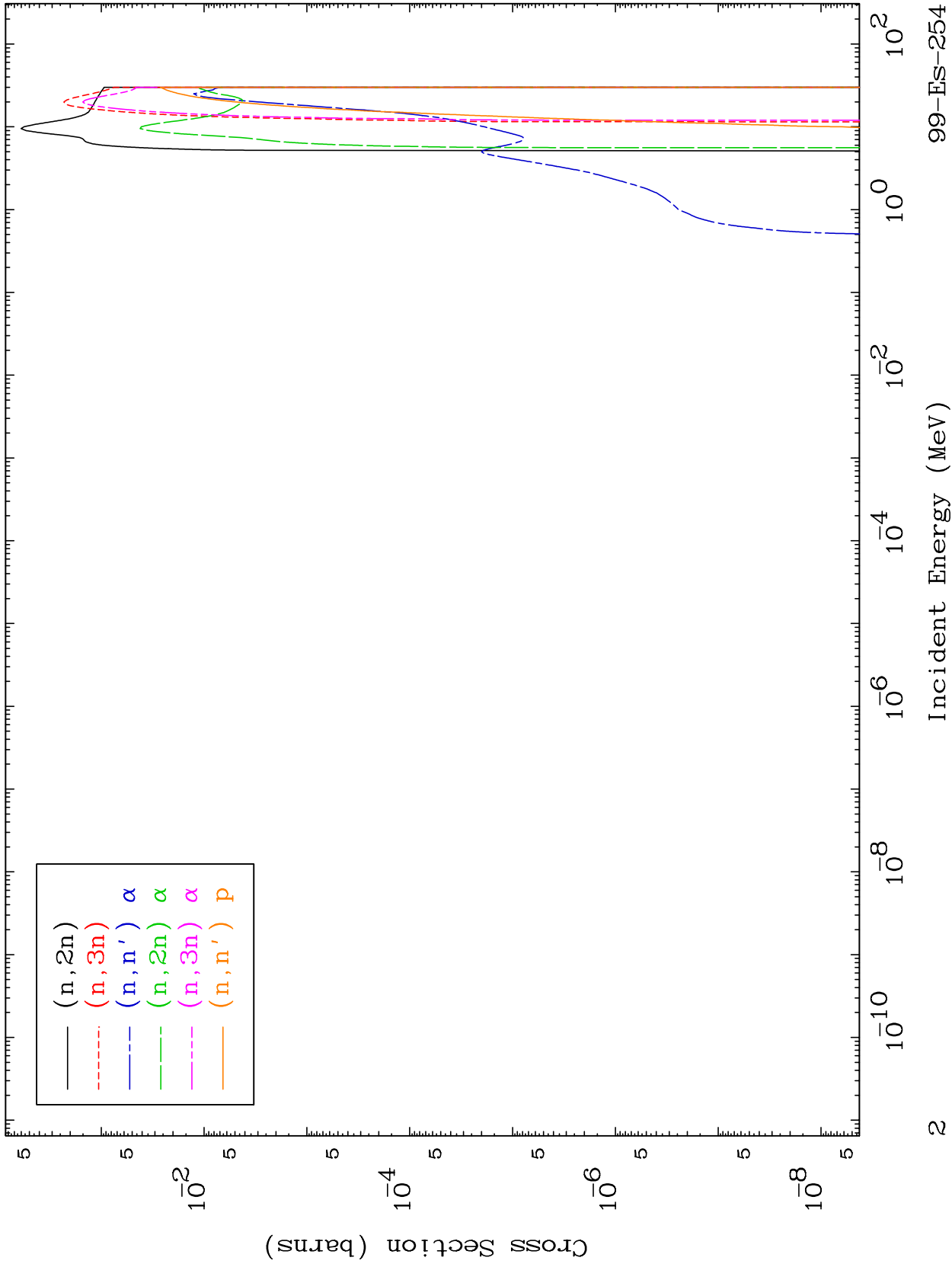
99-Es-254



MAT 9914

Neutron Production  
293 Kelvin Cross Sections

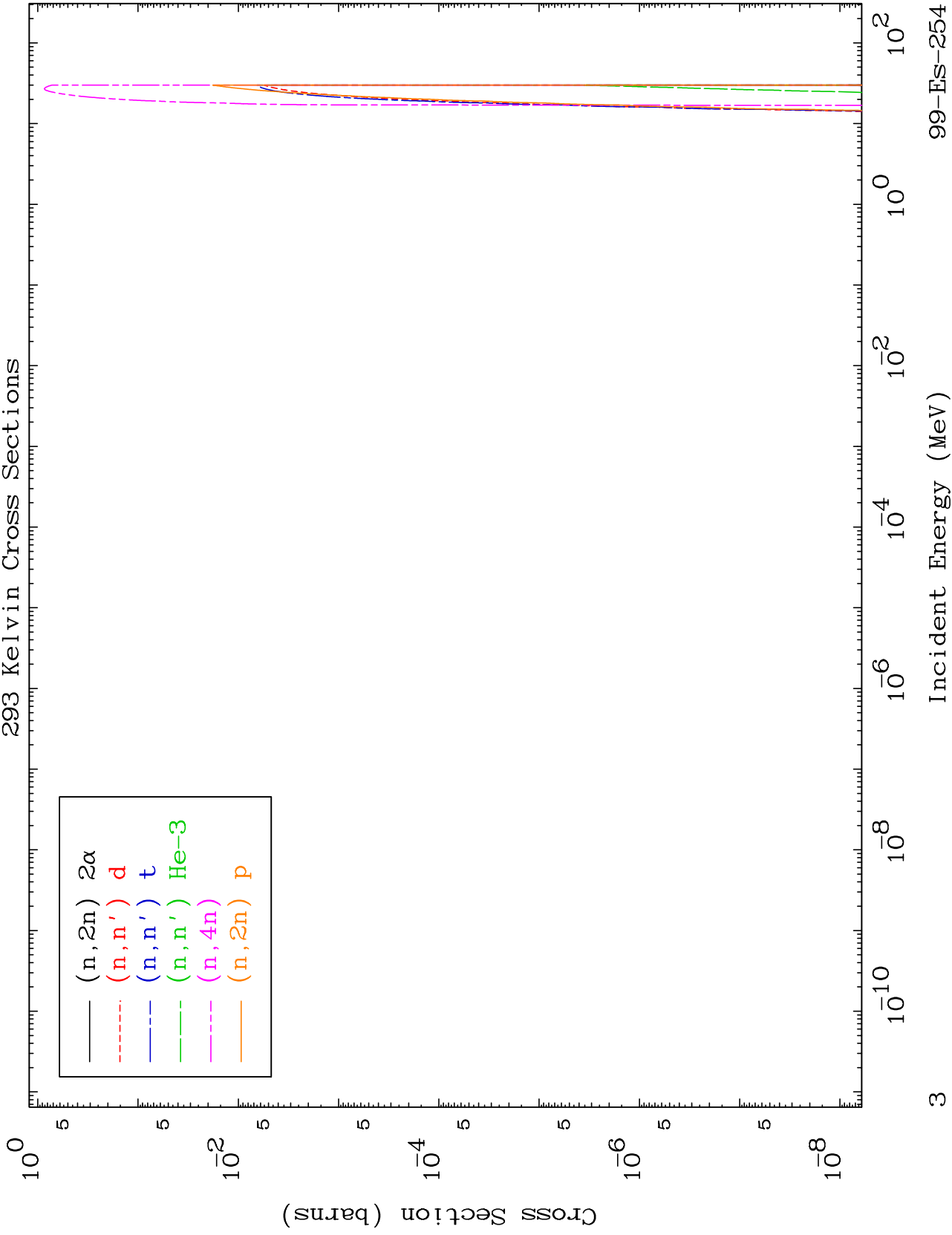
99-Es-254



MAT 9914

Neutron Production  
293 Kelvin Cross Sections

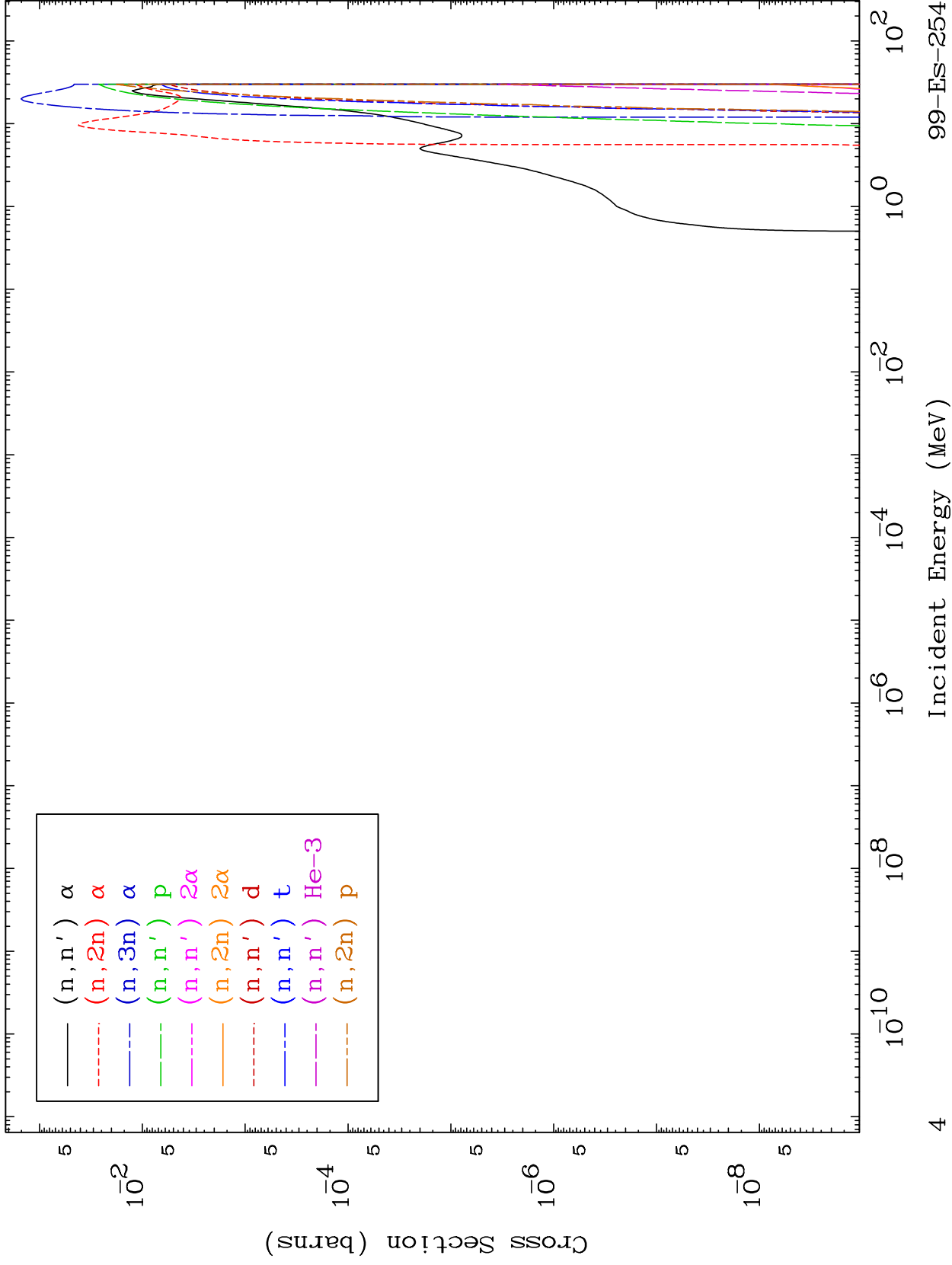
99-Es-254



MAT 9914

Charged Particle  
293 Kelvin Cross Sections

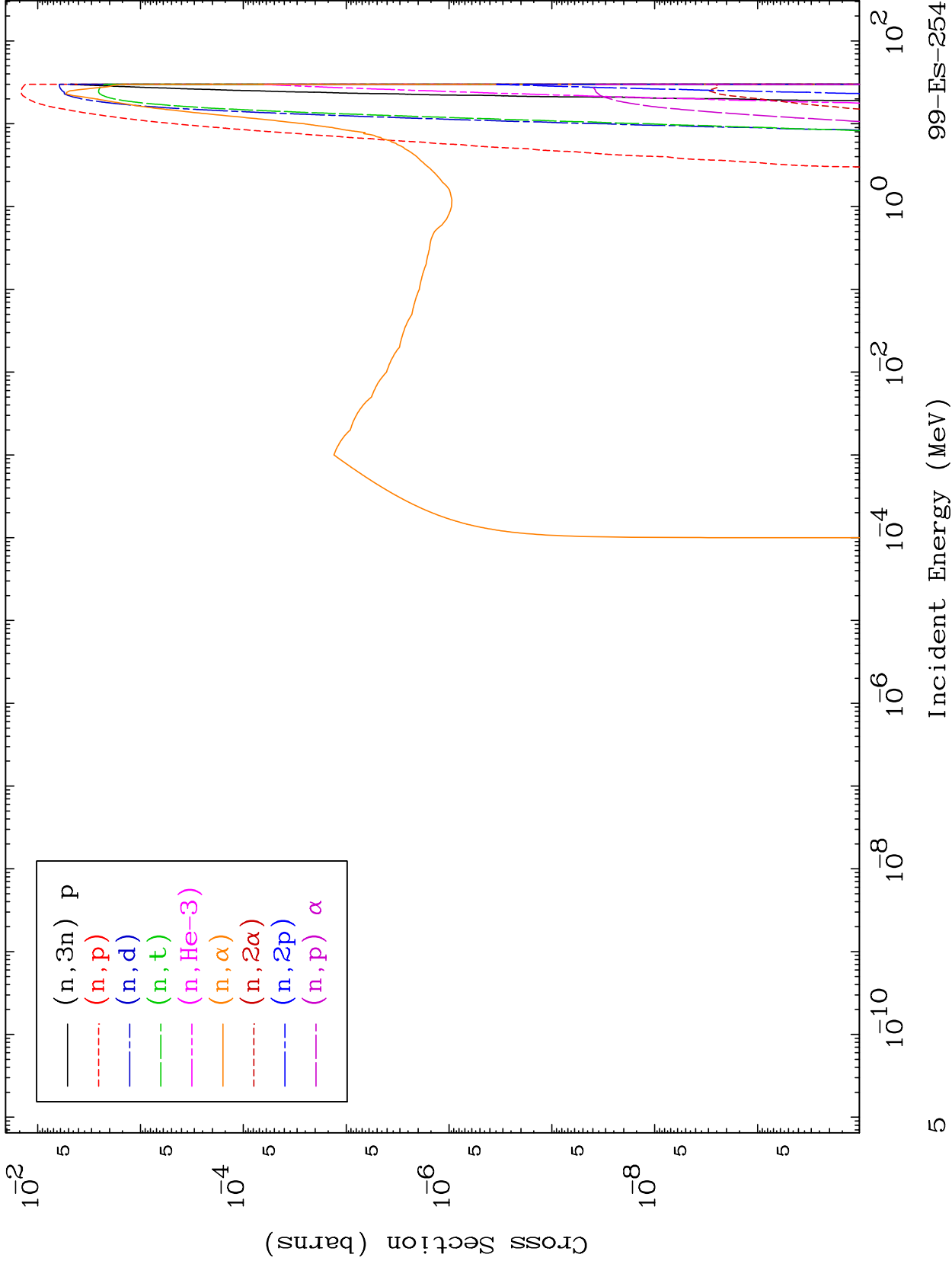
