

Program EVALPLOT
(Version 2015-2)

by

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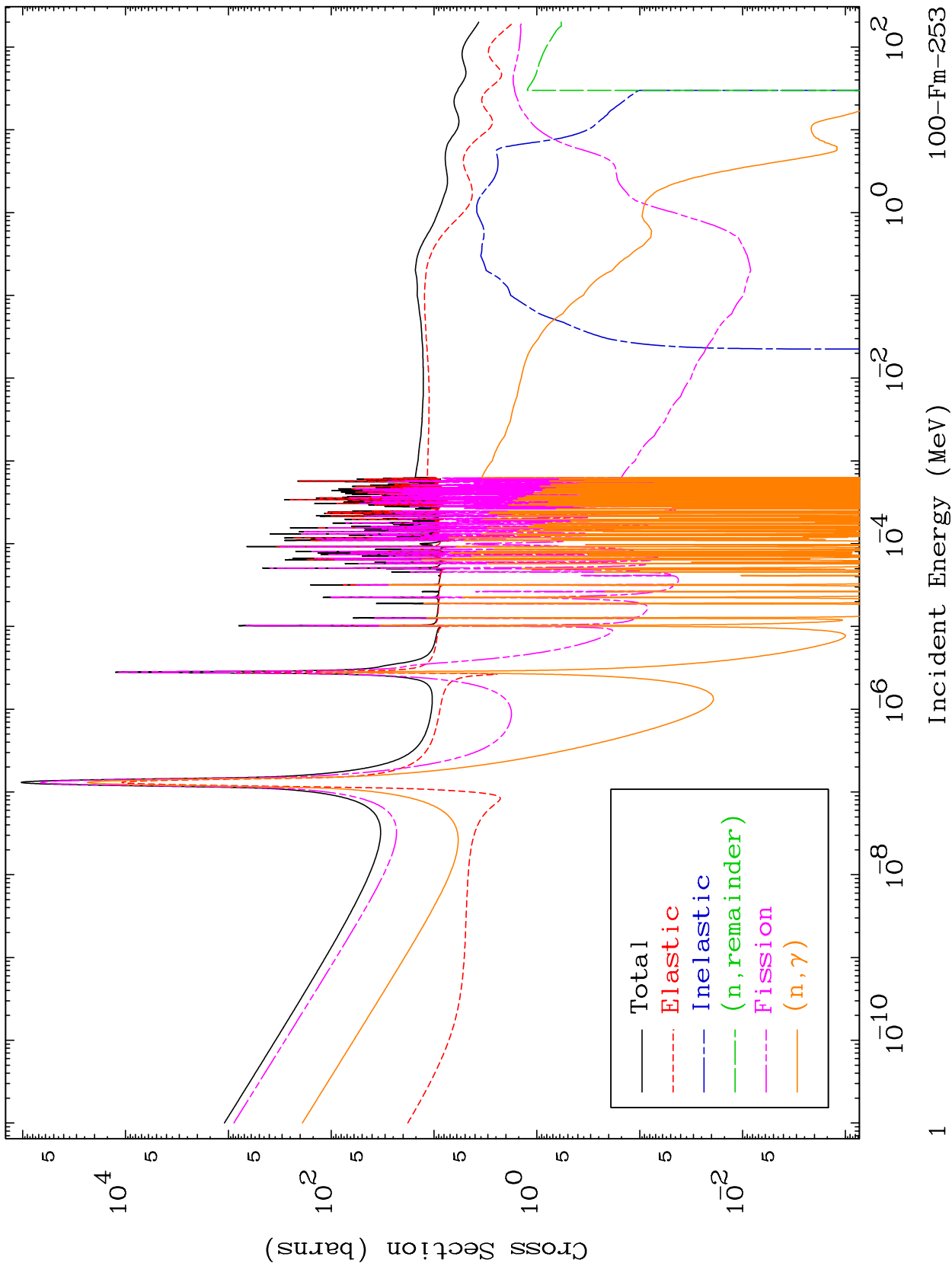
Press Mouse Button to Start

