

Program EVALPLOT
(Version 2015-2)

by

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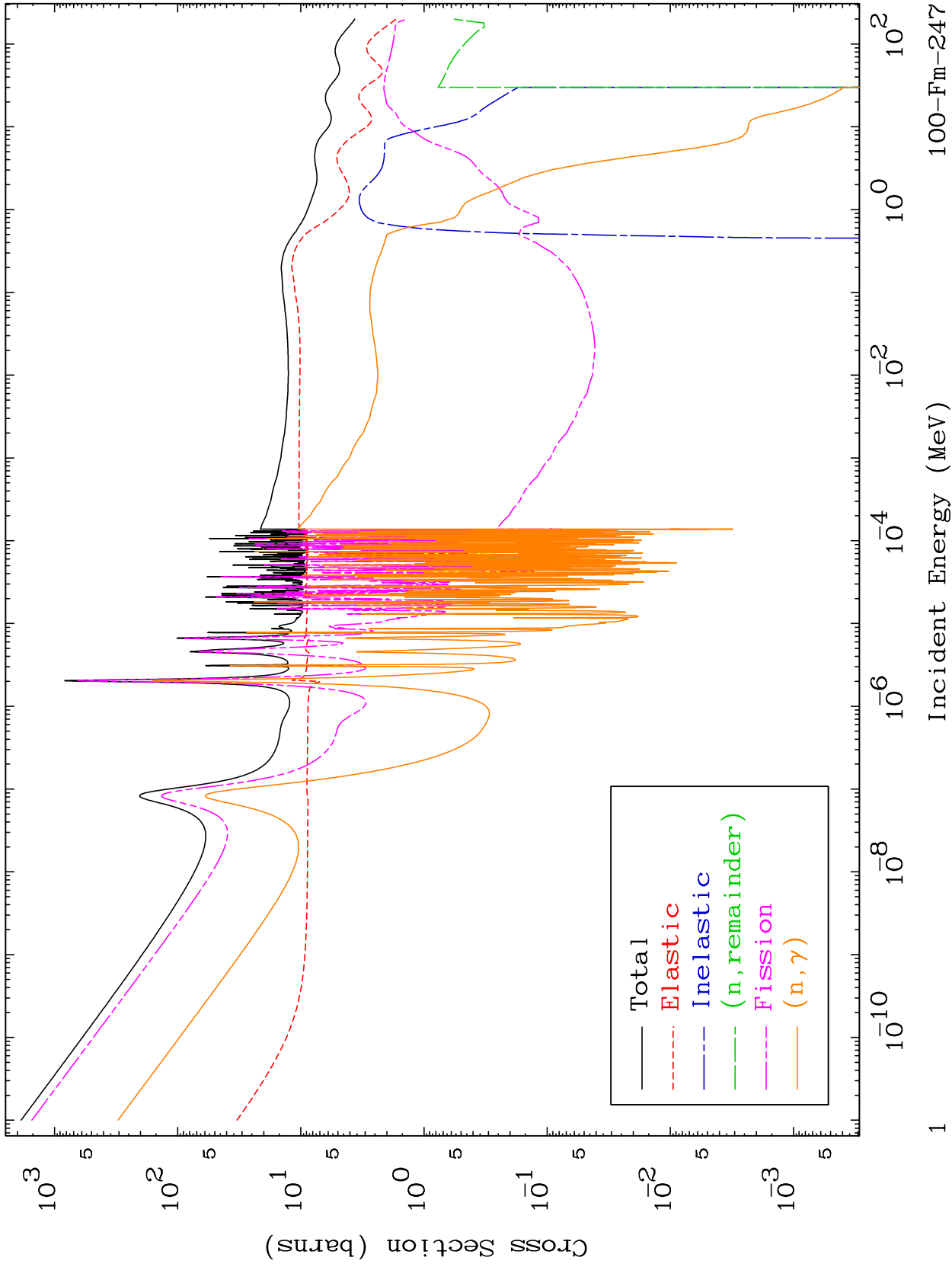
E.Mail:redcullen1@comcast.net
Web:home.comcast.net/~redcullen1