99-Es-254



MAT 9914

Charged Particle  
293 Kelvin Cross Sections

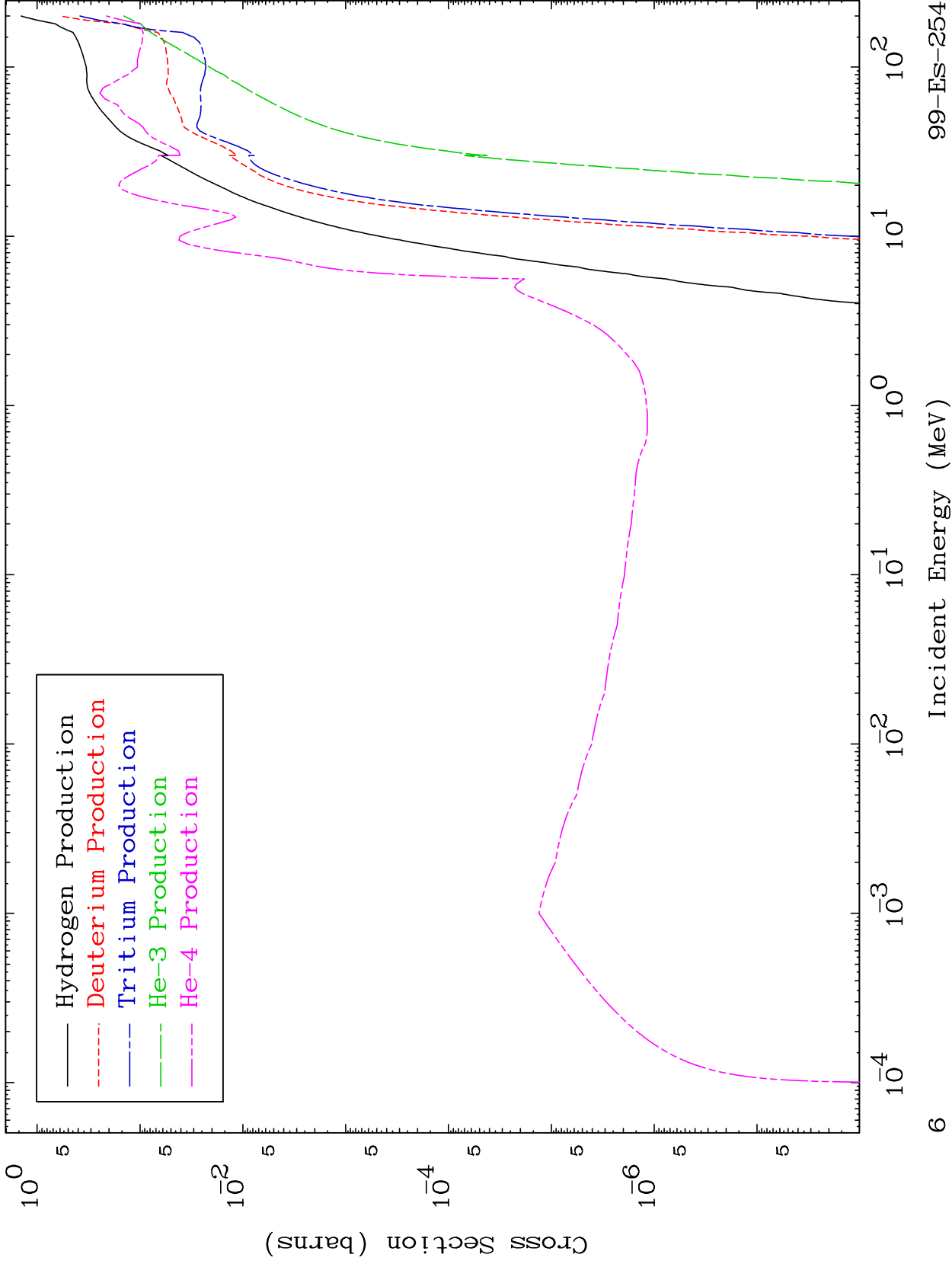
99-Es-254



MAT 9914

Particle Production  
293 Kelvin Cross Sections

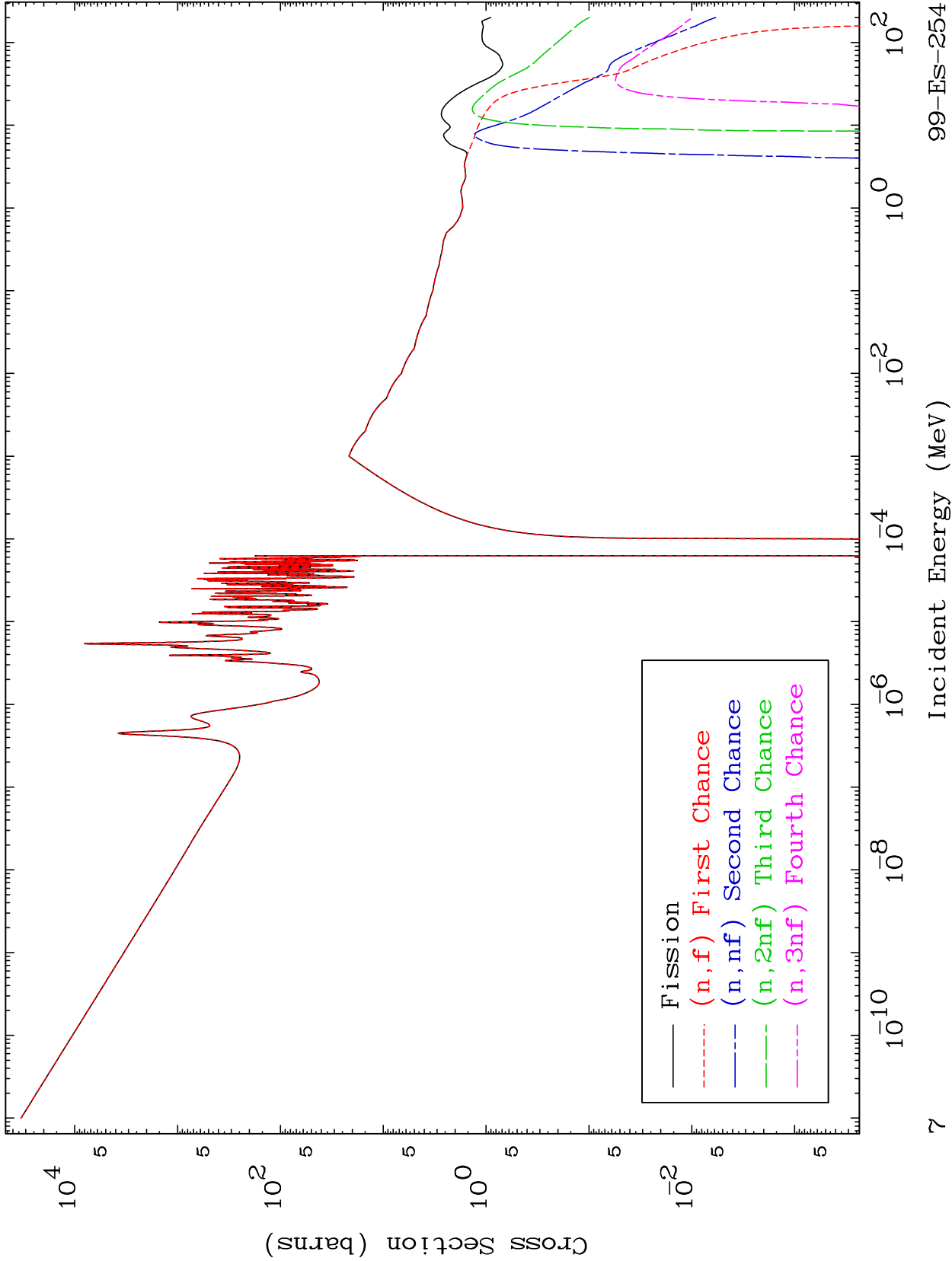
99-Es-254



MAT 9914

Fission  
293 Kelvin Cross Sections

99-Es-254



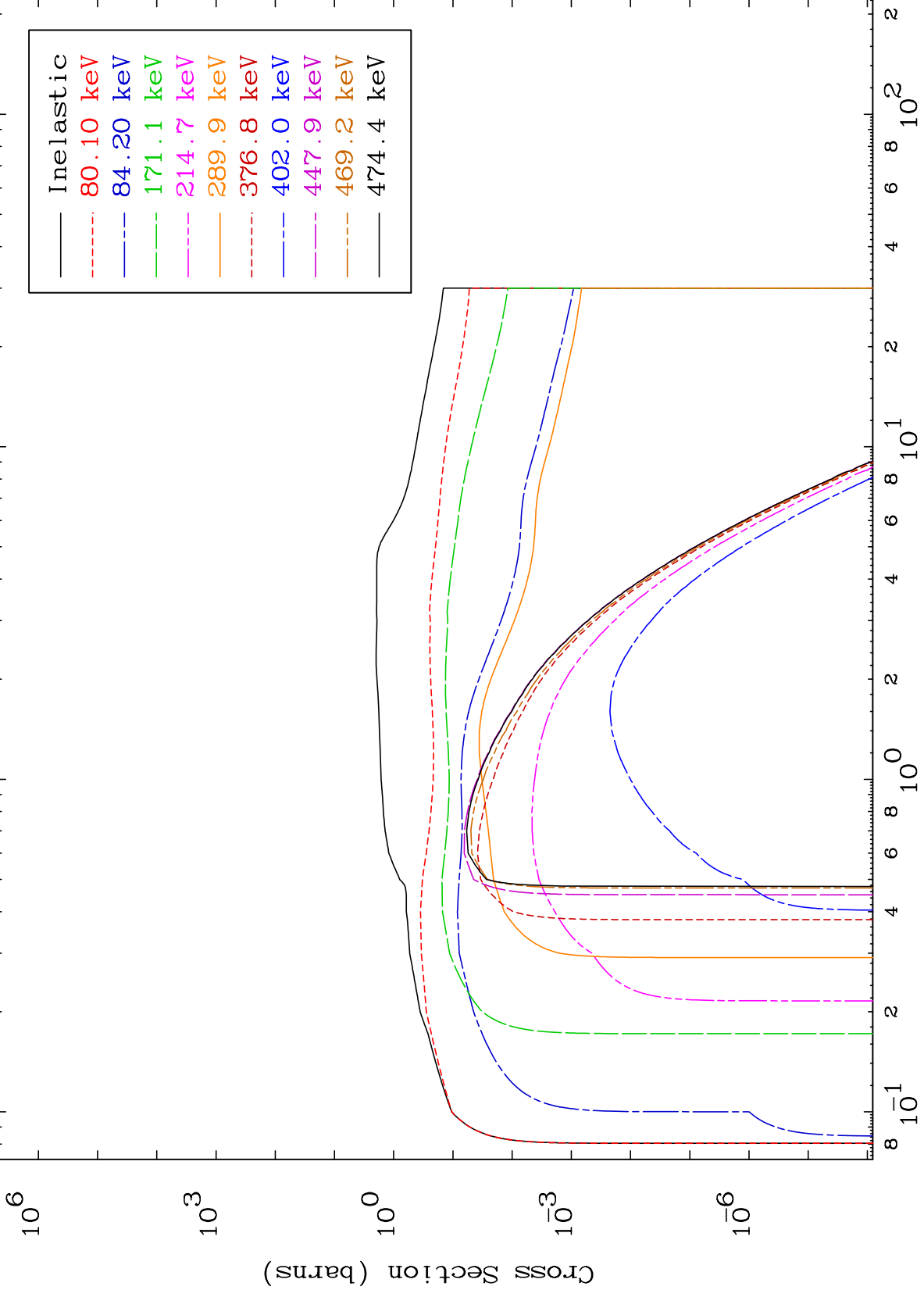


MAT 9914

