

Program EVALPLOT  
(Version 2015-2)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

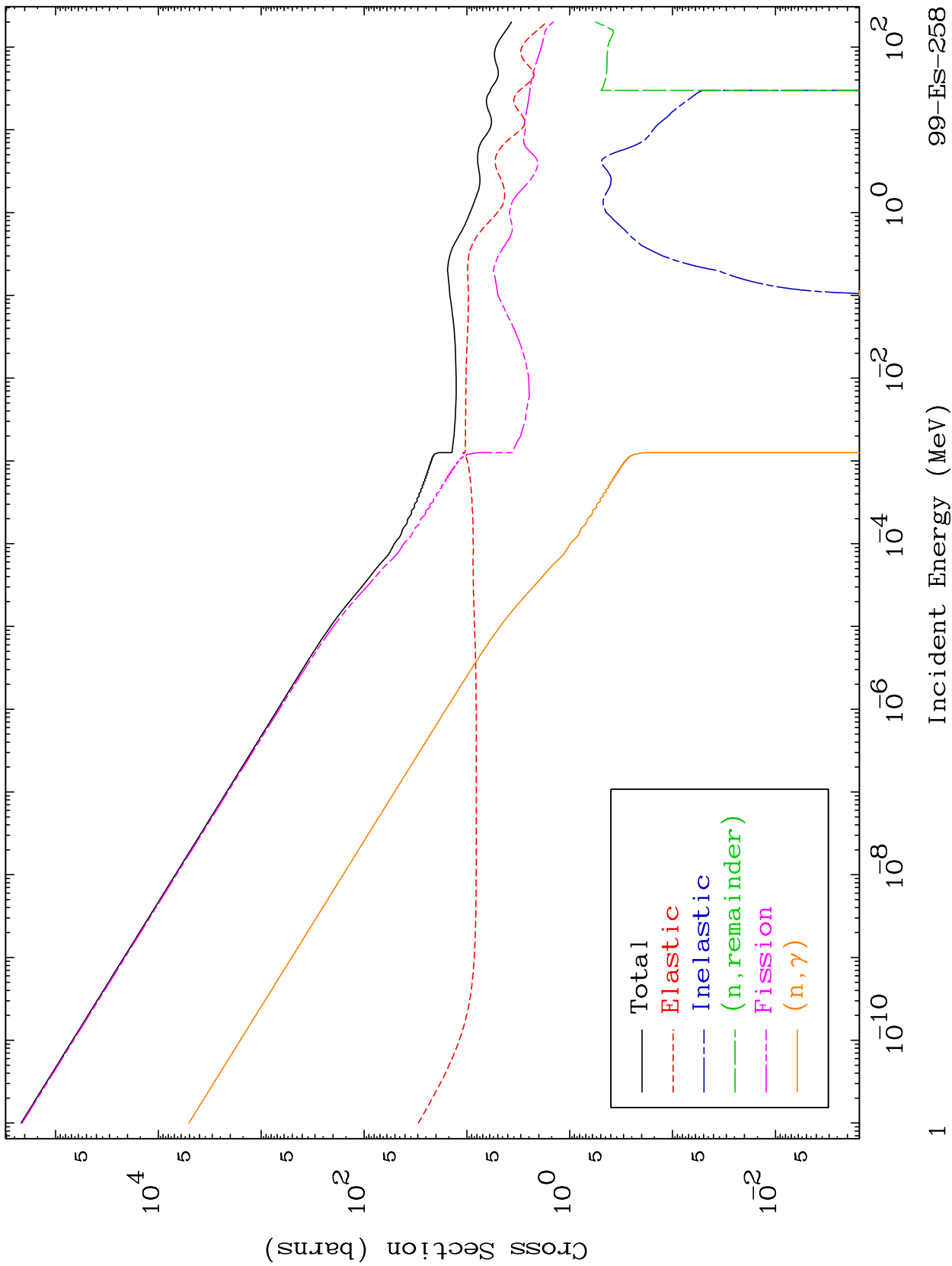
Press Mouse Button to Start

MAT 9918

Major

293 Kelvin Cross Sections

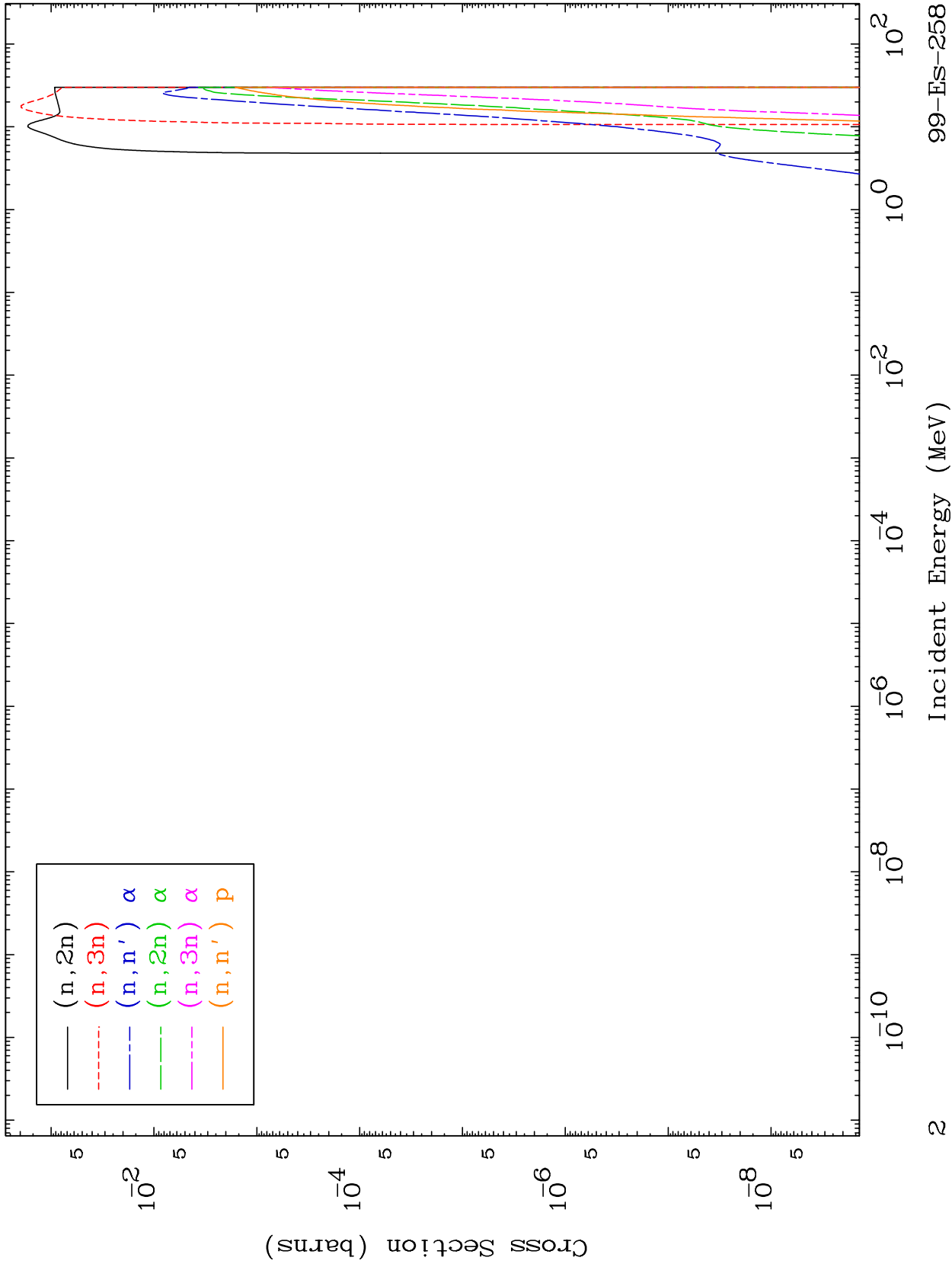
99-Es-258



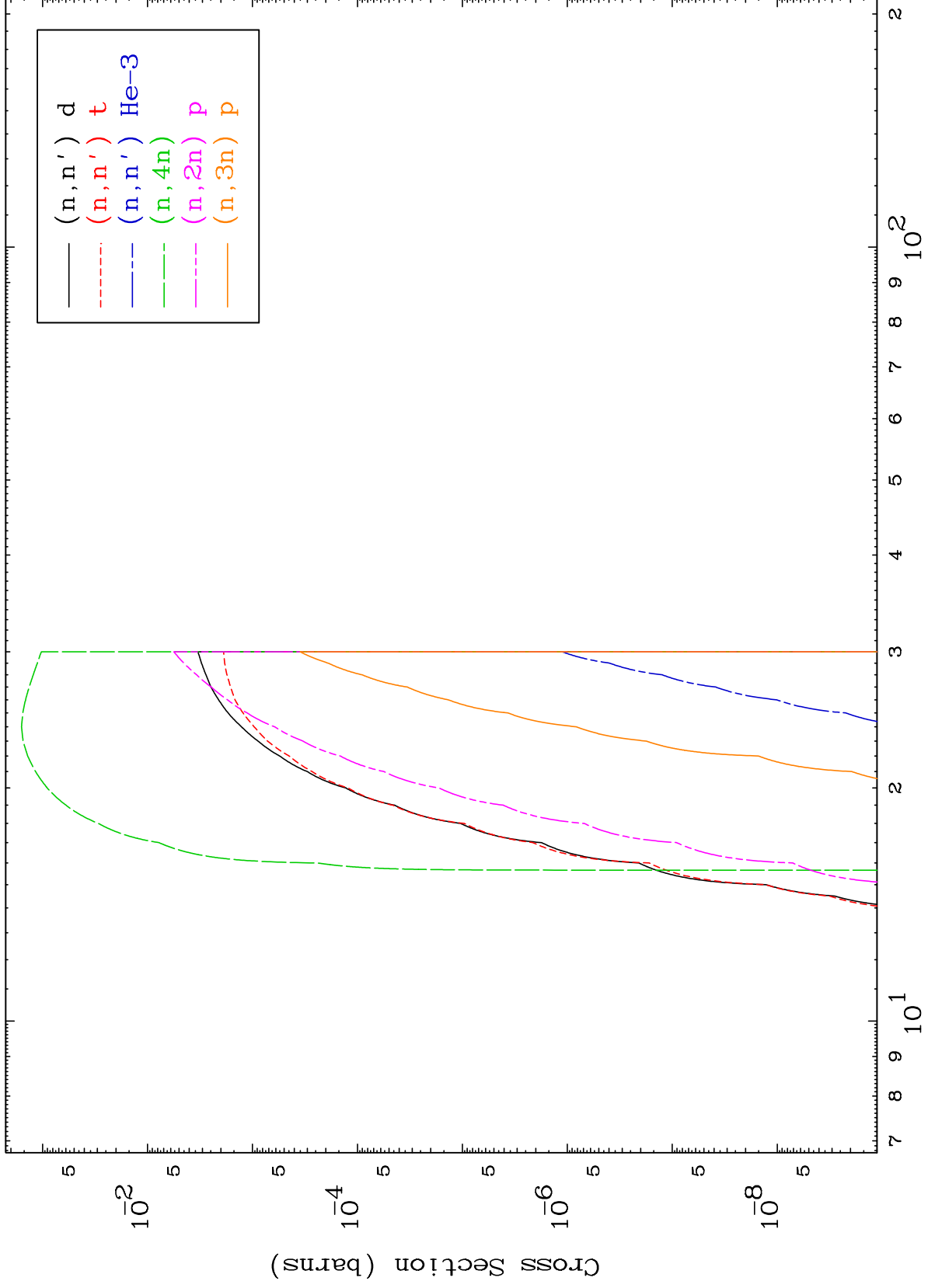
MAT 9918

Neutron Production  
293 Kelvin Cross Sections

99-Es-258



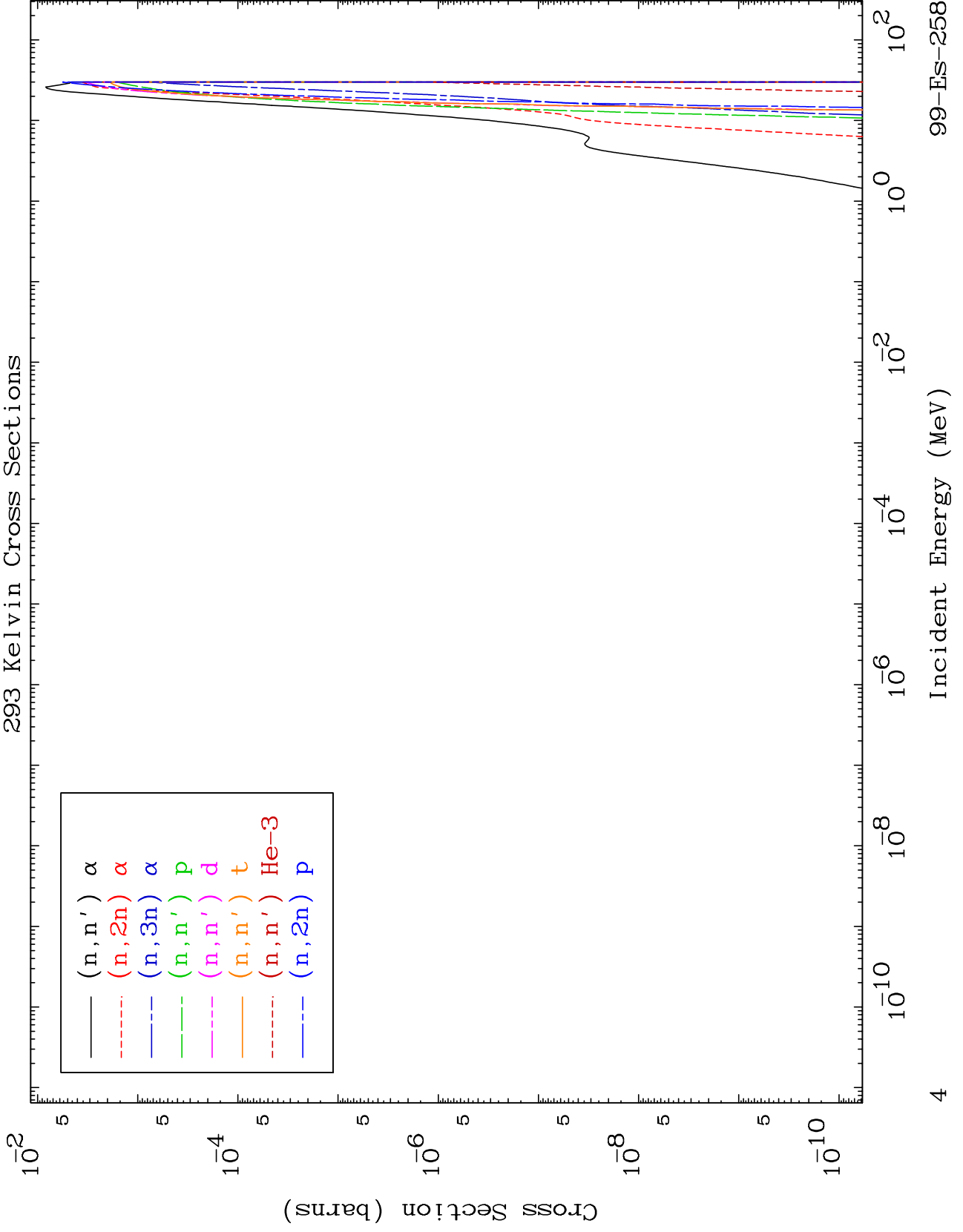
