

Program EVALPLOT  
(Version 2015-2)

by

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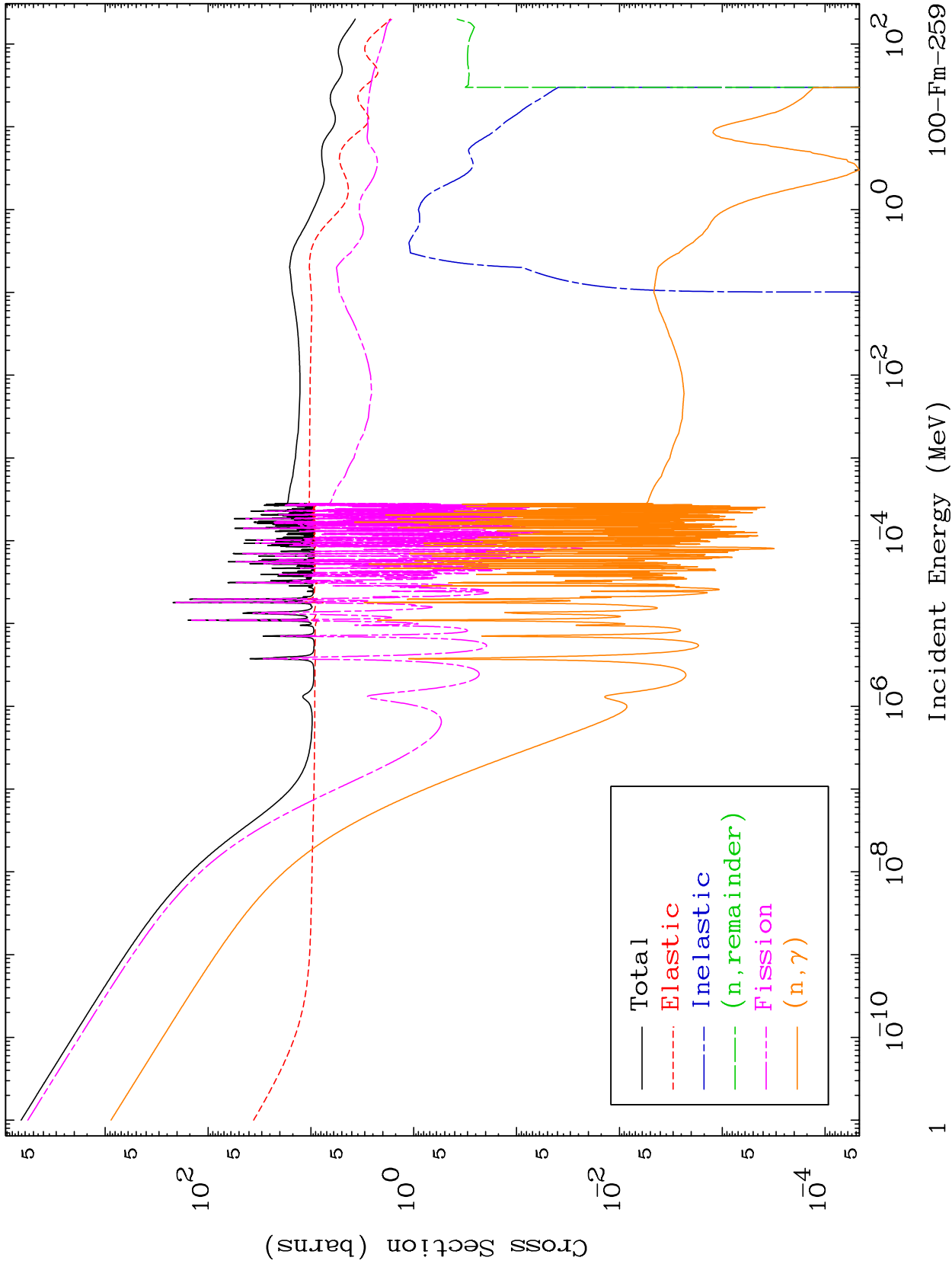
Press Mouse Button to Start