MAT 9934

Major

293 Kelvin Cross Sections

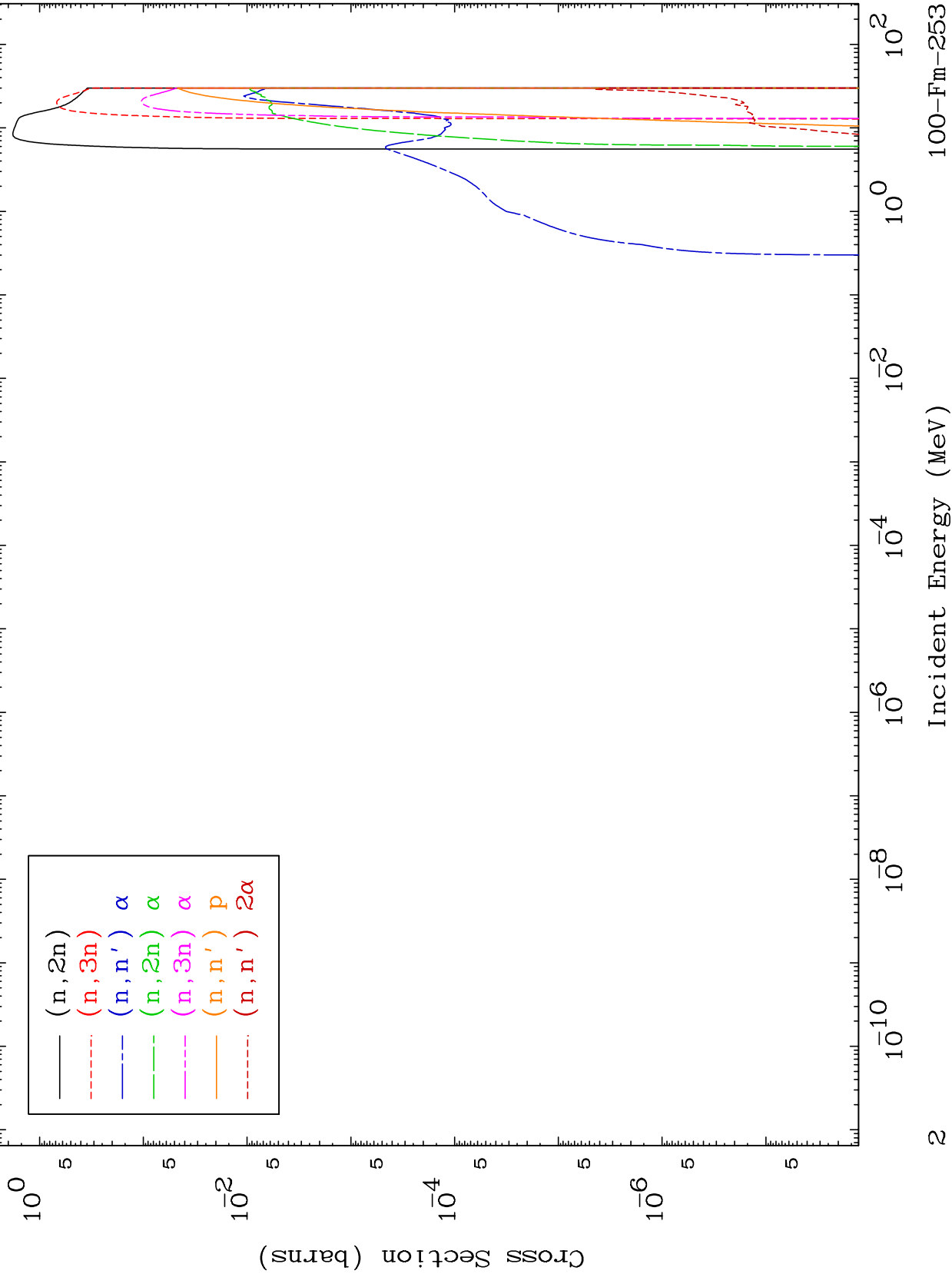
100-Fm-253



MAT 9934

Neutron Production
293 Kelvin Cross Sections

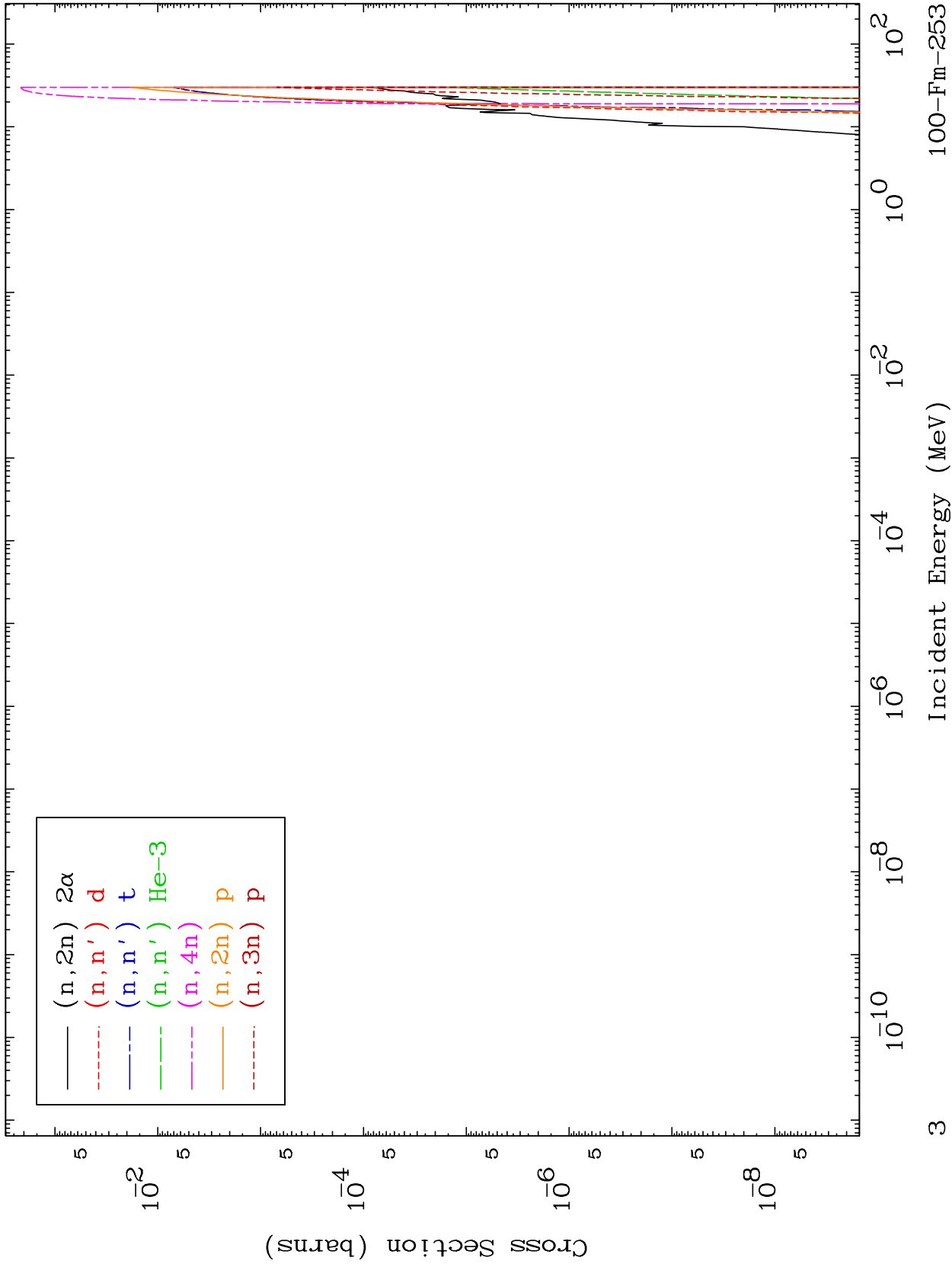
100-Fm-253



MAT 9934

Neutron Production
293 Kelvin Cross Sections

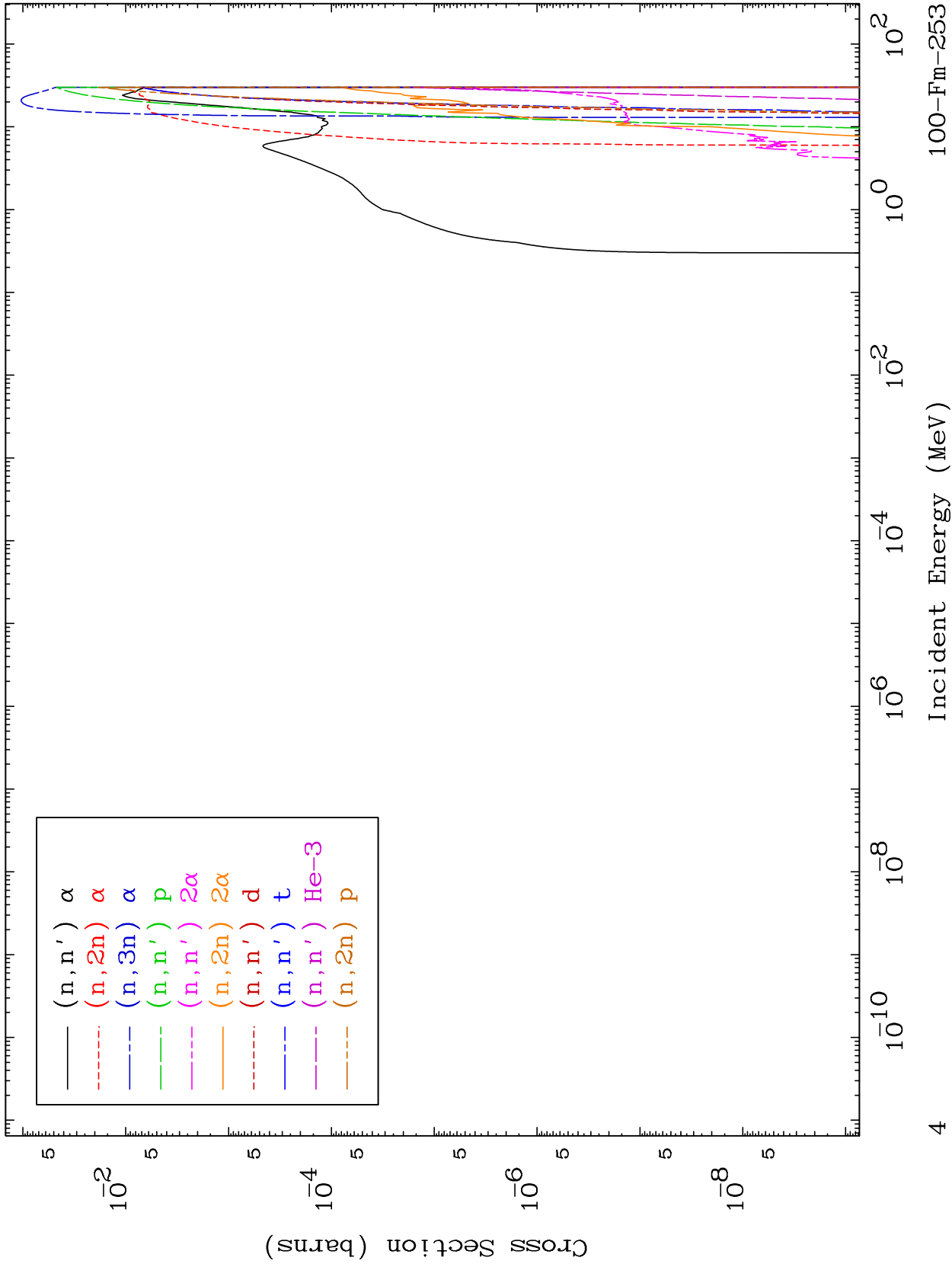
100-Fm-253



MAT 9934

Charged Particle
293 Kelvin Cross Sections

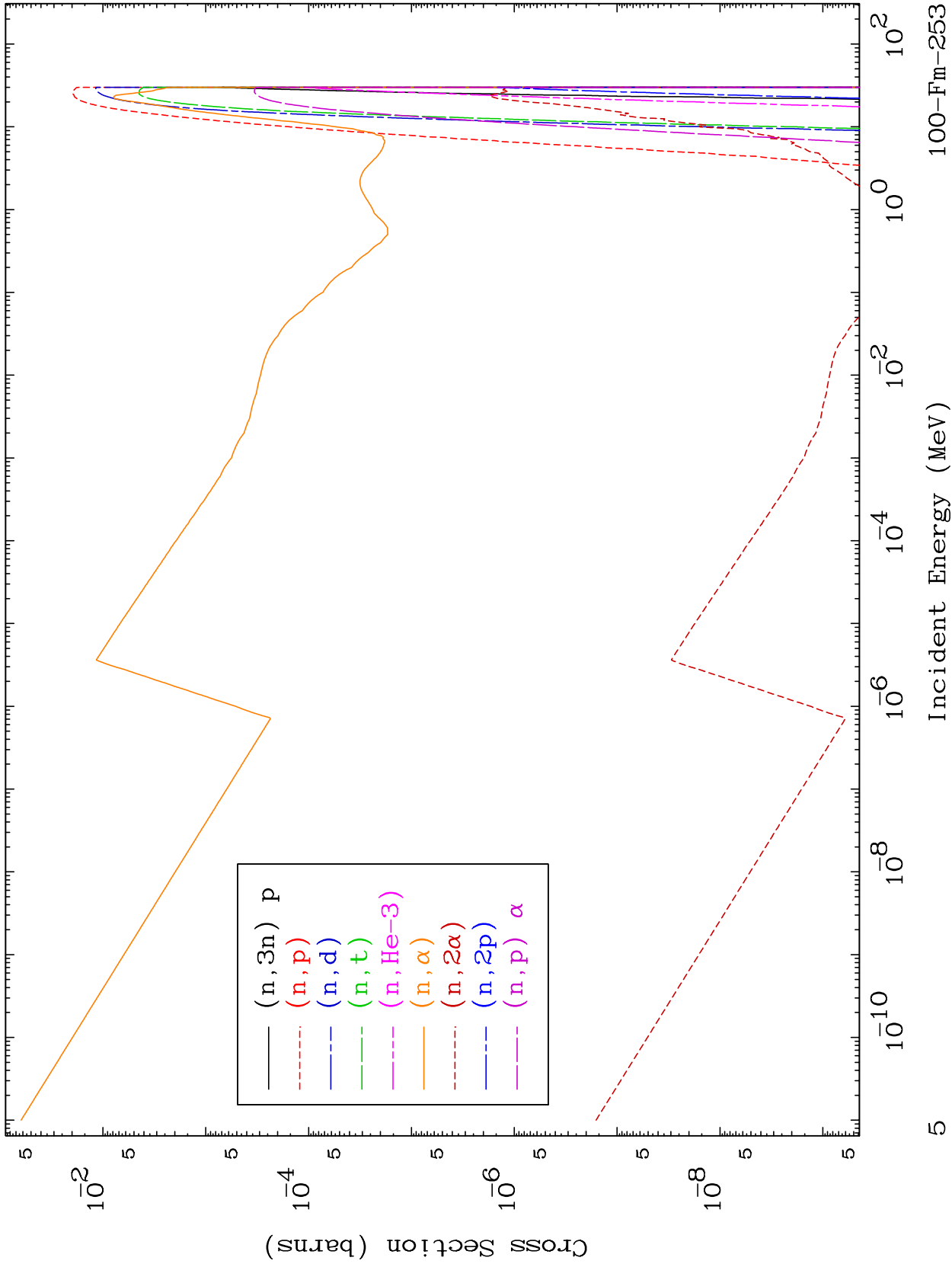
100-Fm-253



MAT 9934

Charged Particle
293 Kelvin Cross Sections

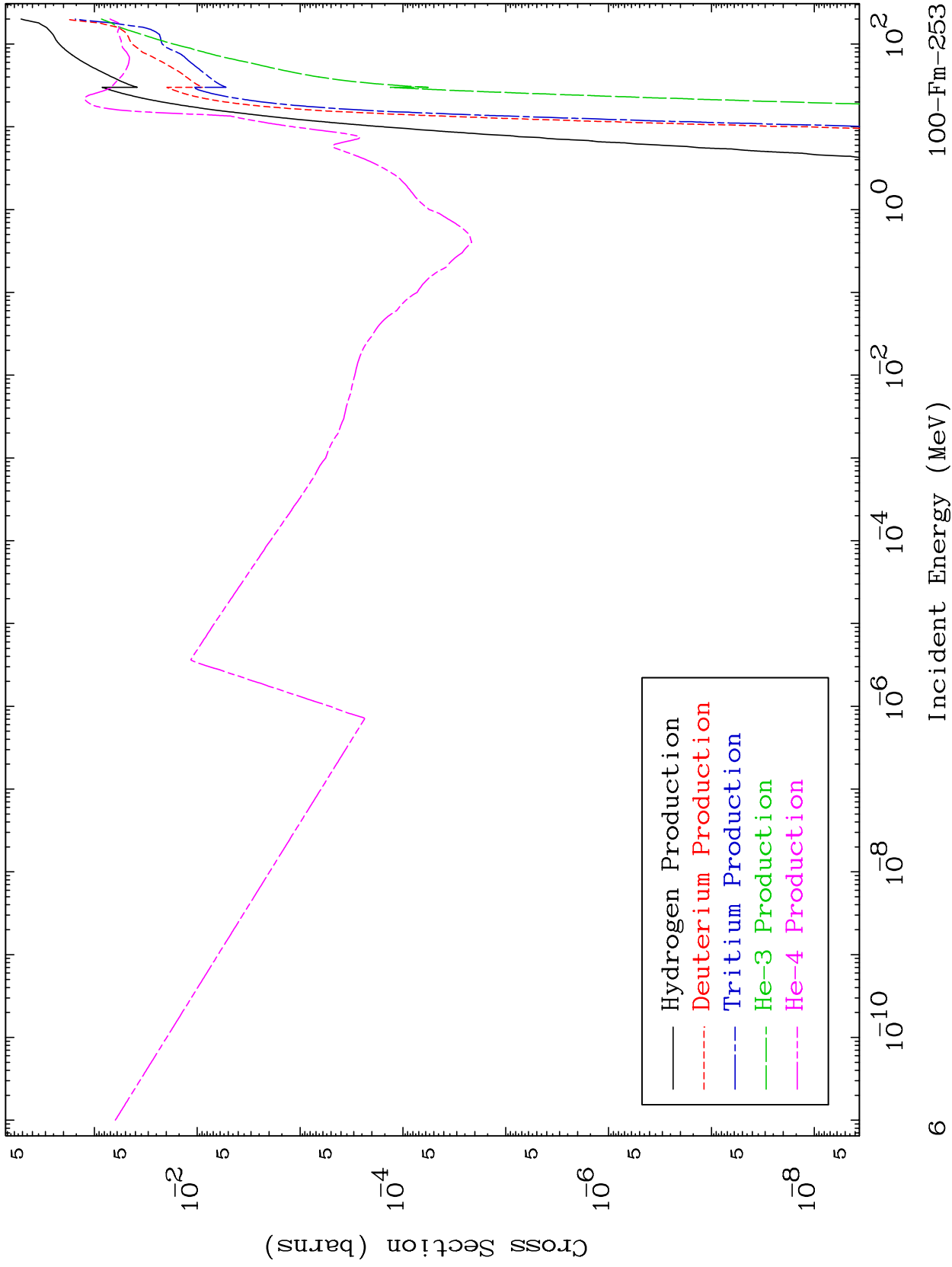
100-Fm-253



MAT 9934