Neutron Production  
293 Kelvin Cross Sections



MAT 9918

Charged Particle  
293 Kelvin Cross Sections

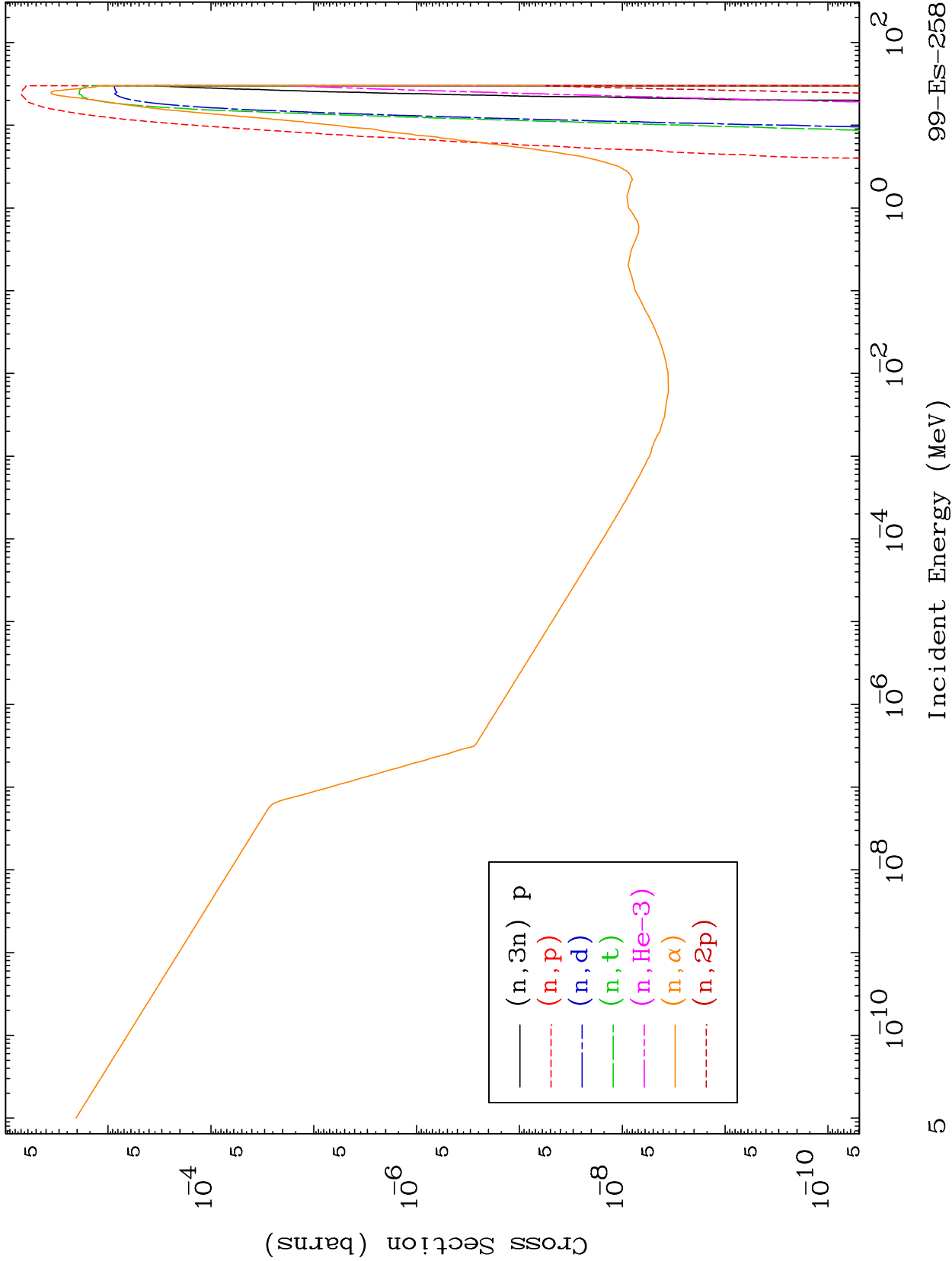
99-Es-258



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Charged Particle  
293 Kelvin Cross Sections

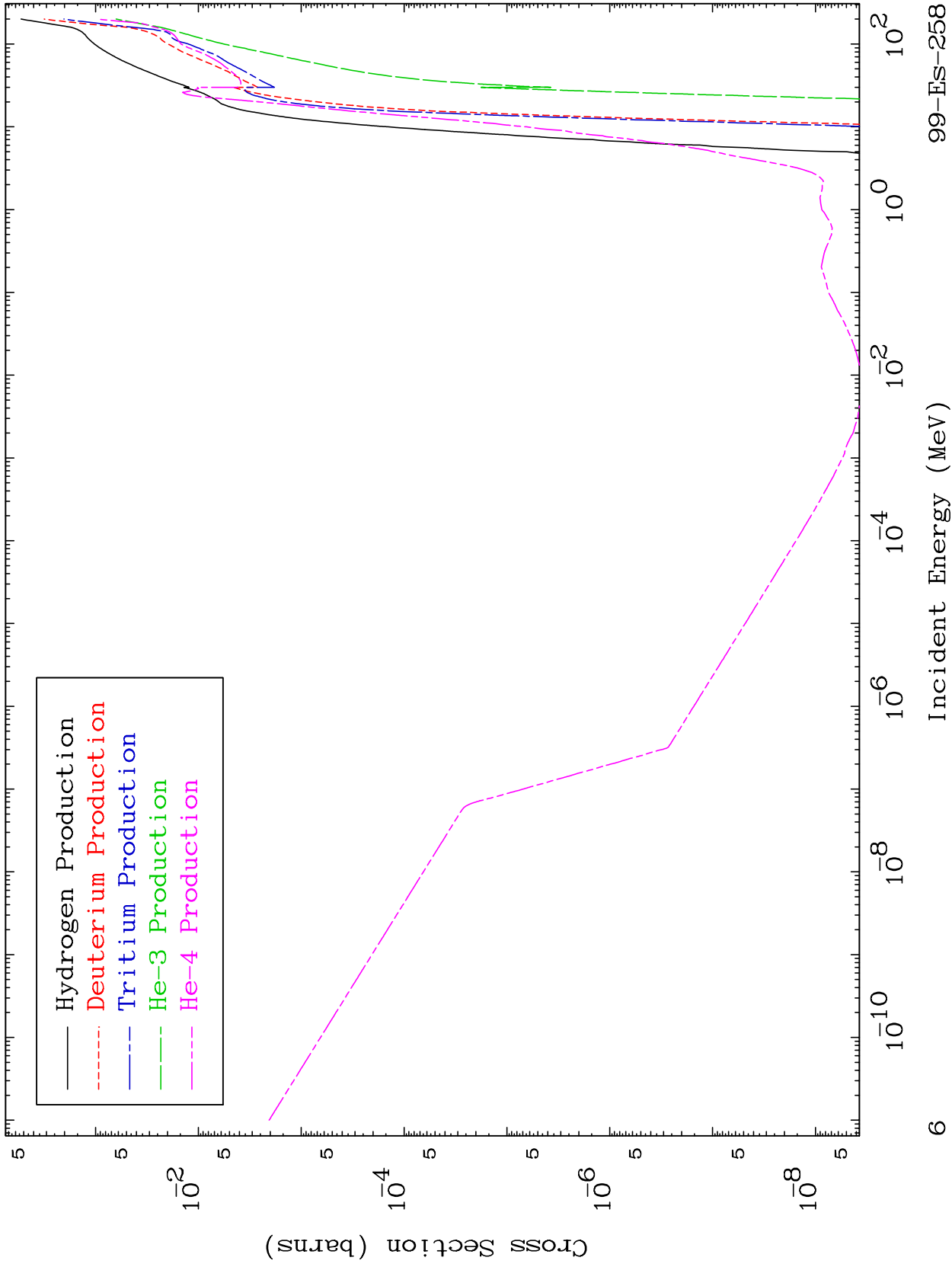
99-Es-258



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Particle Production  
293 Kelvin Cross Sections