Press Mouse Button to Start

MAT 9928

Major
293 Kelvin Cross Sections

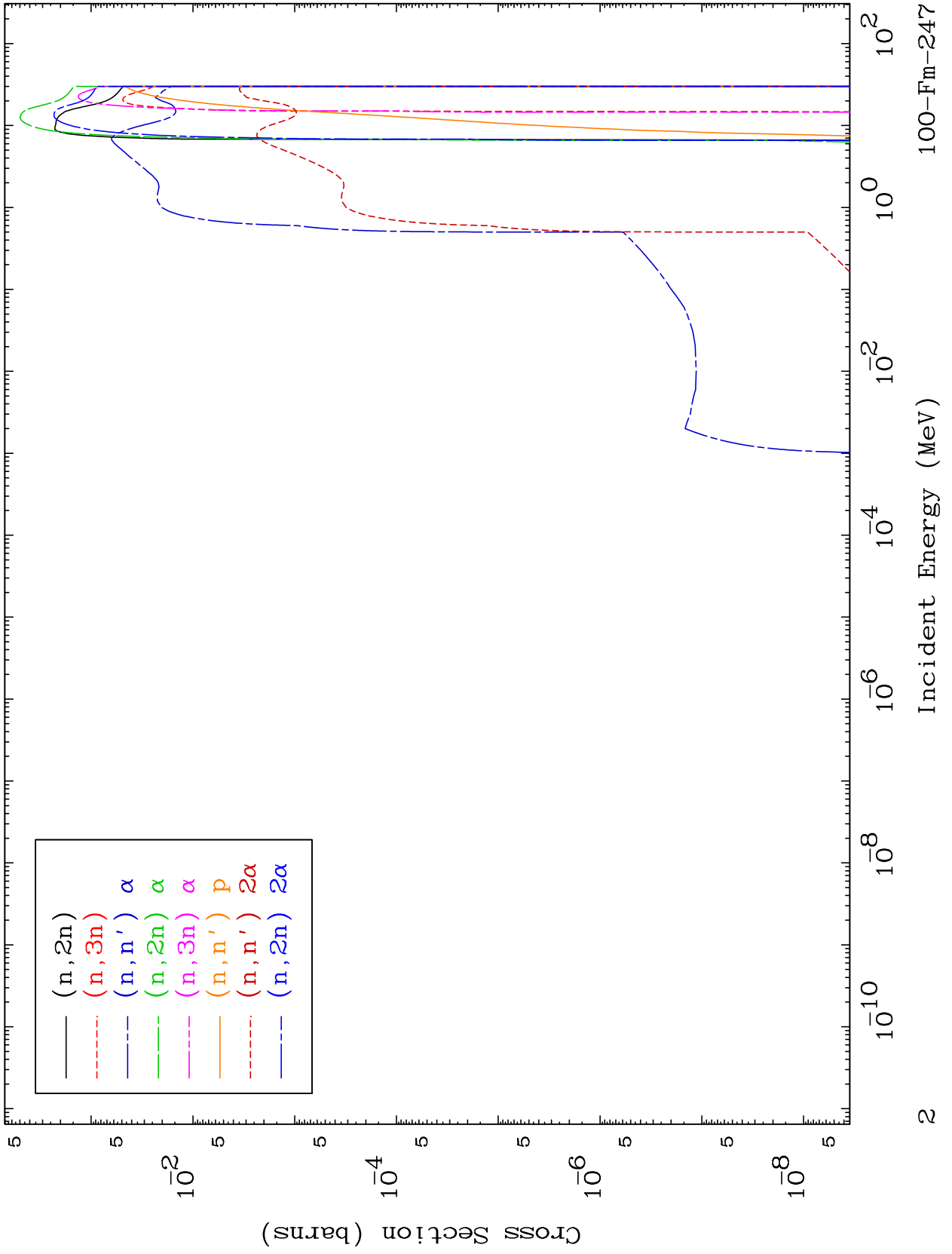
100-Fm-247

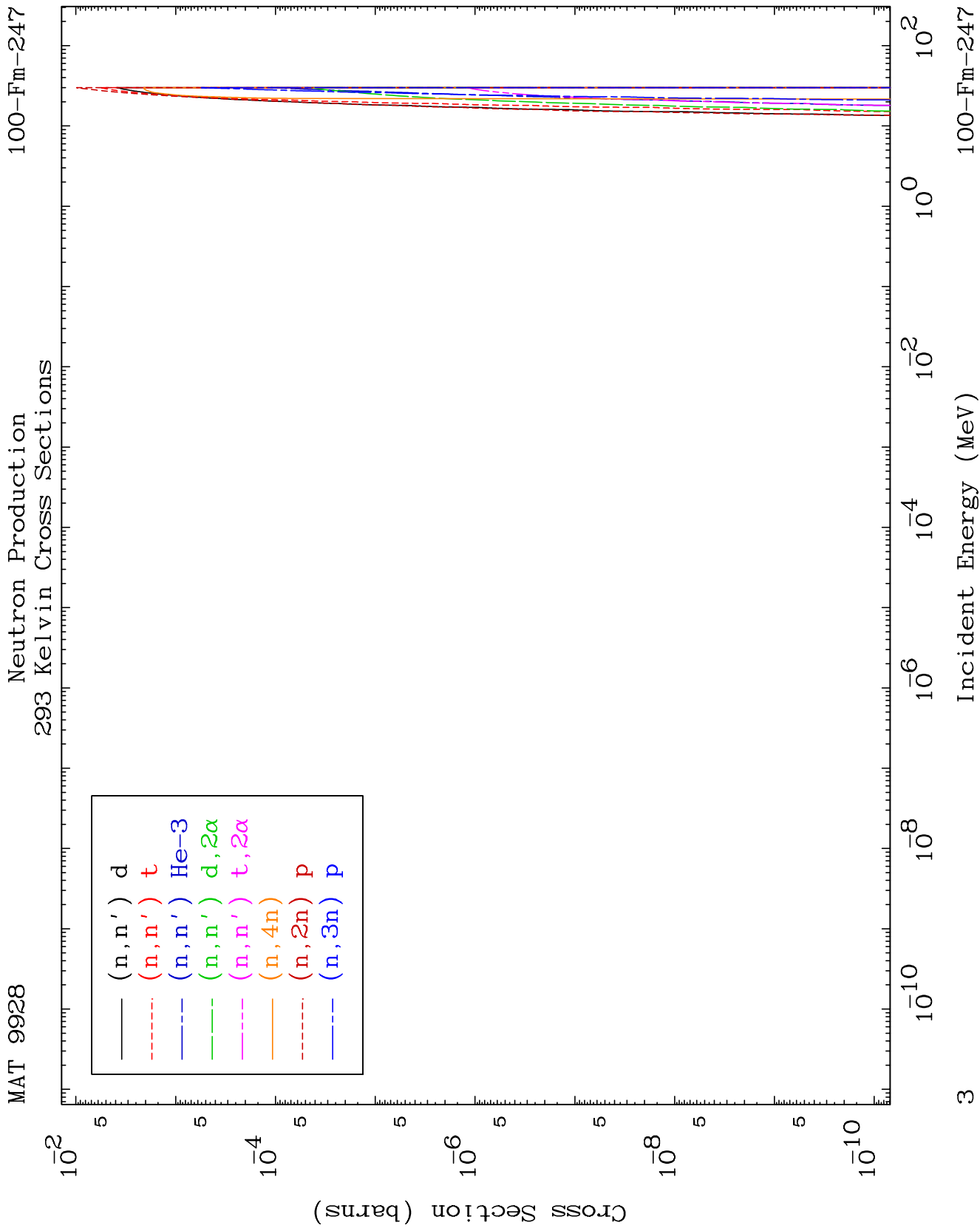


MAT 9928

Neutron Production
293 Kelvin Cross Sections

100-Fm-247

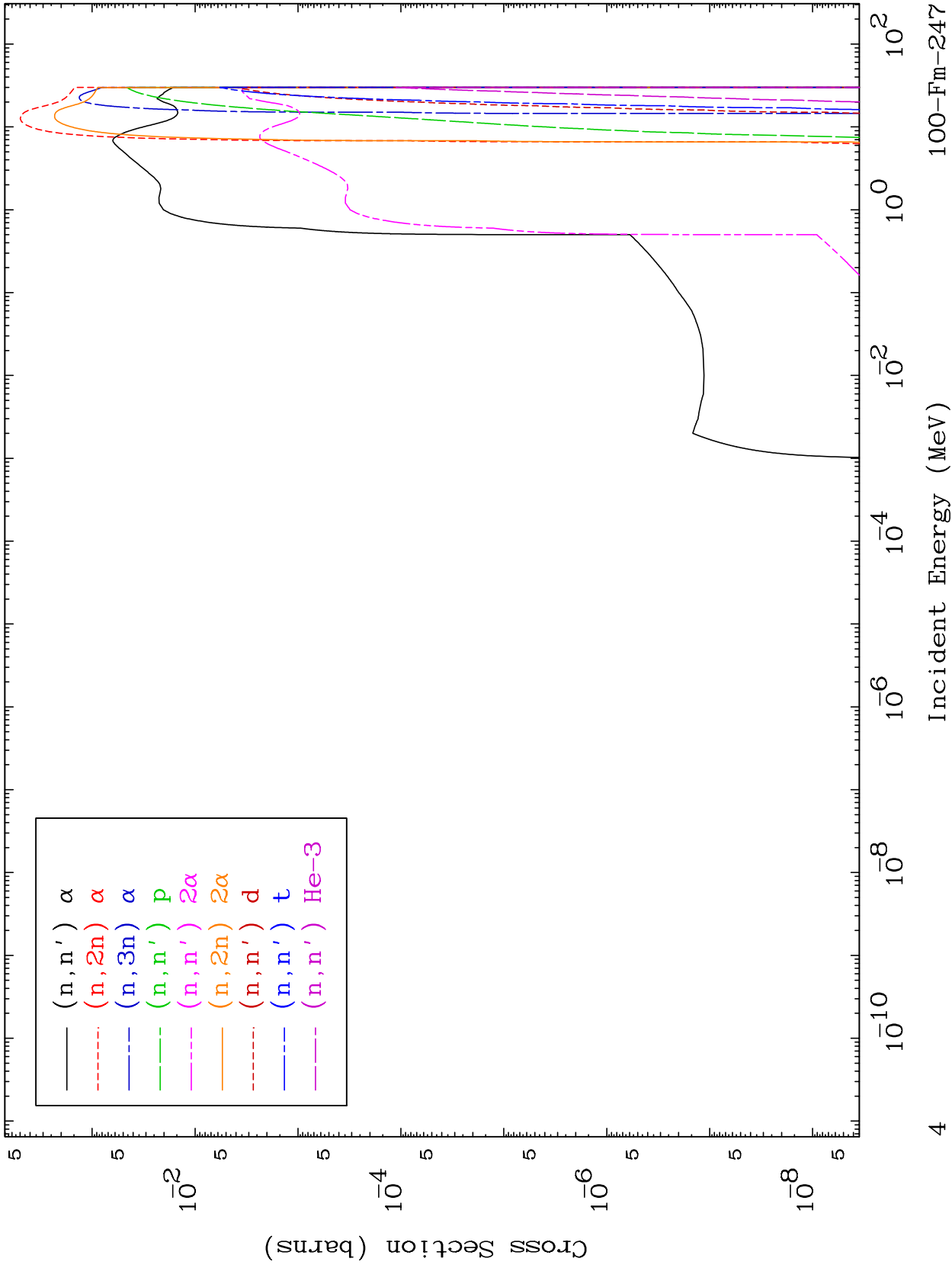




MAT 9928

Charged Particle
293 Kelvin Cross Sections

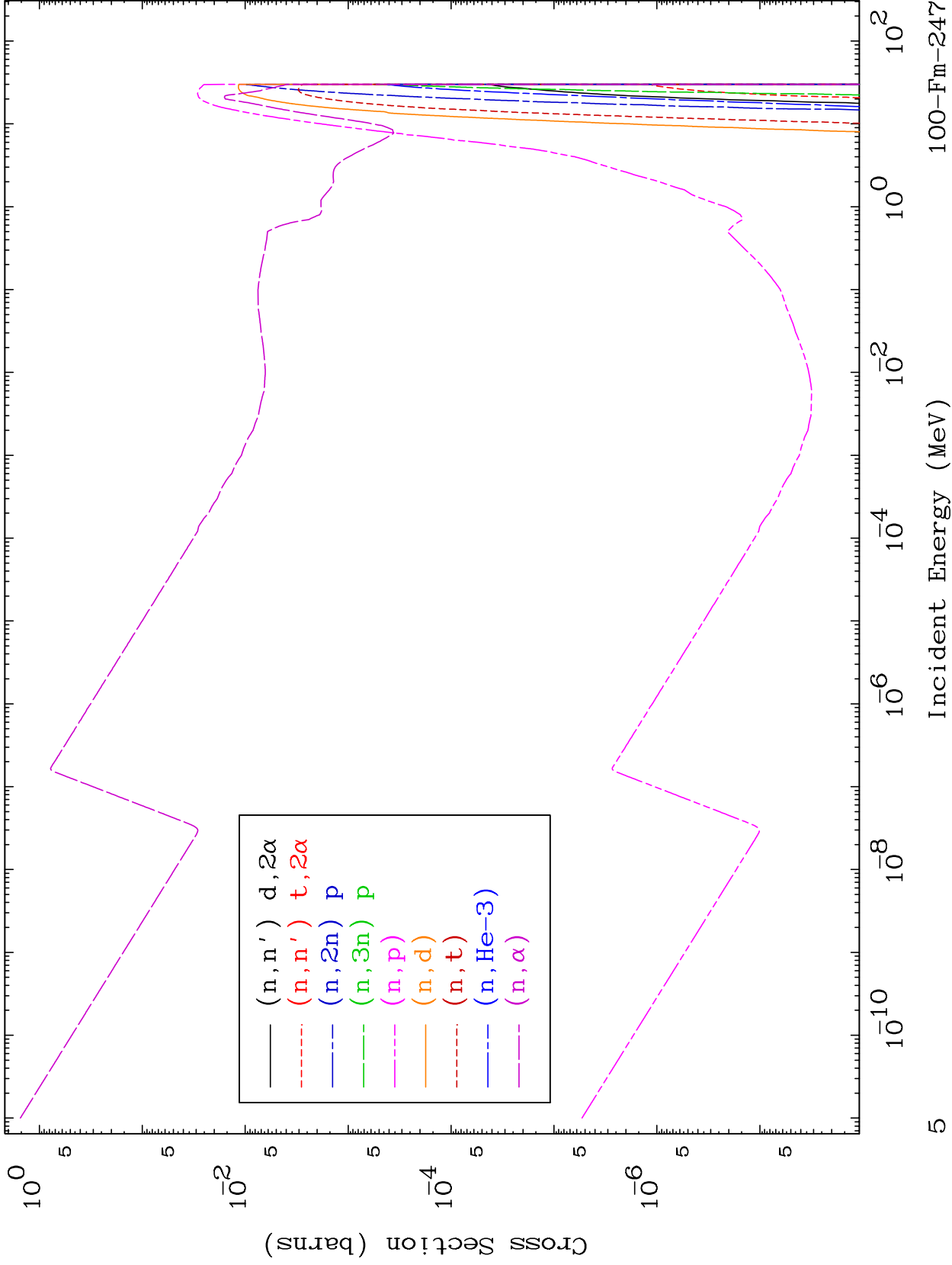
100-Fm-247



MAT 9928

Charged Particle
293 Kelvin Cross Sections

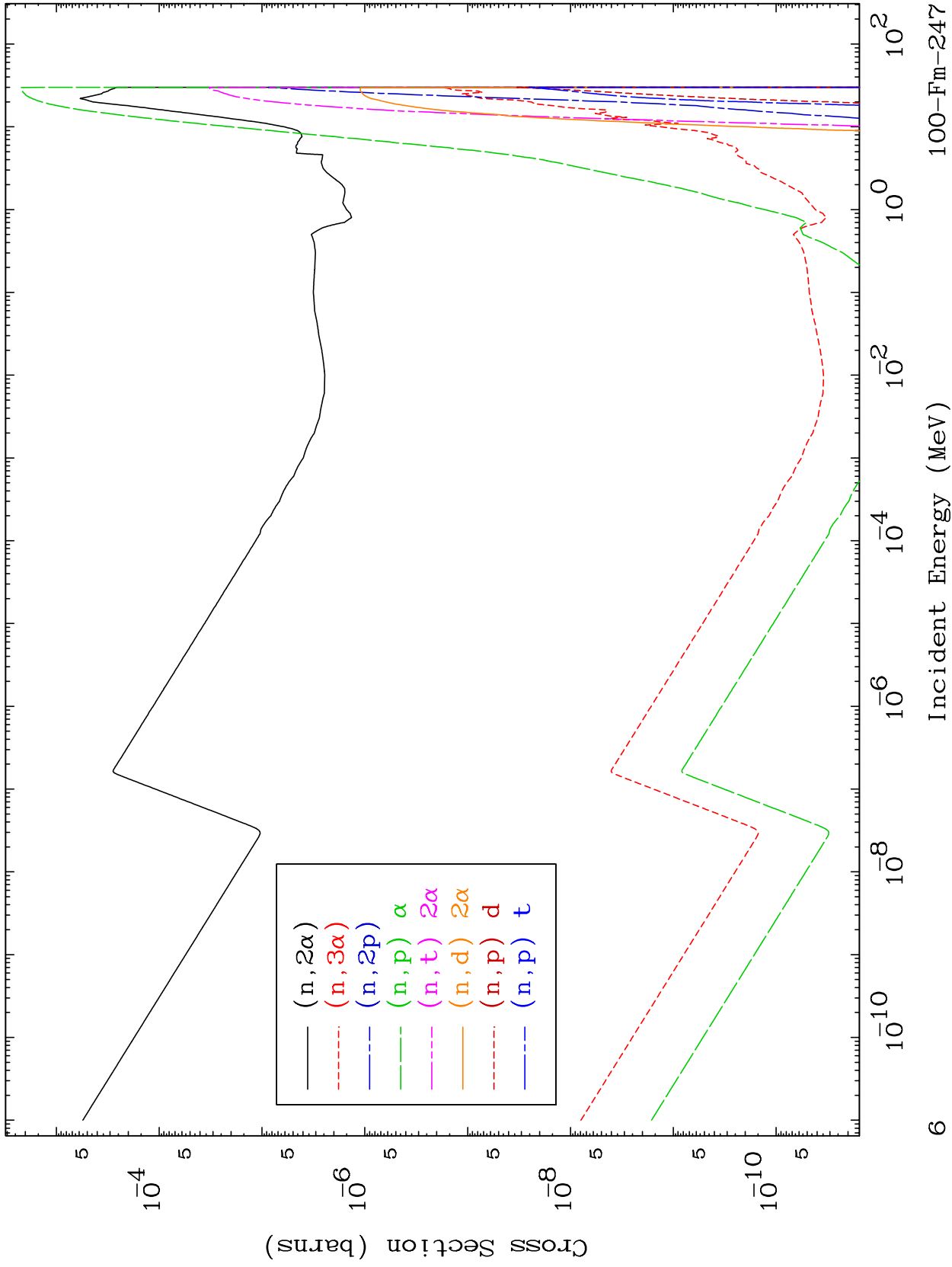
100-Fm-247



MAT 9928

Charged Particle
293 Kelvin Cross Sections

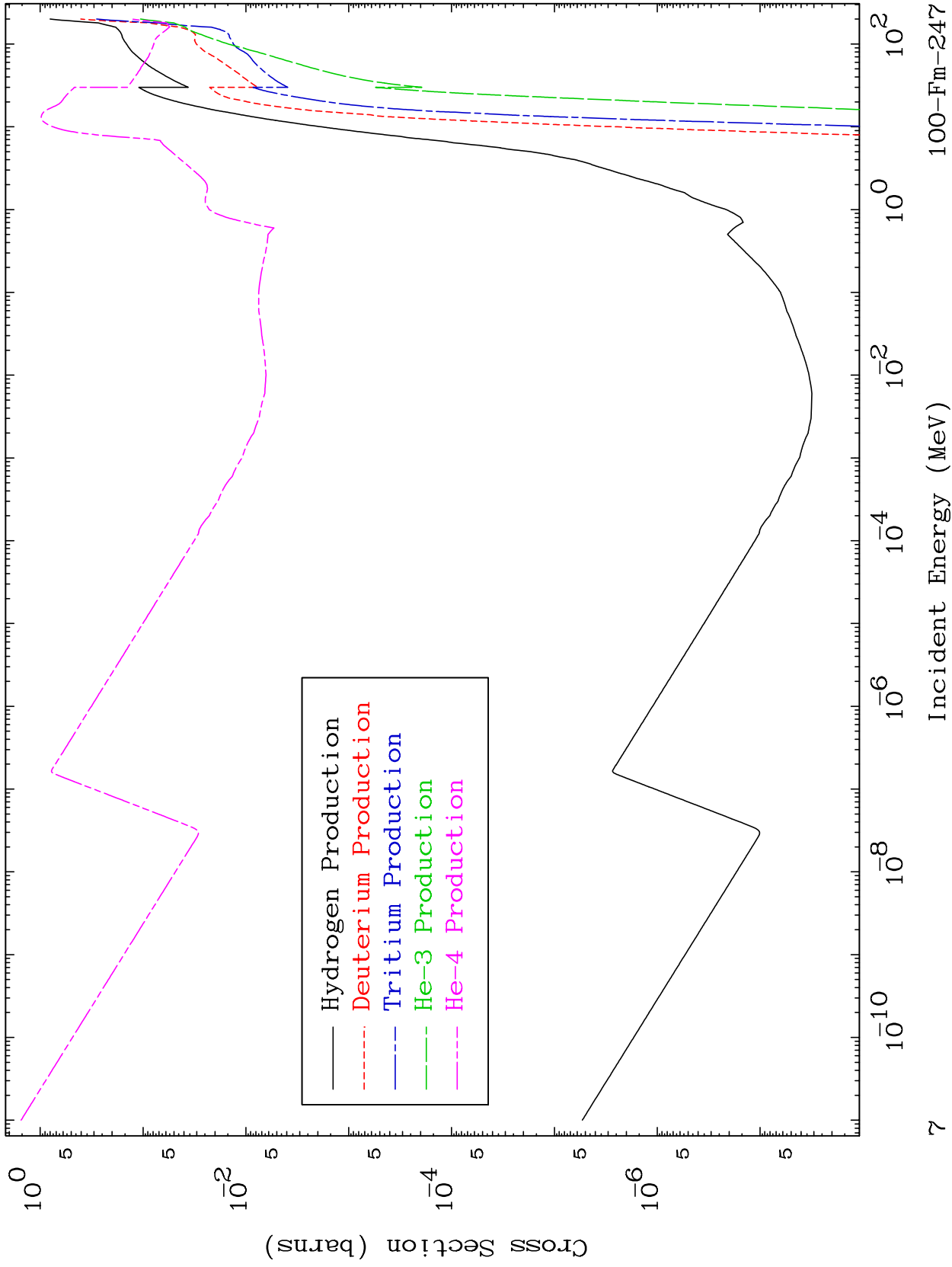
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MAT 9928