Particle Production
293 Kelvin Cross Sections

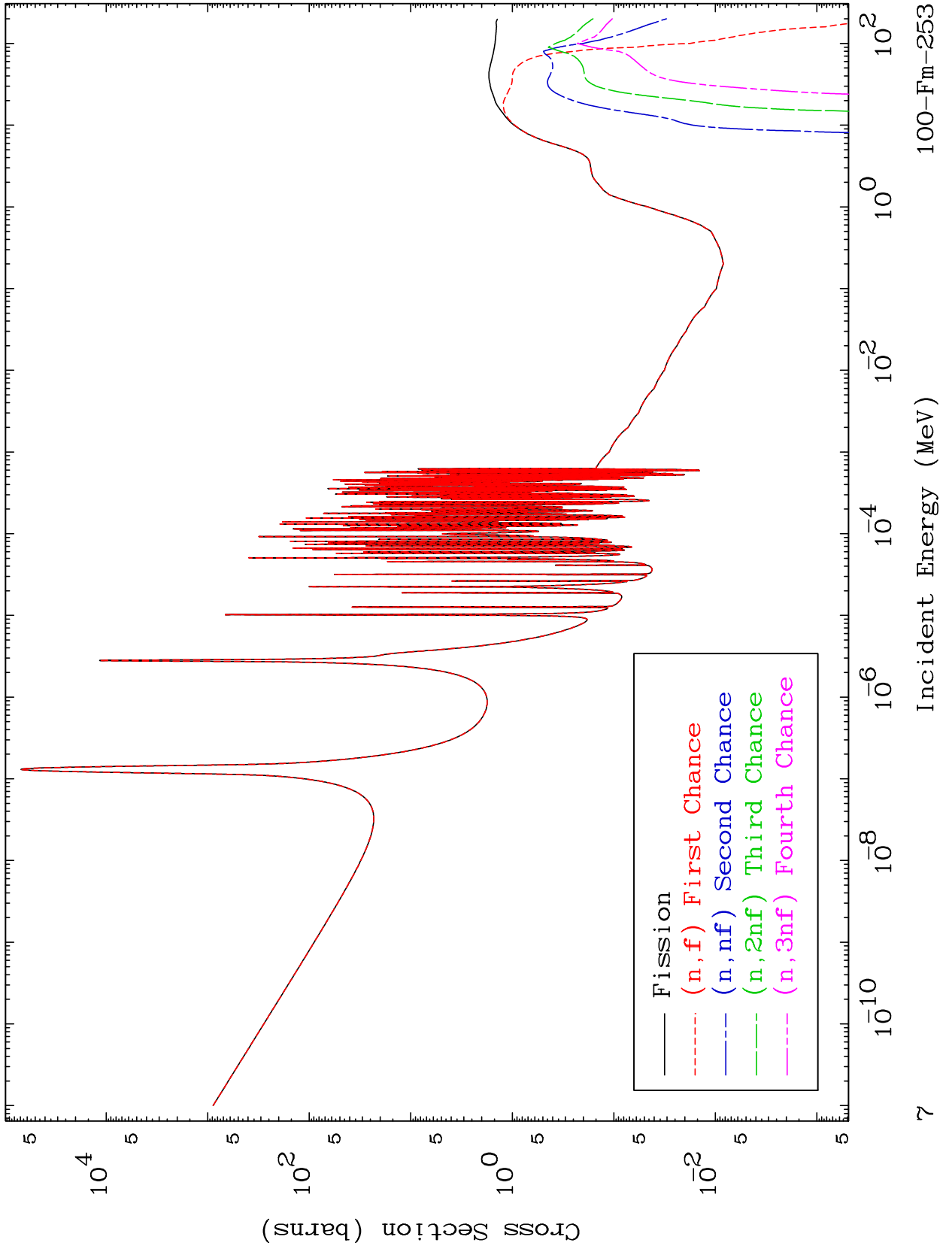
100-Fm-253

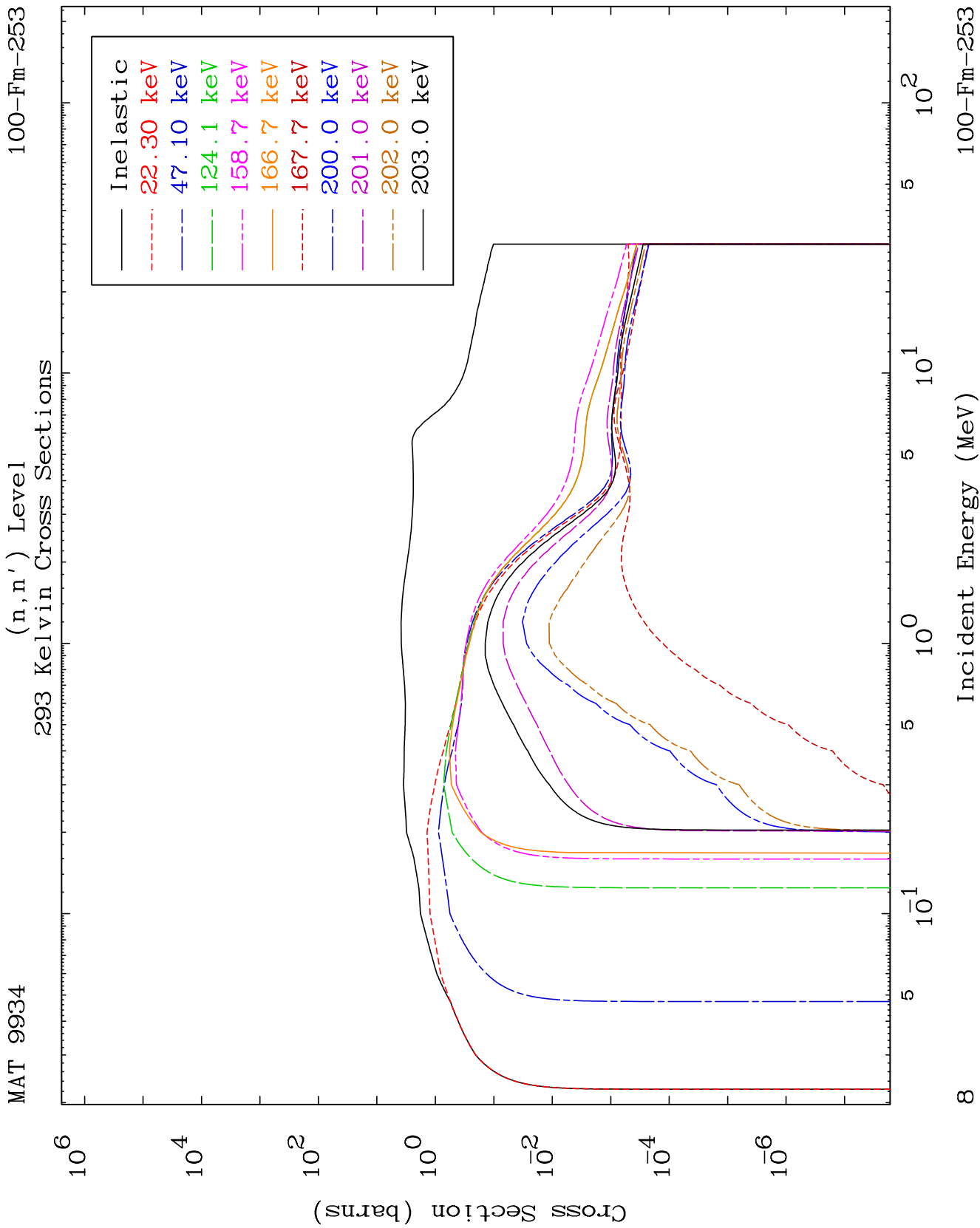


MAT 9934

Fission
293 Kelvin Cross Sections

100-Fm-253

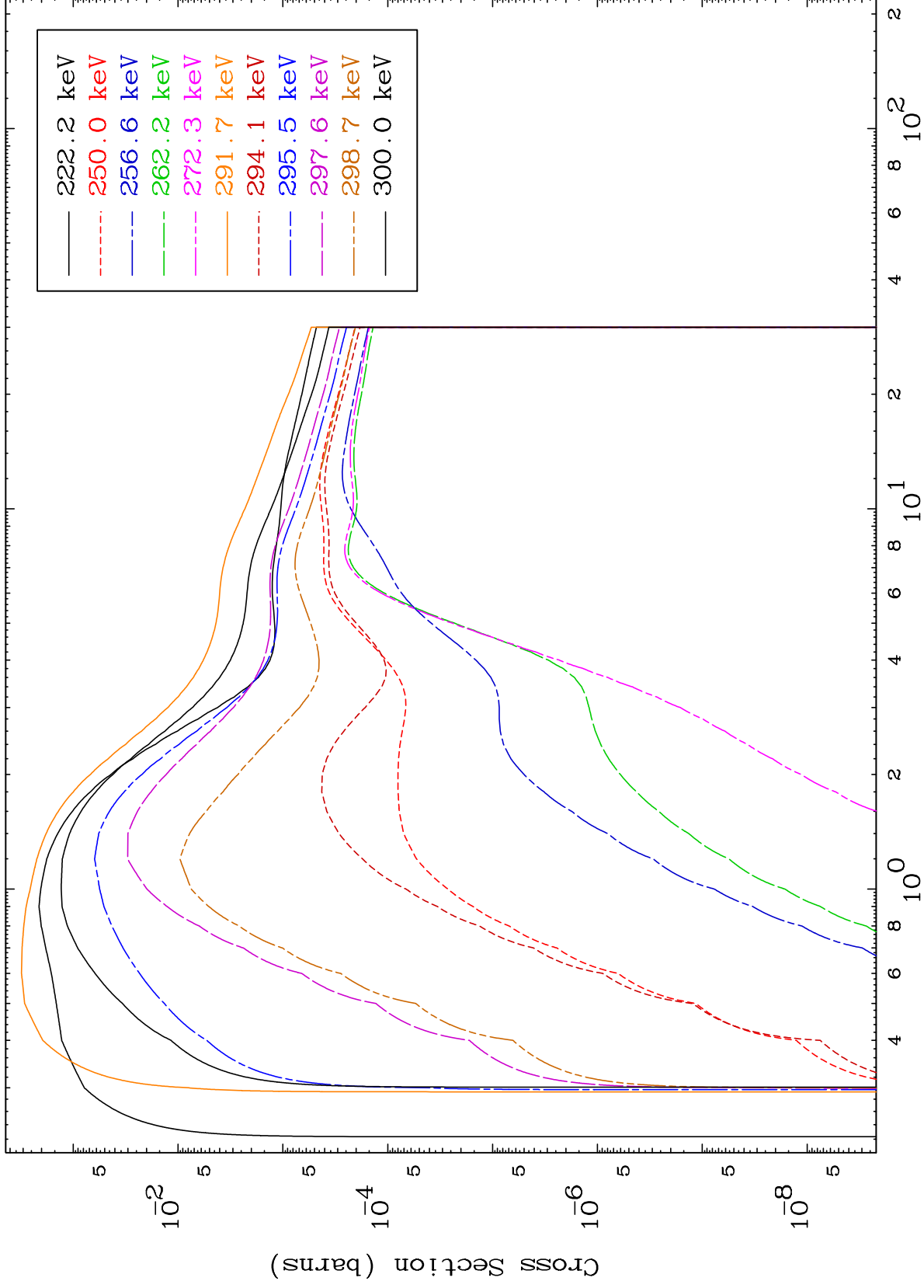




MAT 9934

(n,n') Level
293 Kelvin Cross Sections

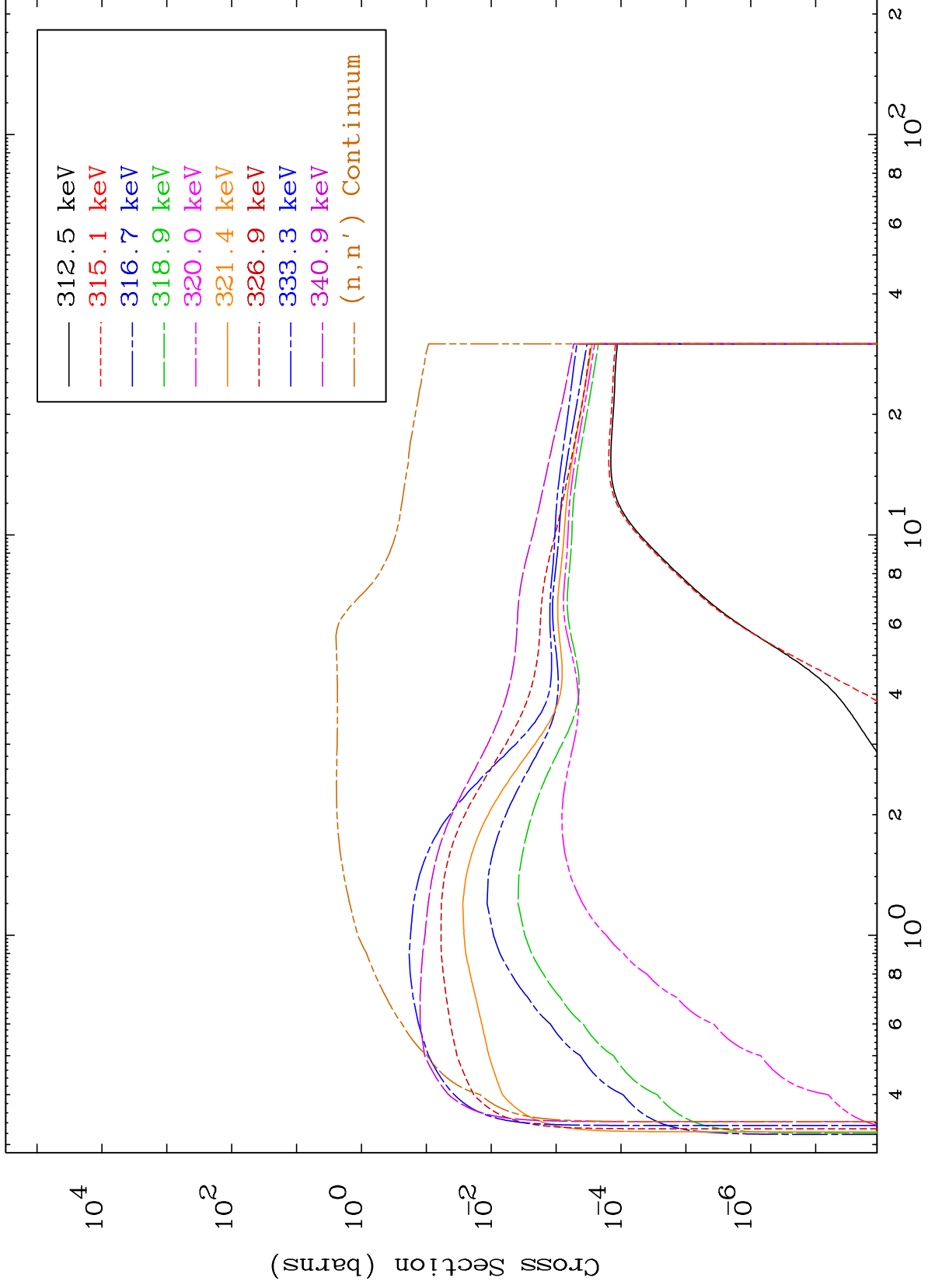
100-Fm-253



MAT 9934

(n,n') Level
293 Kelvin Cross Sections

100-Fm-253



10

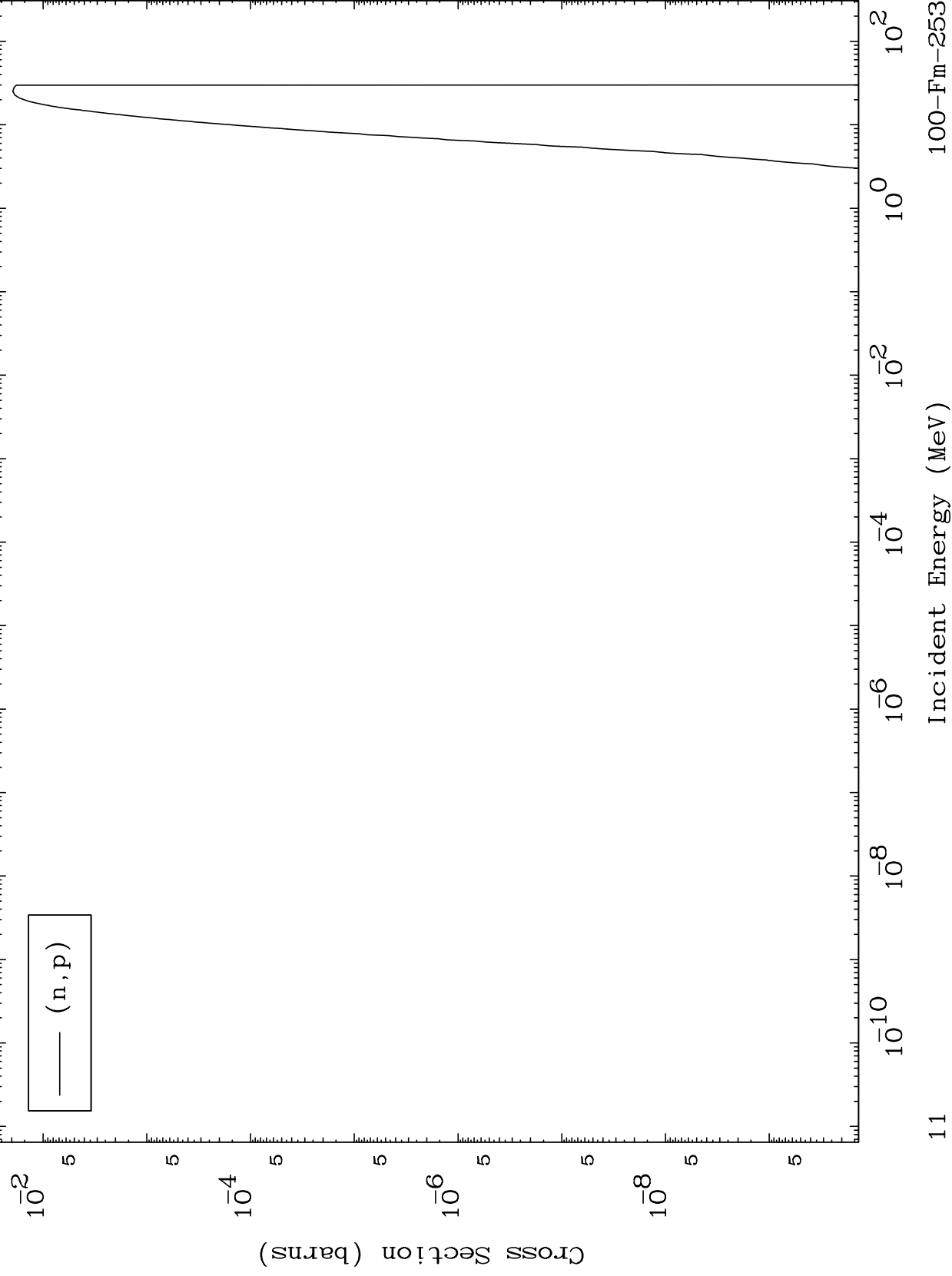
Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,p) Levels
293 Kelvin Cross Sections

