

Program EVALPLOT
(Version 2015-2)

by

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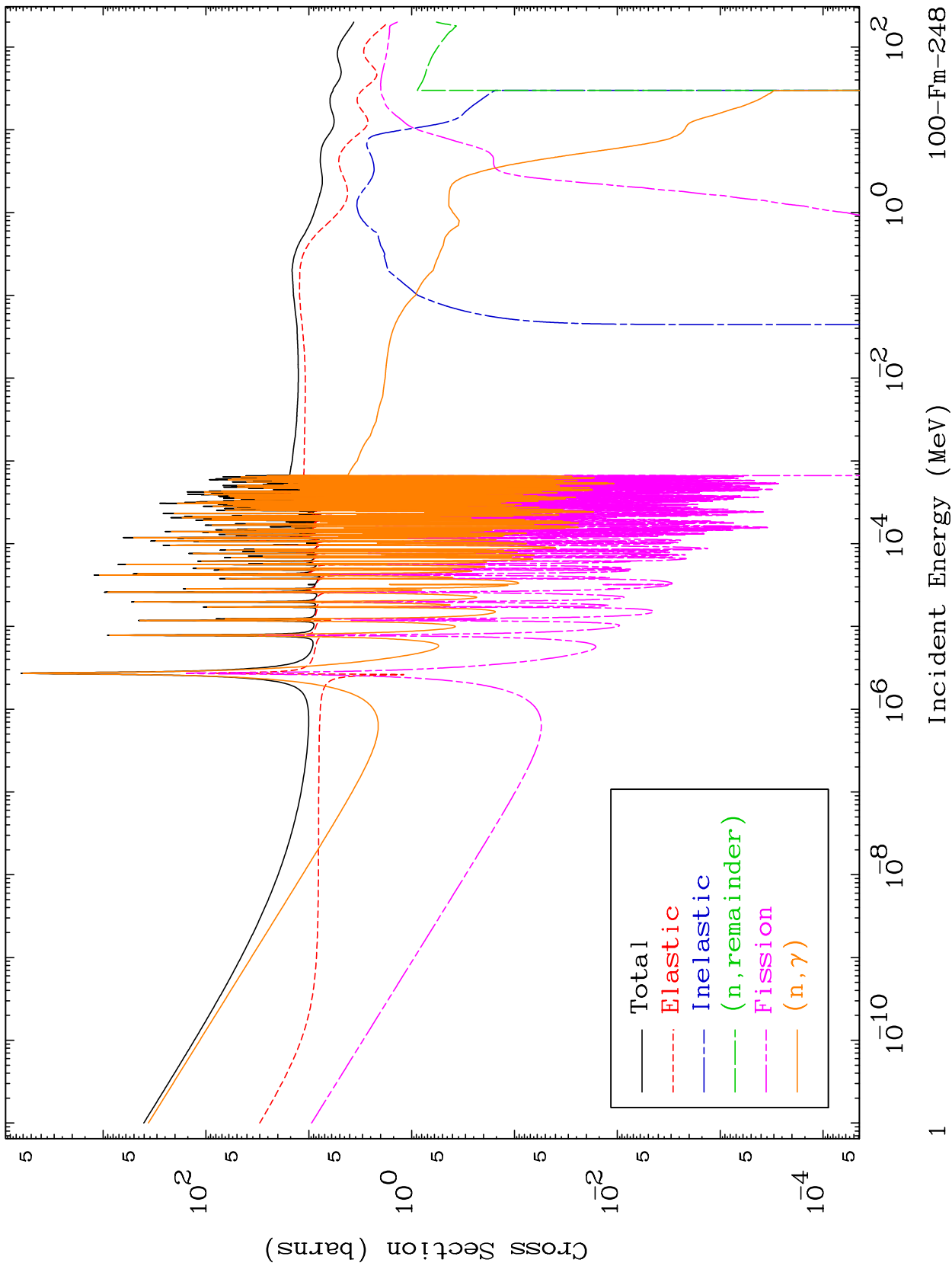
Press Mouse Button to Start