Particle Production
293 Kelvin Cross Sections

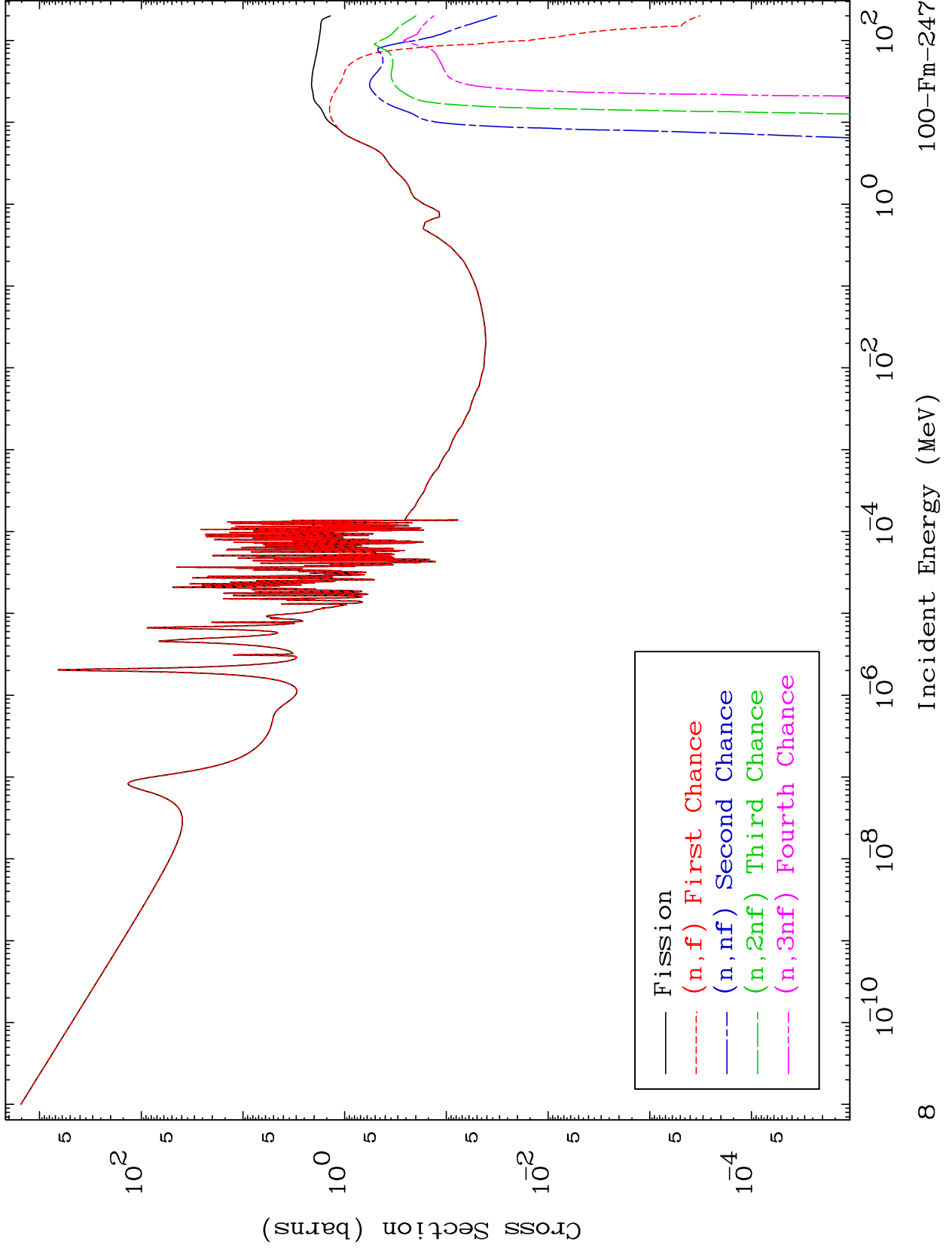
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MAT 9928

Fission
293 Kelvin Cross Sections

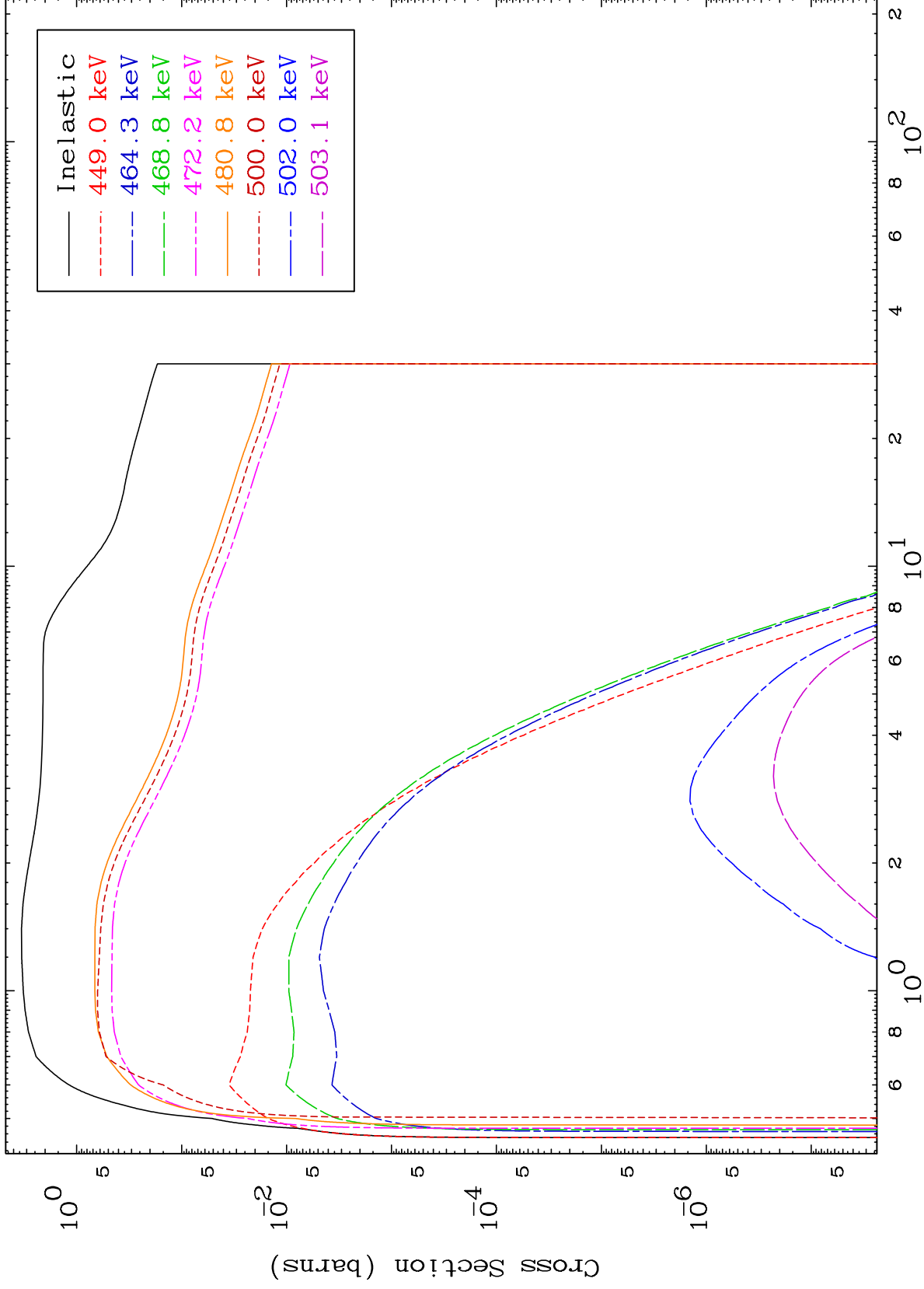
100-Fm-247



MAT 9928

(n,n') Level
293 Kelvin Cross Sections

100-Fm-247



9

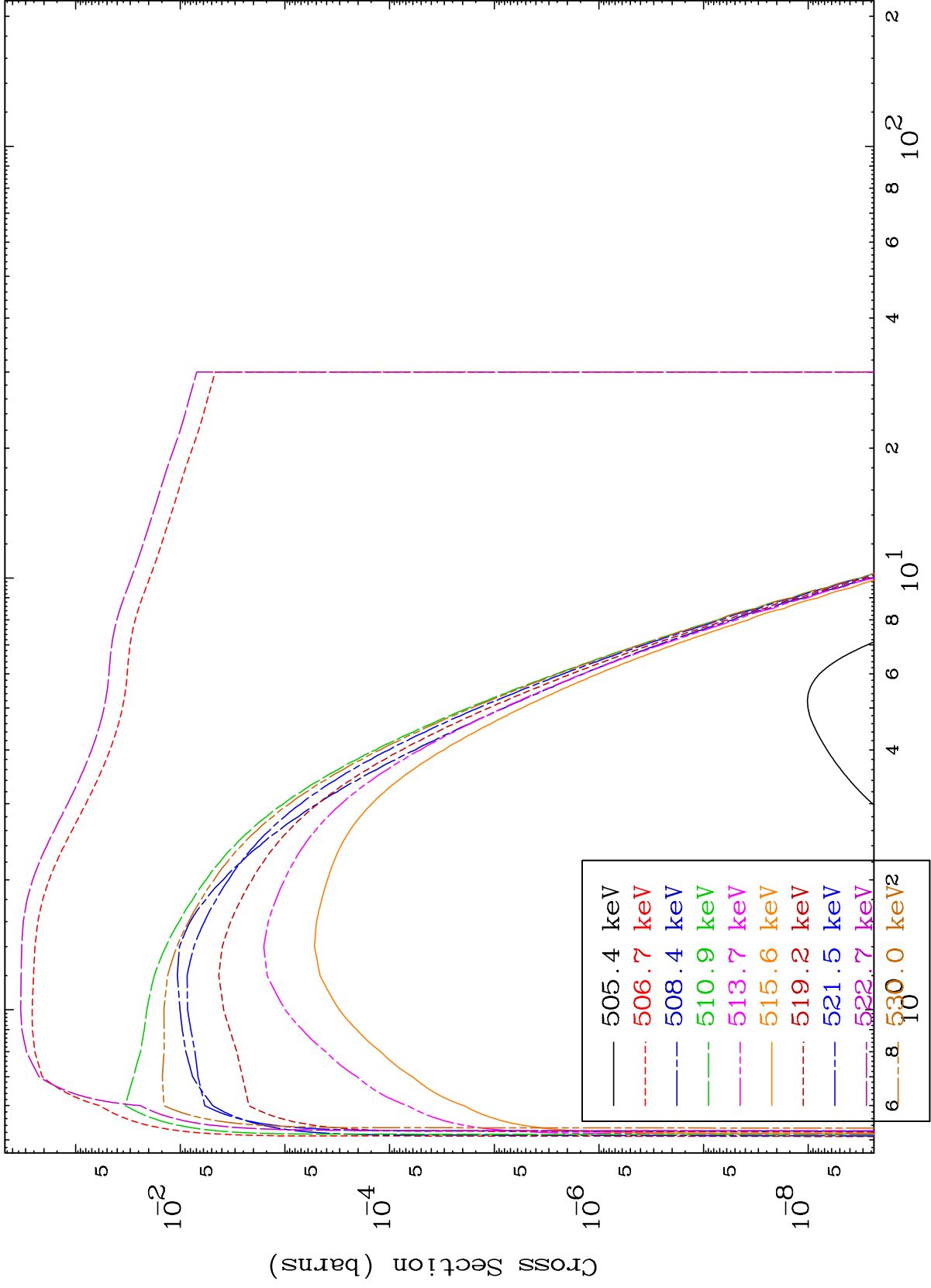
Incident Energy (MeV)

100-Fm-247

MAT 9928

(n,n') Level
293 Kelvin Cross Sections

100-Fm-247



10

Incident Energy (MeV)

