

Program EVALPLOT
(Version 2015-2)

by

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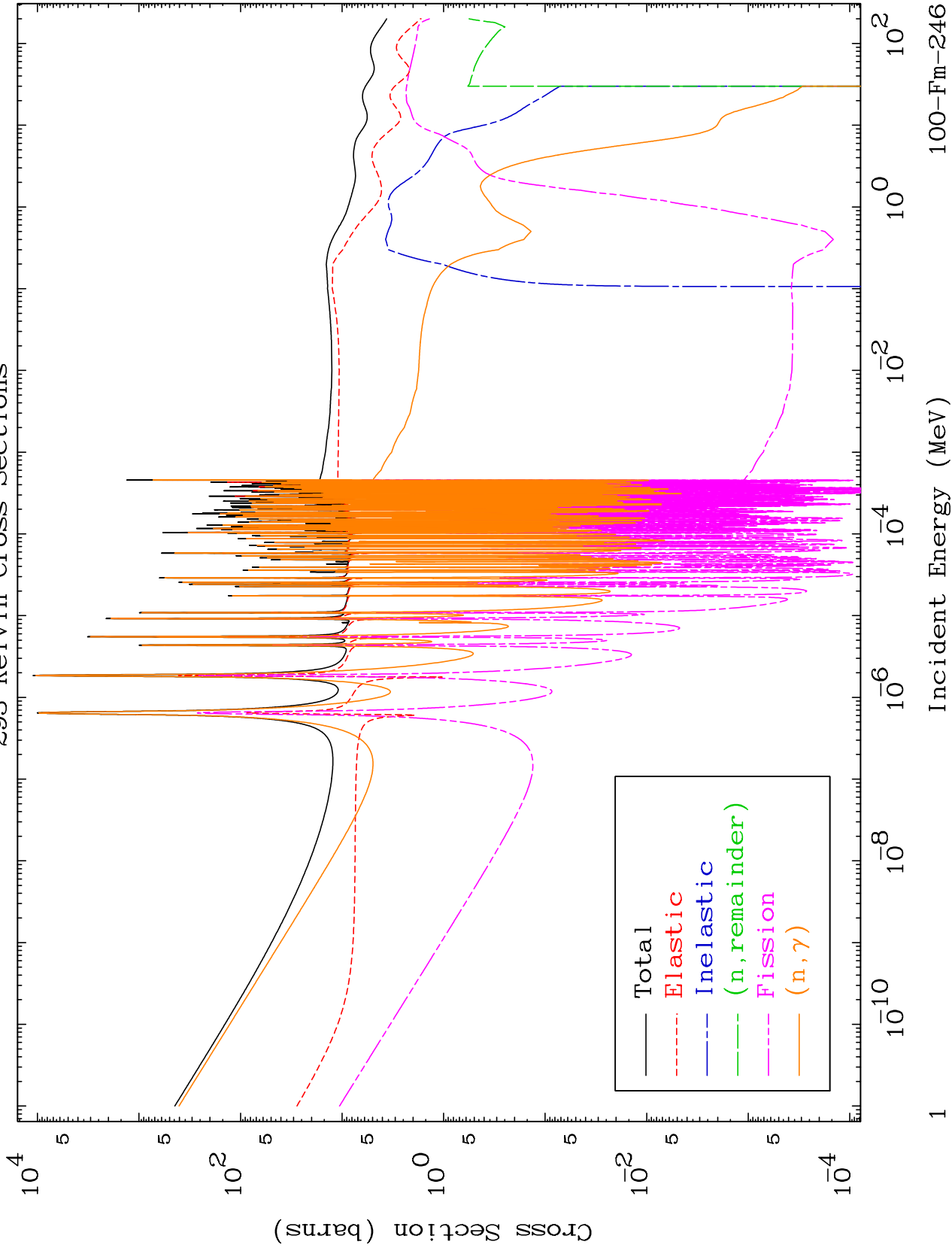
Press Mouse Button to Start

