

Program EVALPLOT  
(Version 2015-2)

by

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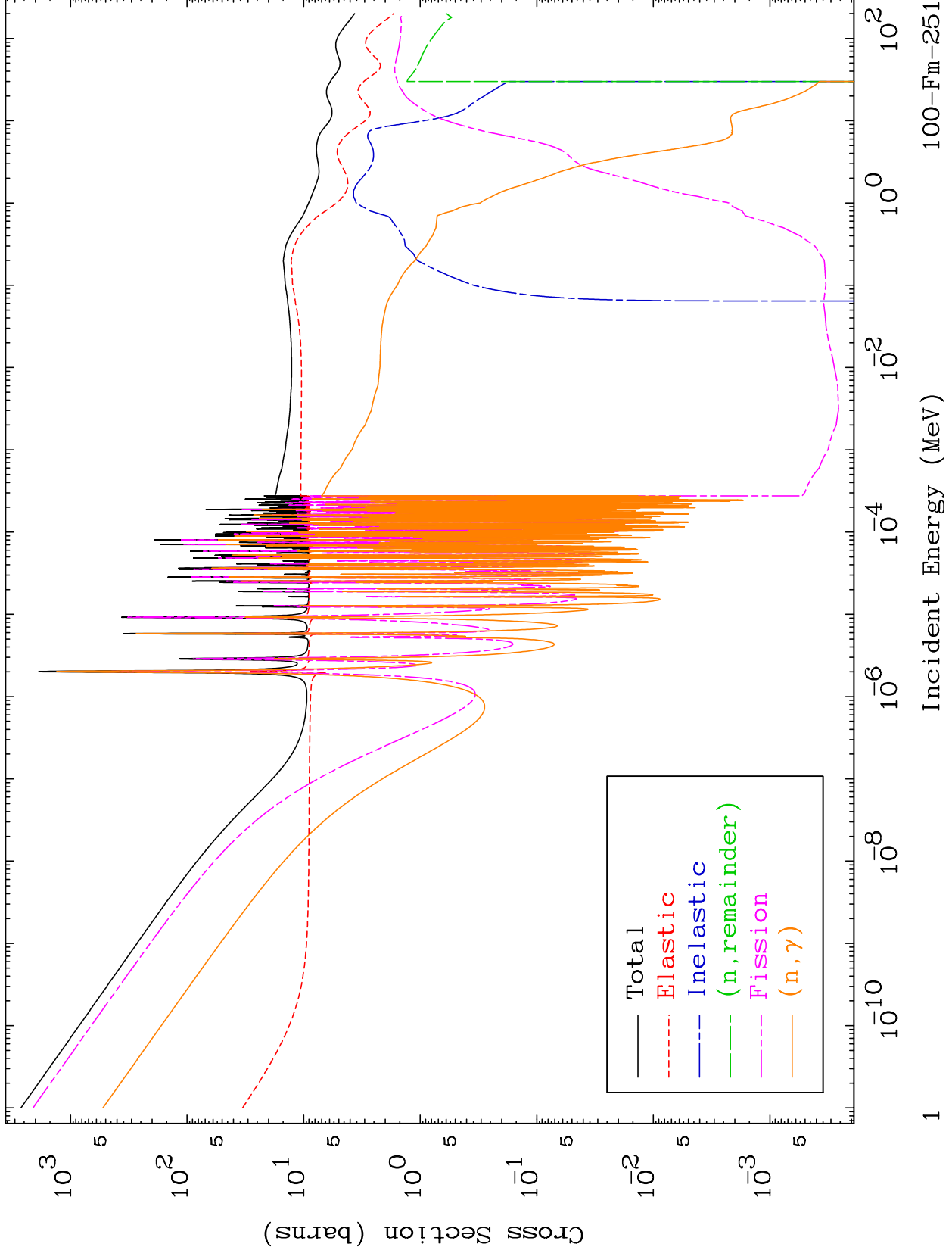
Press Mouse Button to Start

