

Program EVALPLOT  
(Version 2015-2)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net  
Web:home.comcast.net/~redcullen1

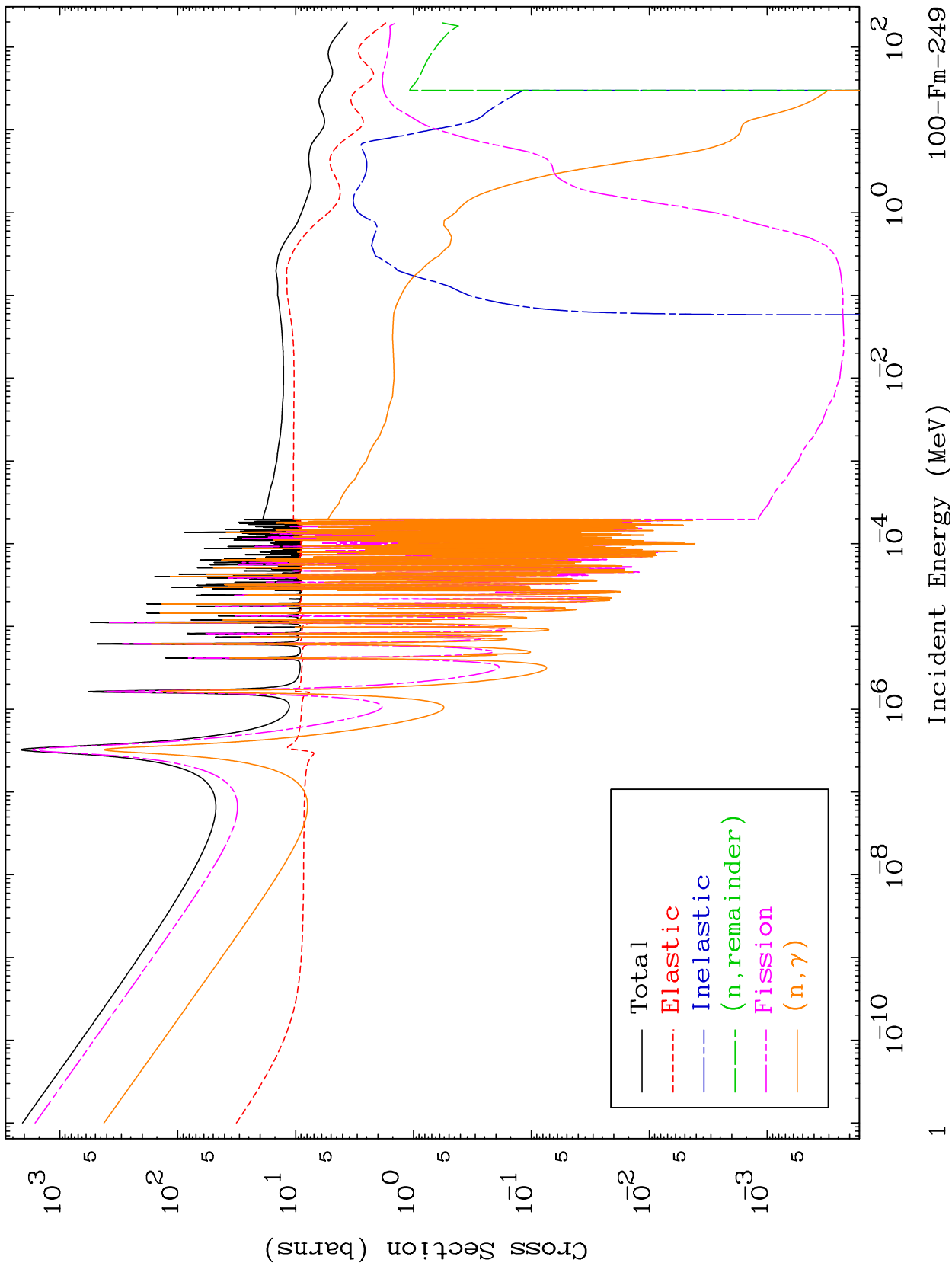
Press Mouse Button to Start

MAT 9930

Major

293 Kelvin Cross Sections

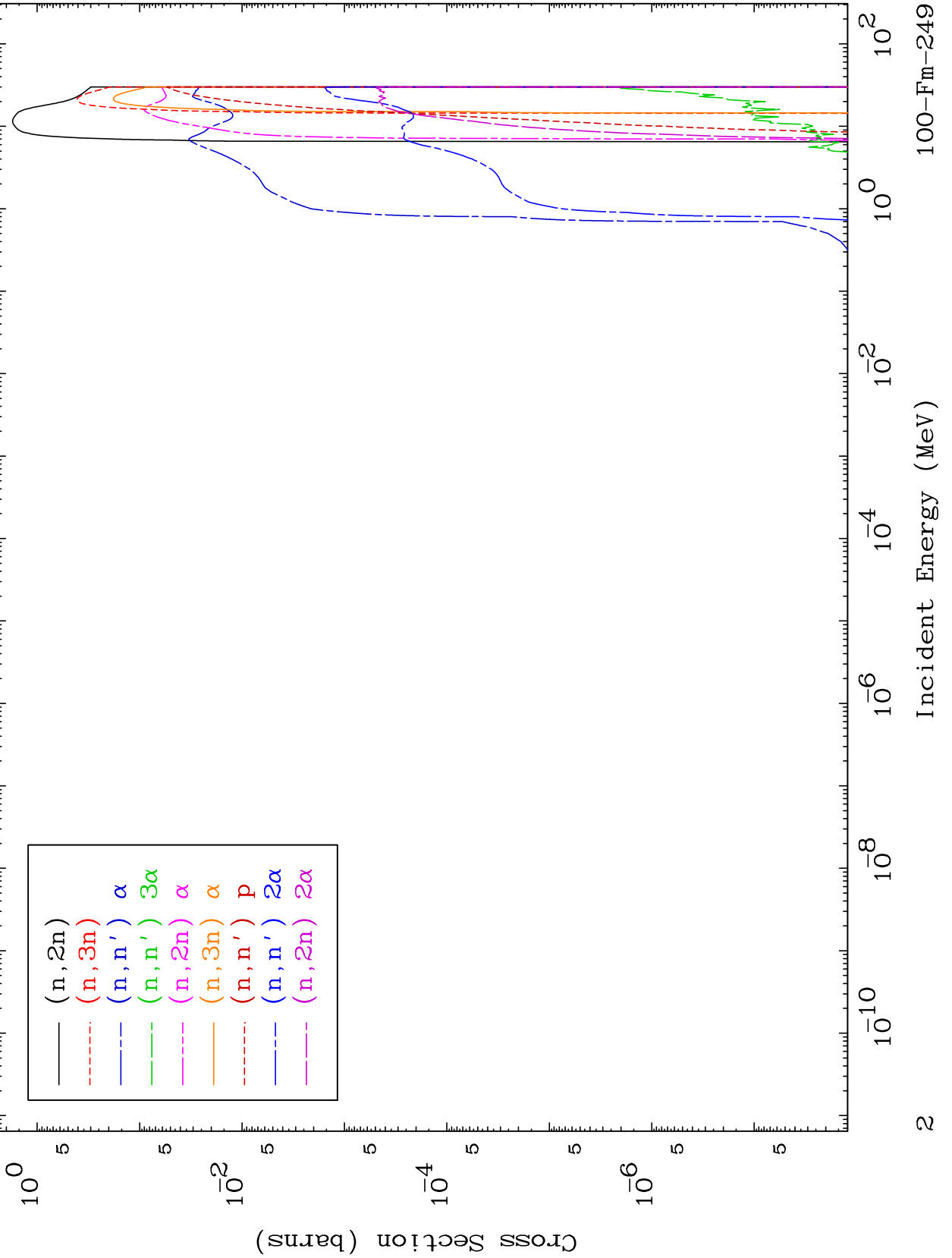
100-Fm-249



MAT 9930

Neutron Production  
293 Kelvin Cross Sections

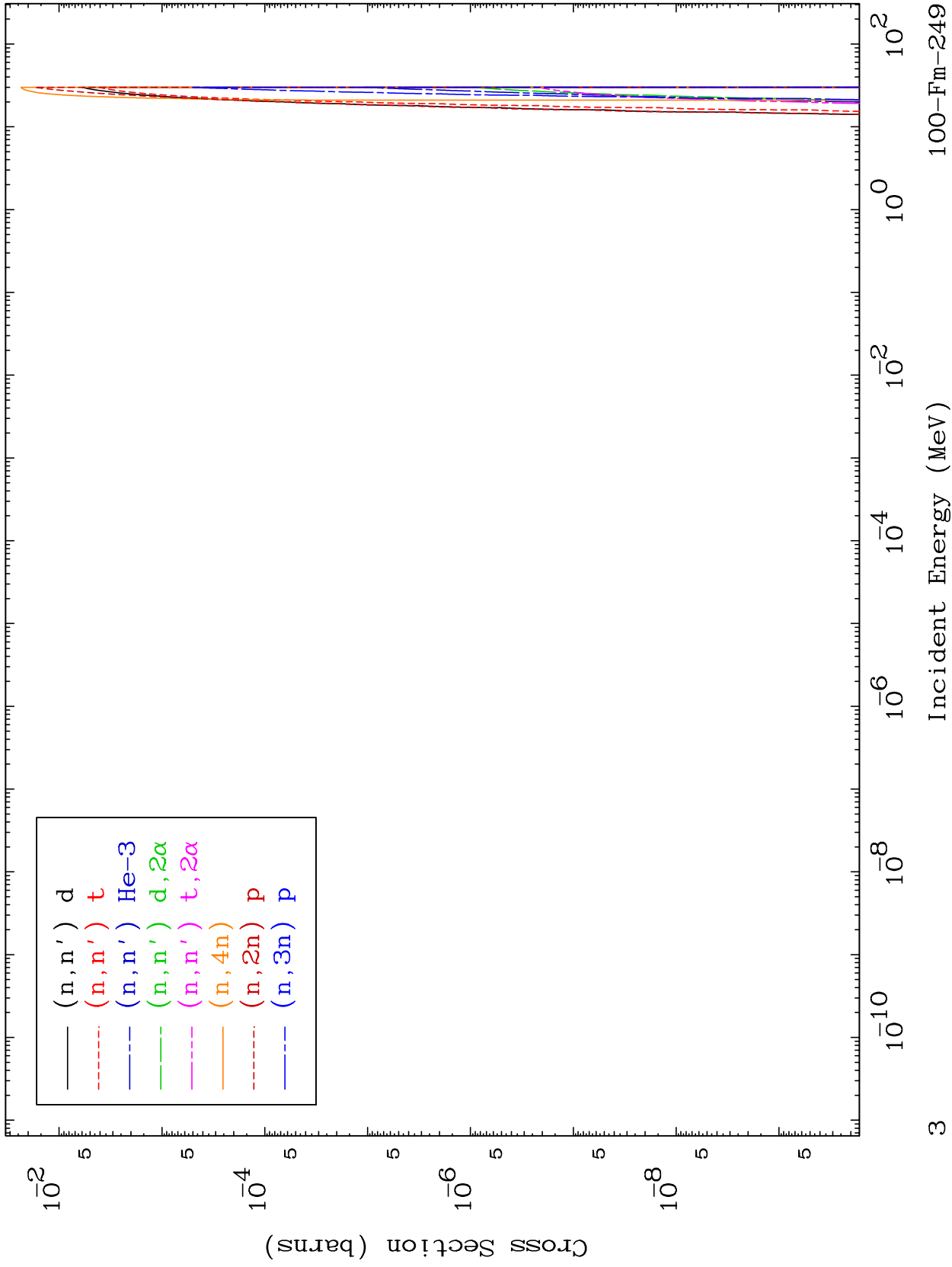
100-Fm-249



MAT 9930

Neutron Production  
293 Kelvin Cross Sections