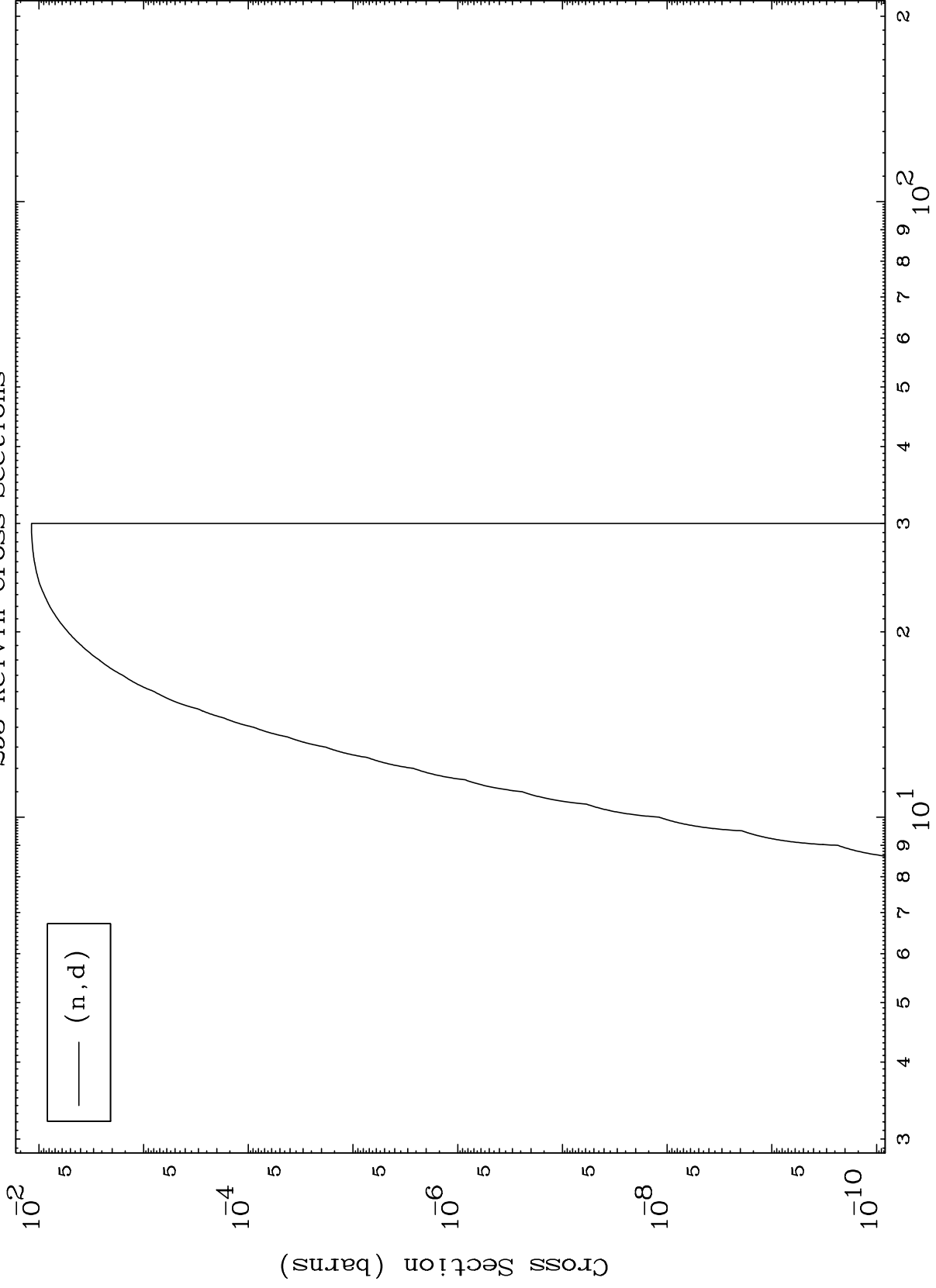
100-Fm-253



MAT 9934

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-253



12

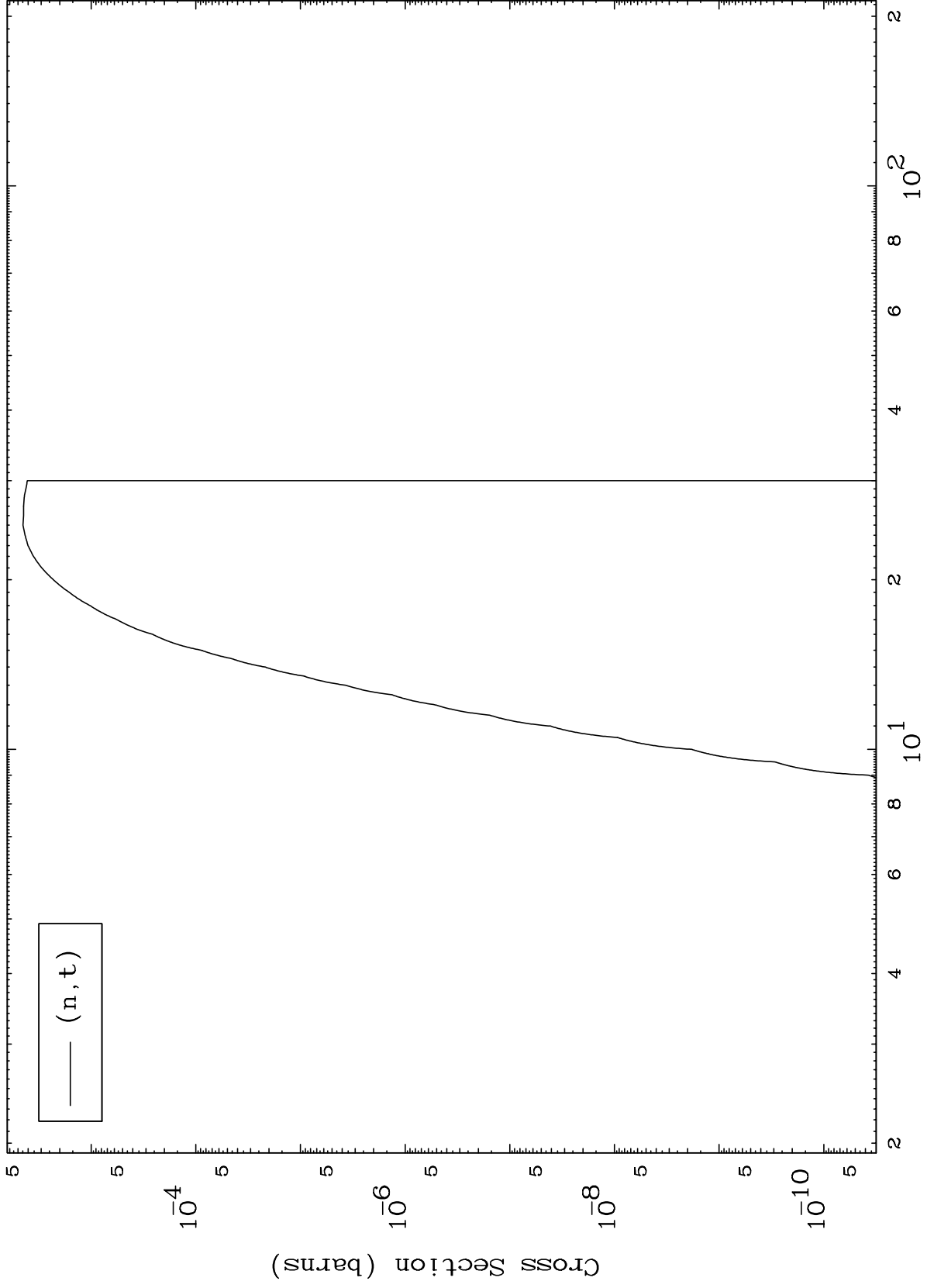
Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-253



13

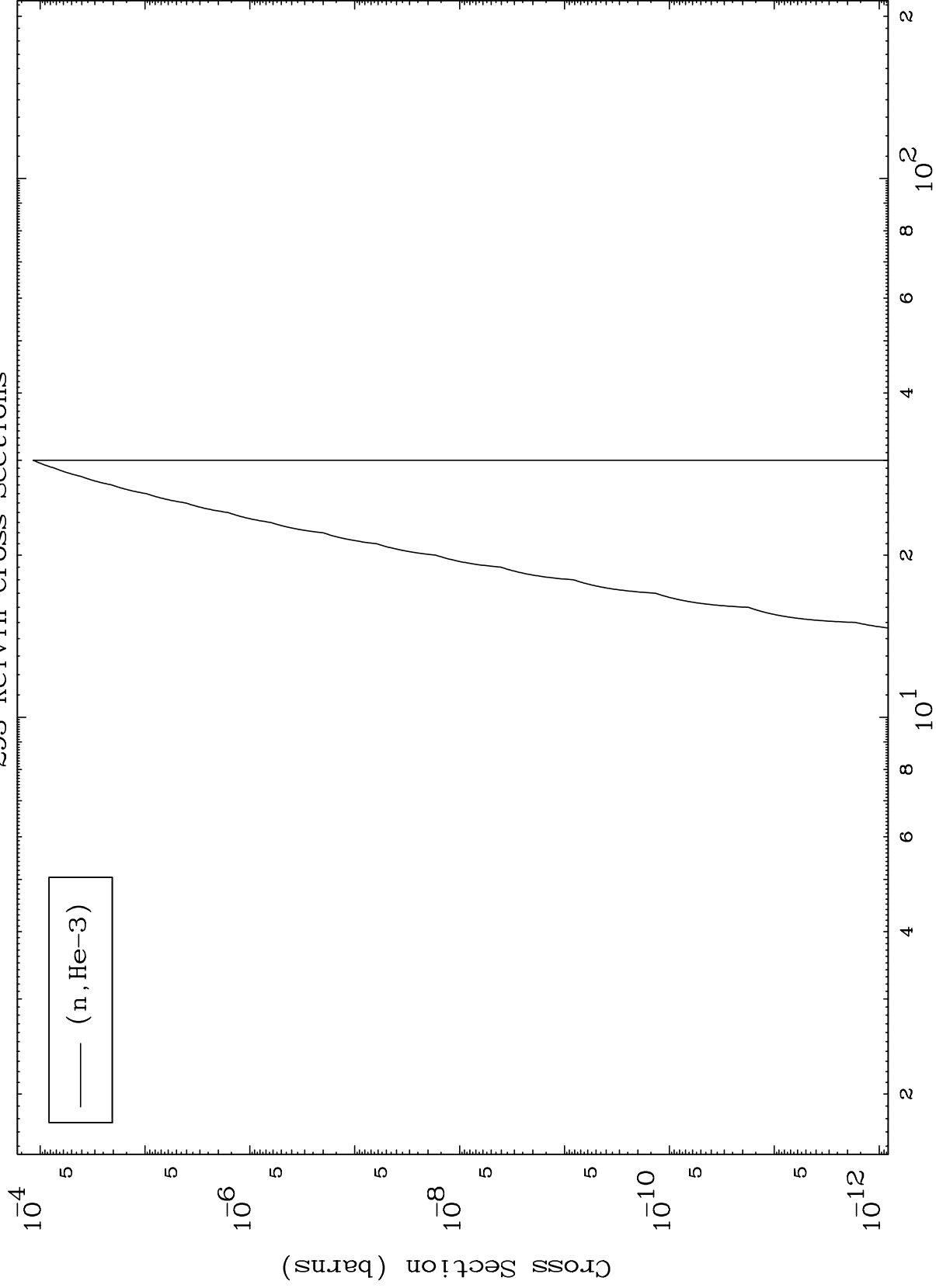
Incident Energy (MeV)

100-Fm-253

MAT 9934

(n,He3) Levels
293 Kelvin Cross Sections

100-Fm-253



14

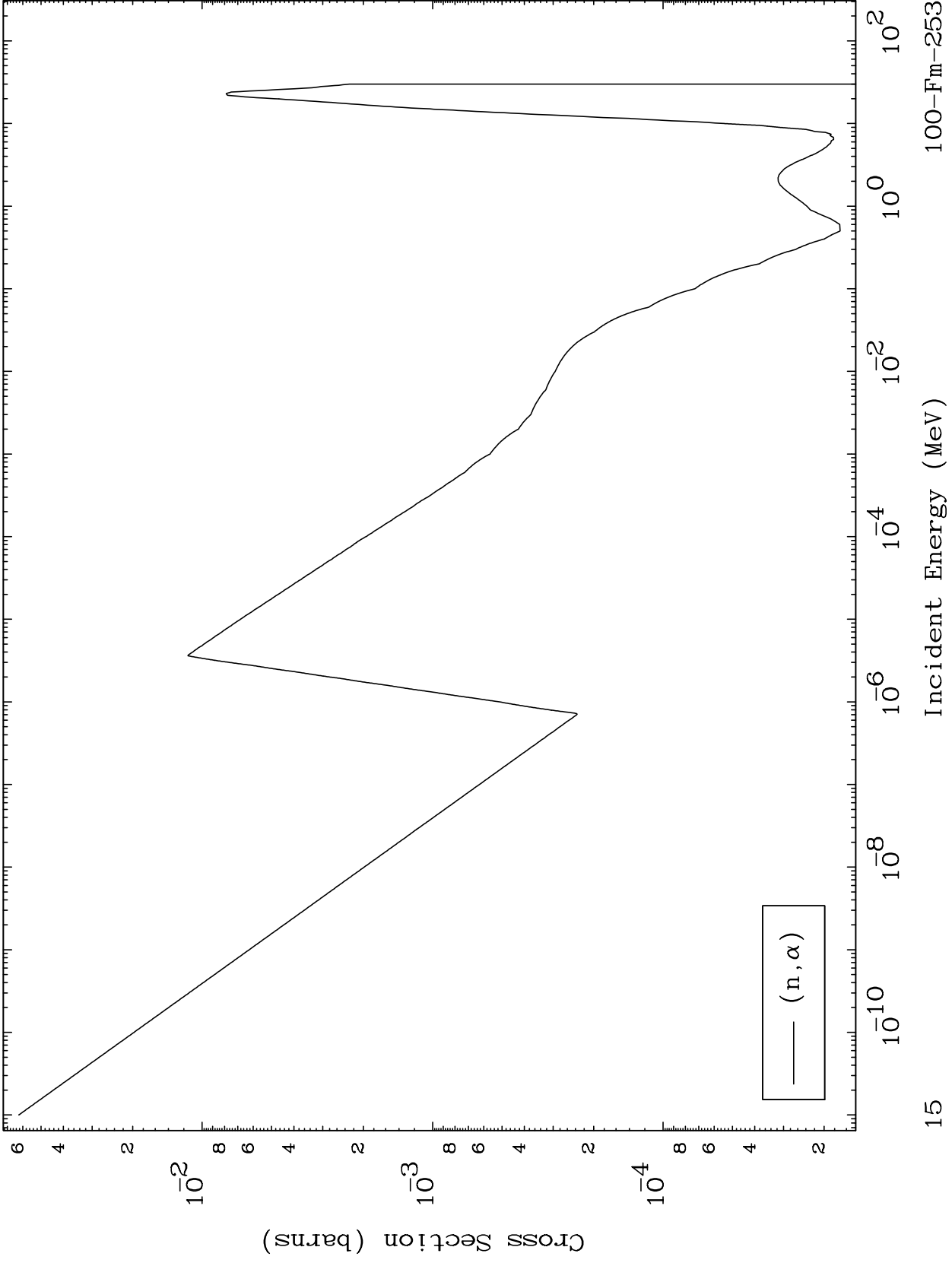
Incident Energy (MeV)