MAT 9932

Major

293 Kelvin Cross Sections

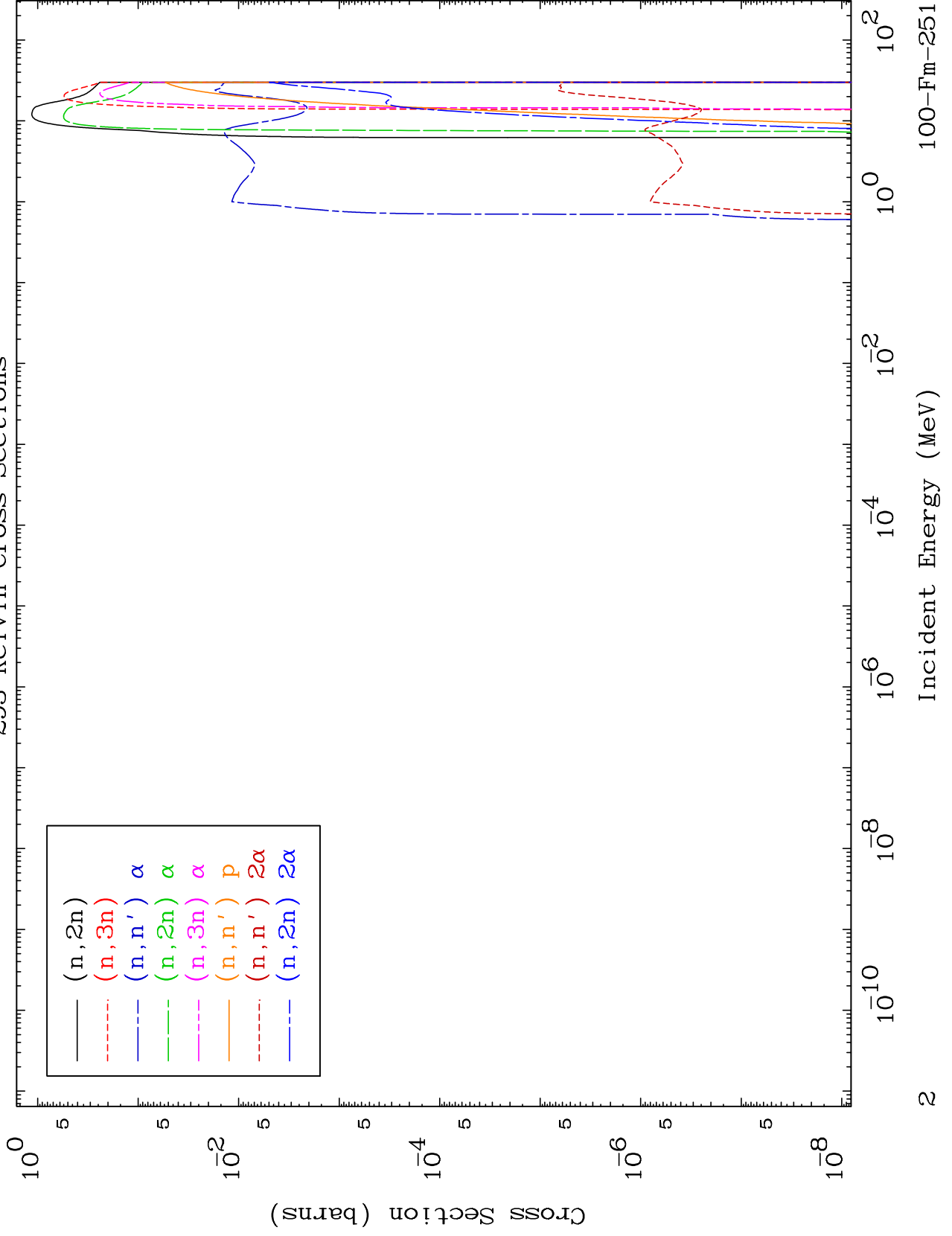
100-Fm-251



MAT 9932

Neutron Production  
293 Kelvin Cross Sections

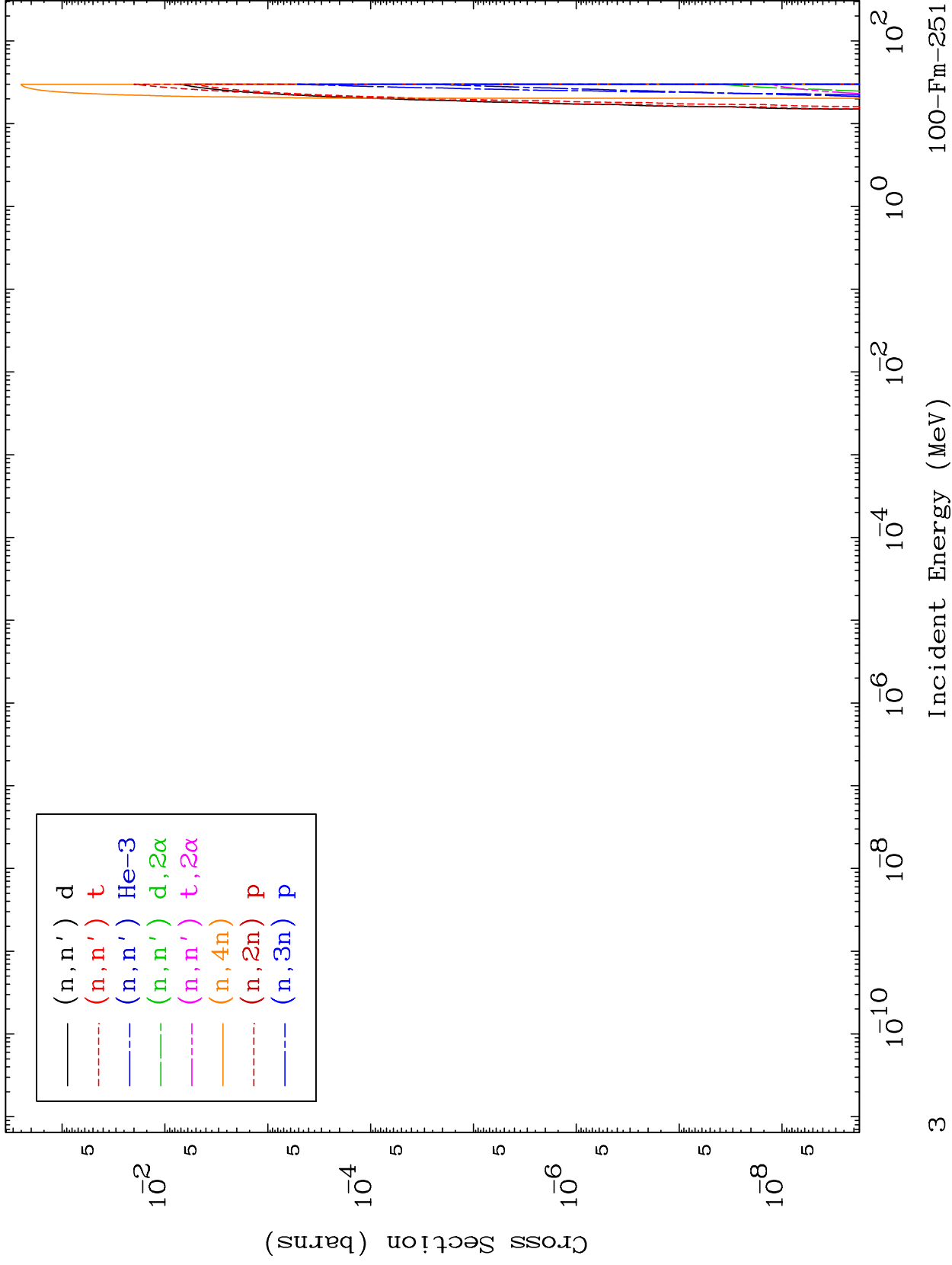
100-Fm-251



MAT 9932

Neutron Production  
293 Kelvin Cross Sections

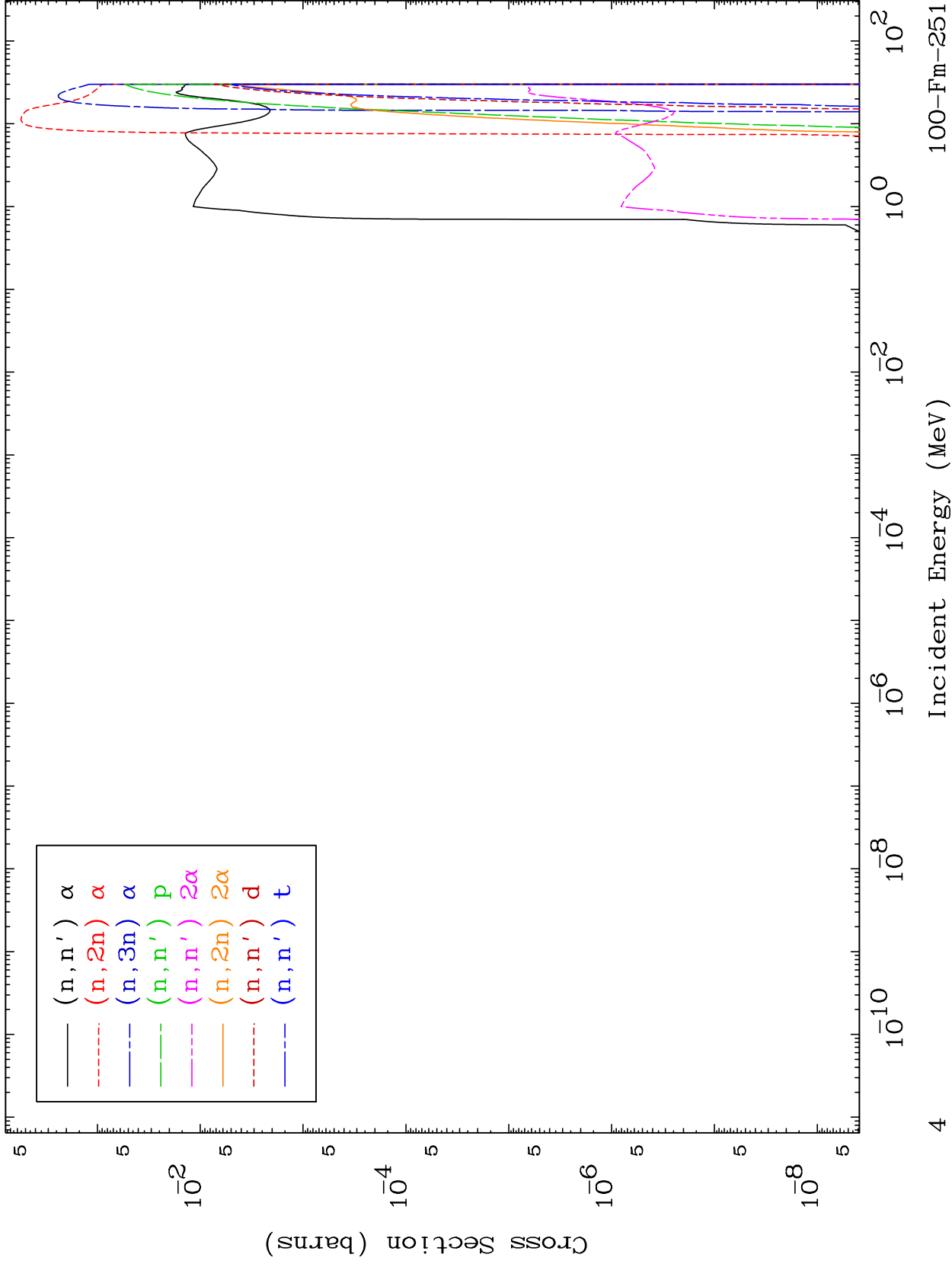
100-Fm-251



MAT 9932

Charged Particle  
293 Kelvin Cross Sections

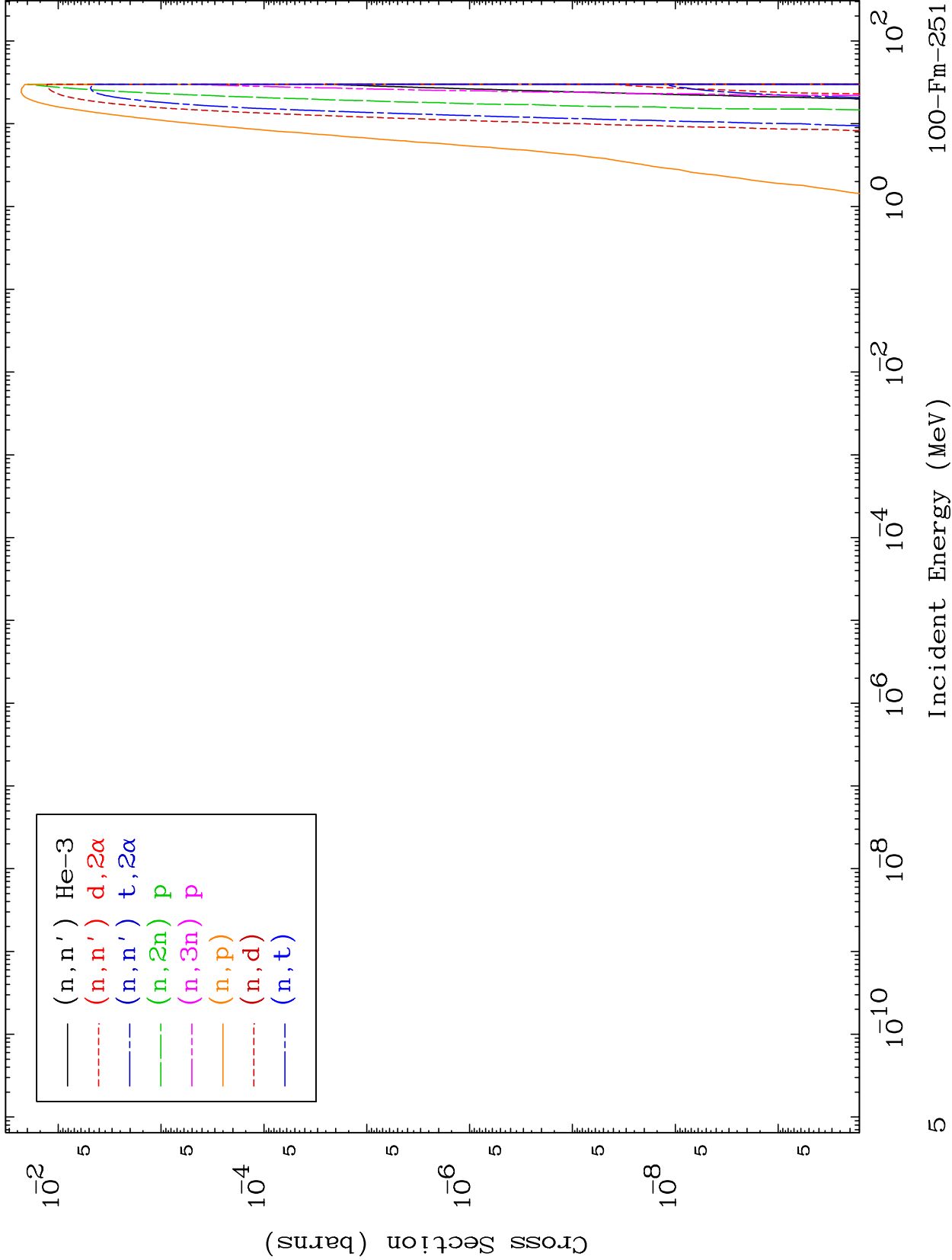
100-Fm-251



MAT 9932

Charged Particle  
293 Kelvin Cross Sections

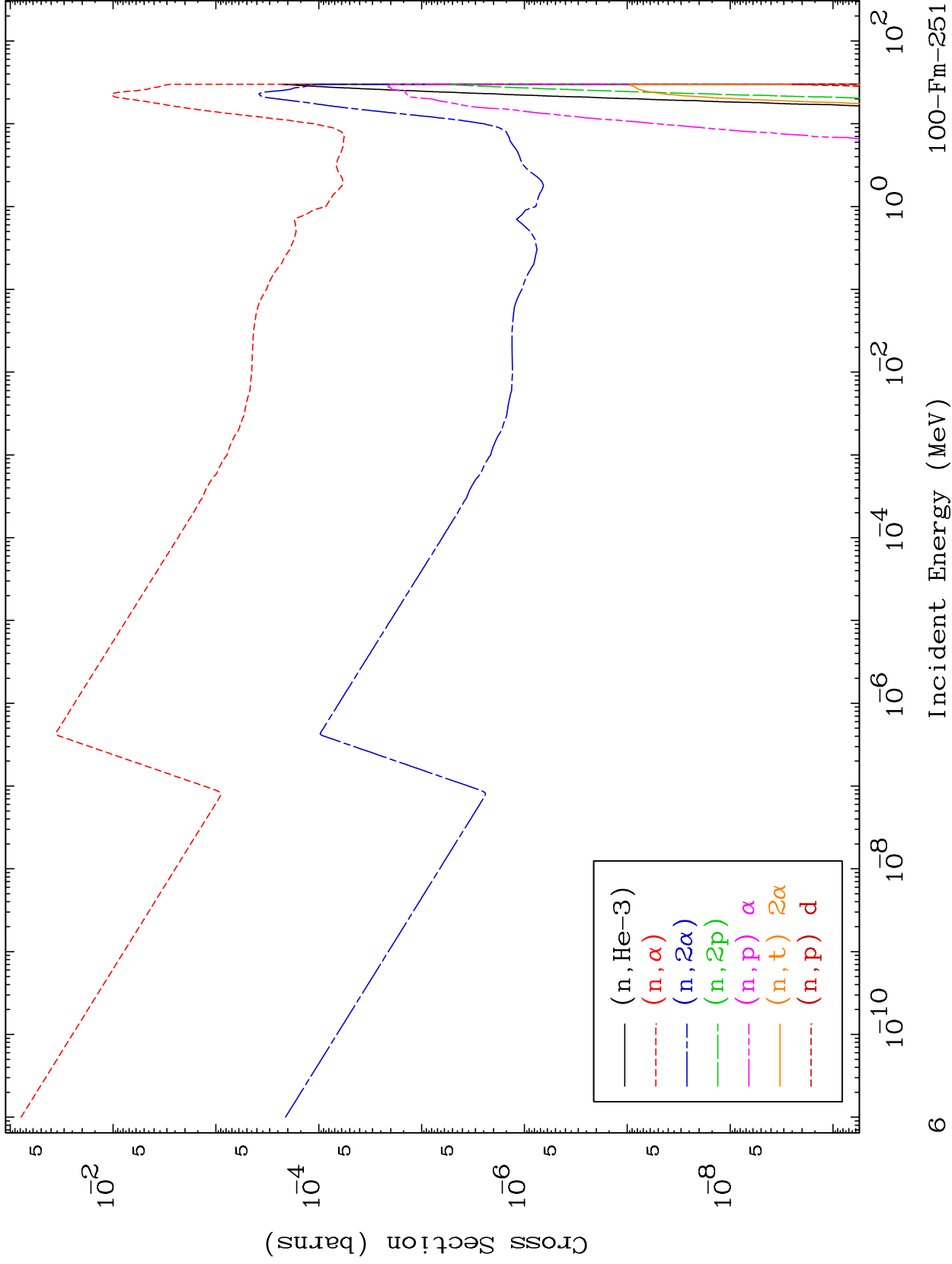
100-Fm-251



MAT 9932

Charged Particle  
293 Kelvin Cross Sections

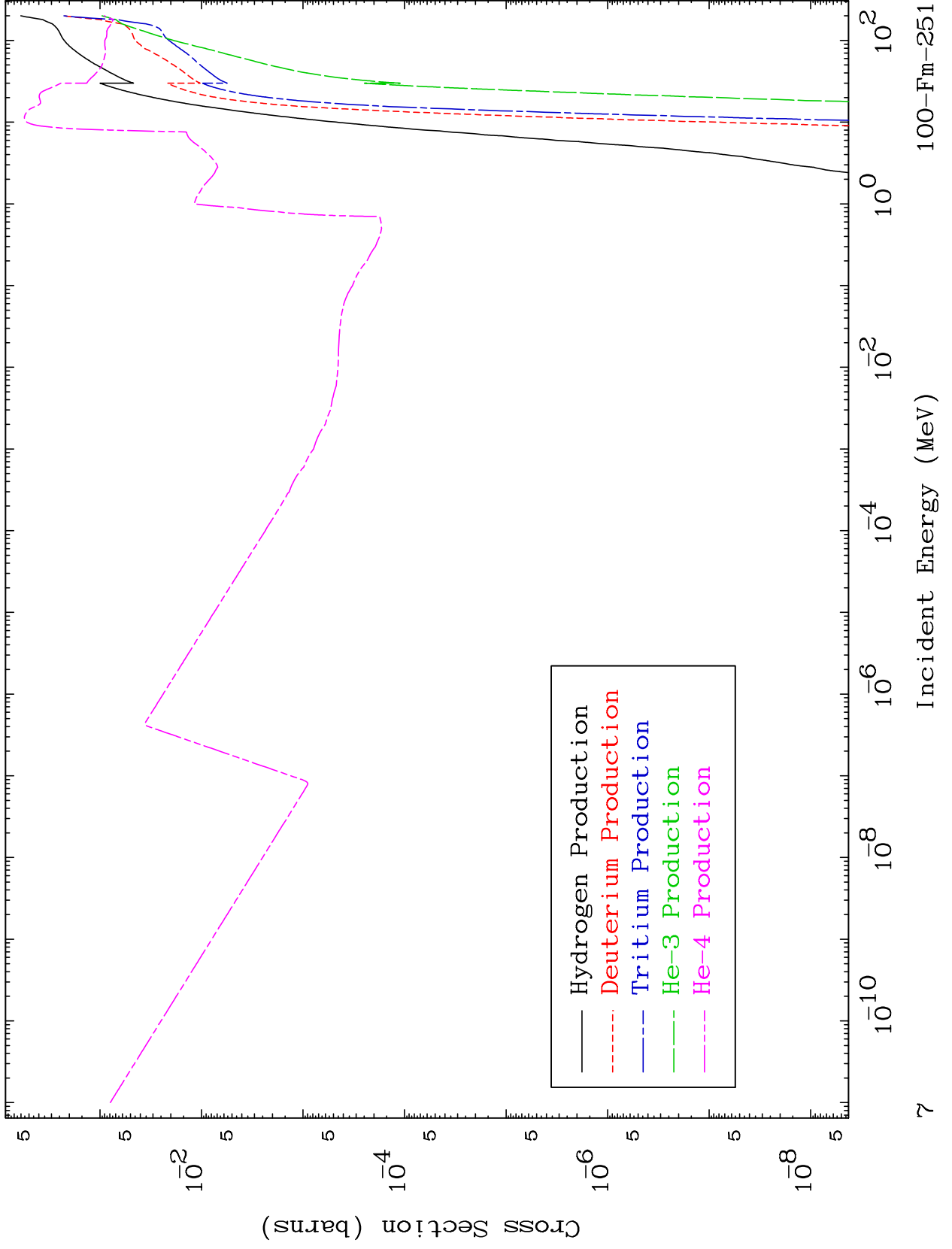
100-Fm-251



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Particle Production  
293 Kelvin Cross Sections

100-Fm-251

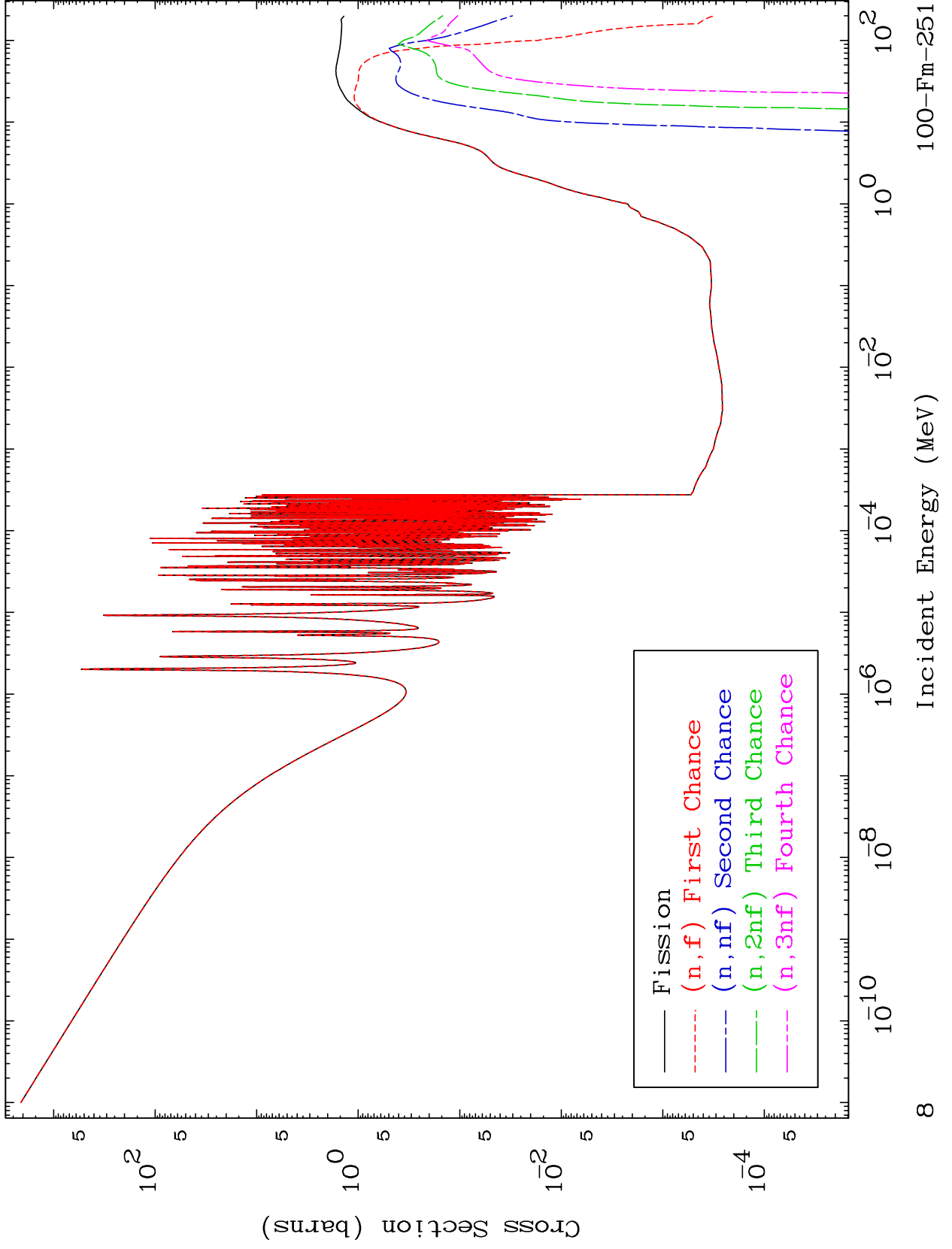




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Fission  
293 Kelvin Cross Sections