(n,n') Level

99-Es-254

293 Kelvin Cross Sections



8

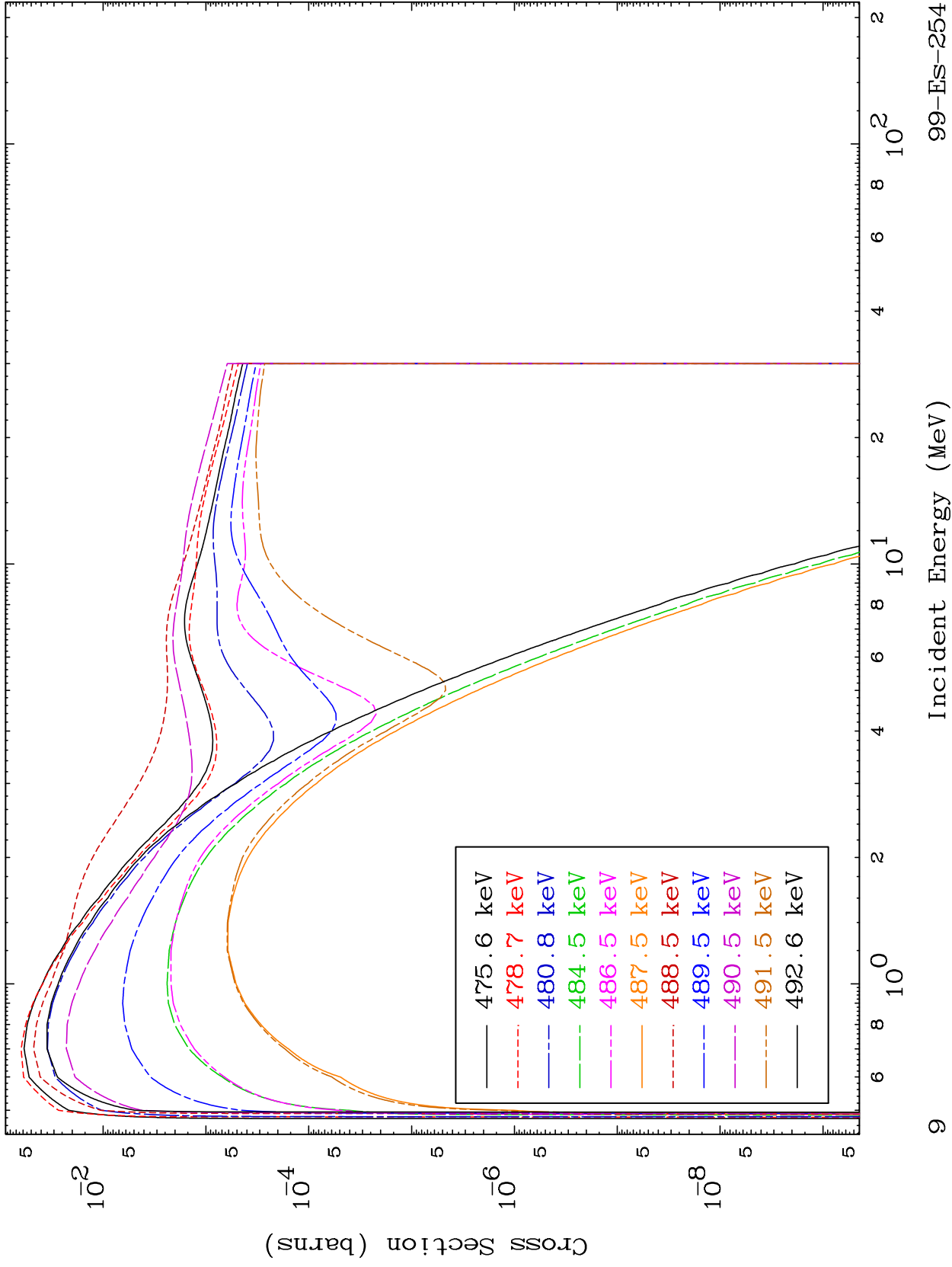
Incident Energy (MeV)

99-Es-254

MAT 9914

(n,n') Level  
293 Kelvin Cross Sections

99-Es-254

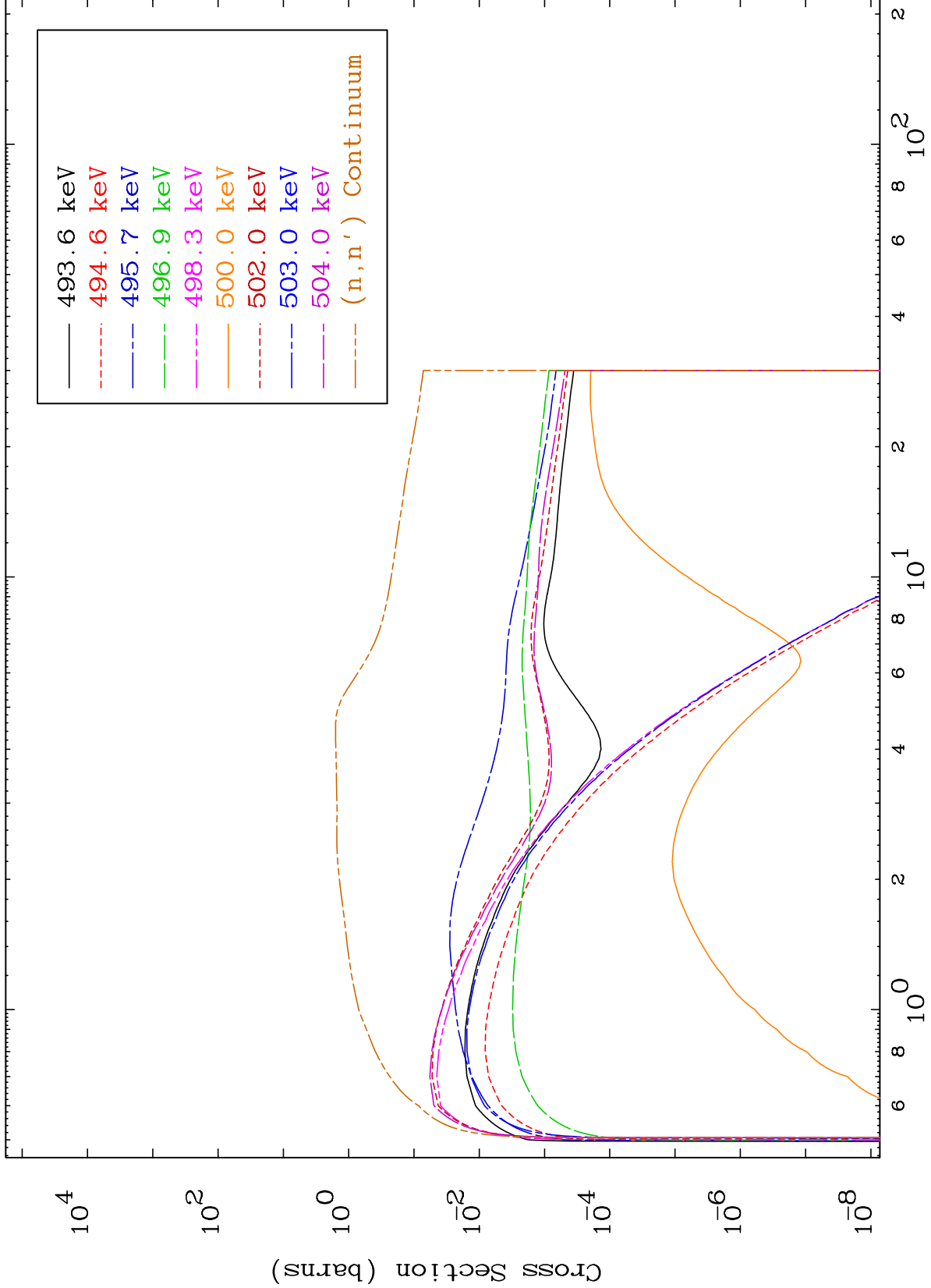


MAT 9914

(n,n') Level

99-Es-254

293 Kelvin Cross Sections



10

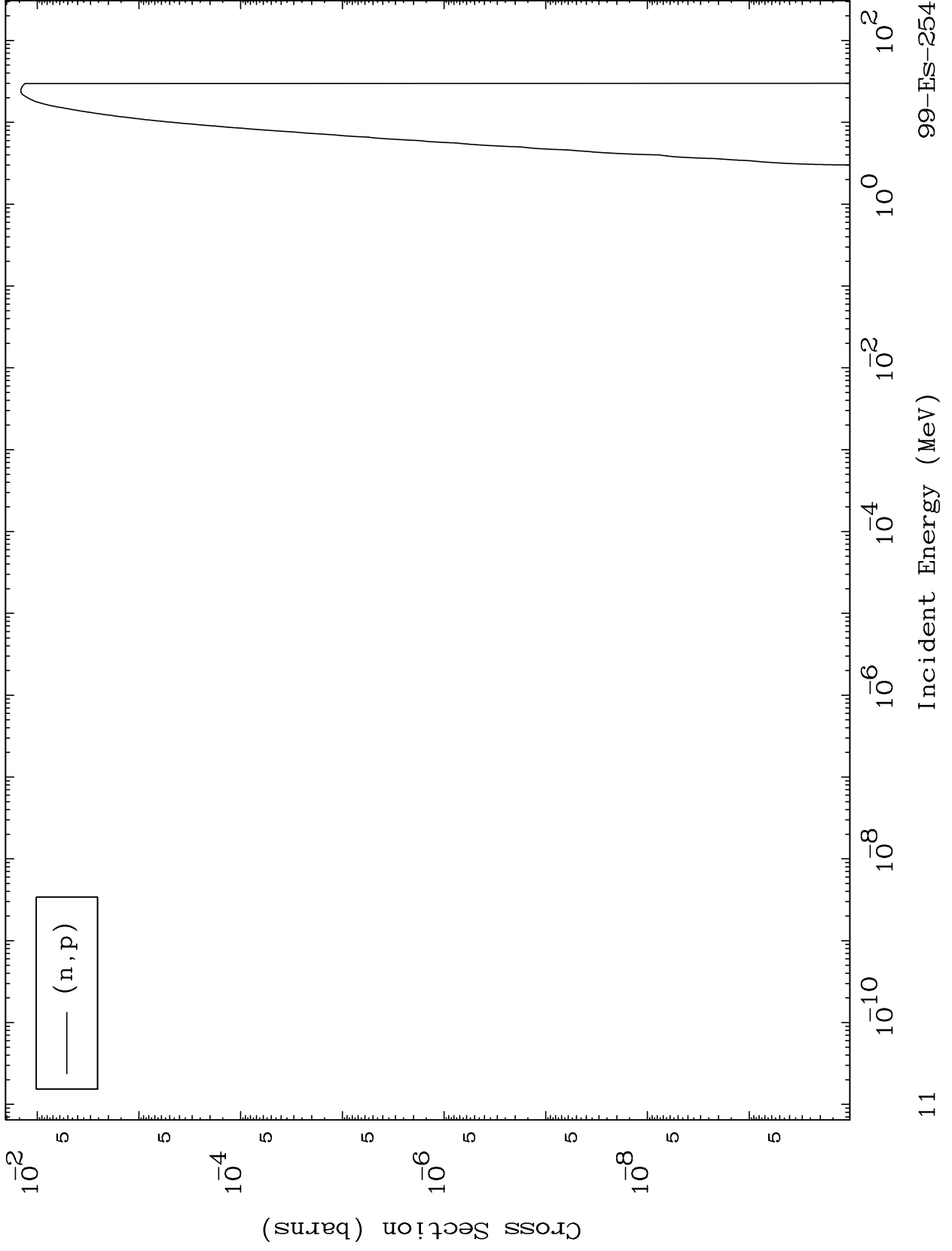
Incident Energy (MeV)