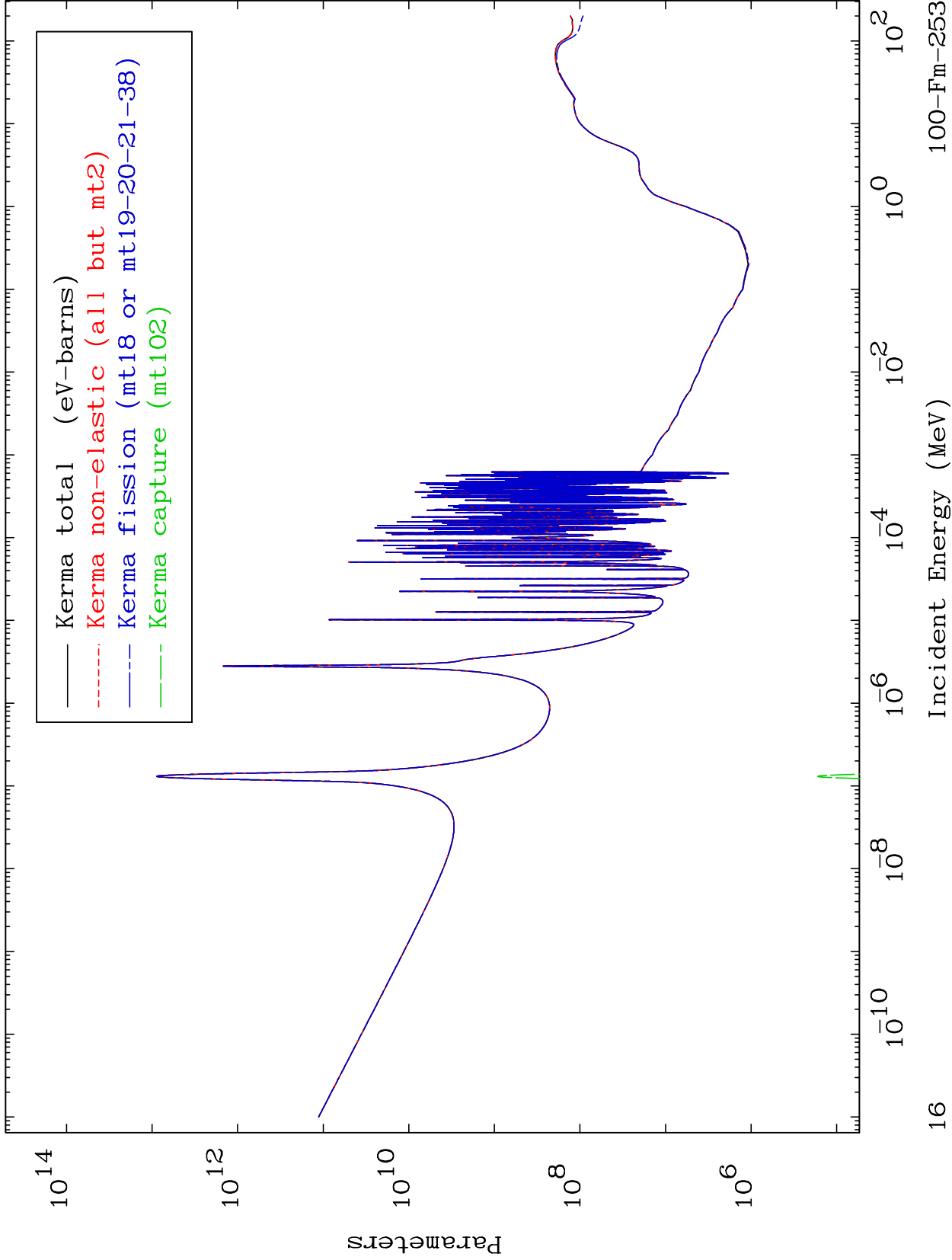
100-Fm-253

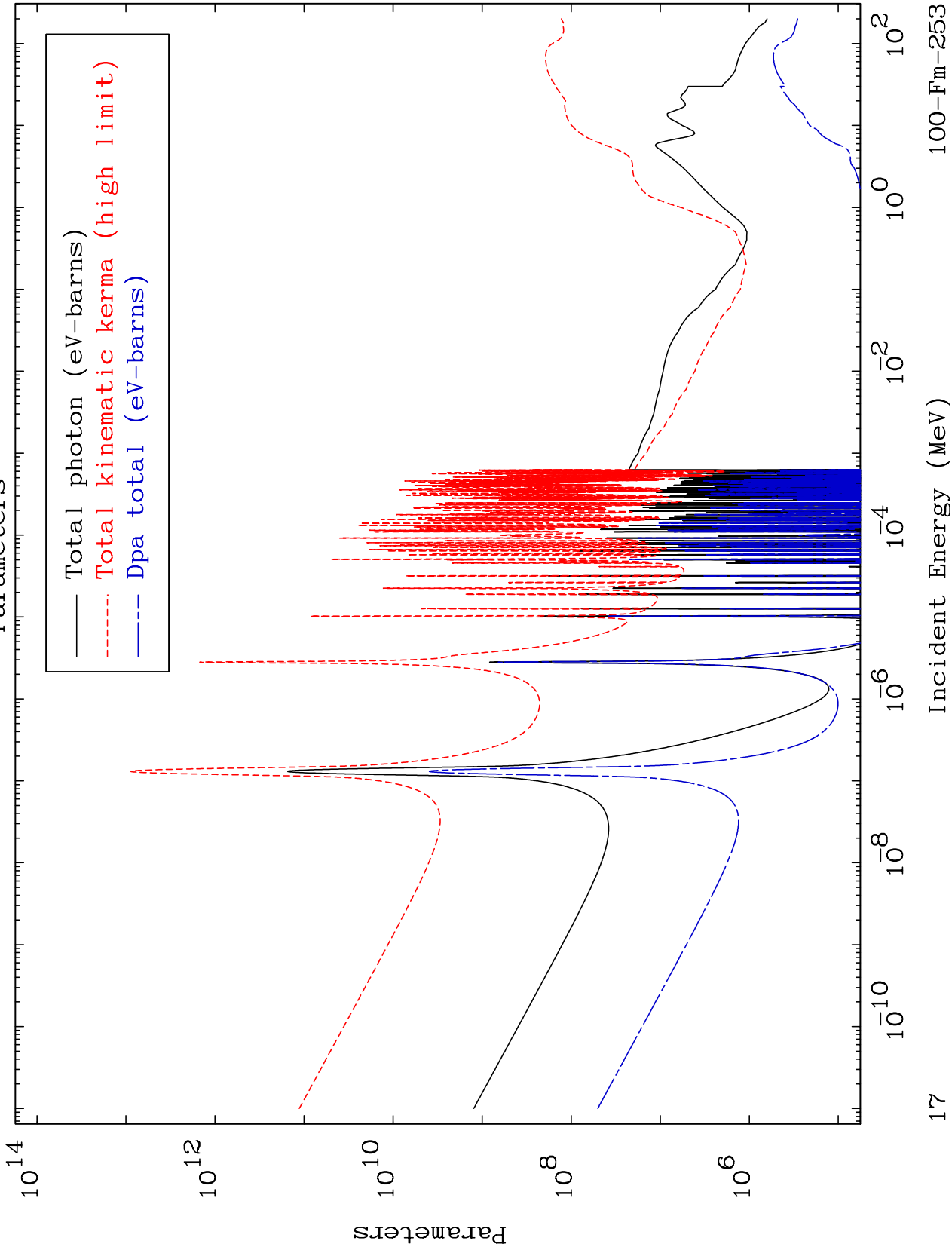
MAT 9934

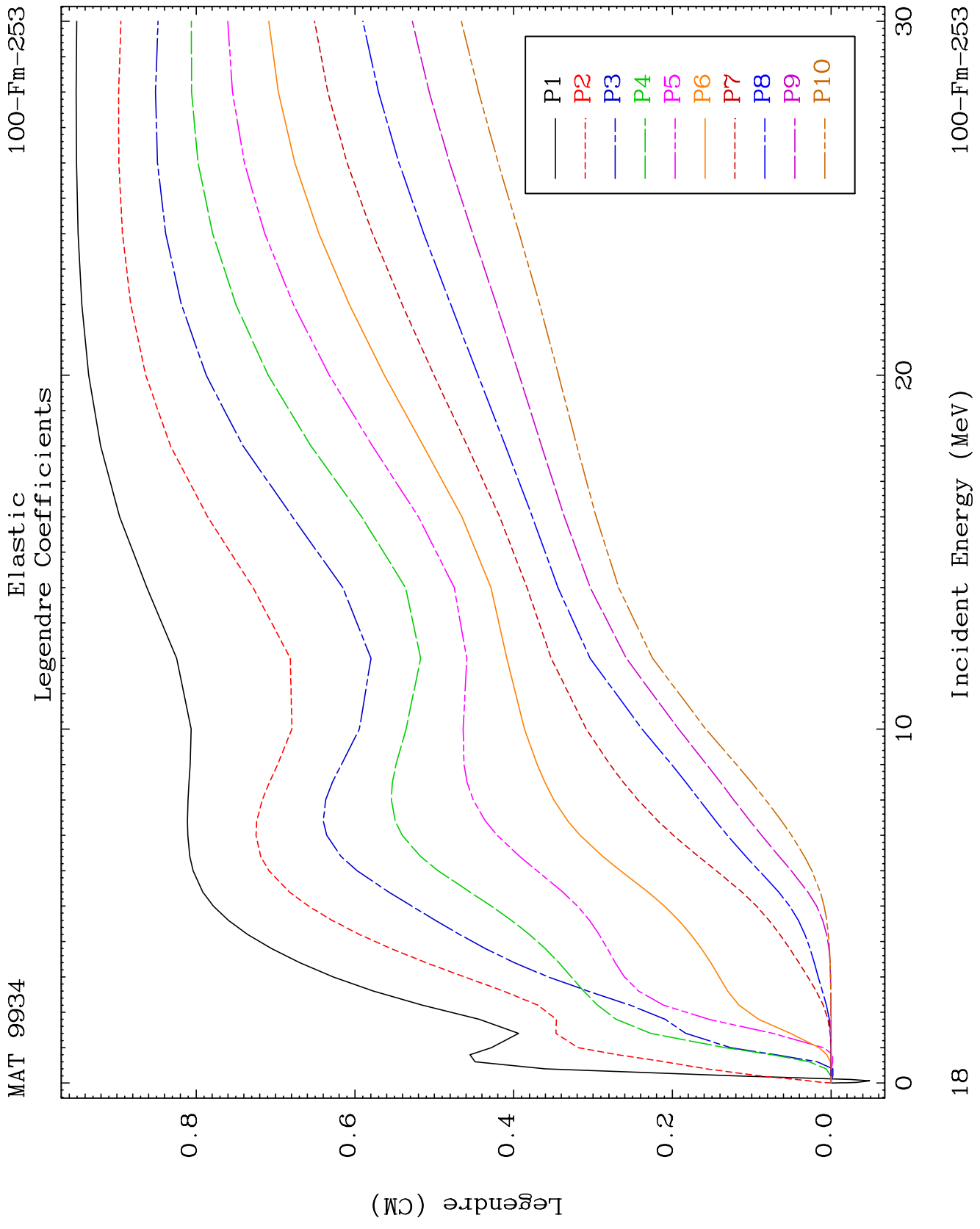
(n,α) Levels
293 Kelvin Cross Sections

100-Fm-253





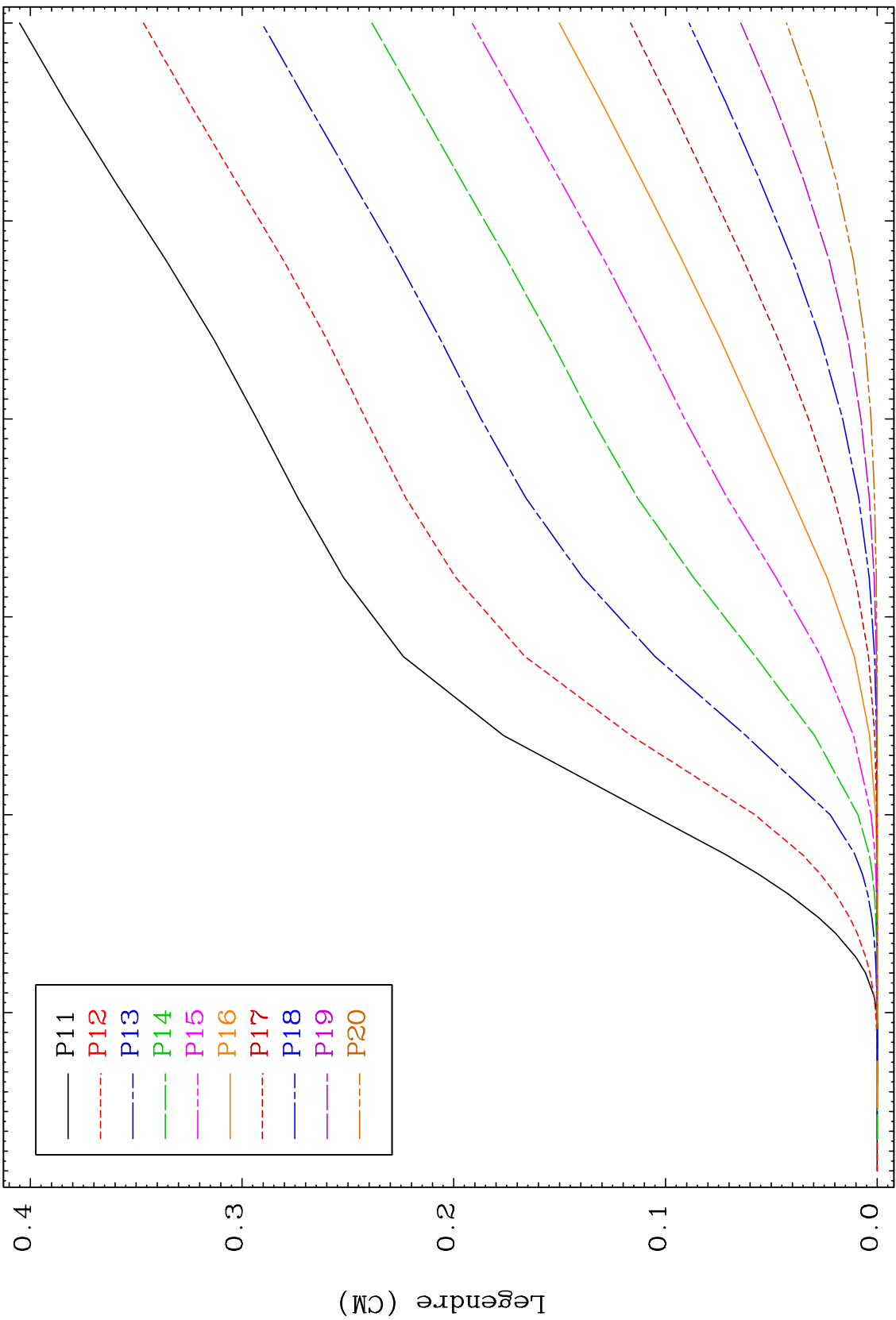




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Elastic Legendre Coefficients

100-Fm-253



19

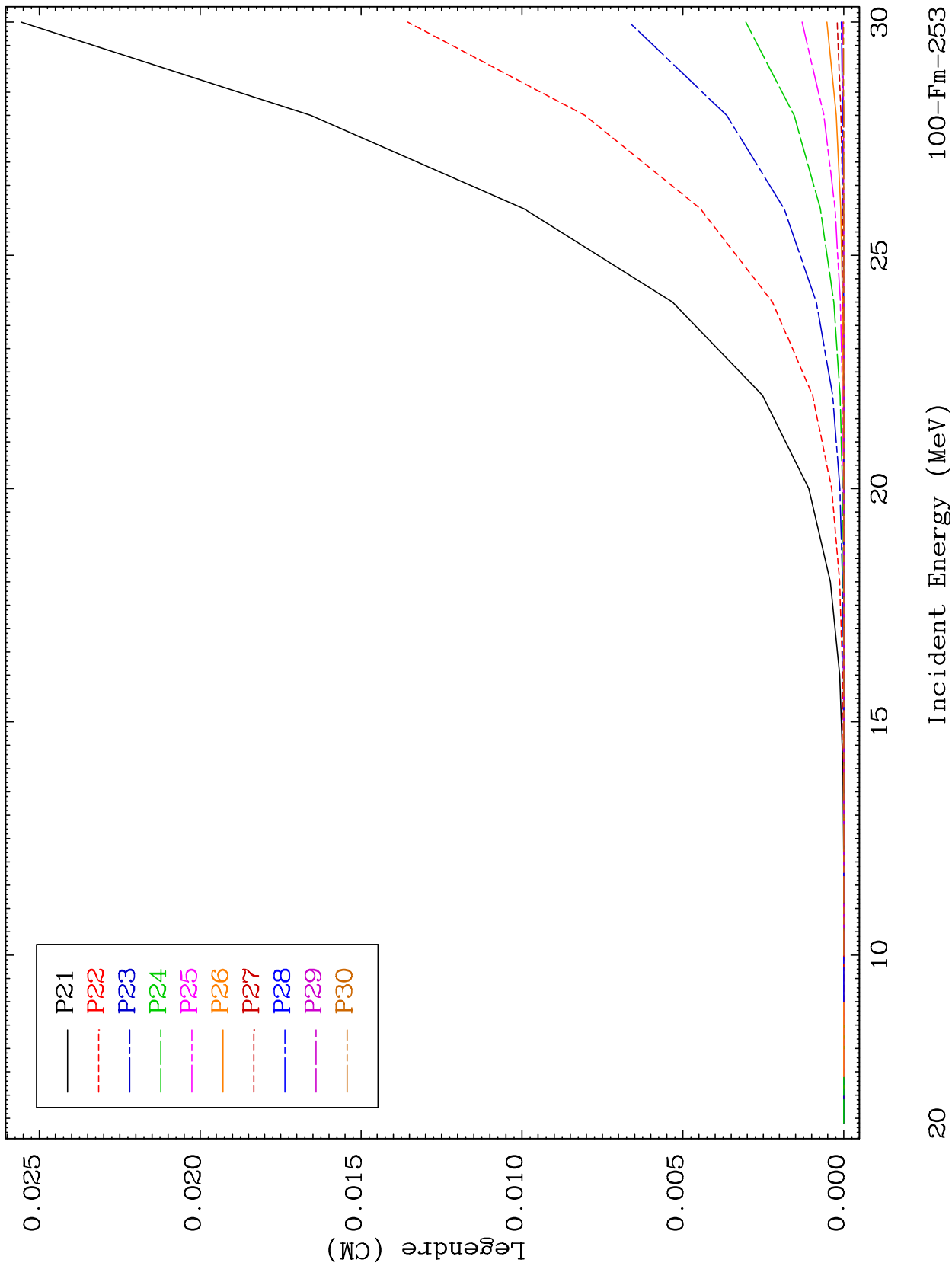
Incident Energy (MeV)

100-Fm-253

MAT 9934

Elastic Legendre Coefficients

100-Fm-253



20

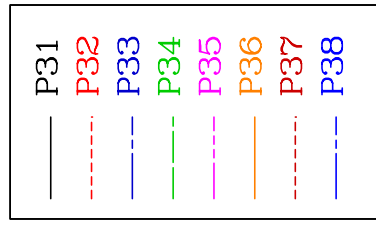
Incident Energy (MeV)

100-Fm-253

MAT 9934

Elastic Legendre Coefficients

100-Fm-253



$\times 10^{-63}$

Legendre (CM)