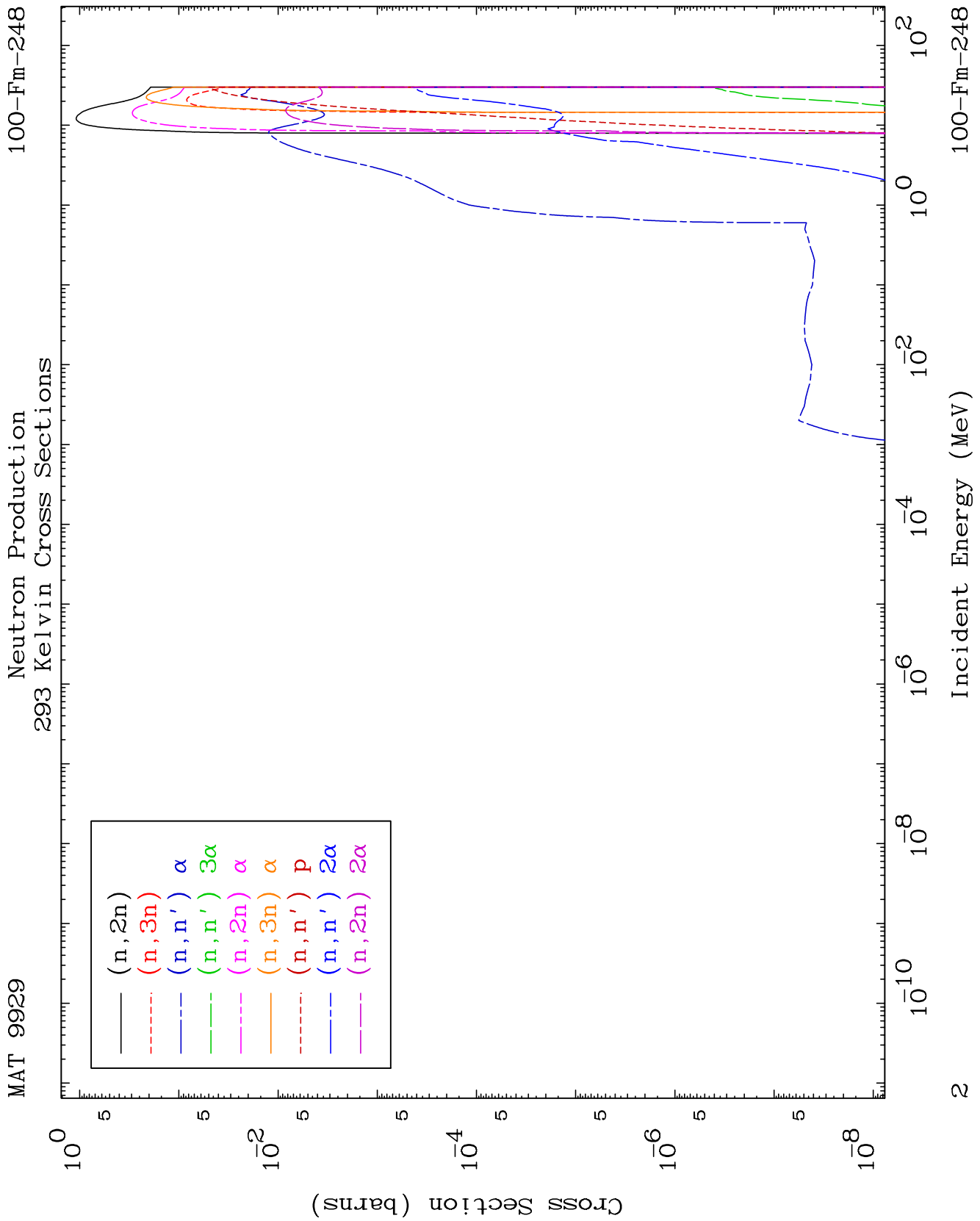
MAT 9929

Major

293 Kelvin Cross Sections

100-Fm-248

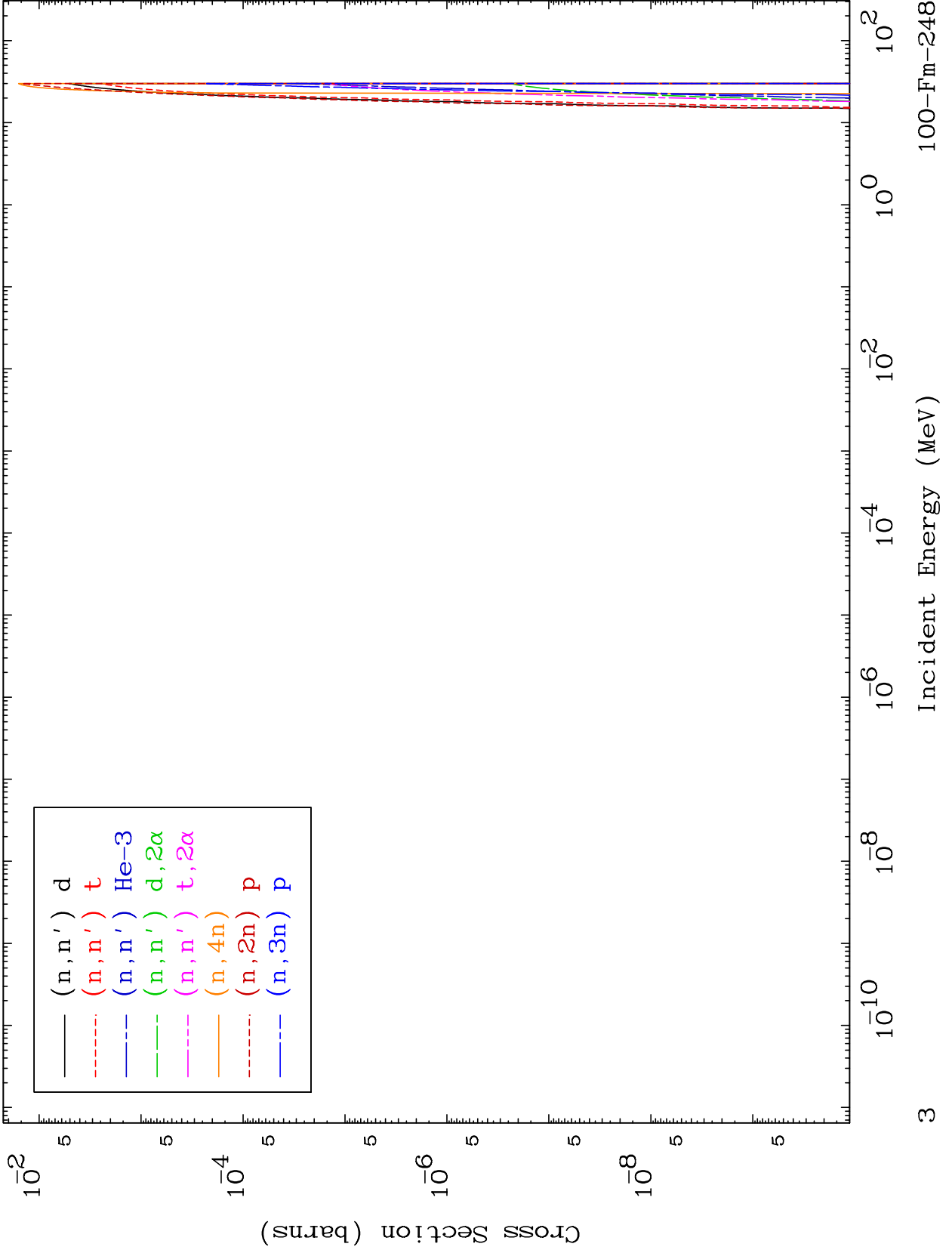




MAT 9929

Neutron Production
293 Kelvin Cross Sections

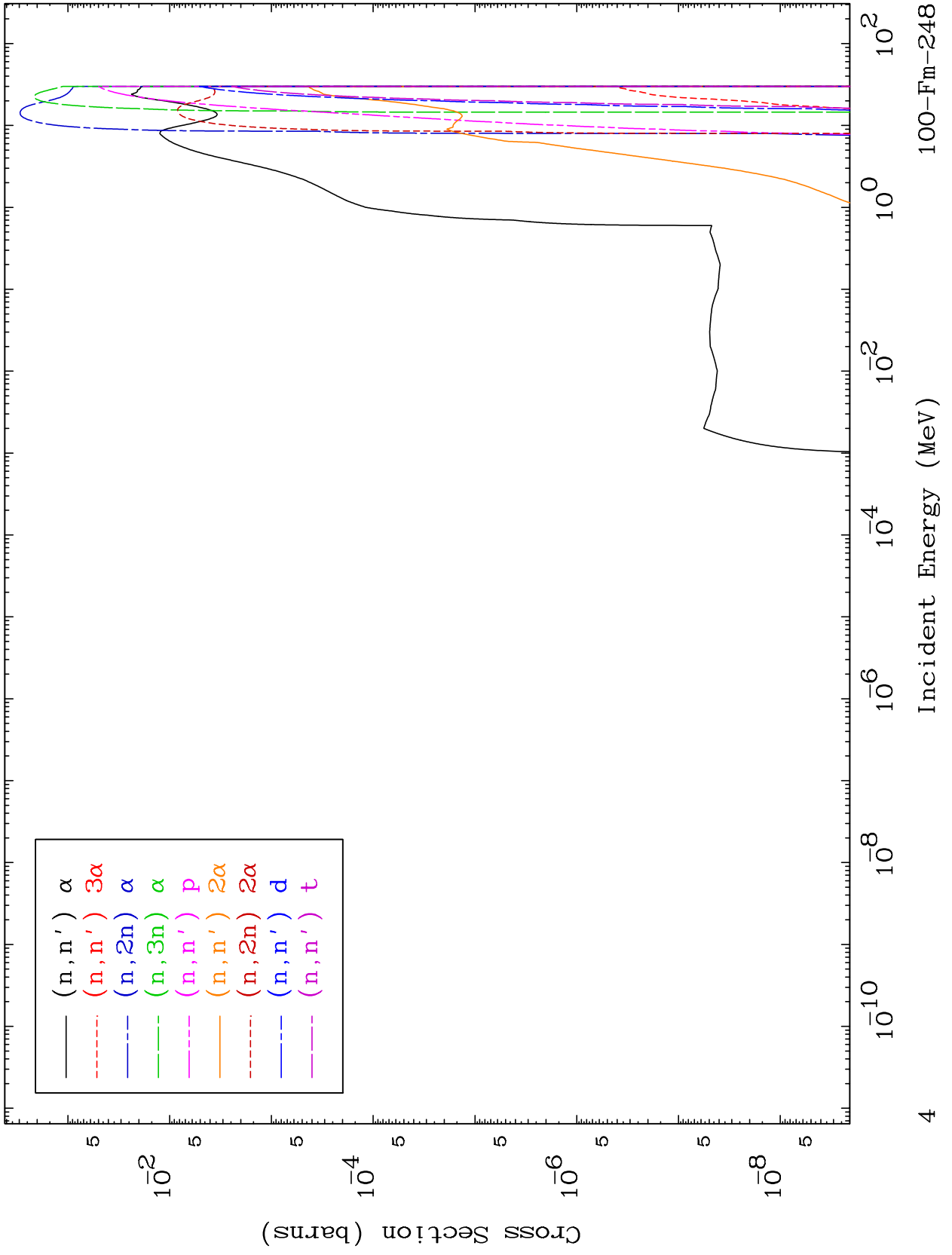
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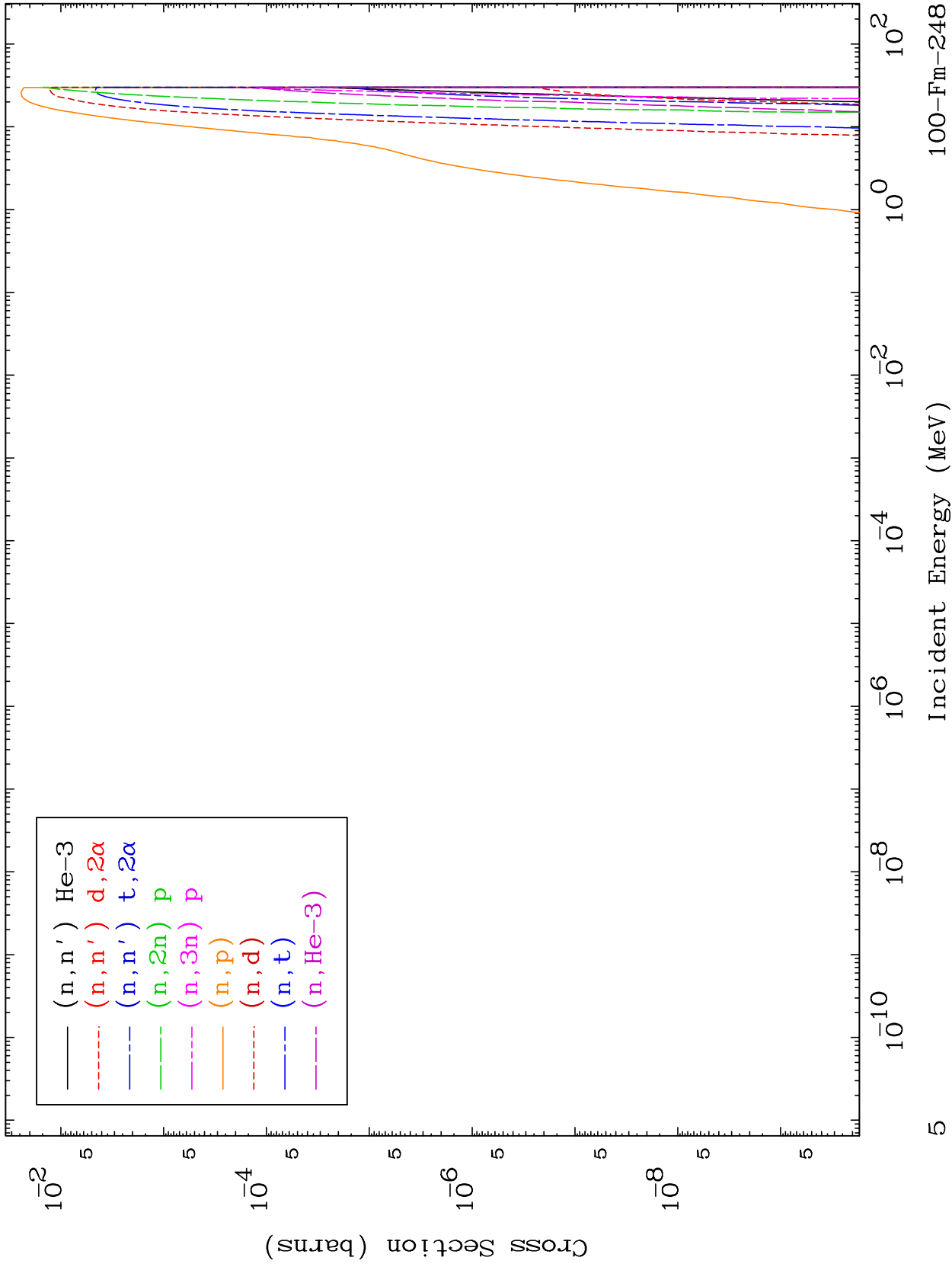


MAT 9929

Charged Particle
293 Kelvin Cross Sections

100-Fm-248

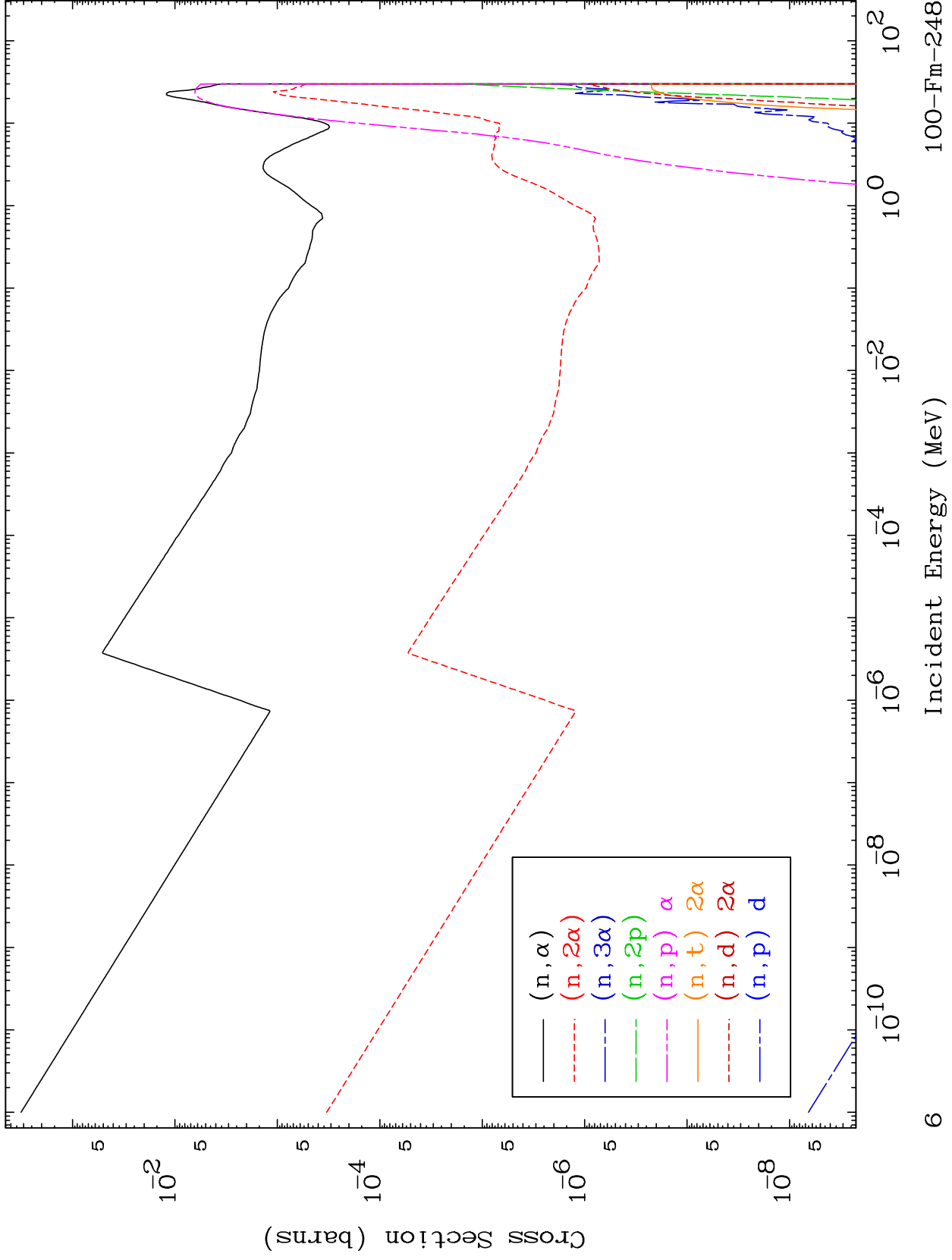


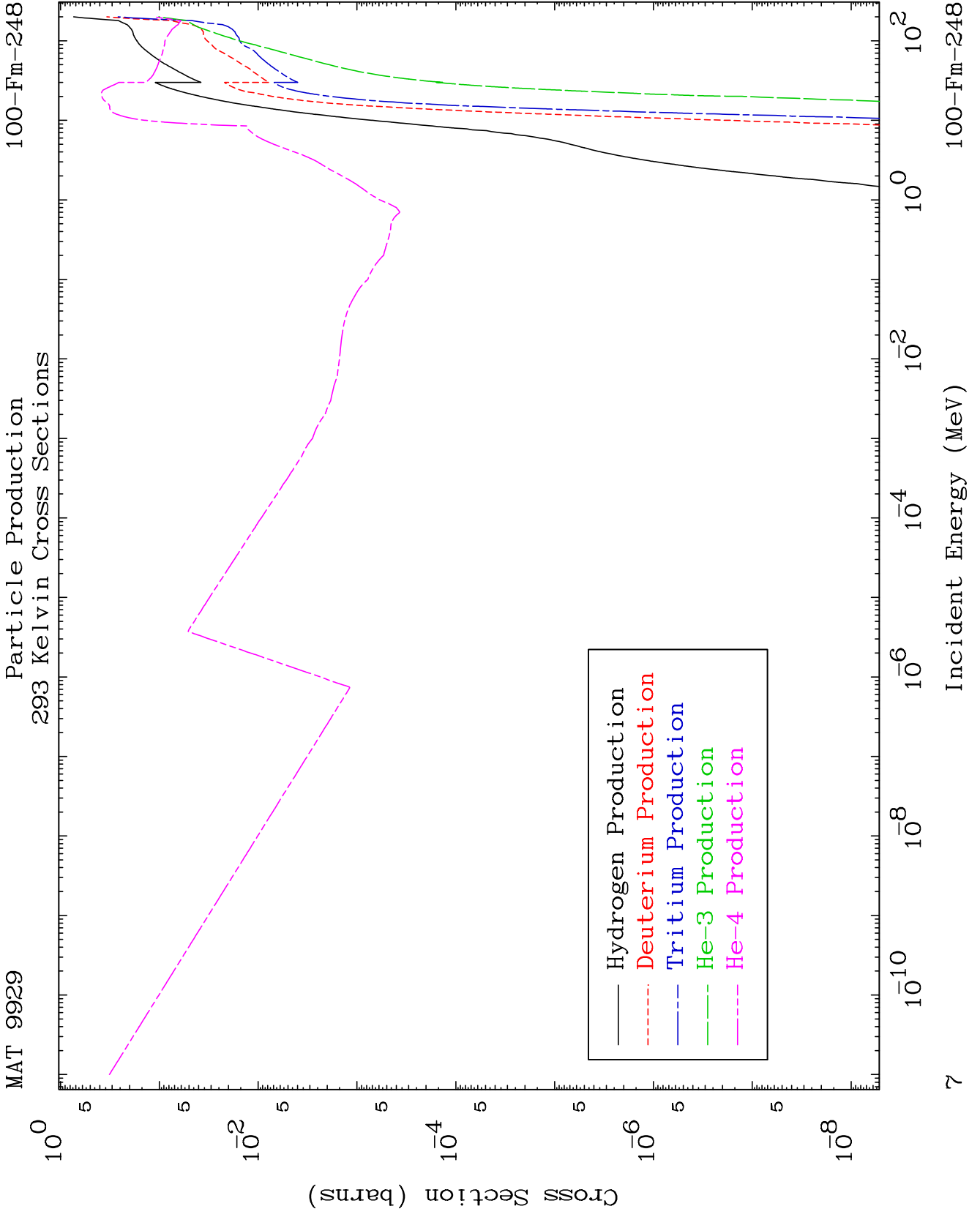


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Charged Particle
293 Kelvin Cross Sections