99-Es-254

MAT 9914

(n,p) Levels  
293 Kelvin Cross Sections

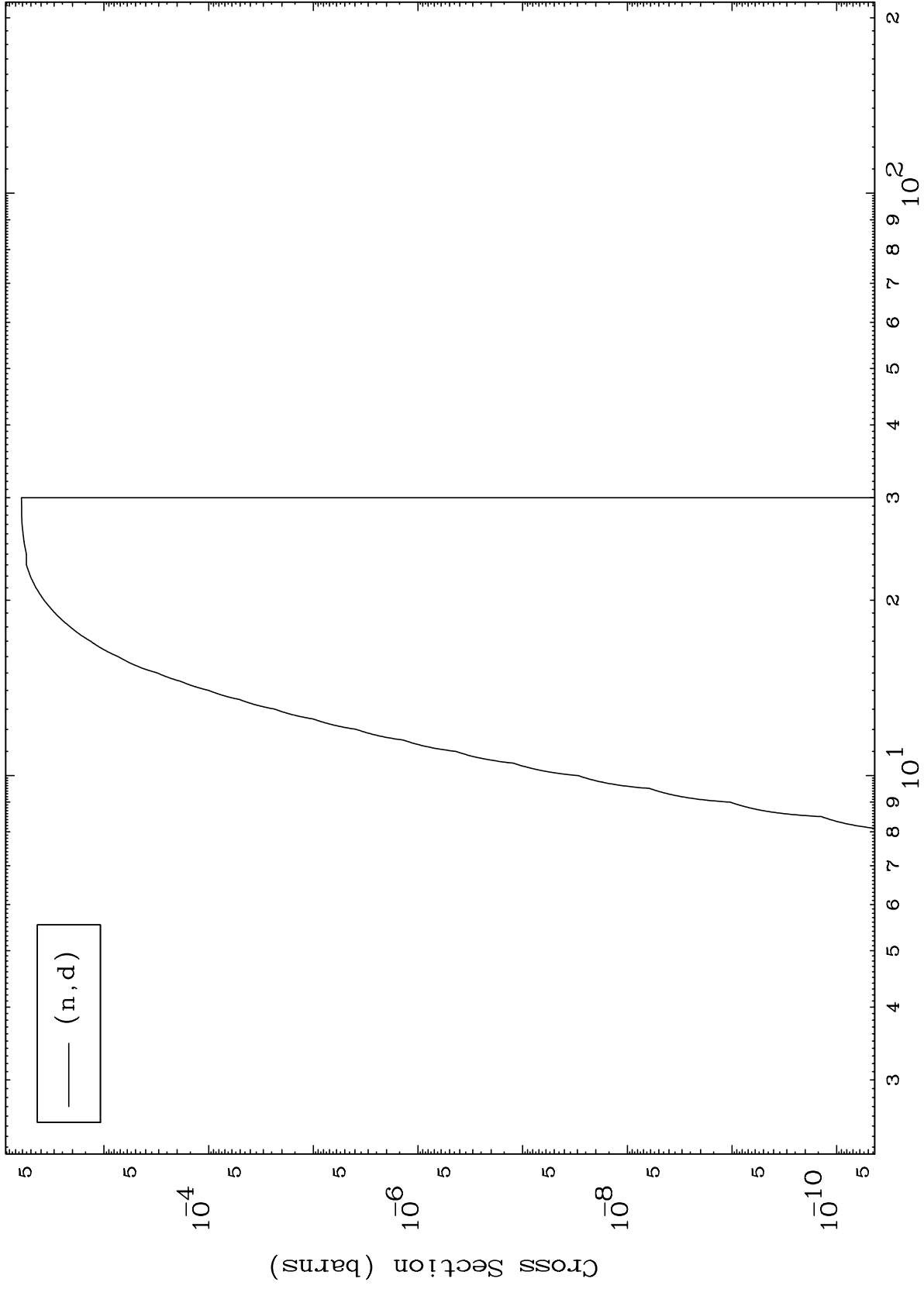
99-Es-254



MAT 9914

(n,d) Levels  
293 Kelvin Cross Sections

99-Es-254



12

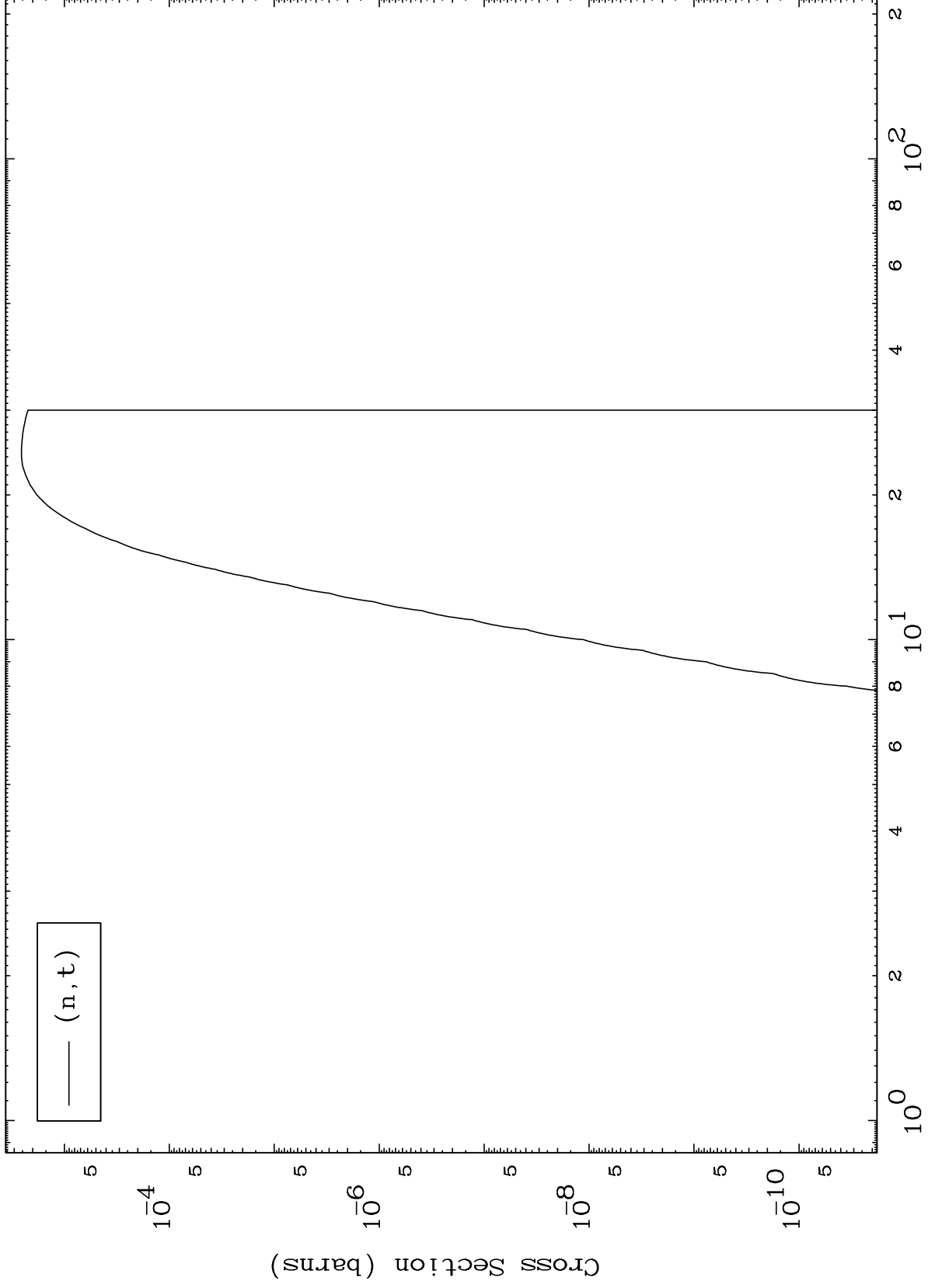
Incident Energy (MeV)

99-Es-254

MAT 9914

(n,t) Levels  
293 Kelvin Cross Sections

99-Es-254



13

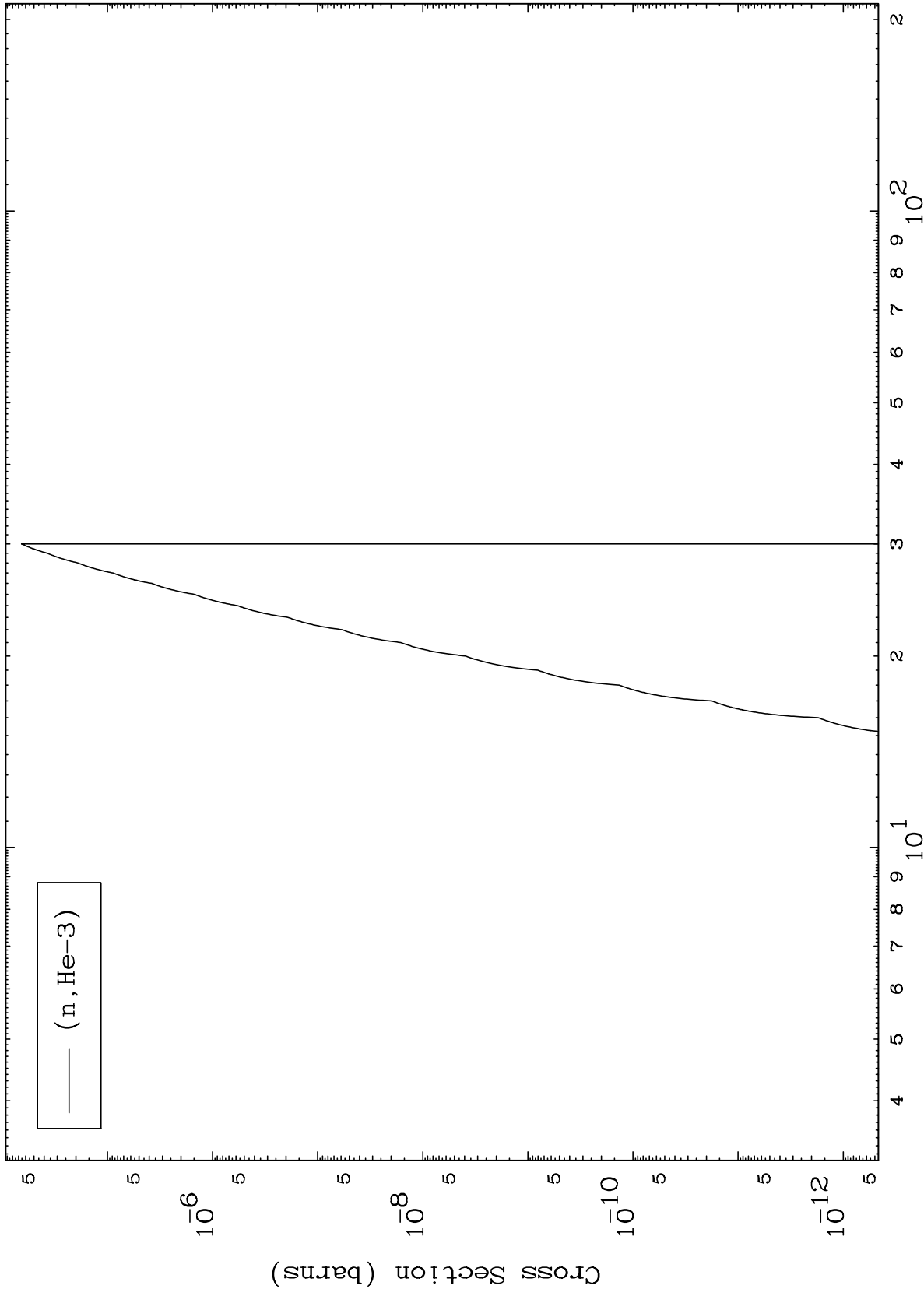
Incident Energy (MeV)

99-Es-254

MAT 9914

(n,He3) Levels  
293 Kelvin Cross Sections

99-Es-254



14

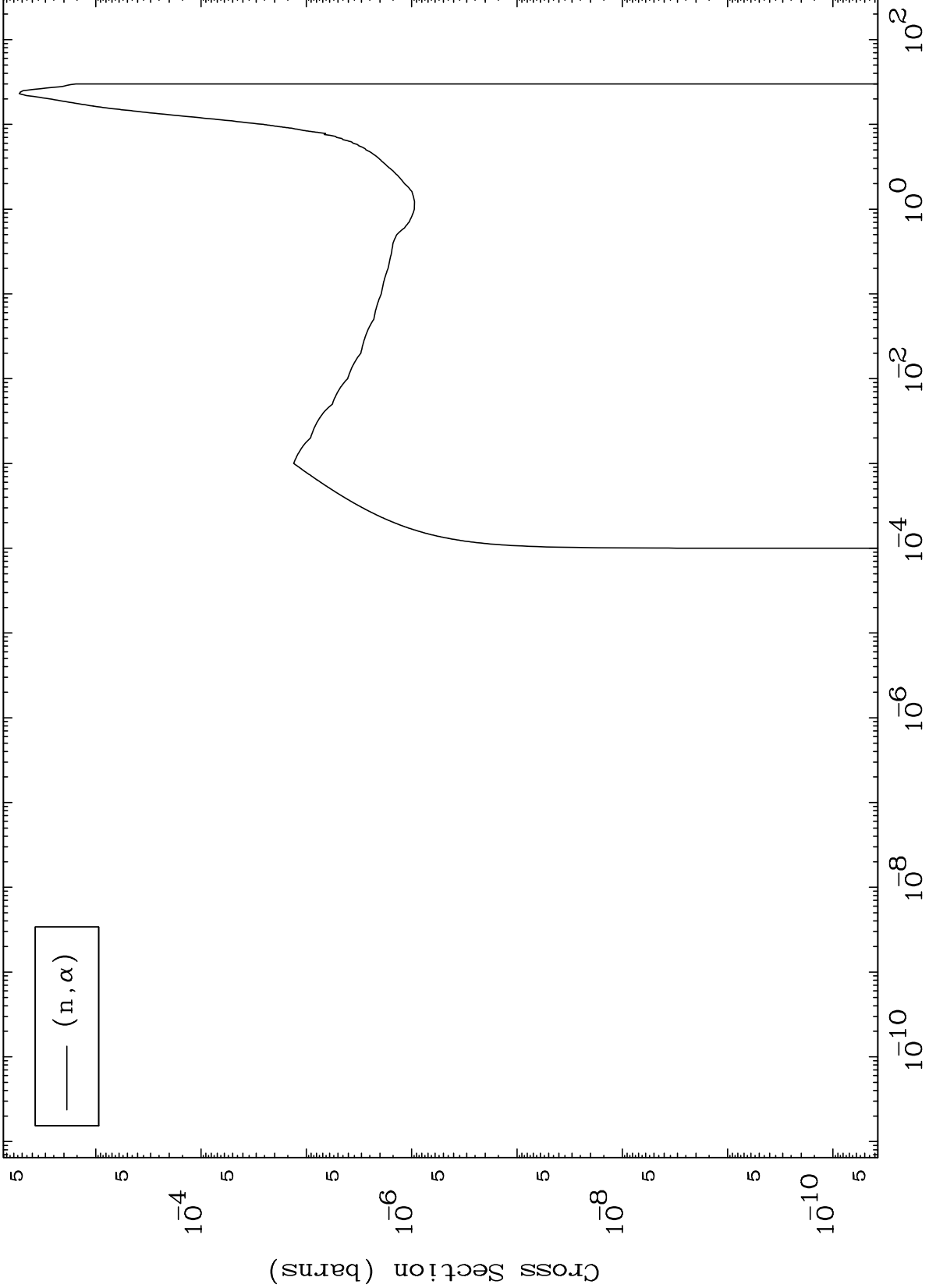
Incident Energy (MeV)