2

1

0

15

20

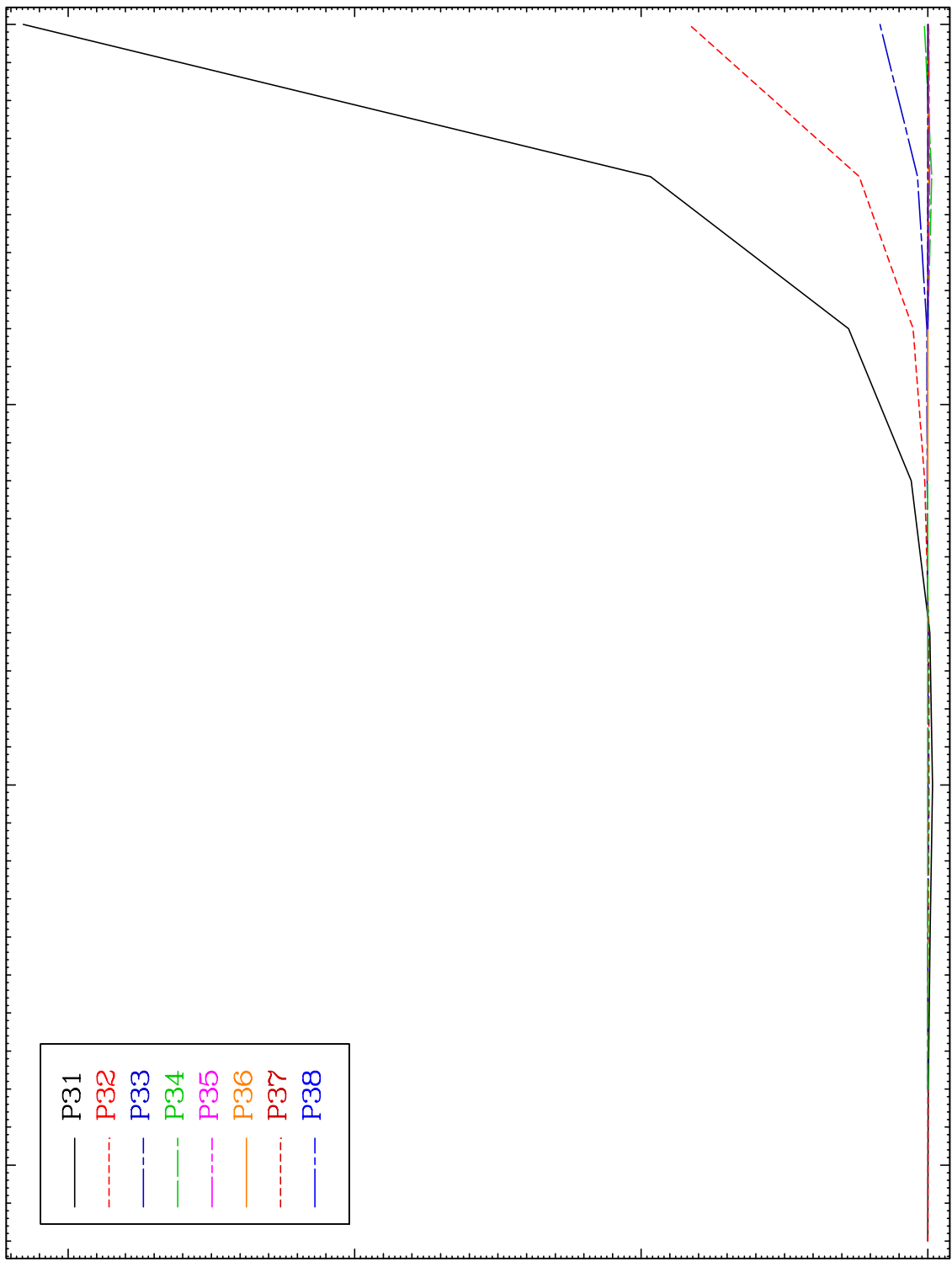
25

30

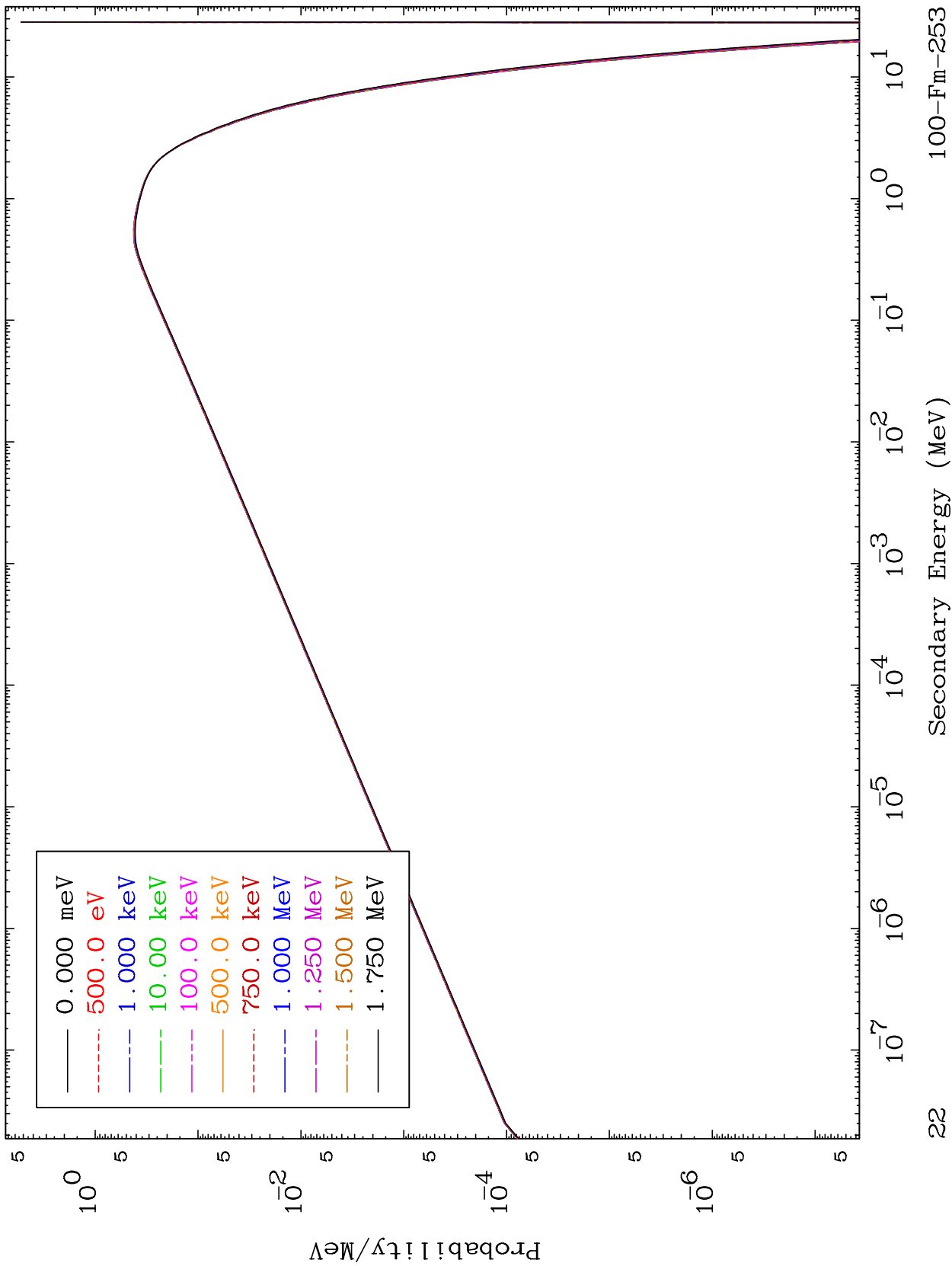
21

Incident Energy (MeV)