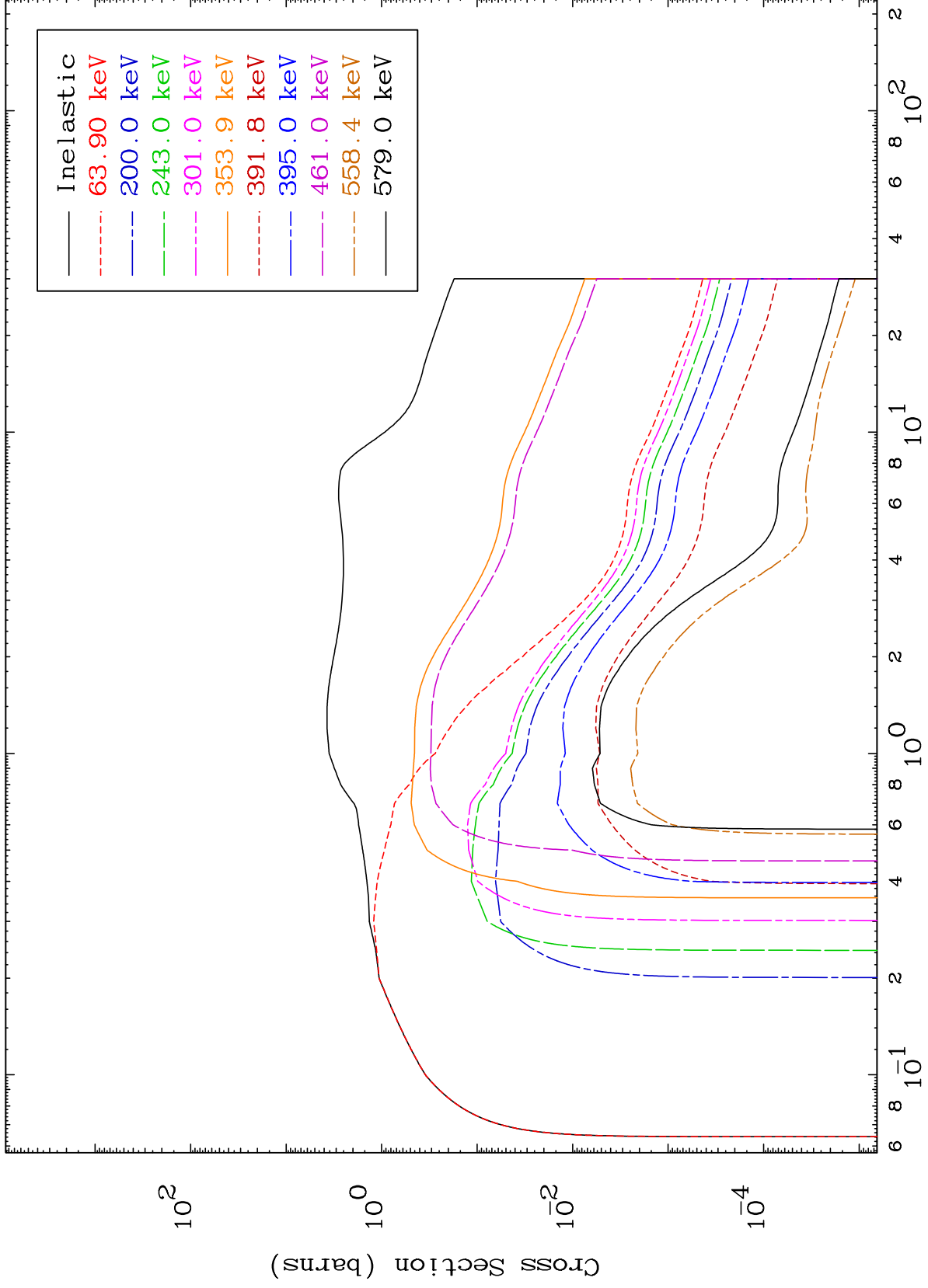
100-Fm-251



MAT 9932

(n,n') Level  
293 Kelvin Cross Sections

100-Fm-251



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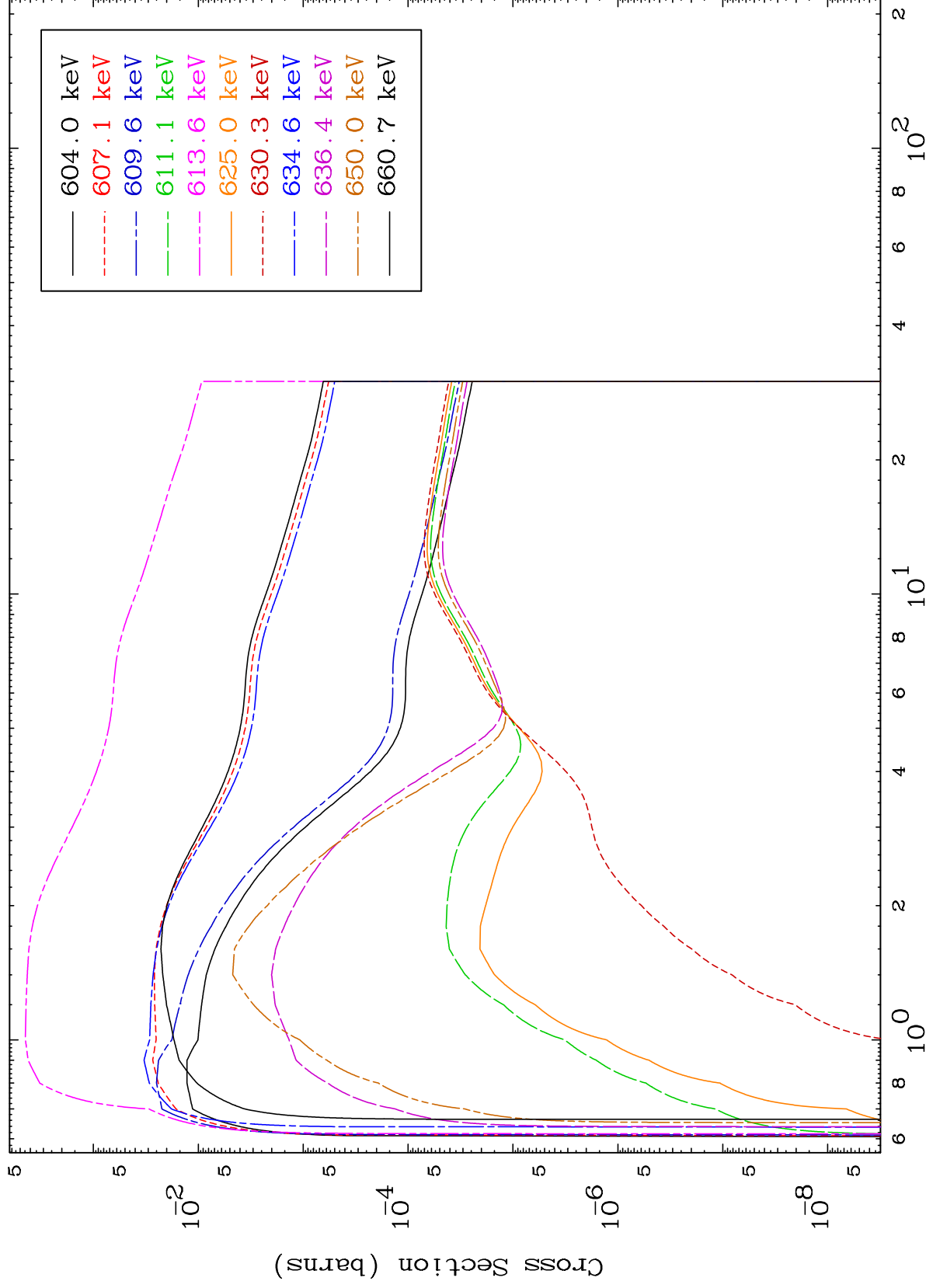
Incident Energy (MeV)

100-Fm-251

MAT 9932

(n,n') Level  
293 Kelvin Cross Sections

100-Fm-251



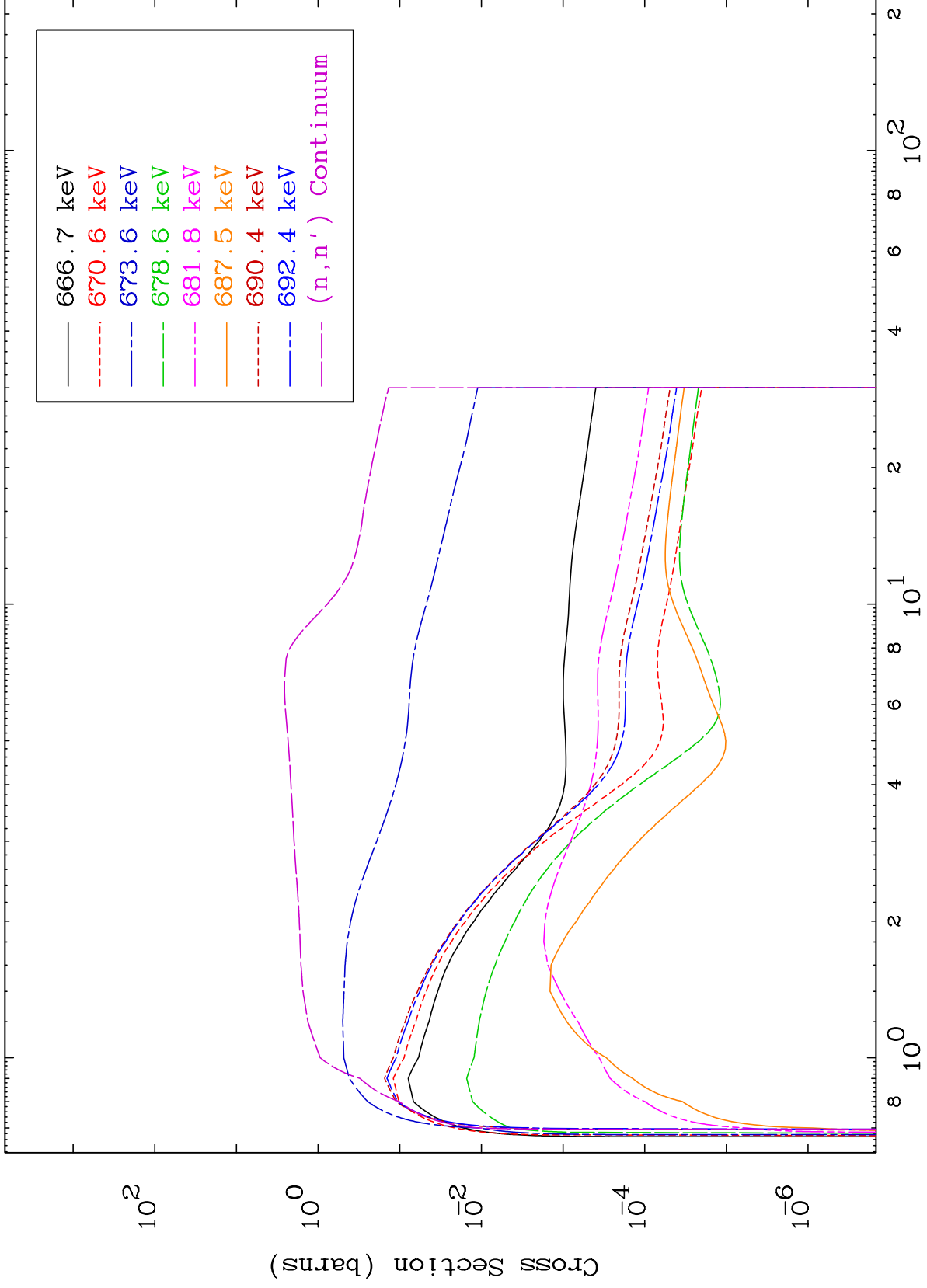
Incident Energy (MeV)