100-Fm-248

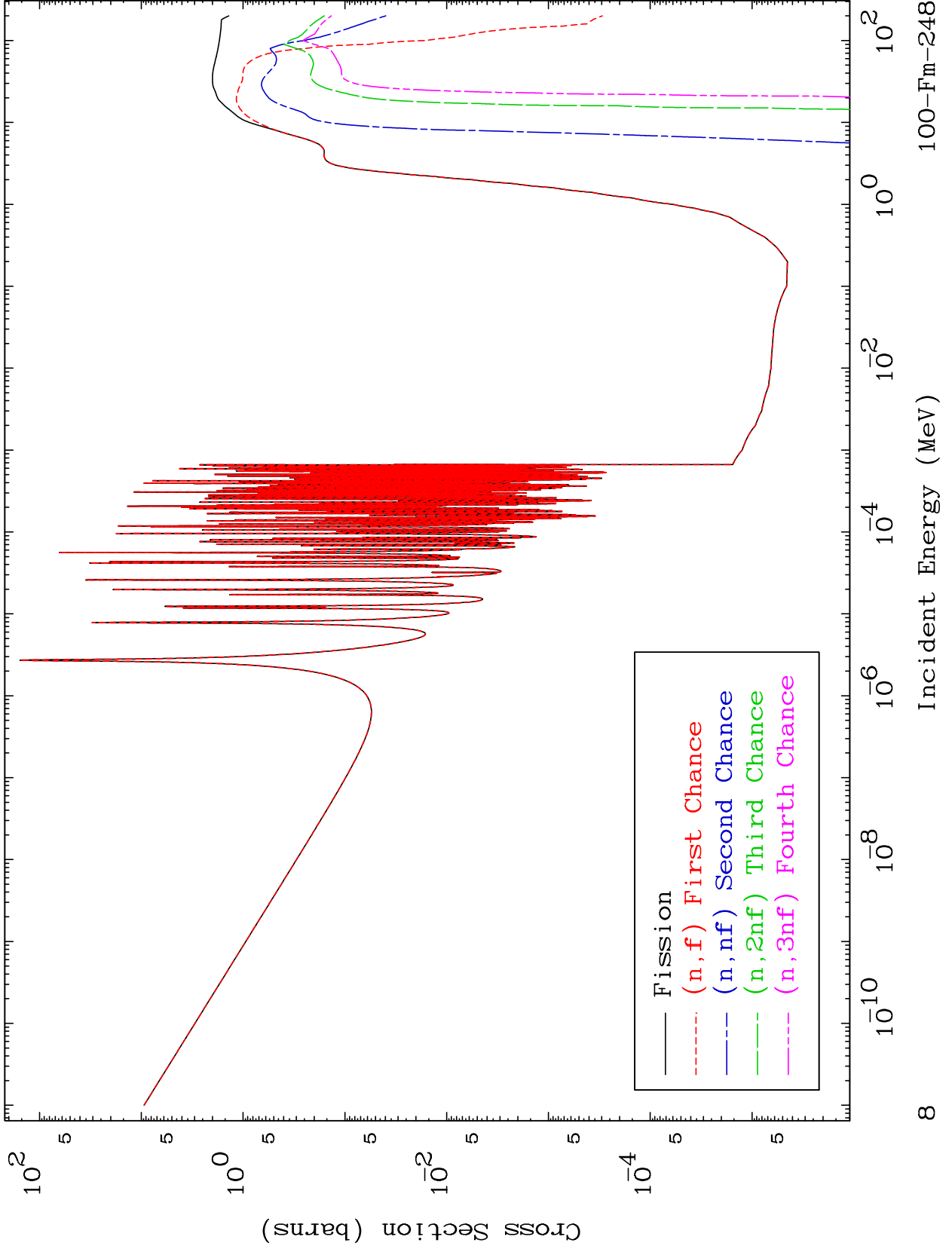




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Fission
293 Kelvin Cross Sections

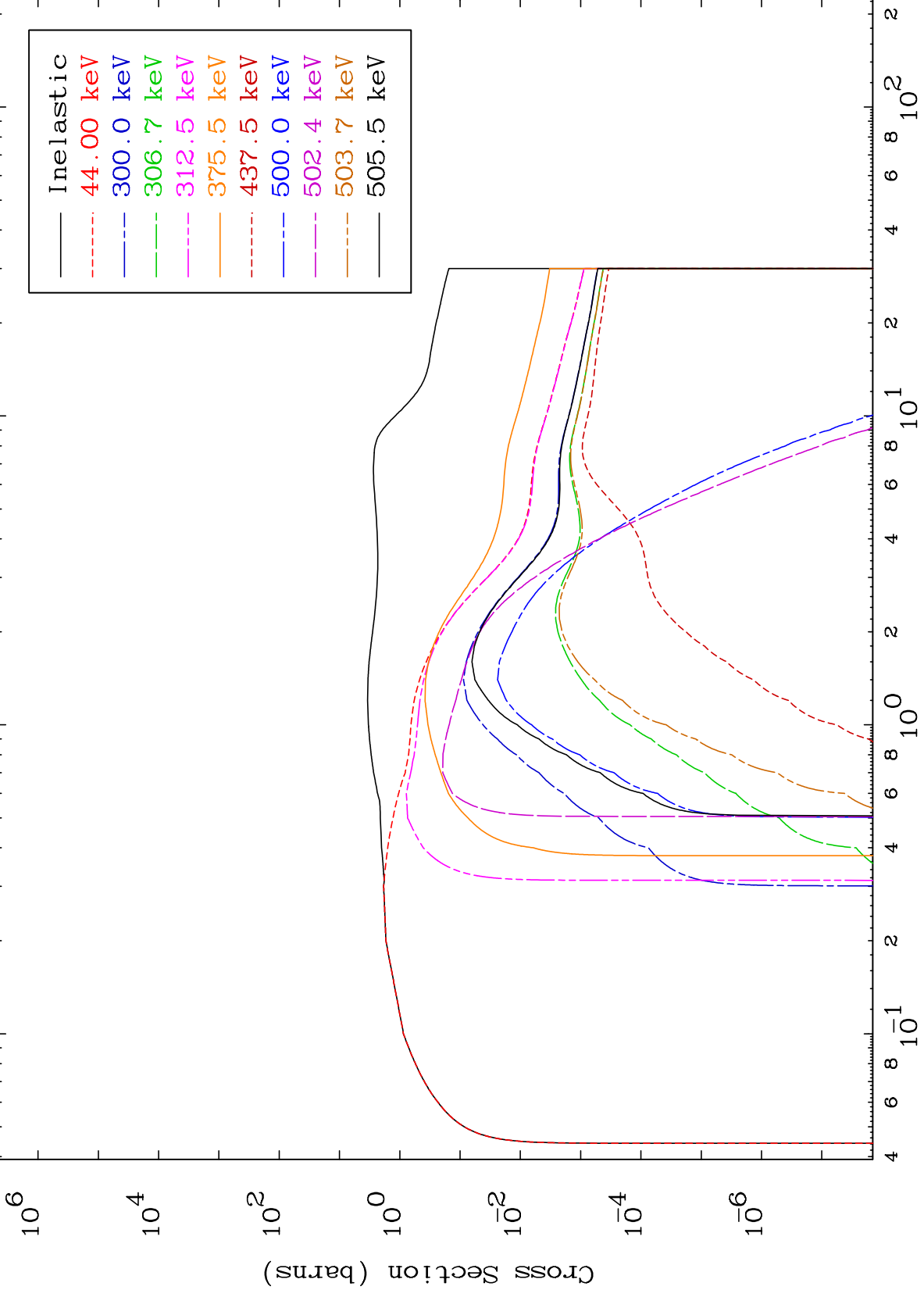
100-Fm-248



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(n,n') Level
293 Kelvin Cross Sections

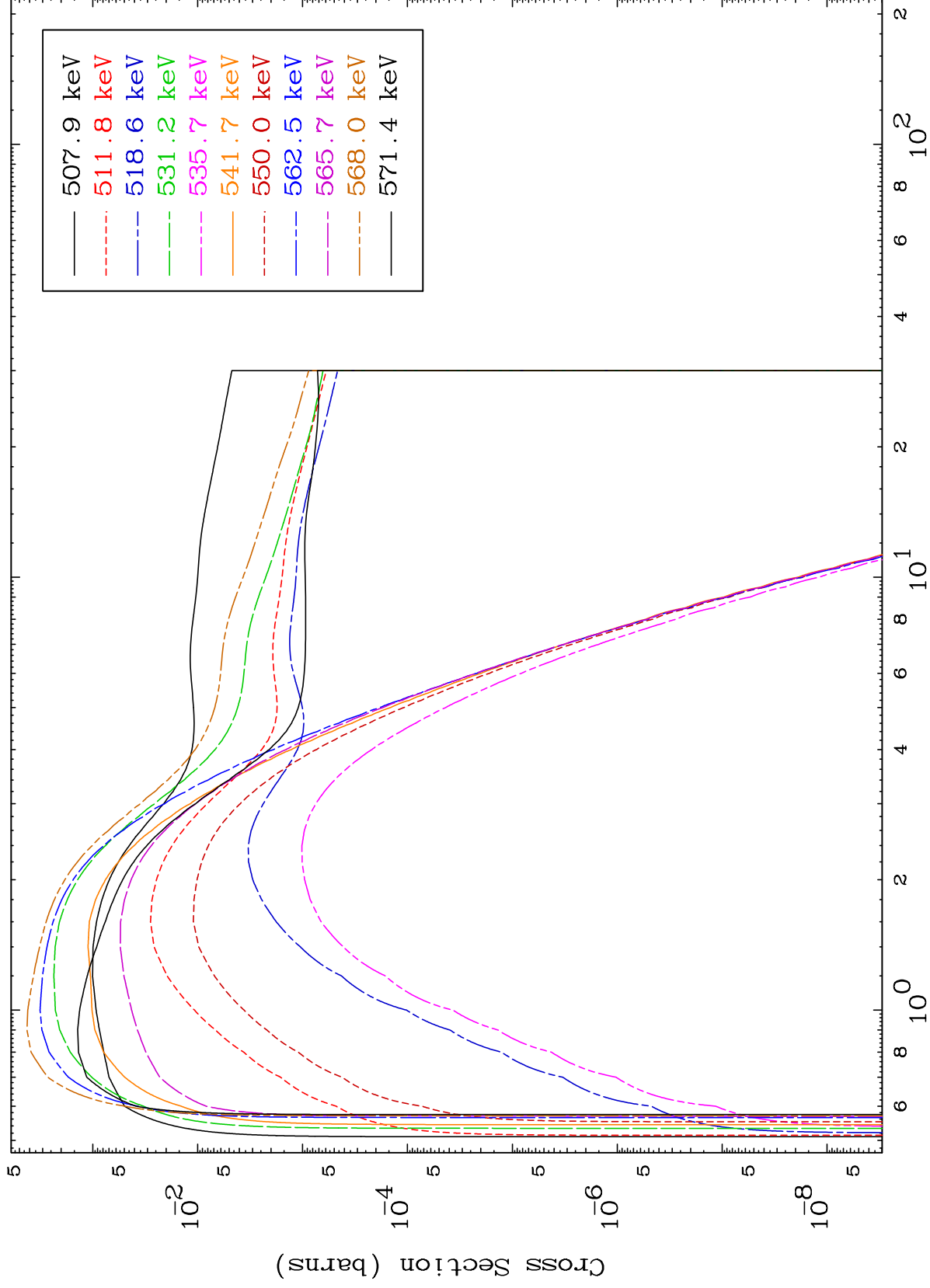
100-Fm-248



MAT 9929

(n,n') Level
293 Kelvin Cross Sections

100-Fm-248



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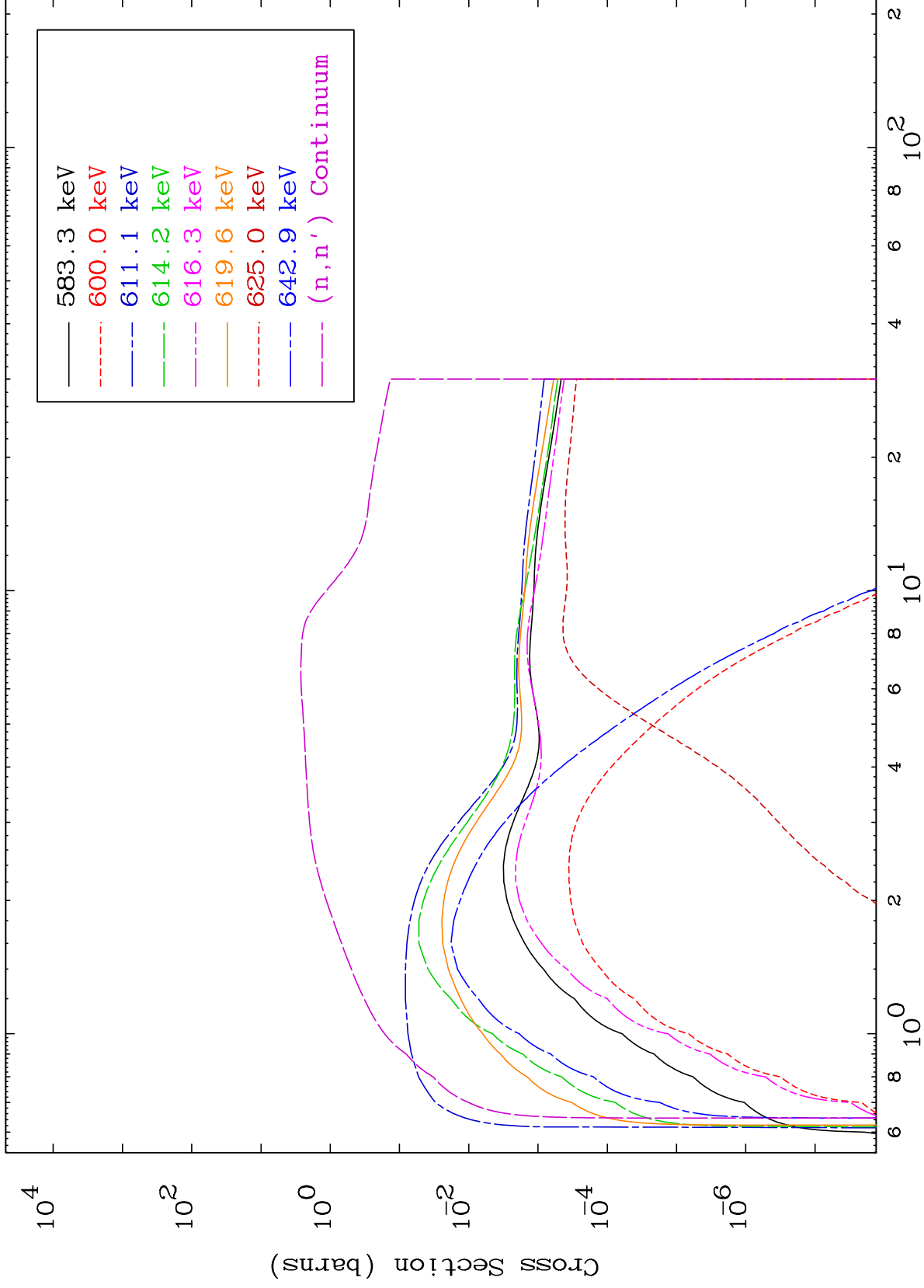
Incident Energy (MeV)