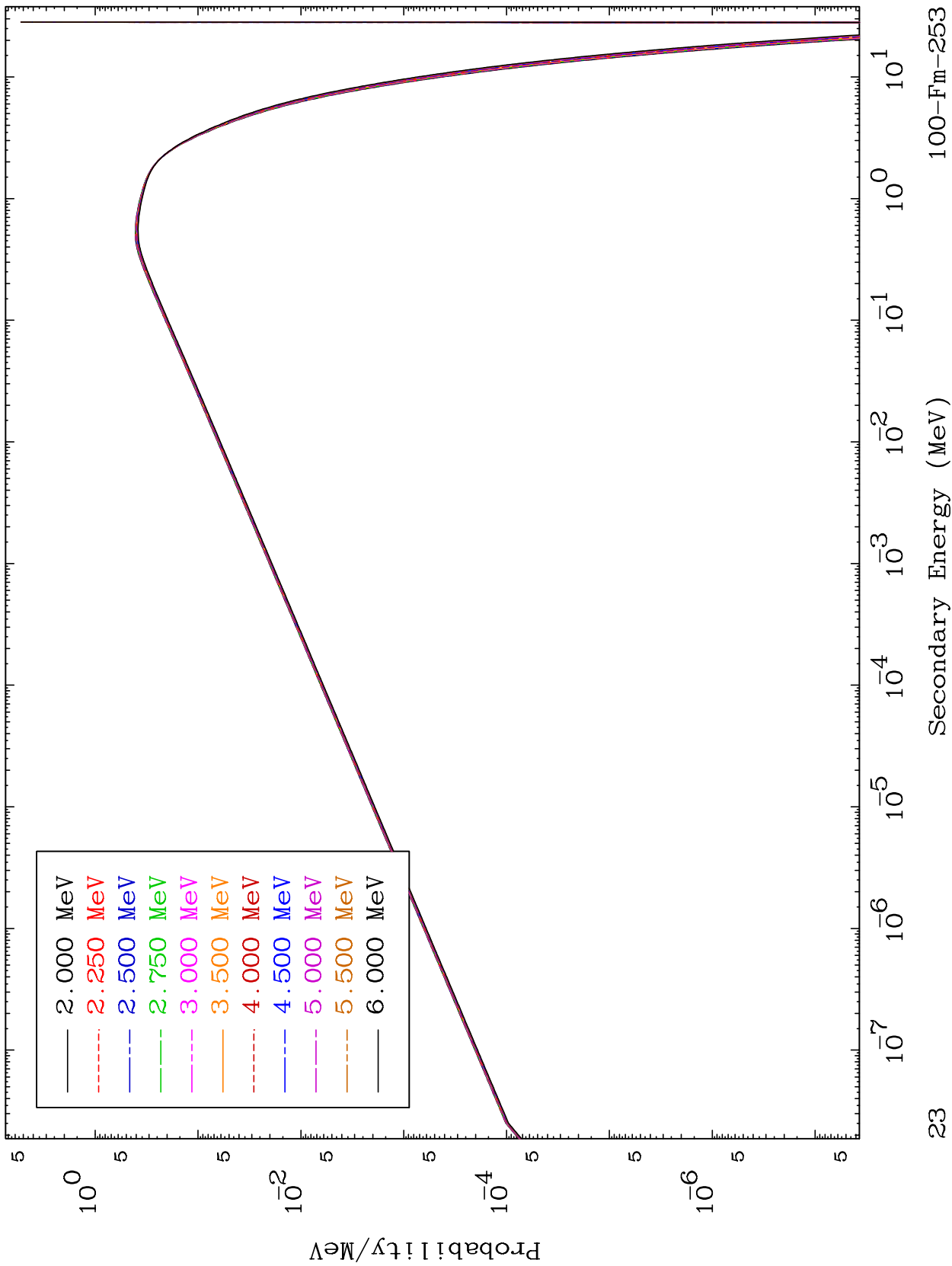
100-Fm-253



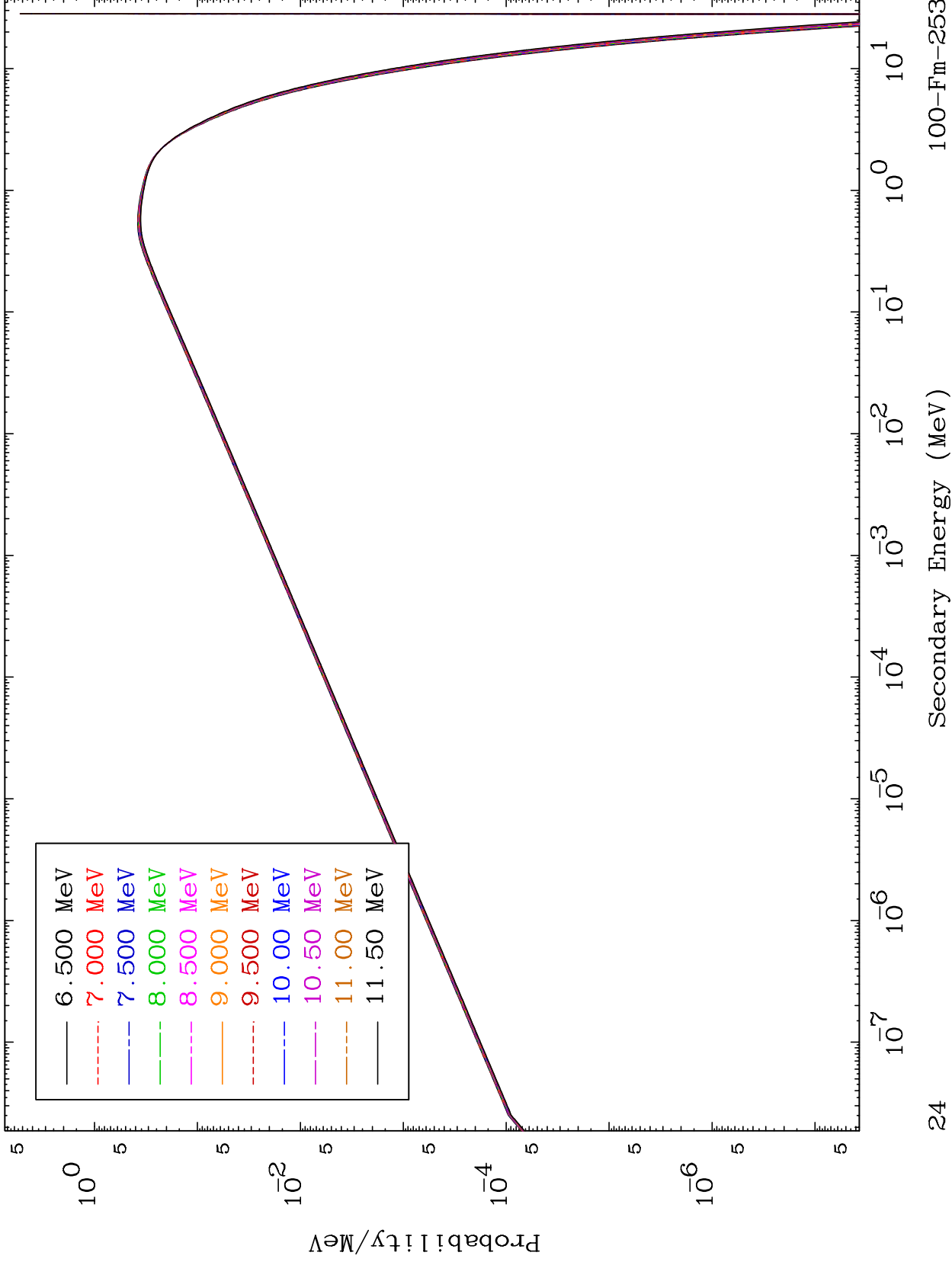
Fission Energy Distribution

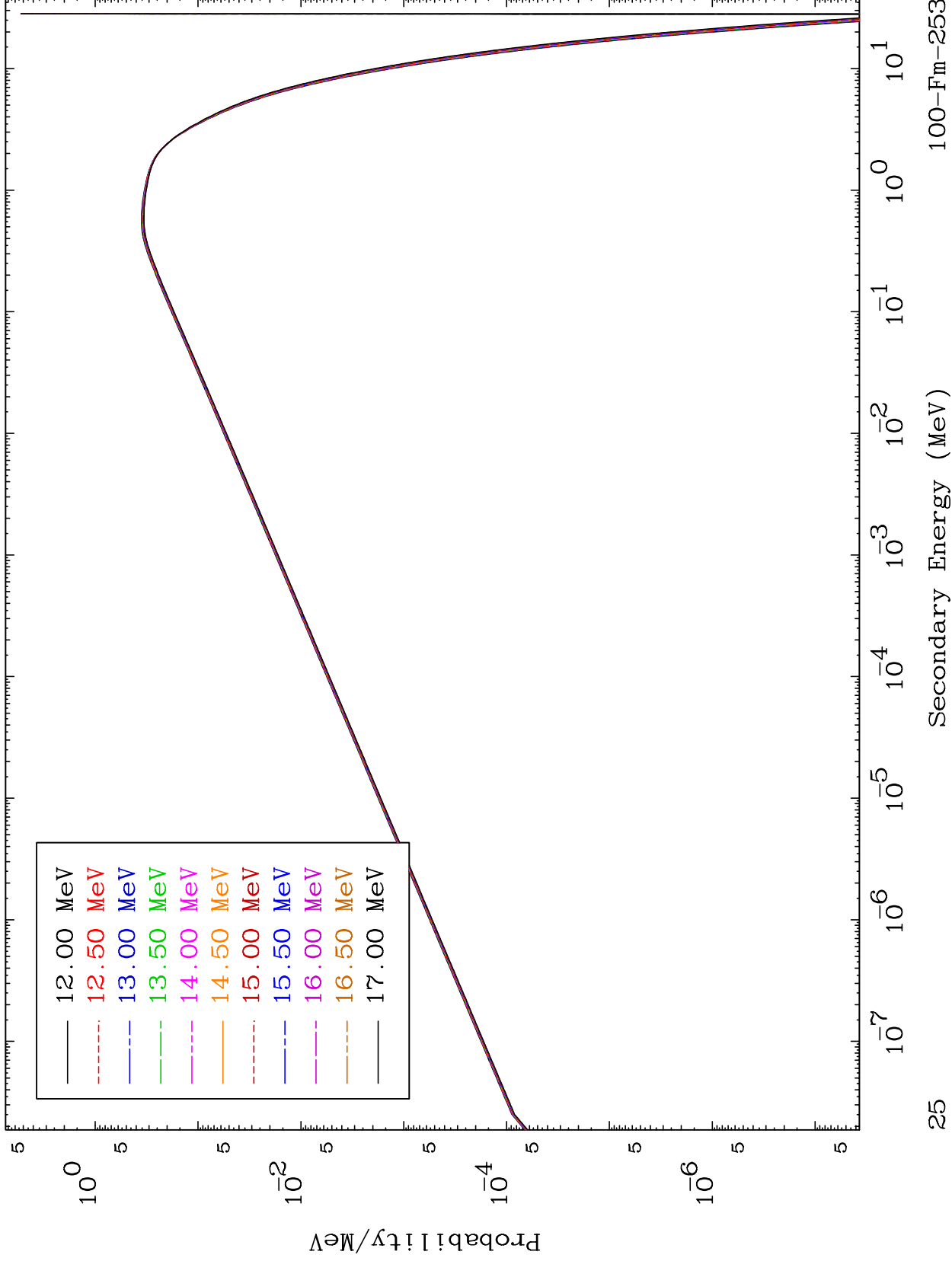


Fission
Energy Distribution

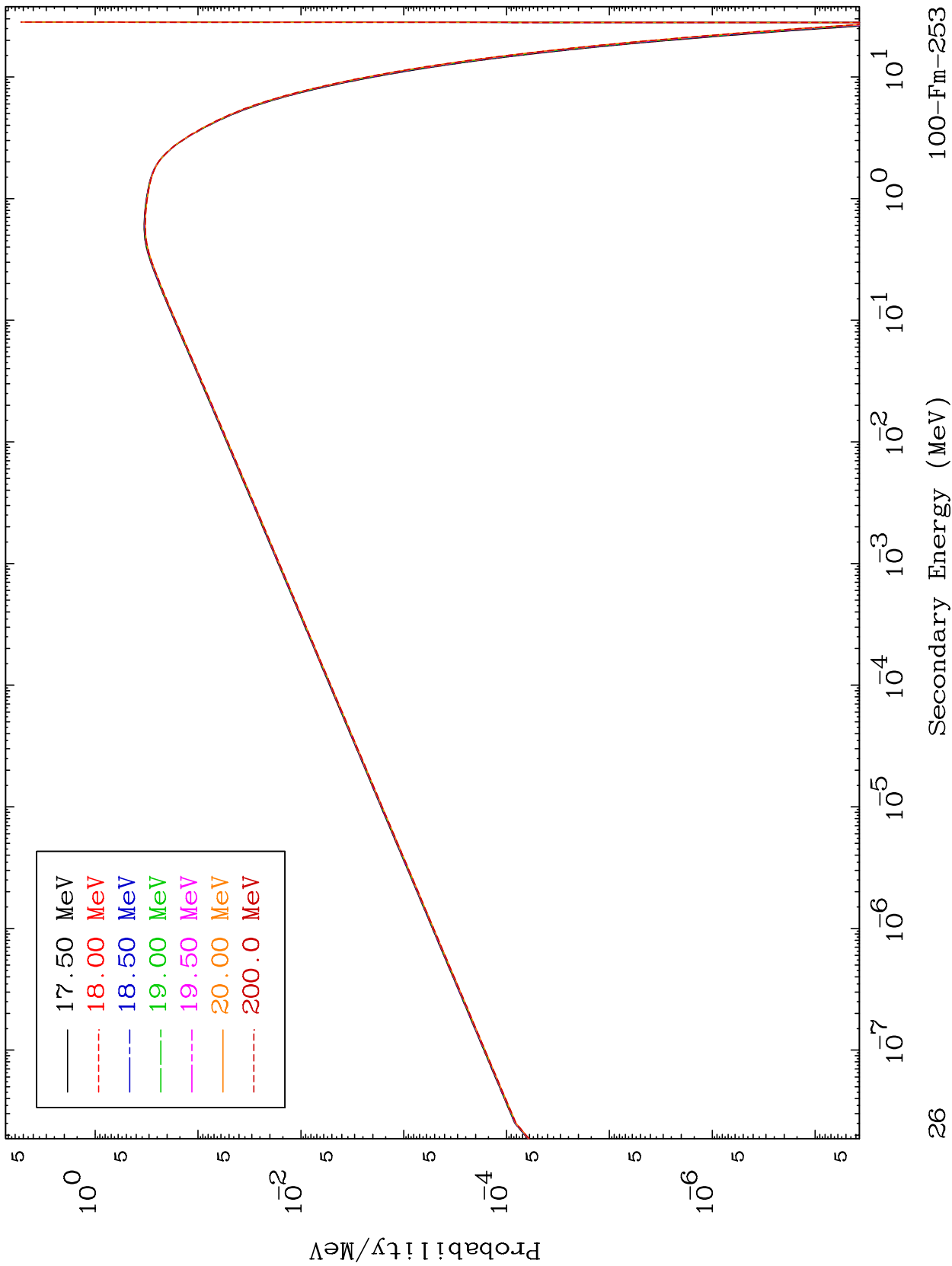


Fission
Energy Distribution





Fission
Energy Distribution

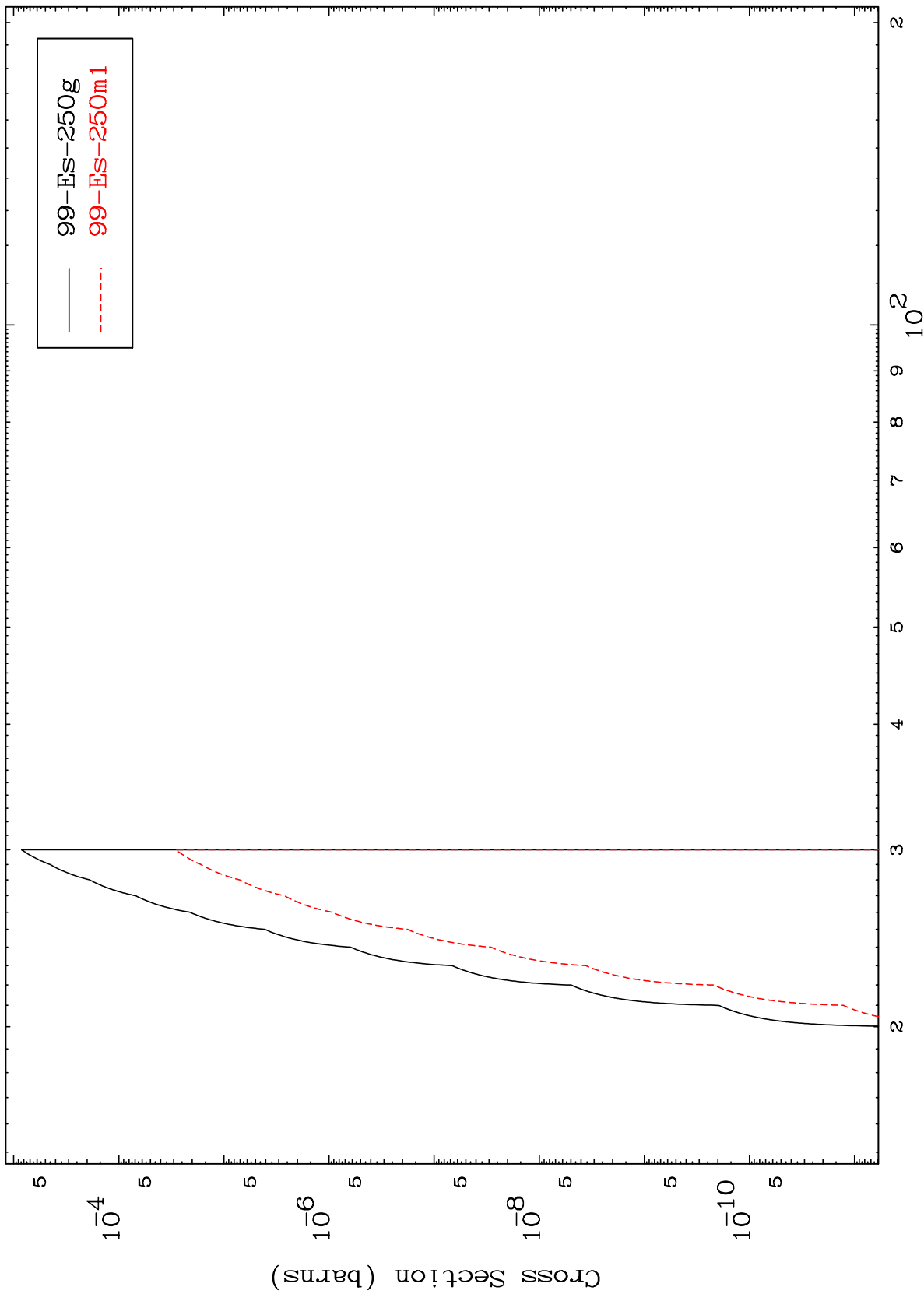


MAT 9934

(n,2n) d

100-Fm-253

Radionuclide Production Cross Section



27

Incident Energy (MeV)

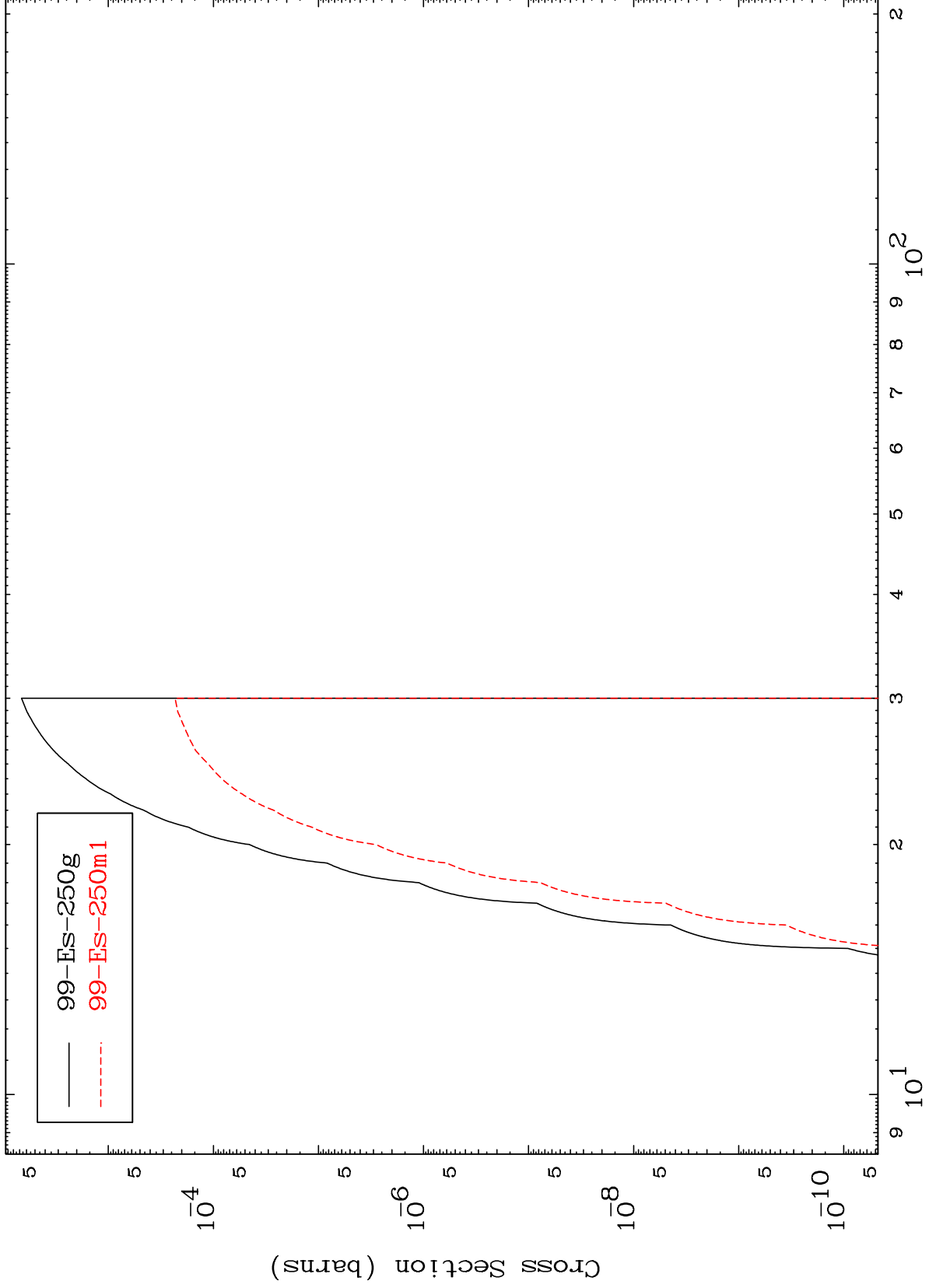
100-Fm-253

MAT 9934

(n,n') t

100-Fm-253

Radionuclide Production Cross Section



28

Incident Energy (MeV)