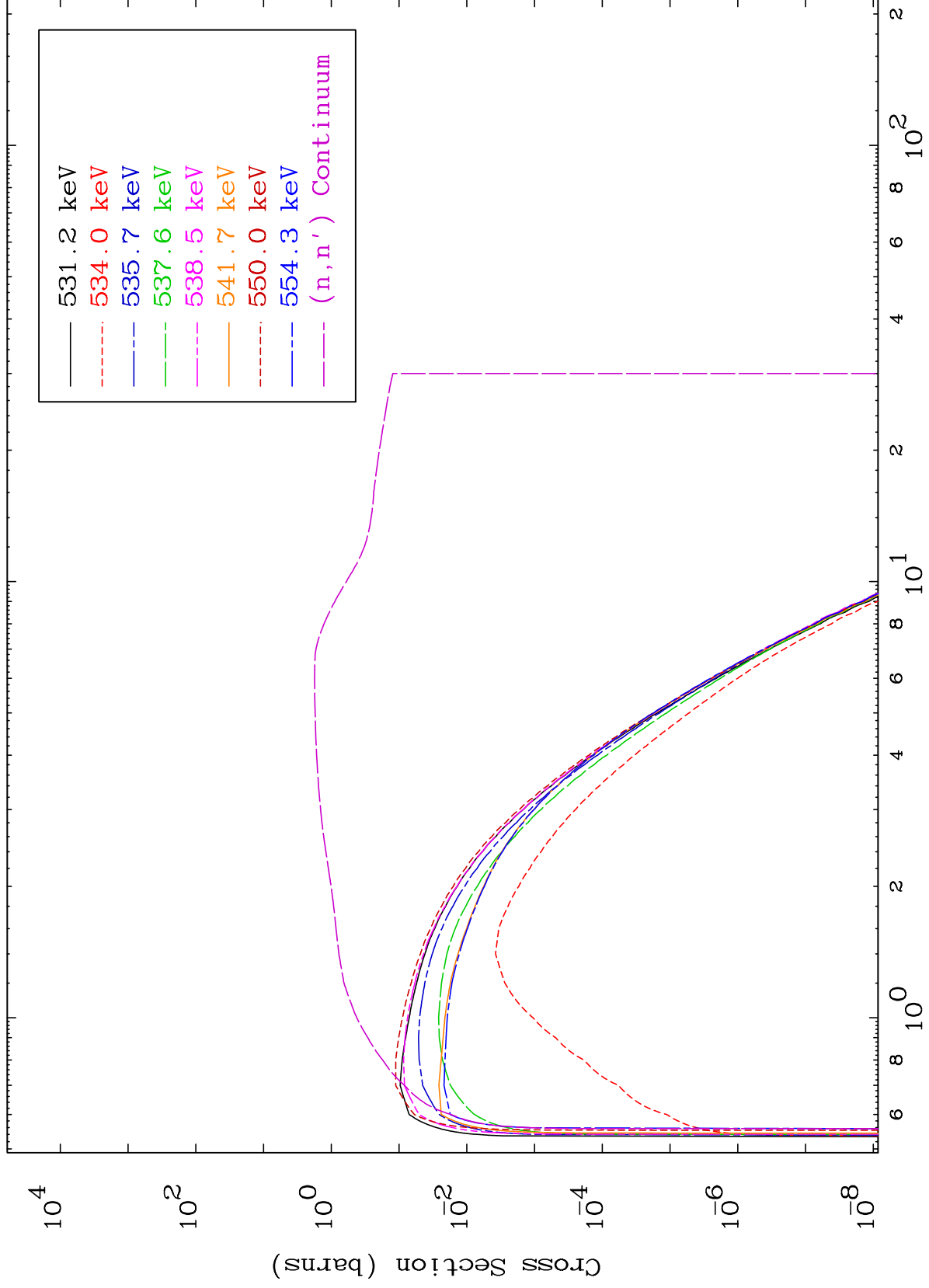
100-Fm-247

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(n,n') Level

100-Fm-247

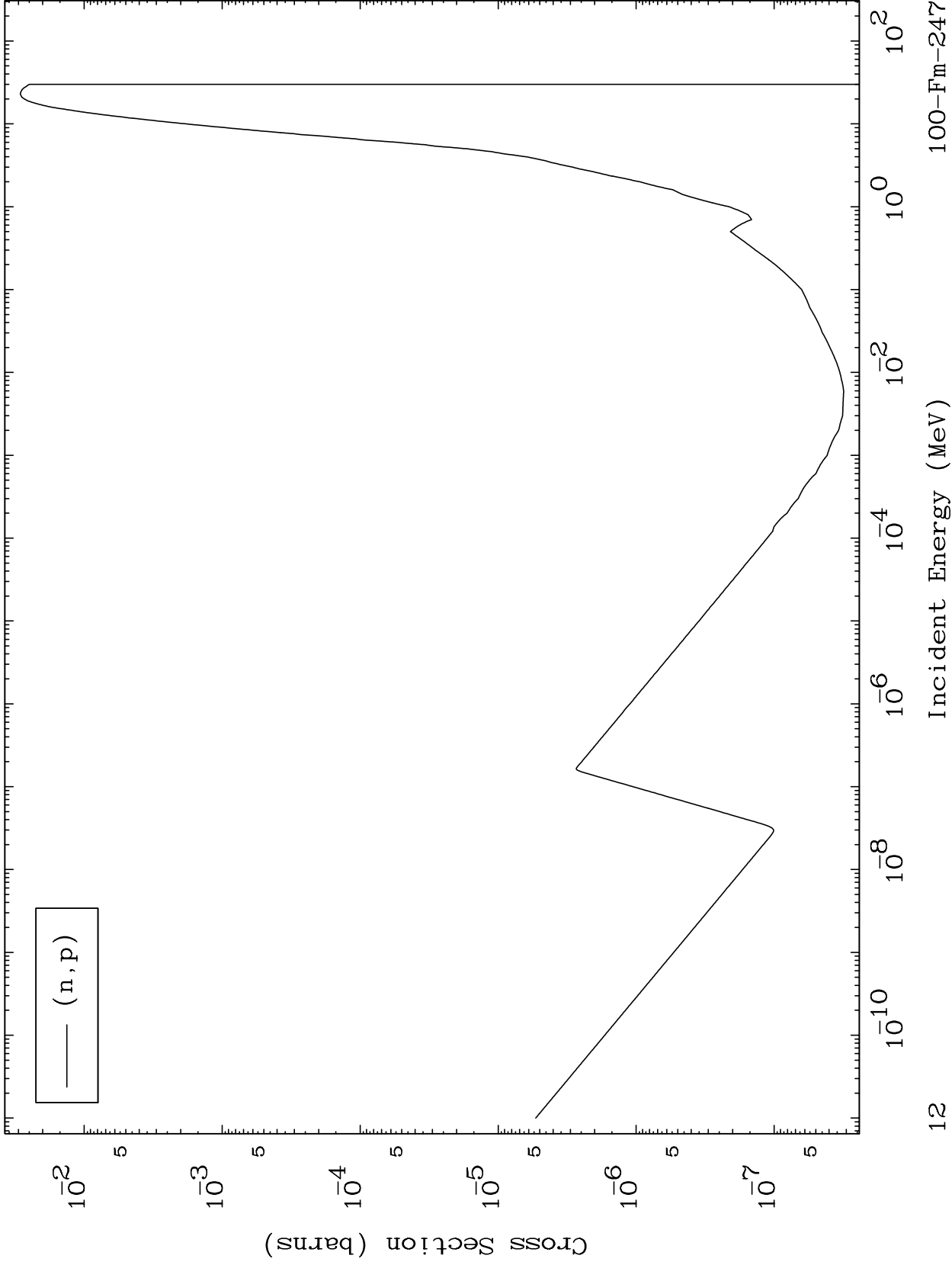
293 Kelvin Cross Sections



MAT 9928

(n,p) Levels
293 Kelvin Cross Sections

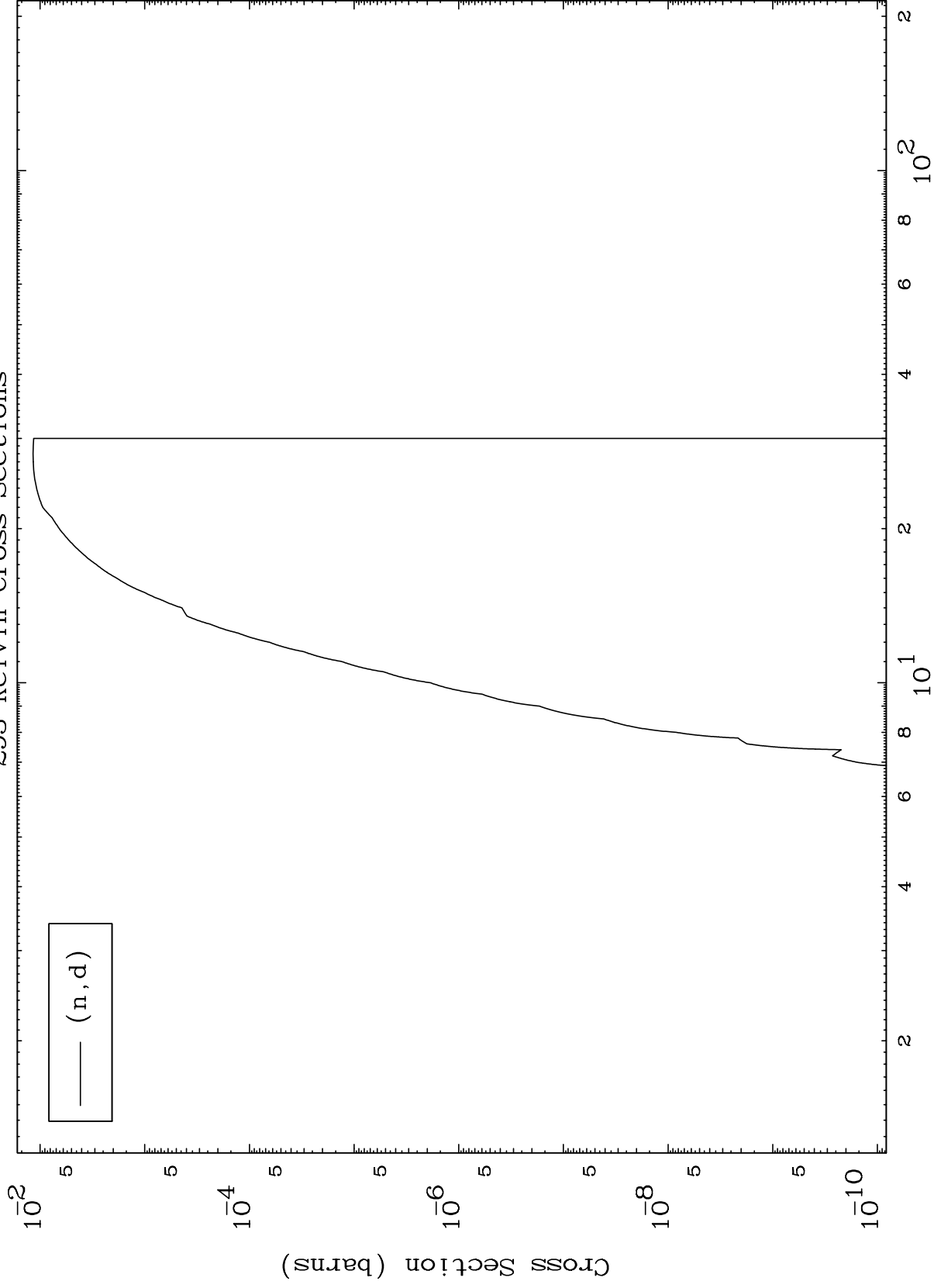
100-Fm-247



MAT 9928

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-247



13

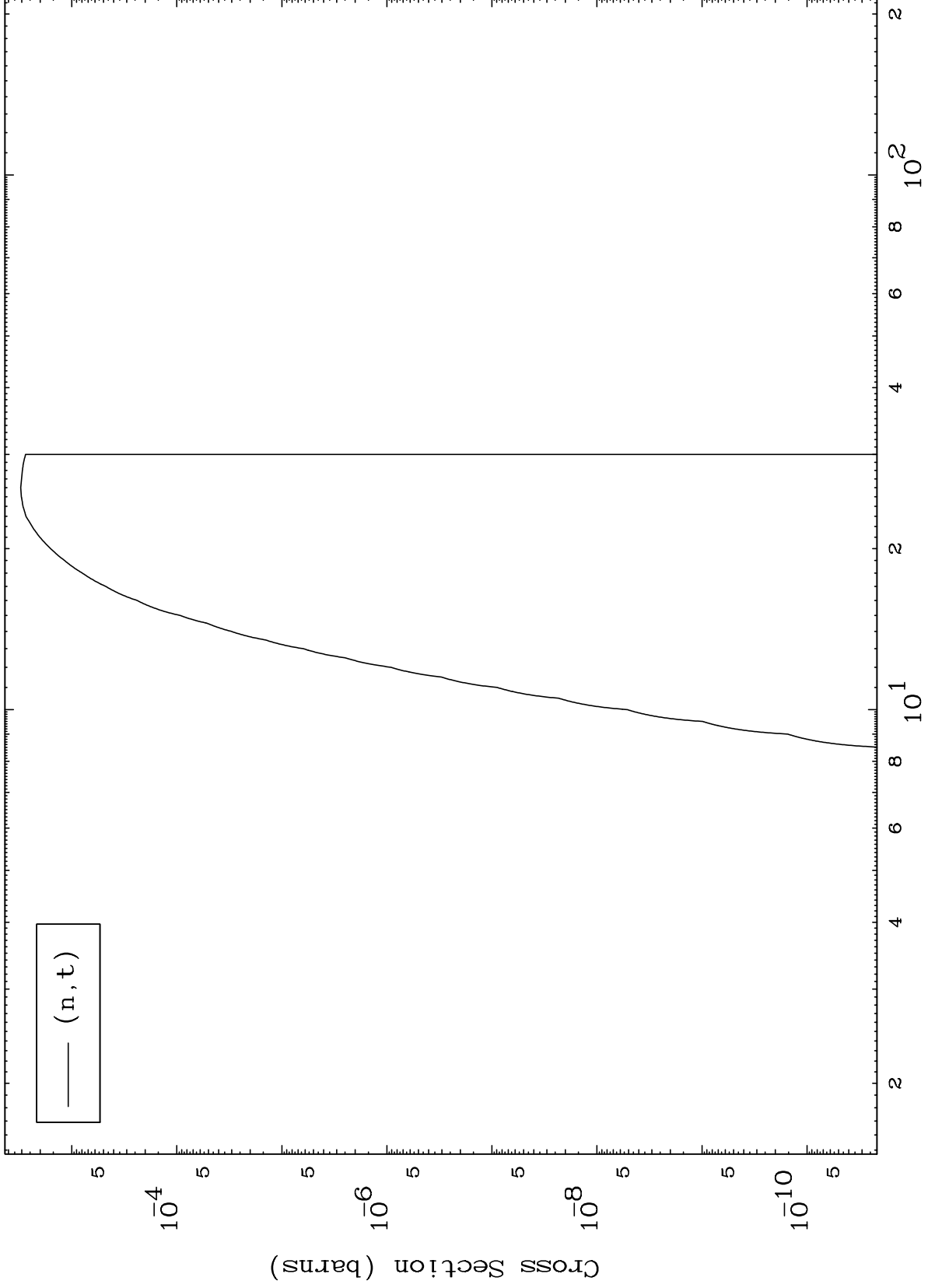
Incident Energy (MeV)

100-Fm-247

MAT 9928

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-247



14

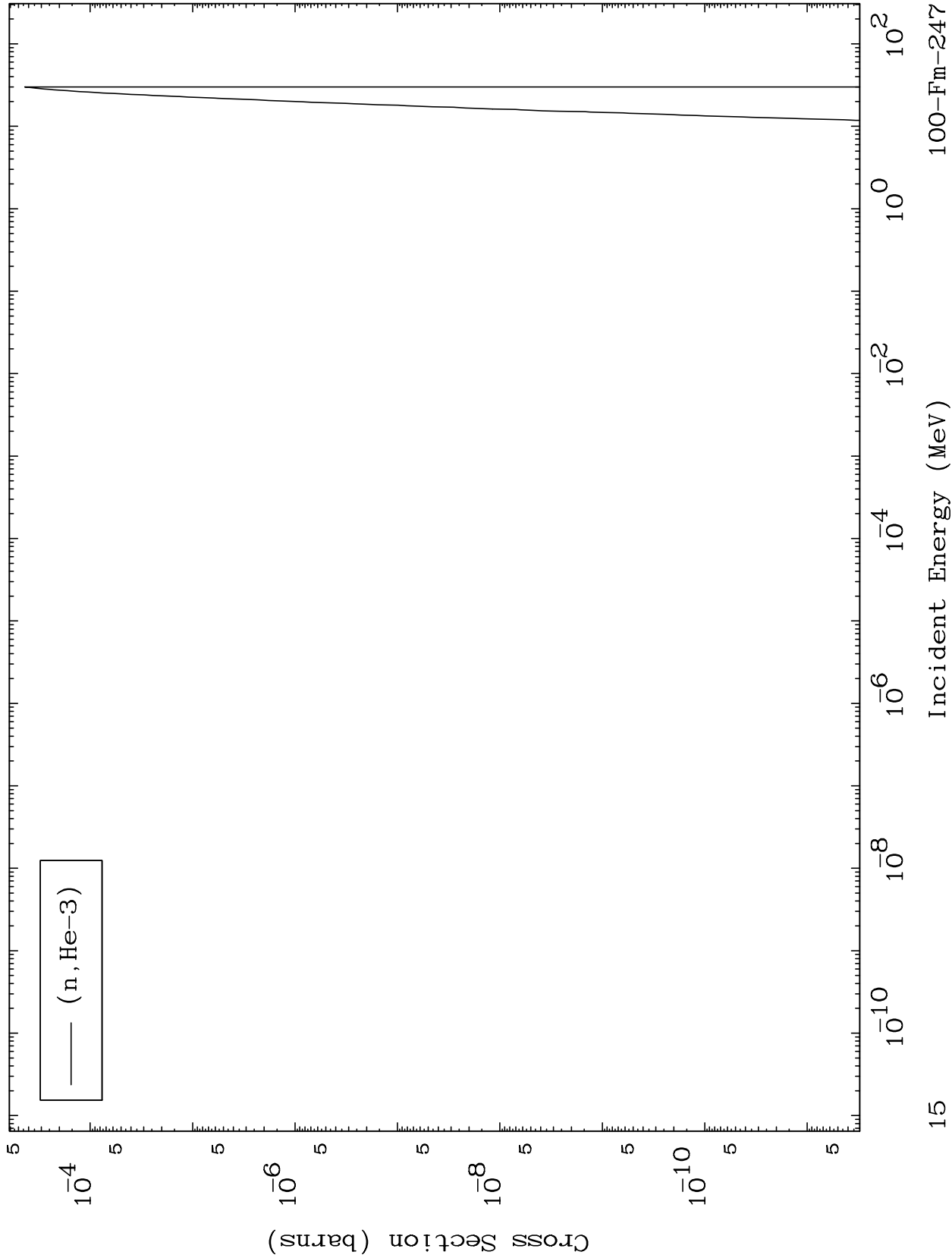
Incident Energy (MeV)