100-Fm-248

MAT 9929

(n,n') Level
293 Kelvin Cross Sections

100-Fm-248



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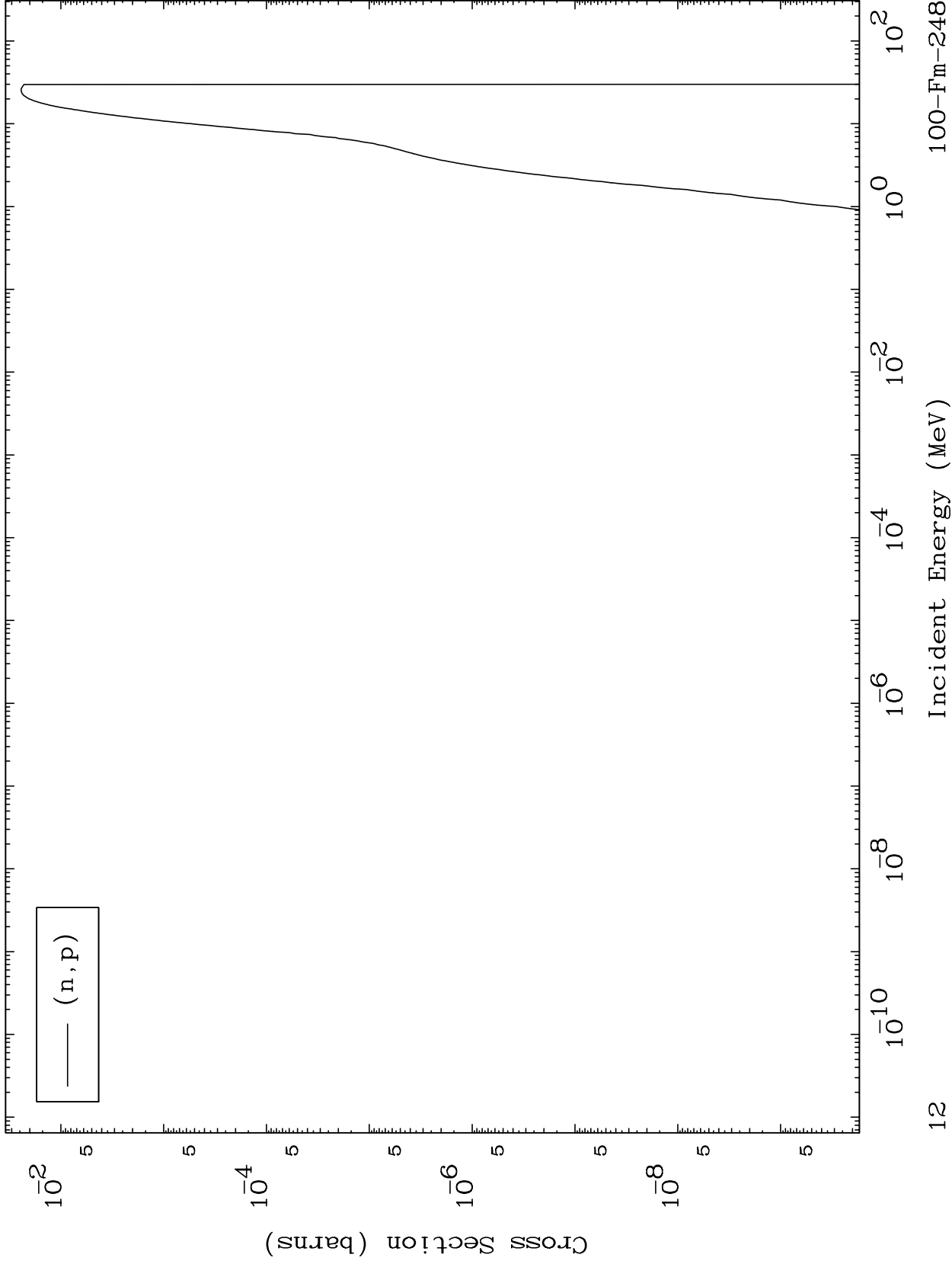
Incident Energy (MeV)