100-Fm-251

MAT 9932

(n,n') Level  
293 Kelvin Cross Sections

100-Fm-251



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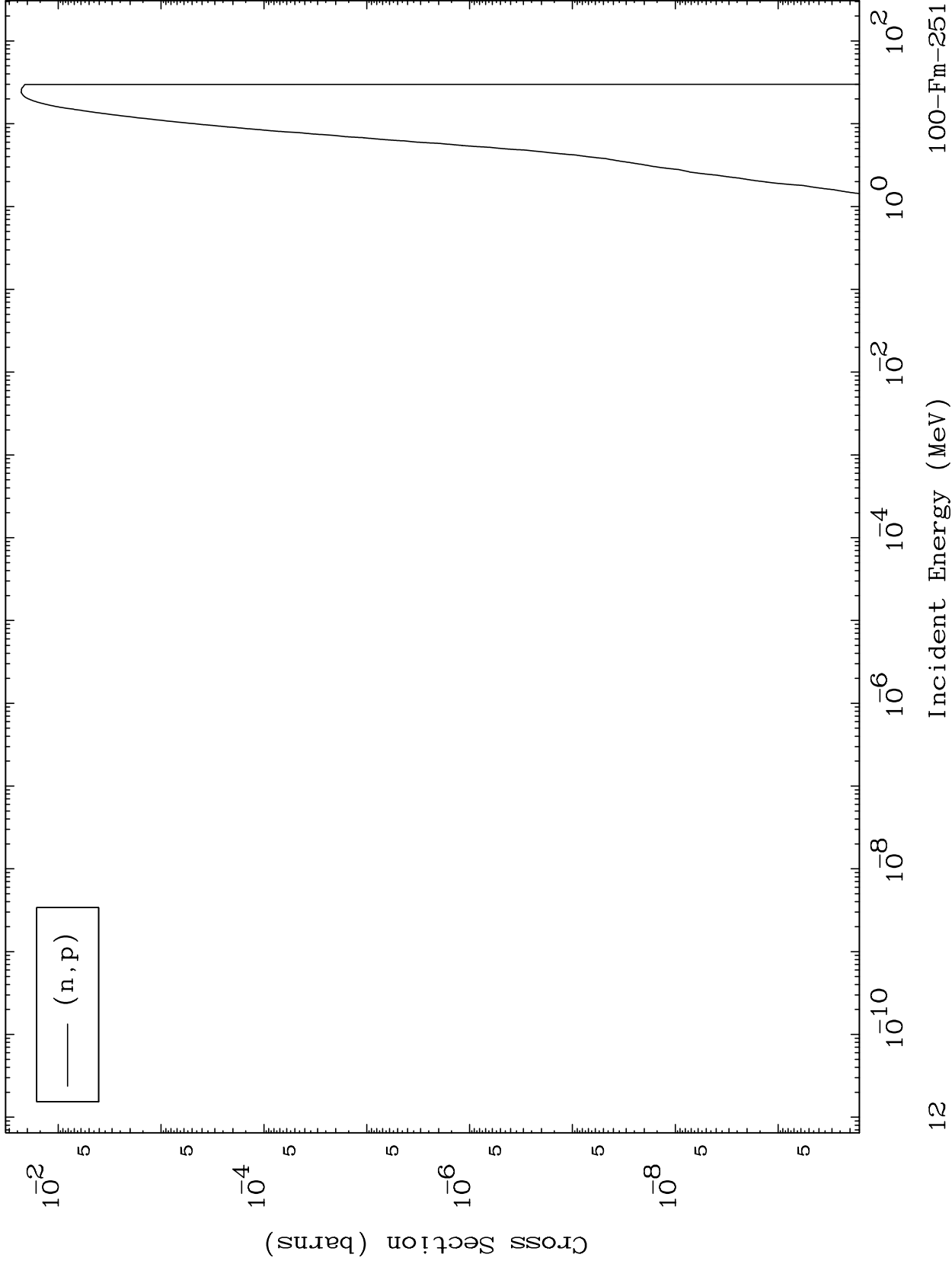
Incident Energy (MeV)

100-Fm-251

MAT 9932

(n,p) Levels  
293 Kelvin Cross Sections

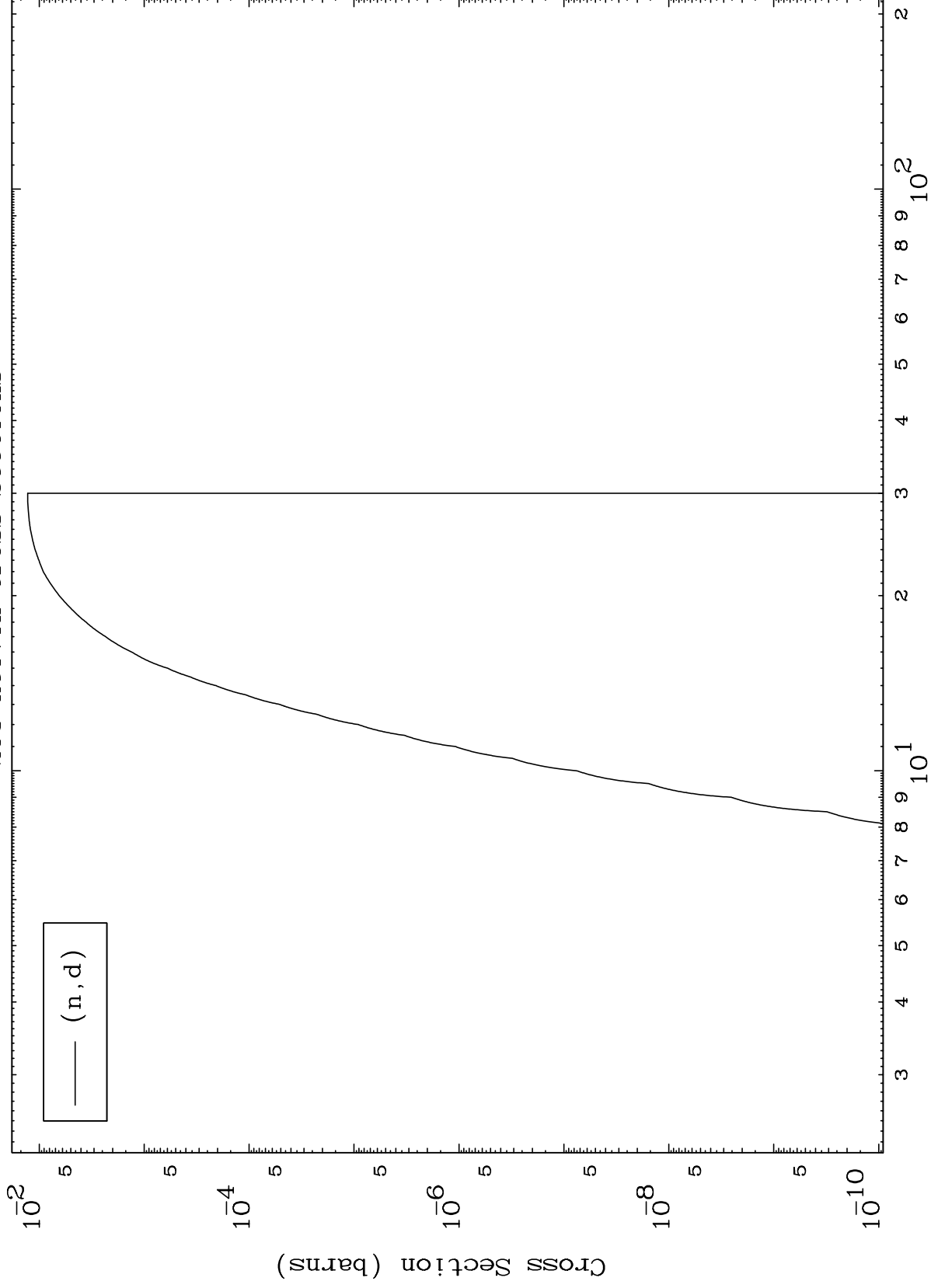
100-Fm-251



MAT 9932

(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-251



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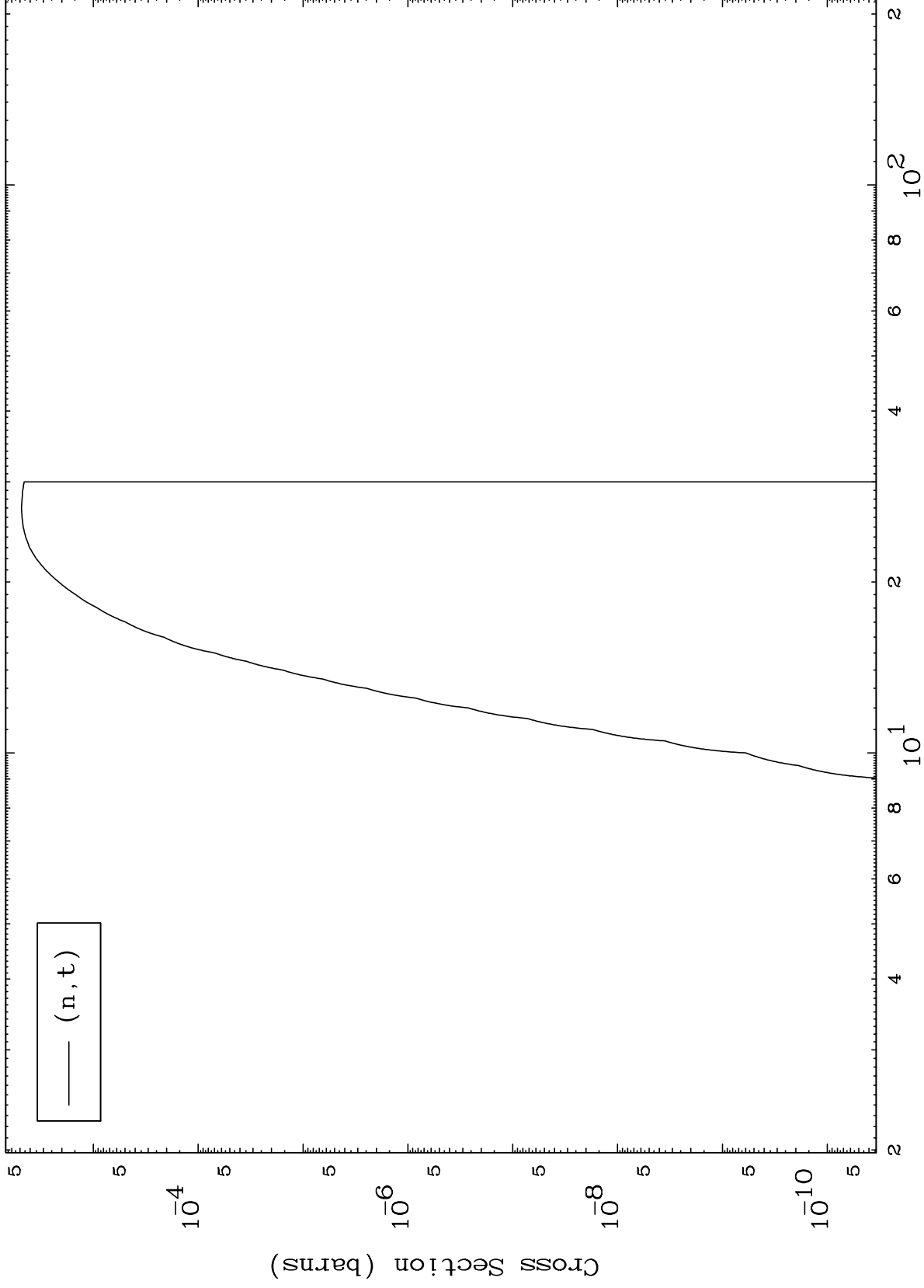
Incident Energy (MeV)

100-Fm-251

MAT 9932

(n,t) Levels  
293 Kelvin Cross Sections

100-Fm-251



14

Incident Energy (MeV)