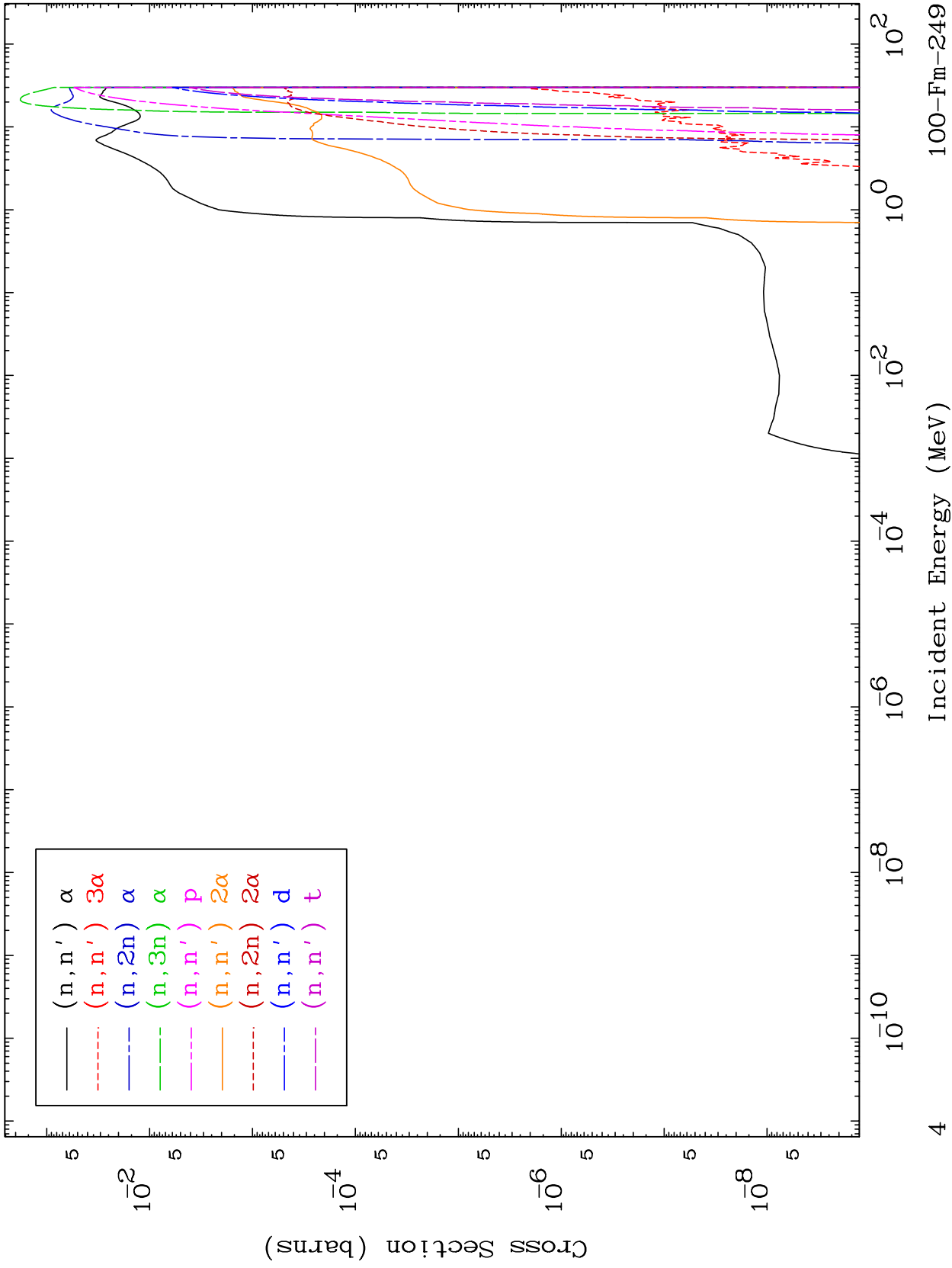
100-Fm-249



MAT 9930

Charged Particle  
293 Kelvin Cross Sections

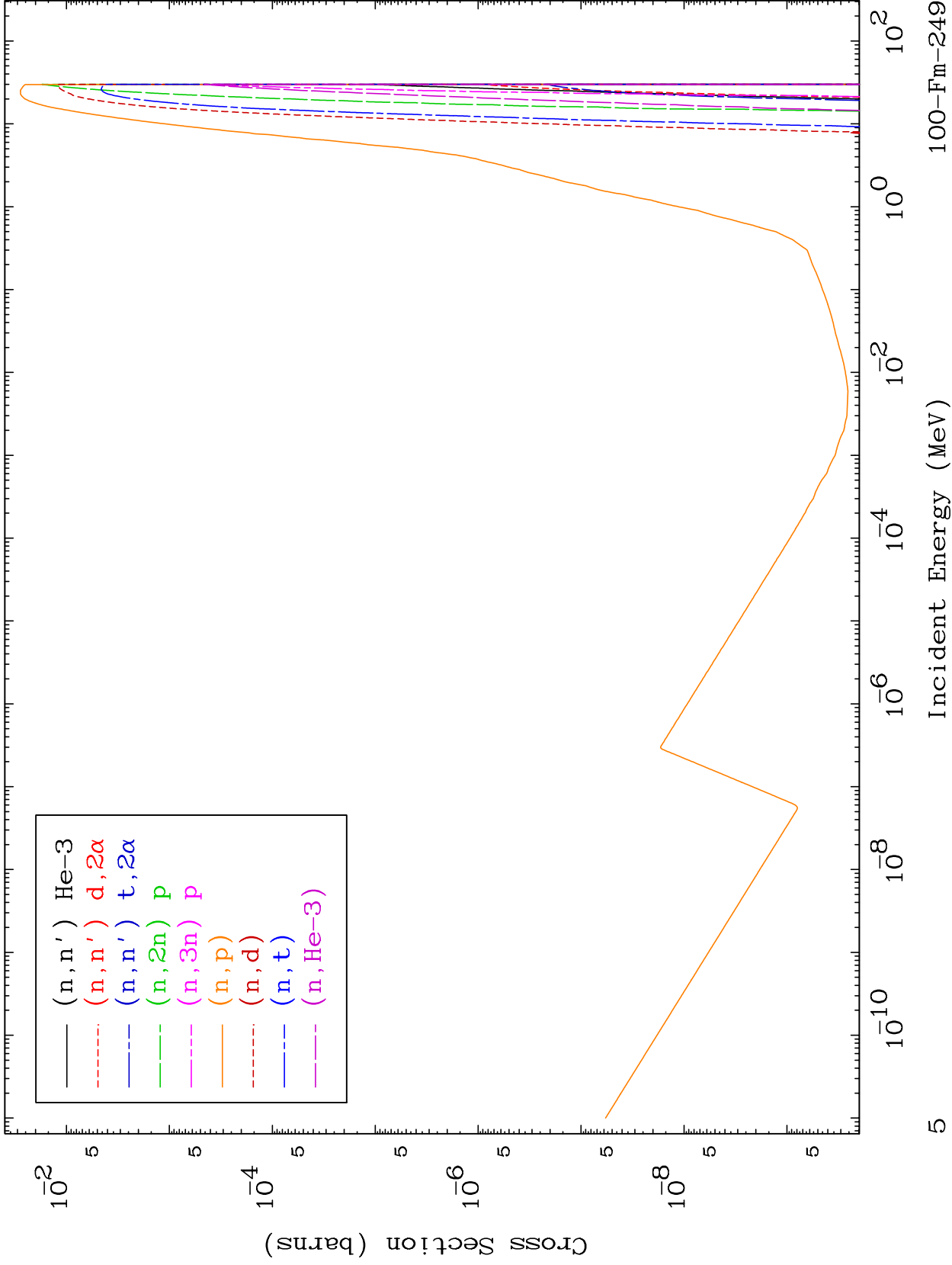
100-Fm-249



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Charged Particle  
293 Kelvin Cross Sections

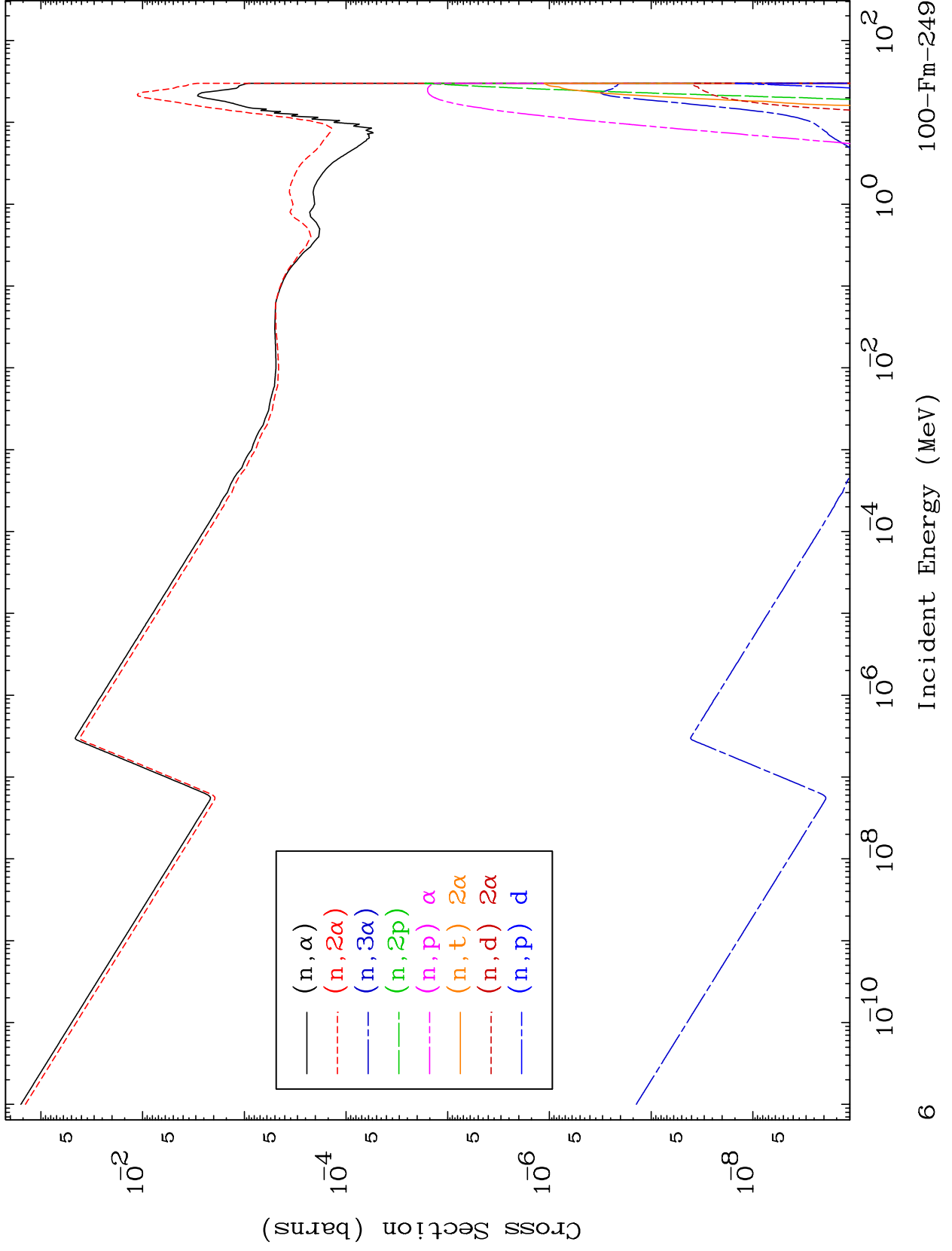
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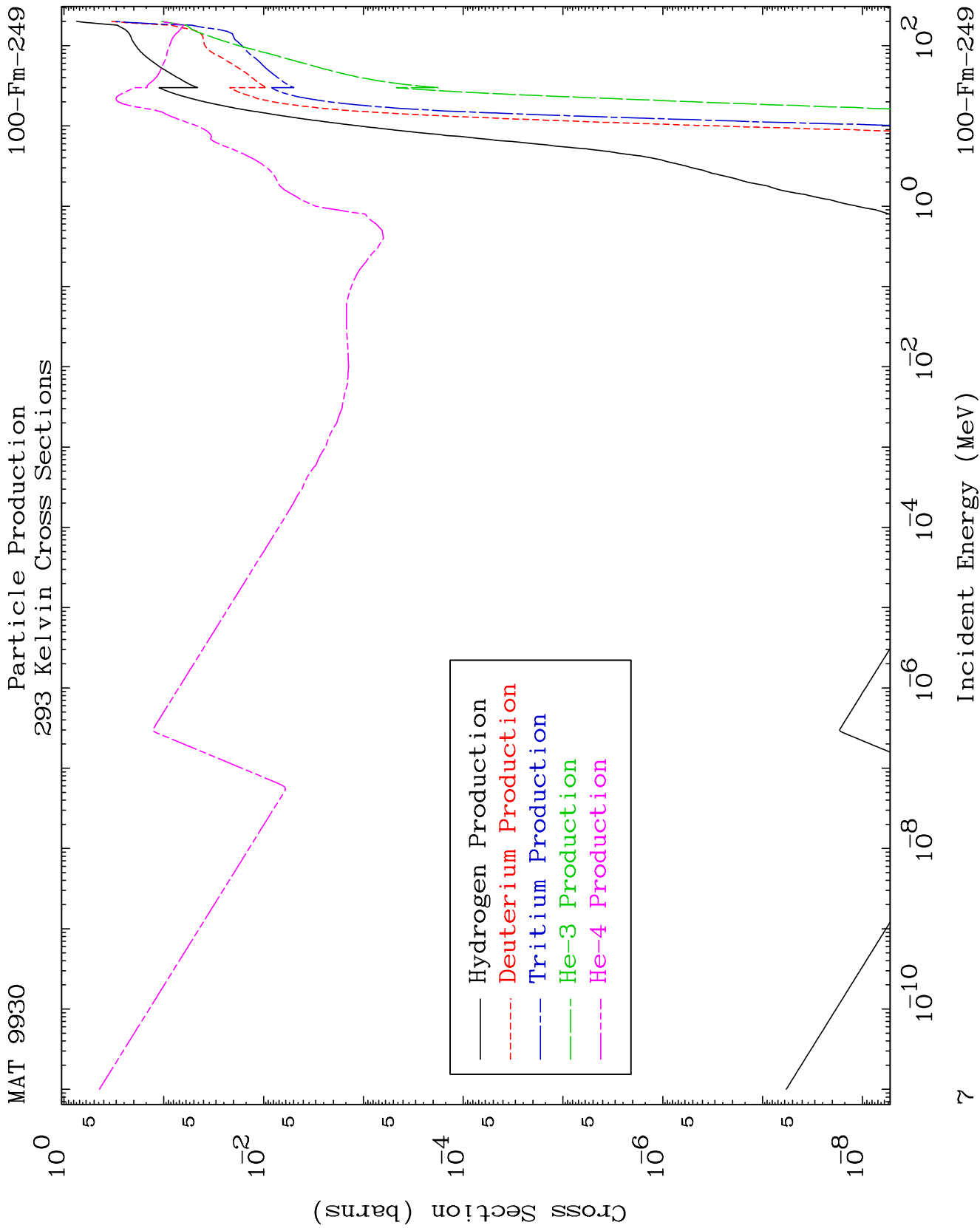


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Charged Particle  
293 Kelvin Cross Sections