100-Fm-248

MAT 9929

(n,p) Levels
293 Kelvin Cross Sections

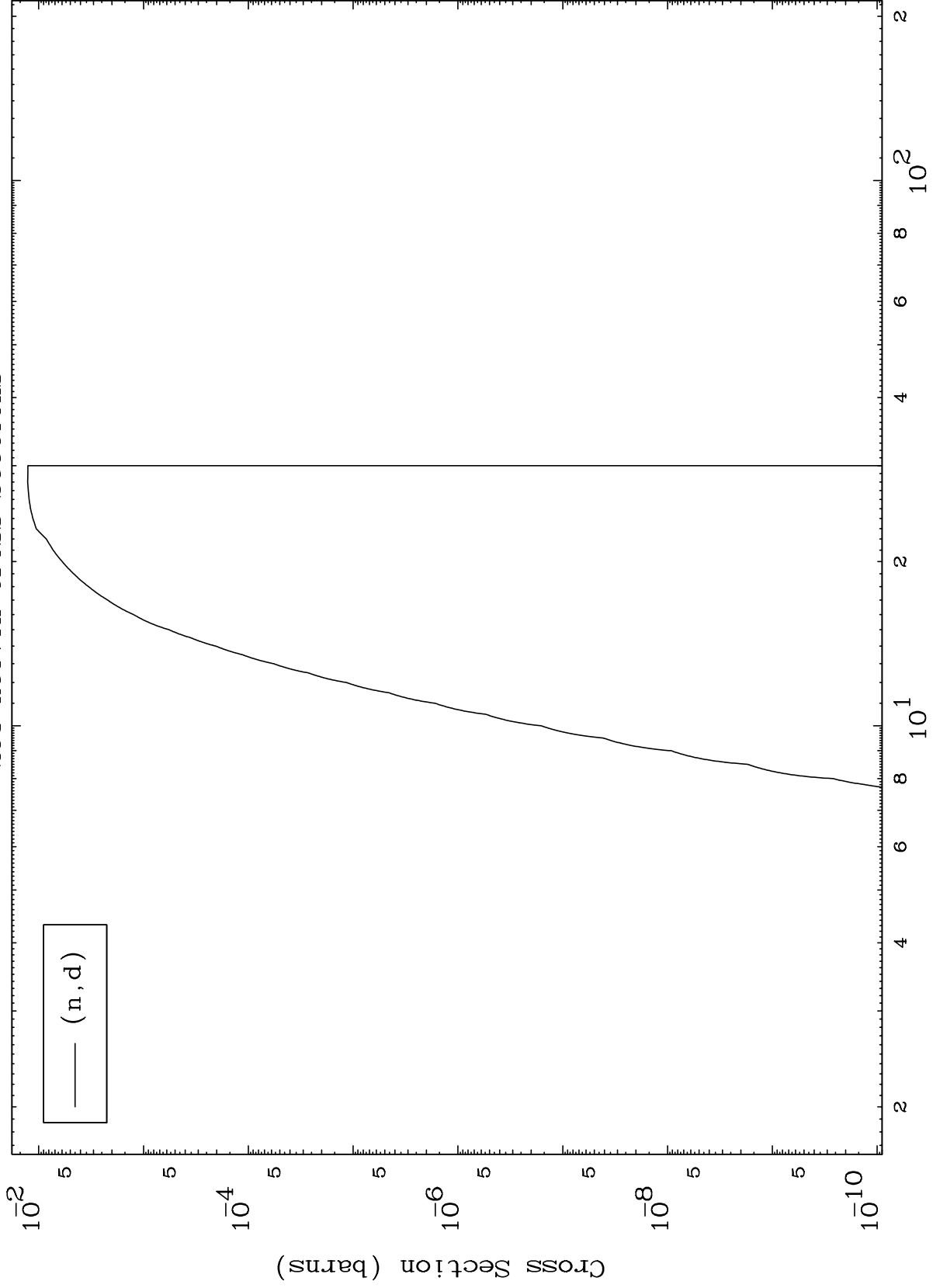
100-Fm-248



MAT 9929

(n,d) Levels
293 Kelvin Cross Sections

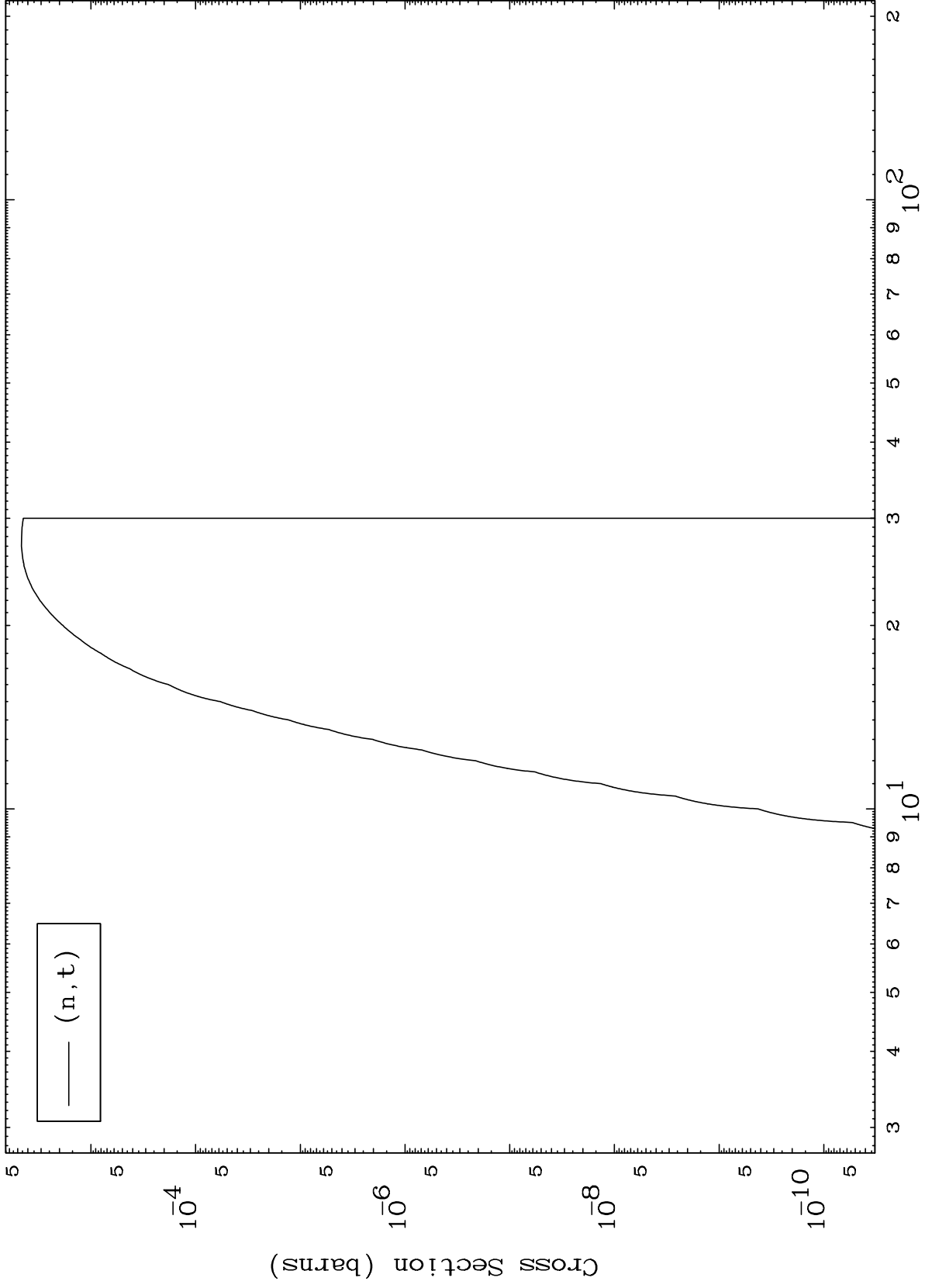
100-Fm-248



MAT 9929

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-248



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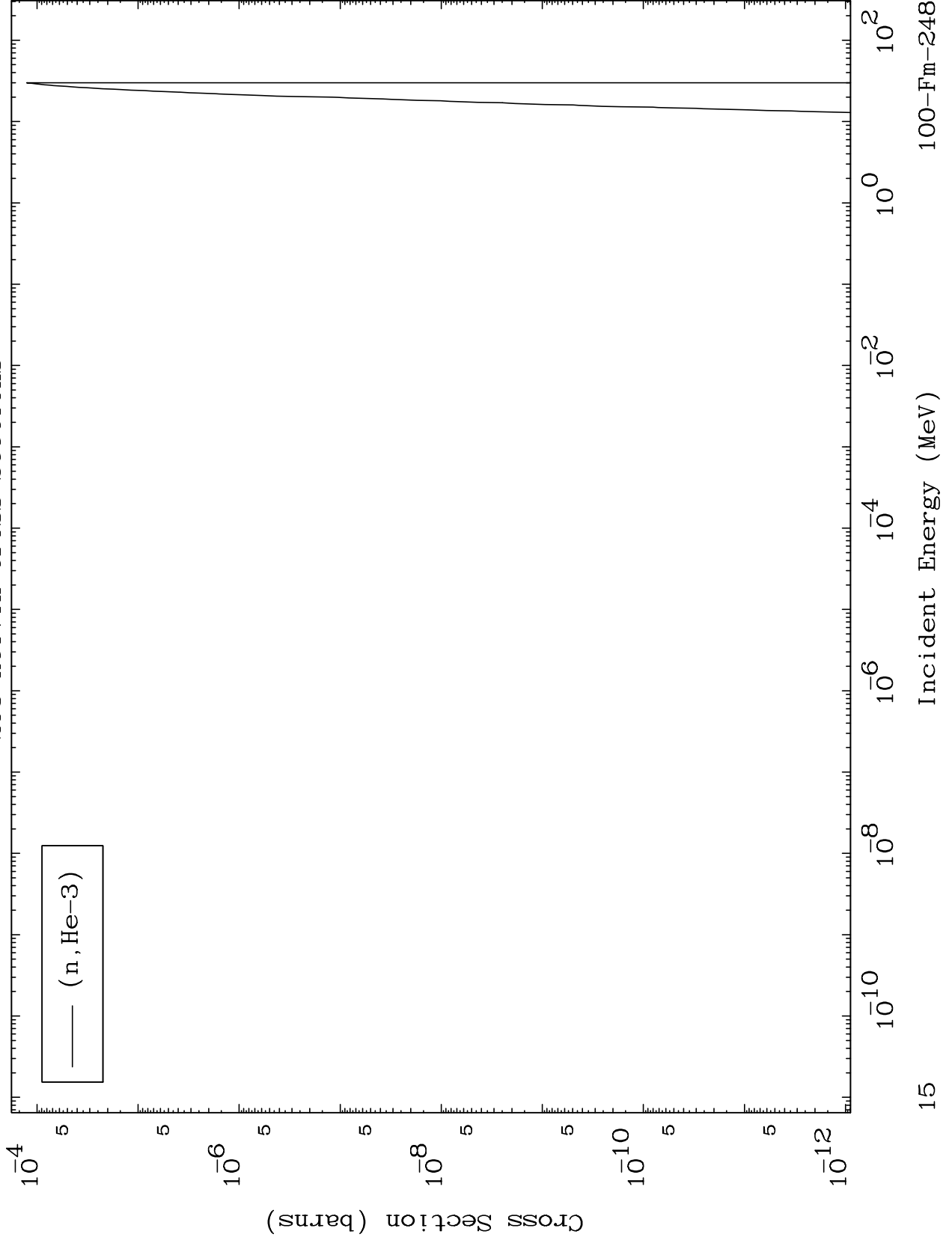
Incident Energy (MeV)

100-Fm-248

MAT 9929

(n,He3) Levels
293 Kelvin Cross Sections

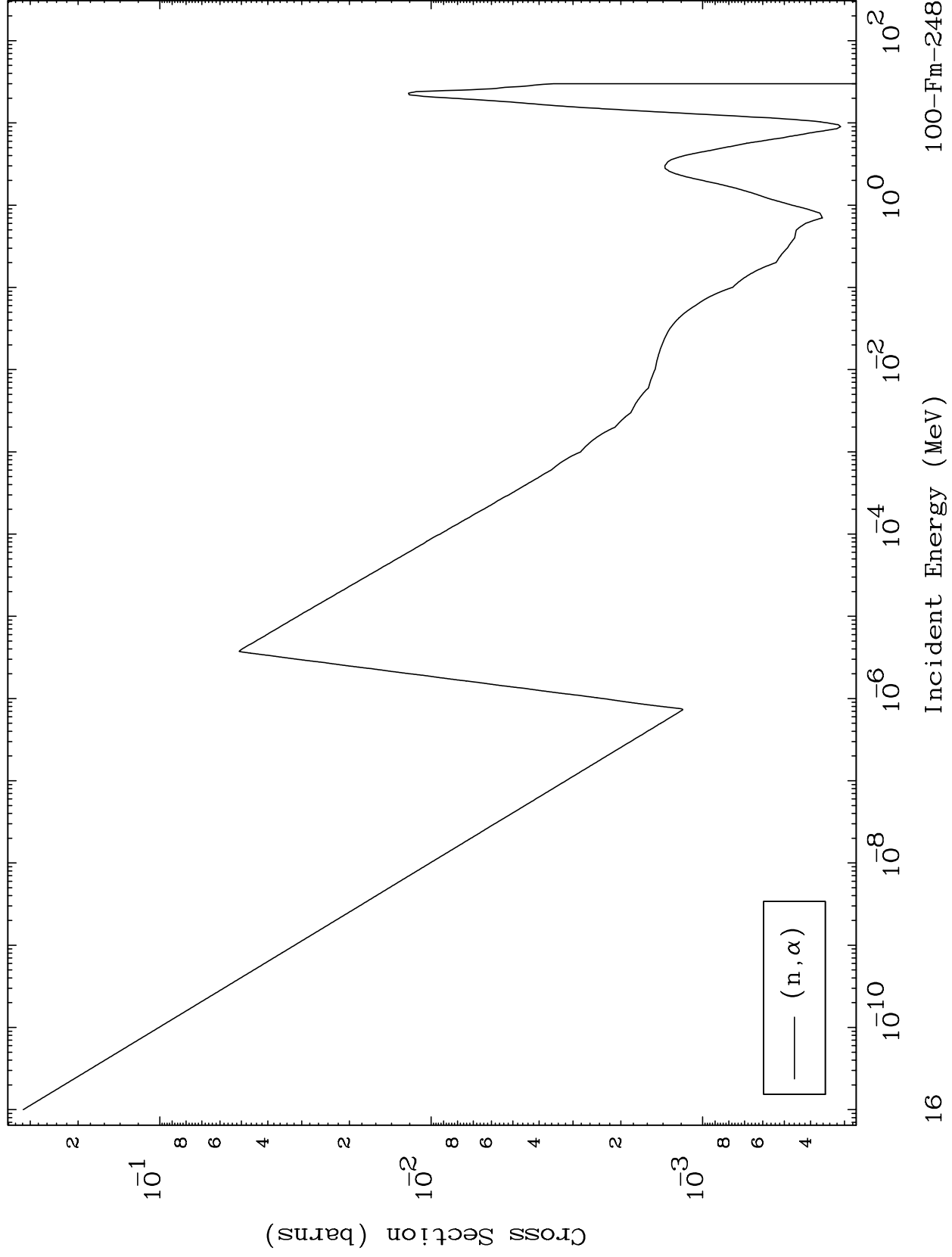
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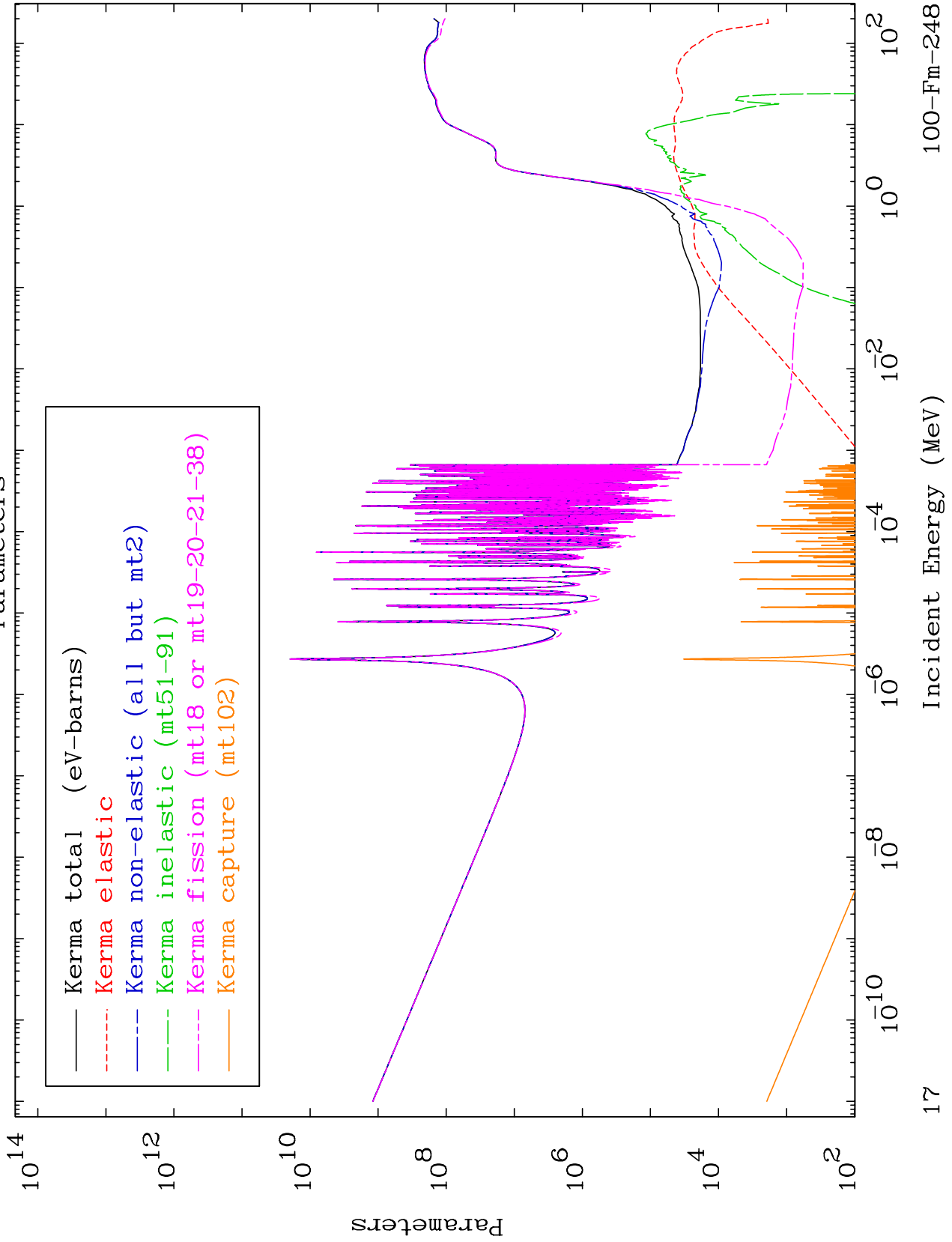