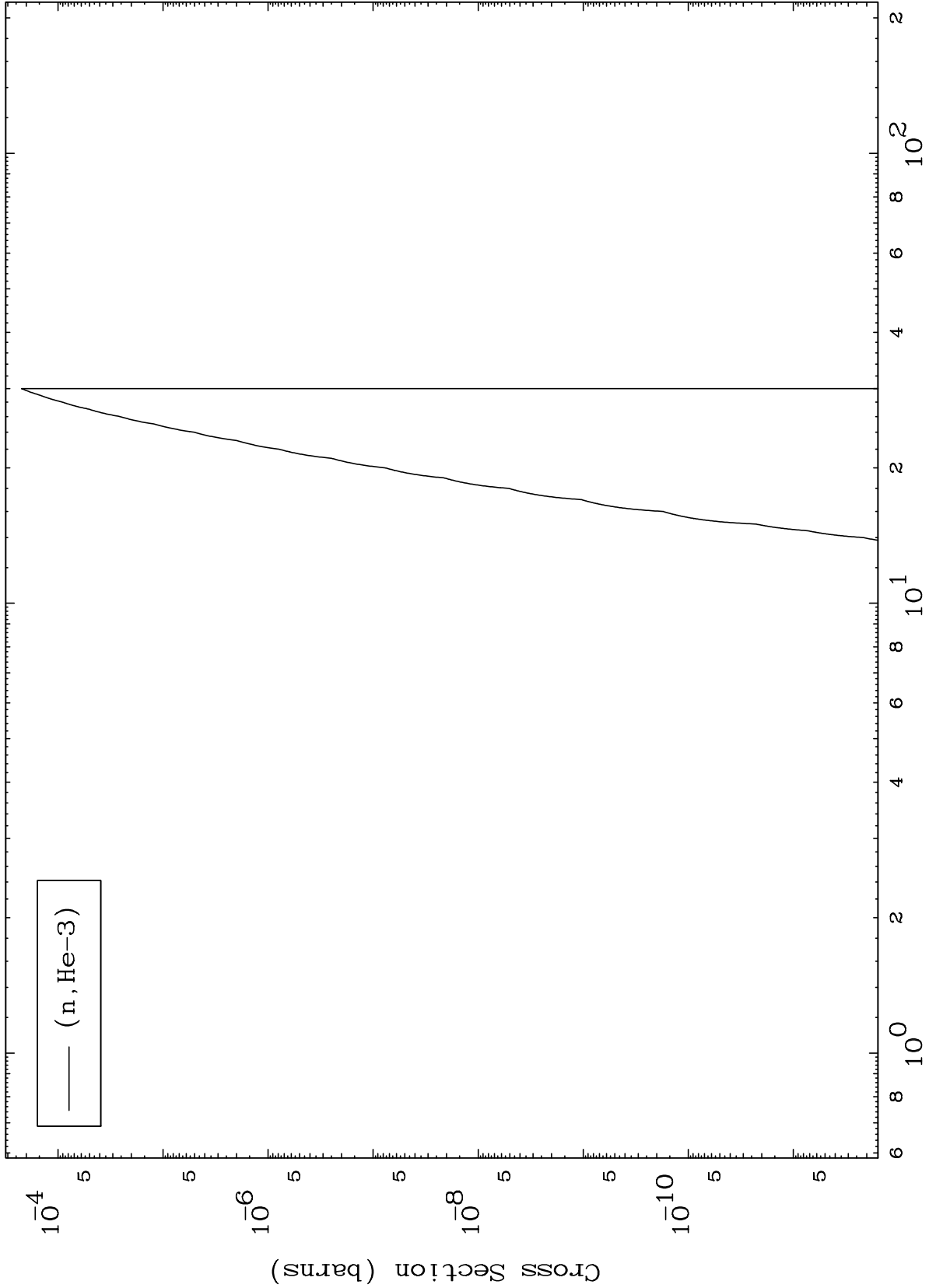
100-Fm-251

MAT 9932

(n,He3) Levels

100-Fm-251

293 Kelvin Cross Sections



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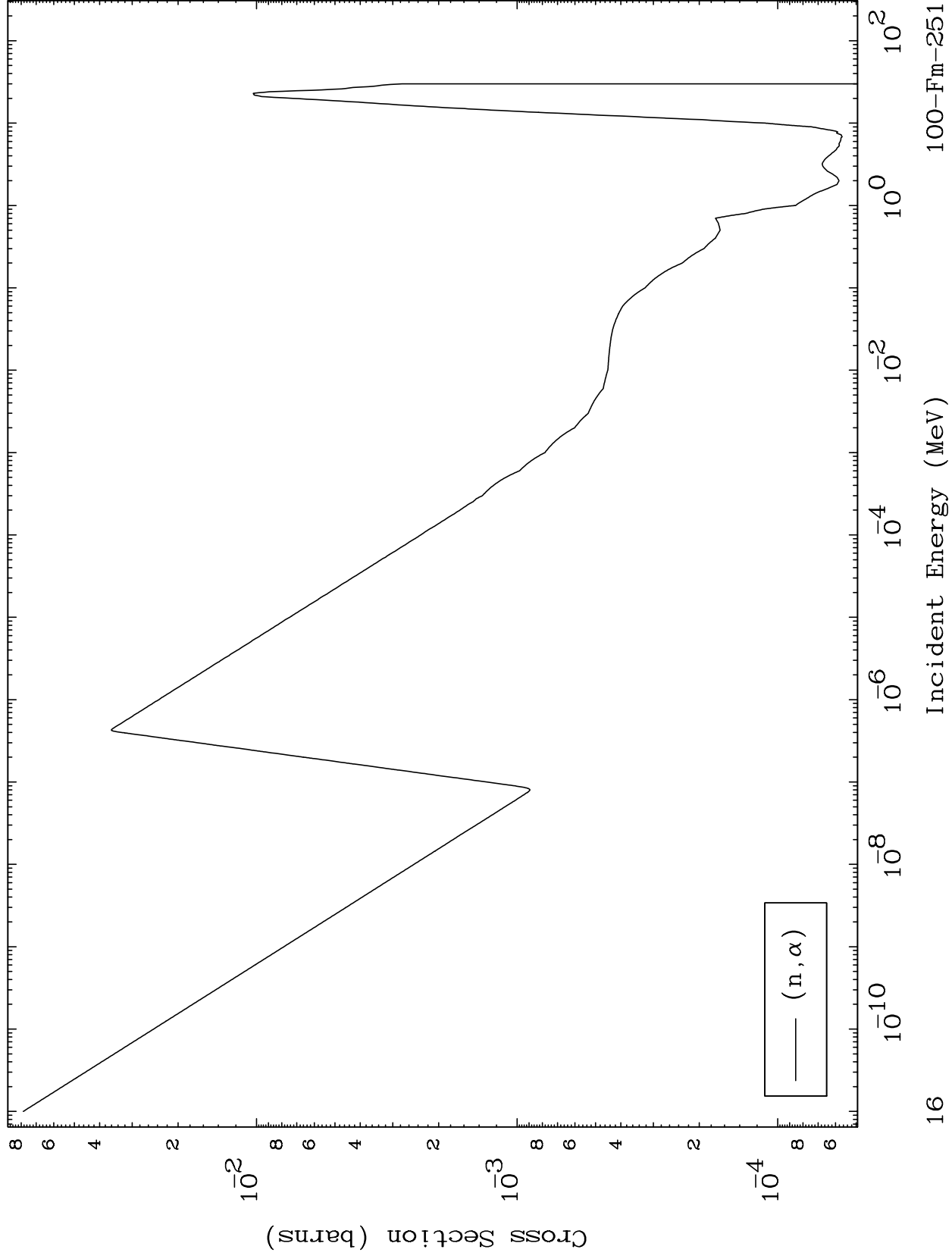
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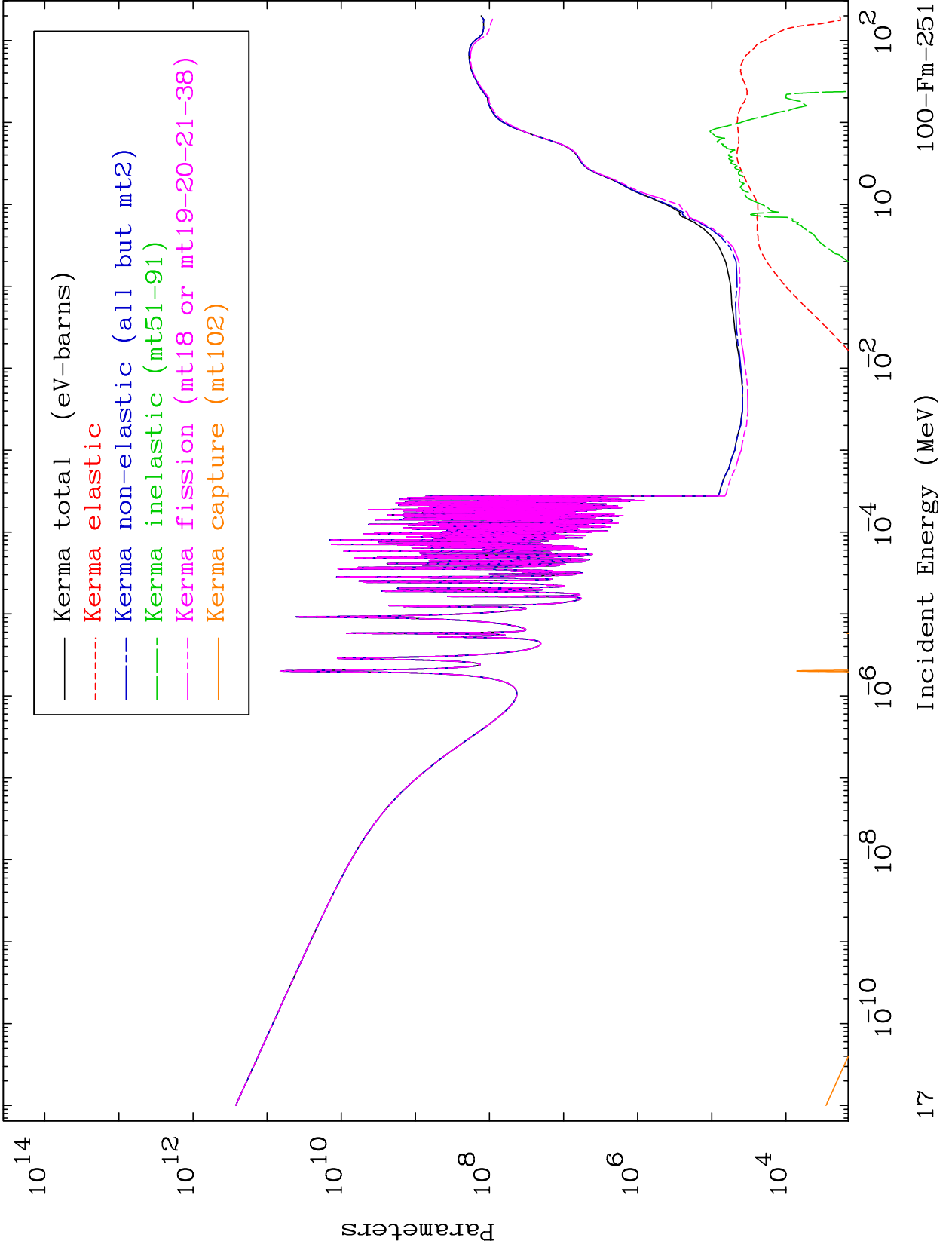


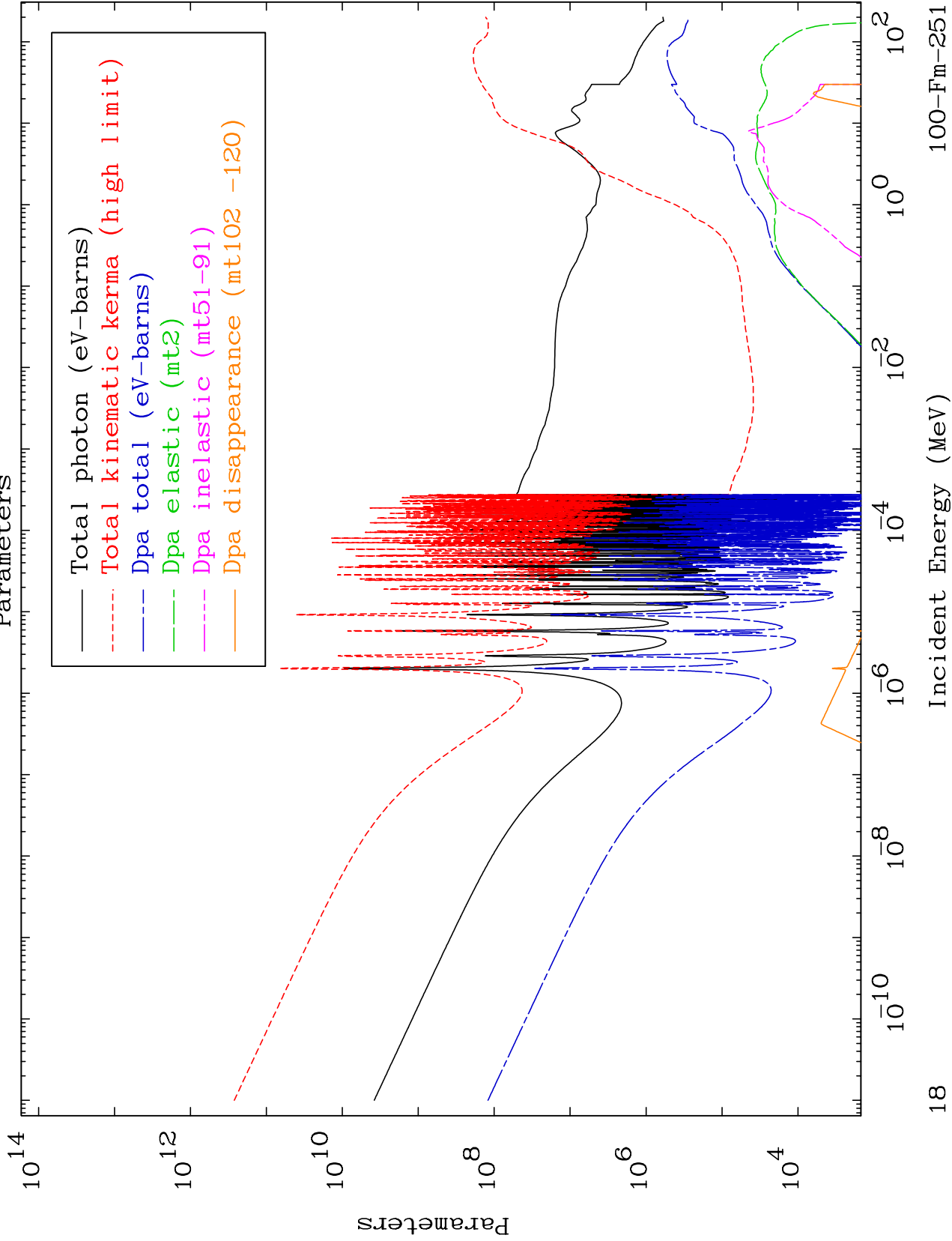
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

100-Fm-251



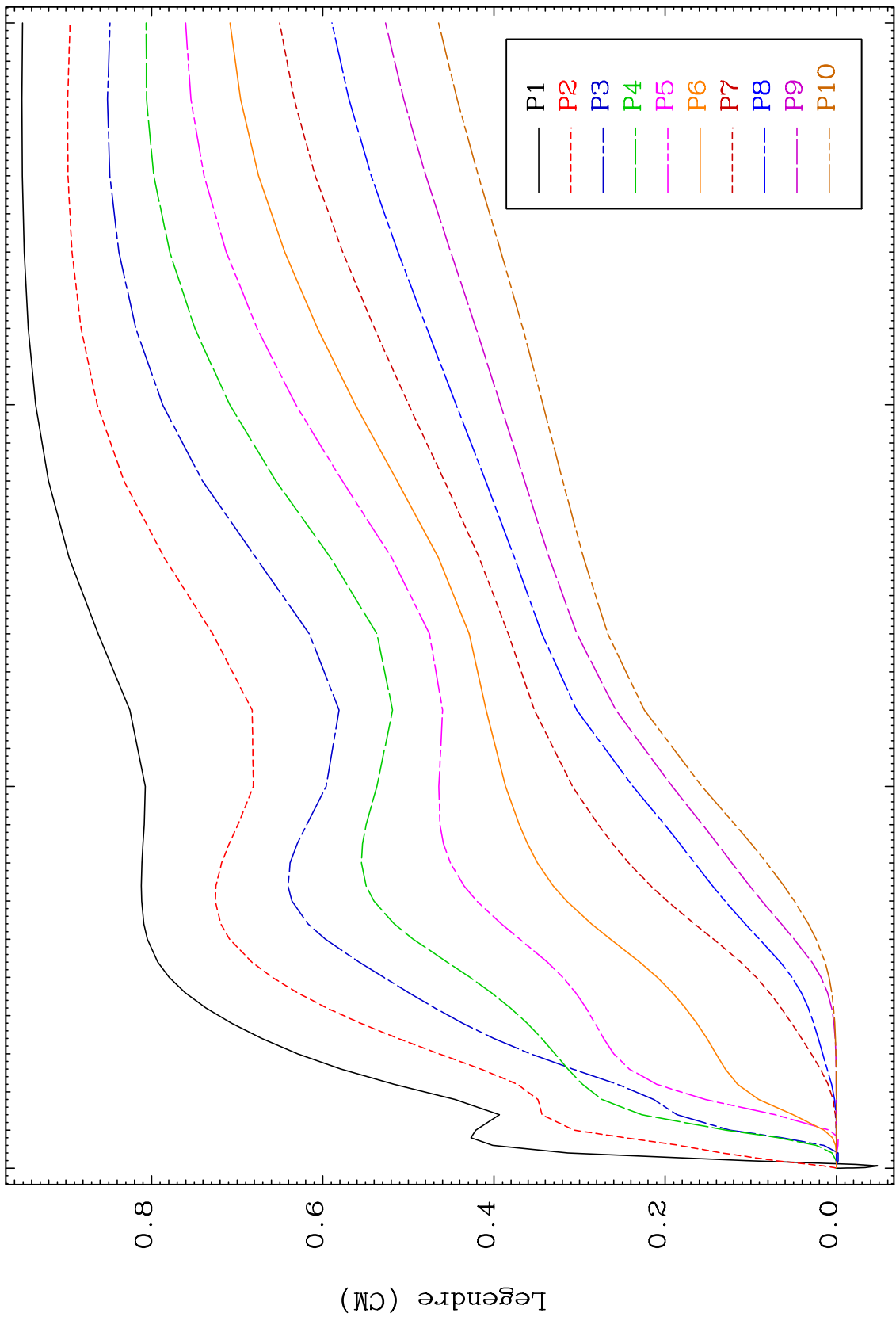




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Elastic Legendre Coefficients