100-Fm-249



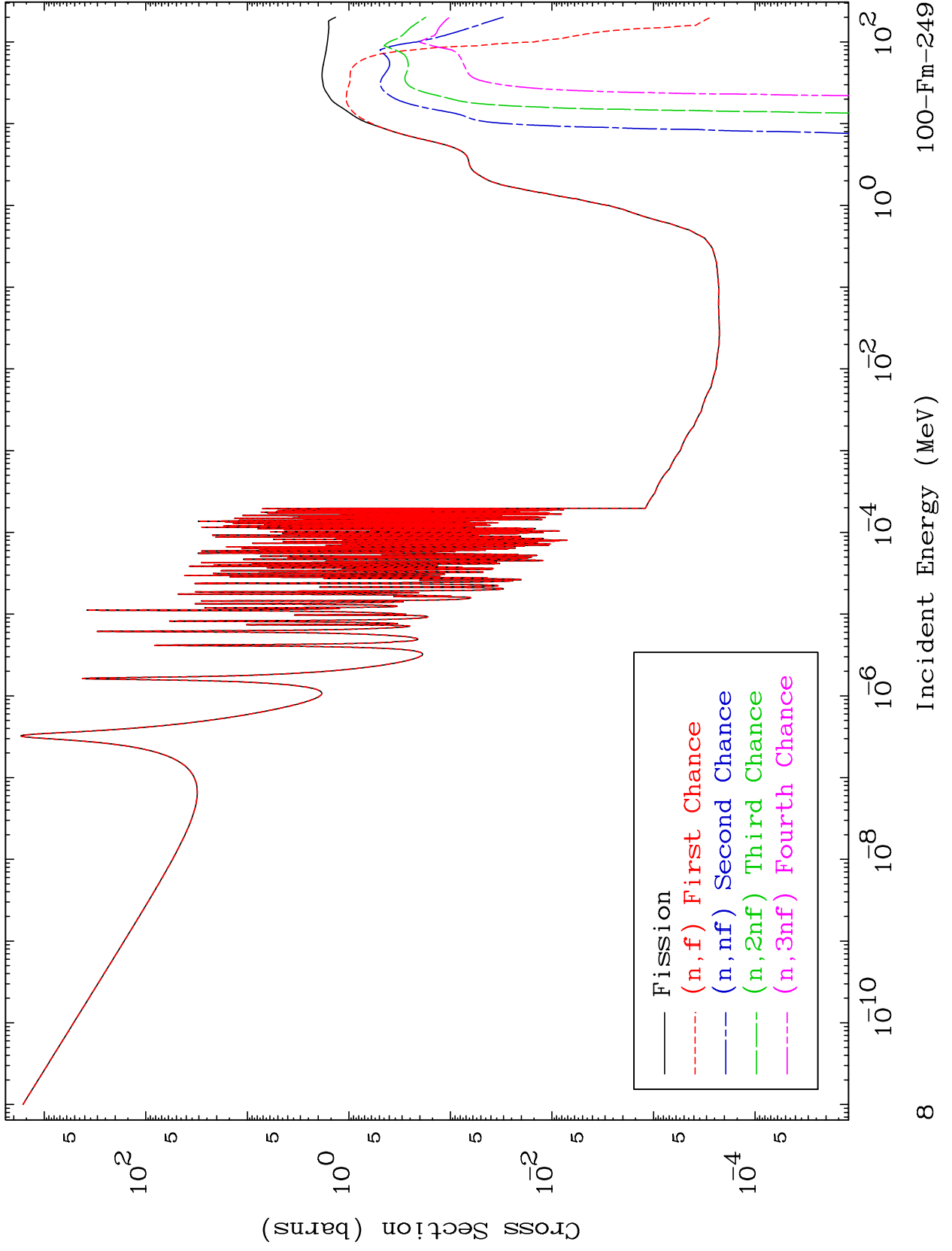




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Fission  
293 Kelvin Cross Sections

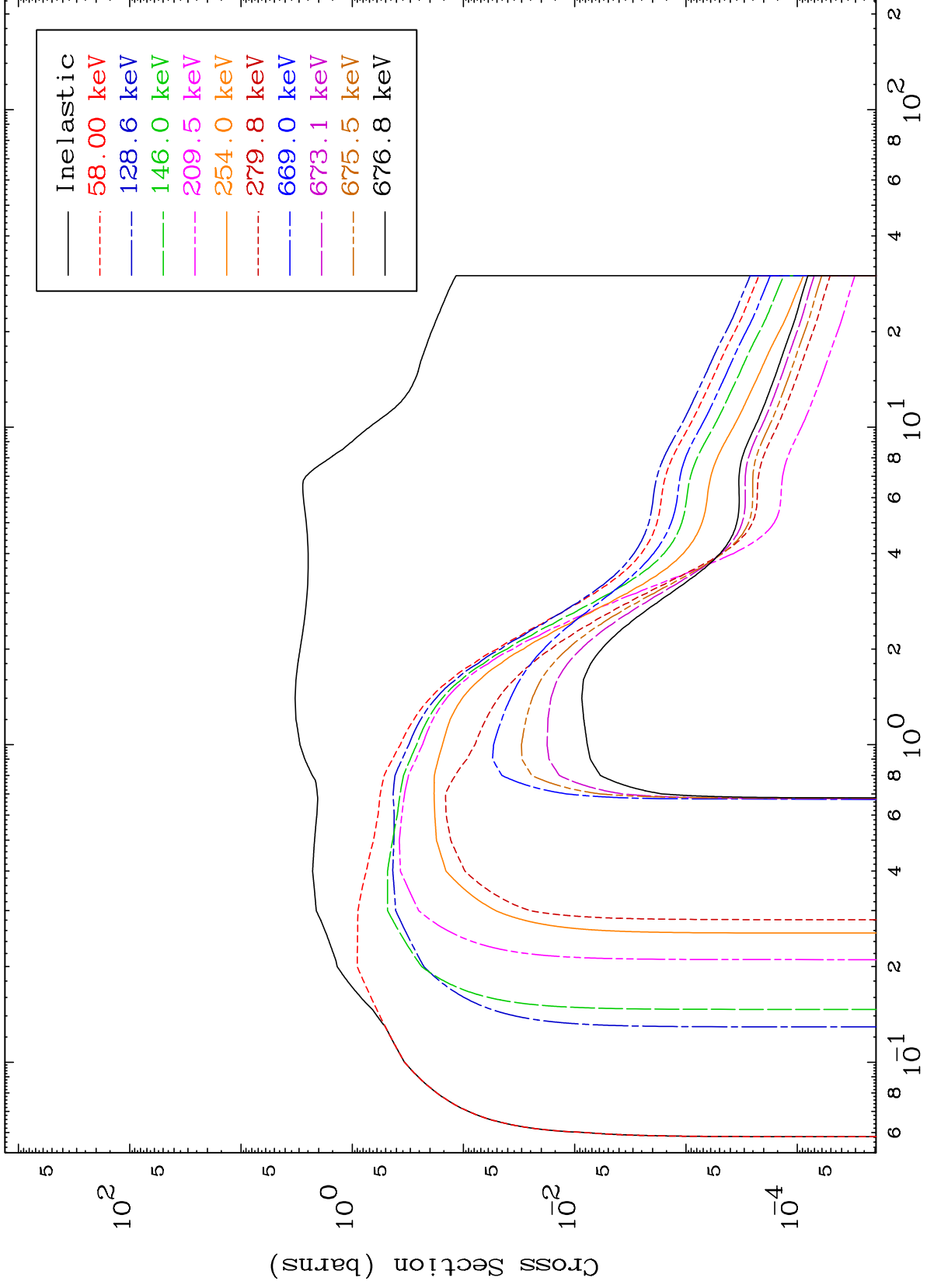
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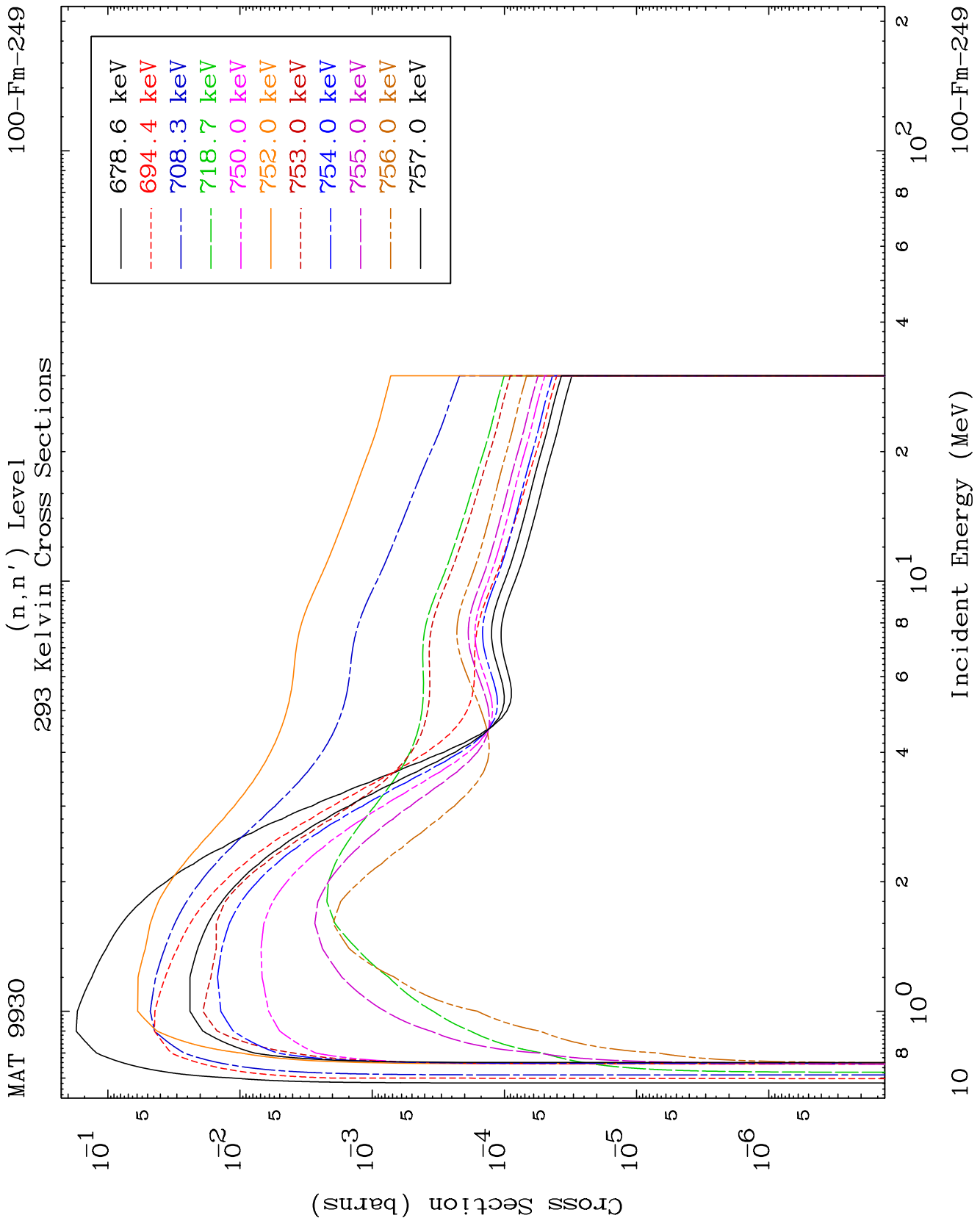


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(n,n') Level  
293 Kelvin Cross Sections

100-Fm-249

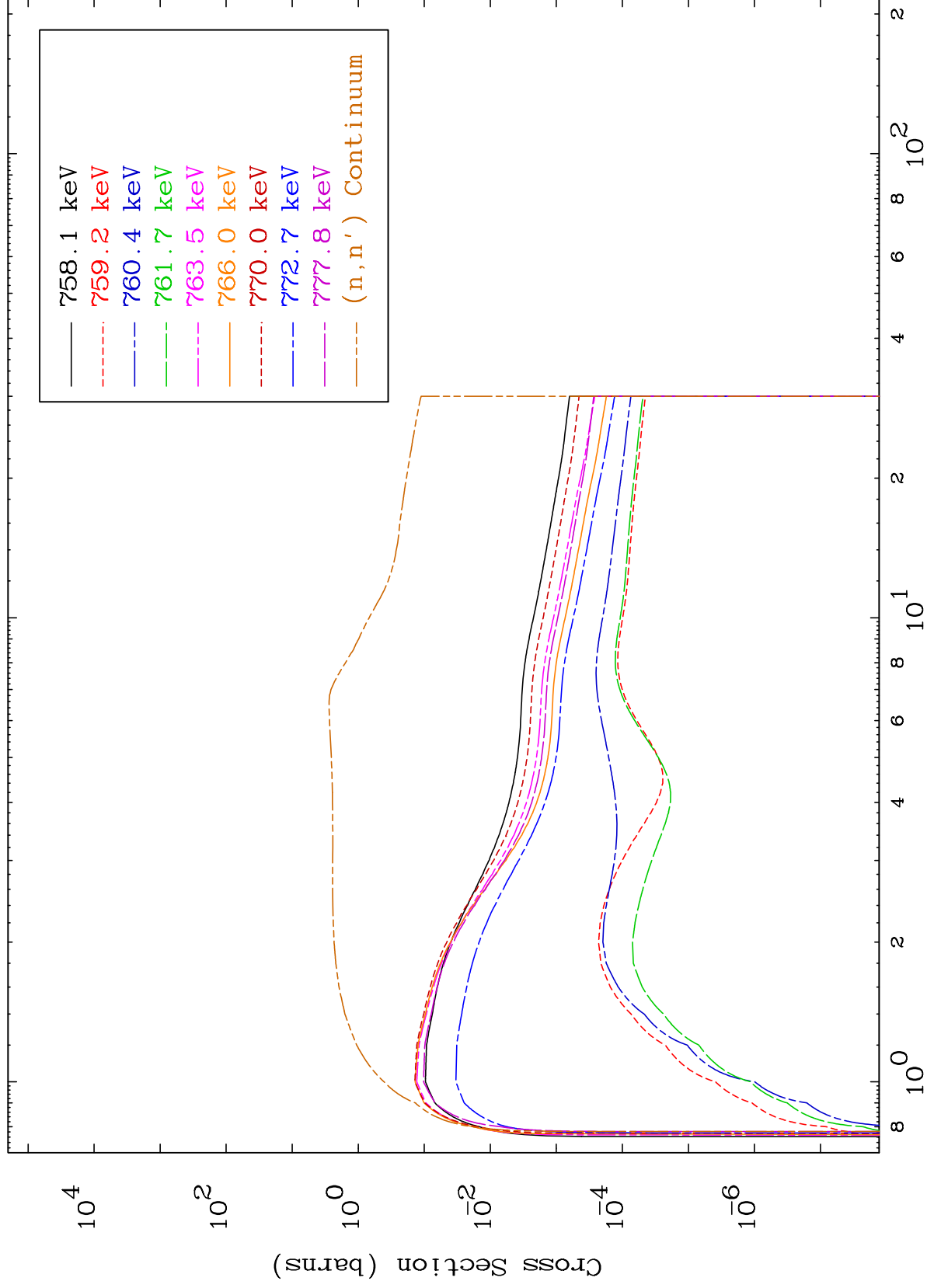




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(n,n') Level  
293 Kelvin Cross Sections