100-Fm-247

MAT 9928

(n,He3) Levels
293 Kelvin Cross Sections

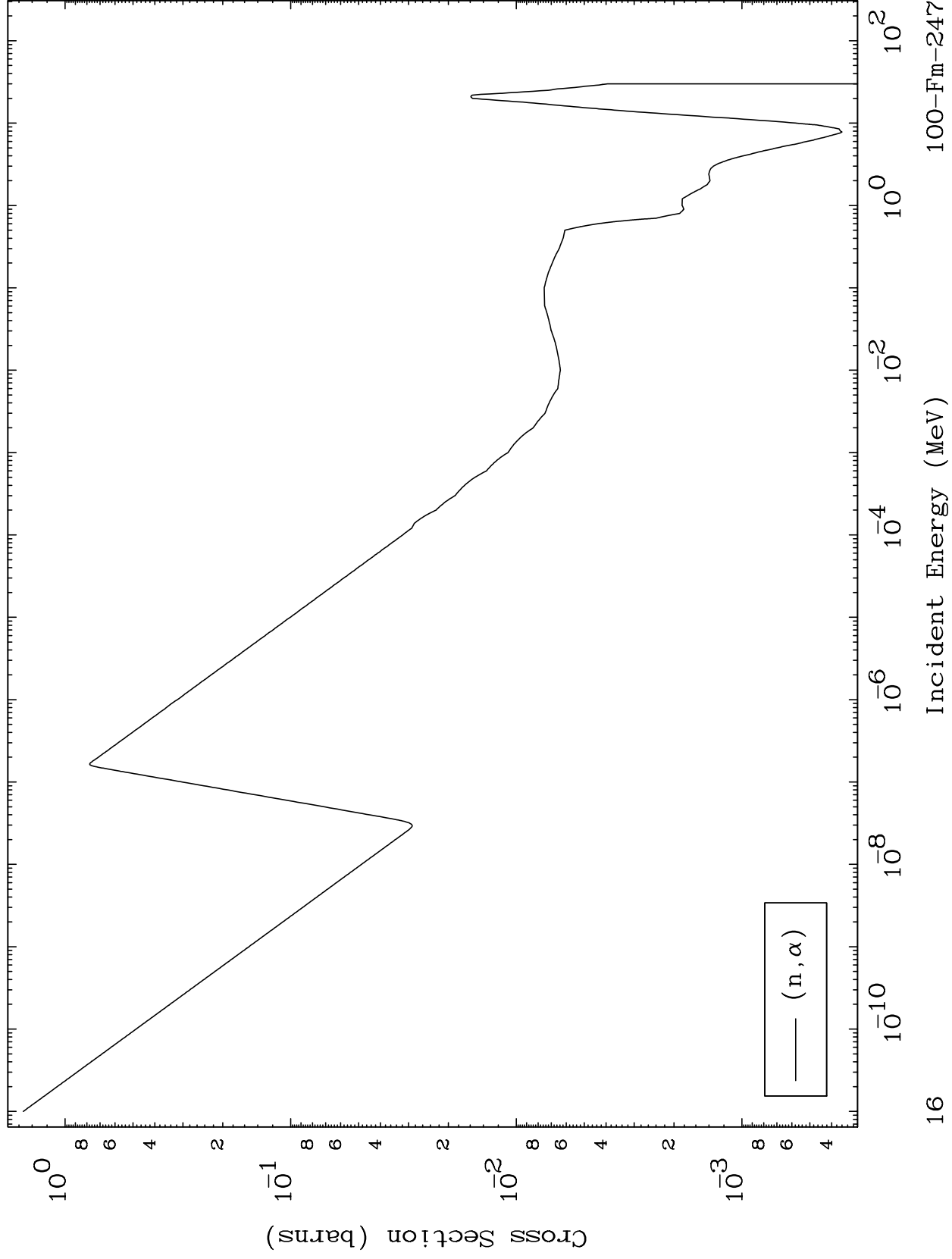
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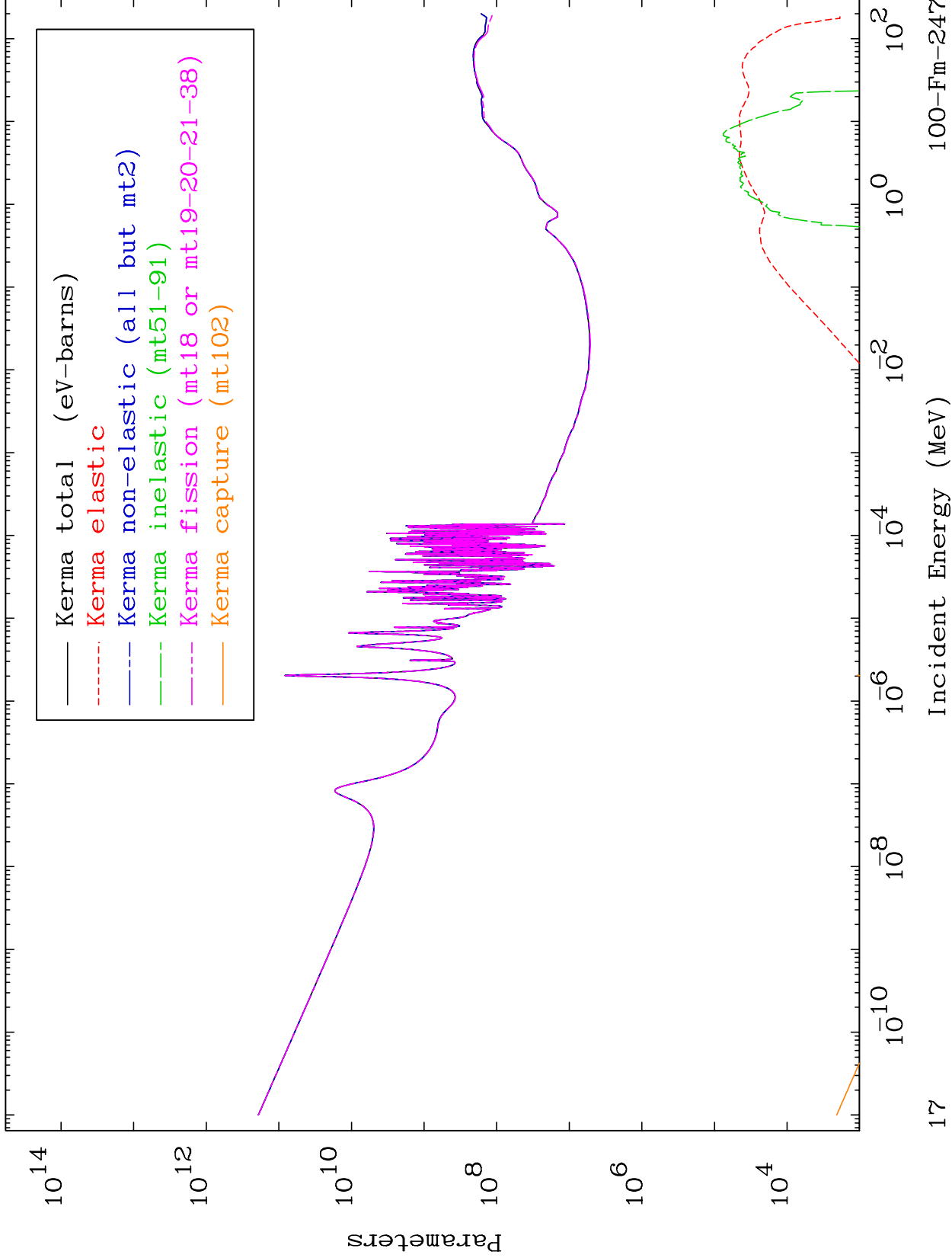