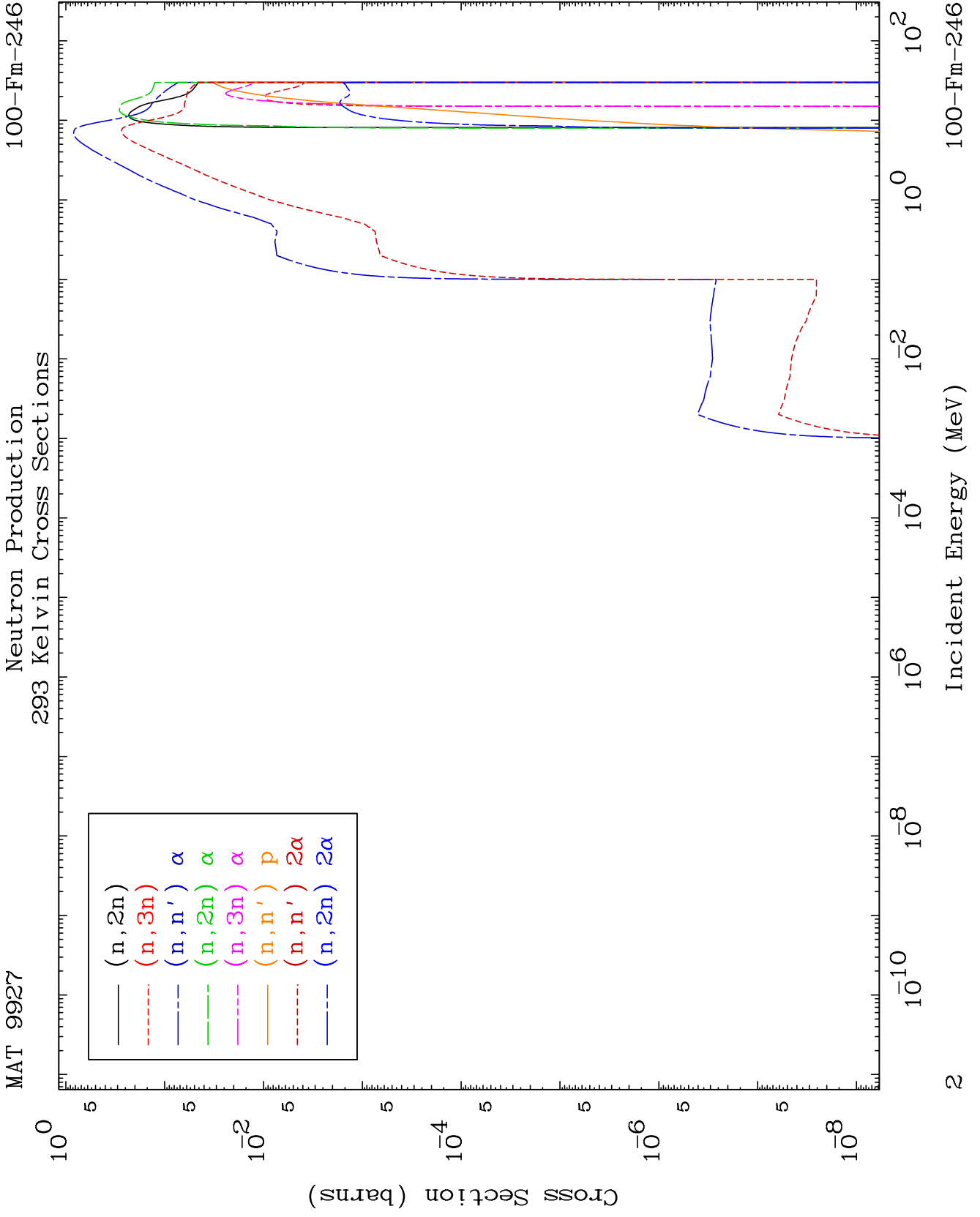
MAT 9927

Major

293 Kelvin Cross Sections

100-Fm-246

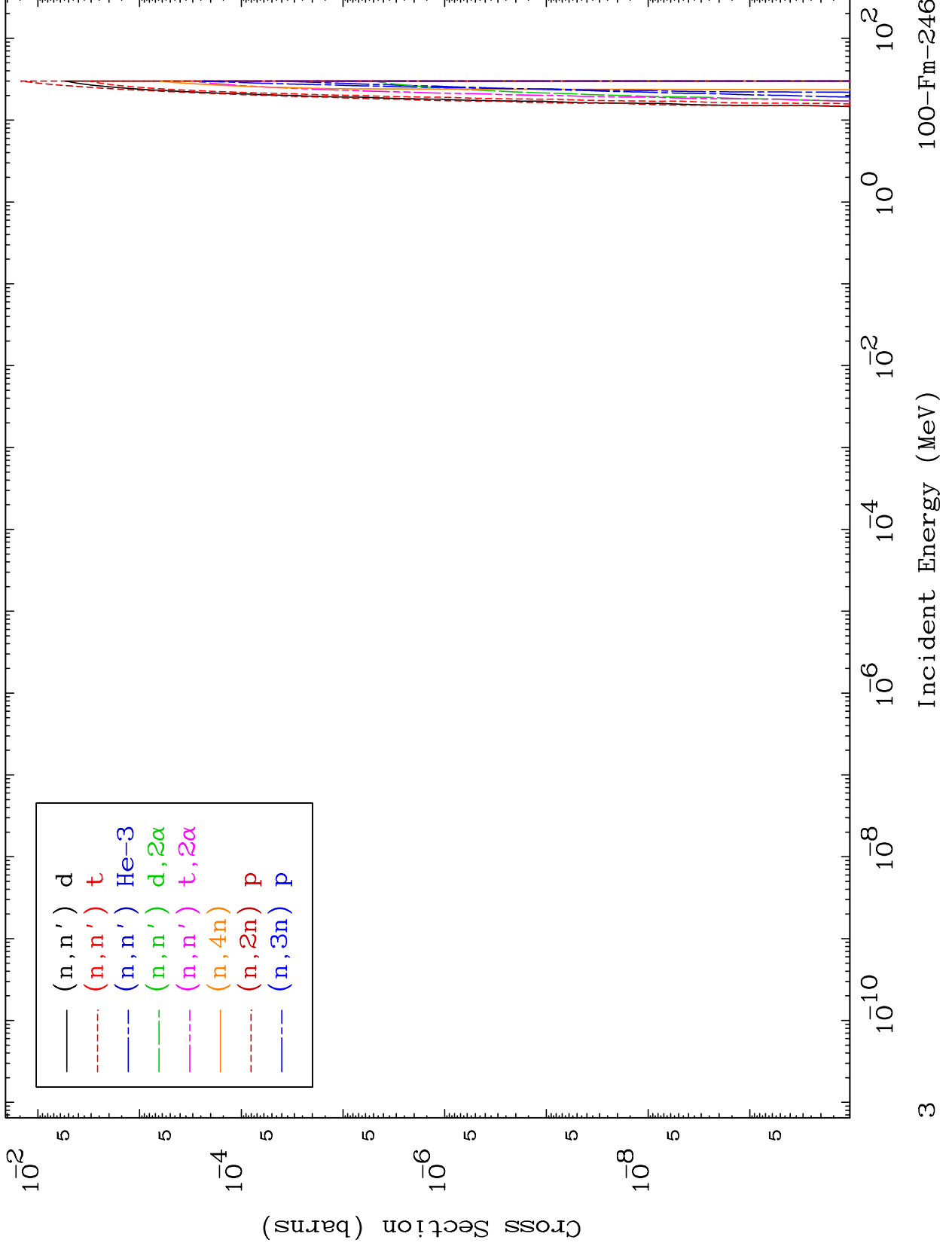




MAT 9927

Neutron Production
293 Kelvin Cross Sections

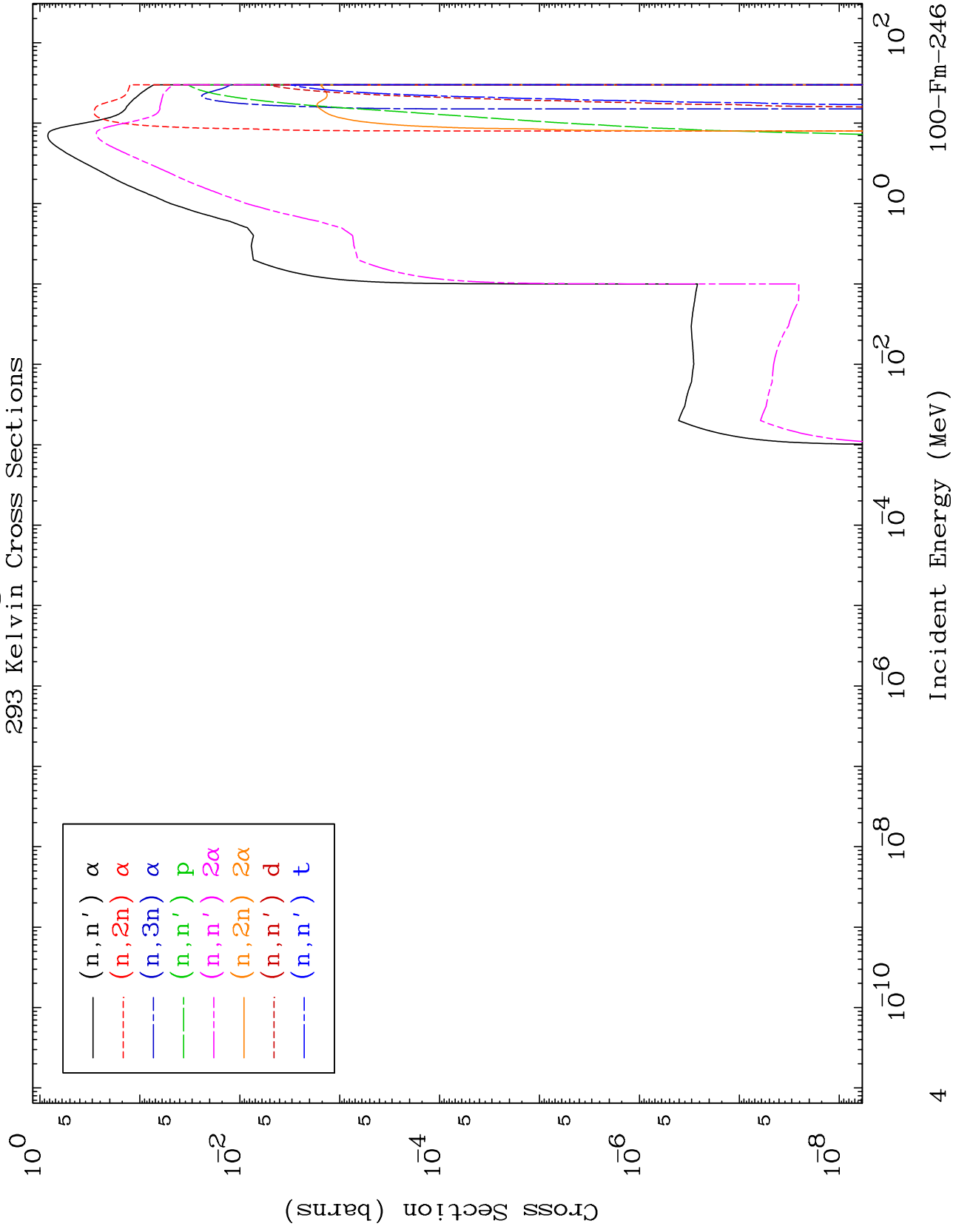
100-Fm-246



MAT 9927

Charged Particle
293 Kelvin Cross Sections

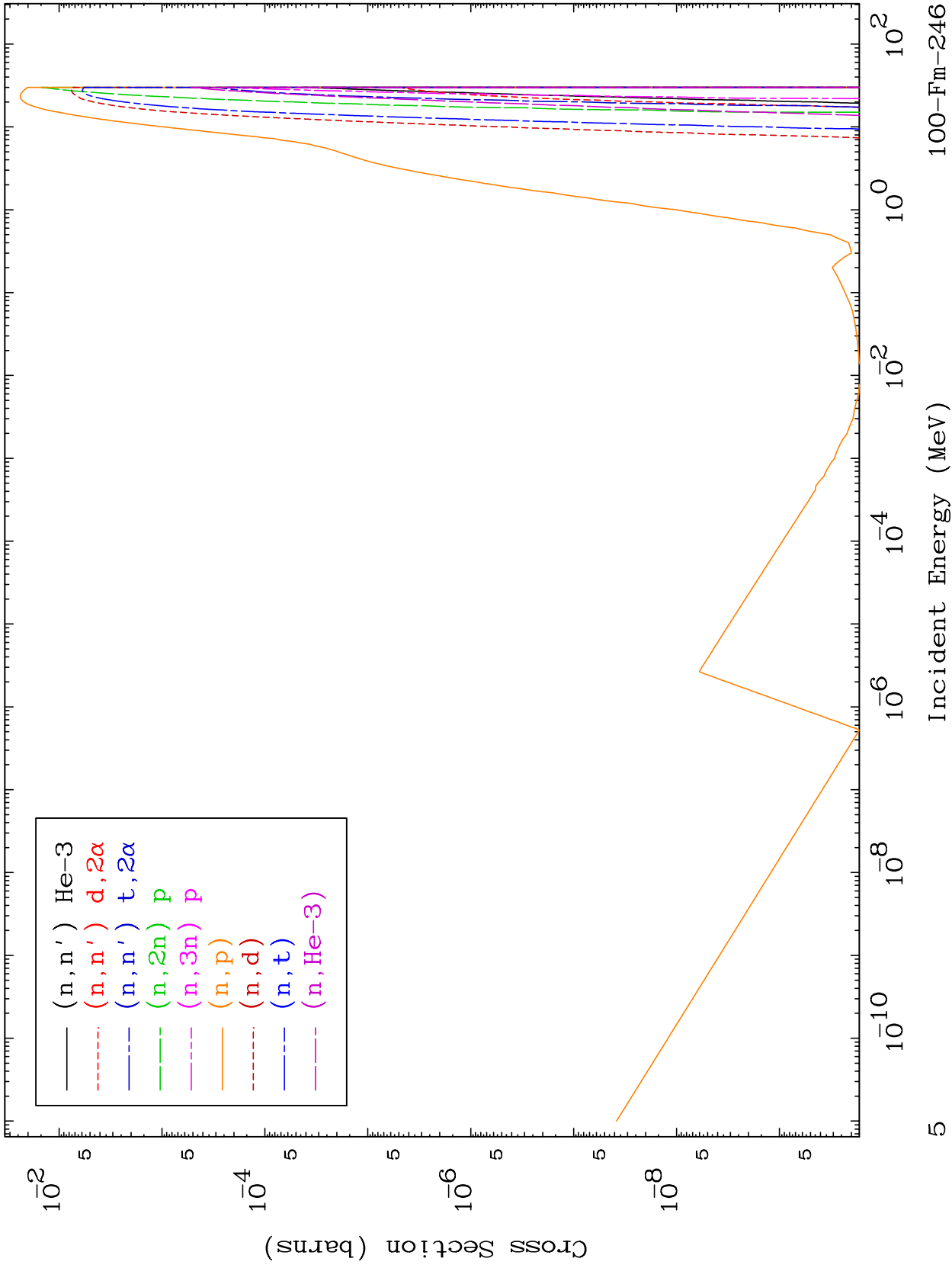
100-Fm-246



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Charged Particle
293 Kelvin Cross Sections

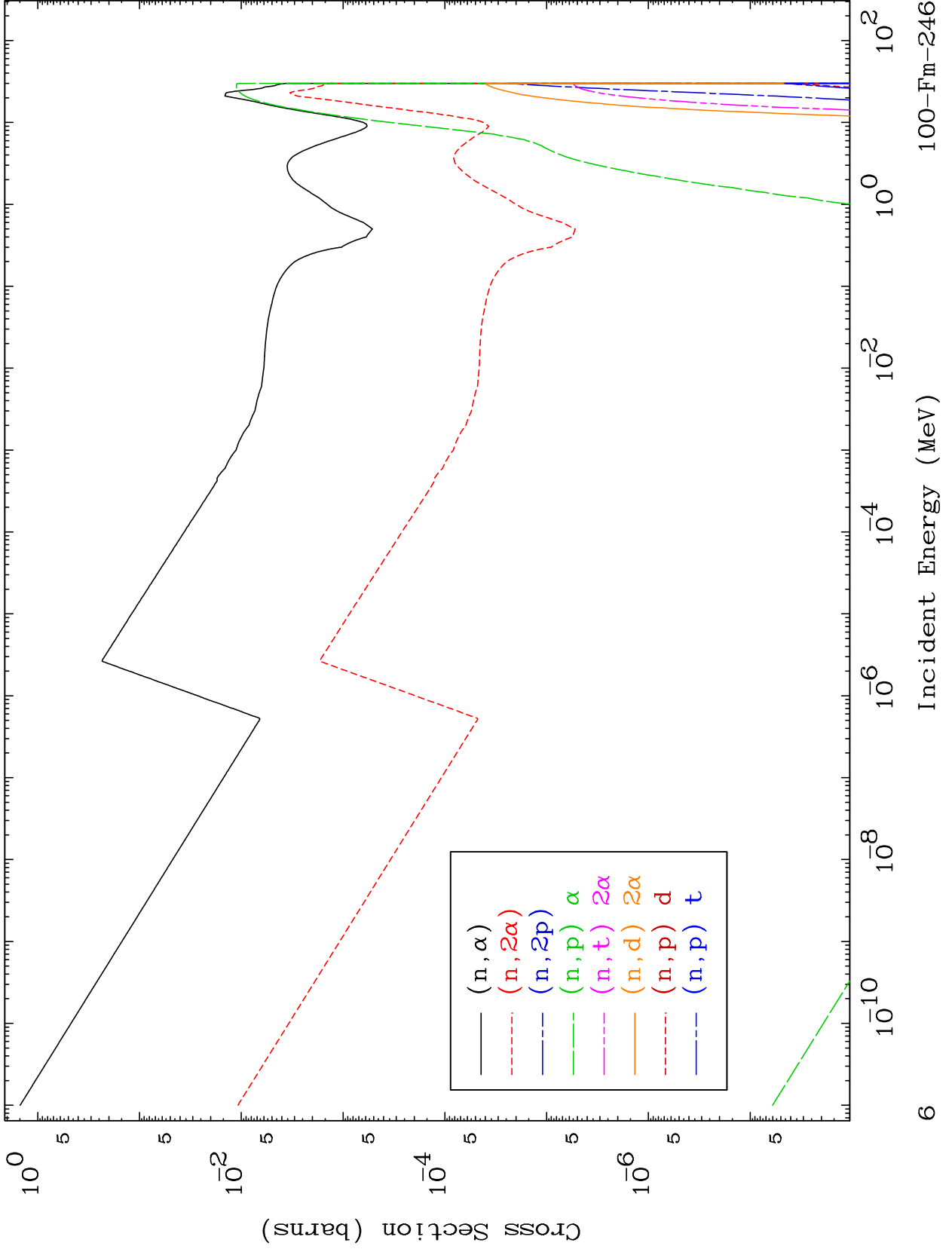
100-Fm-246



MAT 9927

Charged Particle
293 Kelvin Cross Sections

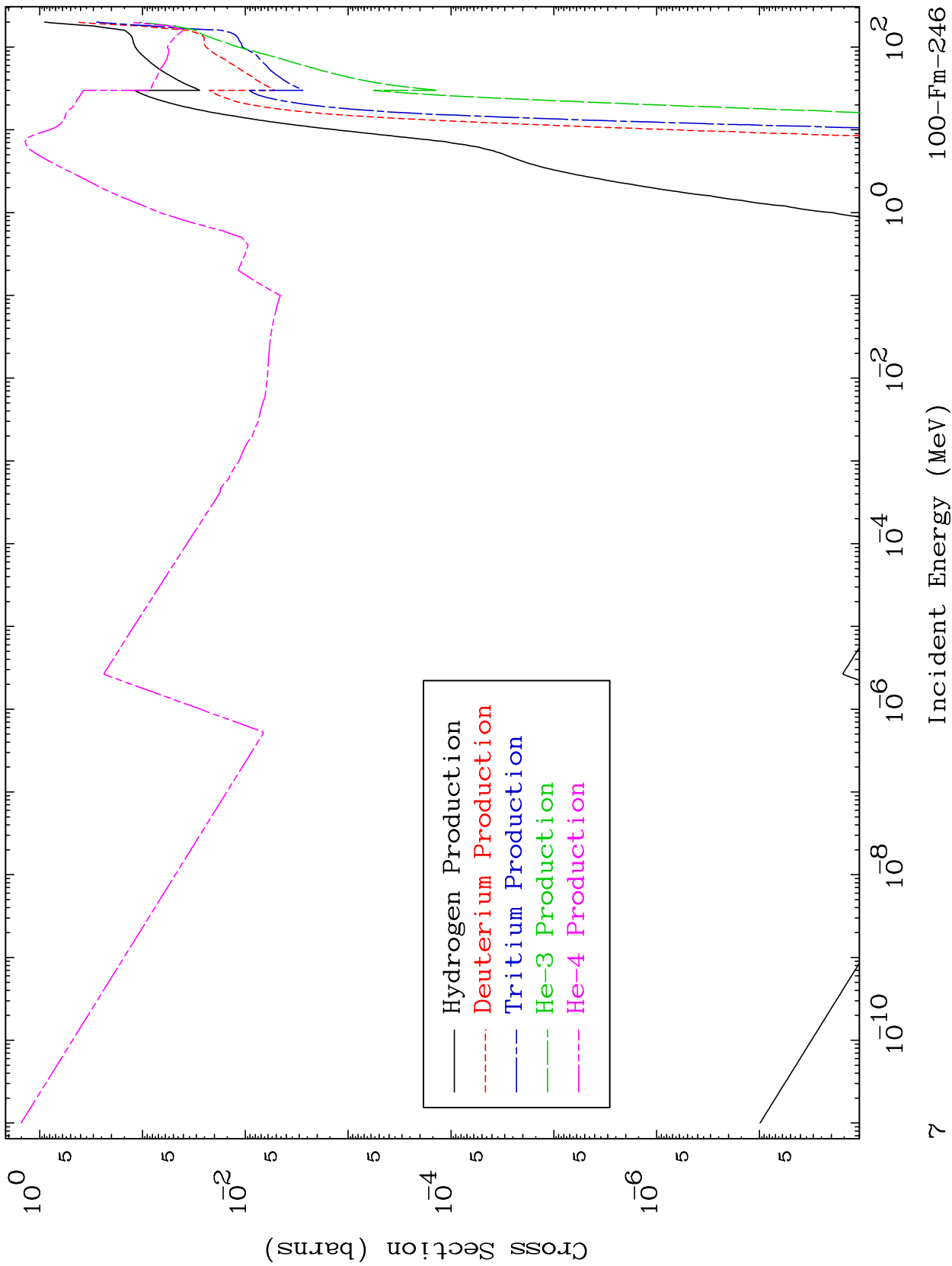
100-Fm-246



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Particle Production
293 Kelvin Cross Sections