100-Fm-251



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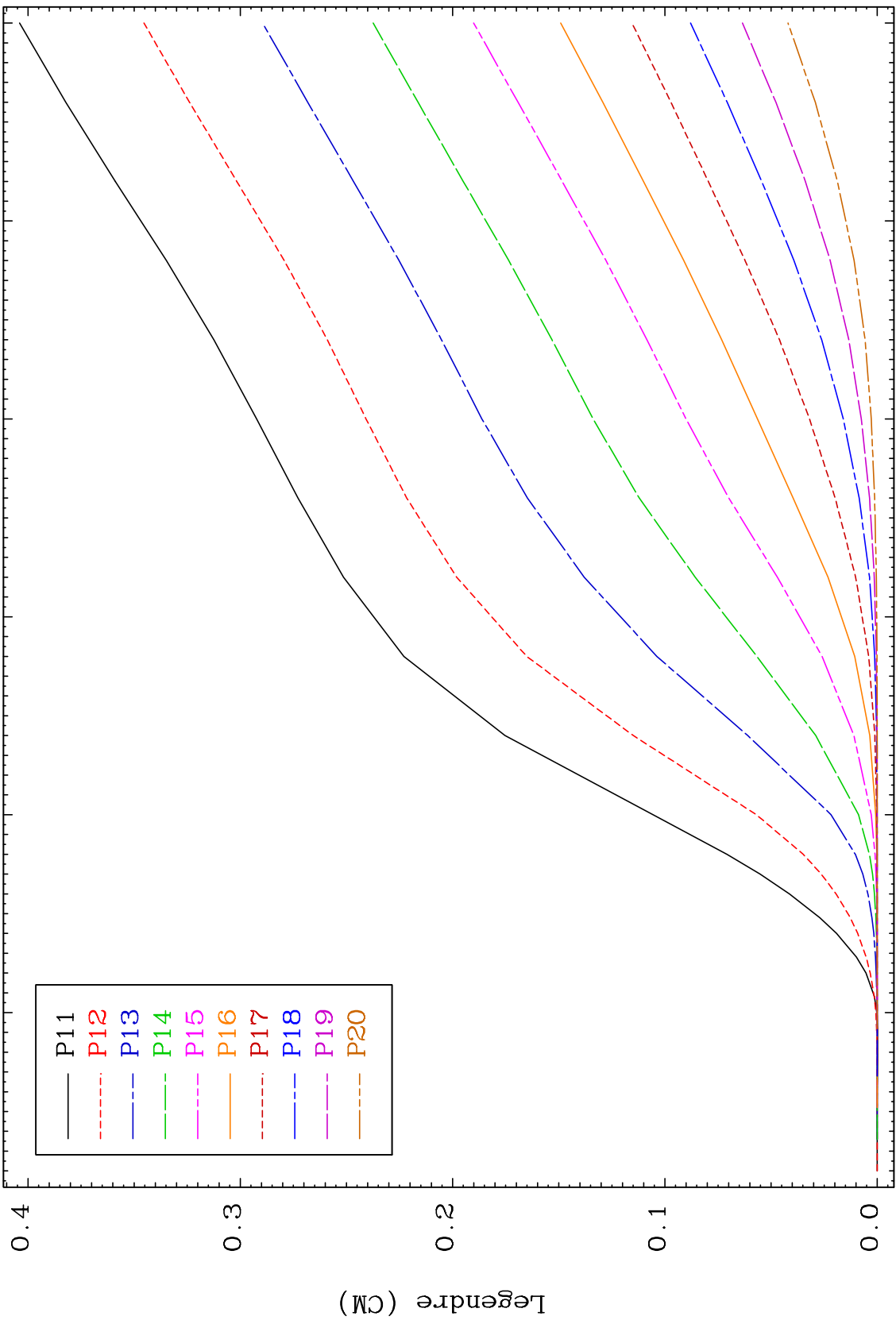
Incident Energy (MeV)

100-Fm-251

MAT 9932

Elastic Legendre Coefficients

100-Fm-251



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Incident Energy (MeV)

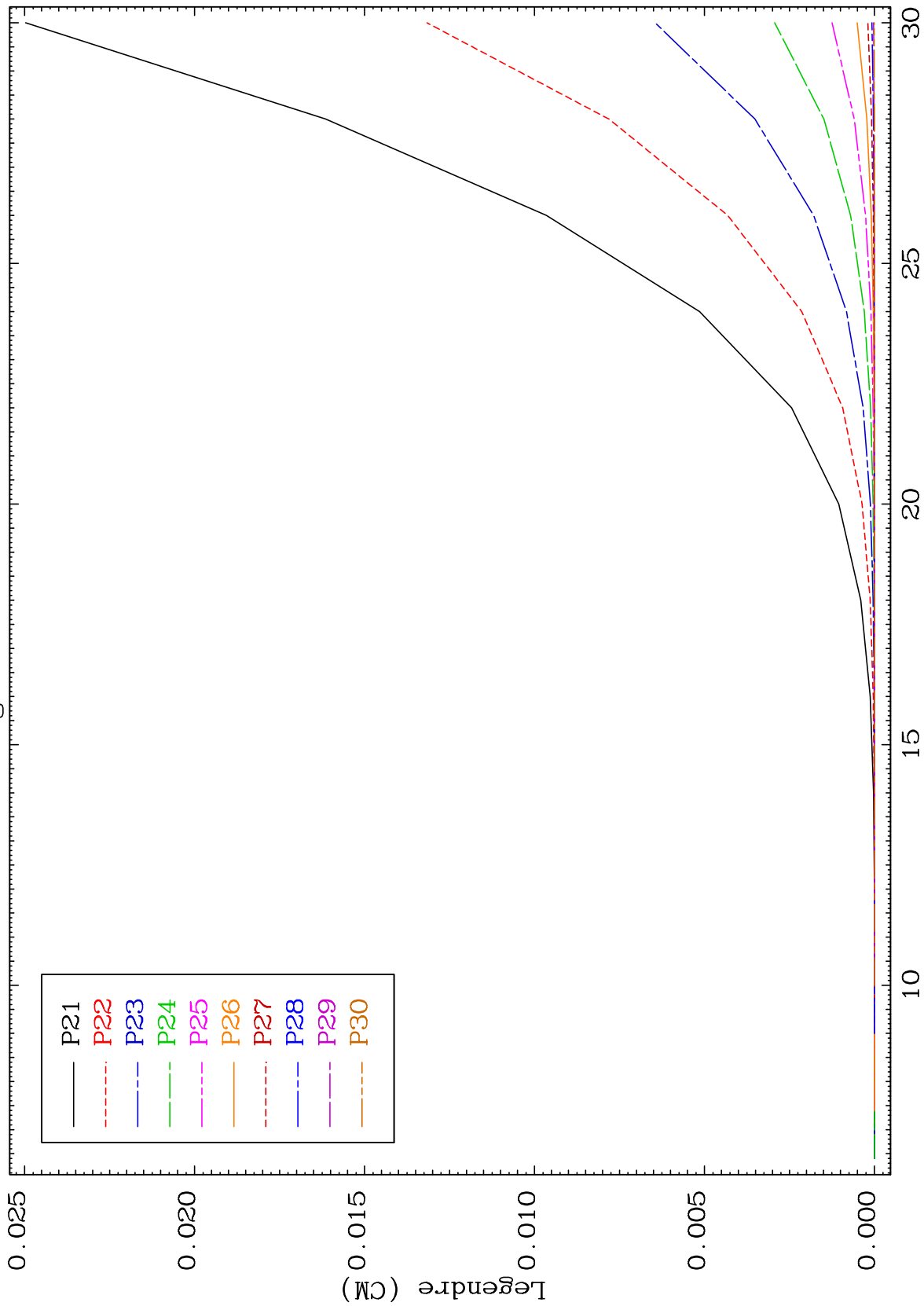
100-Fm-251

- P11
- P12
- P13
- P14
- P15
- P16
- P17
- P18
- P19
- P20

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Elastic  
Legendre Coefficients

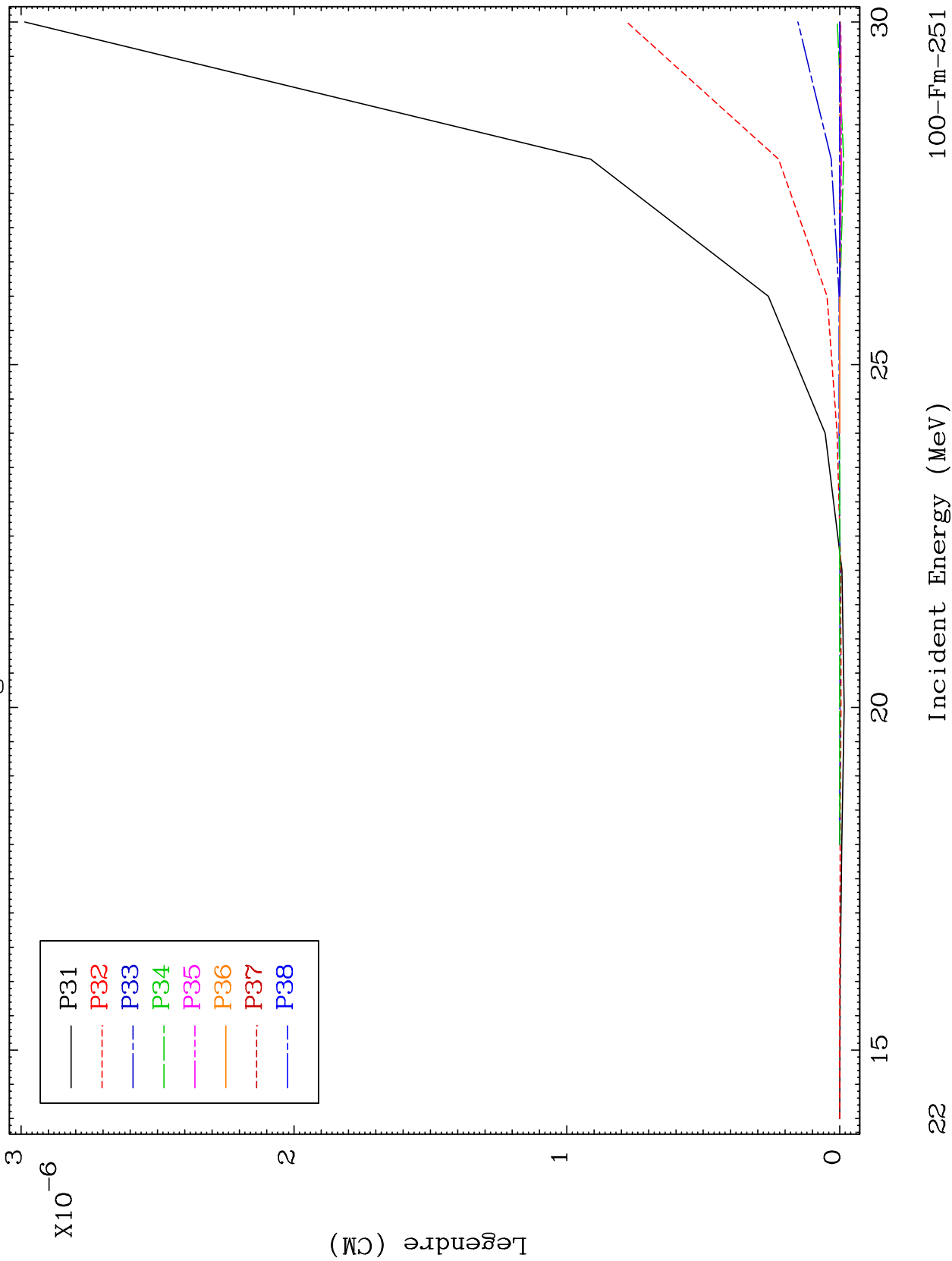
100-Fm-251



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Elastic  
Legendre Coefficients

100-Fm-251

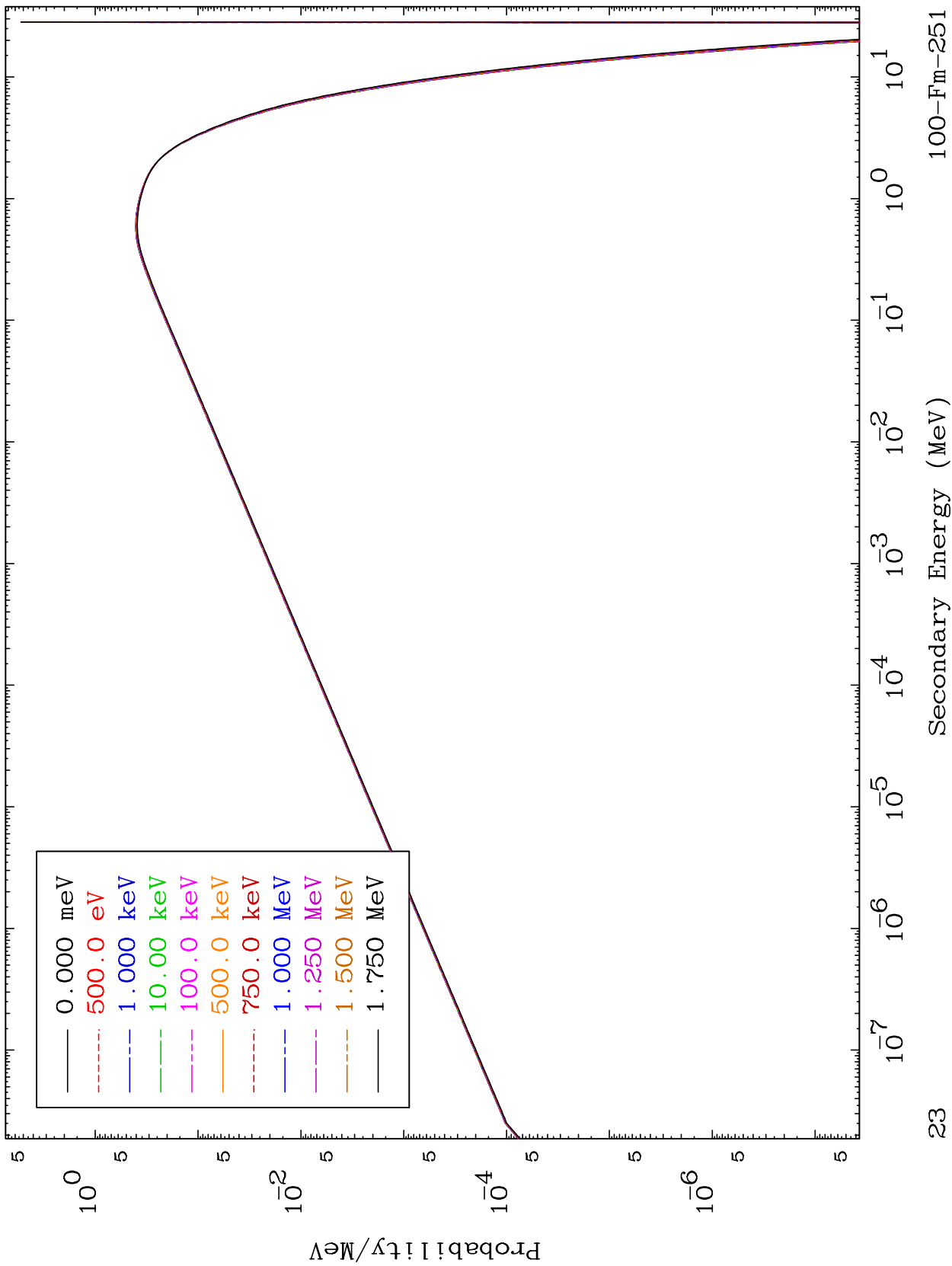


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Incident Energy (MeV)