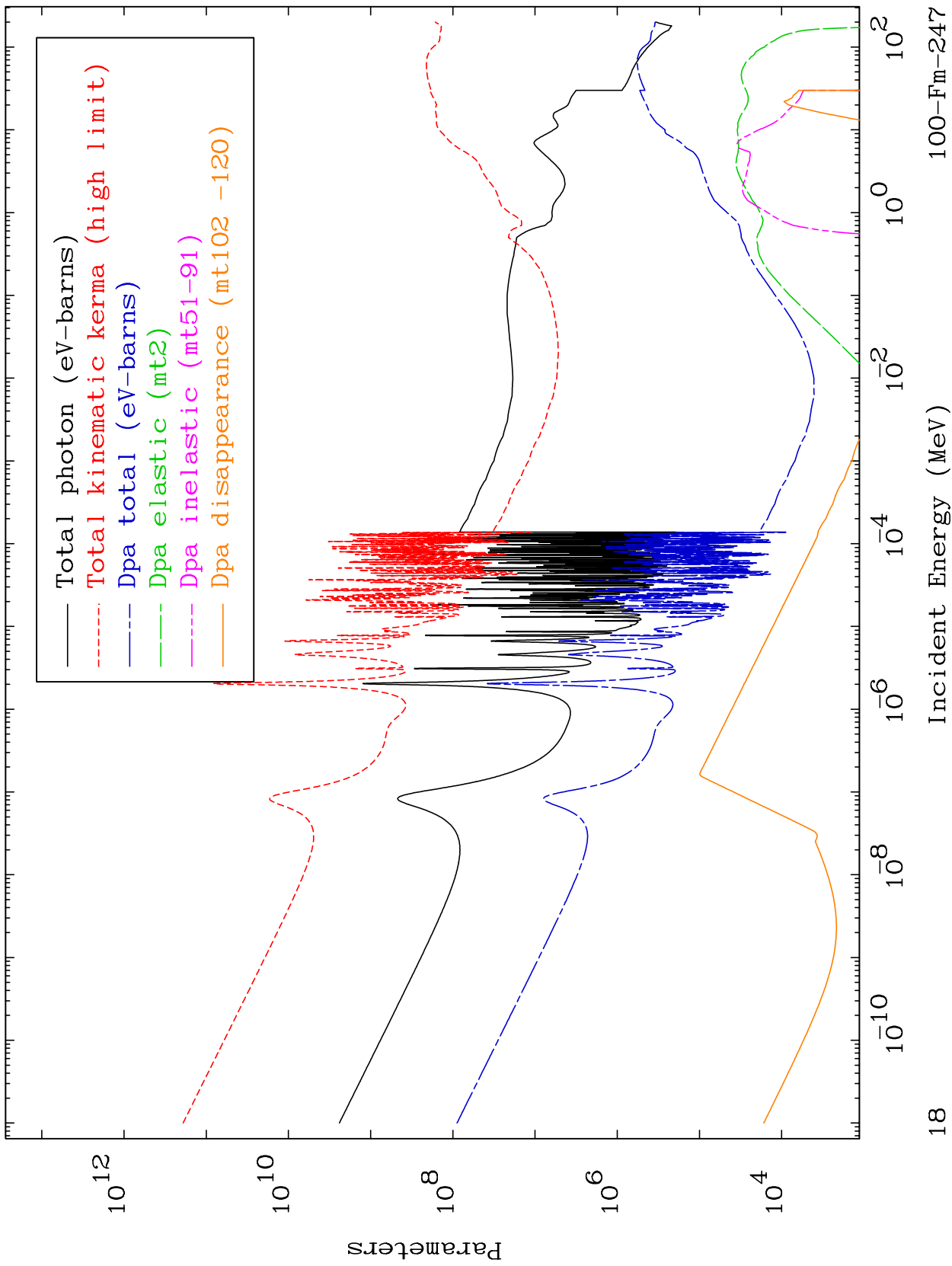
MAT 9928

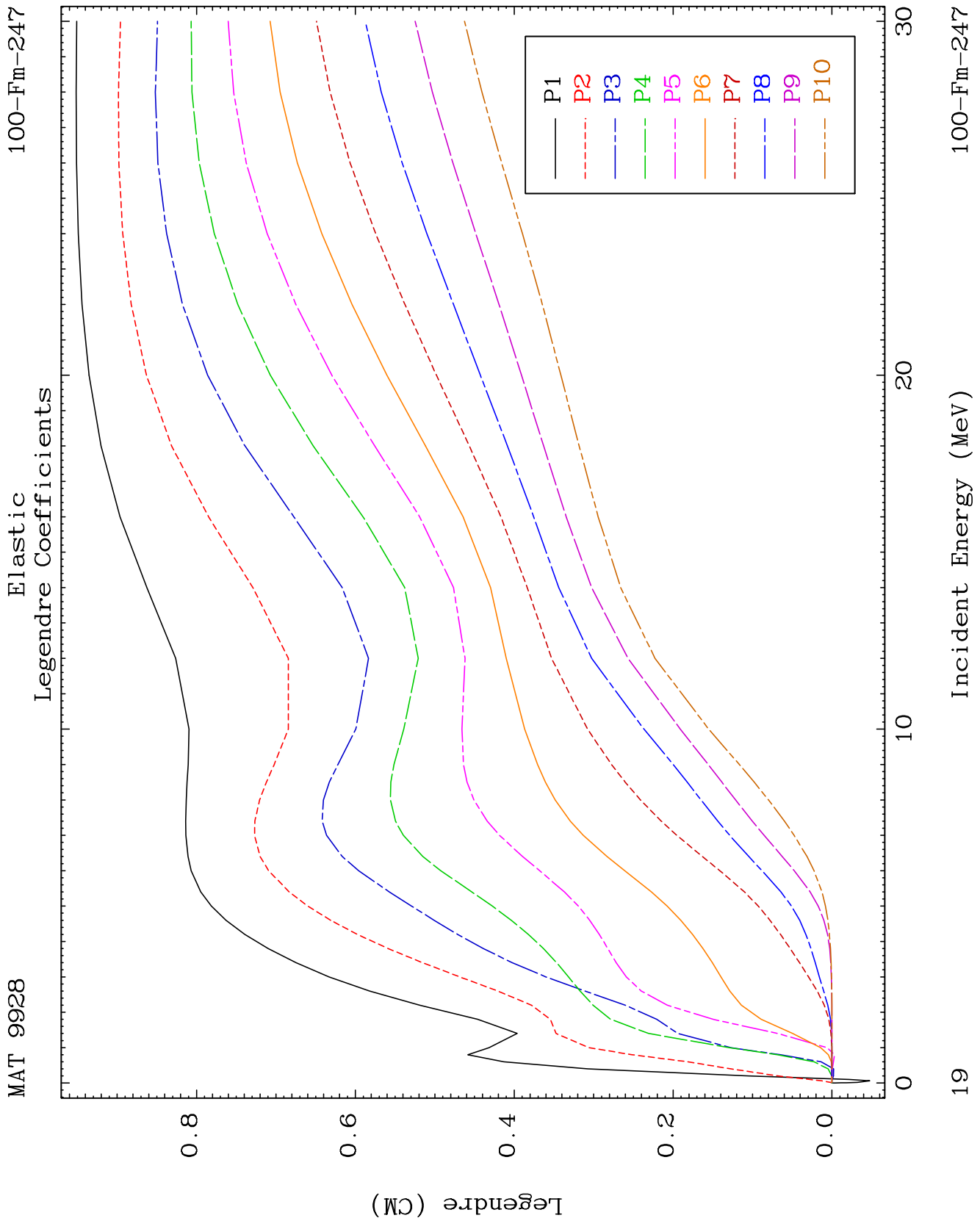
(n,α) Levels
293 Kelvin Cross Sections

100-Fm-247





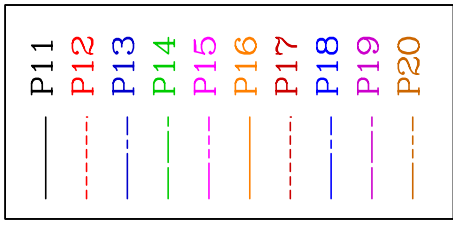
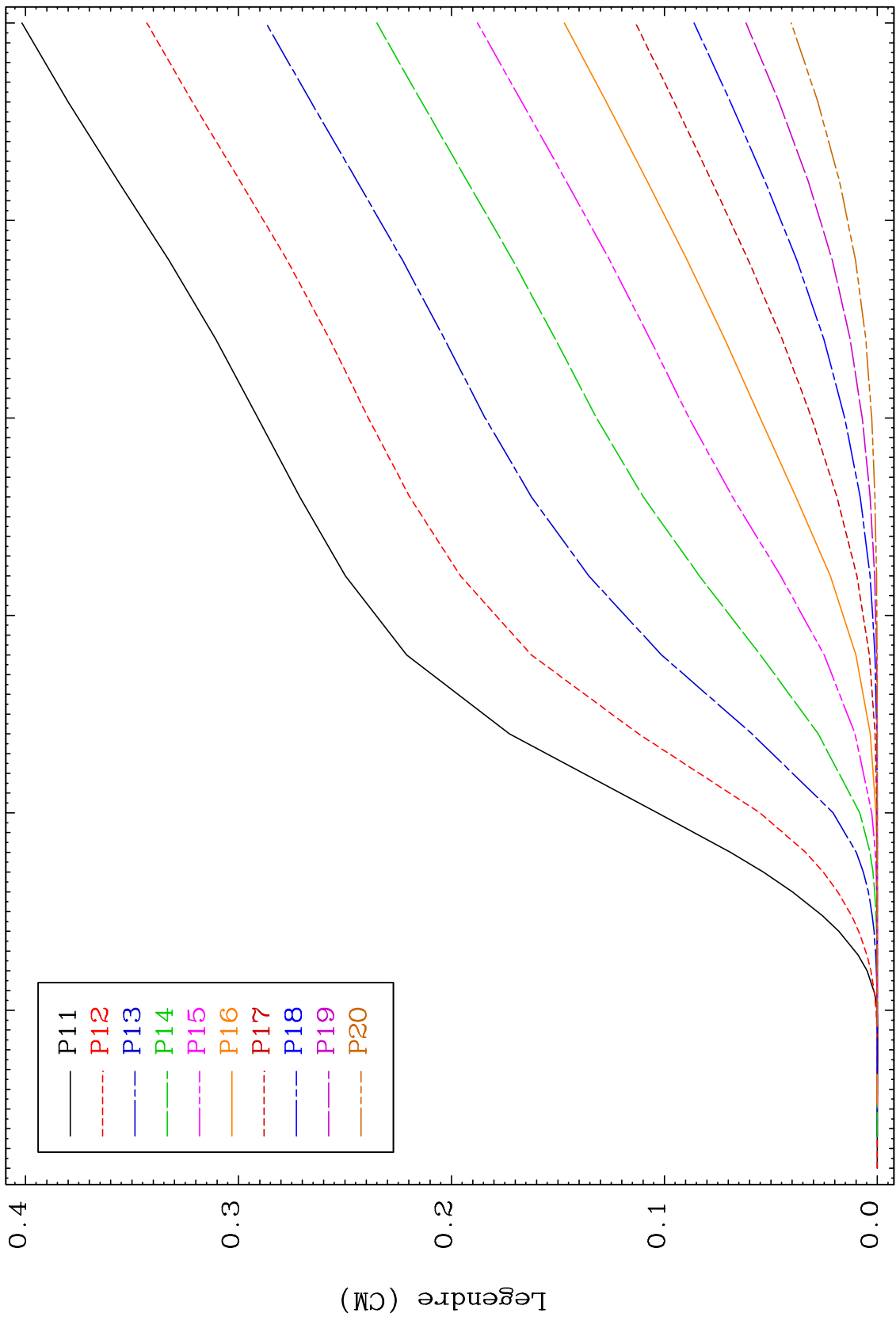




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Elastic Legendre Coefficients

100-Fm-247



100-Fm-247

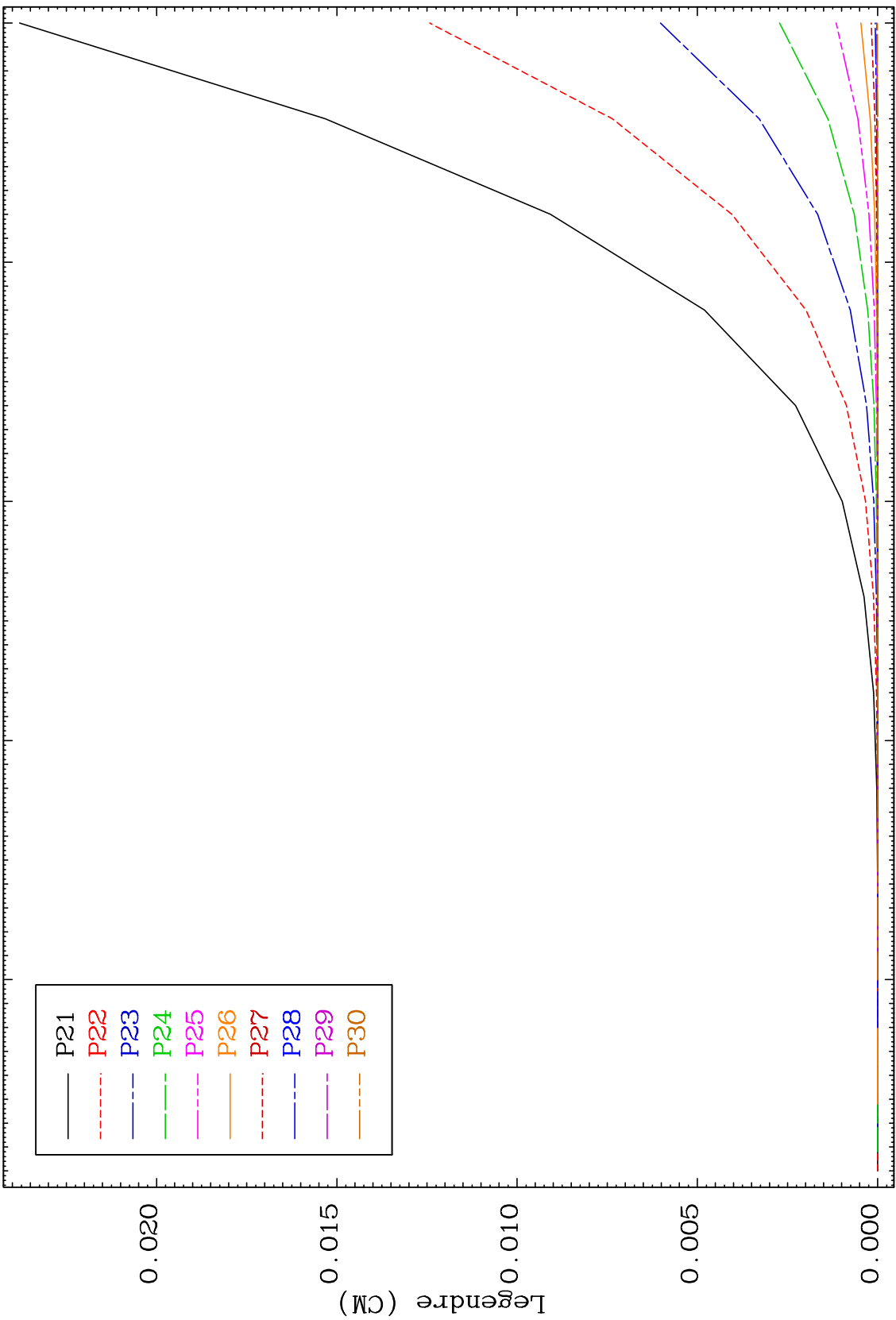
Incident Energy (MeV)

20

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Elastic Legendre Coefficients

100-Fm-247



21

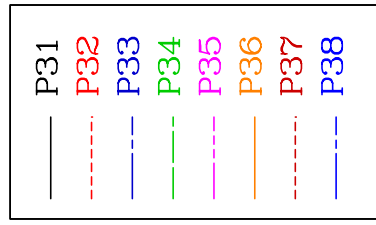
Incident Energy (MeV)

100-Fm-247

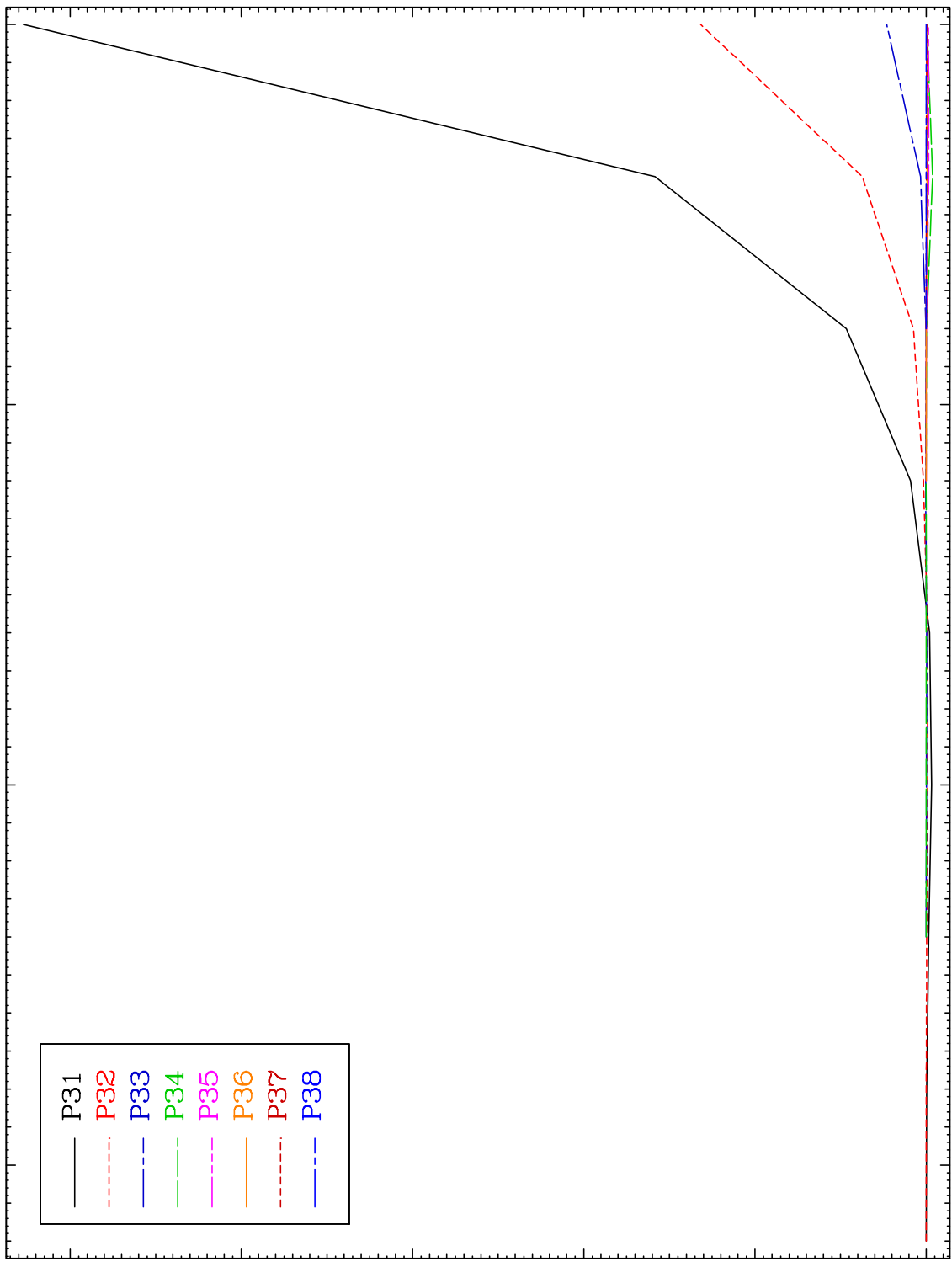
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Elastic Legendre Coefficients