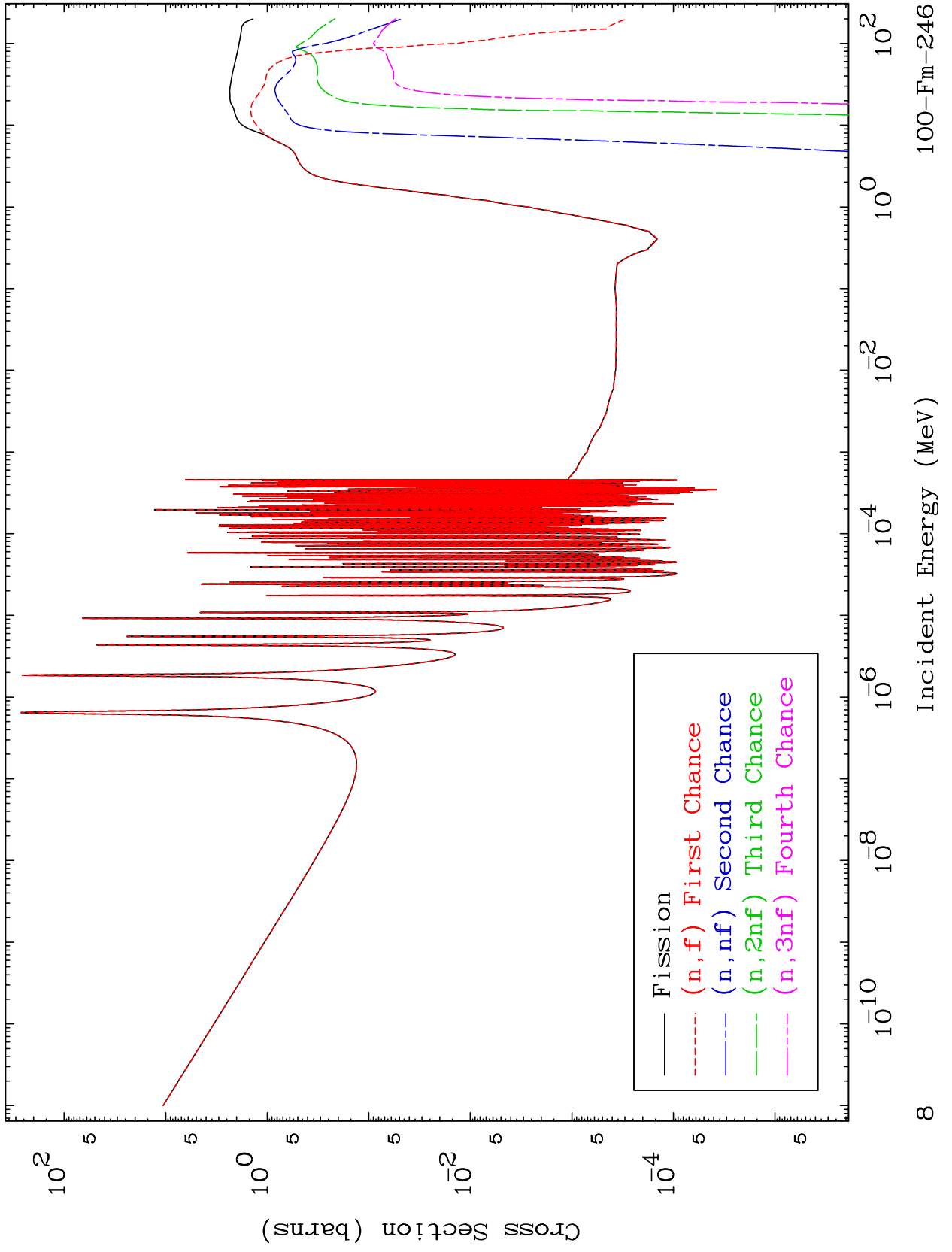
100-Fm-246



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Fission
293 Kelvin Cross Sections

100-Fm-246

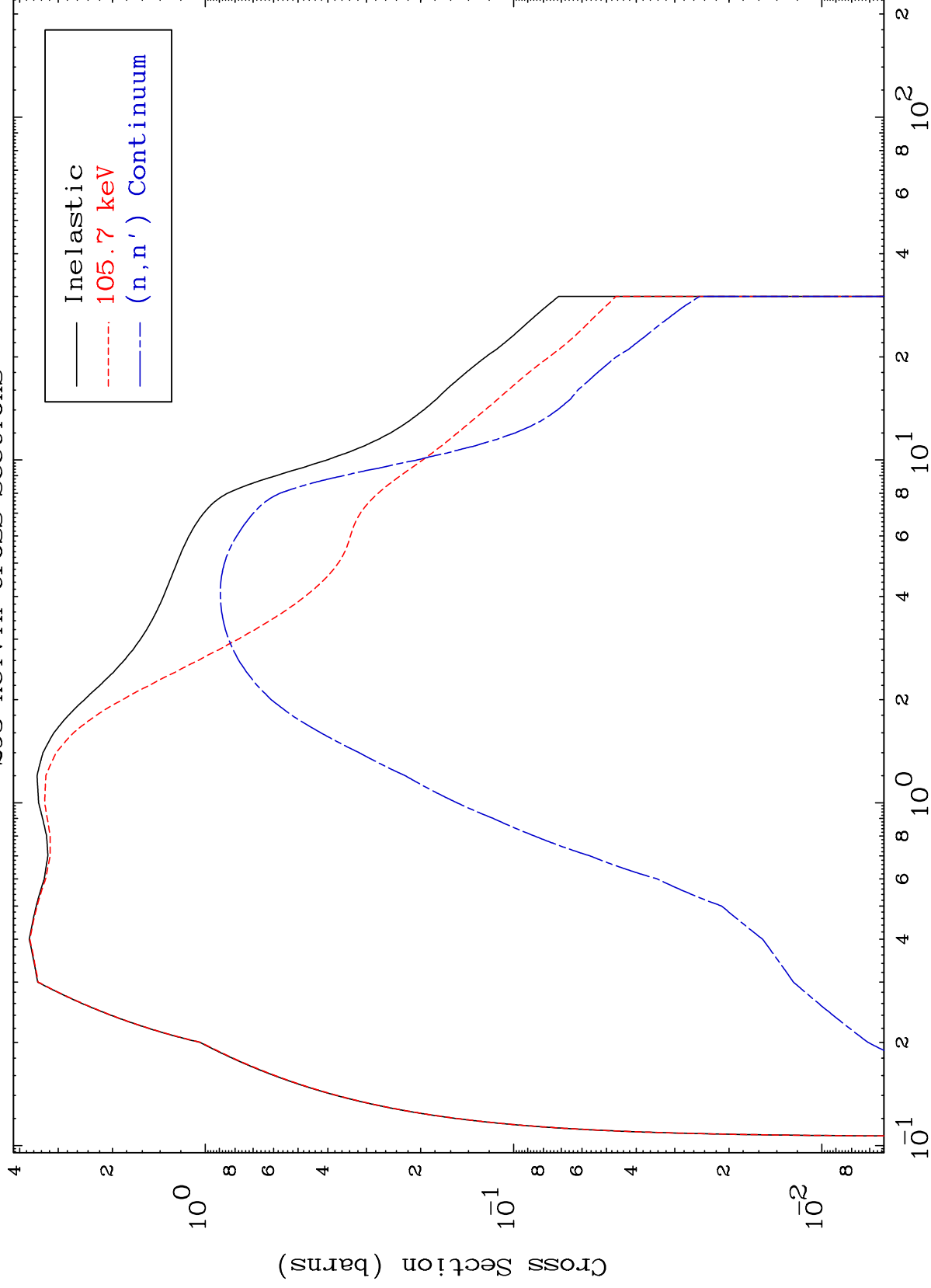


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(n,n') Level

100-Fm-246

293 Kelvin Cross Sections



Incident Energy (MeV)

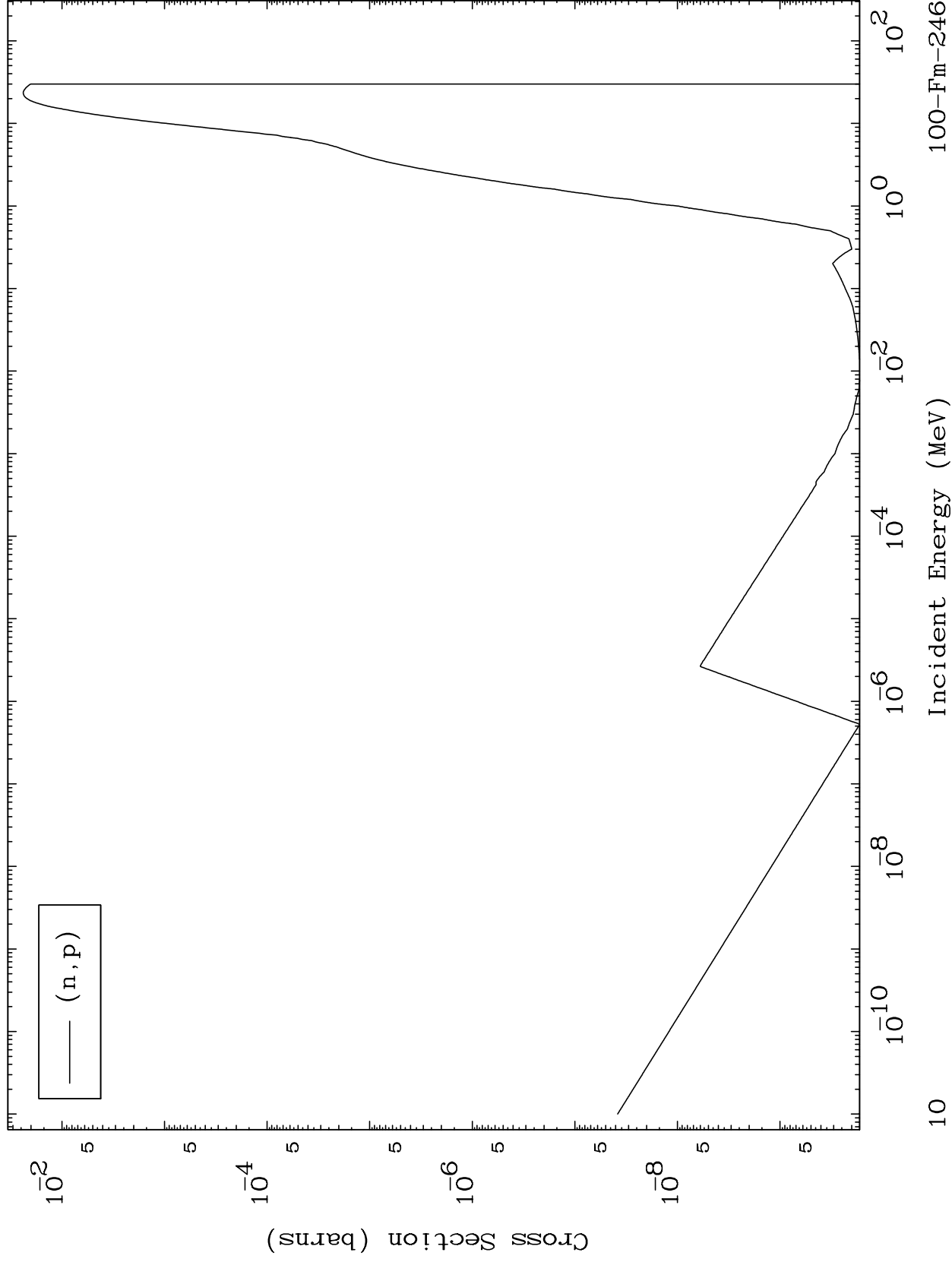
100-Fm-246

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(n,p) Levels
293 Kelvin Cross Sections