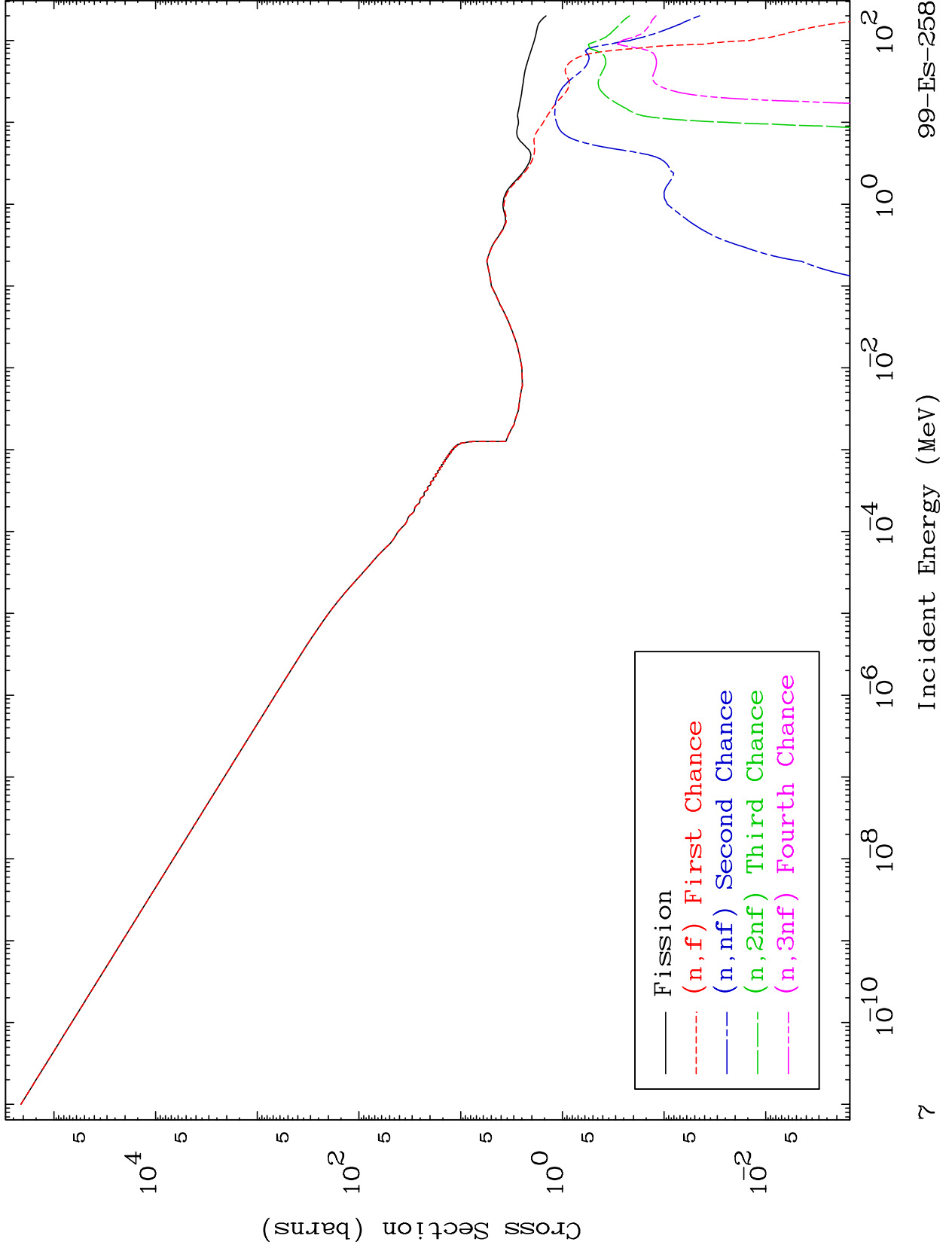
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Fission  
293 Kelvin Cross Sections

99-Es-258



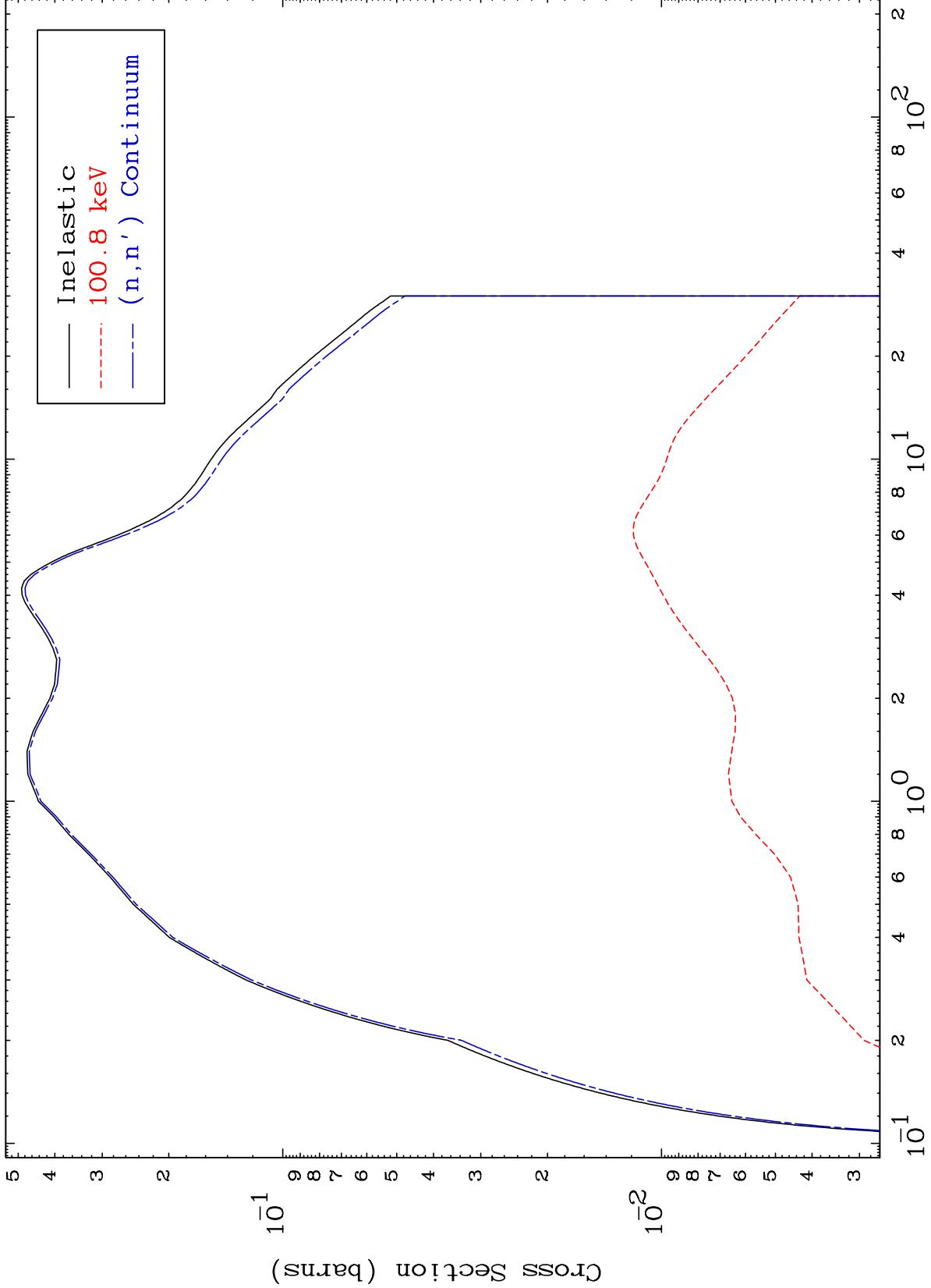


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(n,n') Level

99-Es-258

293 Kelvin Cross Sections



Incident Energy (MeV)