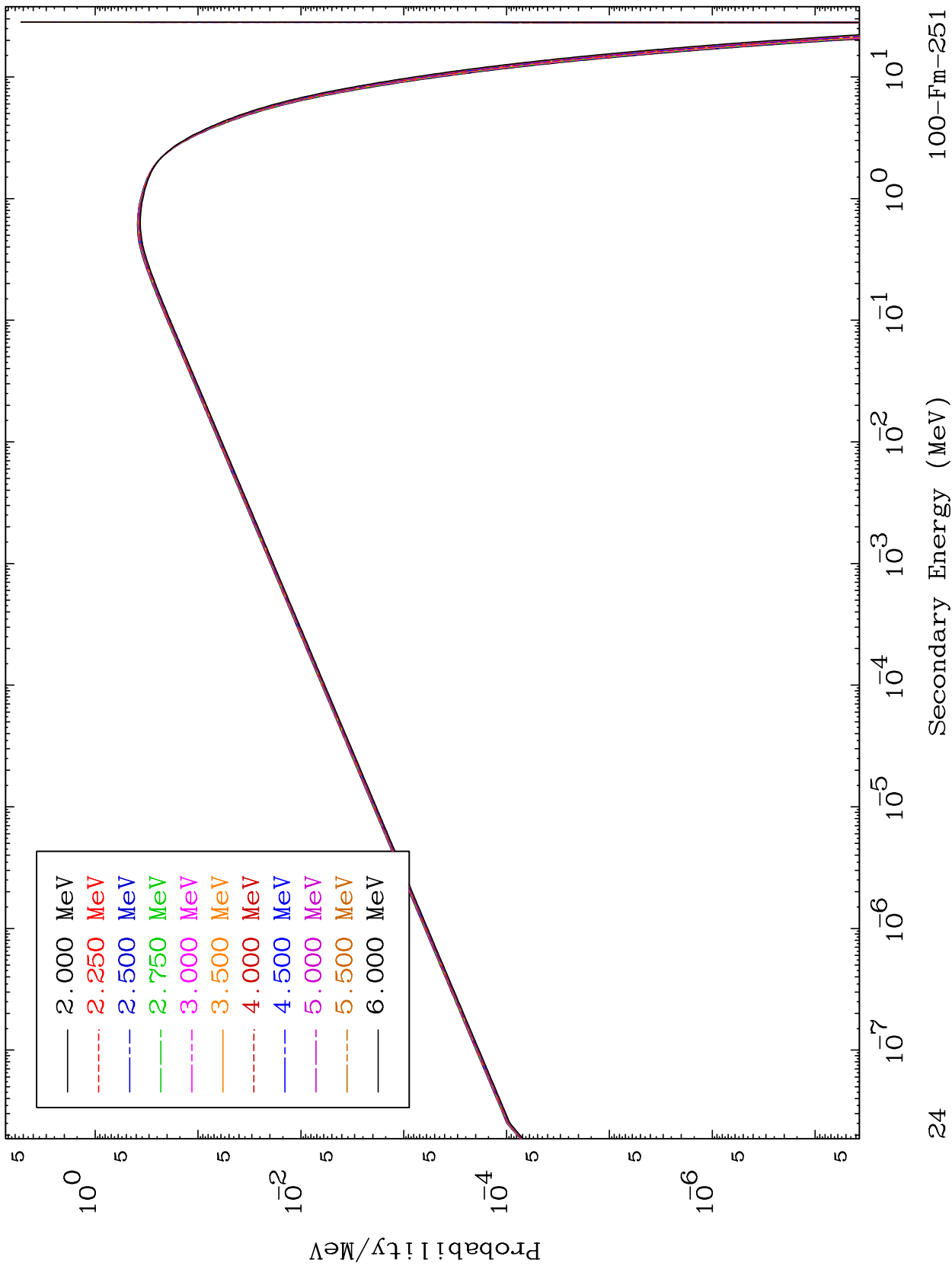
100-Fm-251

Fission Energy Distribution





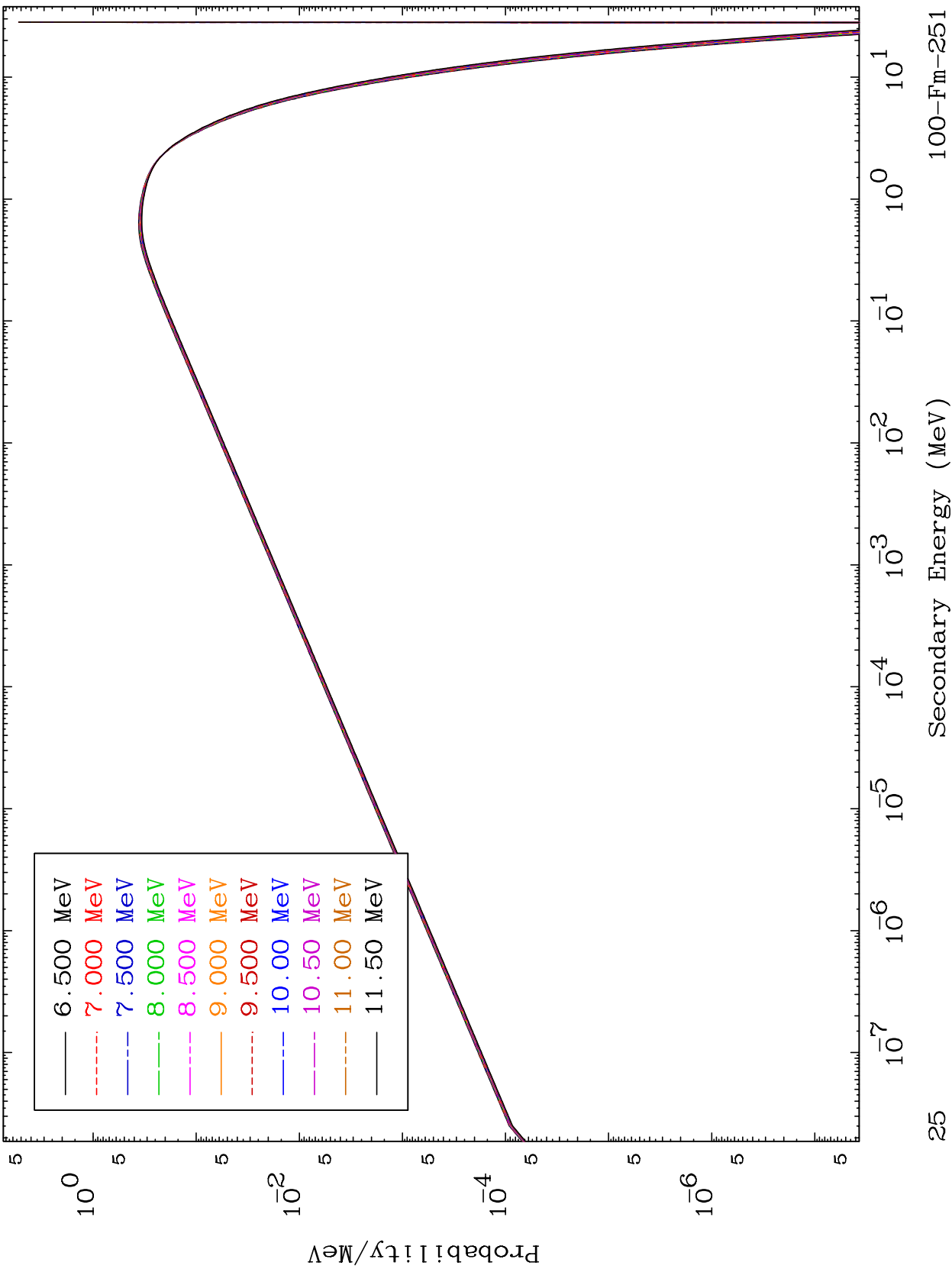
Fission Energy Distribution



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### Fission Energy Distribution

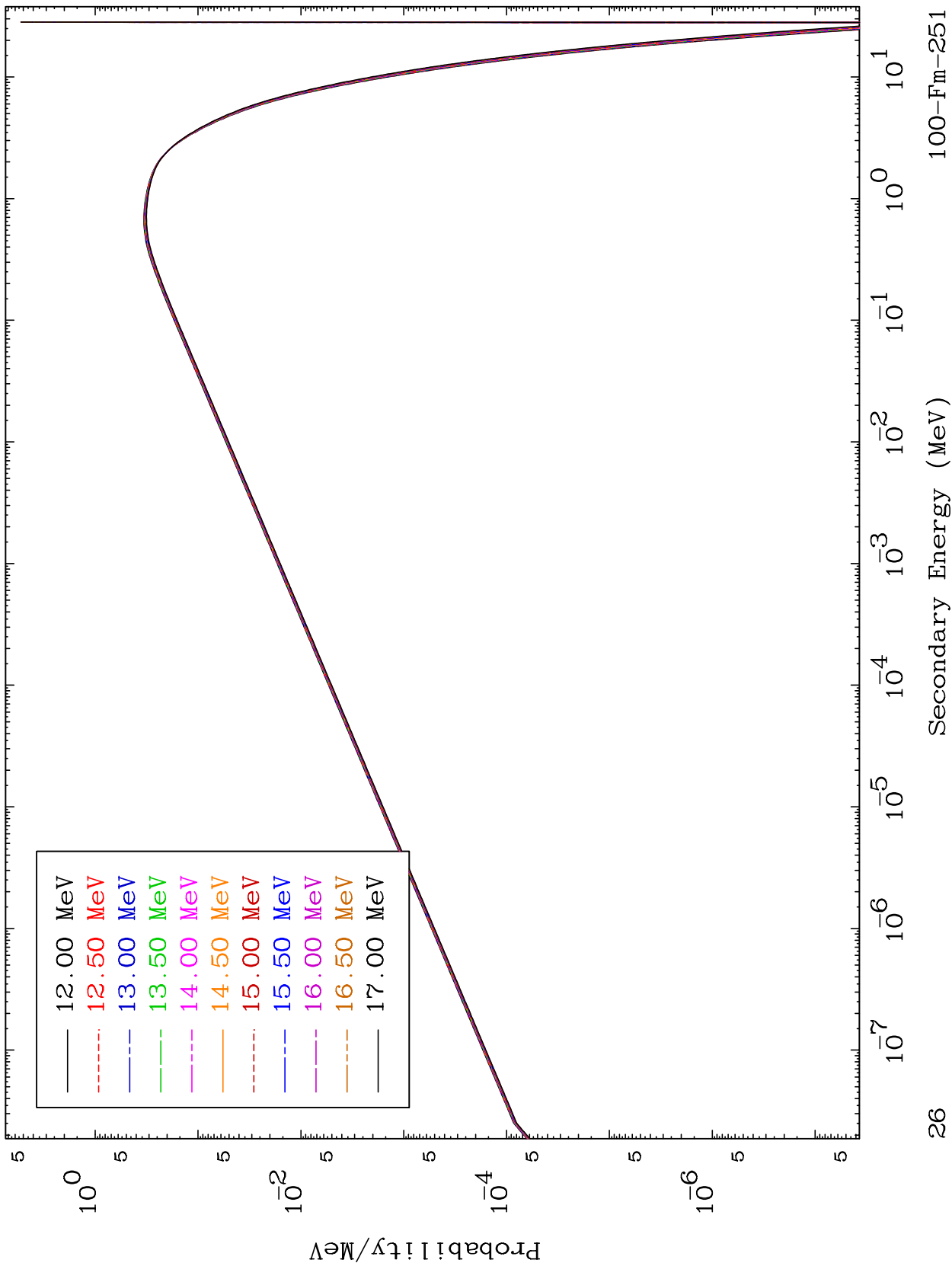
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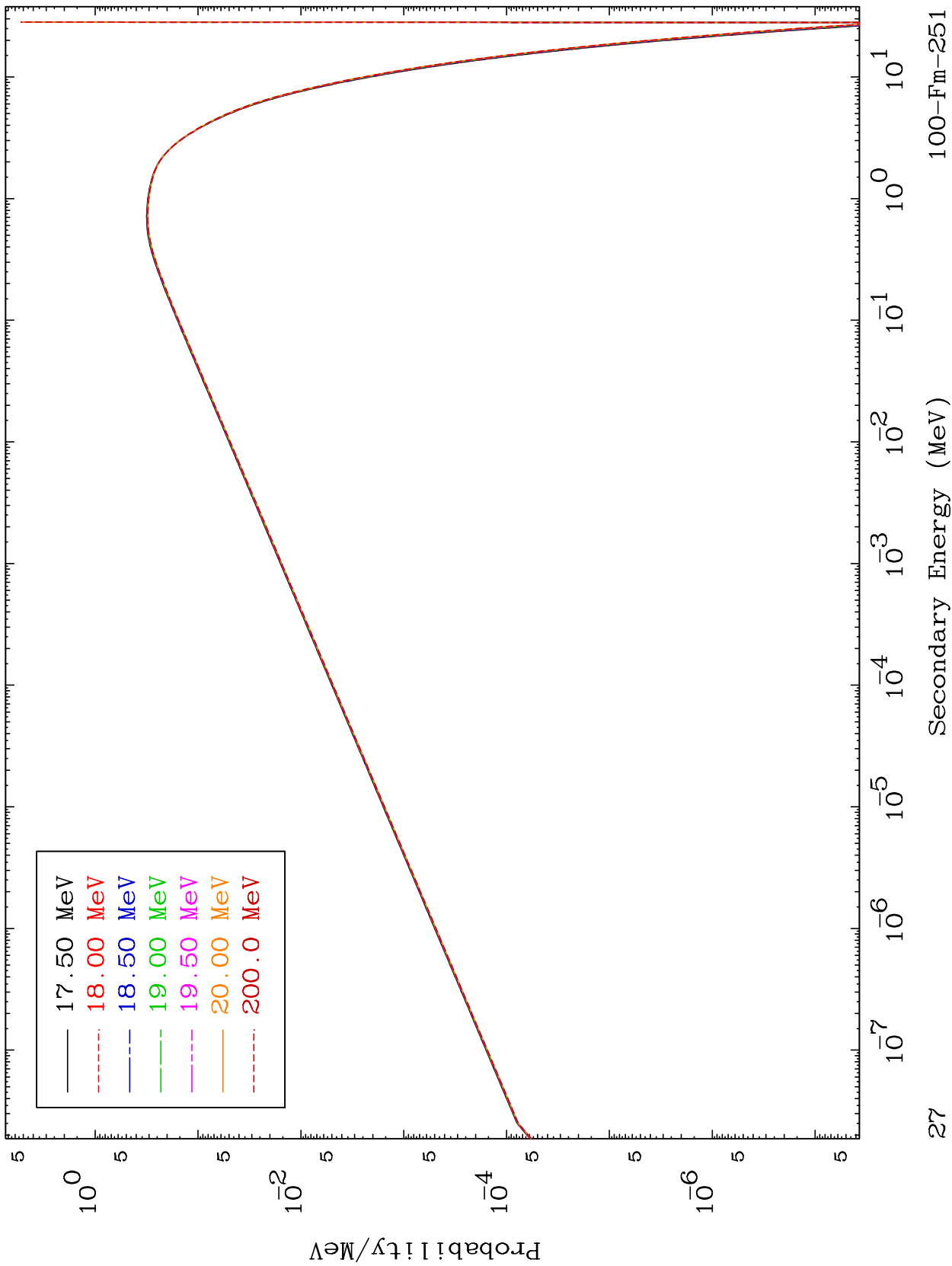
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Fission  
Energy Distribution