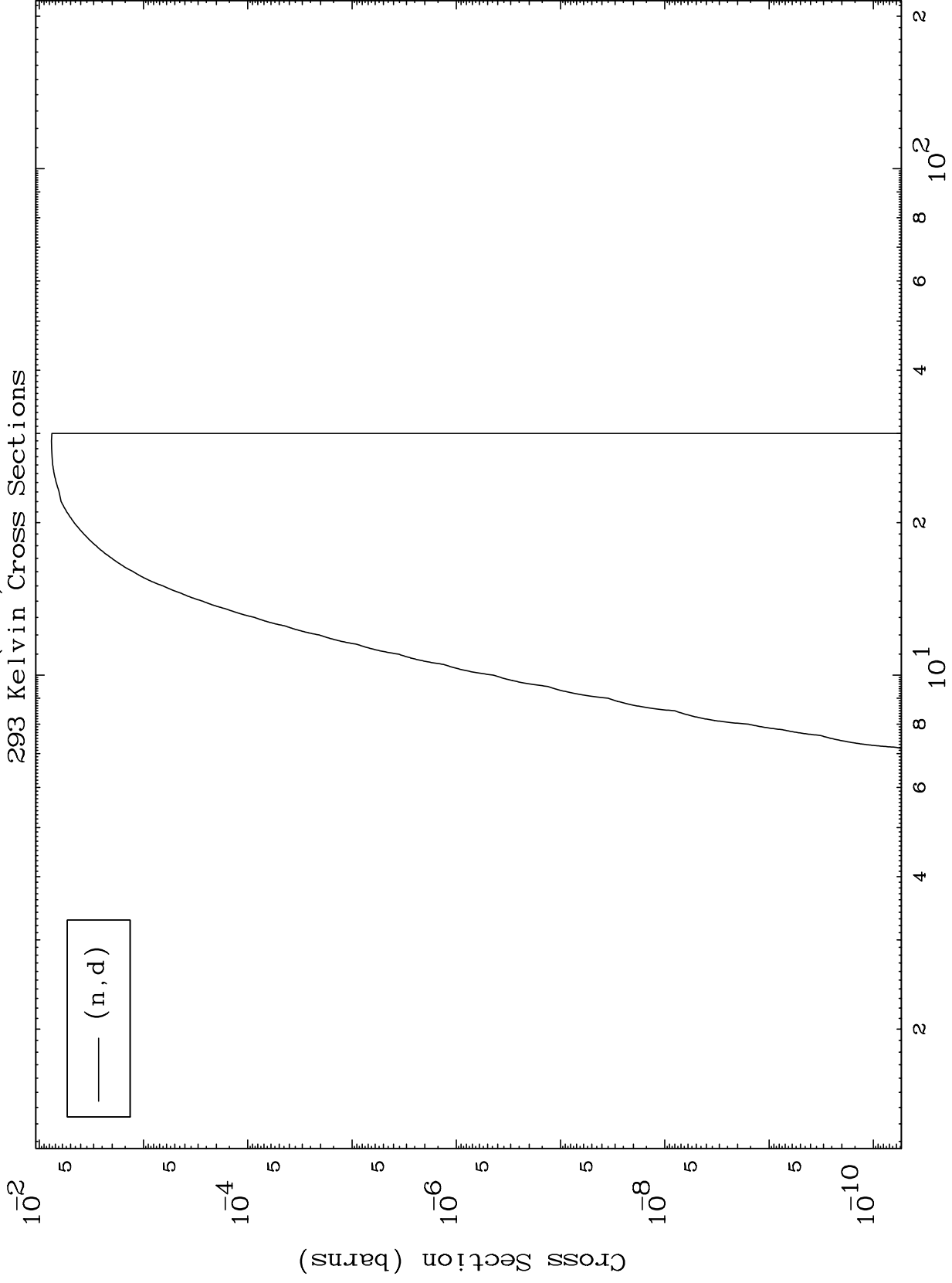
100-Fm-246



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(n,d) Levels
293 Kelvin Cross Sections

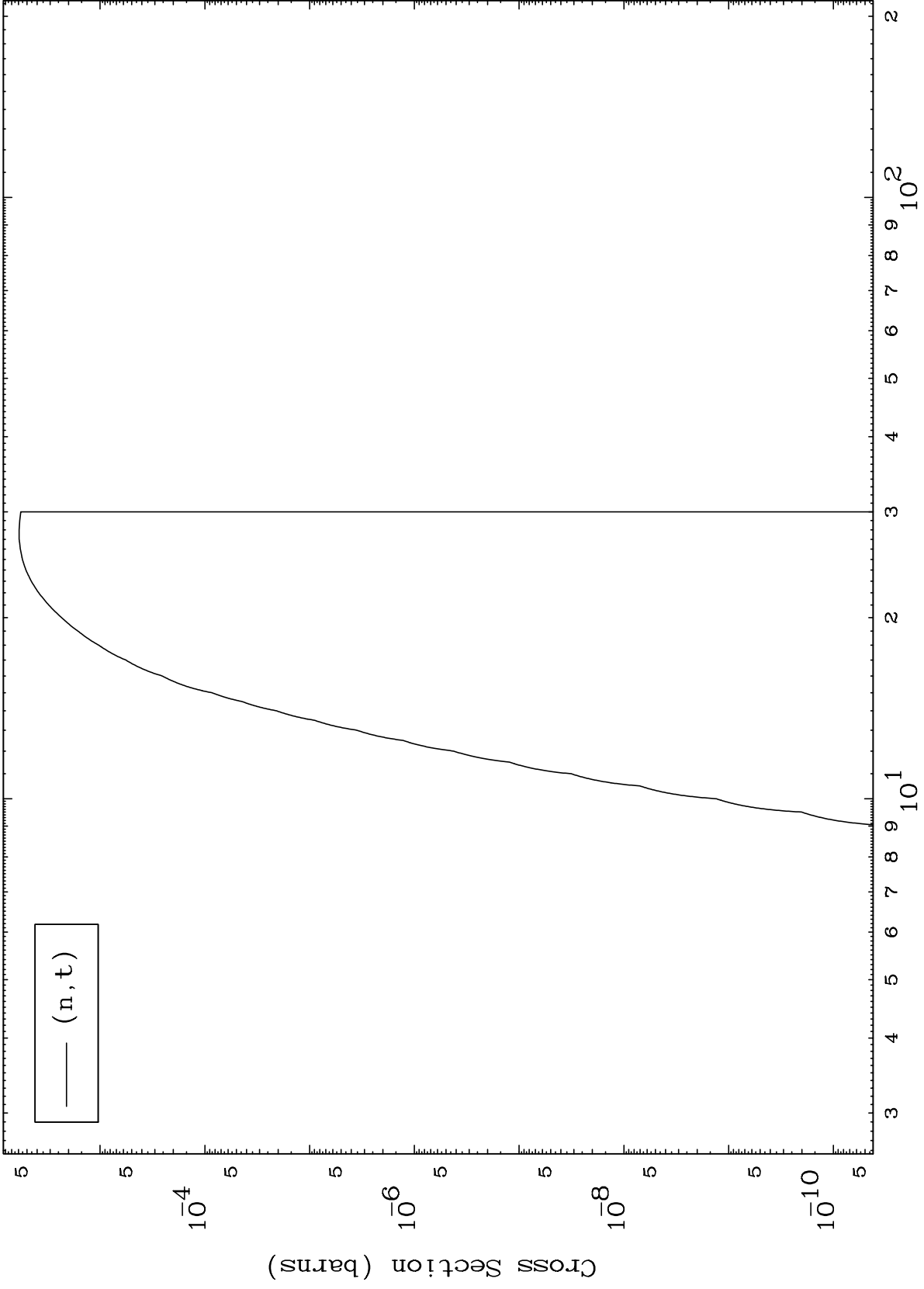
100-Fm-246



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(n,t) Levels
293 Kelvin Cross Sections

100-Fm-246



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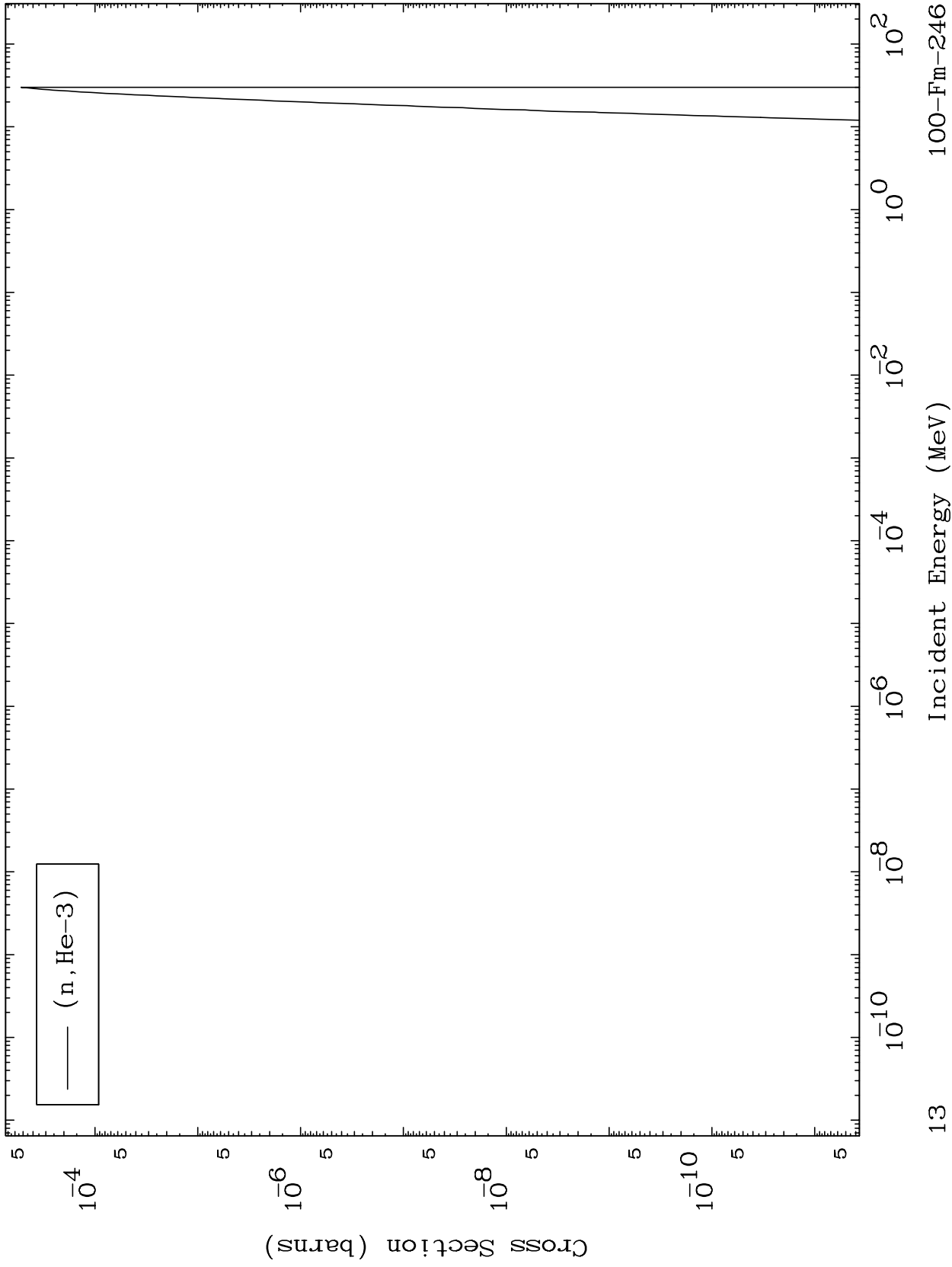
Incident Energy (MeV)