100-Fm-249



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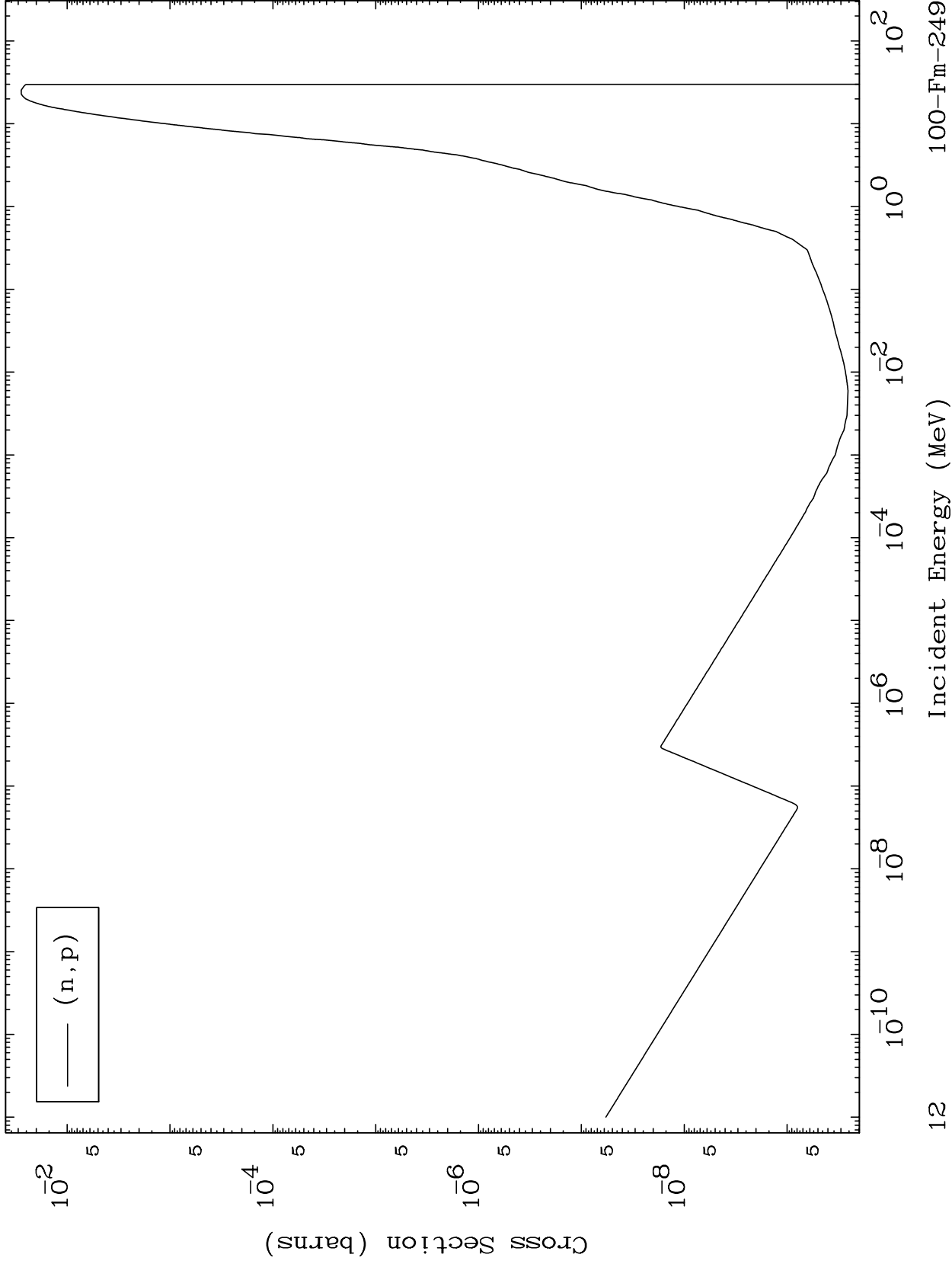
Incident Energy (MeV)

100-Fm-249

MAT 9930

(n,p) Levels  
293 Kelvin Cross Sections

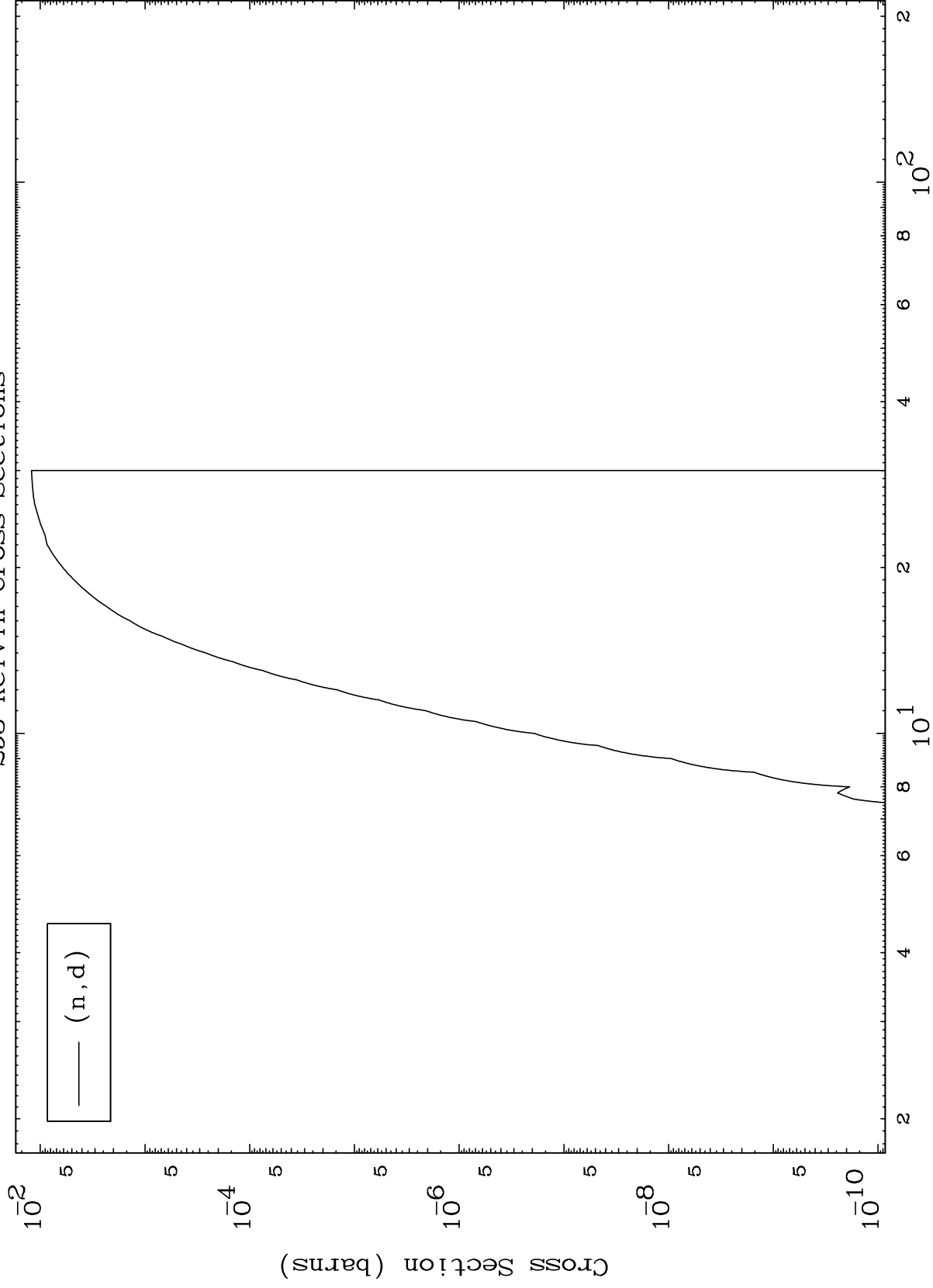
100-Fm-249



MAT 9930

(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-249



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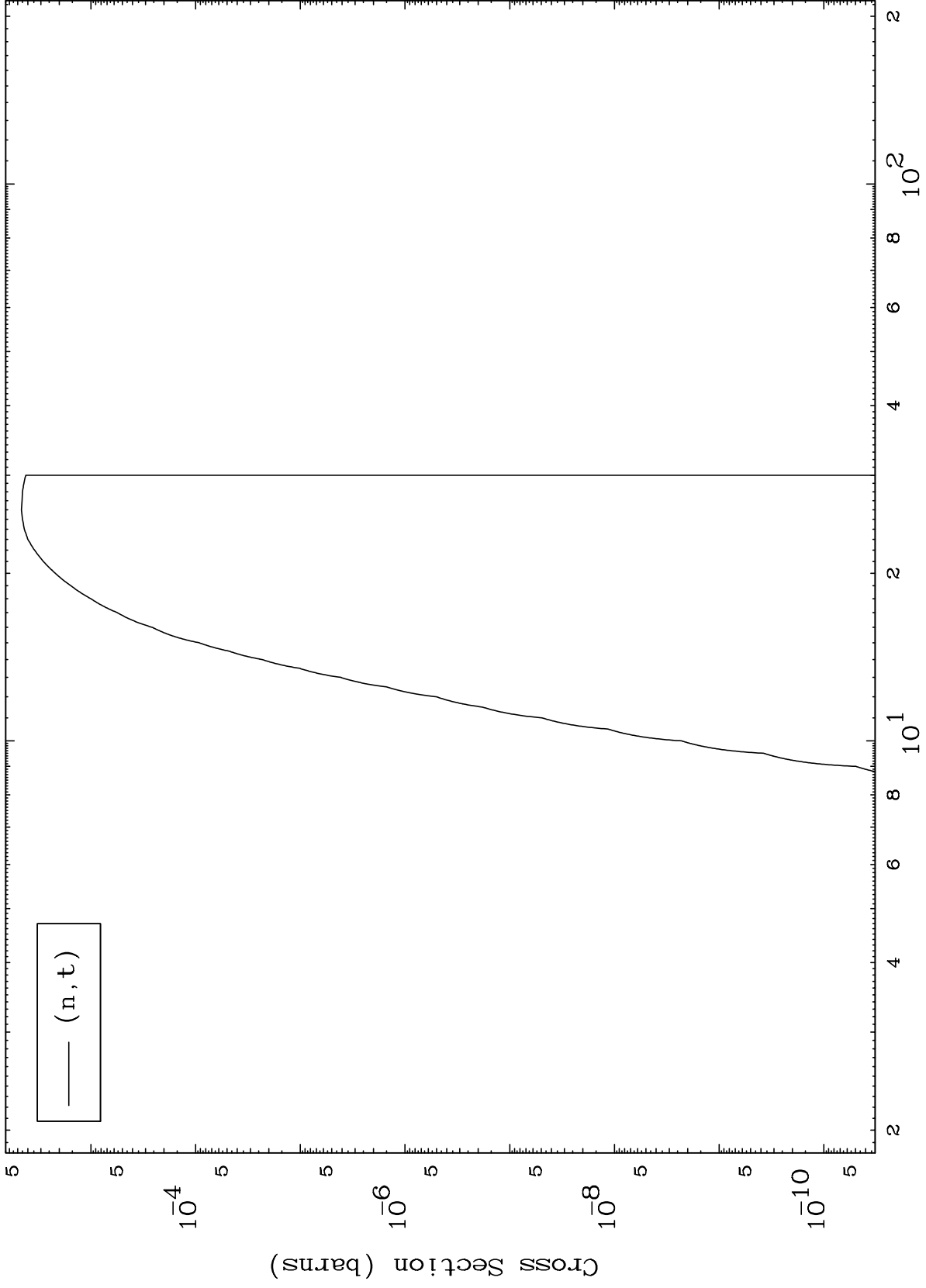
Incident Energy (MeV)

100-Fm-249

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(n,t) Levels  
293 Kelvin Cross Sections