100-Fm-251



Fission  
Energy Distribution

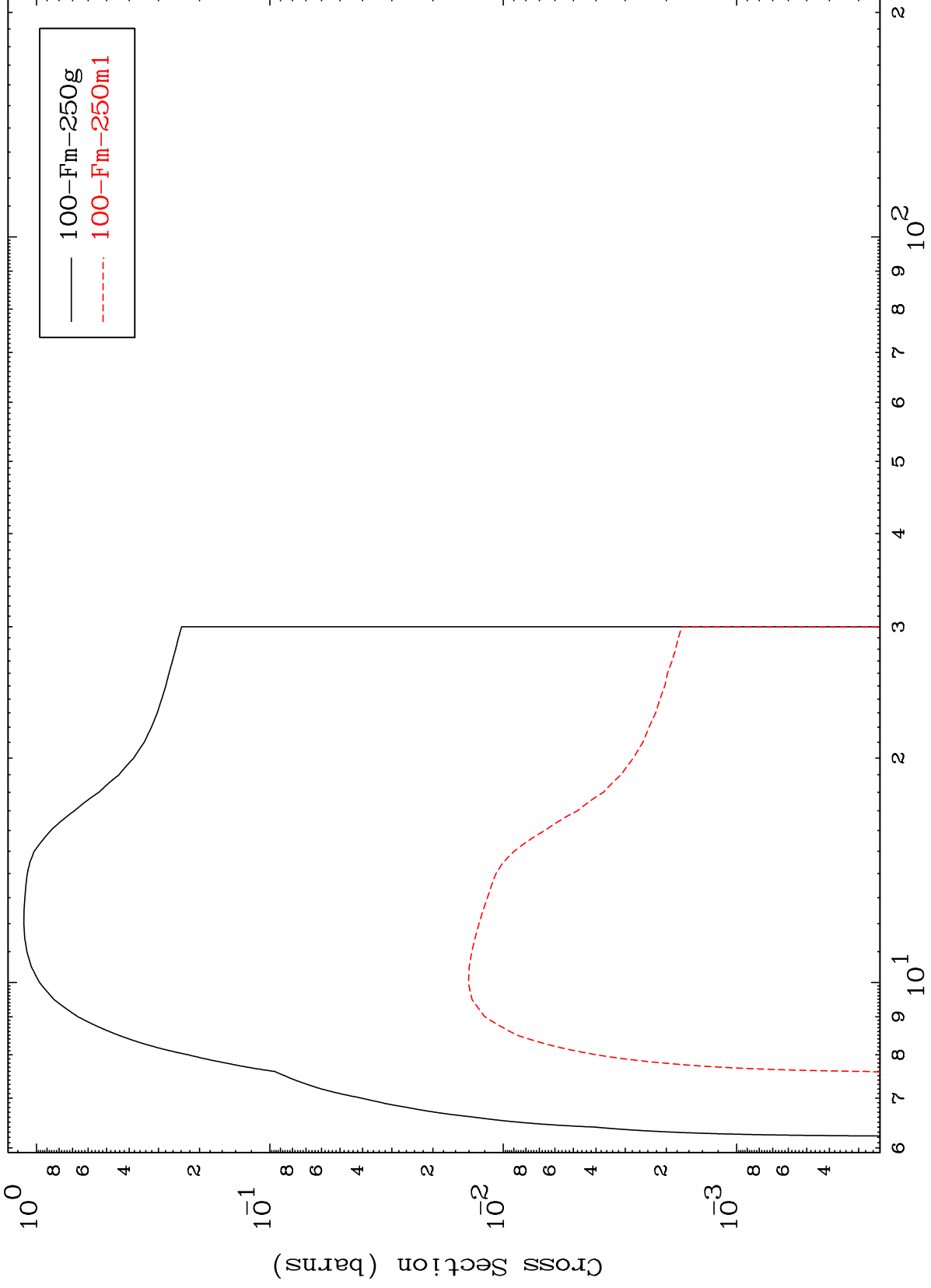


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(n,2n)

100-Fm-251

Radionuclide Production Cross Section



28

Incident Energy (MeV)

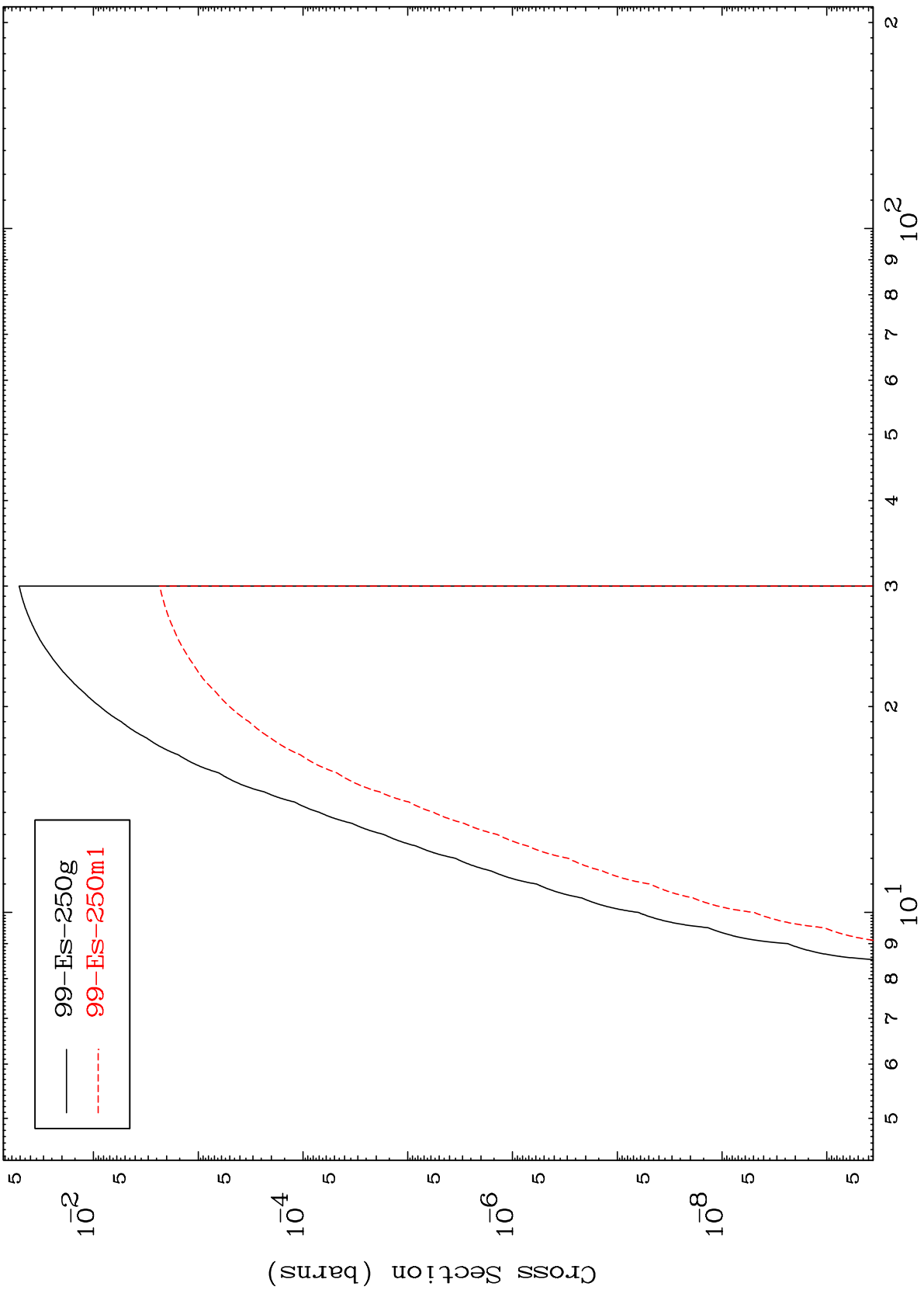
100-Fm-251

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(n,n') p

100-Fm-251

Radionuclide Production Cross Section



29

Incident Energy (MeV)

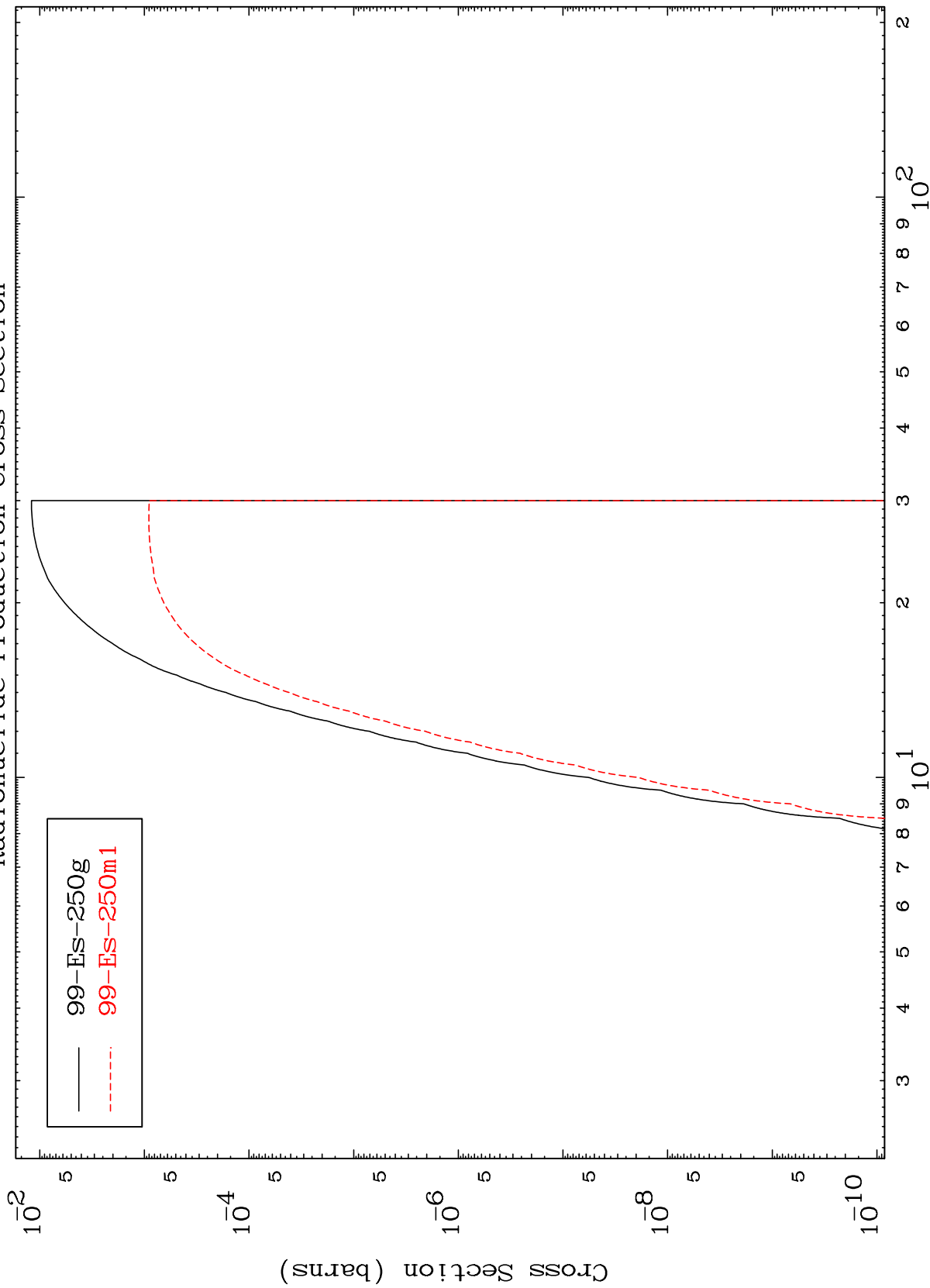
100-Fm-251

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(n,d)

100-Fm-251

Radionuclide Production Cross Section



30

Incident Energy (MeV)

100-Fm-251