99-Es-254

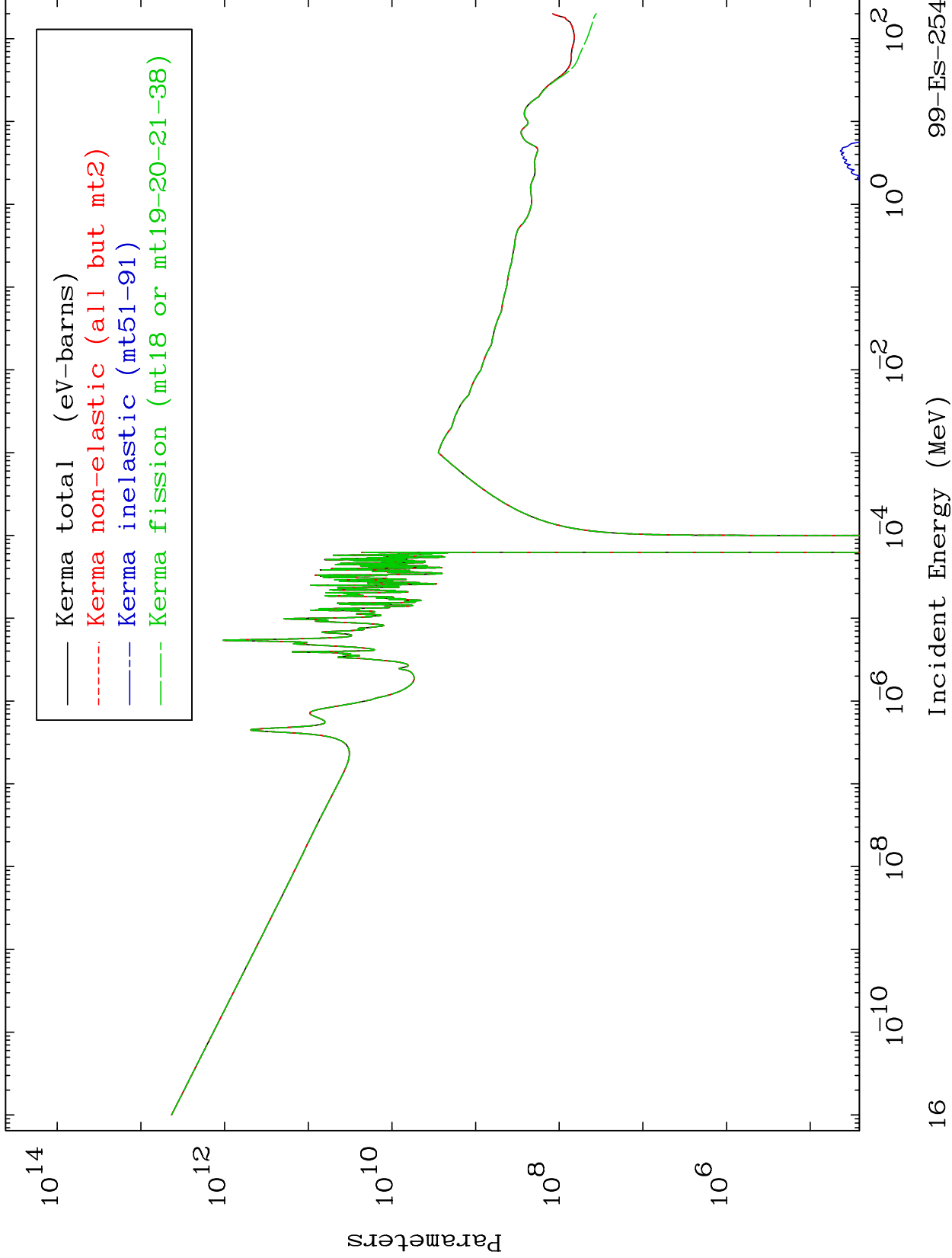
MAT 9914

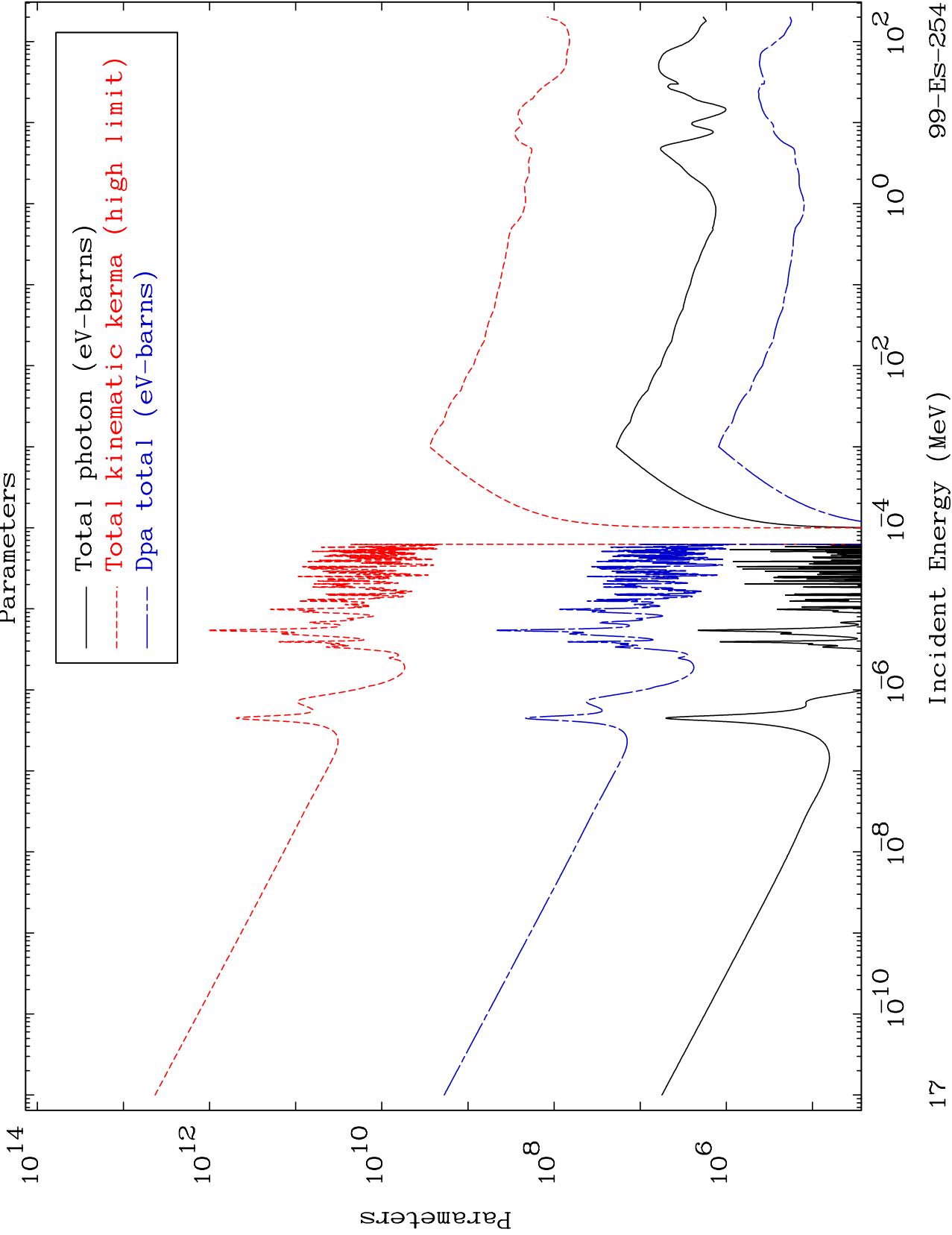
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

99-Es-254





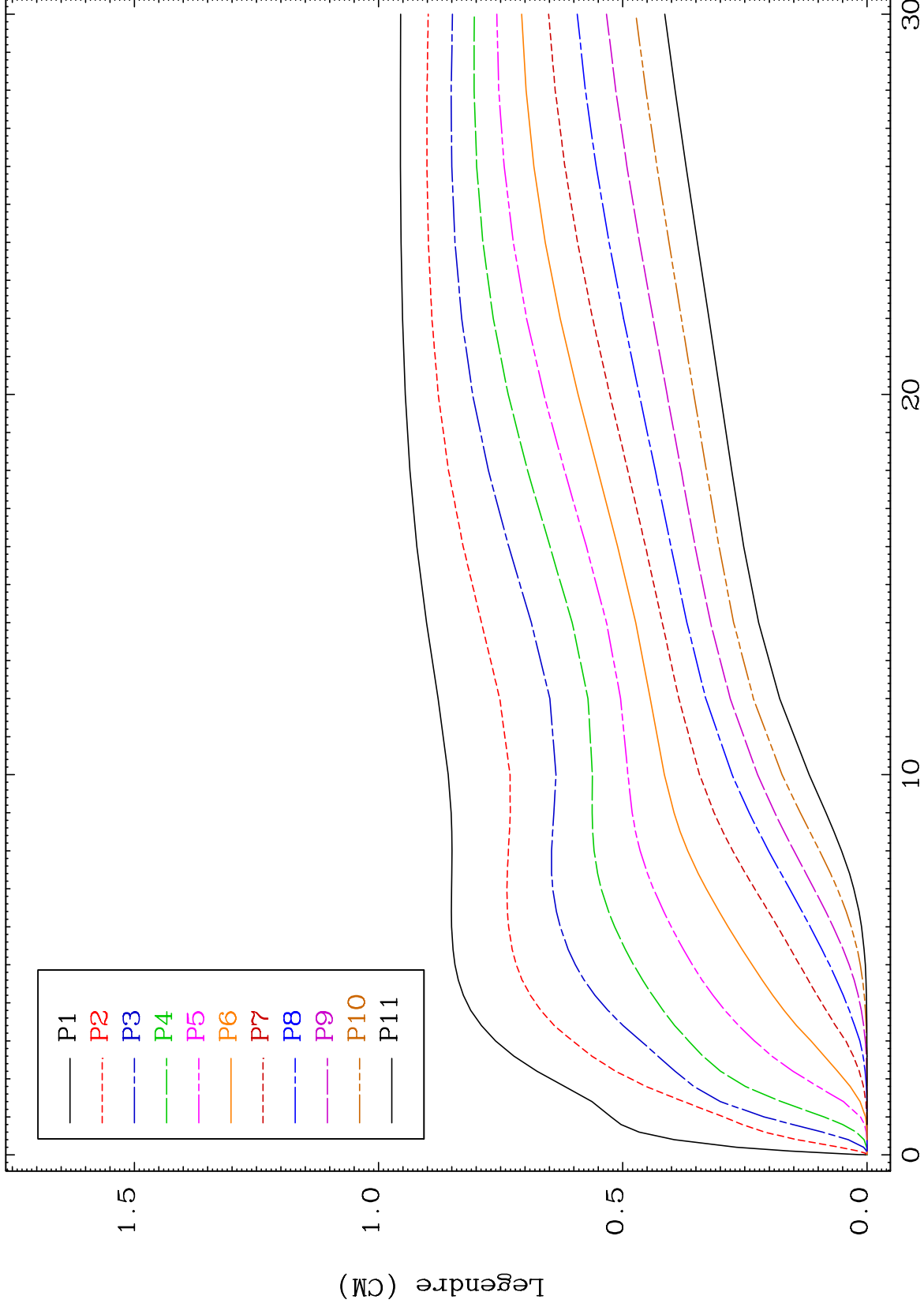




MAT 9914

Elastic Legendre Coefficients

99-Es-254



18

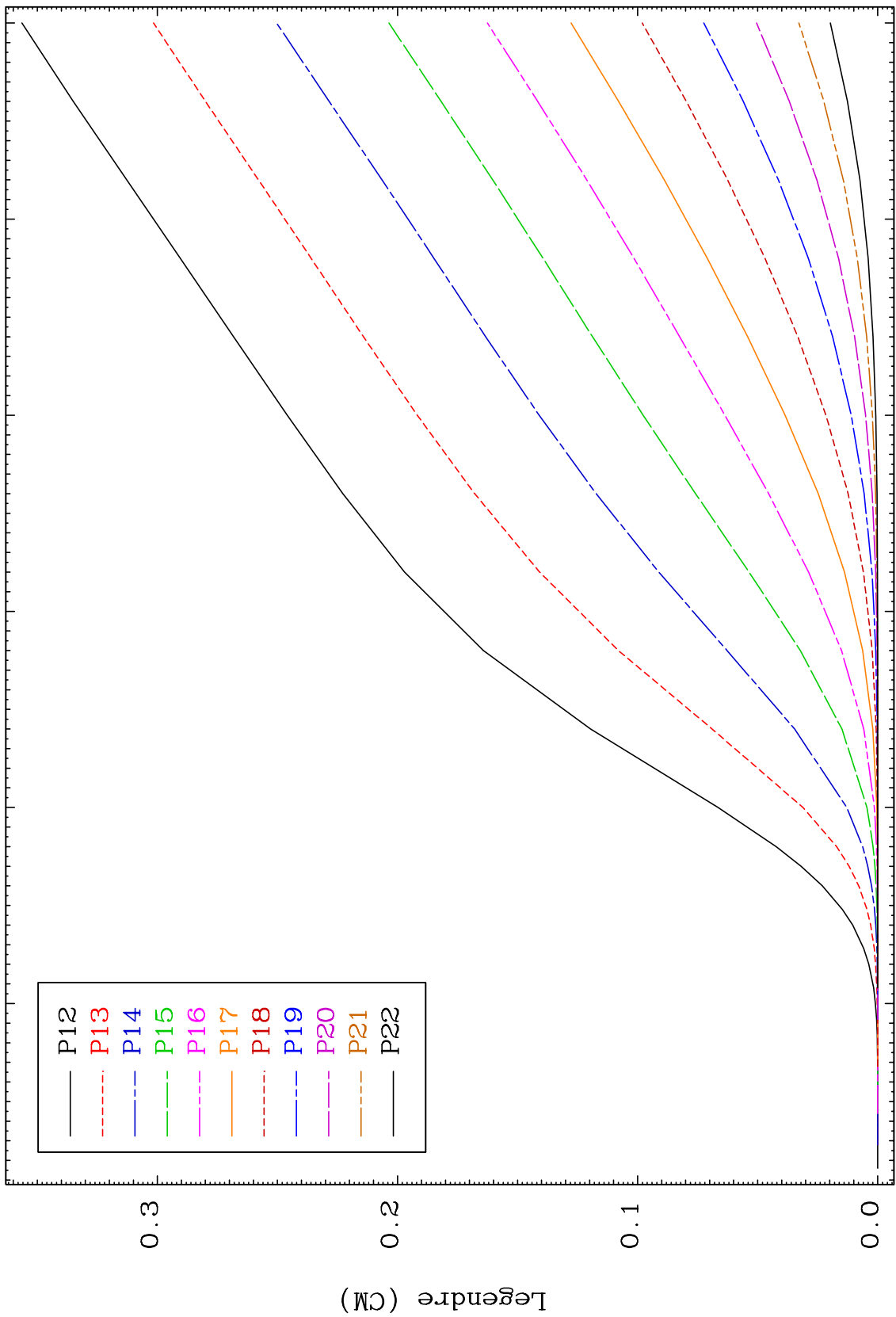
Incident Energy (MeV)