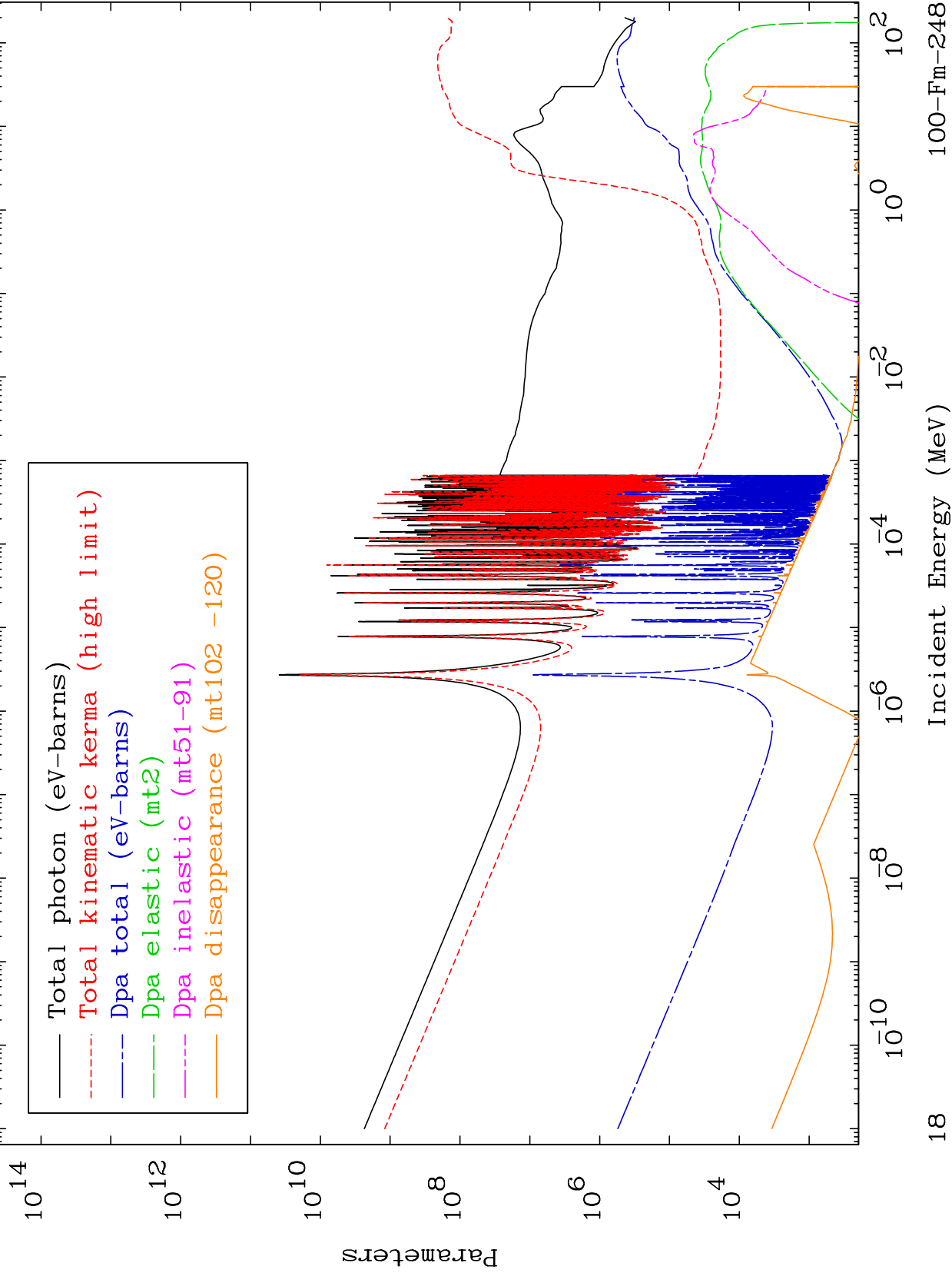
MAT 9929

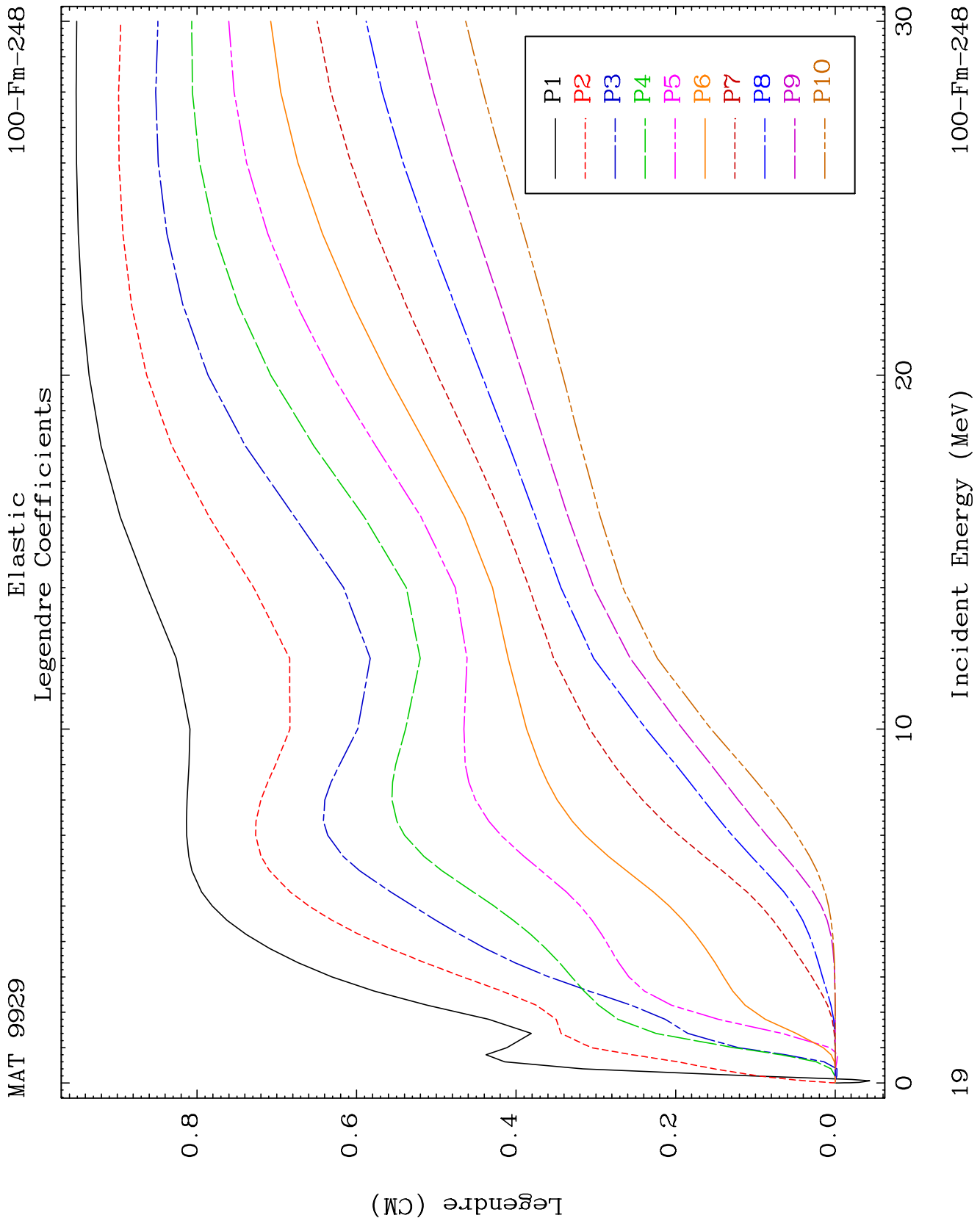
(n,α) Levels
293 Kelvin Cross Sections

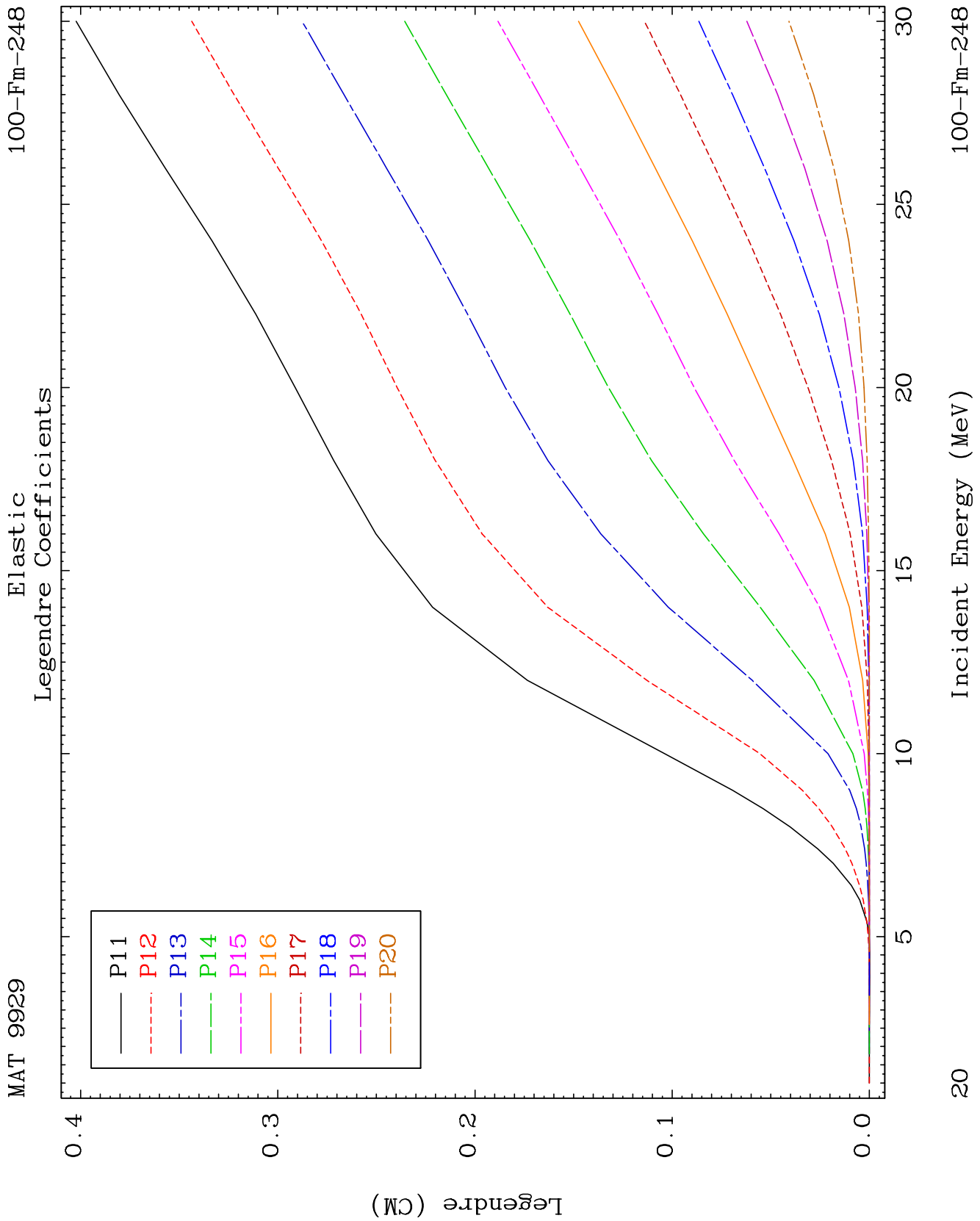
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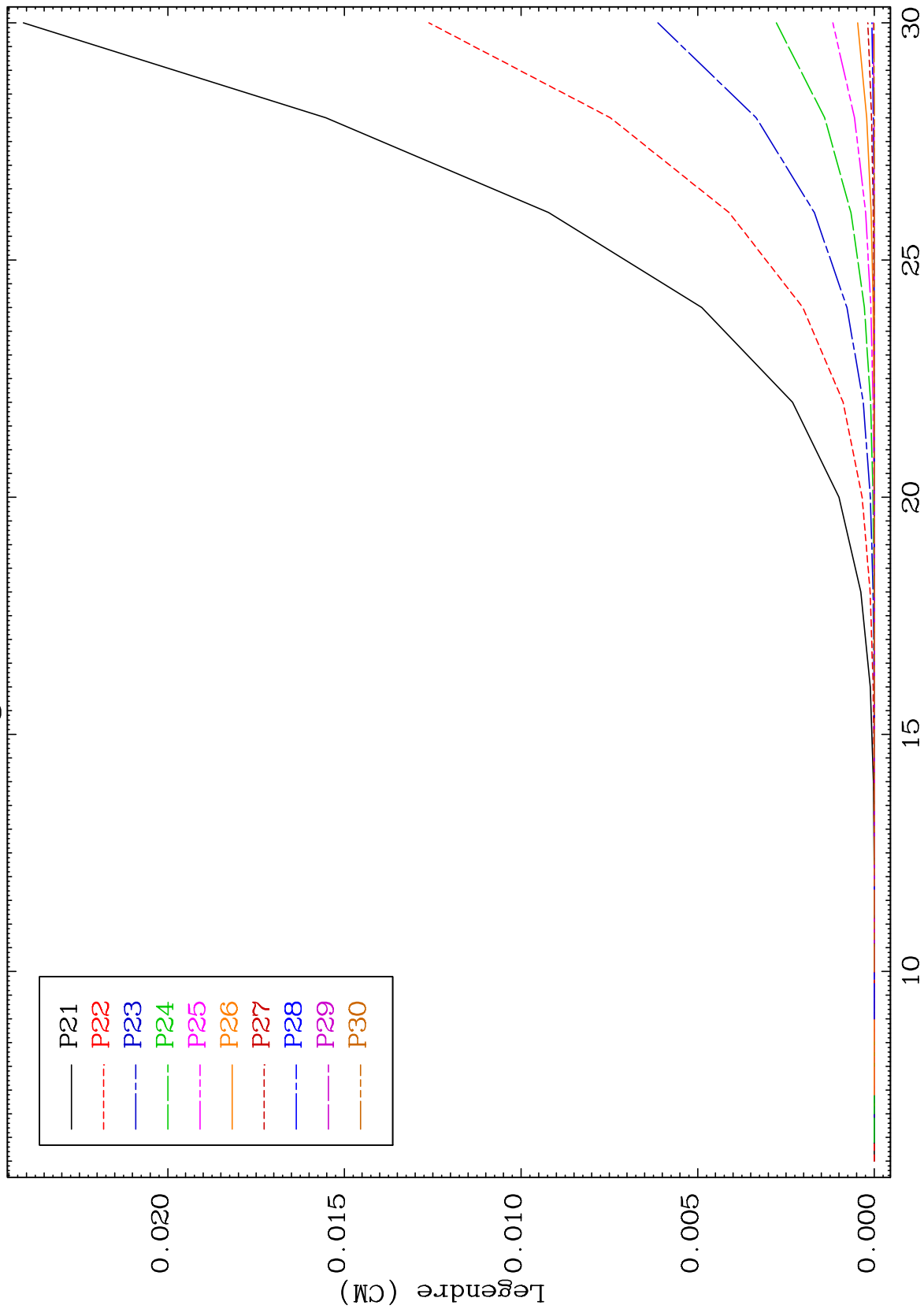




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Elastic Legendre Coefficients