100-Fm-246

MAT 9927

(n,He3) Levels
293 Kelvin Cross Sections

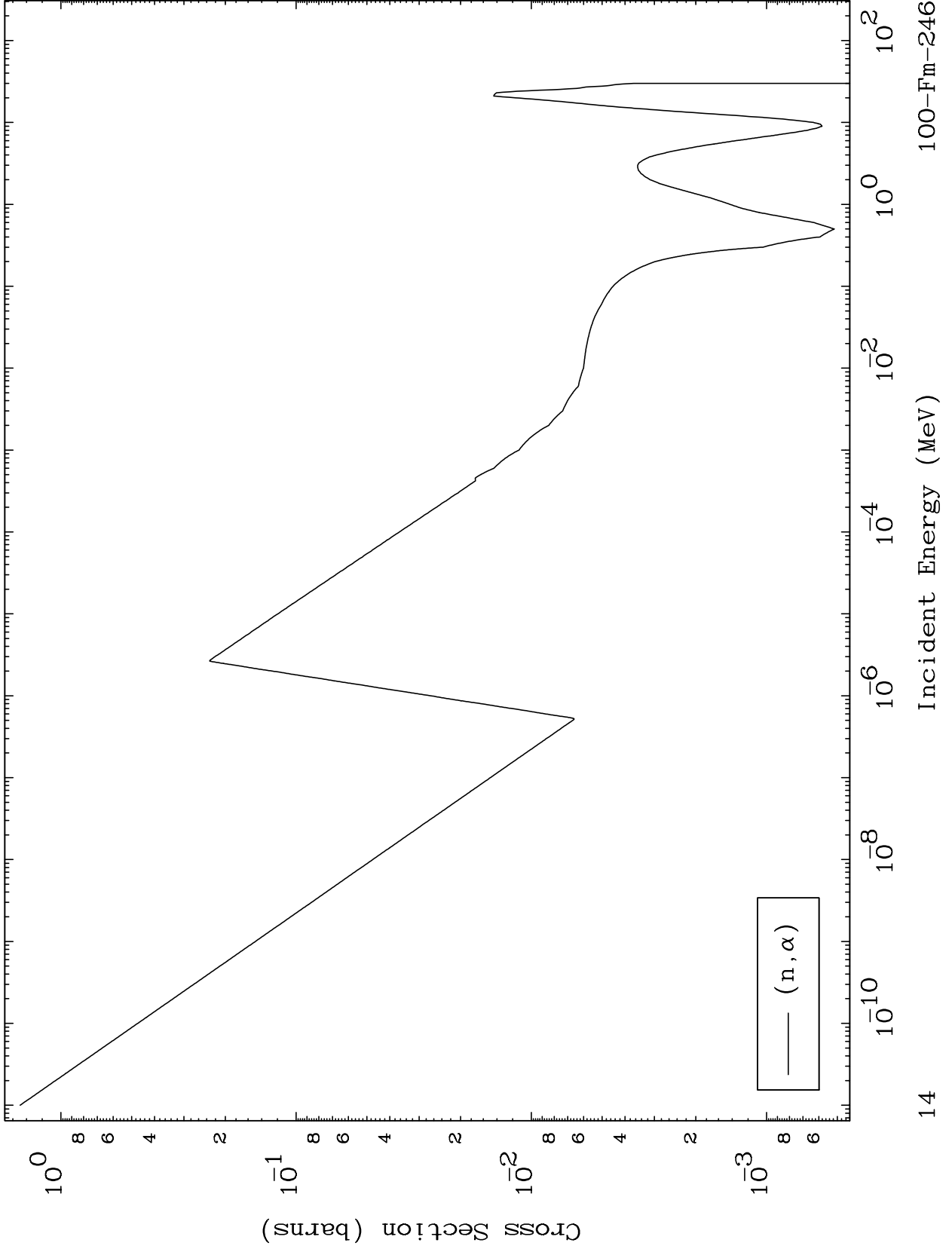
100-Fm-246

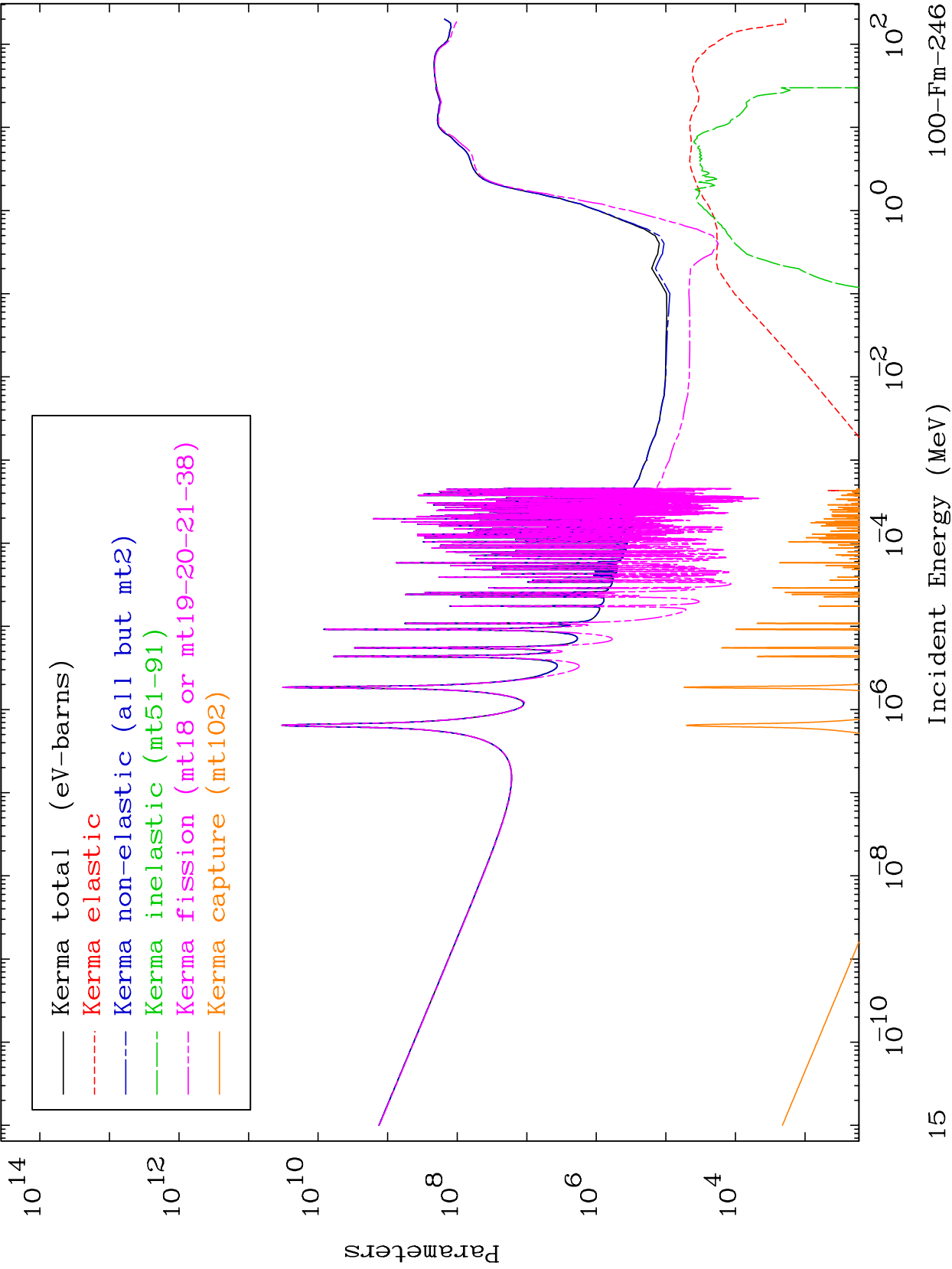


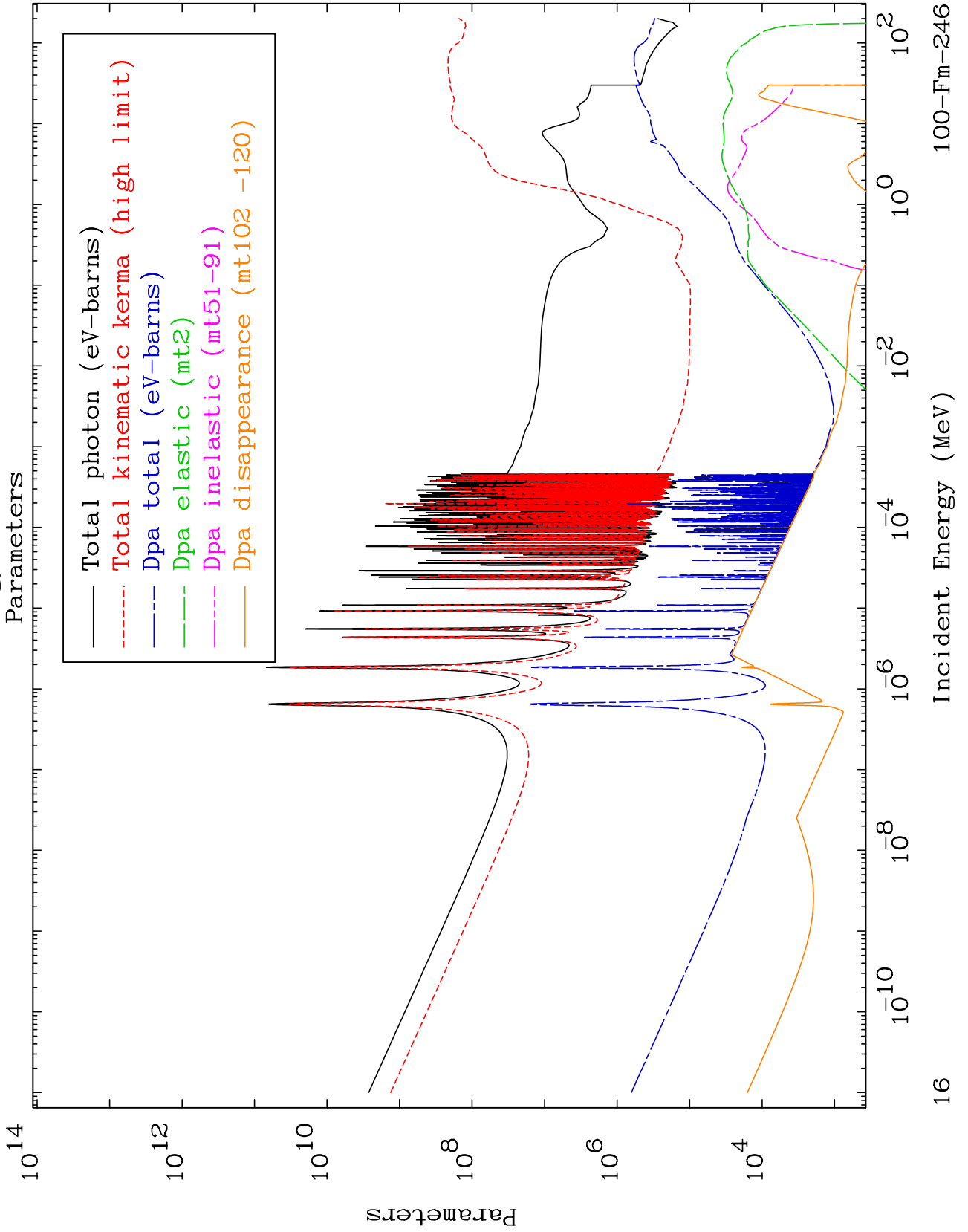
MAT 9927

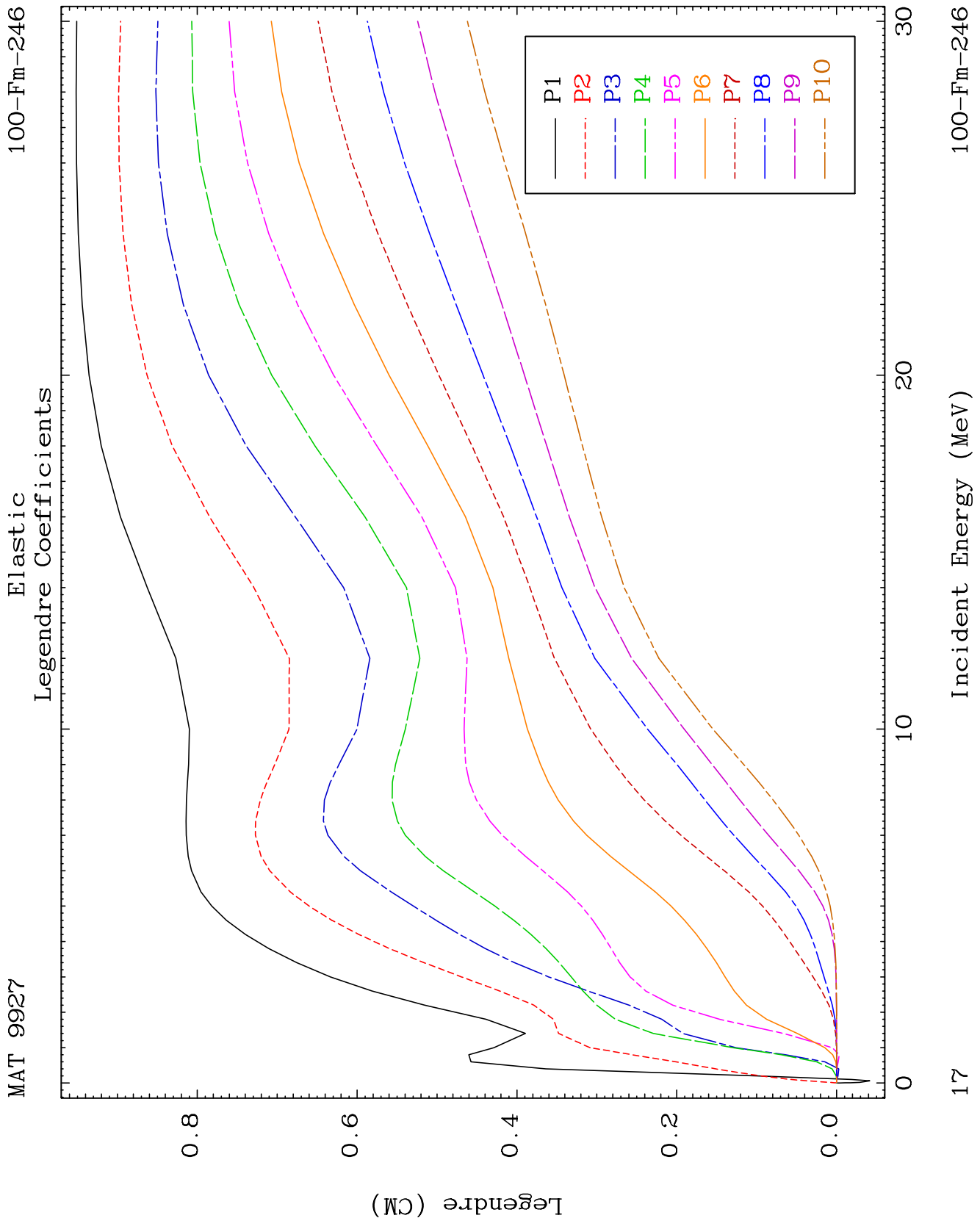
(n, α) Levels
293 Kelvin Cross Sections

100-Fm-246





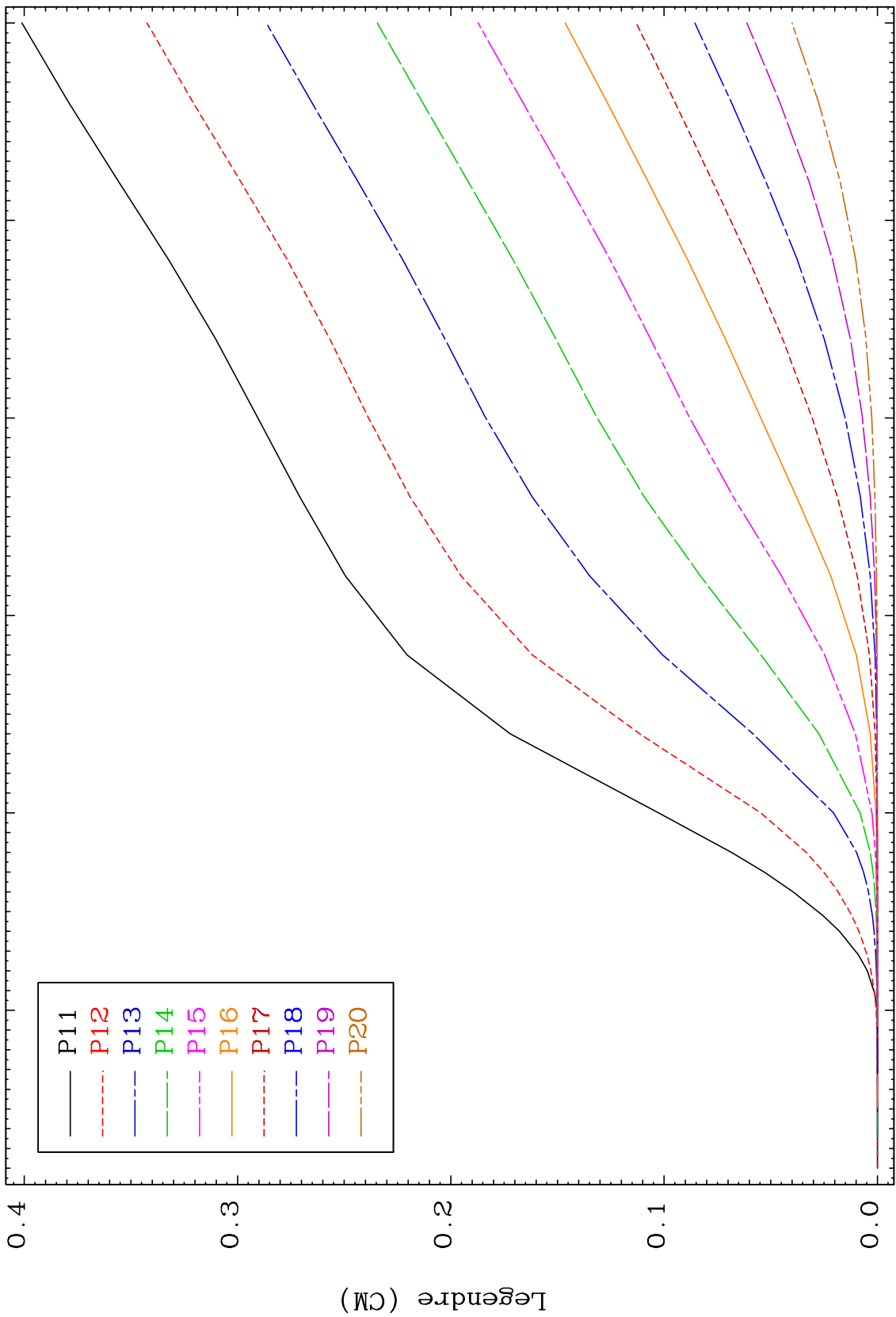




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Elastic Legendre Coefficients