99-Es-254

MAT 9914

Elastic Legendre Coefficients

99-Es-254



19

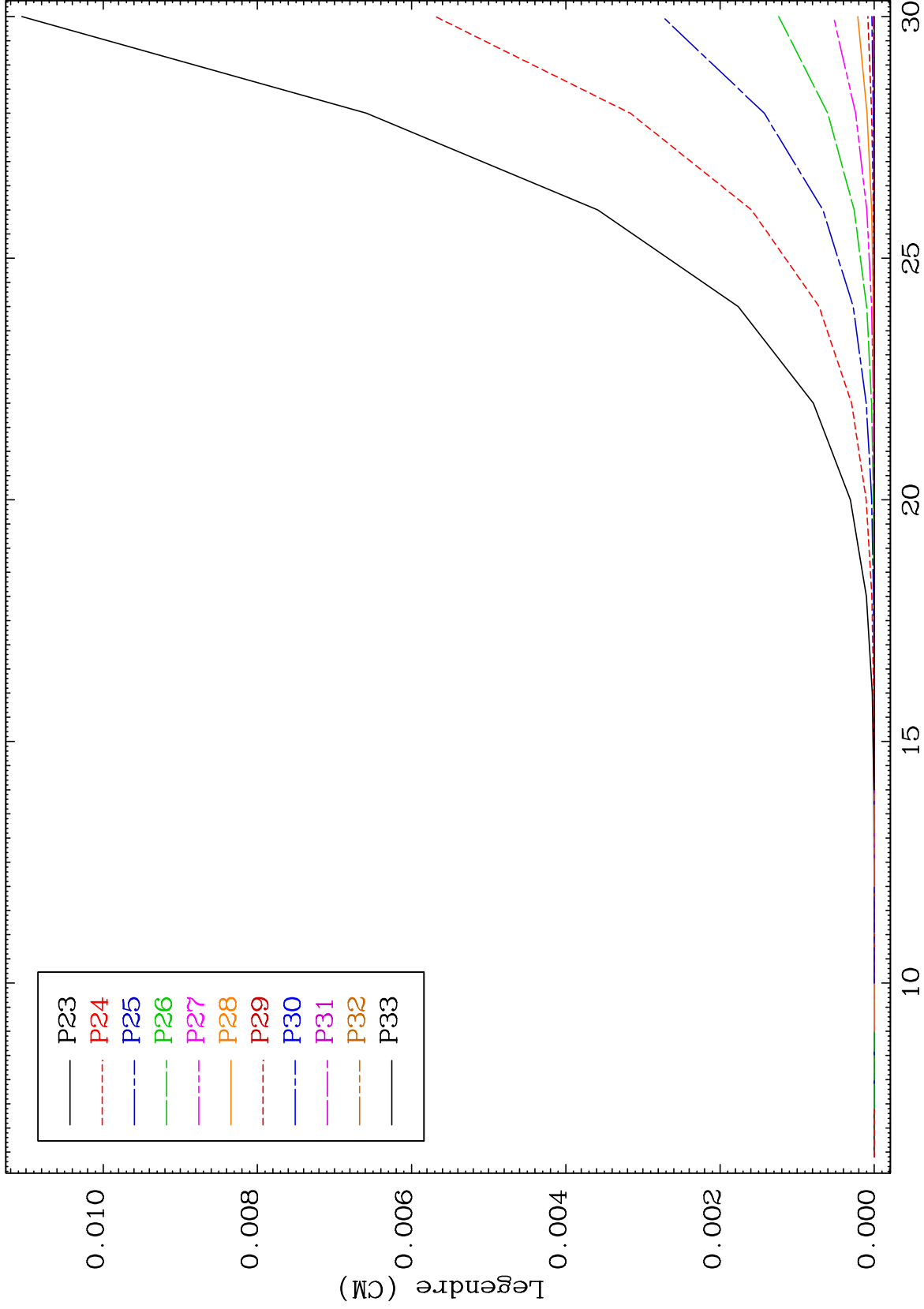
Incident Energy (MeV)

99-Es-254

MAT 9914

Elastic Legendre Coefficients

99-Es-254



20

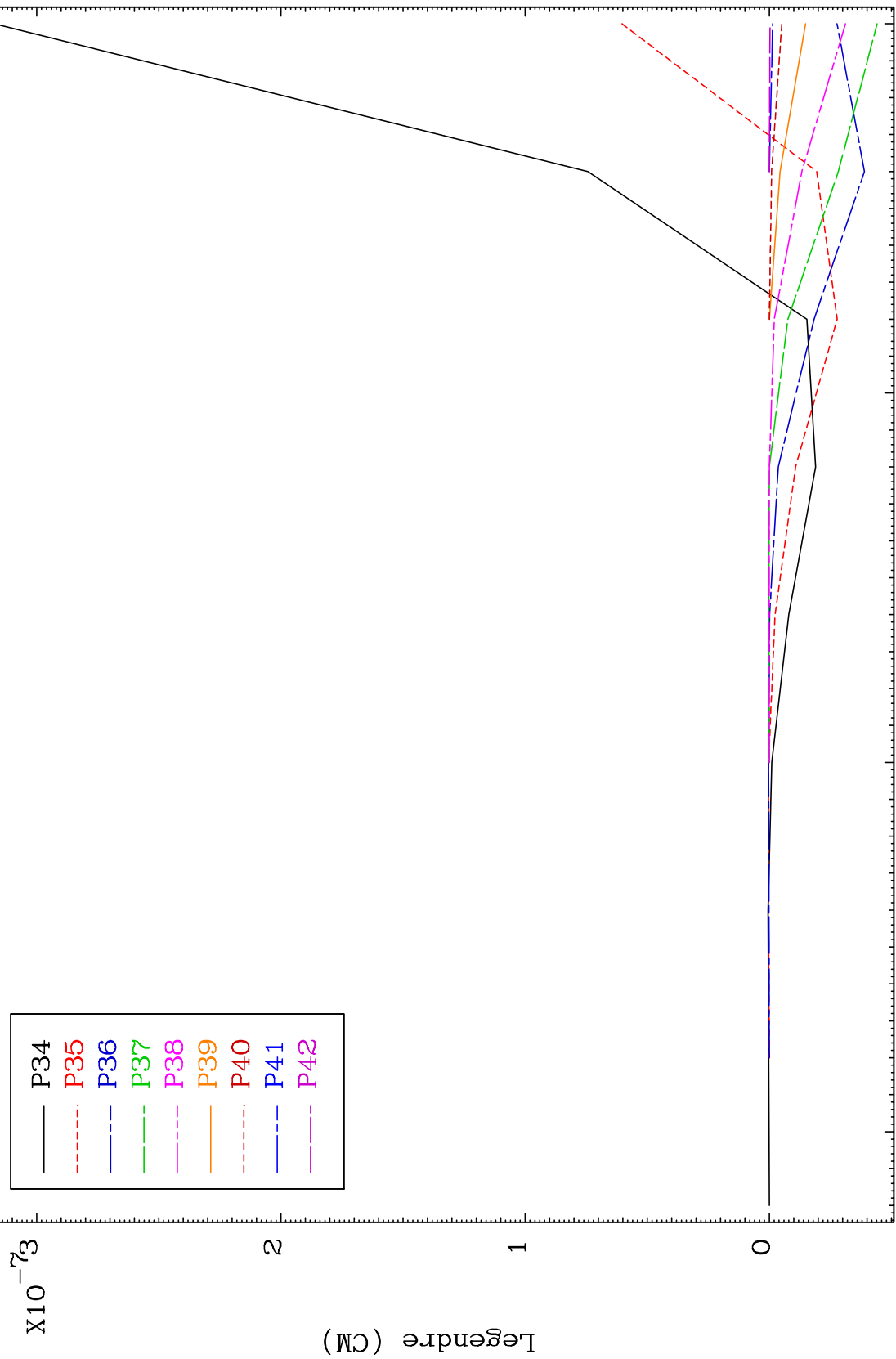
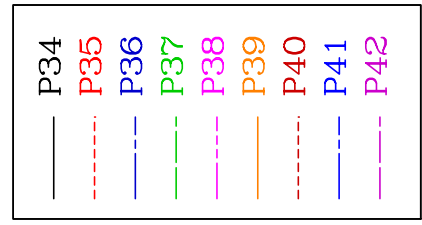
Incident Energy (MeV)