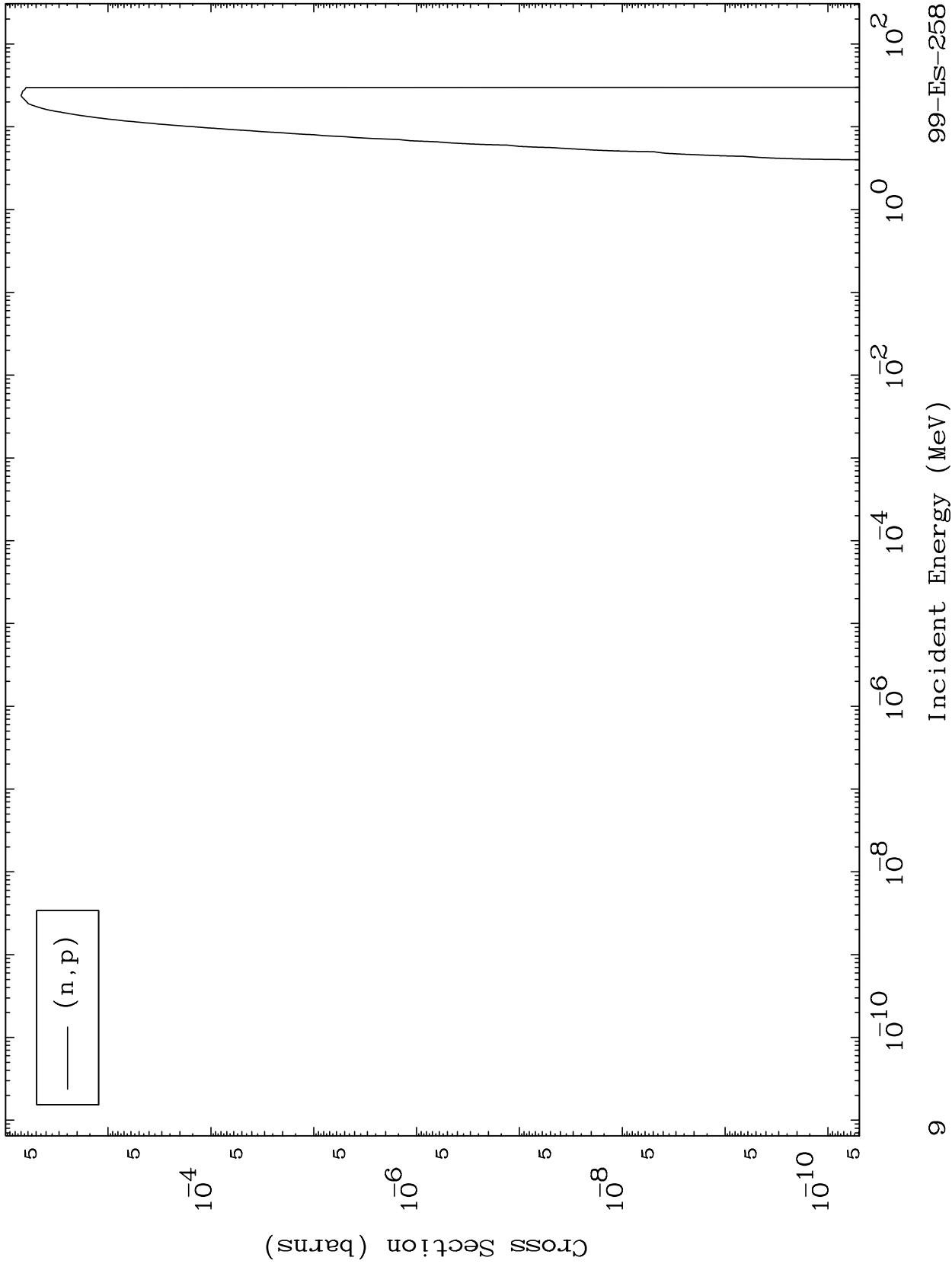
99-Es-258

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MAT 9918

(n,p) Levels  
293 Kelvin Cross Sections

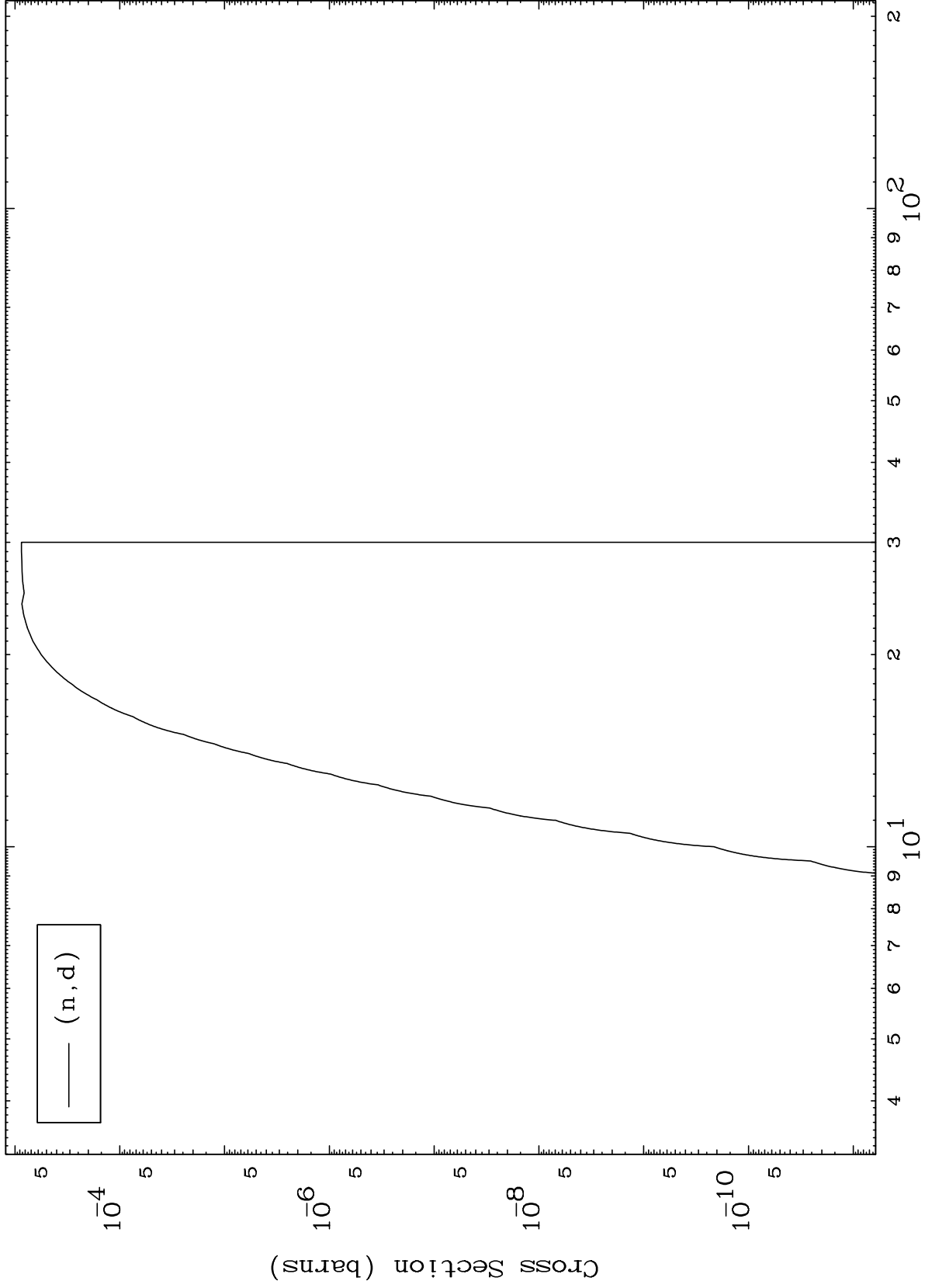
99-Es-258



MAT 9918

(n,d) Levels  
293 Kelvin Cross Sections

99-Es-258



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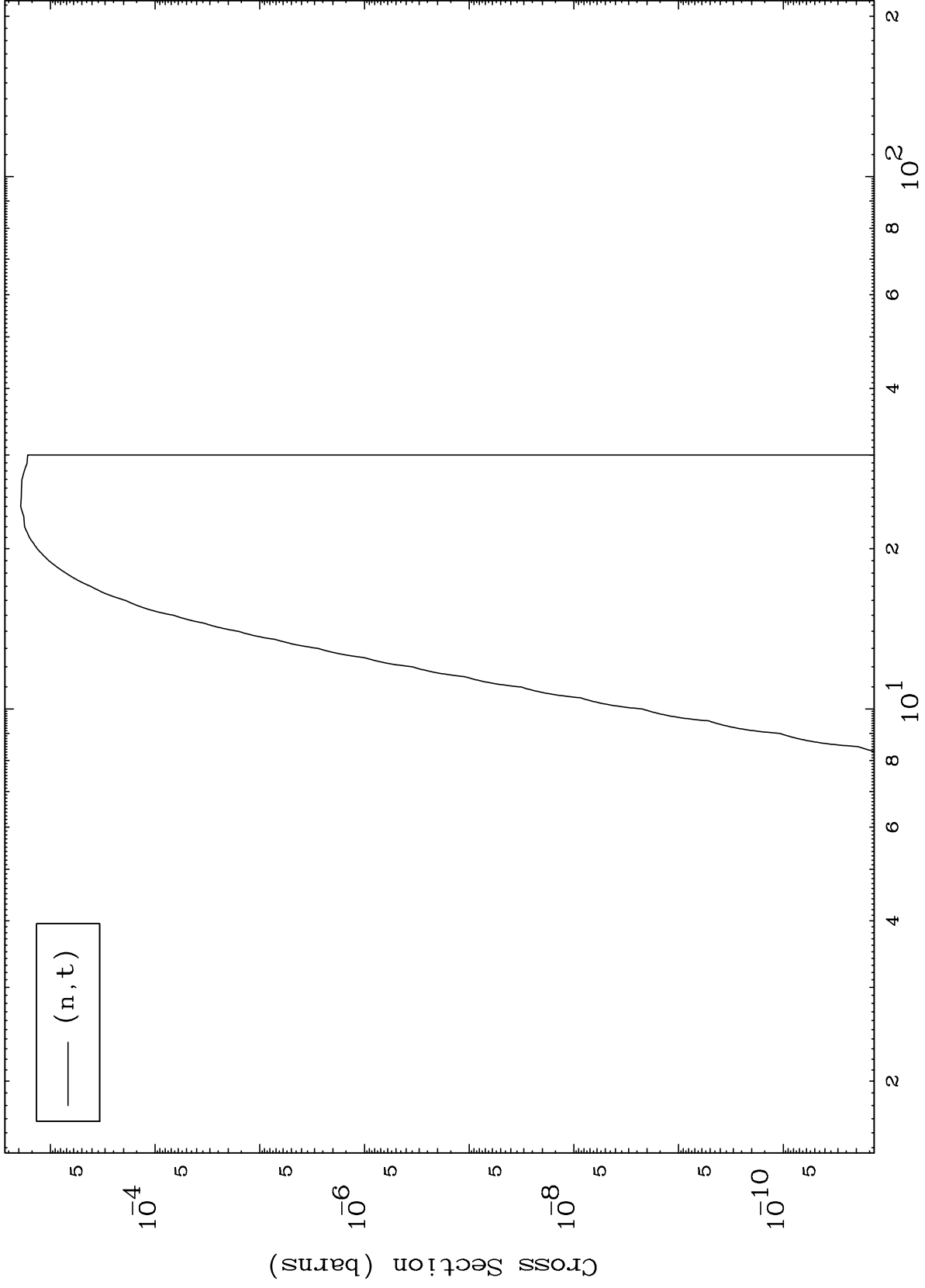
Incident Energy (MeV)