100-Fm-246



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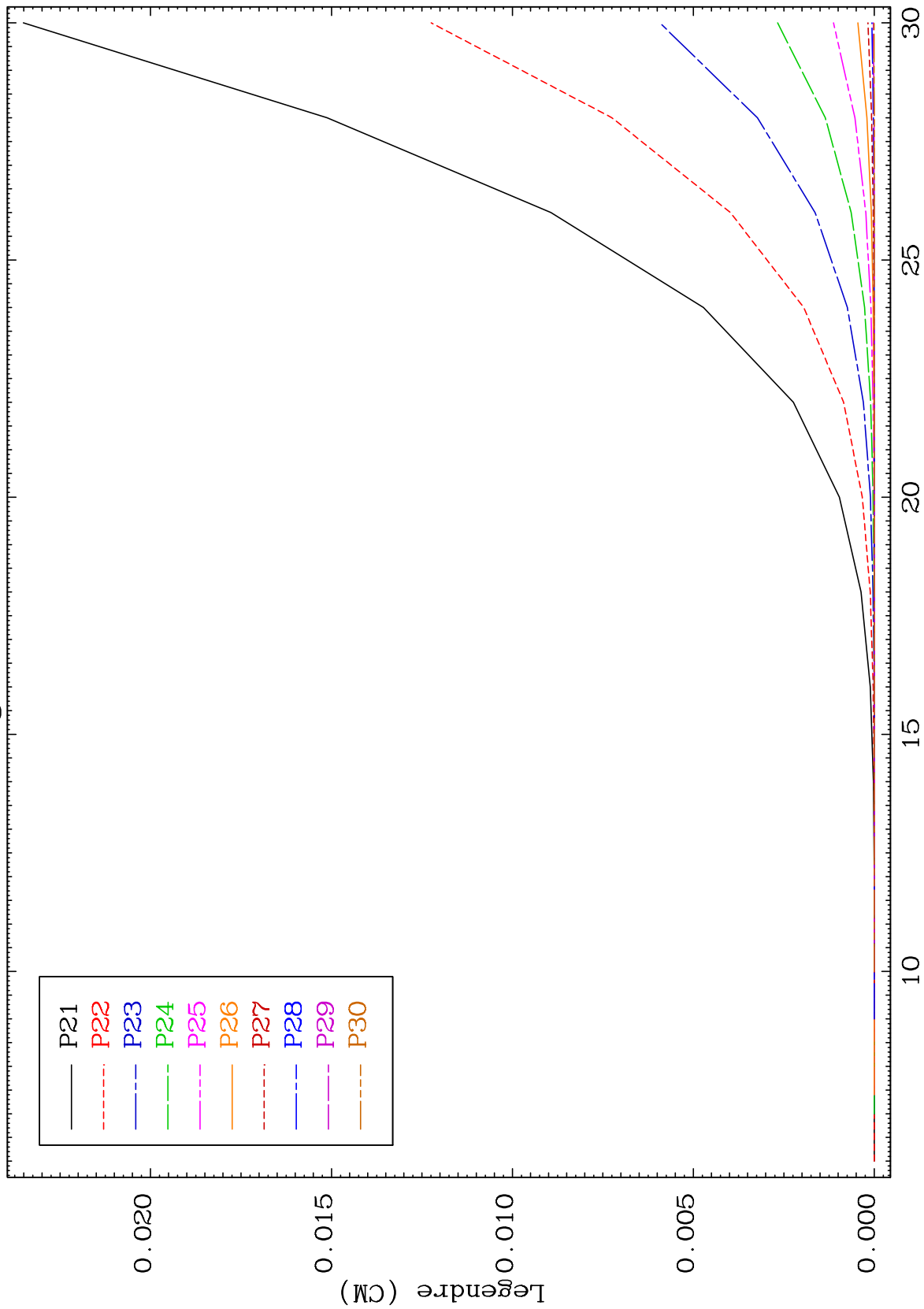
Incident Energy (MeV)

100-Fm-246

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Elastic Legendre Coefficients

100-Fm-246



19

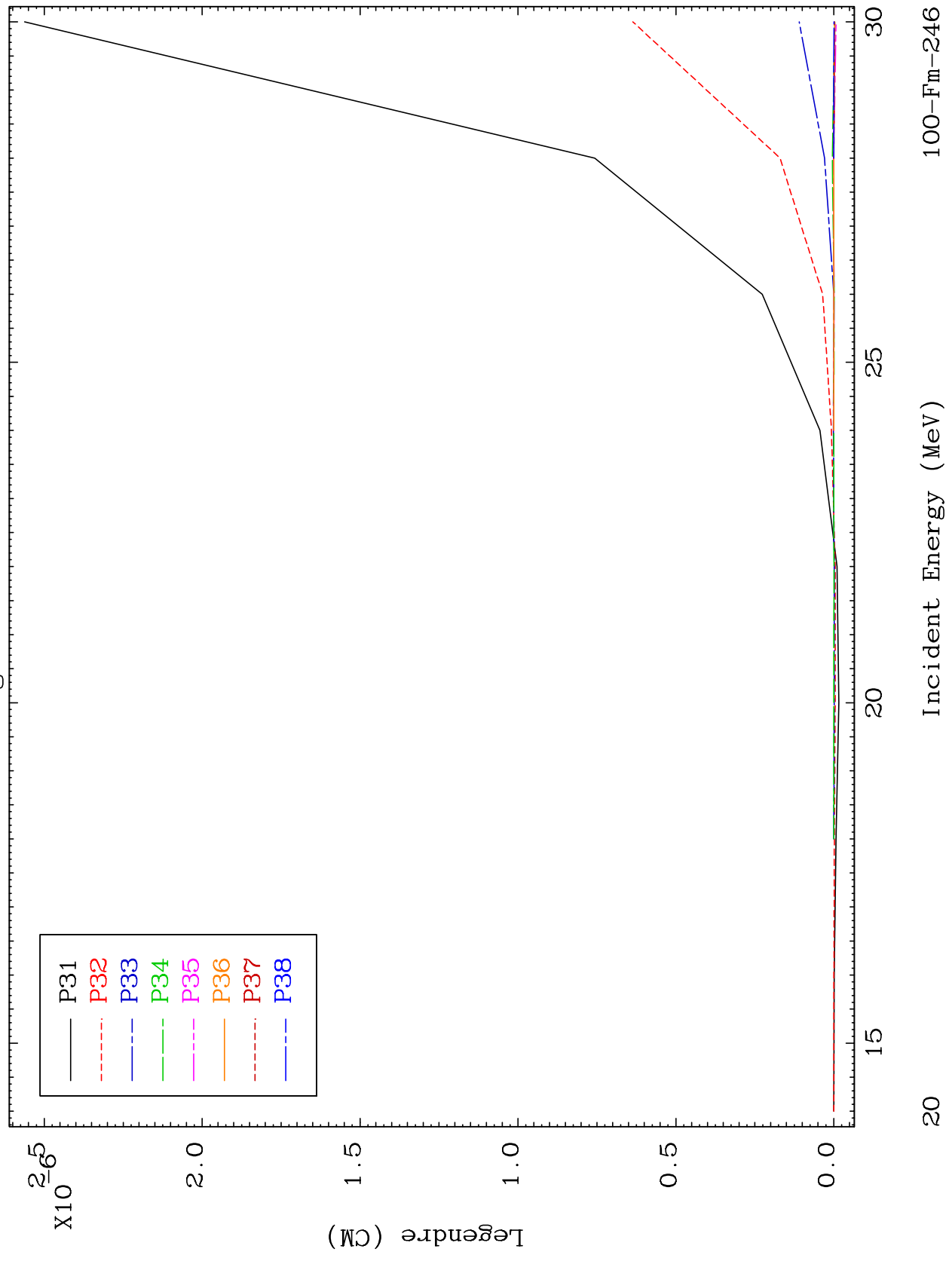
Incident Energy (MeV)

100-Fm-246

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Elastic Legendre Coefficients