99-Es-254

MAT 9914

Elastic Legendre Coefficients

99-Es-254

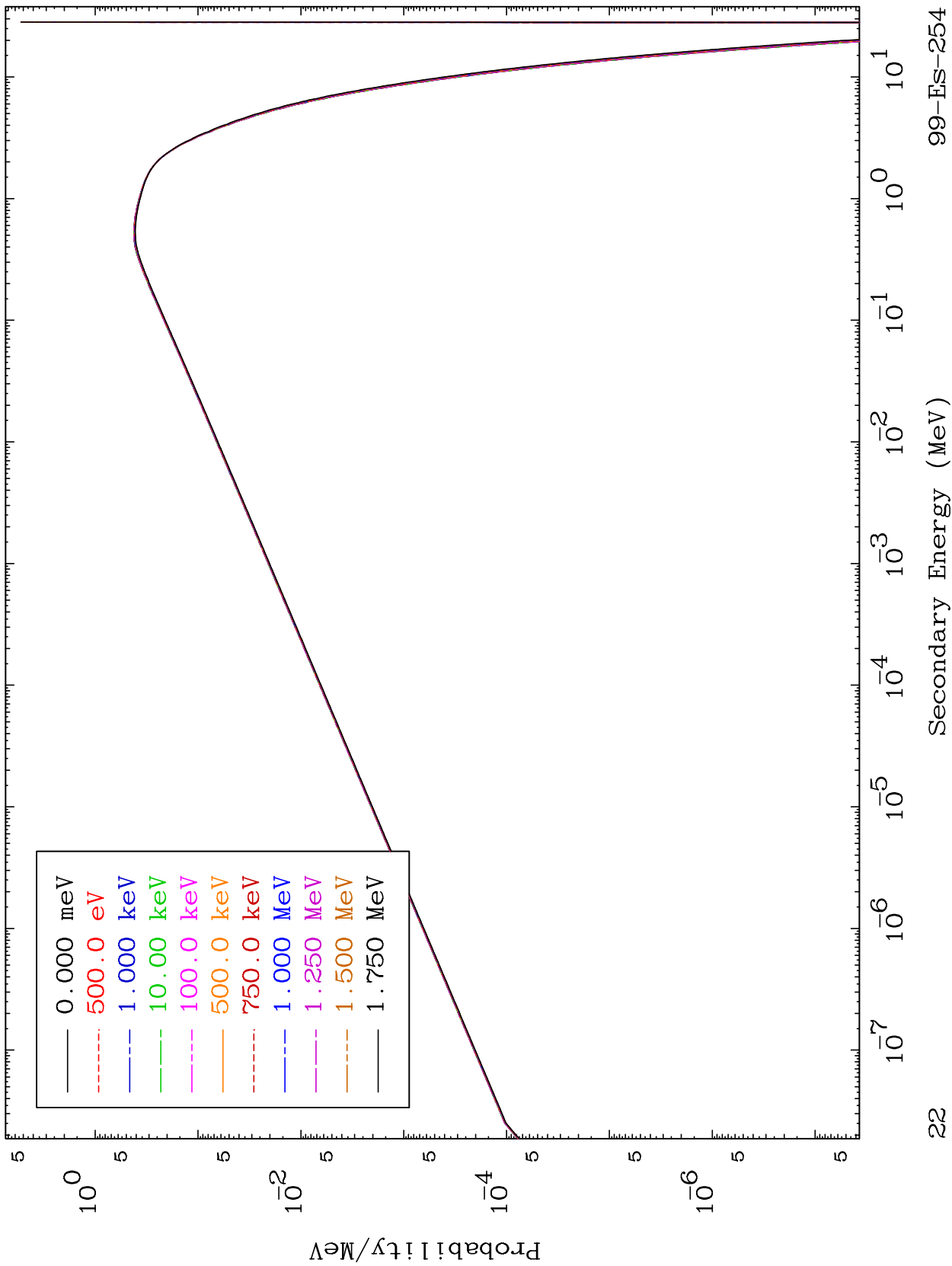


21

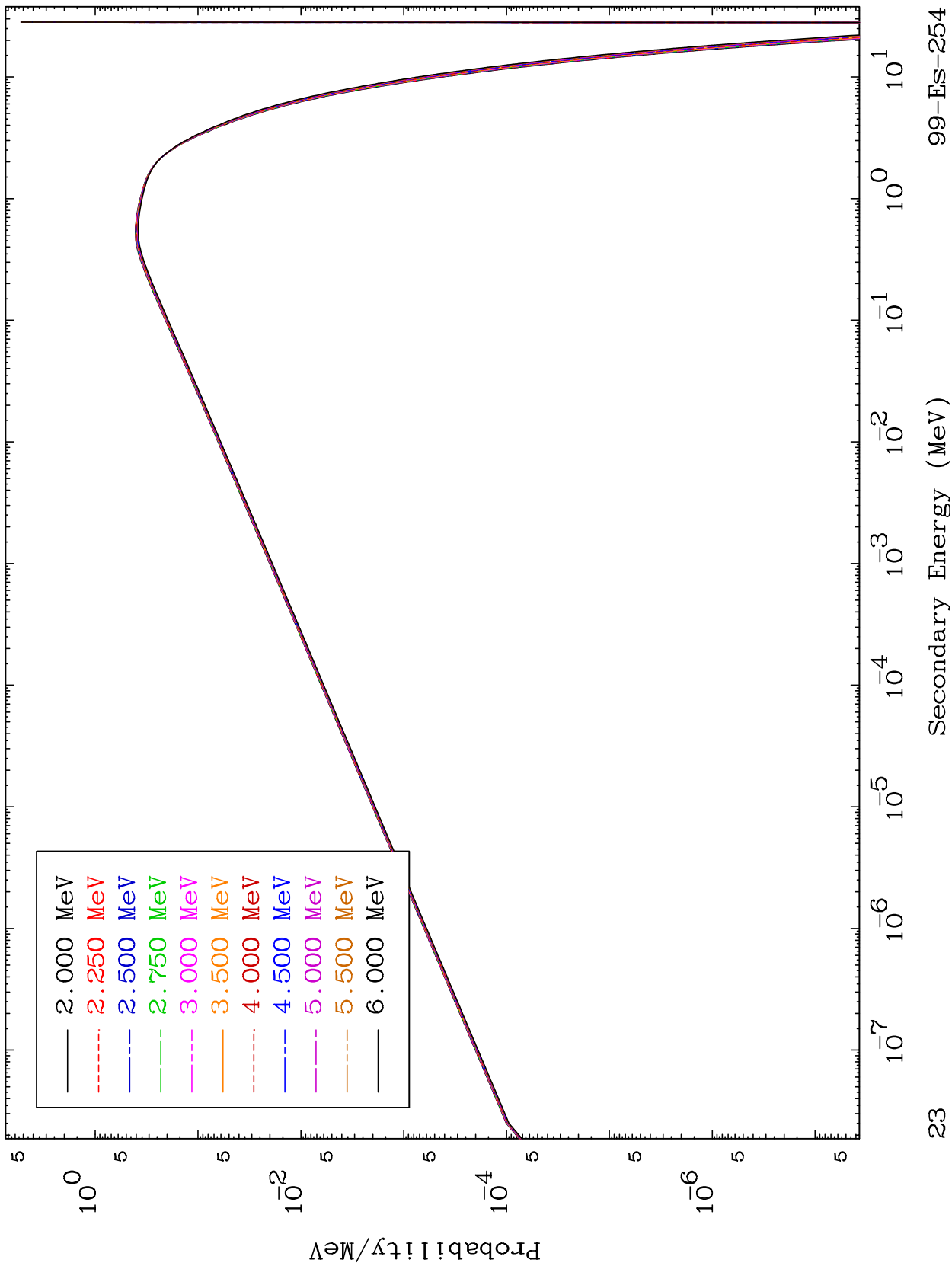
Incident Energy (MeV)