100-Fm-249



14

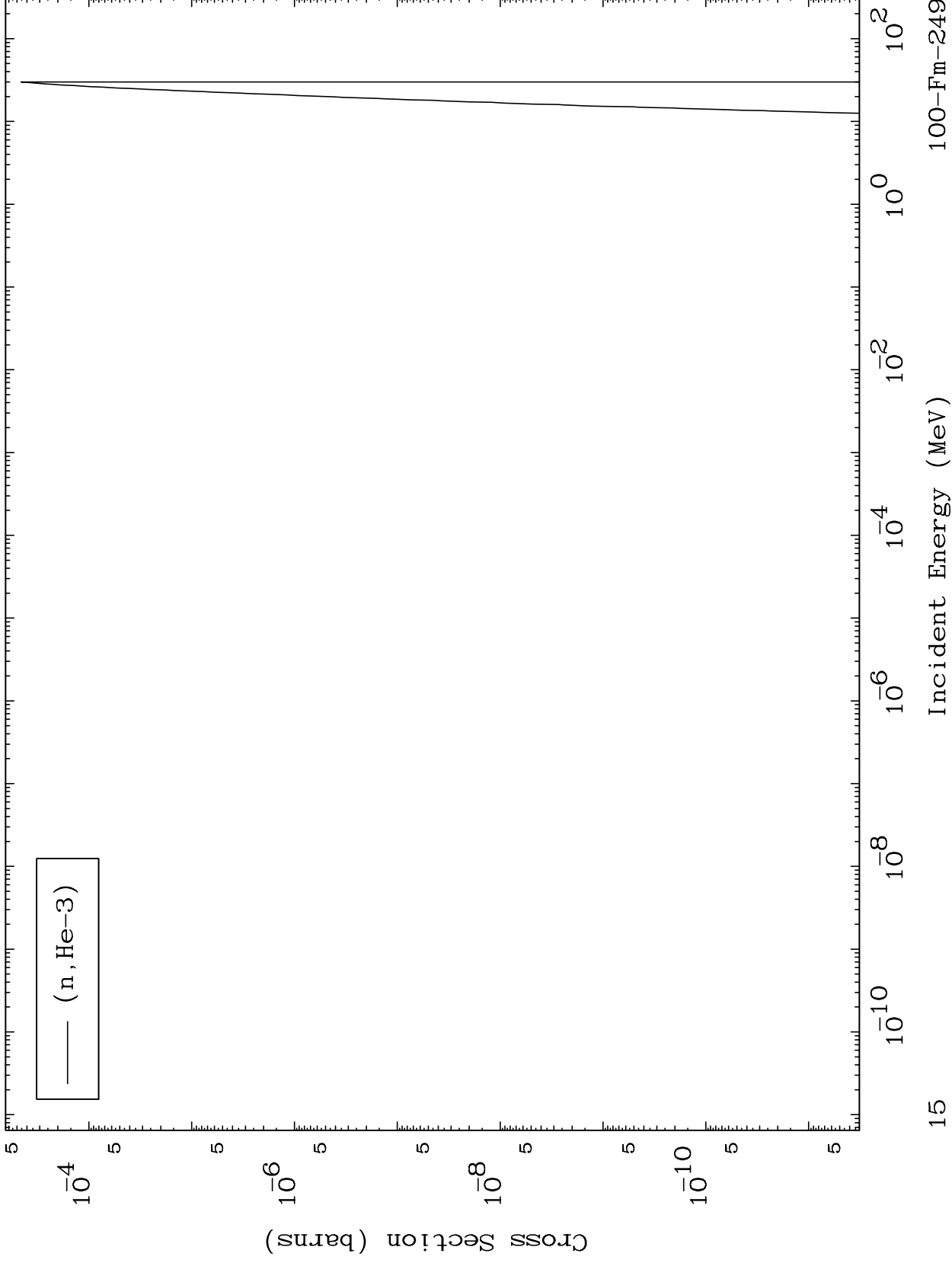
Incident Energy (MeV)

100-Fm-249

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(n,He3) Levels  
293 Kelvin Cross Sections