100-Fm-247



Legendre (CM) $\times 10^6$

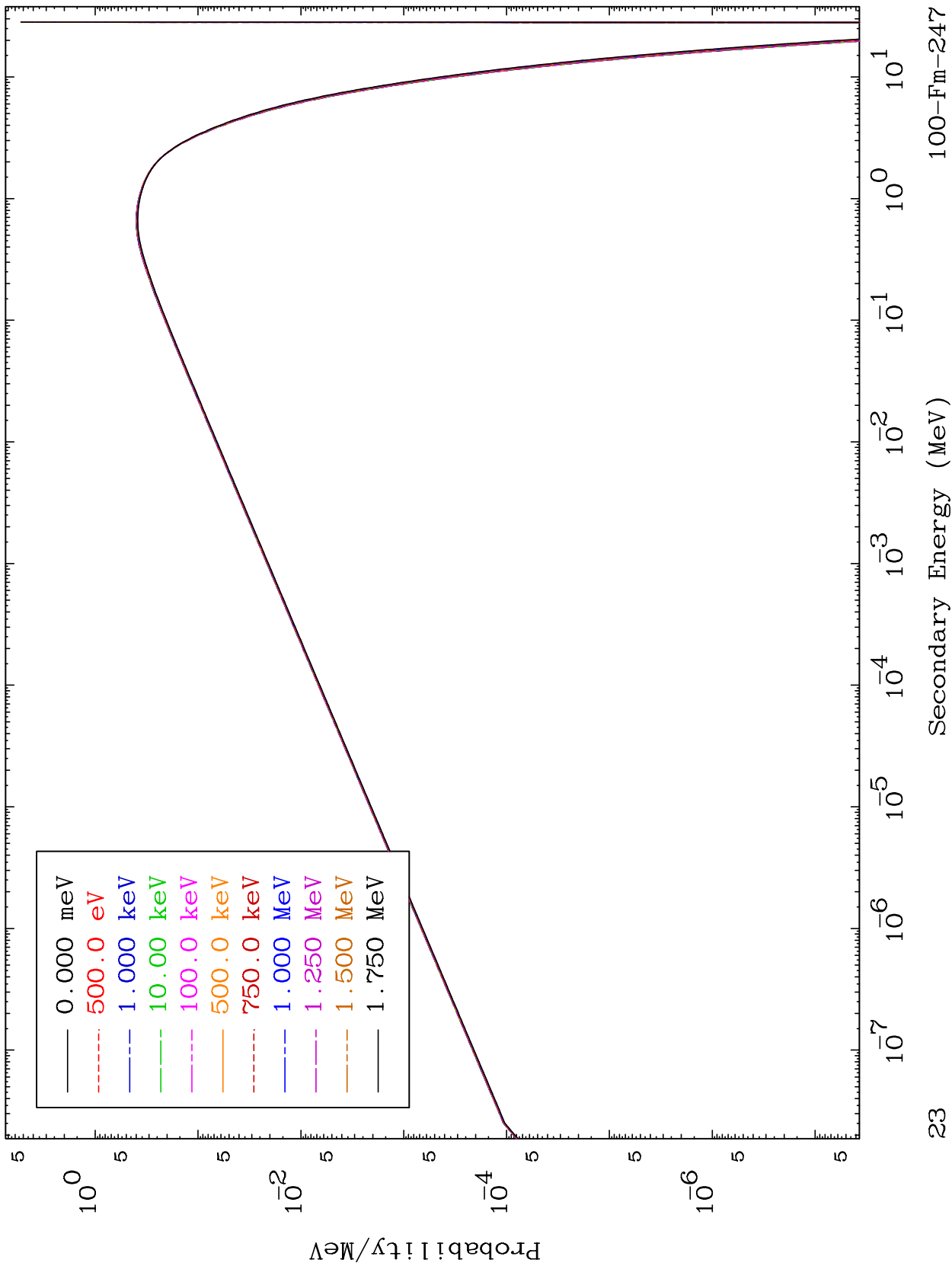


22

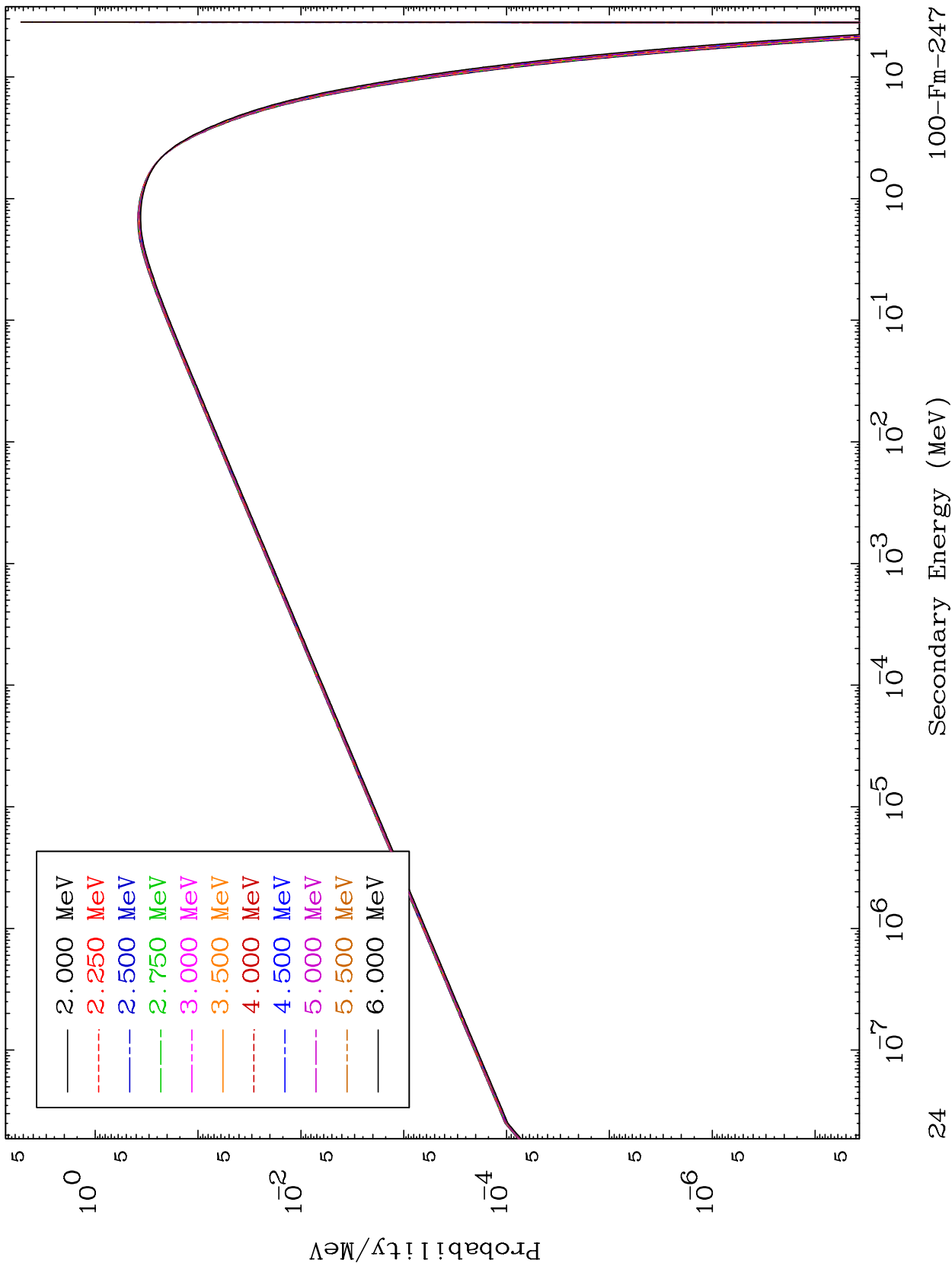
Incident Energy (MeV)