99-Es-258

MAT 9918

(n,t) Levels  
293 Kelvin Cross Sections

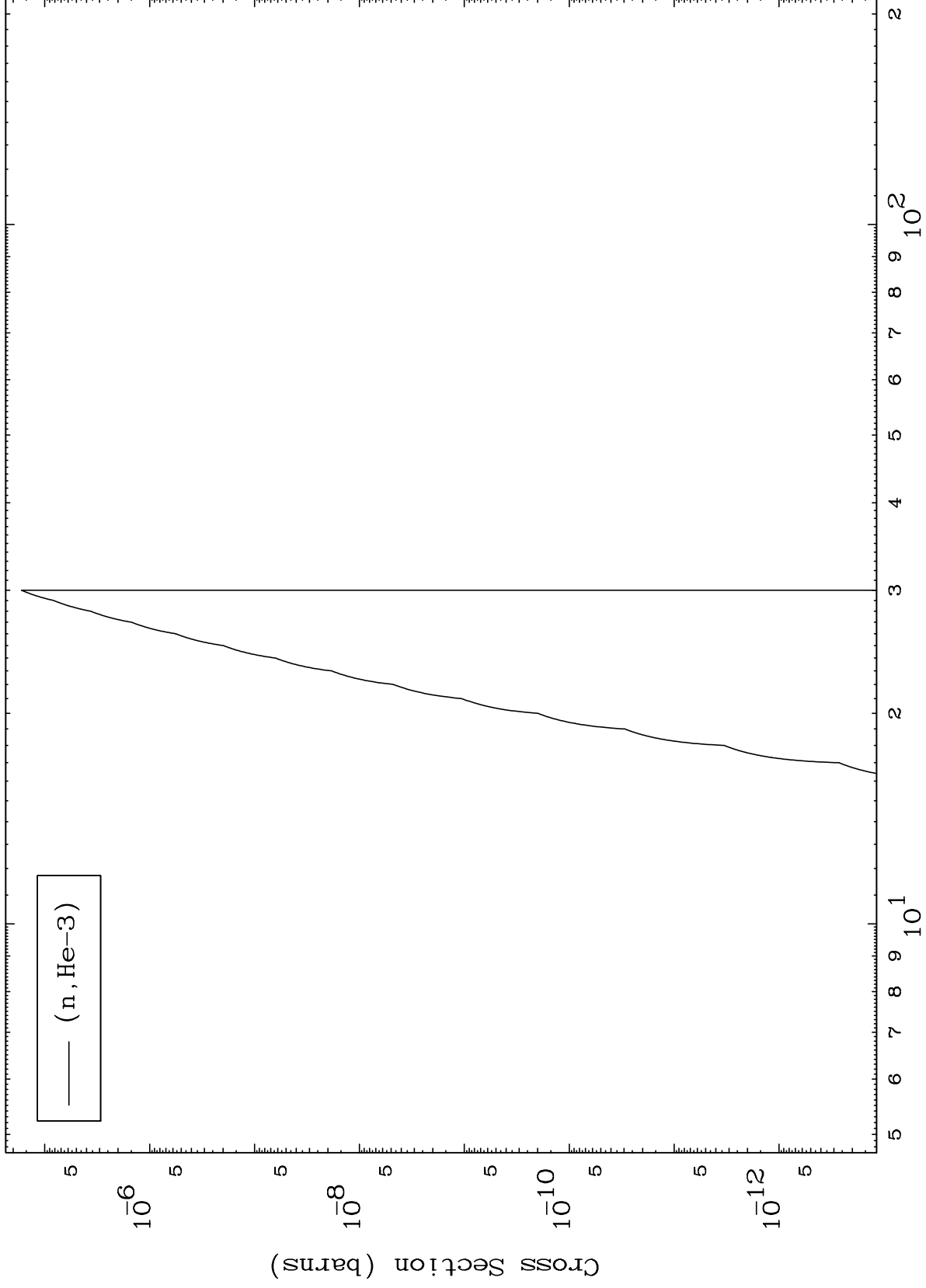
99-Es-258



MAT 9918

(n,He3) Levels  
293 Kelvin Cross Sections

99-Es-258



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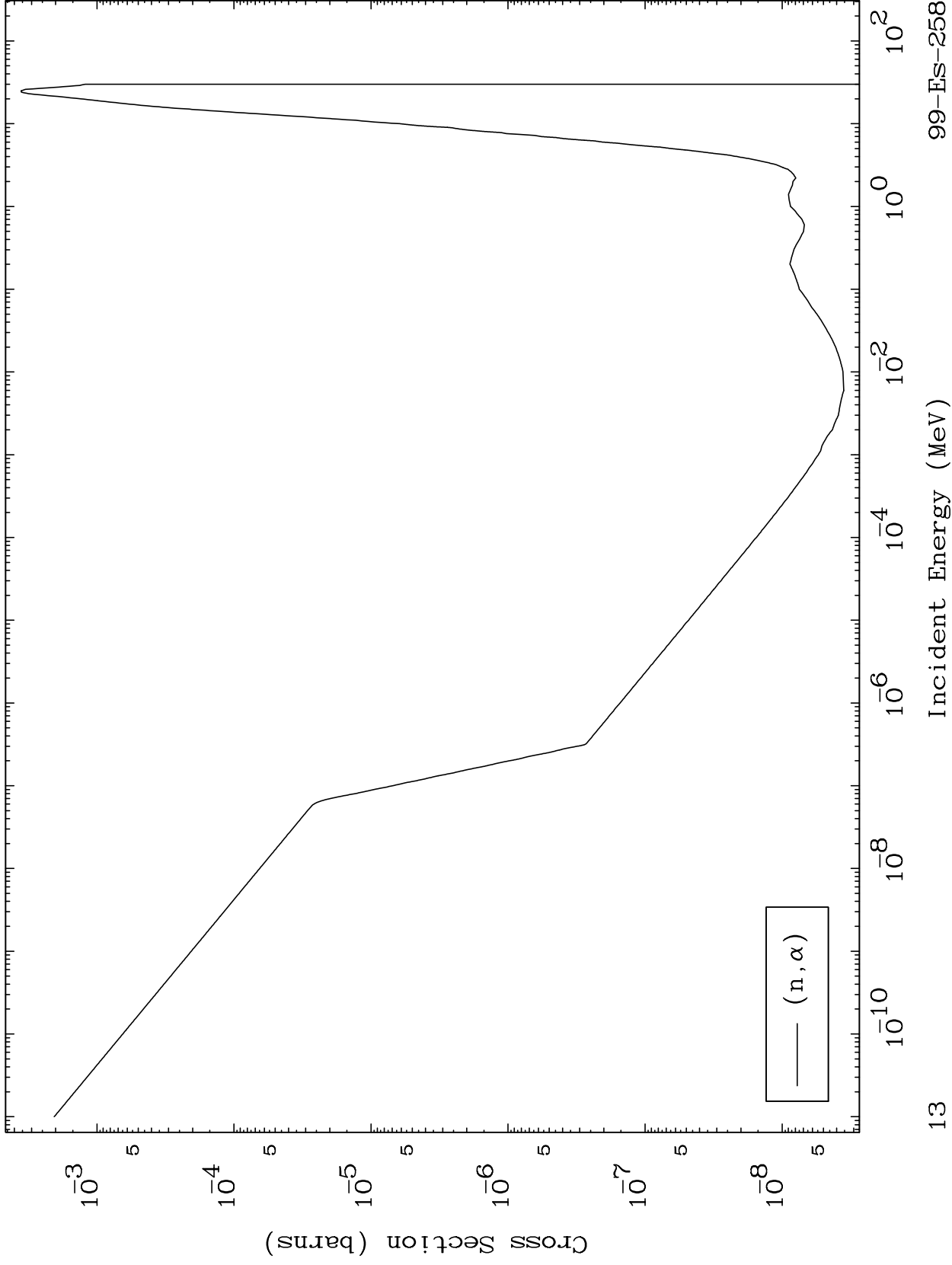
Incident Energy (MeV)

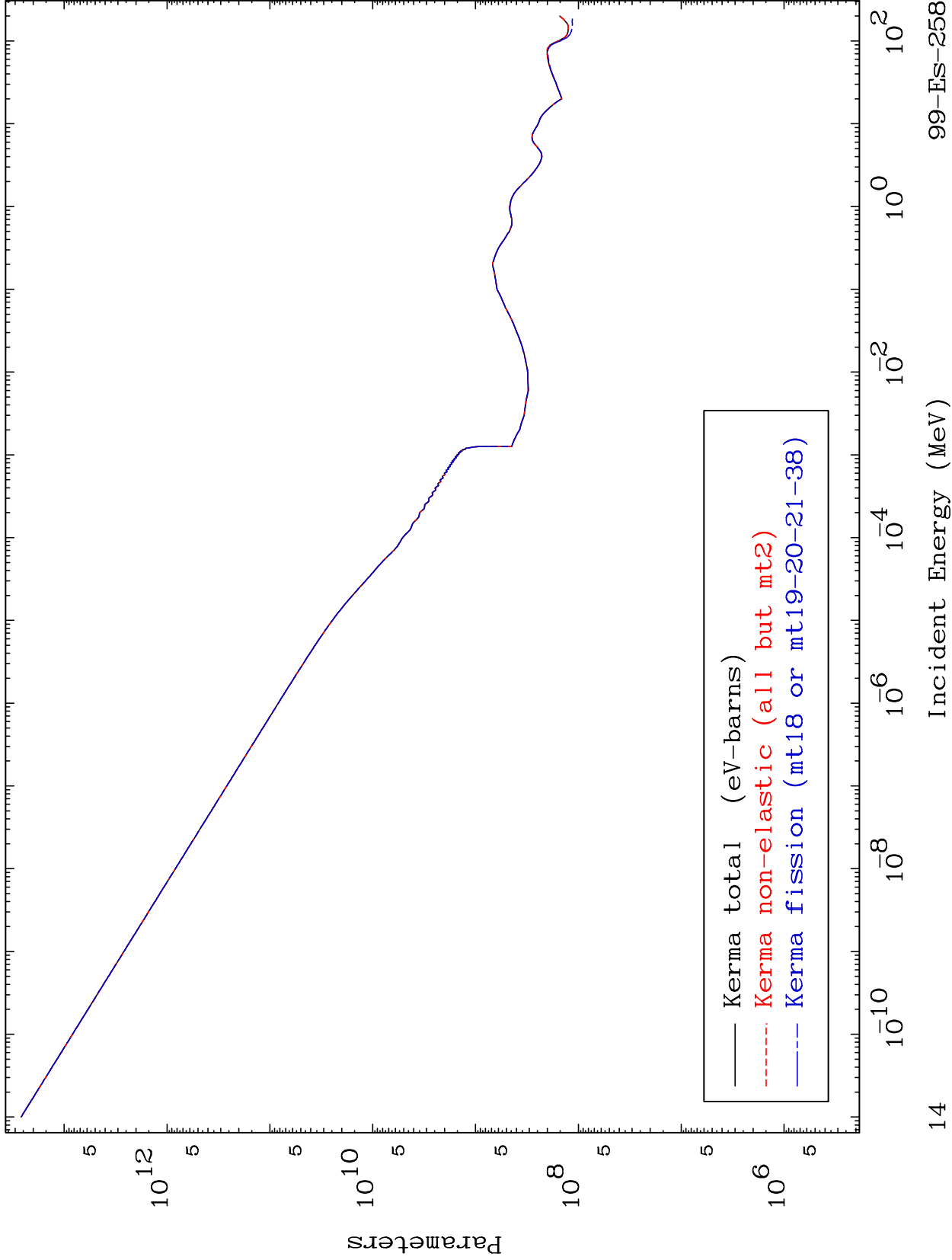
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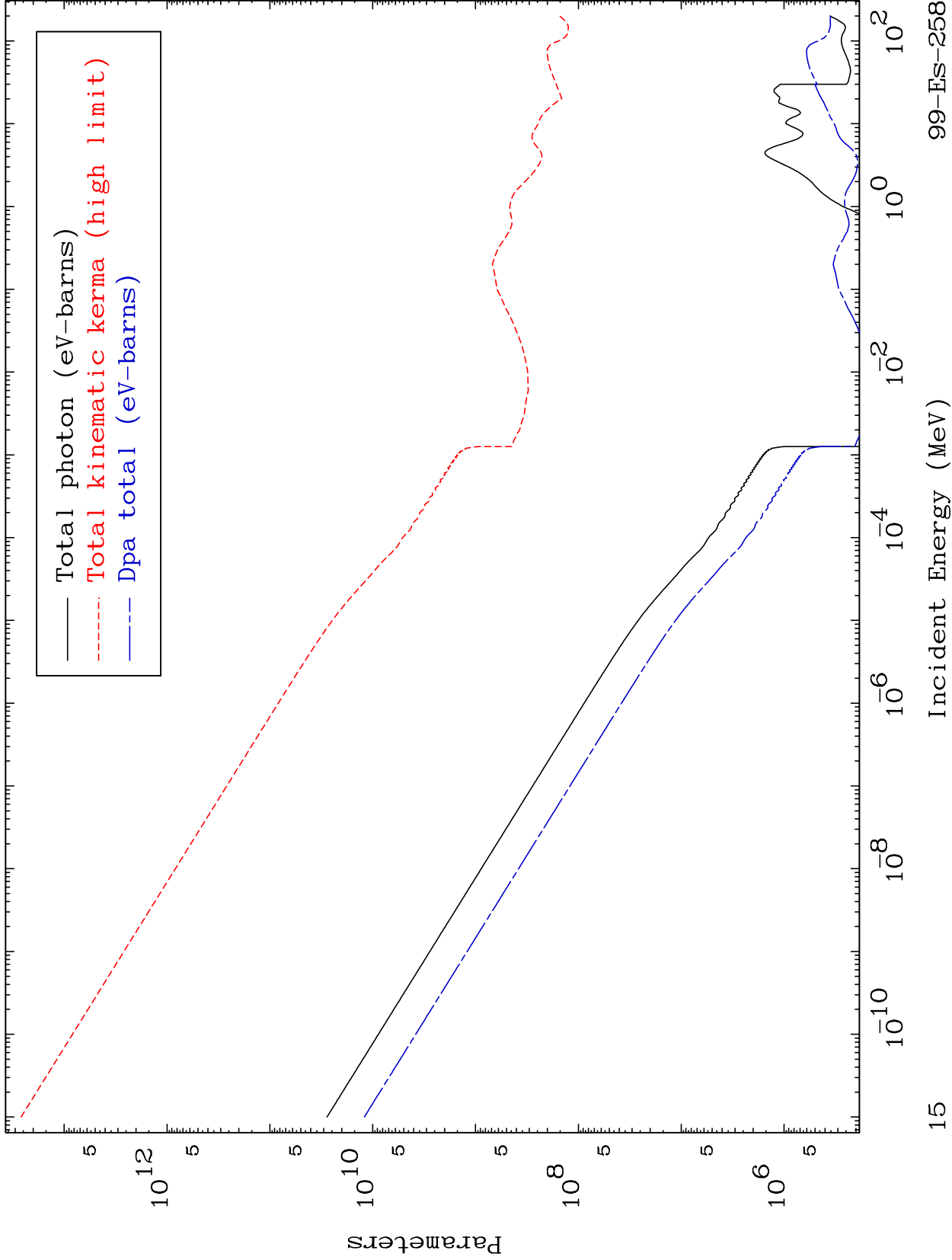
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(n,α) Levels  
293 Kelvin Cross Sections