100-Fm-246



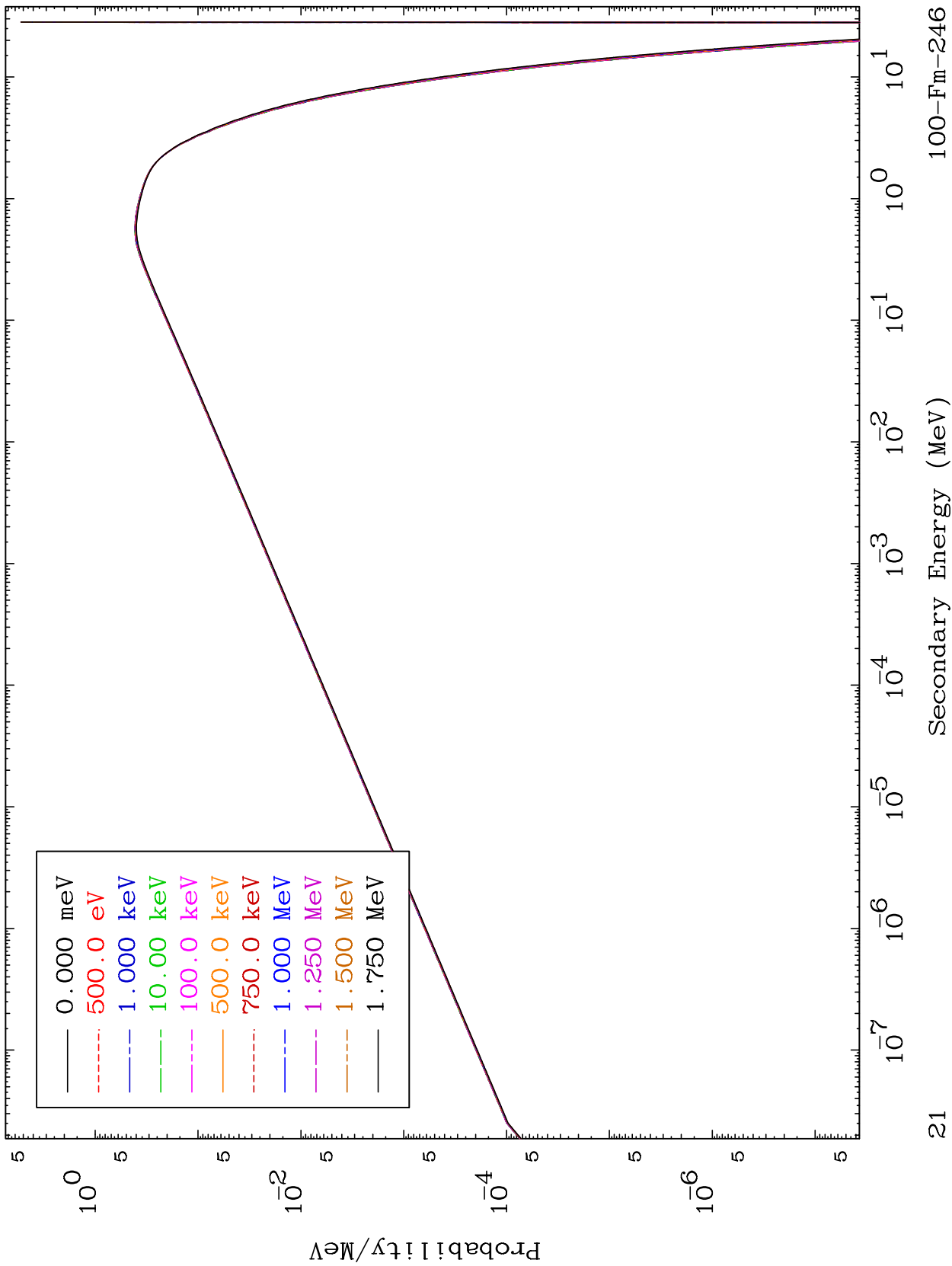
20

Incident Energy (MeV)

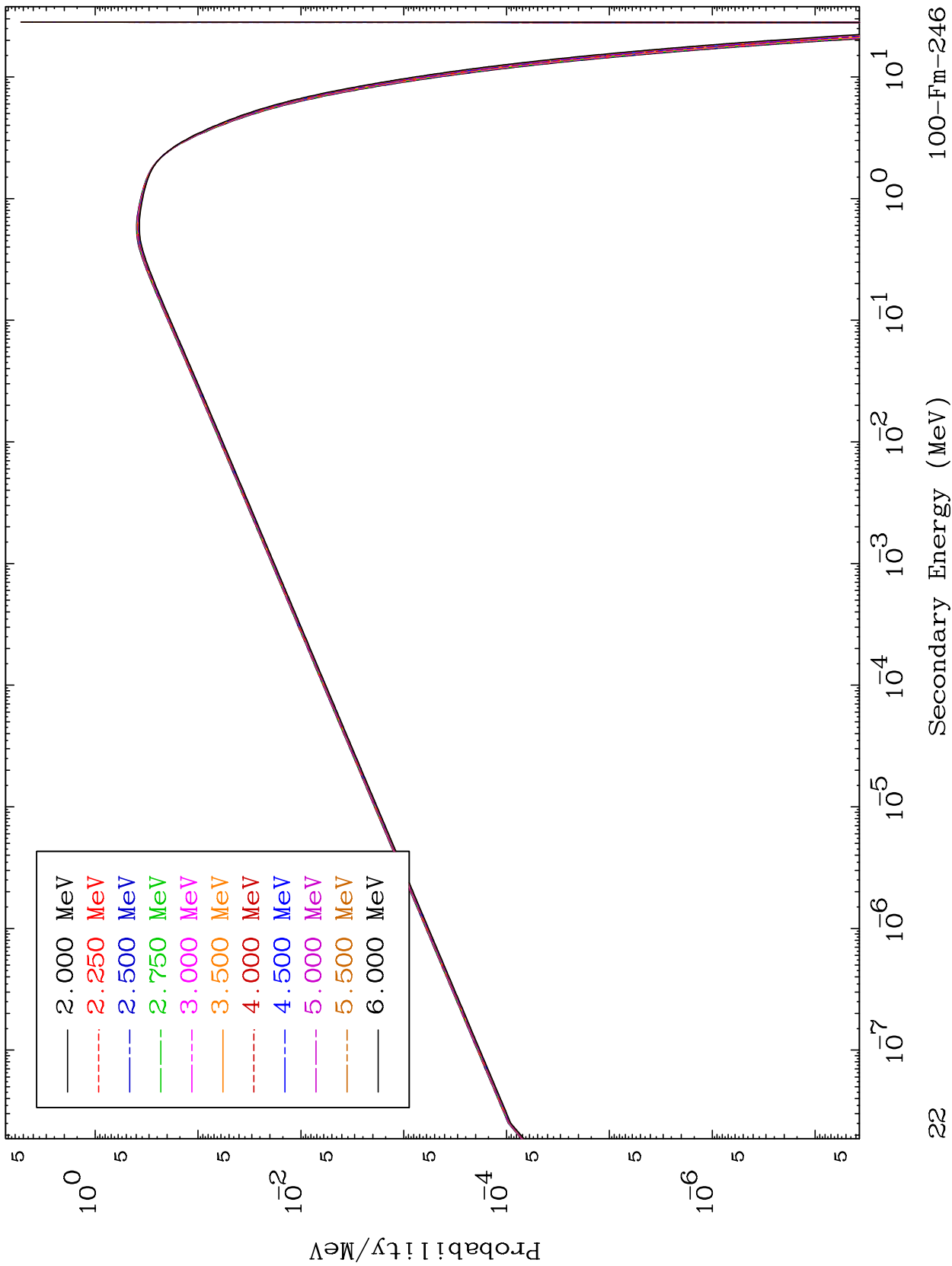
100-Fm-246

$\times 10^{-6}$
Legendre (CM)