100-Fm-248



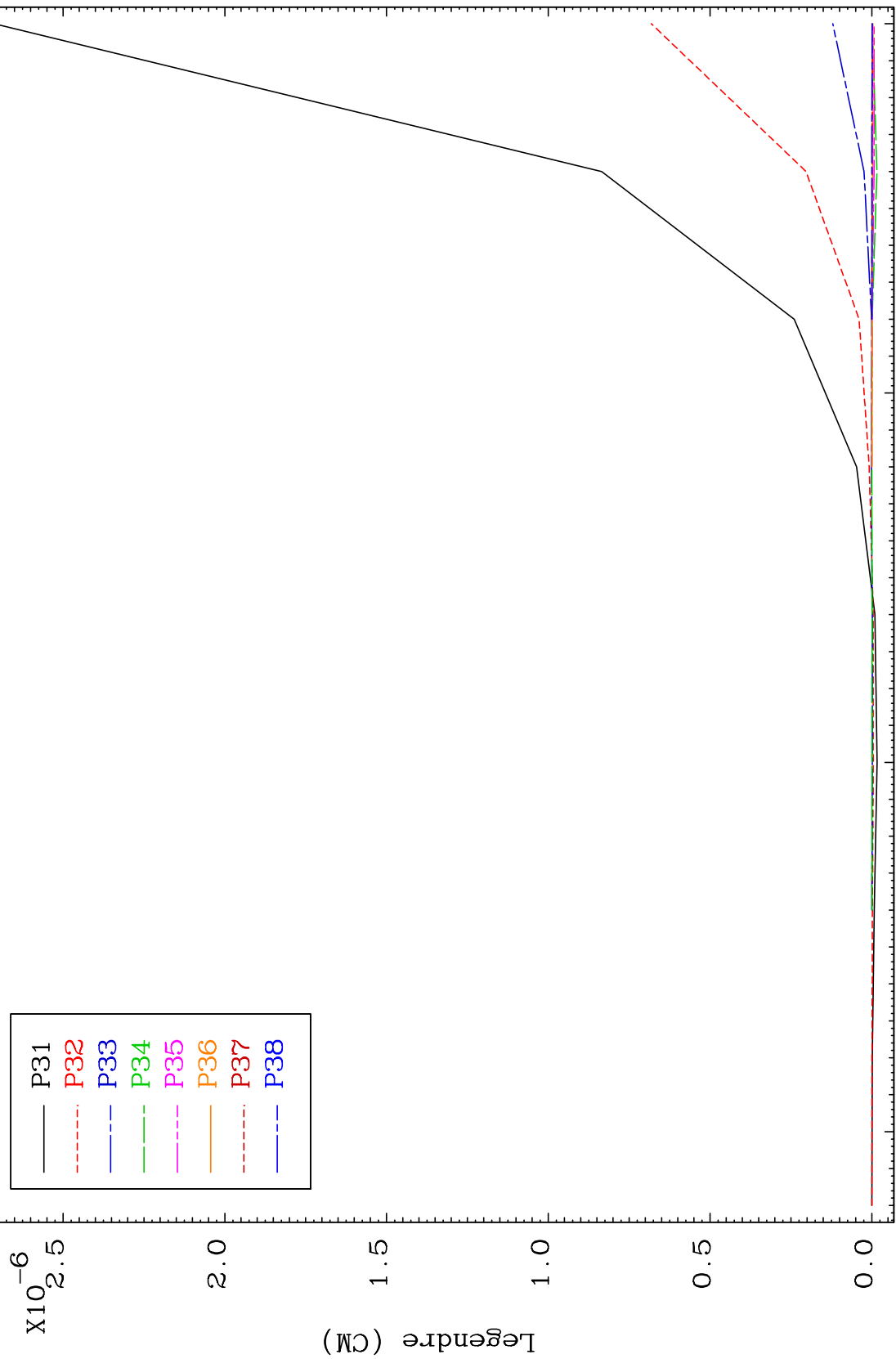
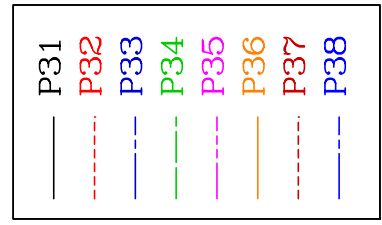
100-Fm-248

Incident Energy (MeV)

MAT 9929

Elastic Legendre Coefficients

100-Fm-248

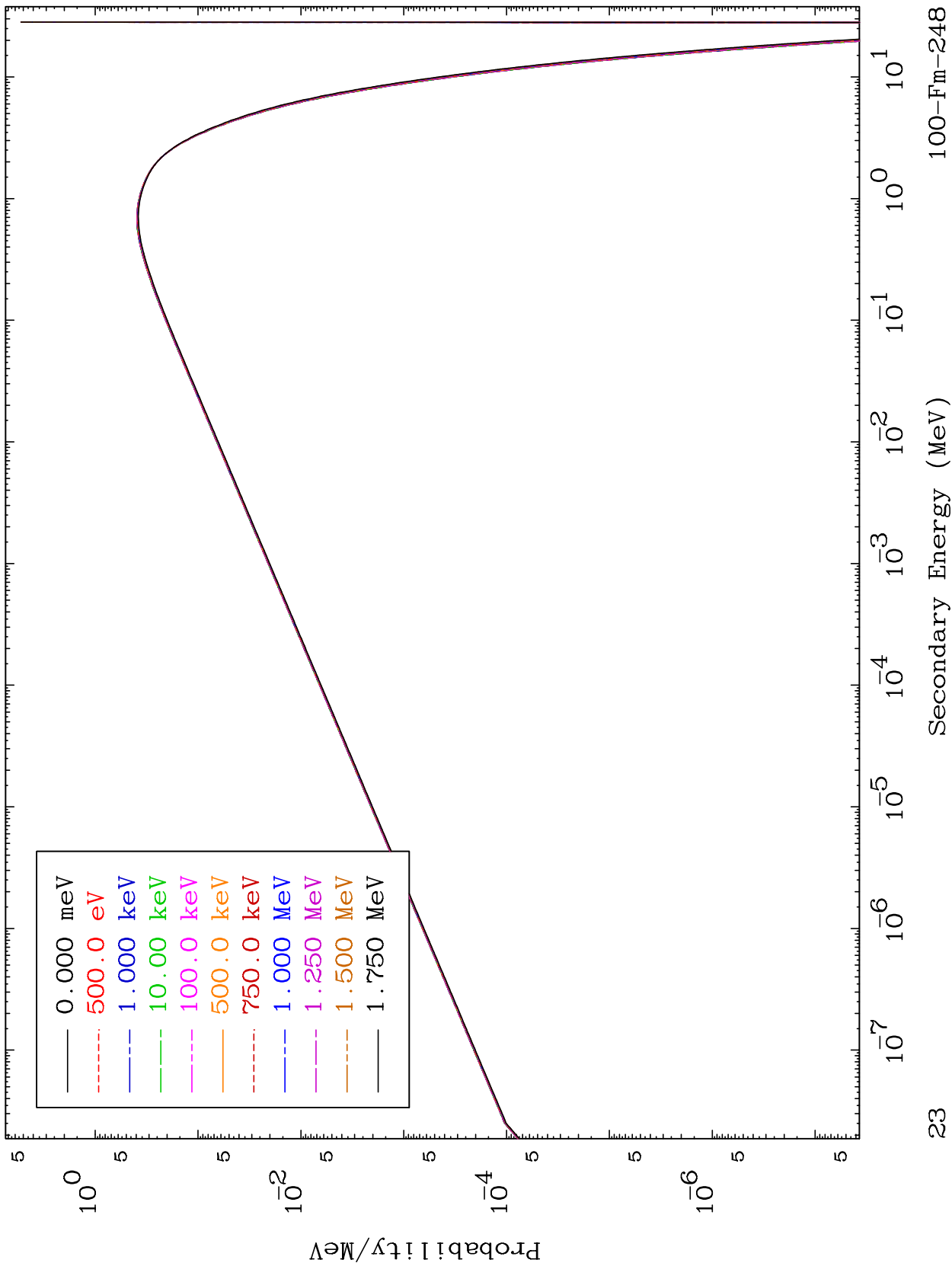


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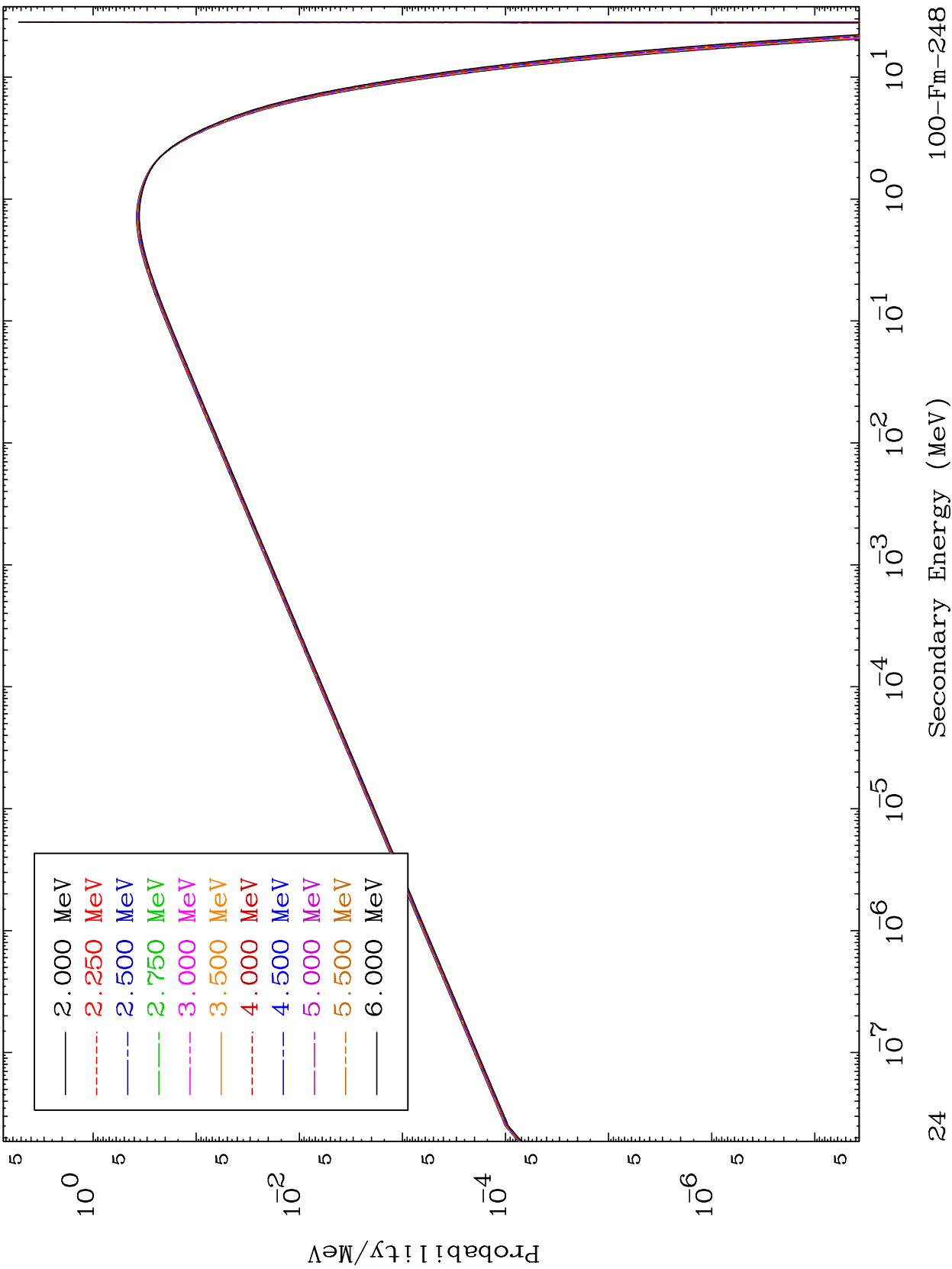
Incident Energy (MeV)