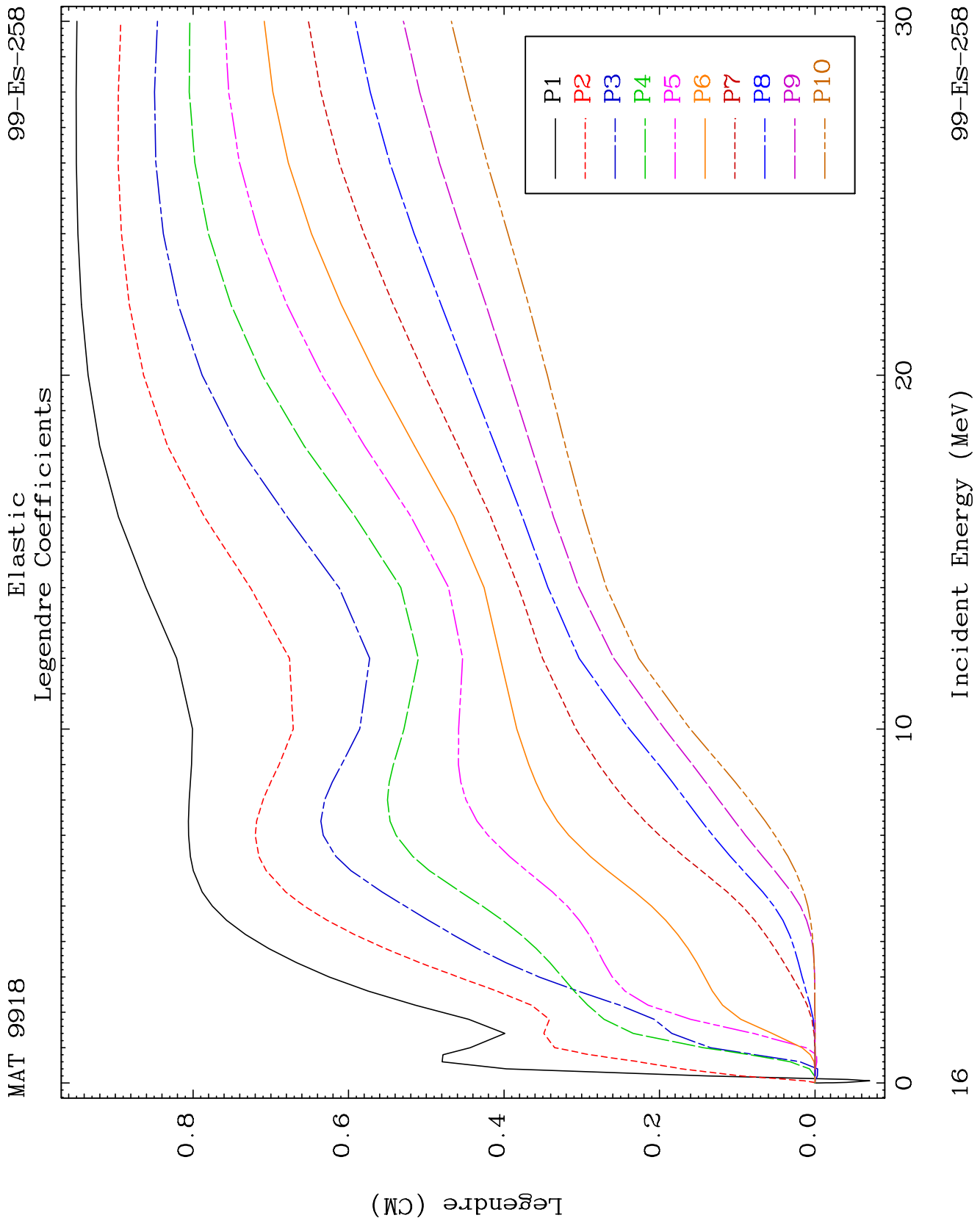
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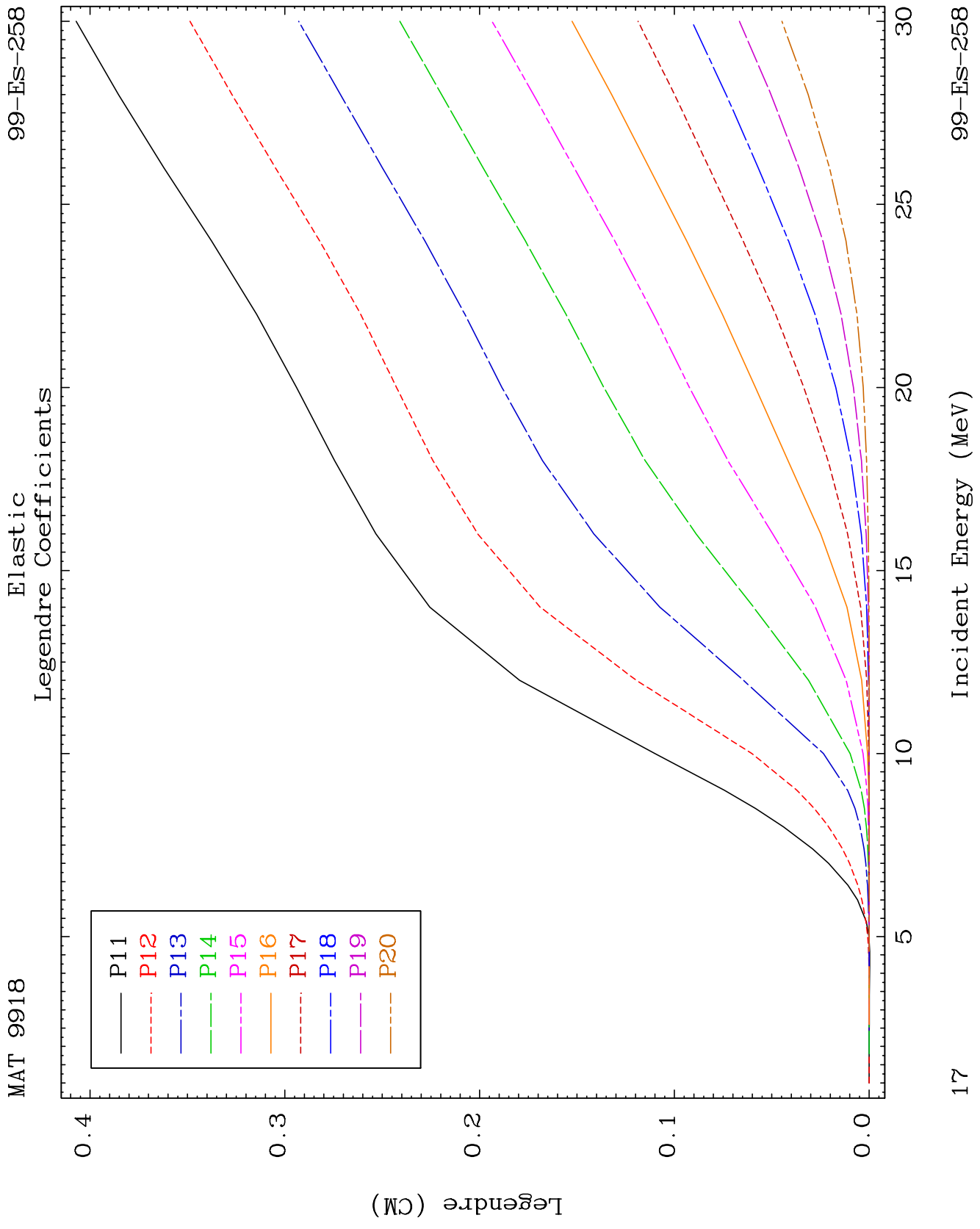








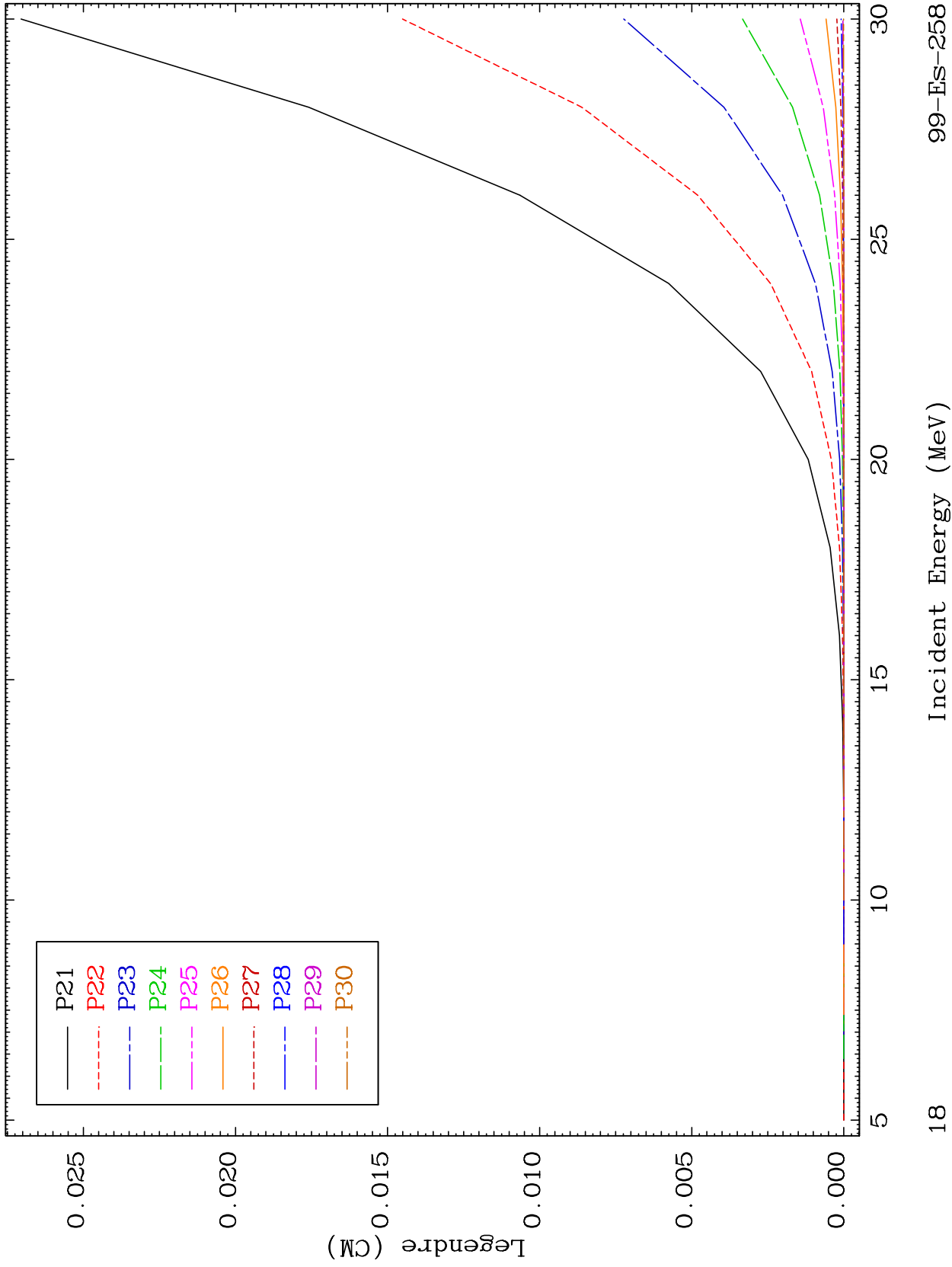




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### Elastic Legendre Coefficients

99-Es-258



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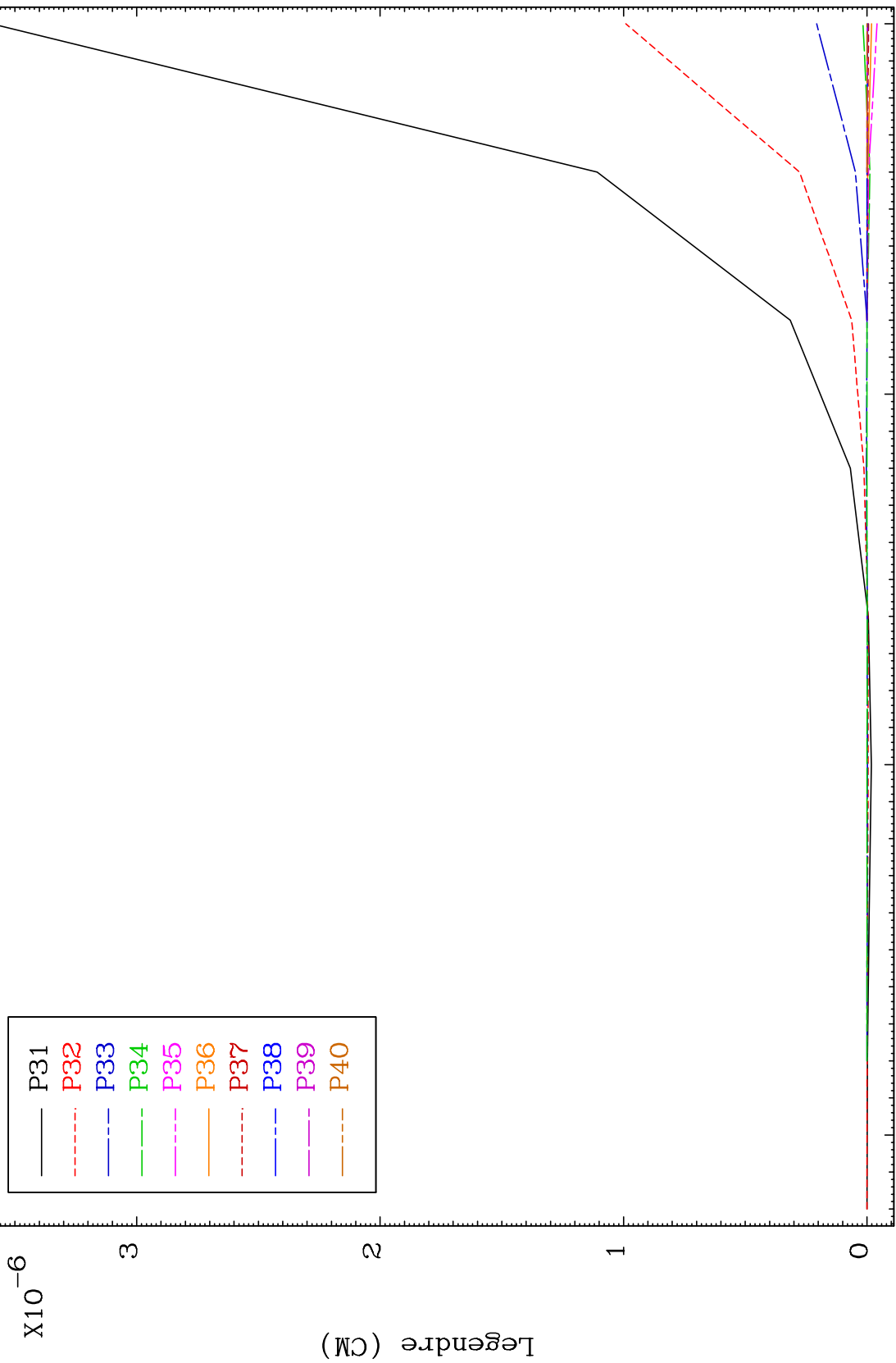
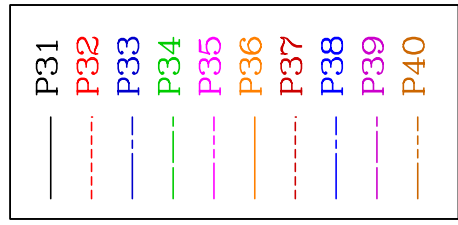
Incident Energy (MeV)