100-Fm-247

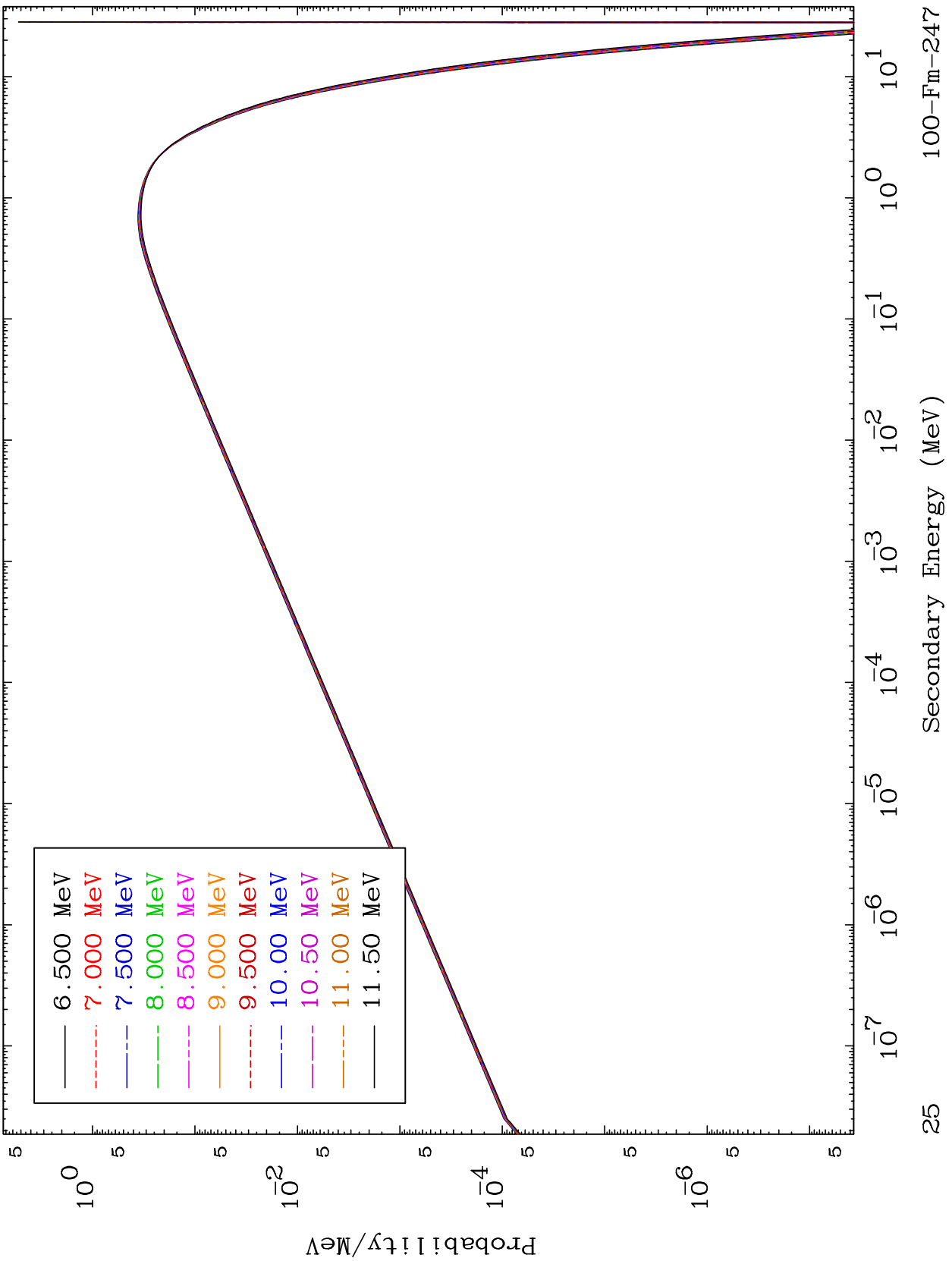
Fission Energy Distribution



Fission
Energy Distribution



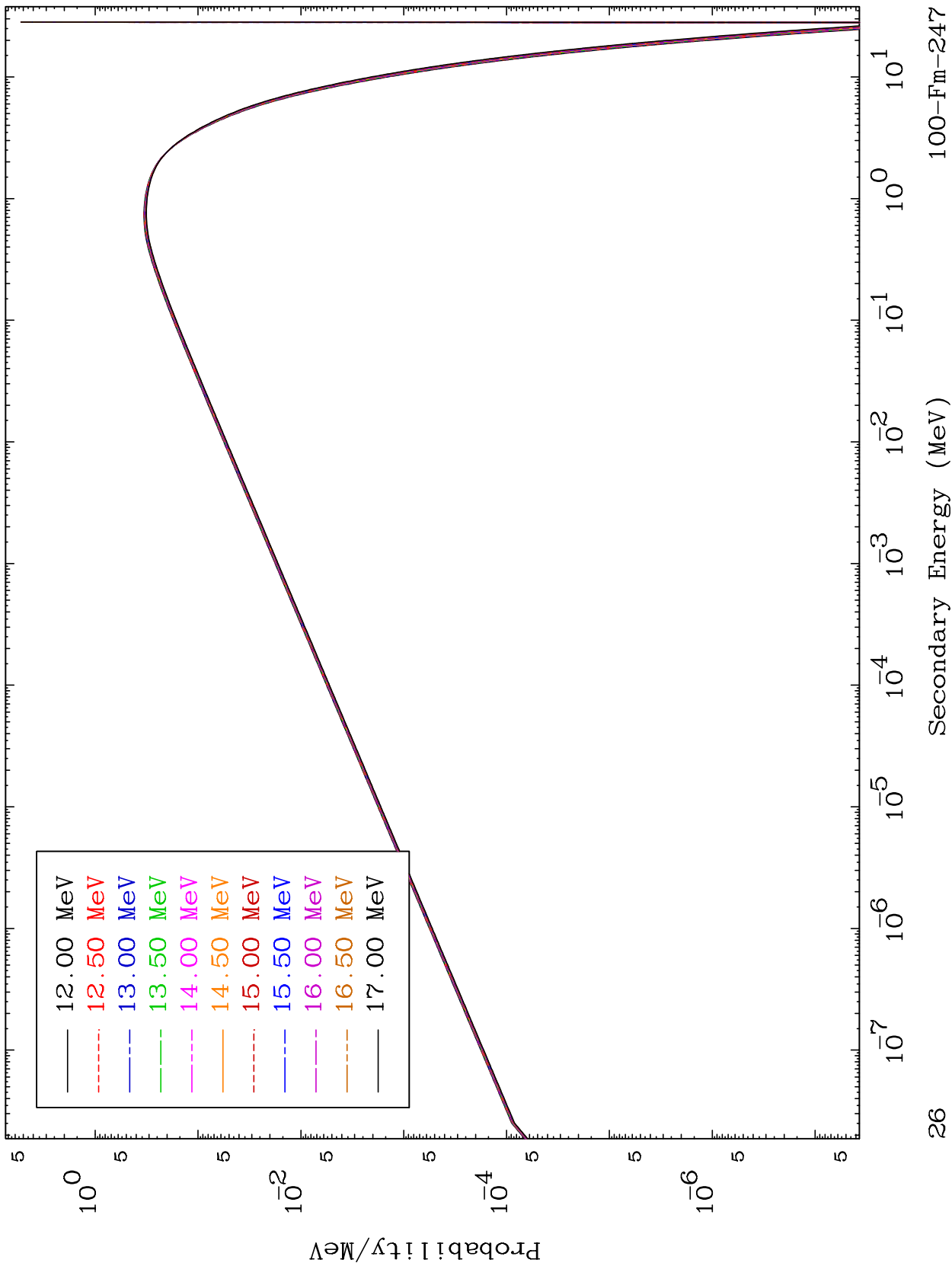
Fission Energy Distribution



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Fission Energy Distribution

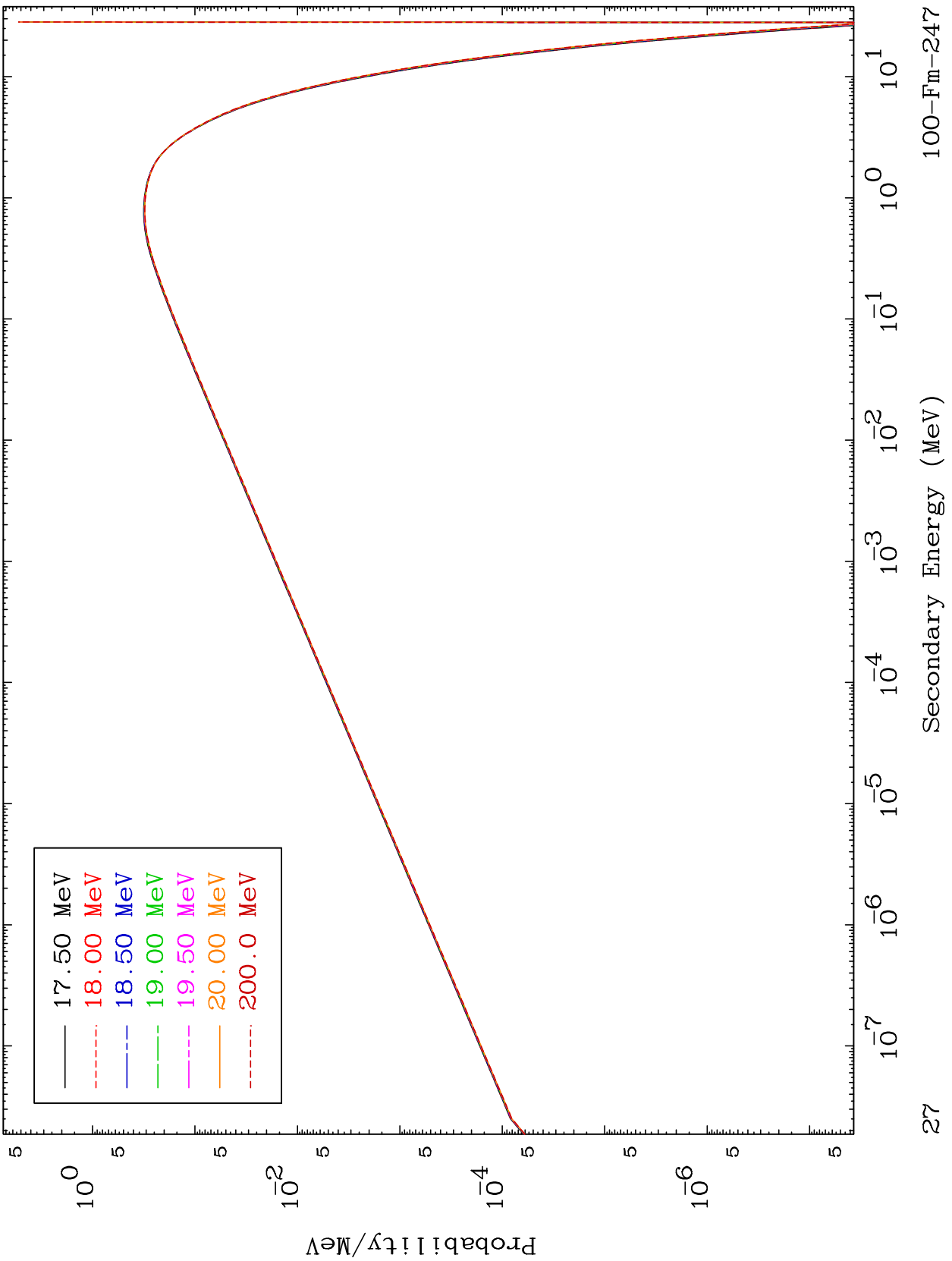
100-Fm-247



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Fission Energy Distribution

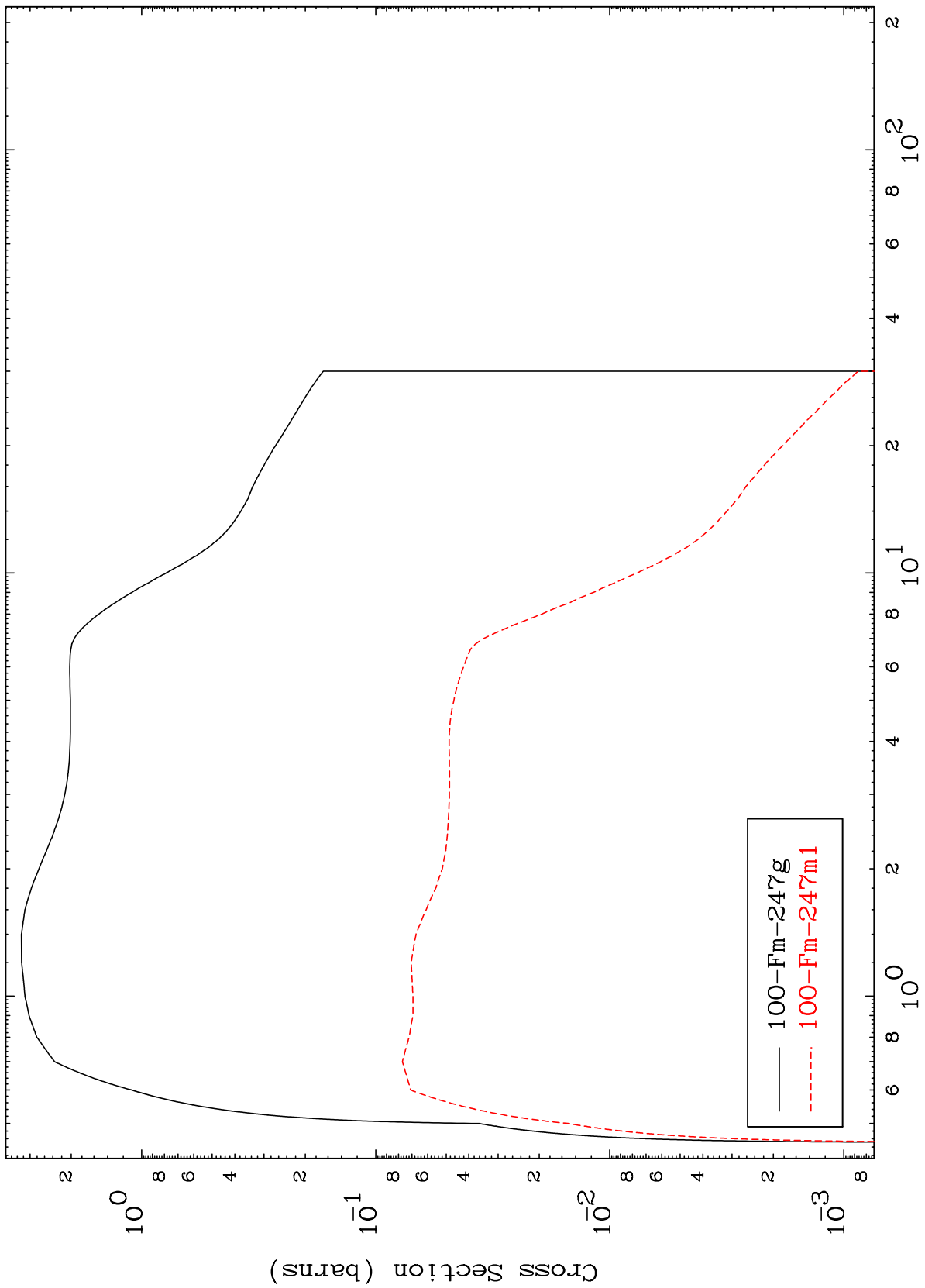
100-Fm-247



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100-Fm-247

Inelastic
Radionuclide Production Cross Section



100-Fm-247

Incident Energy (MeV)

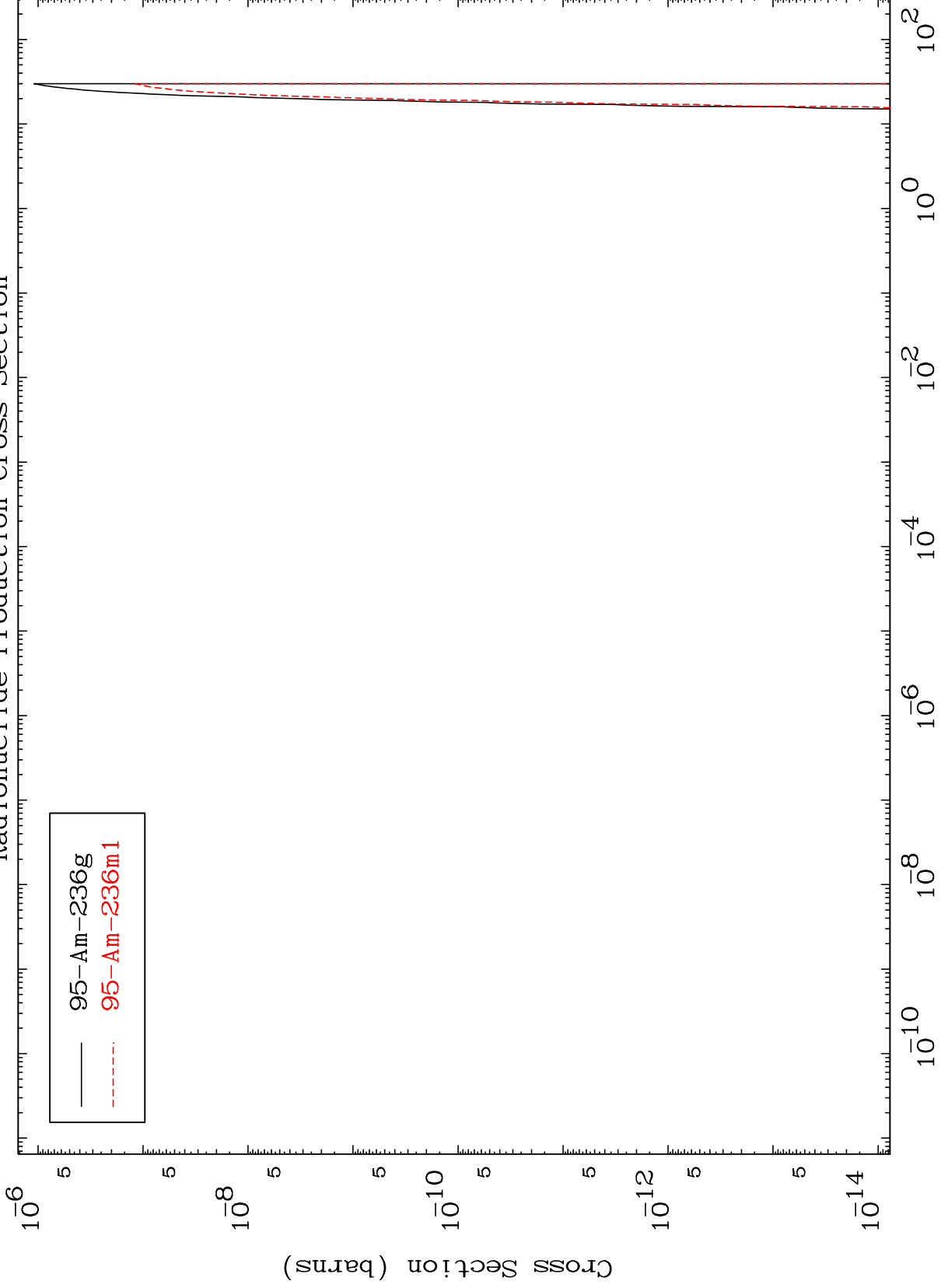
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(n,n') t,2 α

100-Fm-247

Radionuclide Production Cross Section



— 95-Am-236g
- - - 95-Am-236m1

Incident Energy (MeV)

100-Fm-247