99-Es-254

Fission Energy Distribution

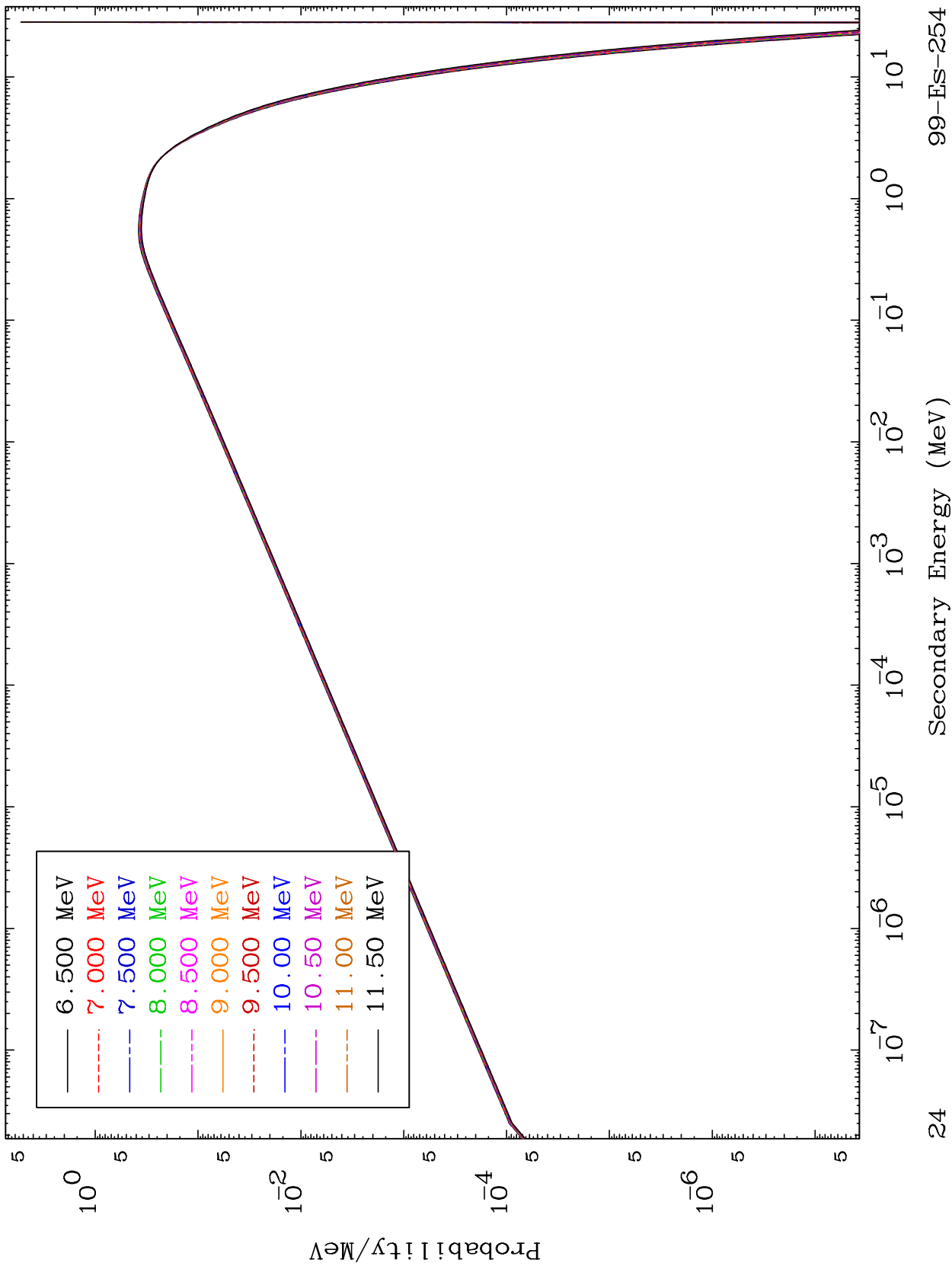


Fission Energy Distribution

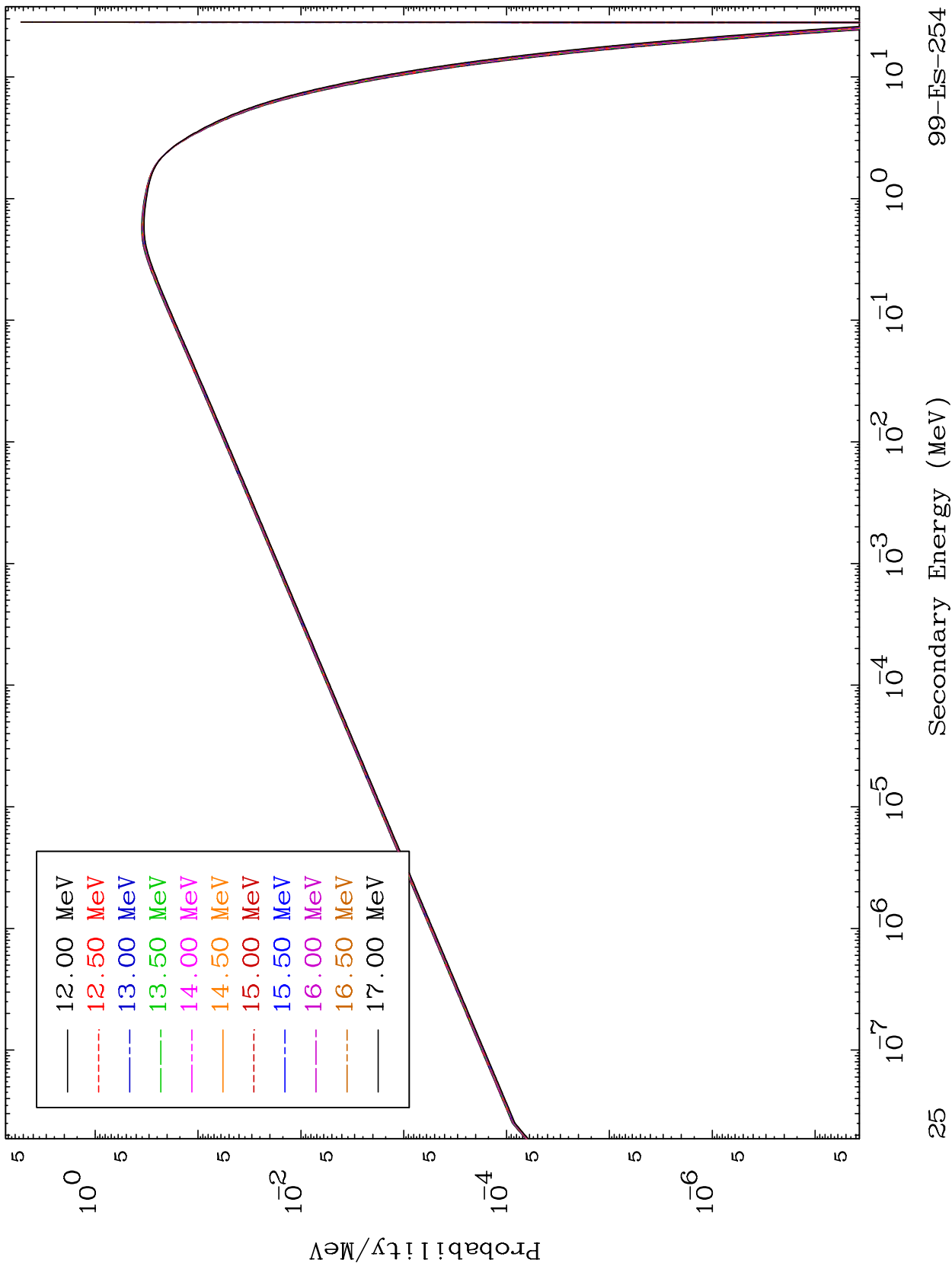




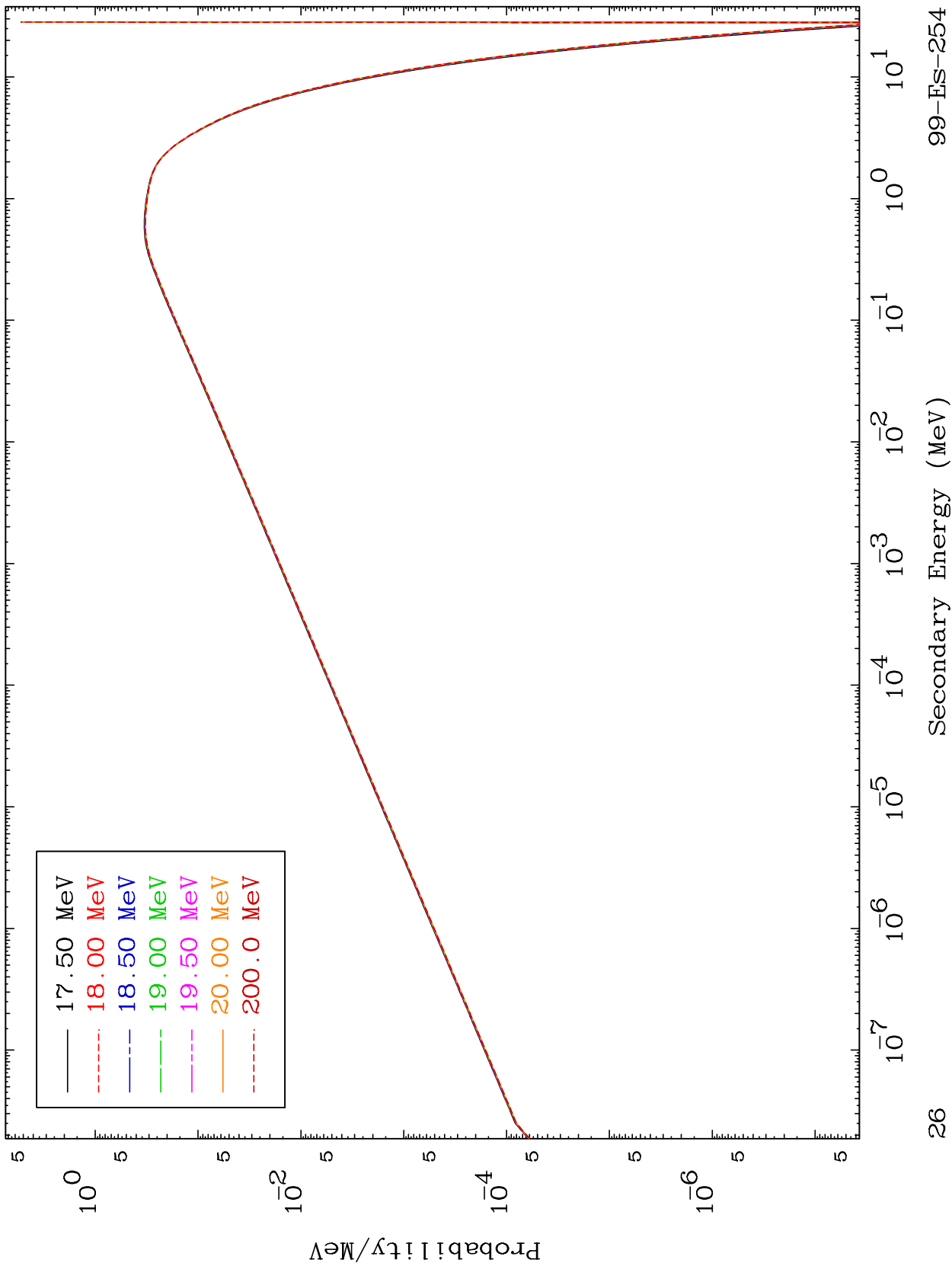
Fission  
Energy Distribution



Fission Energy Distribution



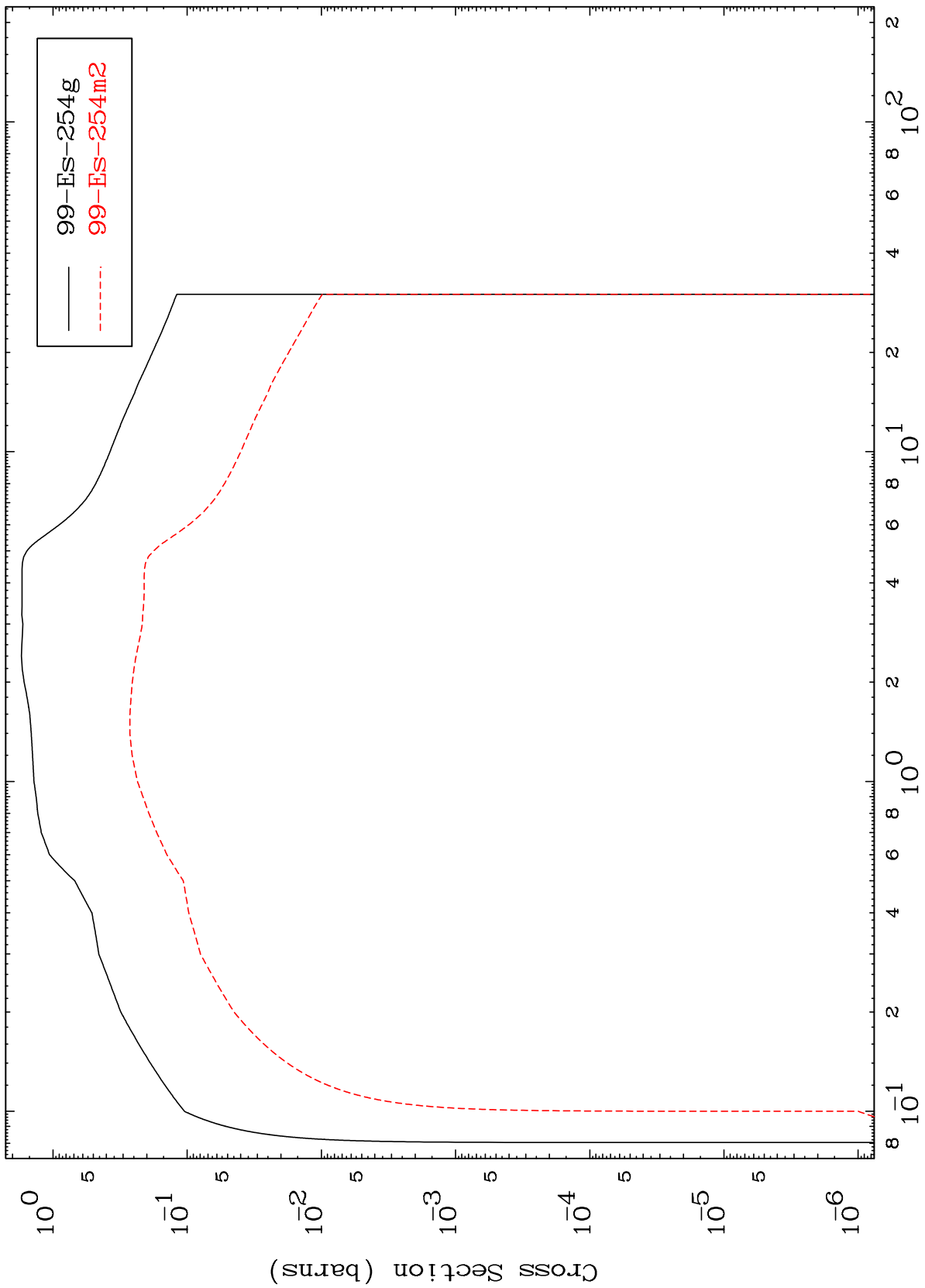
Fission  
Energy Distribution



MAT 9914

99-Es-254

Inelastic  
Radionuclide Production Cross Section

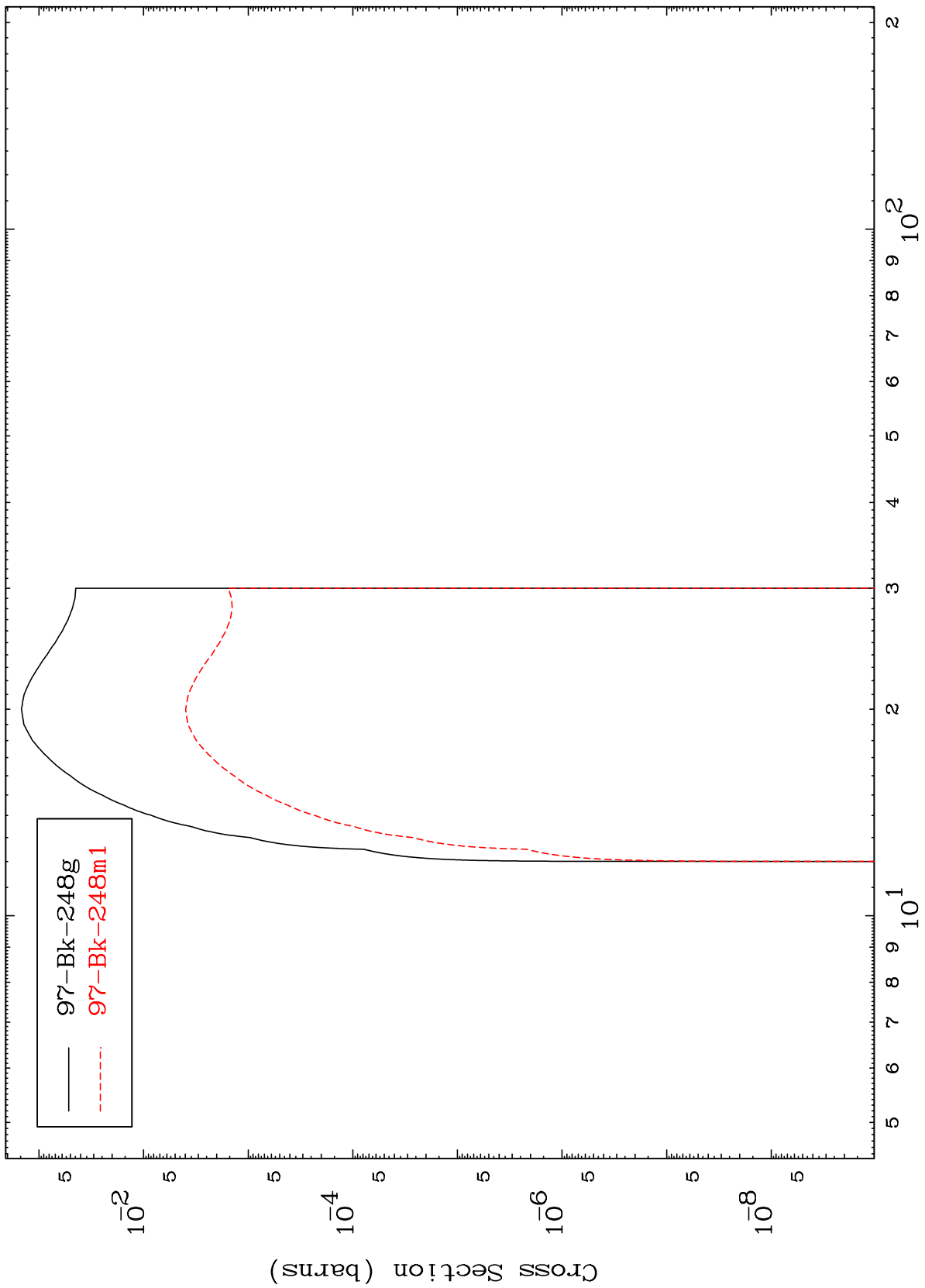


27

Incident Energy (MeV)

99-Es-254

Radionuclide Production Cross Section

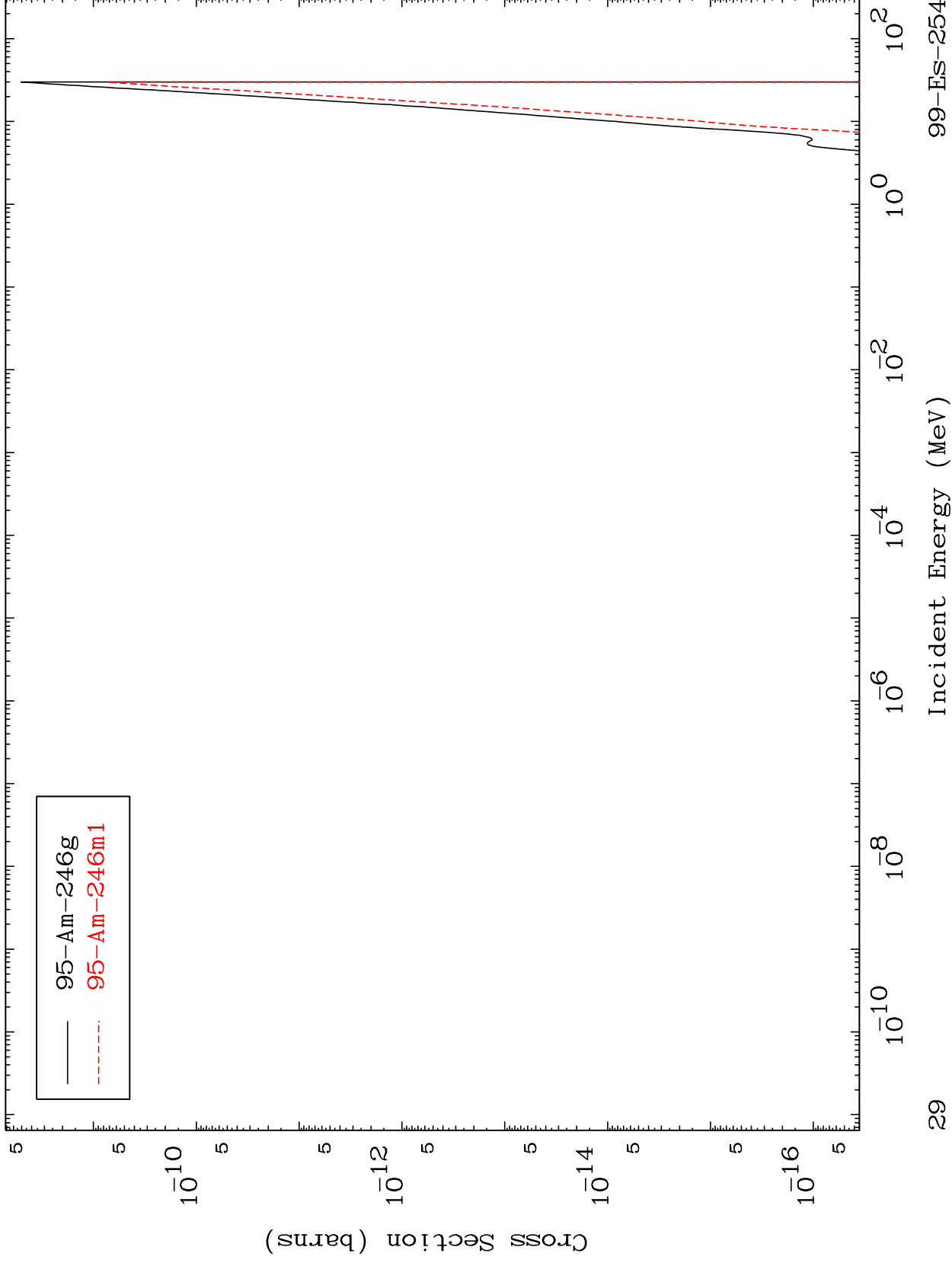


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(n,n') 2 $\alpha$

99-Es-254

Radionuclide Production Cross Section



29

Incident Energy (MeV)

99-Es-254