100-Fm-249

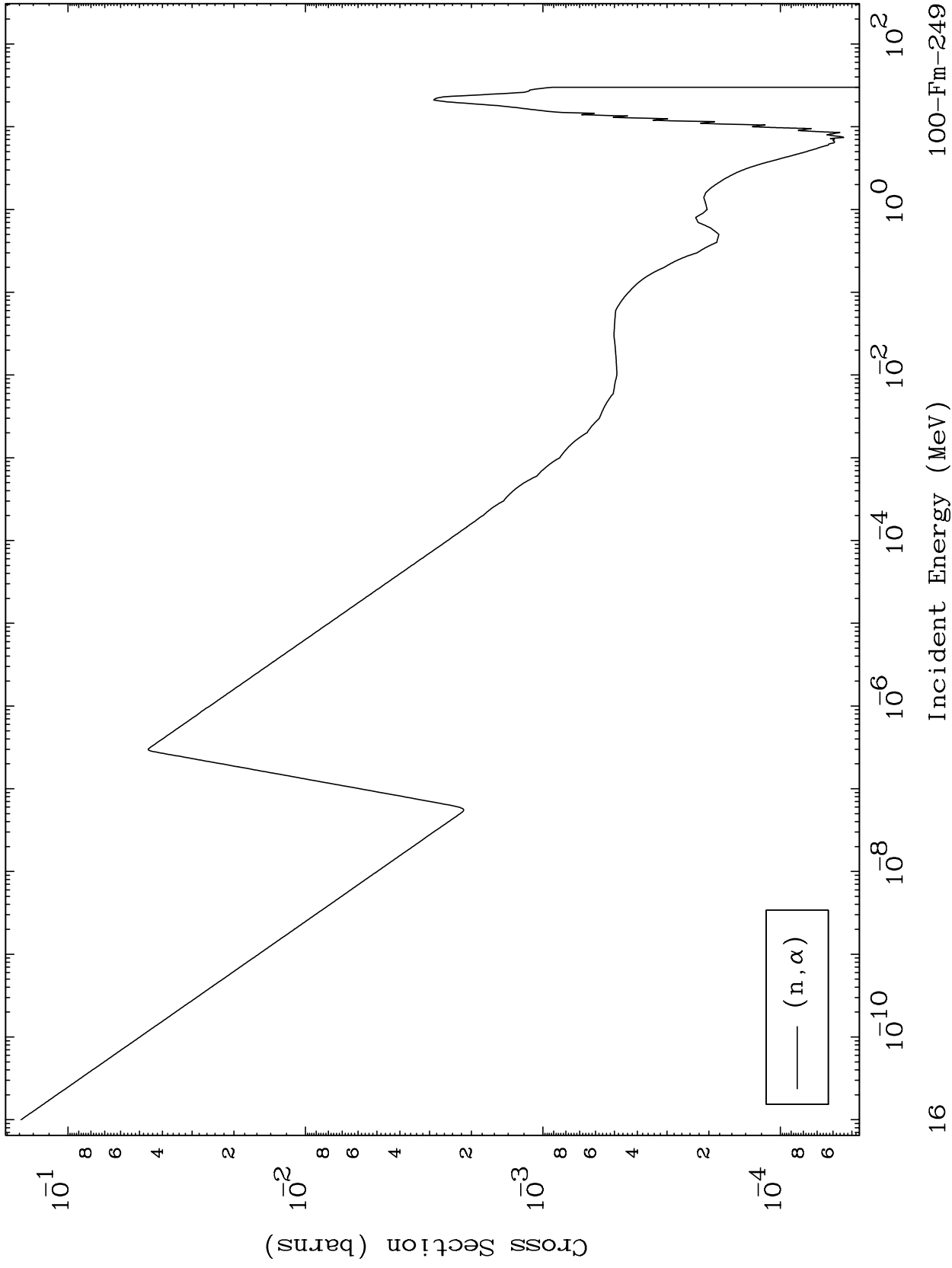


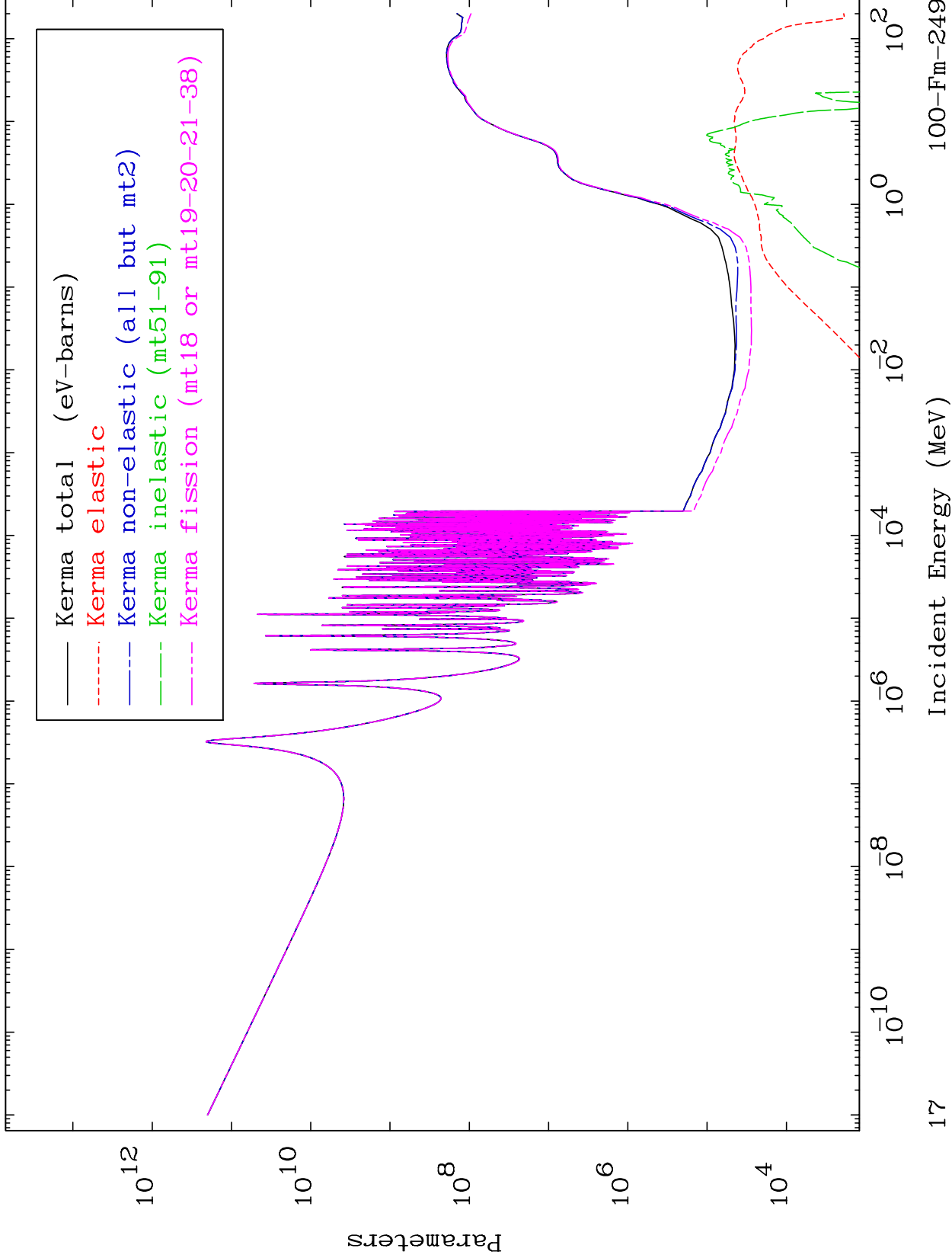


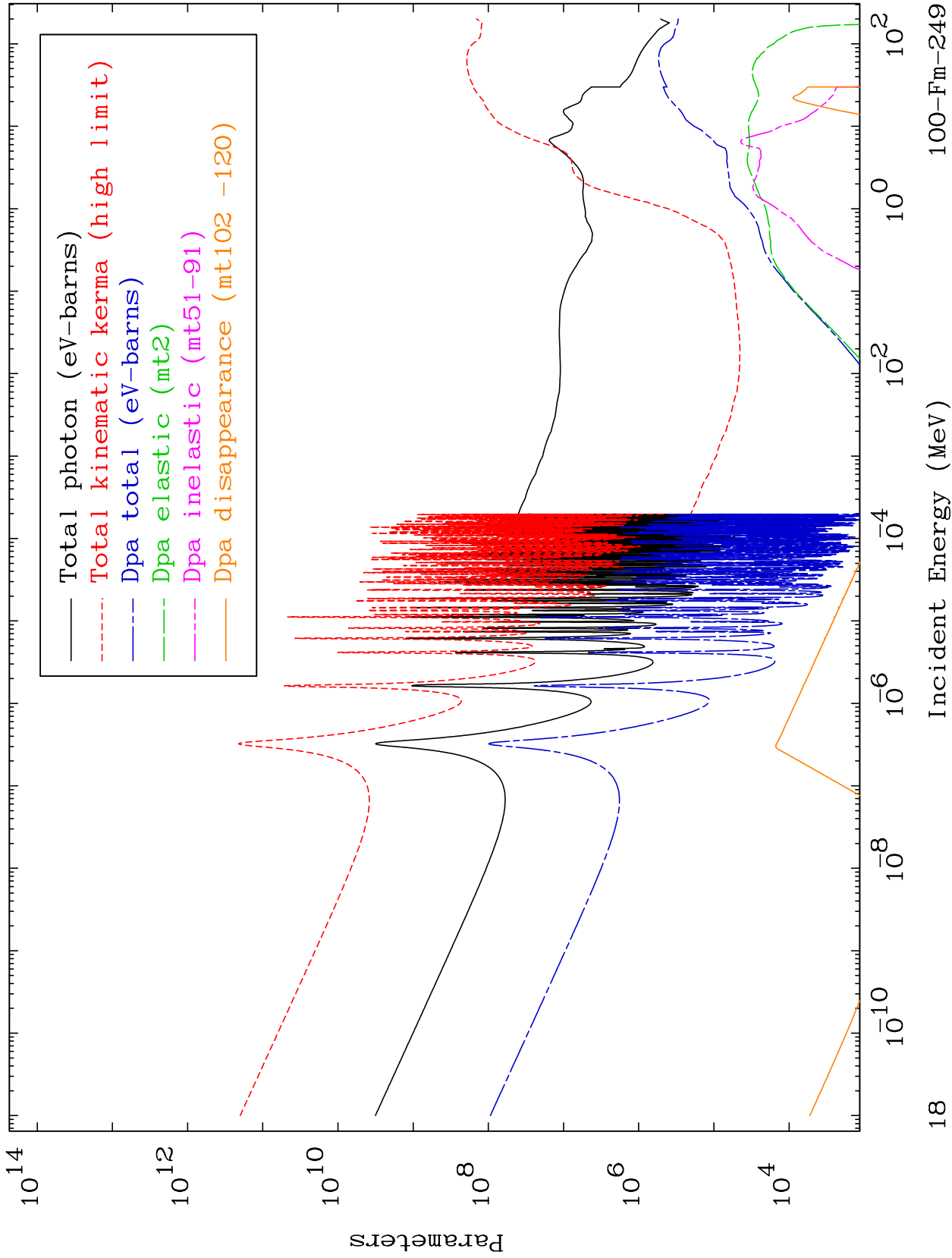
MAT 9930

(n,α) Levels  
293 Kelvin Cross Sections

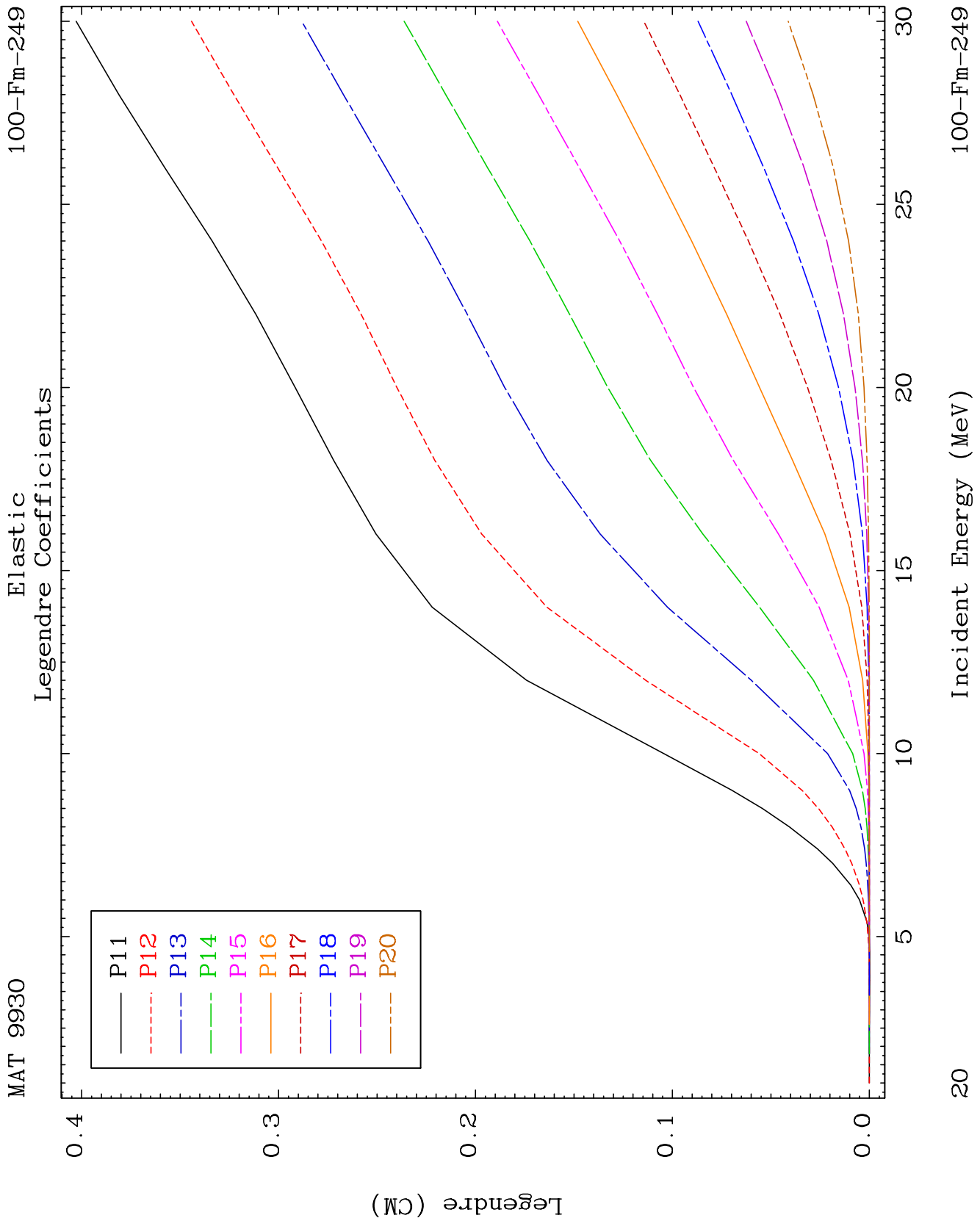
100-Fm-249







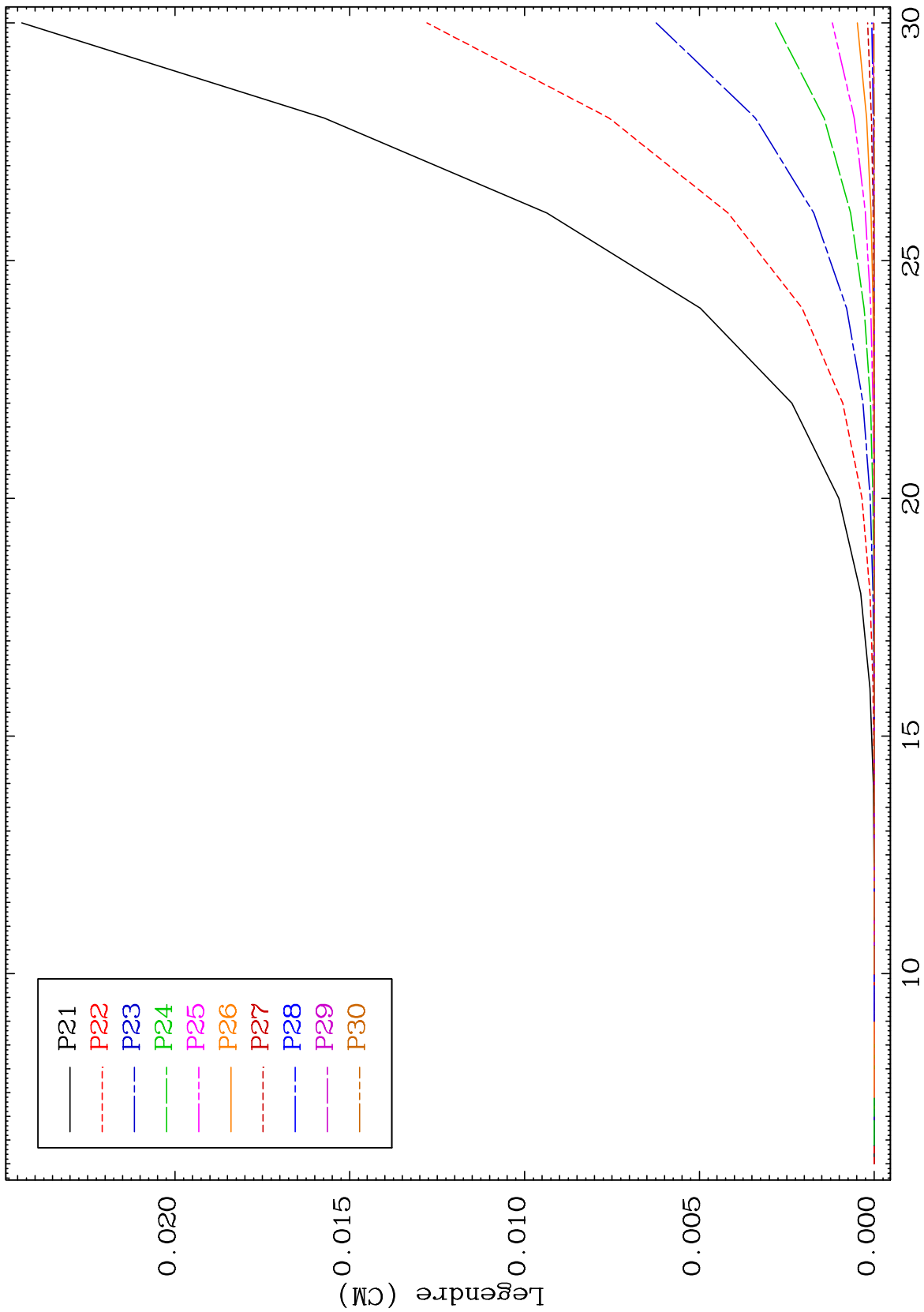




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Elastic Legendre Coefficients

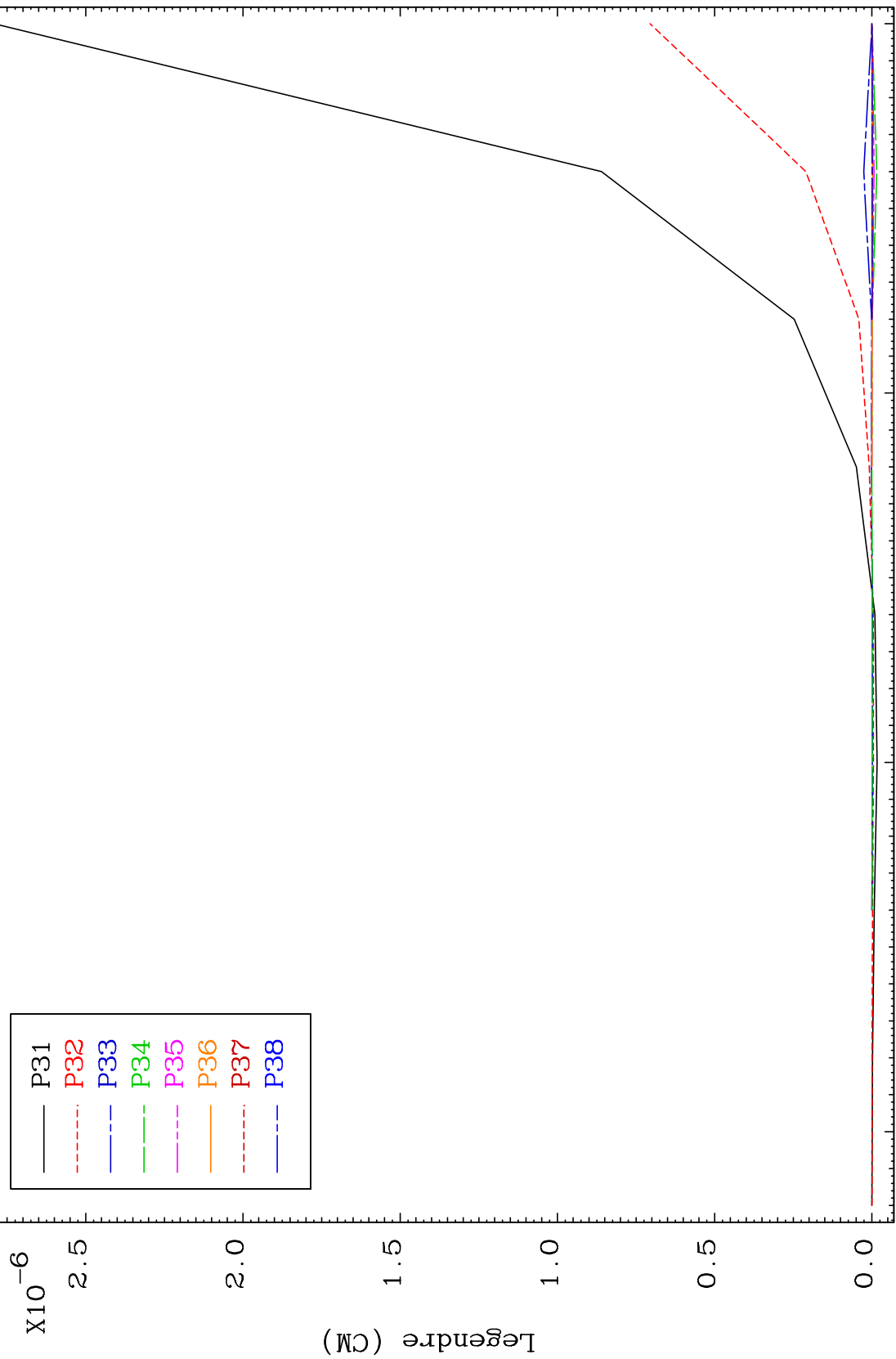
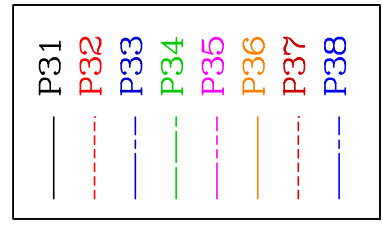
100-Fm-249



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Elastic Legendre Coefficients