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Elastic Legendre Coefficients

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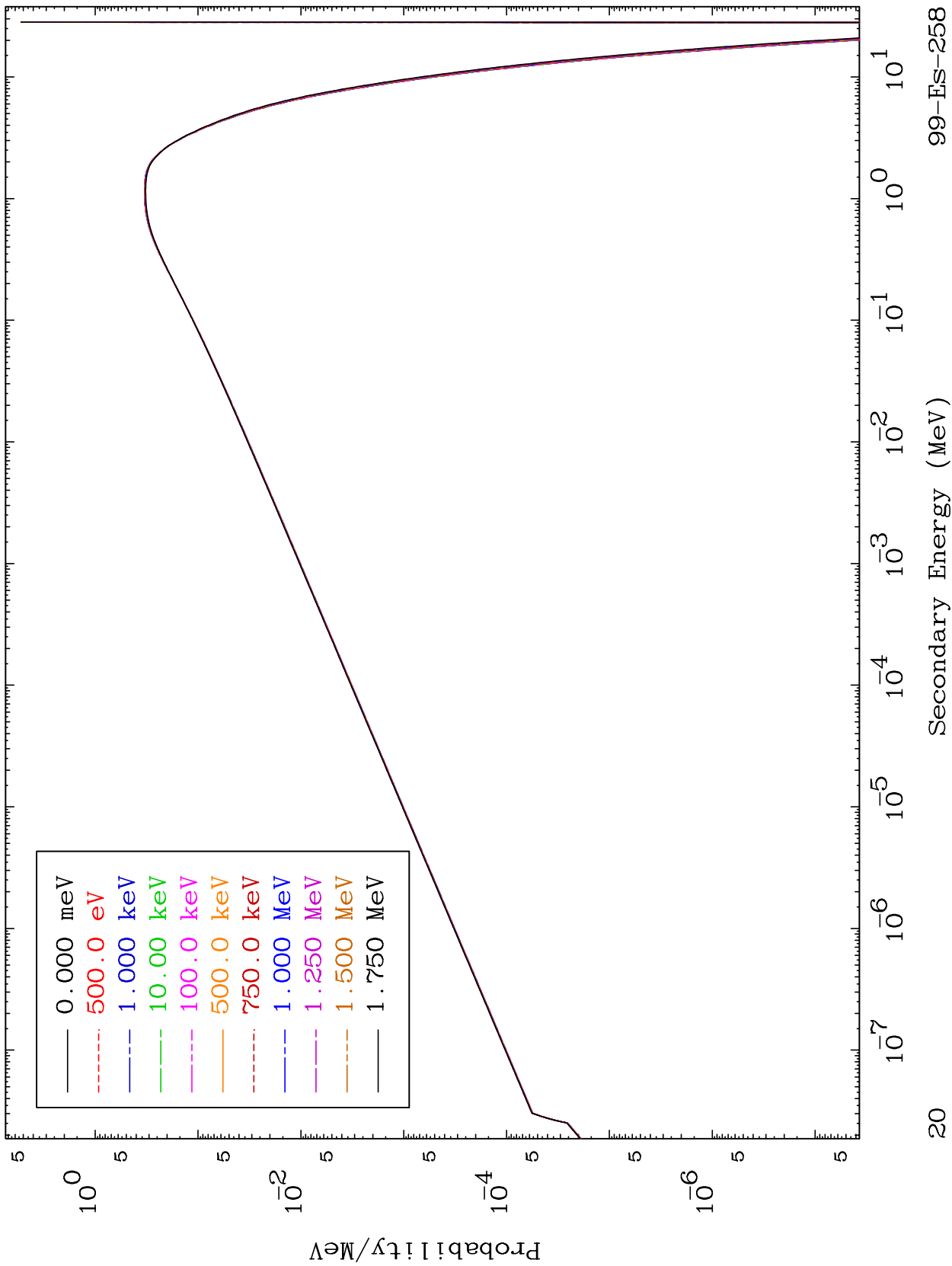


19

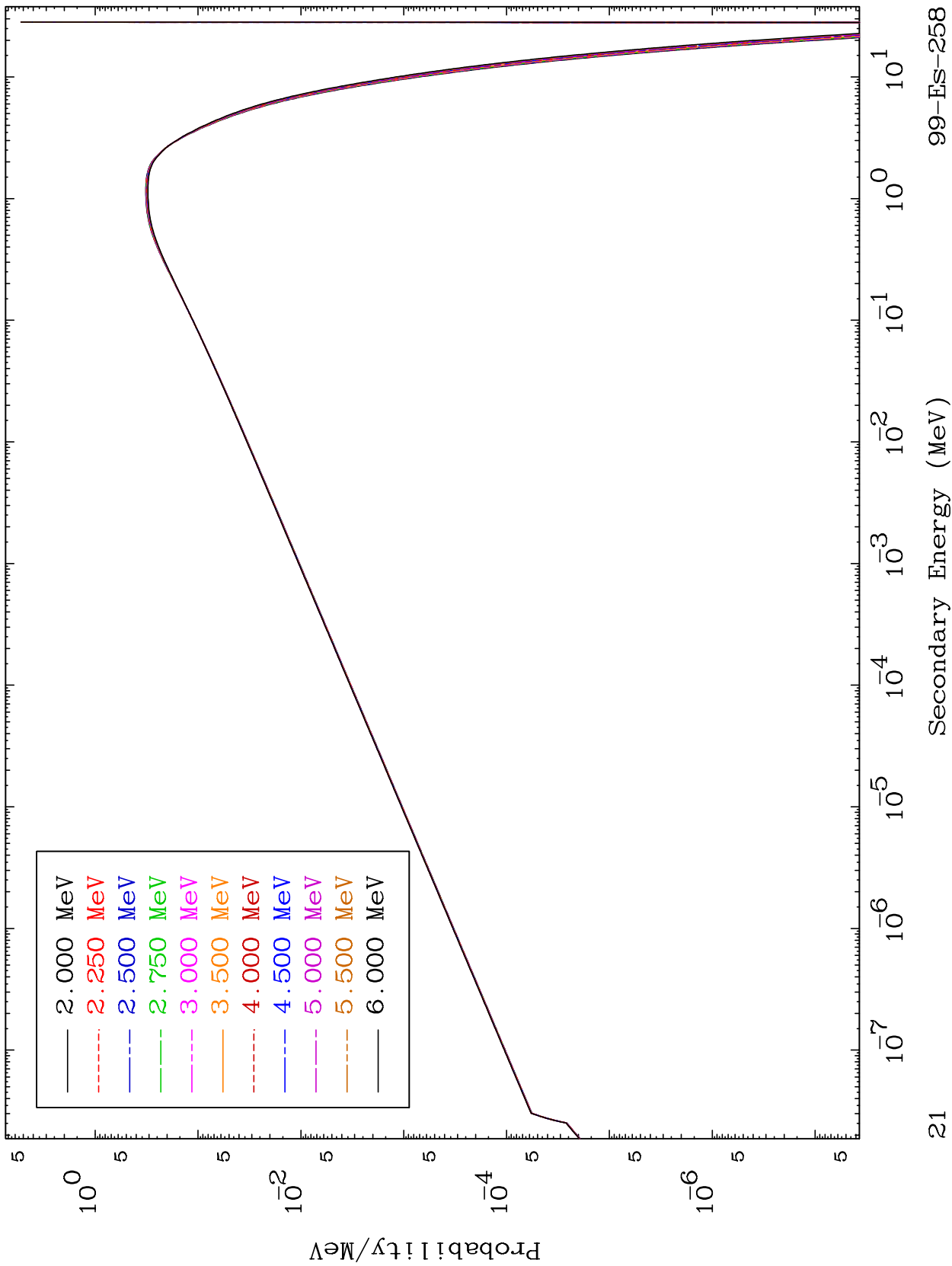
Incident Energy (MeV)