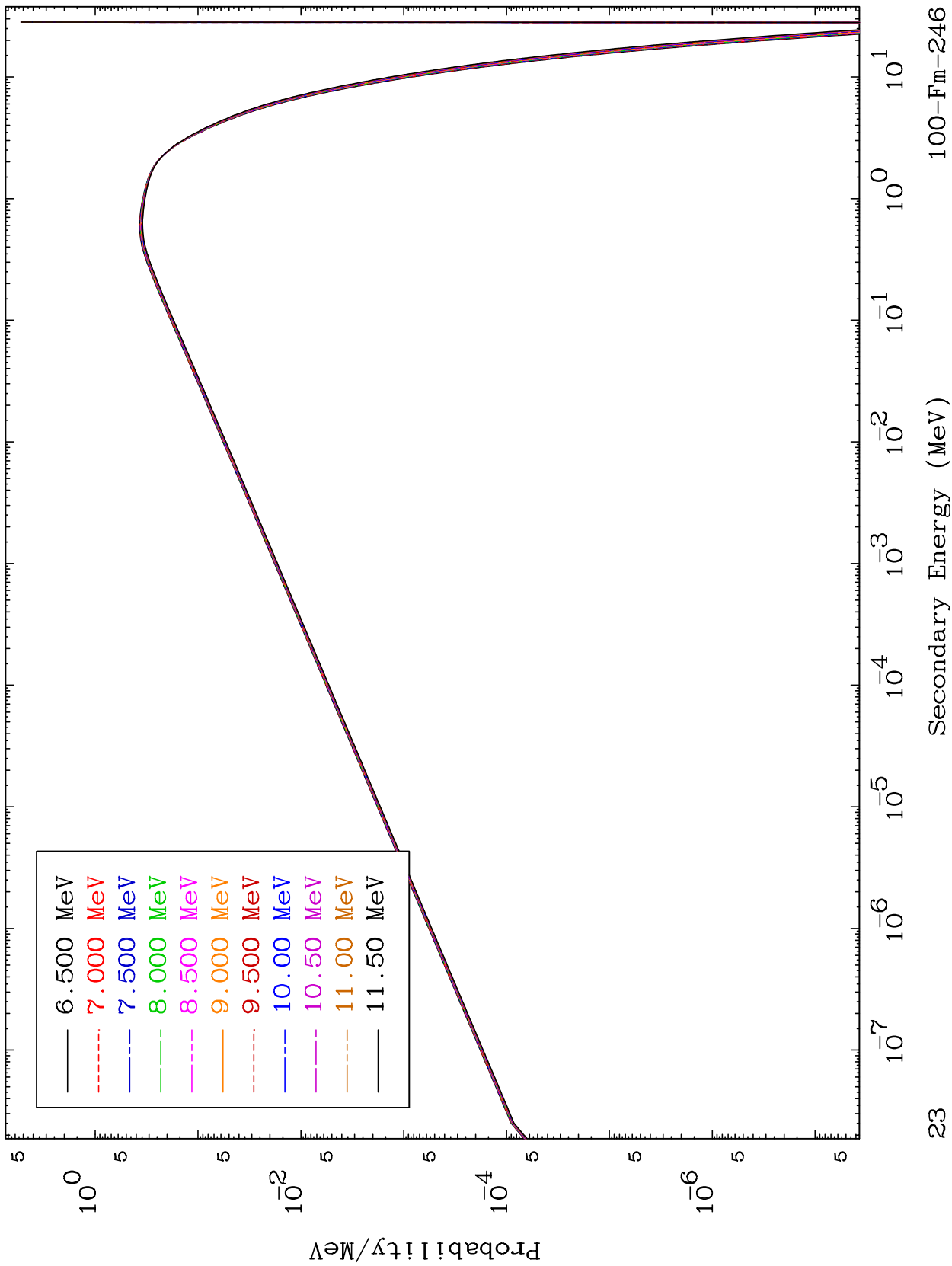
Fission Energy Distribution



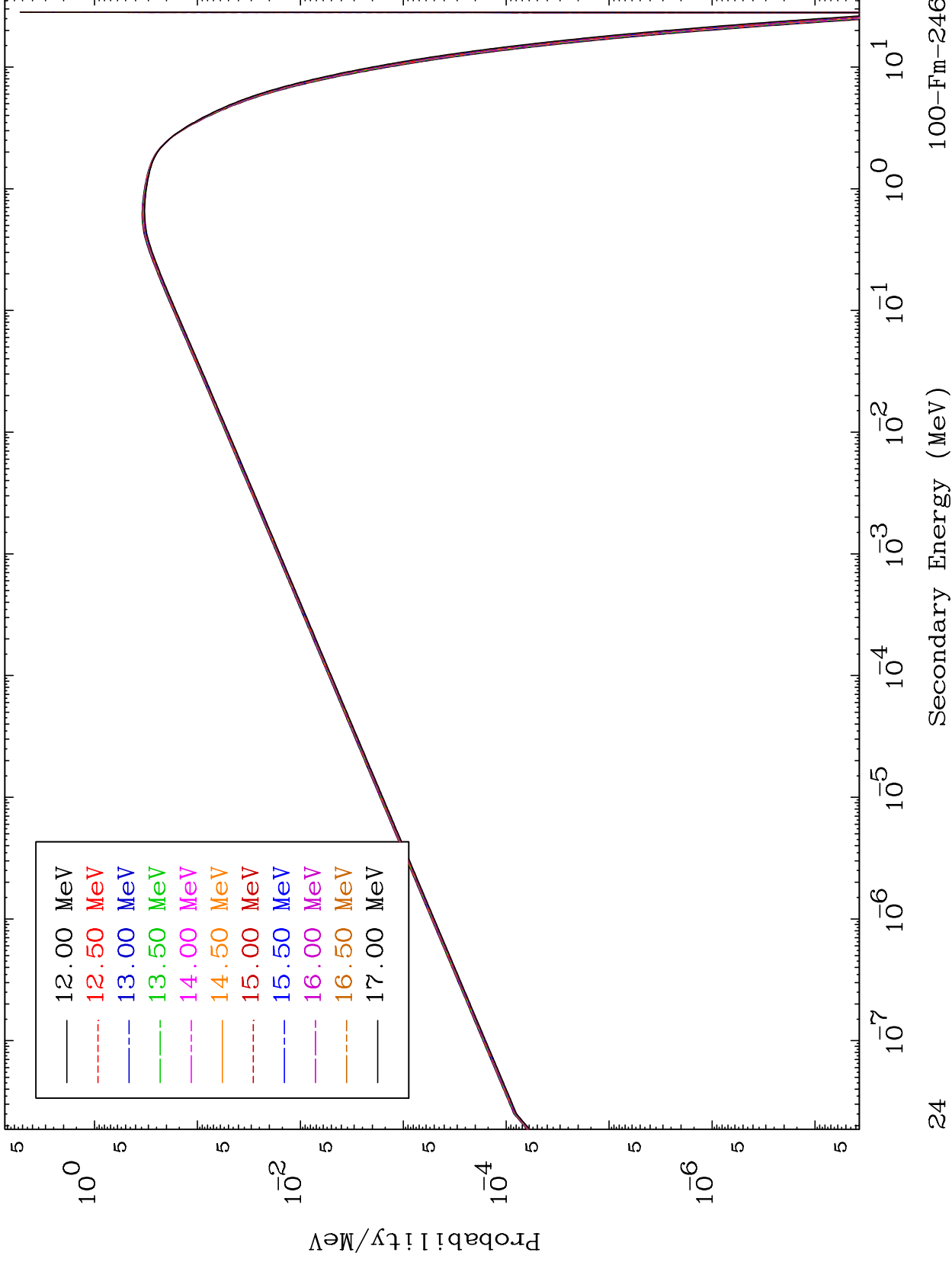
Fission Energy Distribution



Fission Energy Distribution



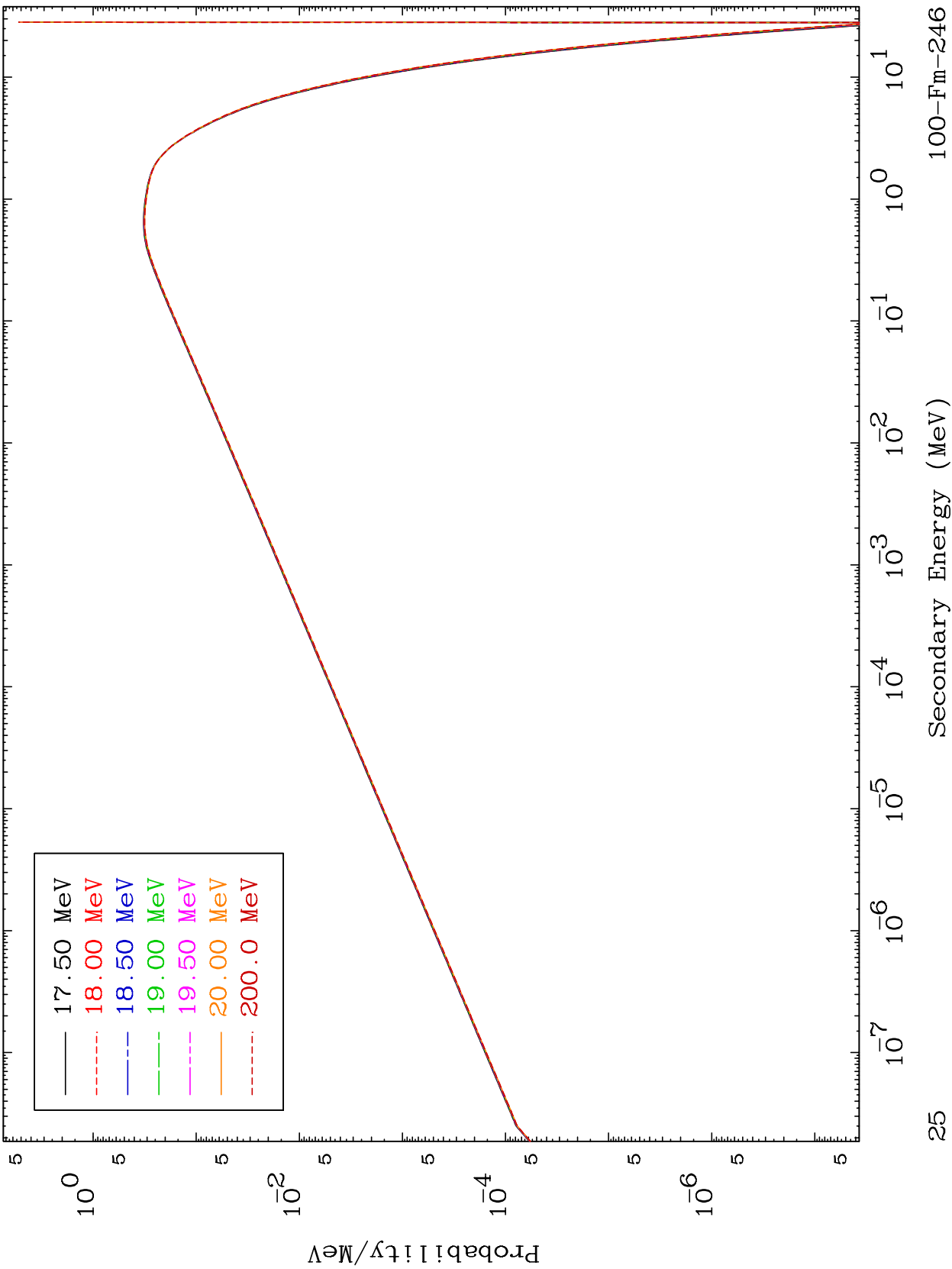
Fission
Energy Distribution



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Fission Energy Distribution

100-Fm-246

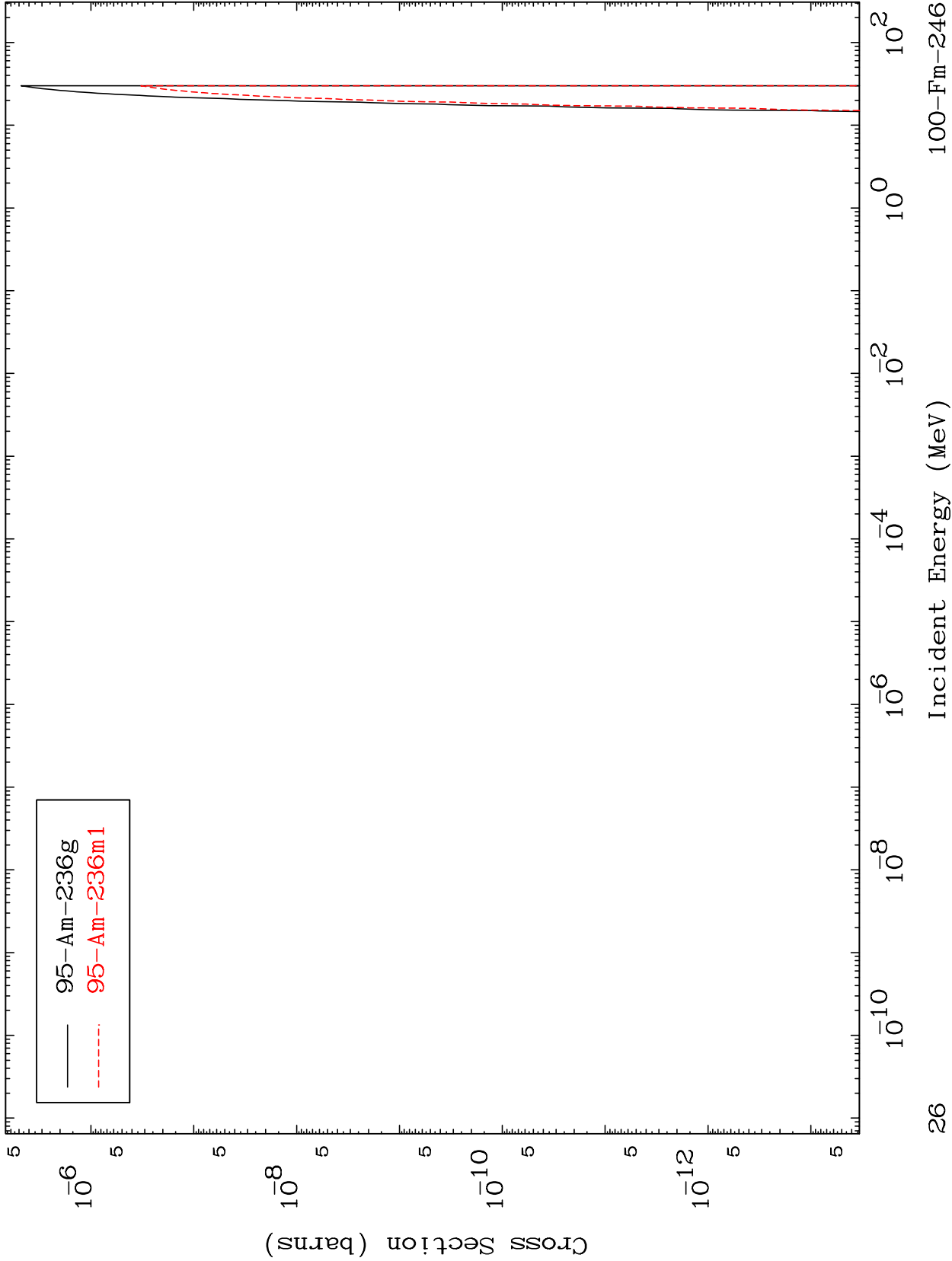


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(n,n') d,2α

100-Fm-246

Radionuclide Production Cross Section

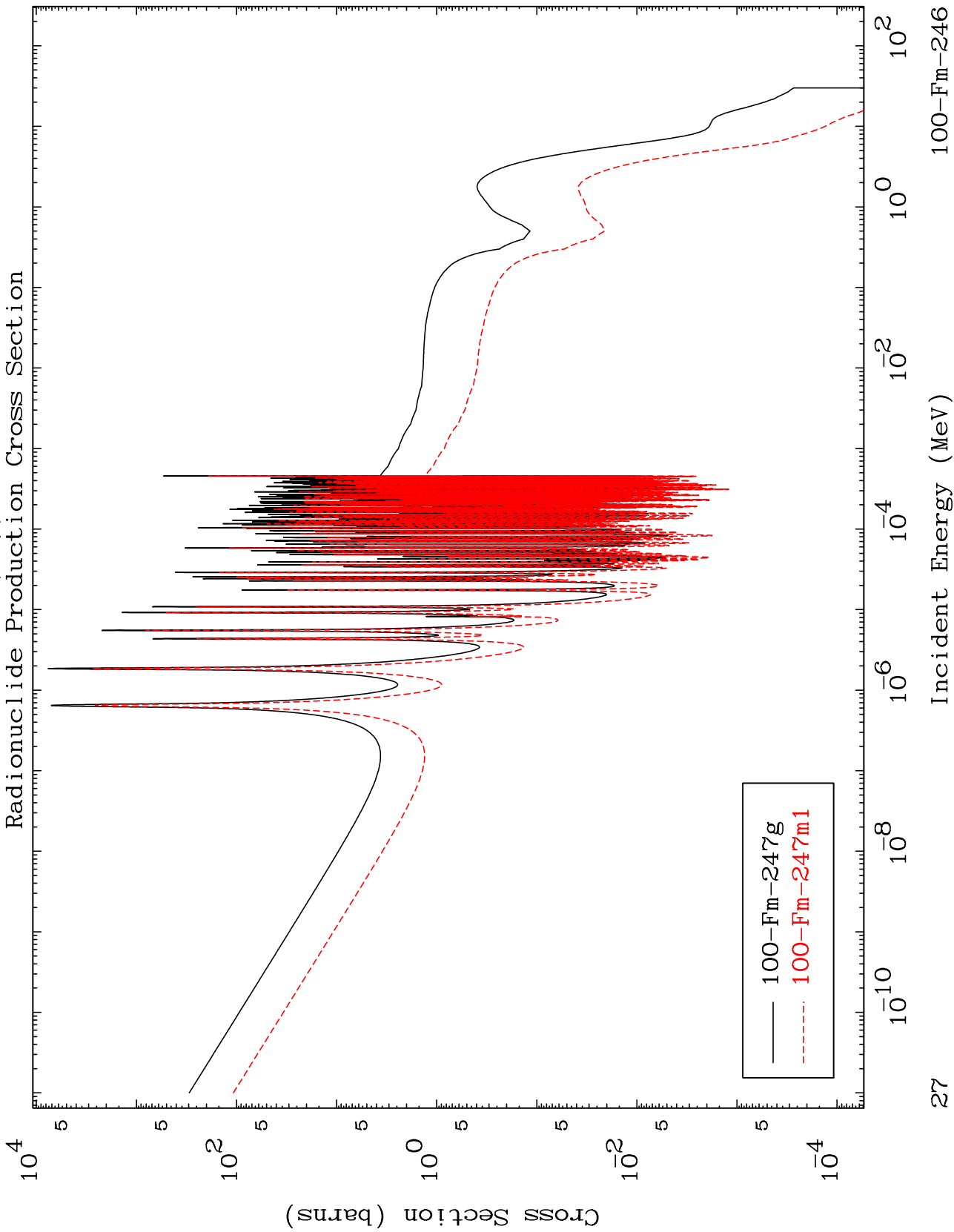


95-Am-236g
95-Am-236m1

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100-Fm-246

Radionuclide Production Cross Section
(n, γ)



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(n,t) 2 α

100-Fm-246

Radionuclide Production Cross Section