100-Fm-248

Fission Energy Distribution



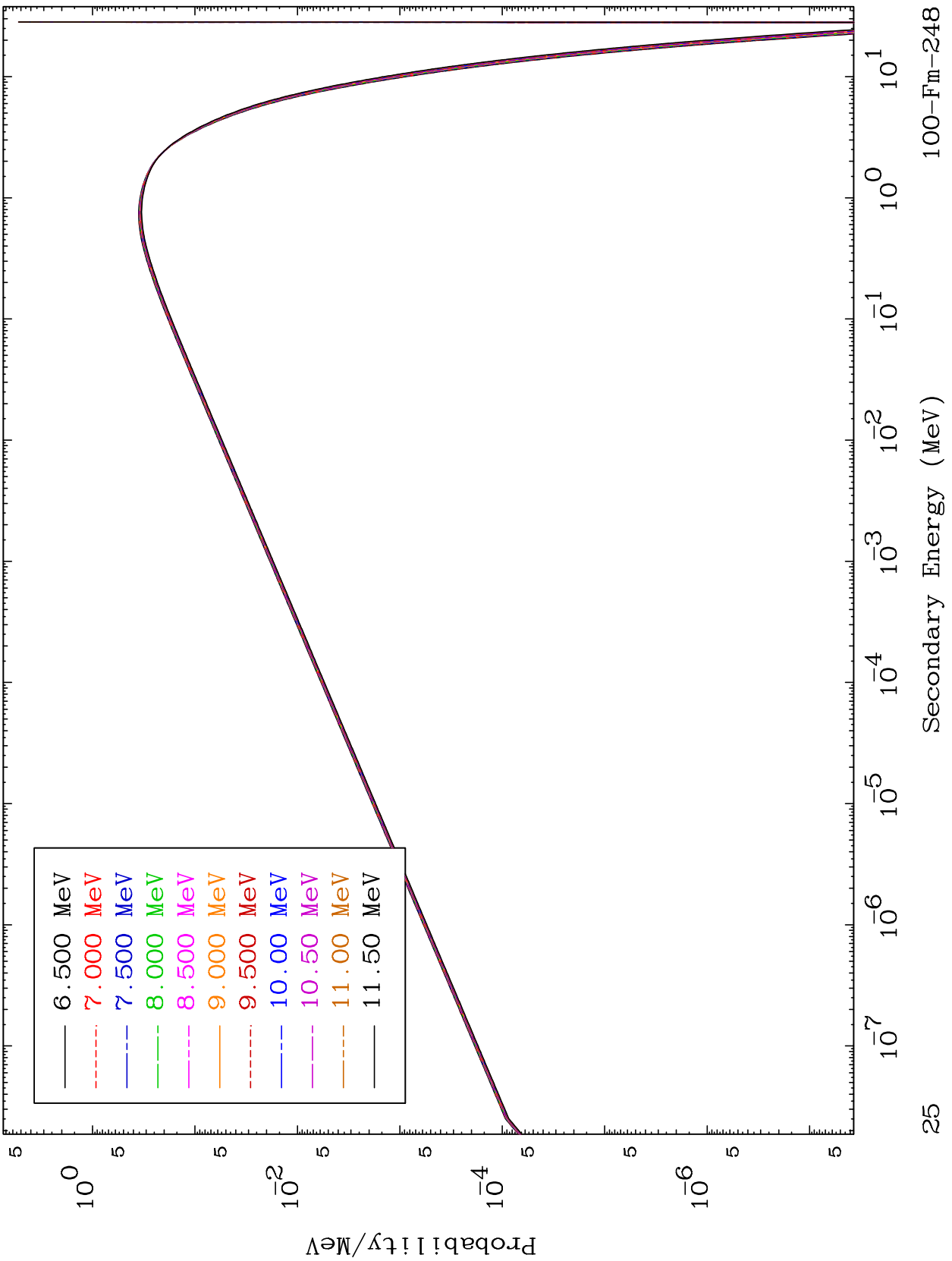
Fission Energy Distribution



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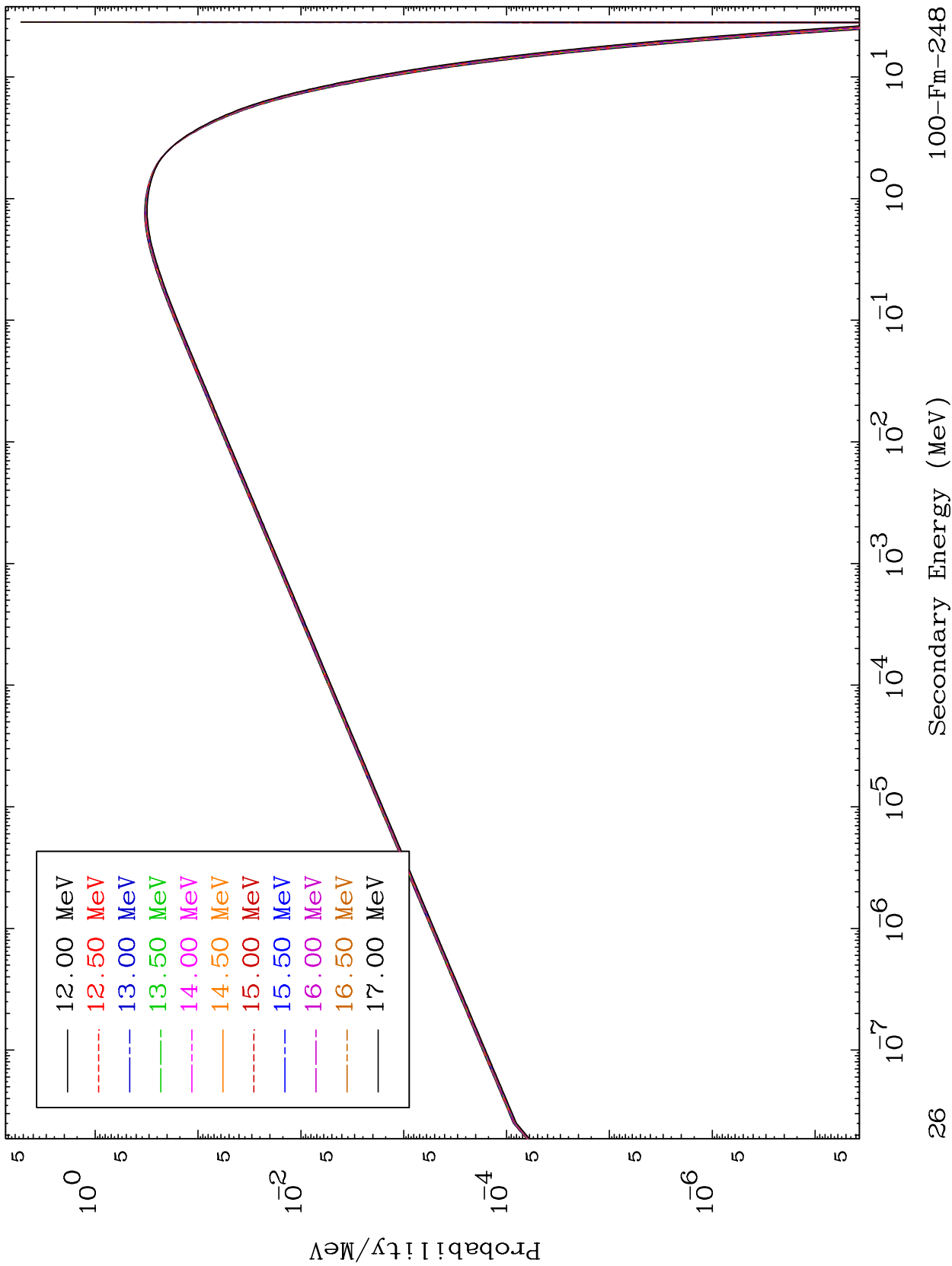
Fission Energy Distribution

100-Fm-248

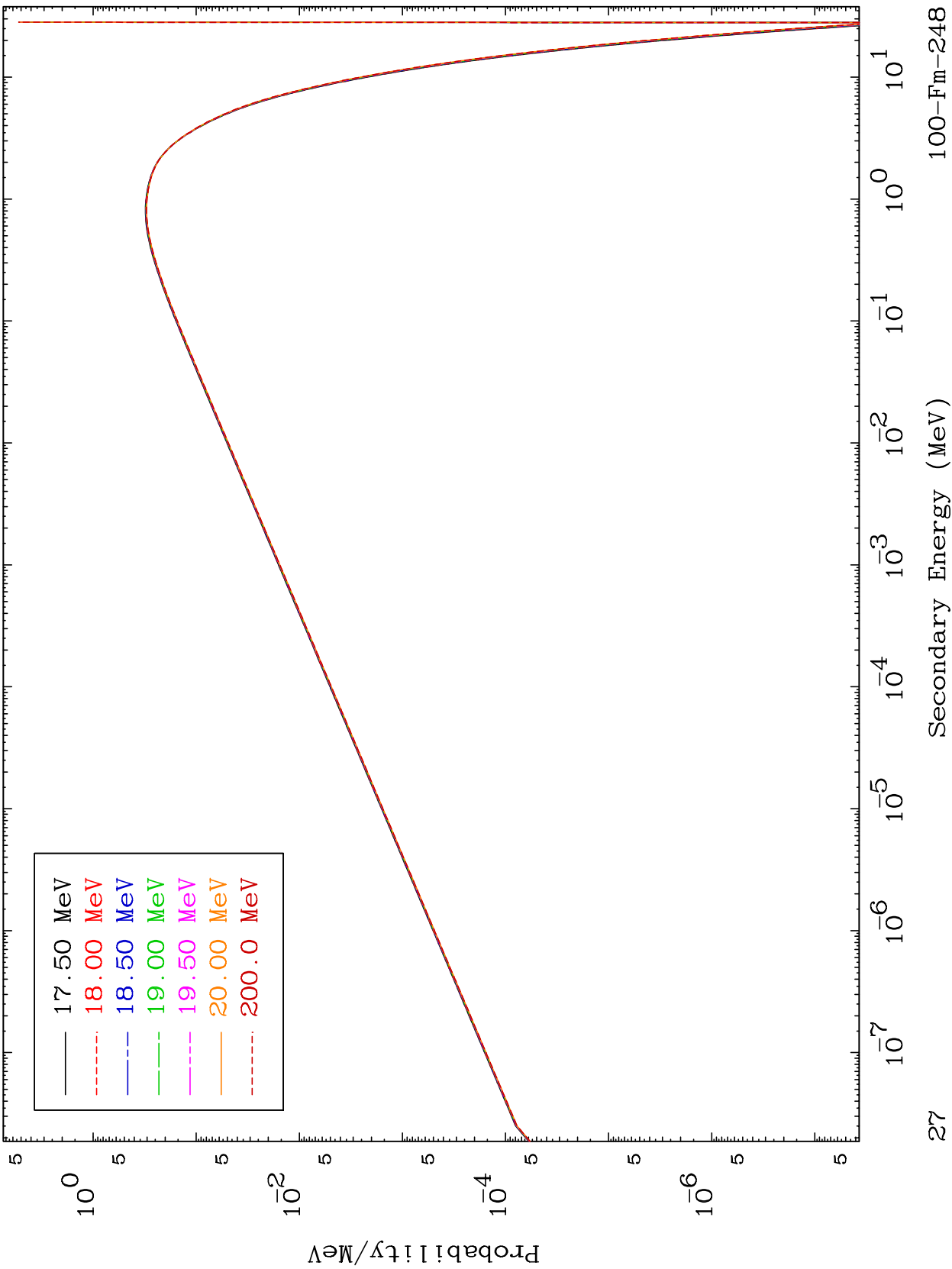


100-Fm-248

Fission Energy Distribution



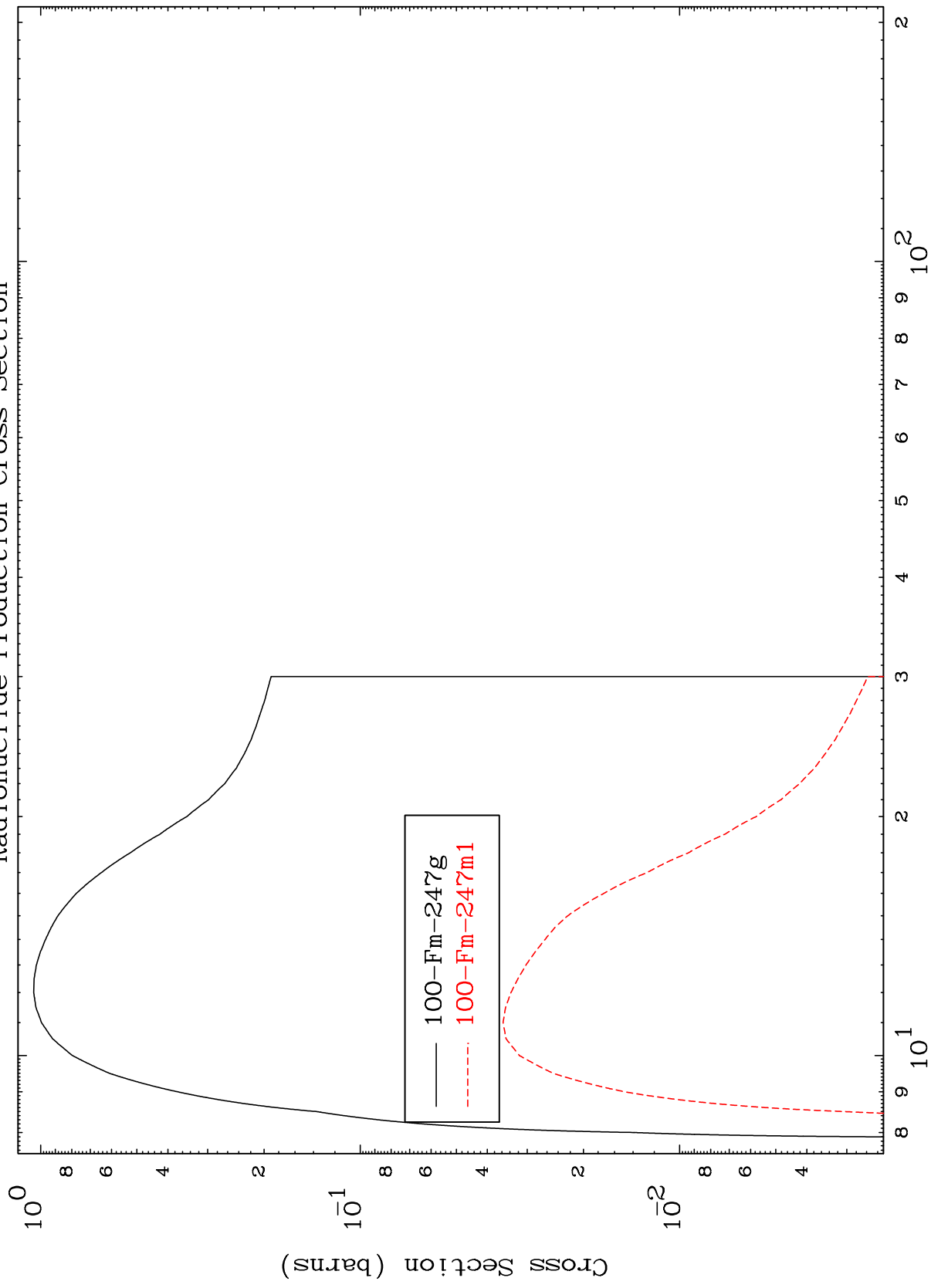
Fission
Energy Distribution



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100-Fm-248

(n,2n)
Radionuclide Production Cross Section



100-Fm-248

Incident Energy (MeV)

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(n, 3 α)

100-Fm-248

Radionuclide Production Cross Section