99-Es-258

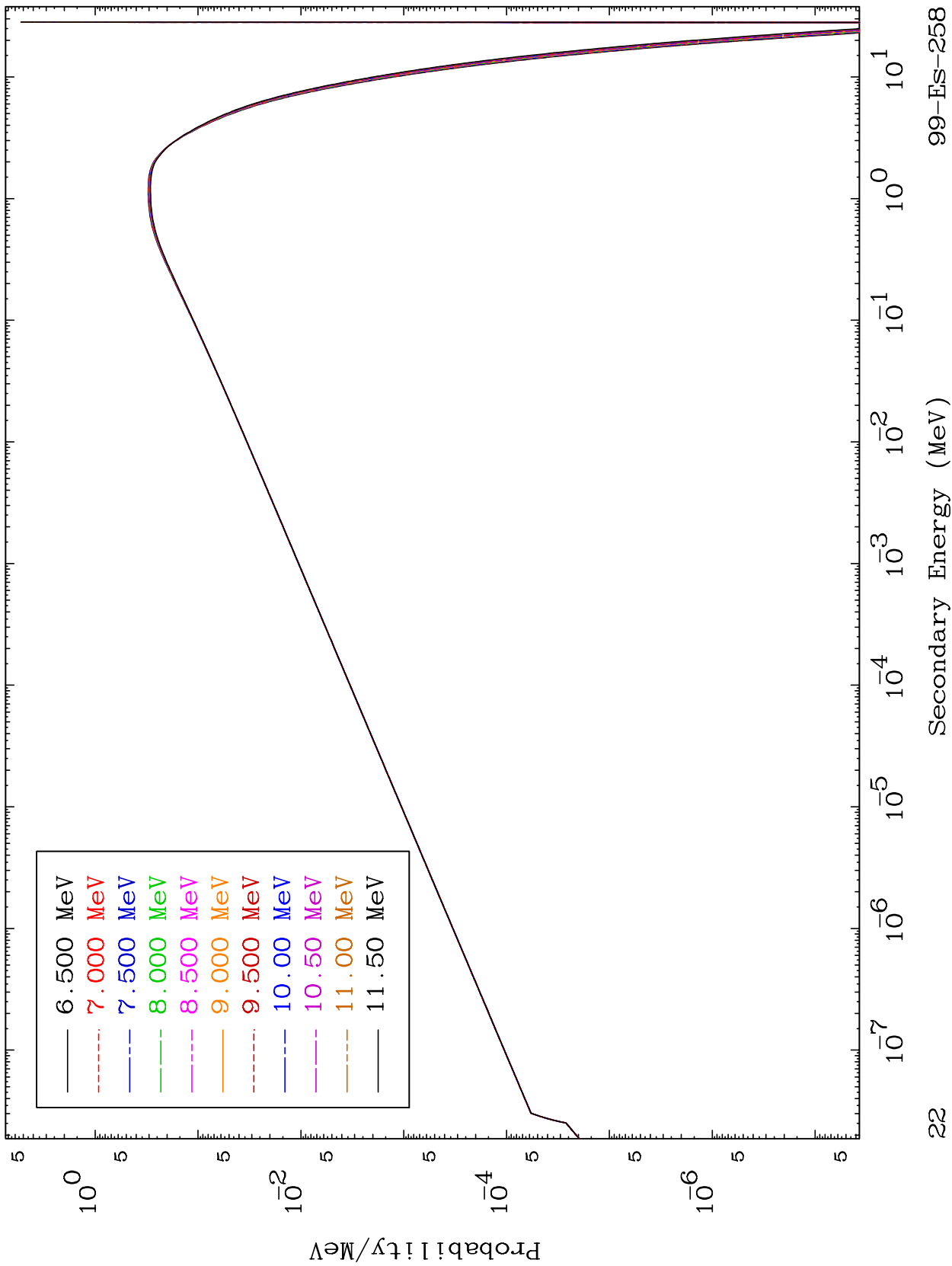
### Fission Energy Distribution



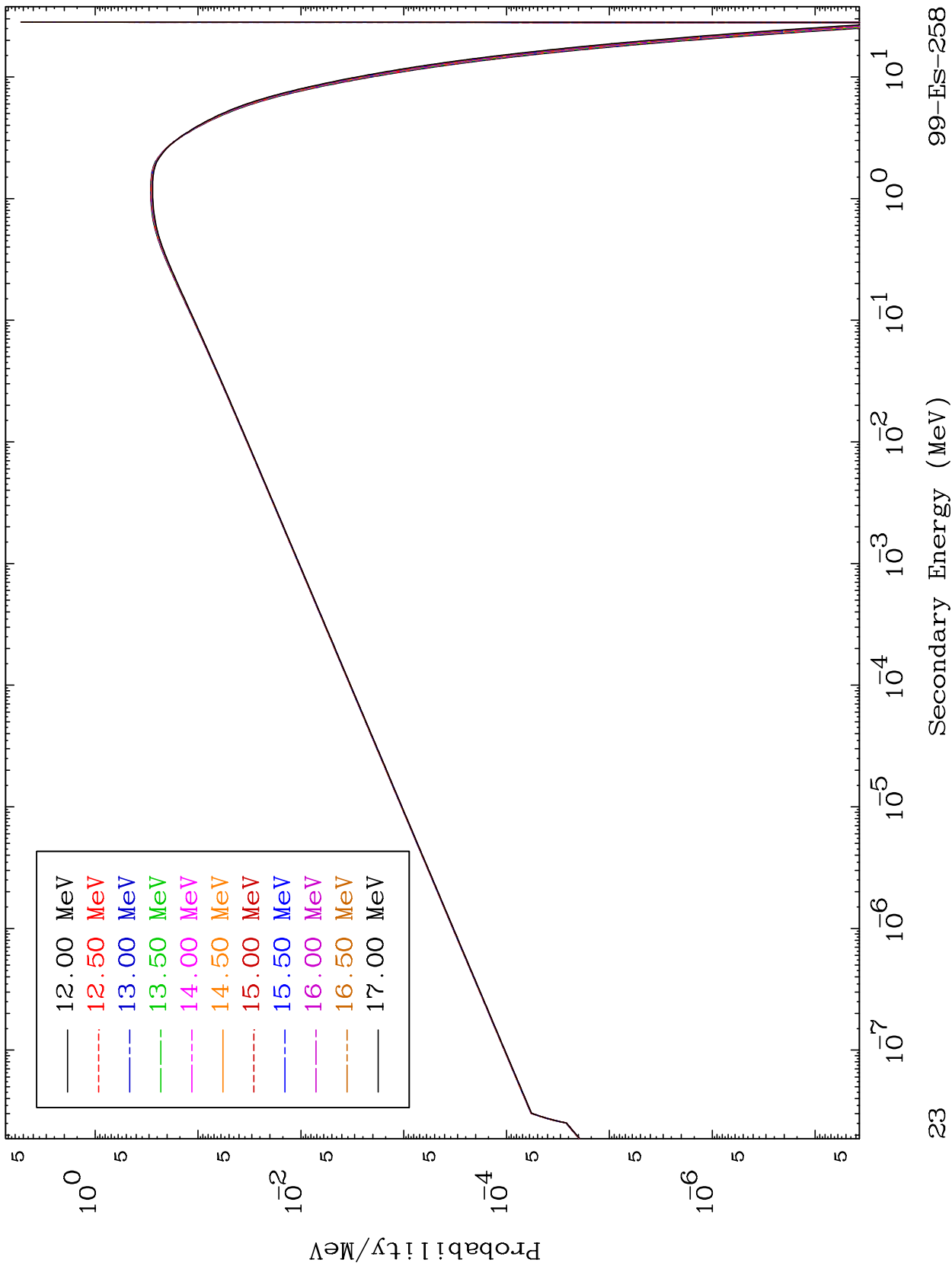
Fission  
Energy Distribution



Fission  
Energy Distribution

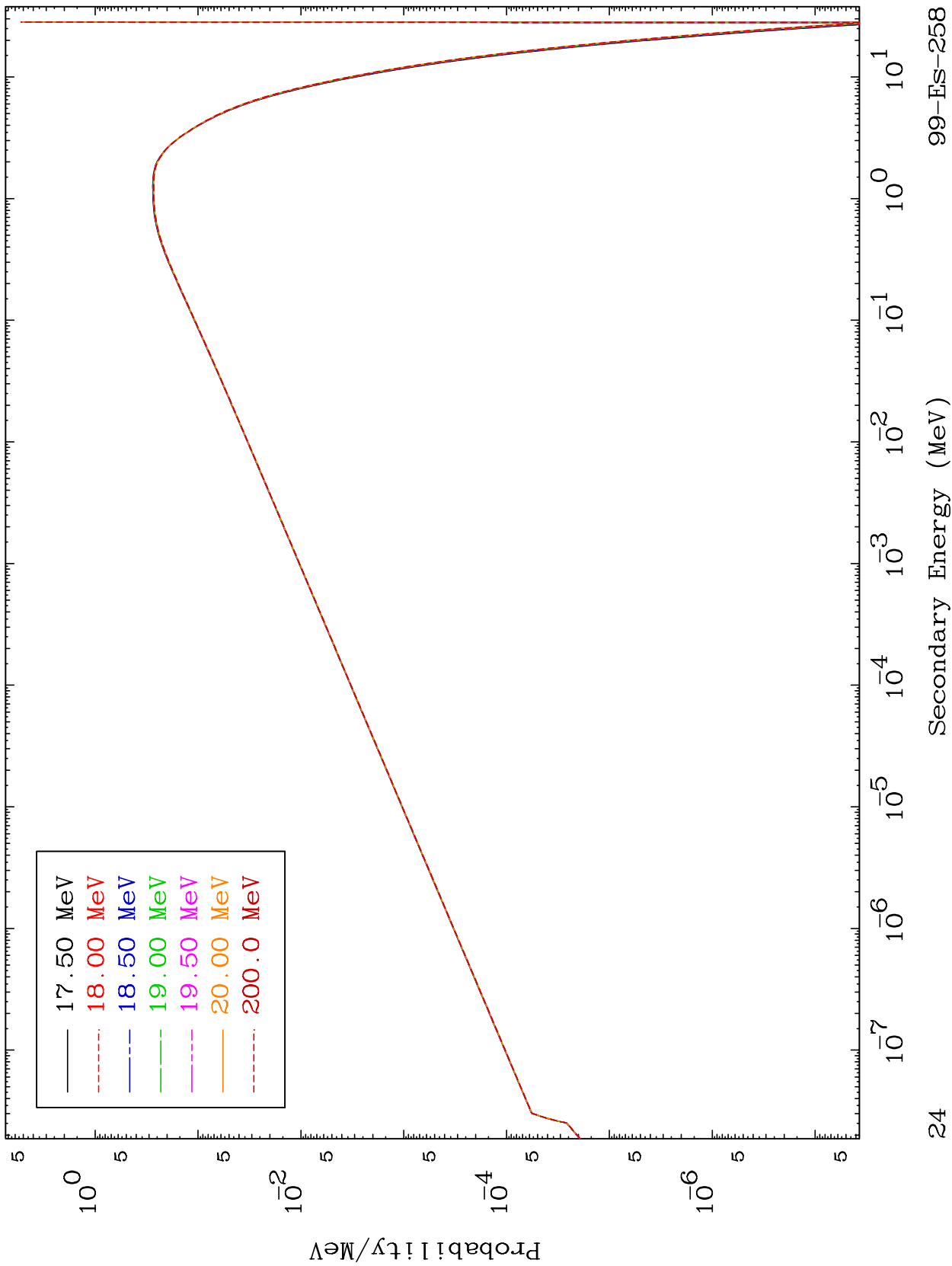


Fission  
Energy Distribution





Fission  
Energy Distribution

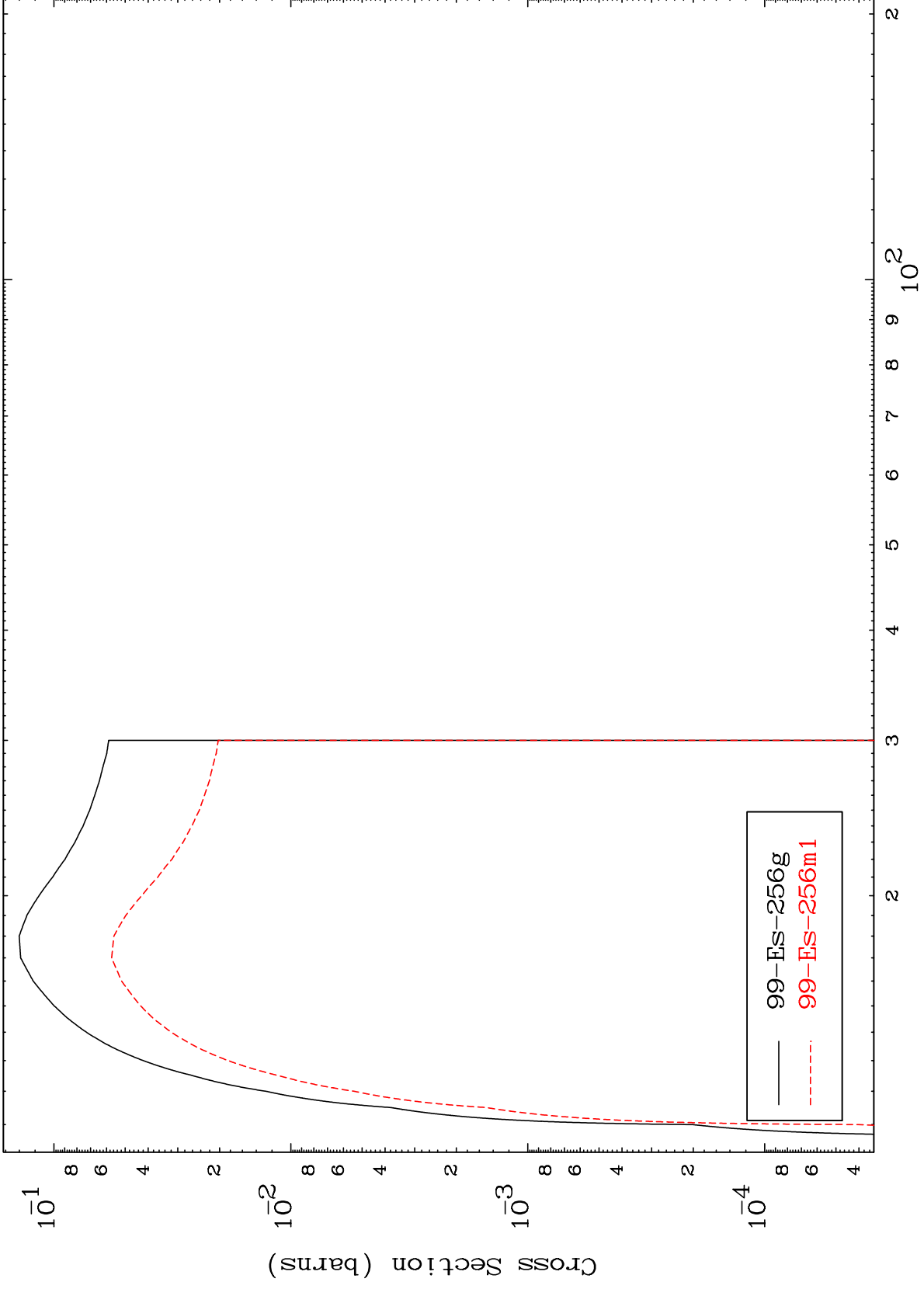


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(n,3n)

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

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