100-Fm-249



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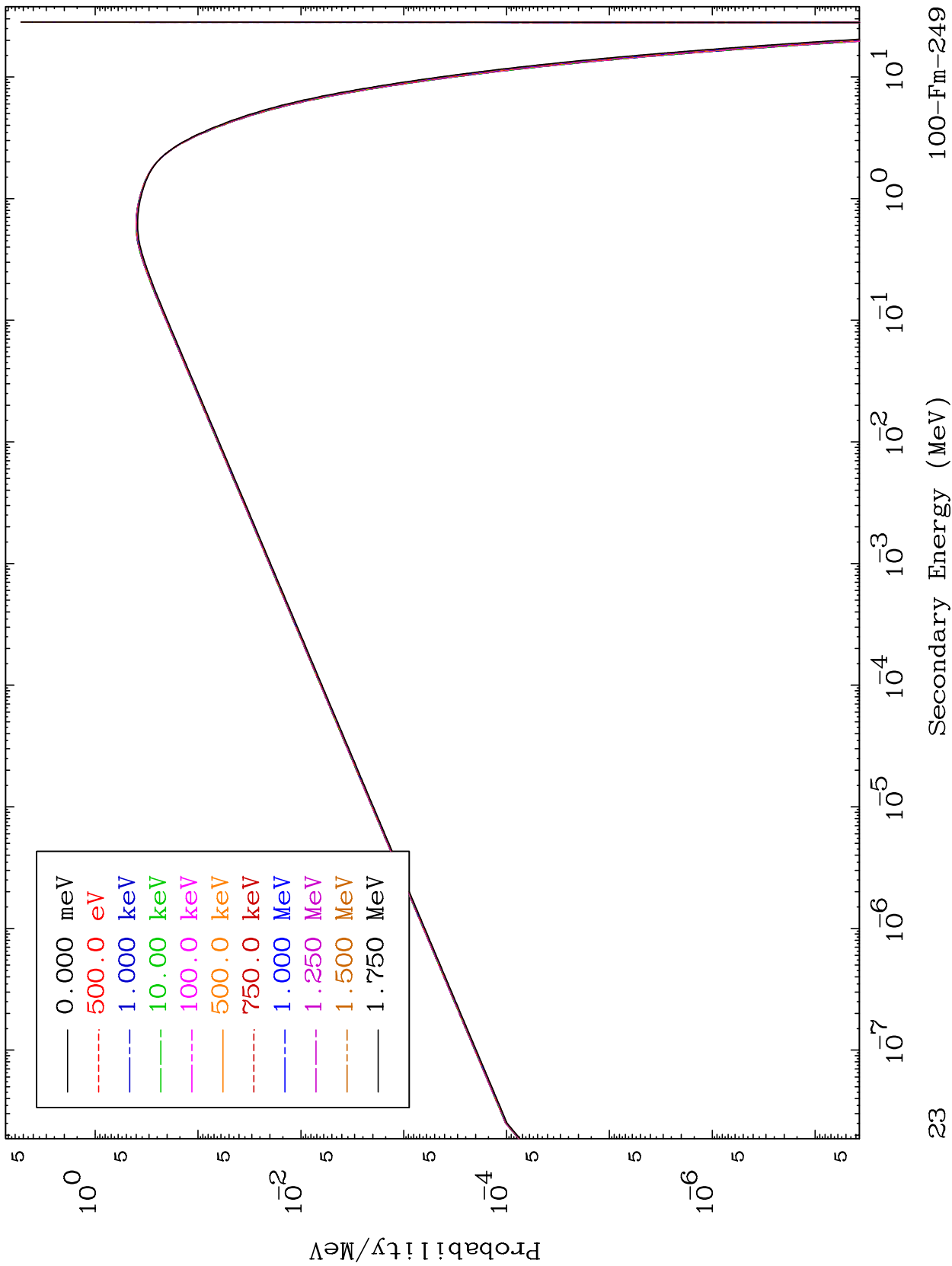
Incident Energy (MeV)

100-Fm-249

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### Fission Energy Distribution

100-Fm-249

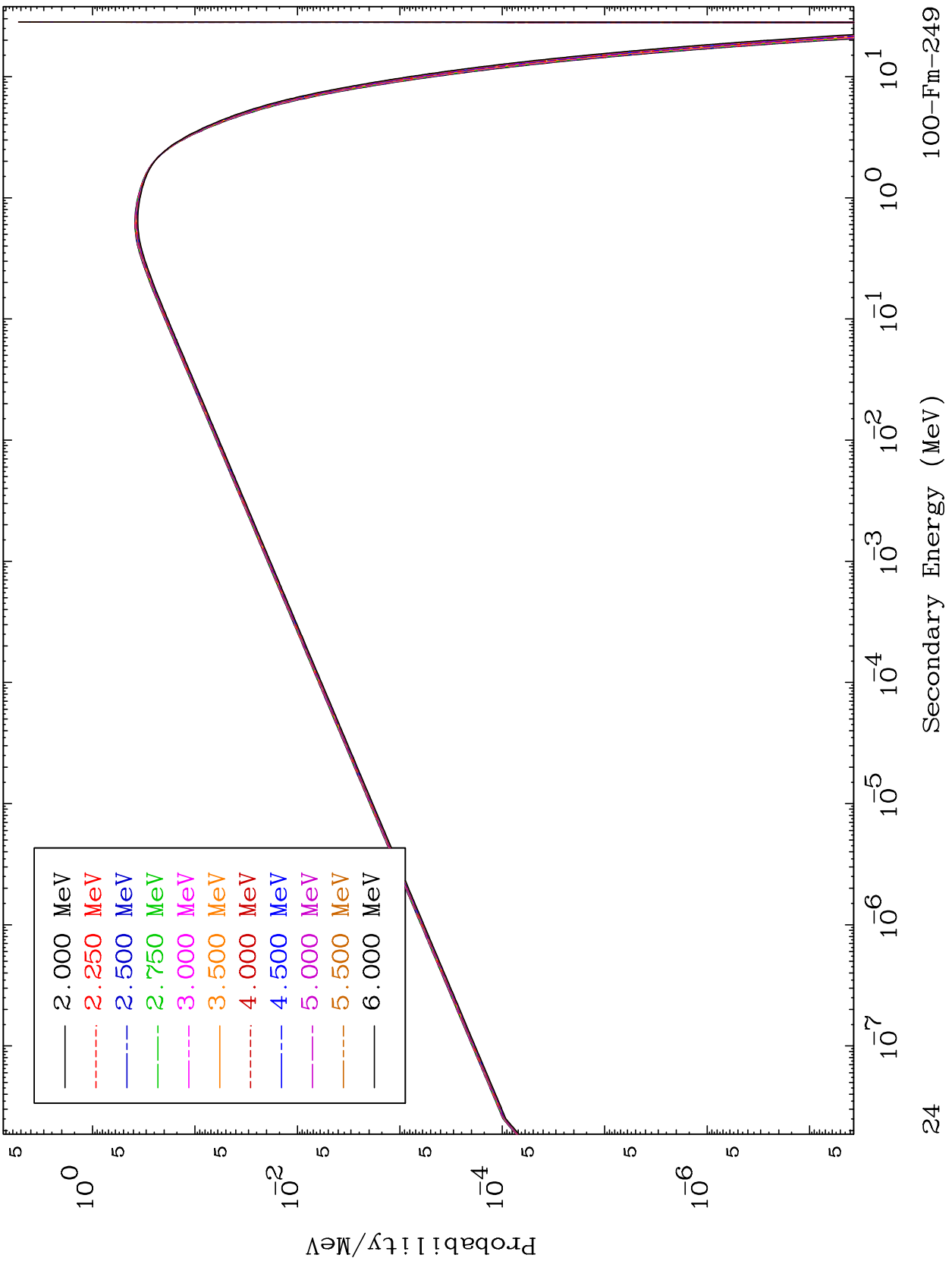




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### Fission Energy Distribution

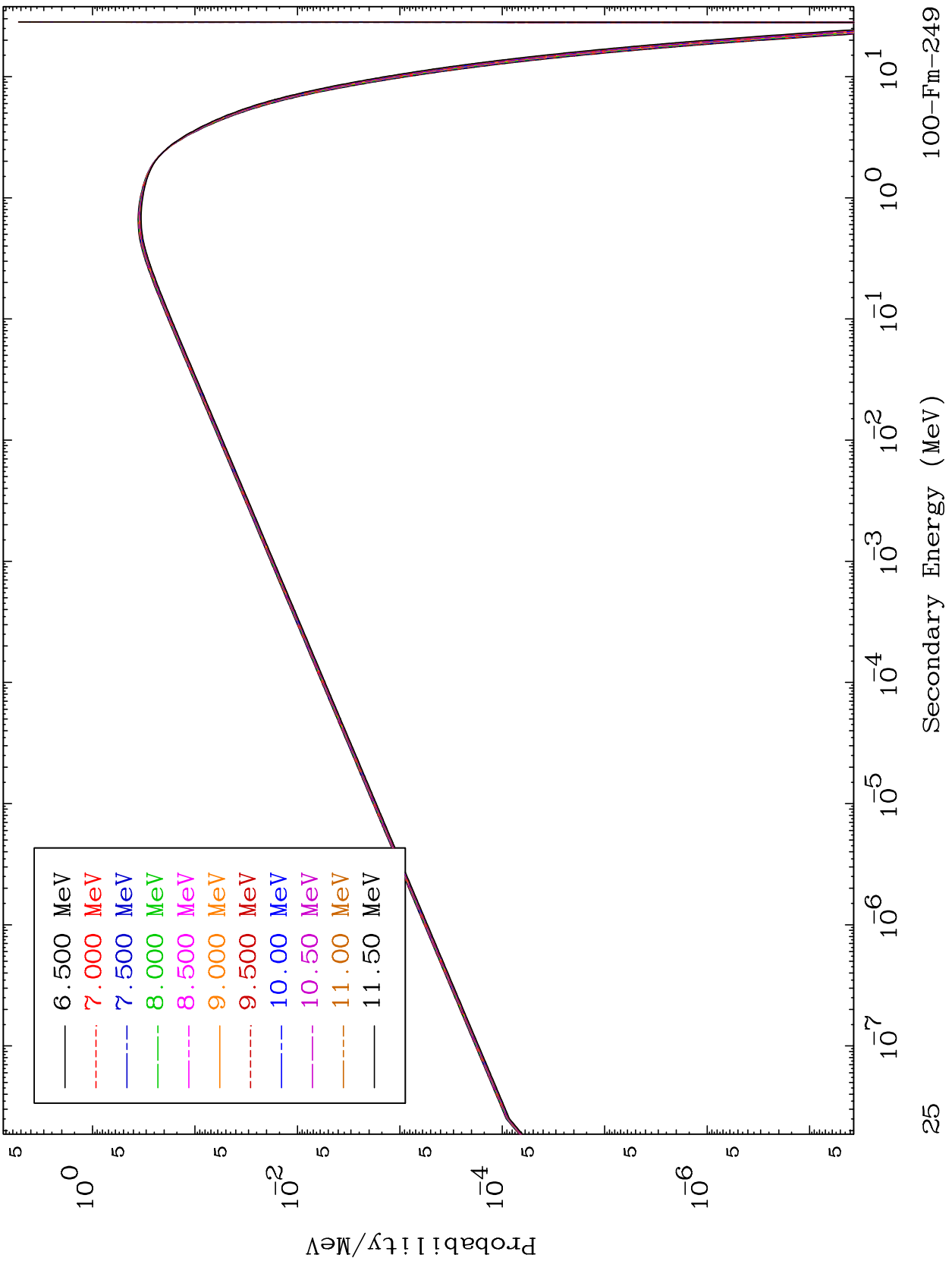
100-Fm-249



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### Fission Energy Distribution

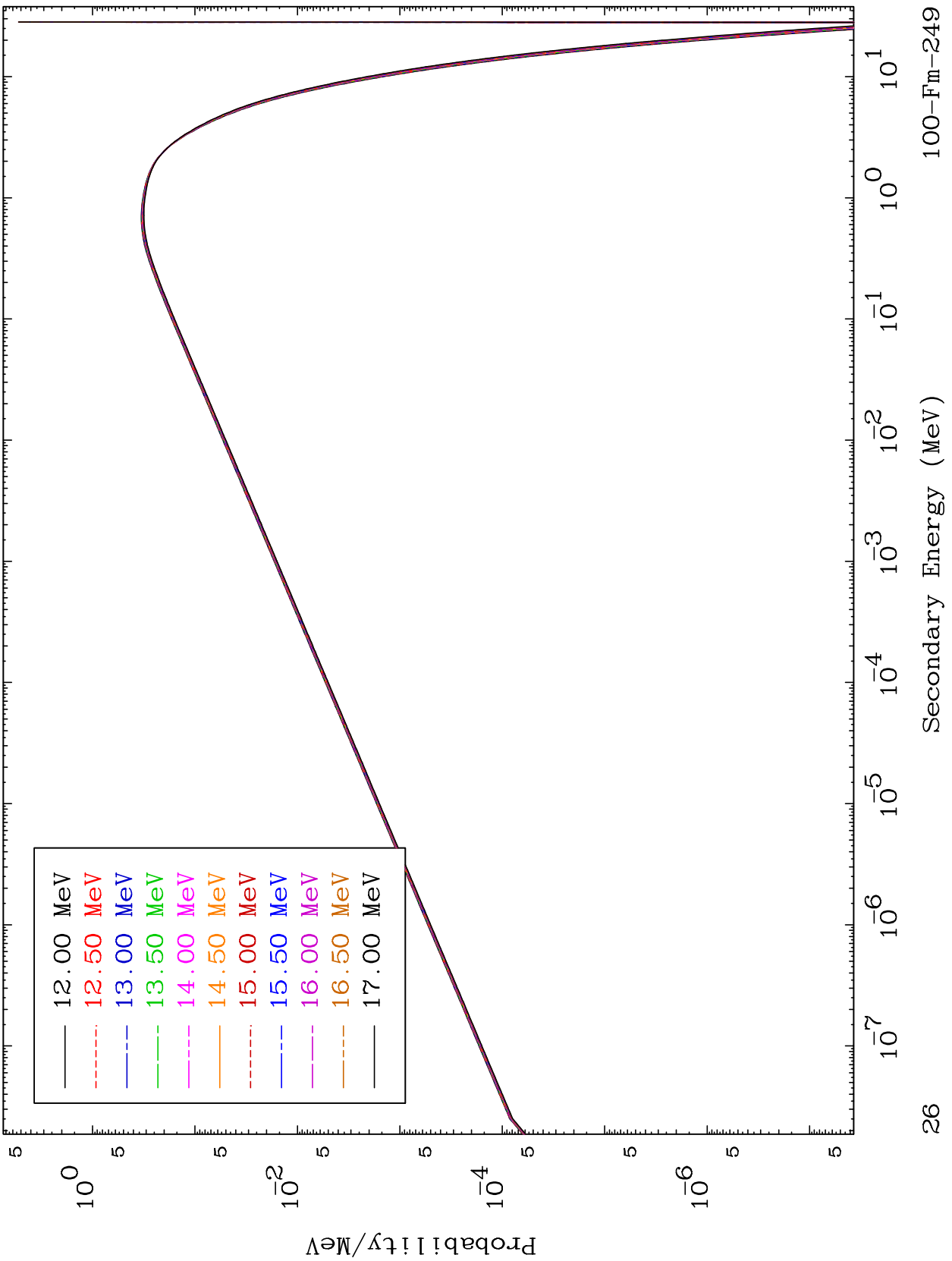
100-Fm-249



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### Fission Energy Distribution