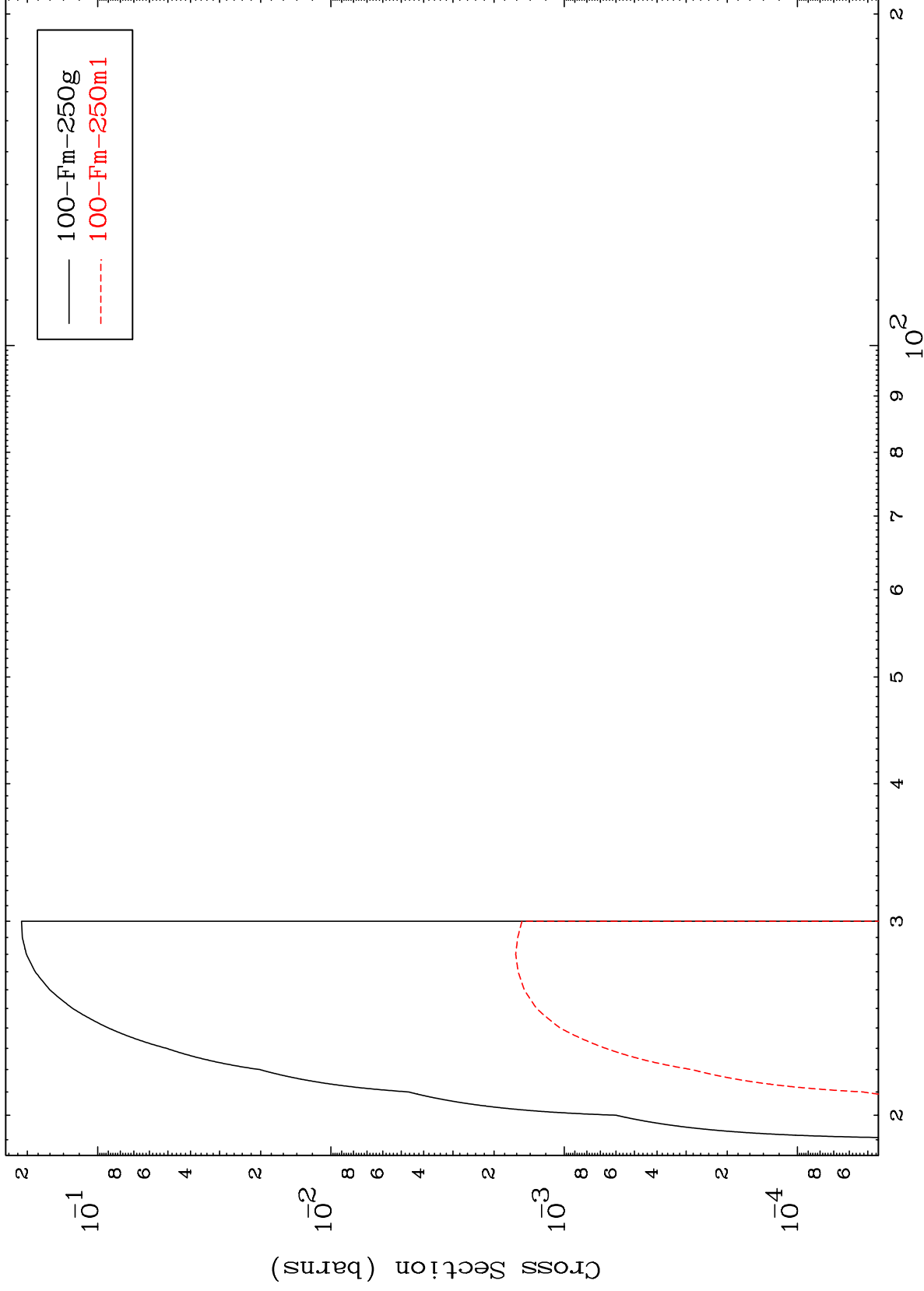
100-Fm-253

MAT 9934

(n,4n)

100-Fm-253

Radionuclide Production Cross Section



29

Incident Energy (MeV)

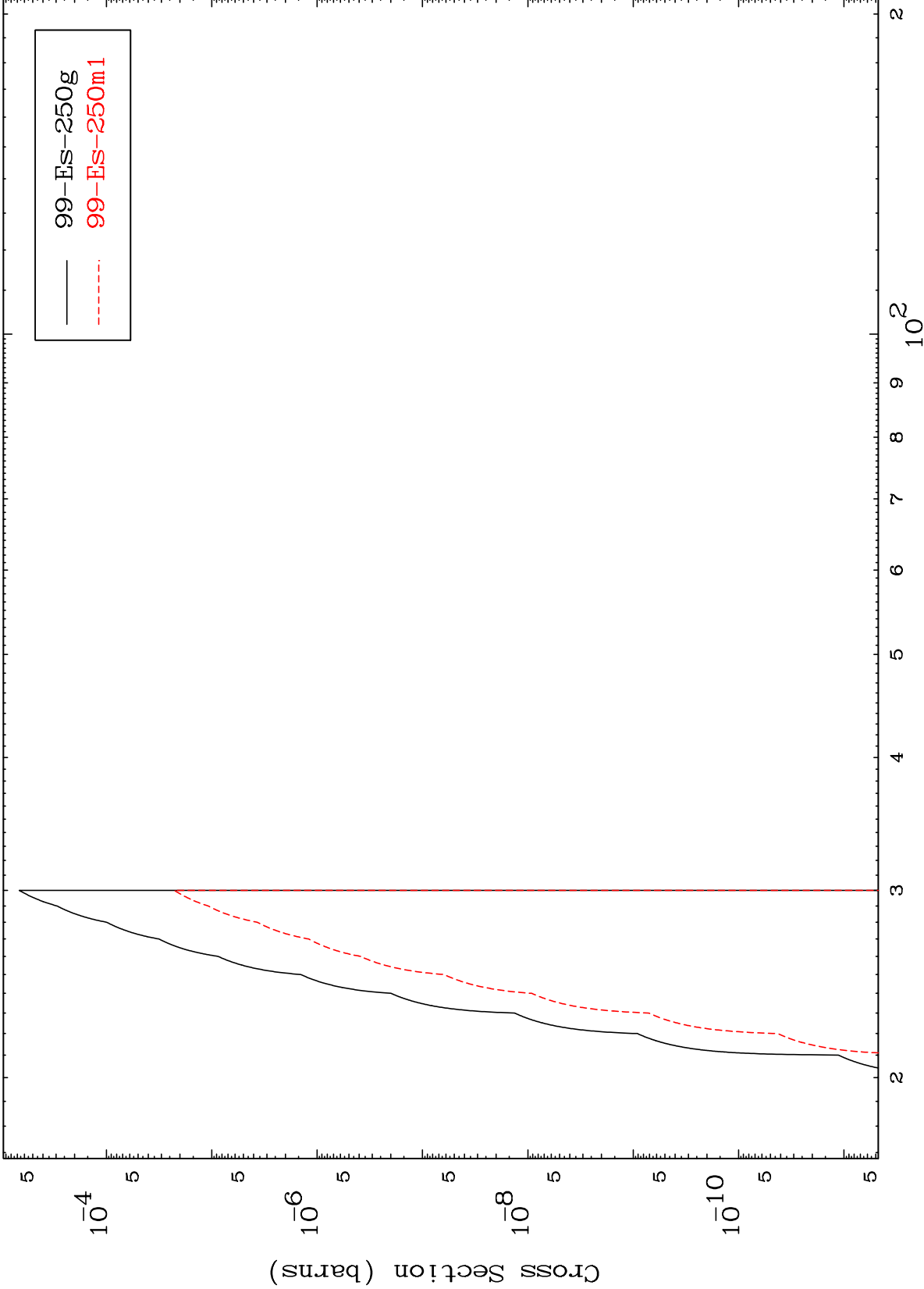
100-Fm-253

MAT 9934

(n,3n) p

100-Fm-253

Radionuclide Production Cross Section



30

Incident Energy (MeV)

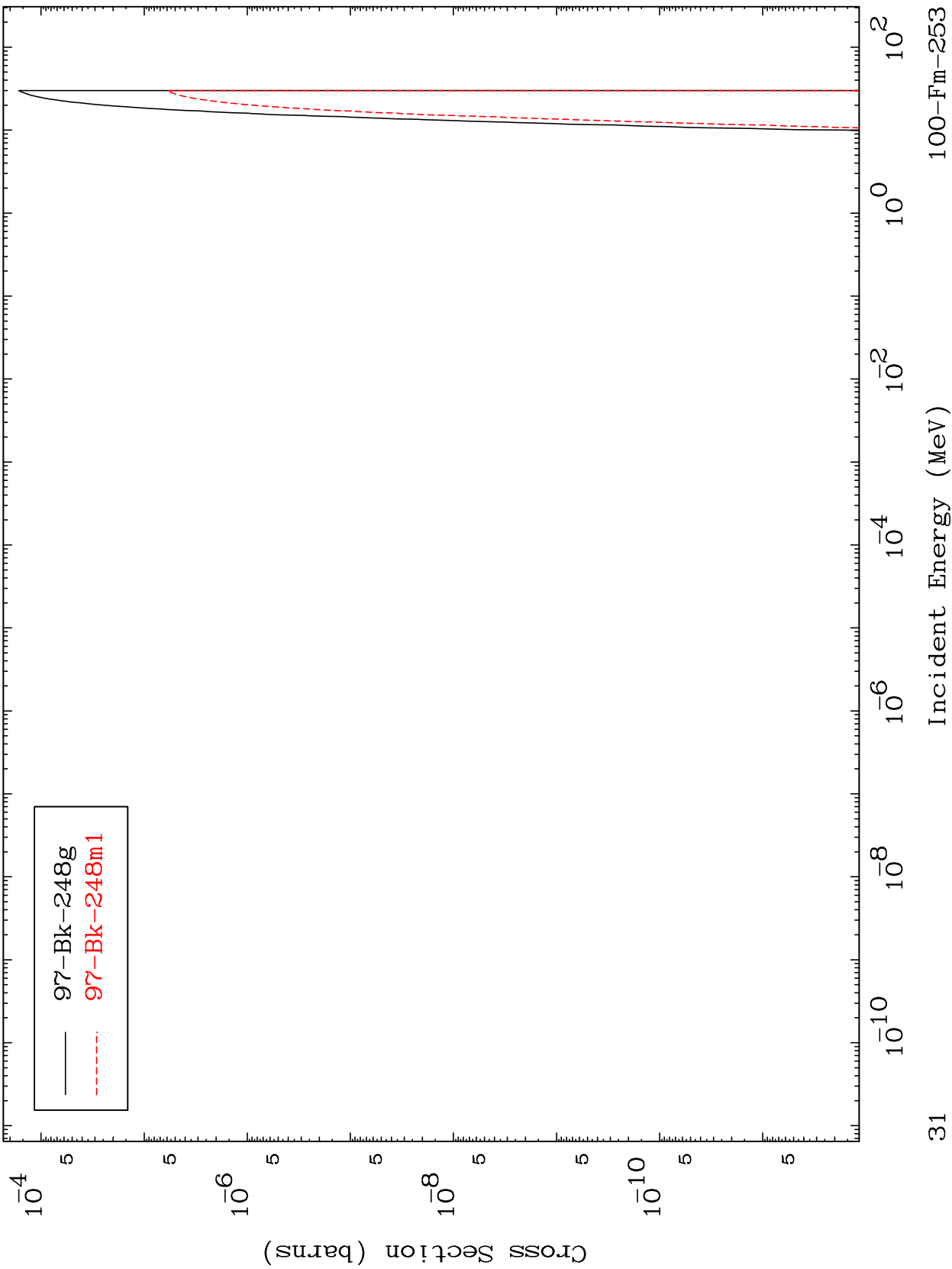
100-Fm-253

MAT 9934

(n,n') p α

100-Fm-253

Radionuclide Production Cross Section



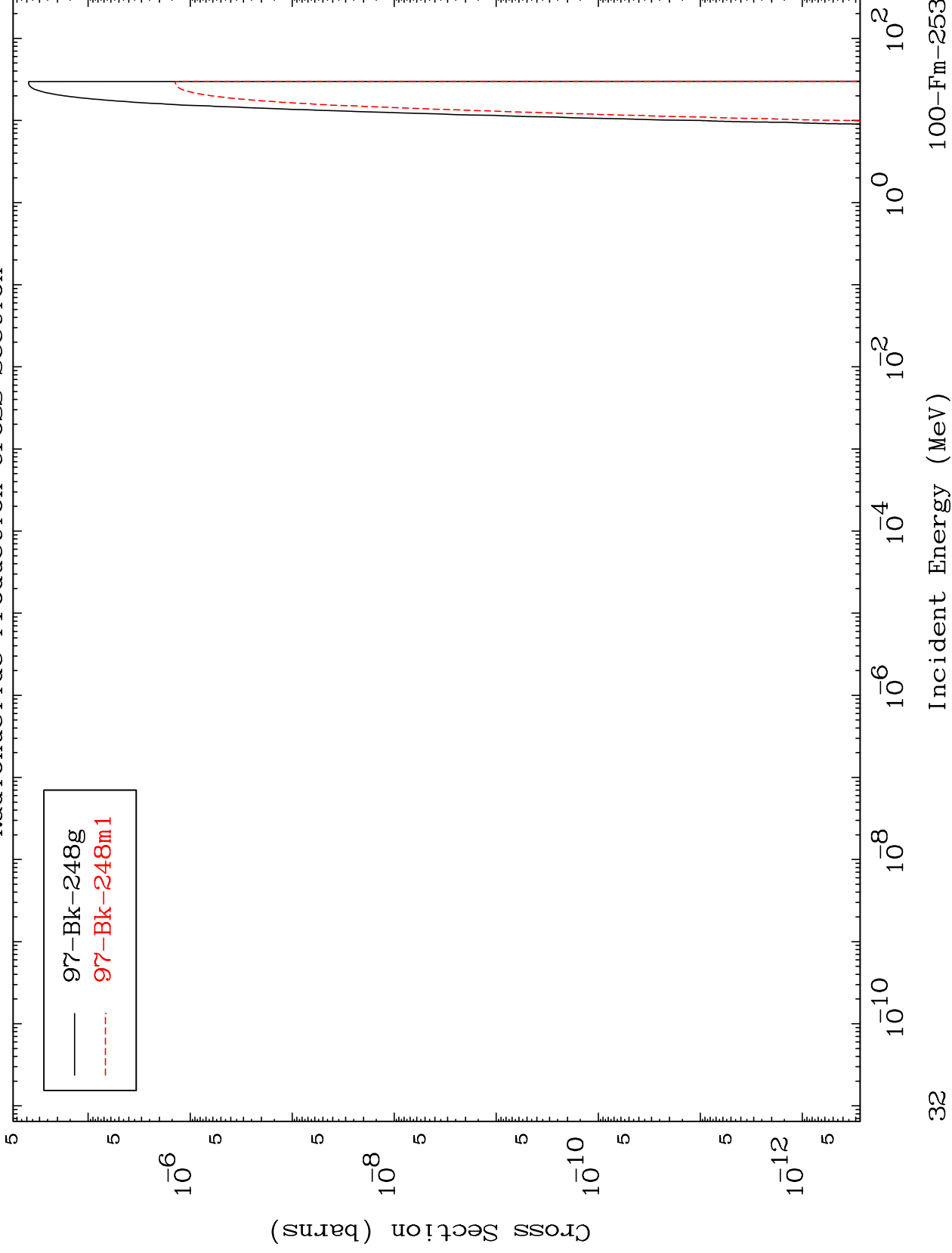
— 97-Bk-248g
- - - 97-Bk-248m1

MAT 9934

(n,d) α

100-Fm-253

Radionuclide Production Cross Section



— 97-Bk-248g
- - - 97-Bk-248m1