100-Fm-249

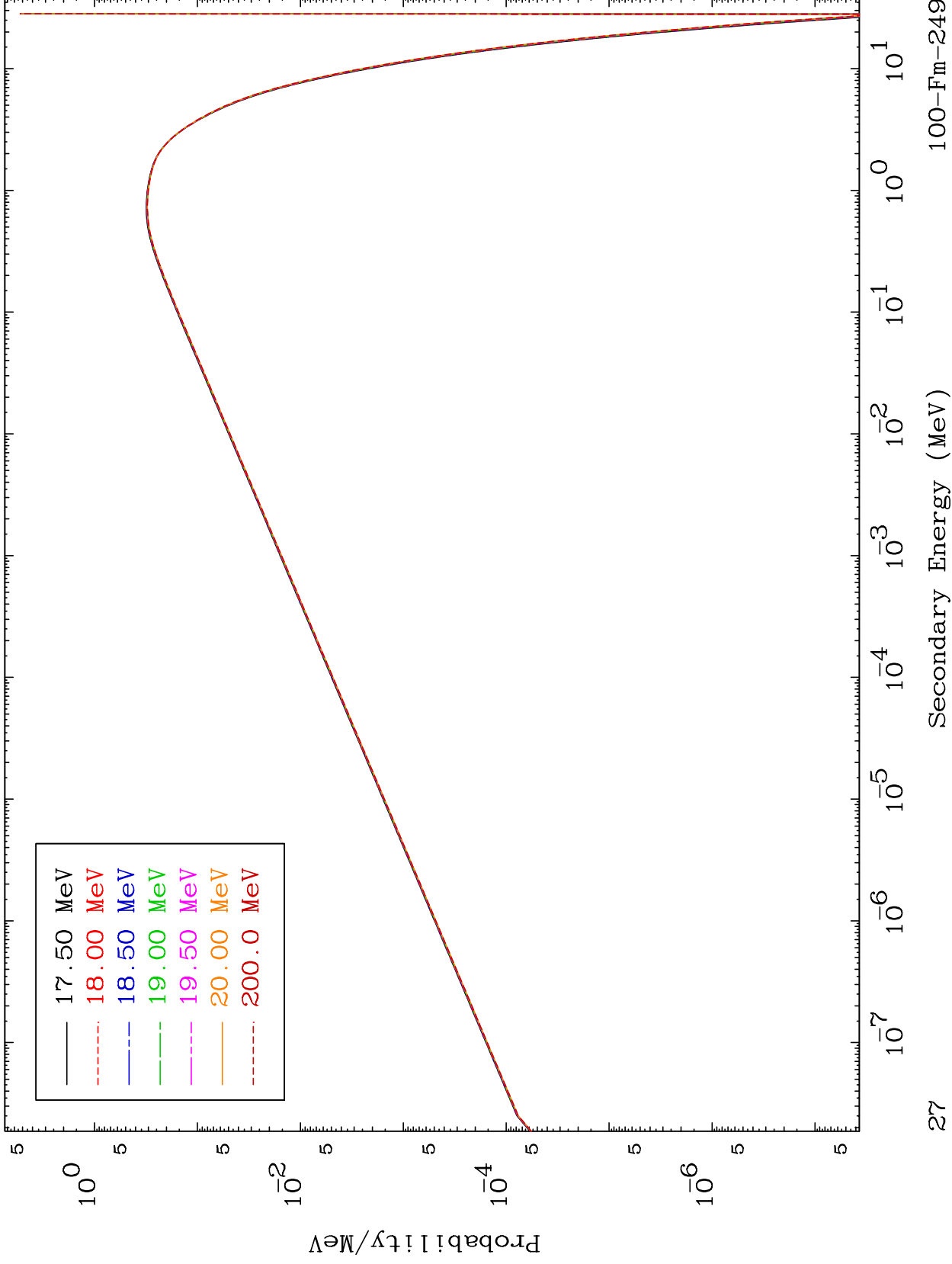


100-Fm-249

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Fission  
Energy Distribution

100-Fm-249

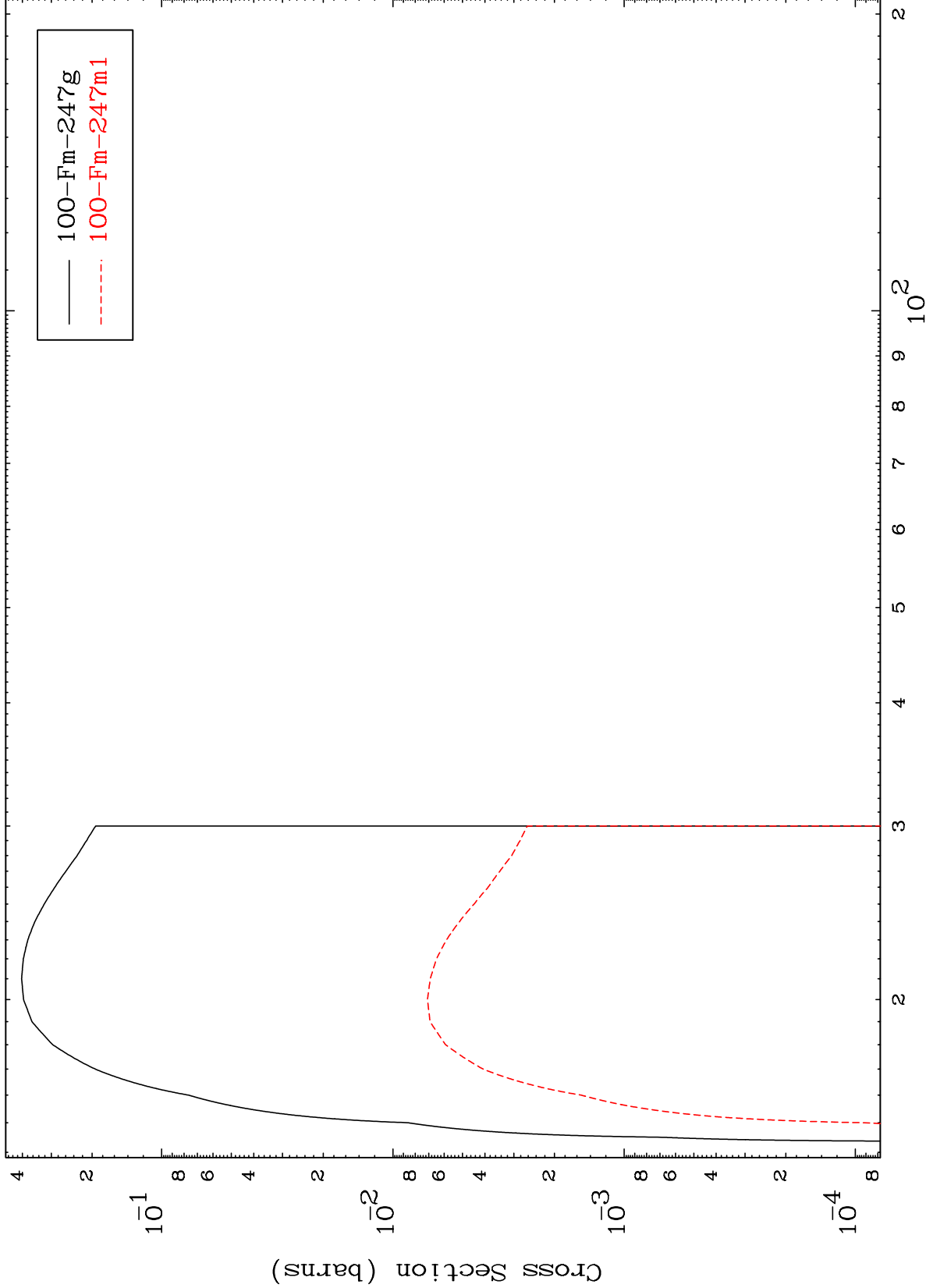


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(n,3n)

100-Fm-249

Radionuclide Production Cross Section



28

Incident Energy (MeV)

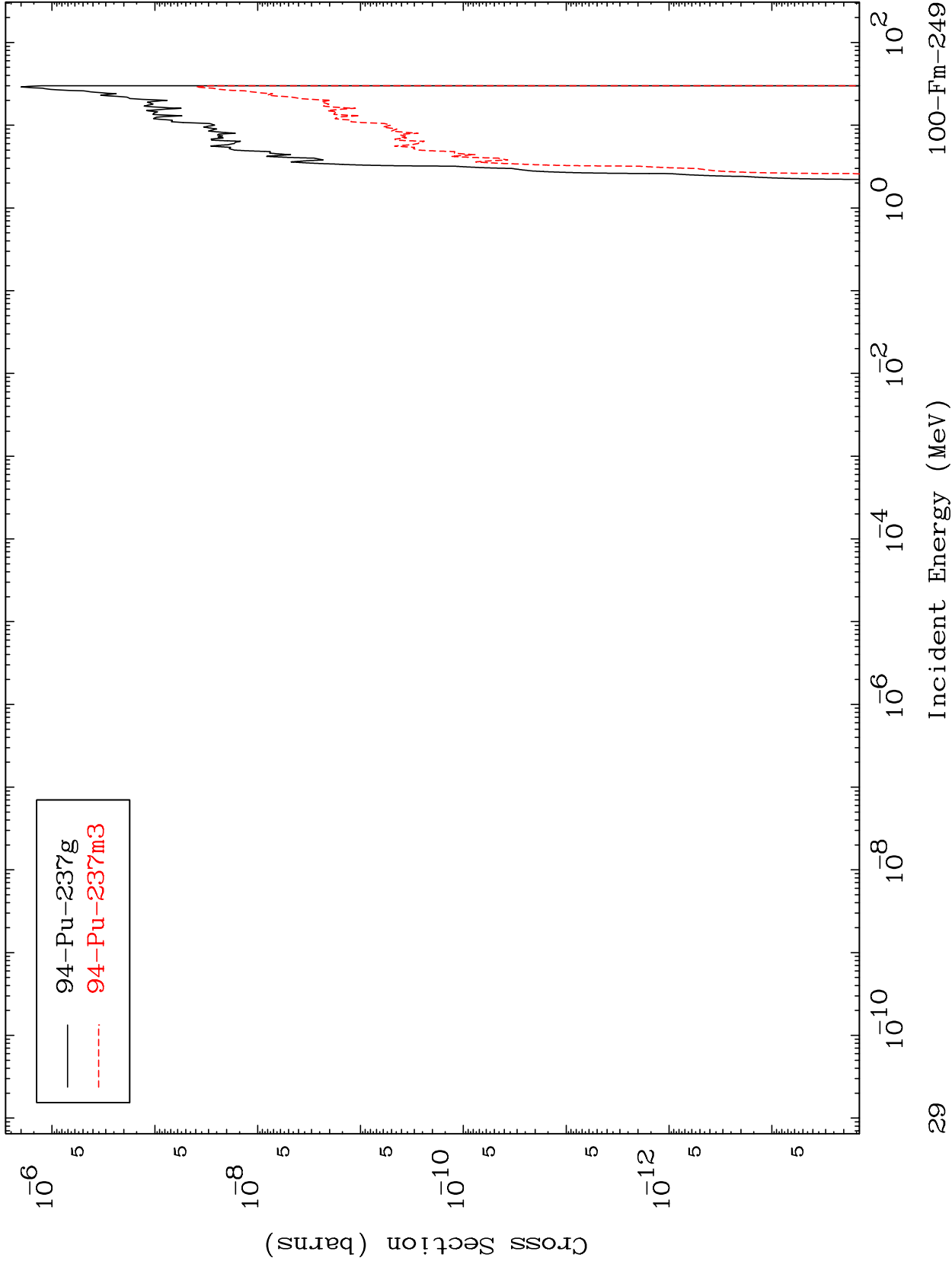
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(n,n') 3 $\alpha$

100-Fm-249

Radionuclide Production Cross Section



MAT 9930

100-Fm-249

Radionuclide Production Cross Section  
(n,  $\gamma$ )