MAT 9940

Major

293 Kelvin Cross Sections

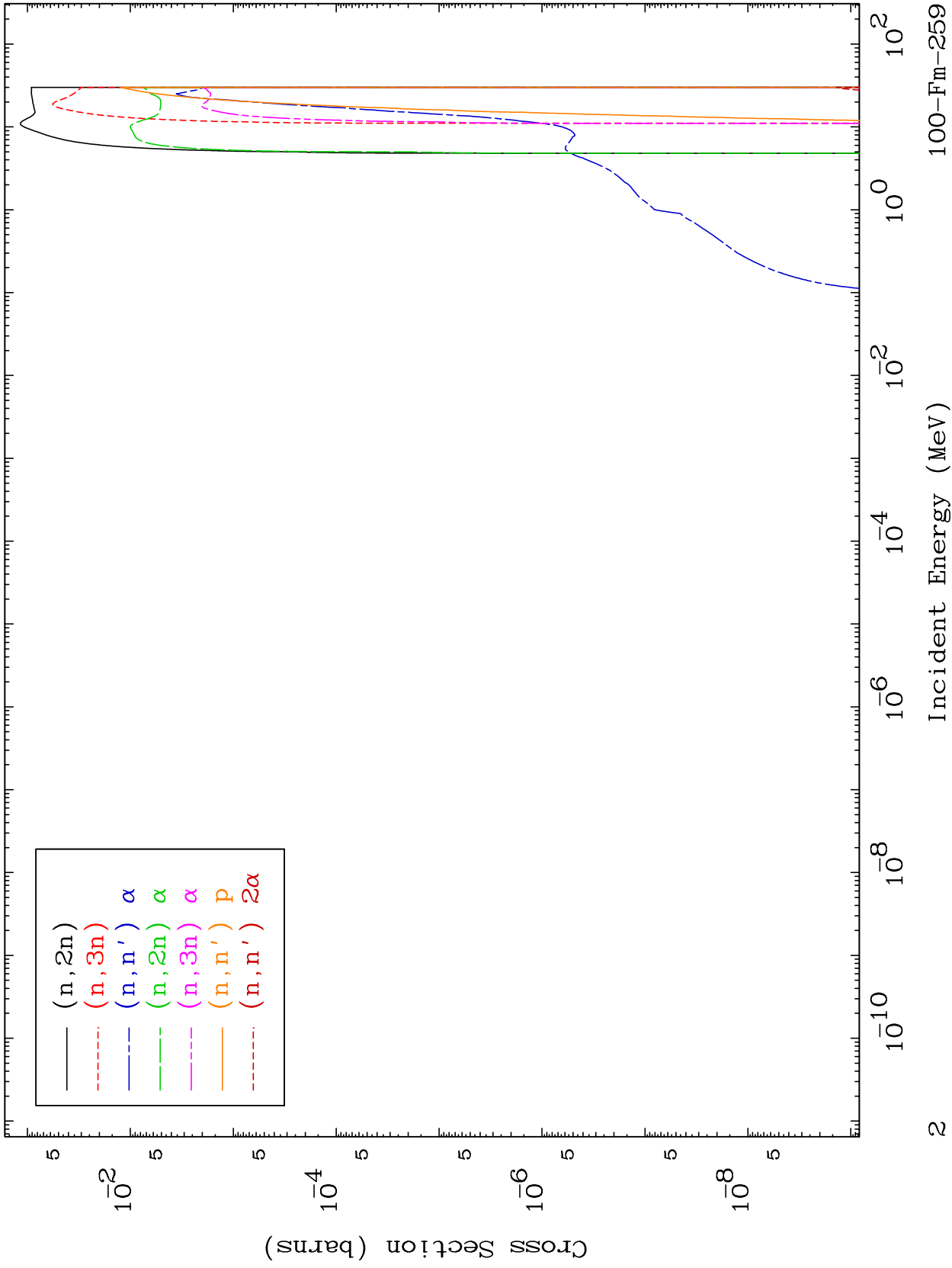
100-Fm-259



MAT 9940

Neutron Production  
293 Kelvin Cross Sections

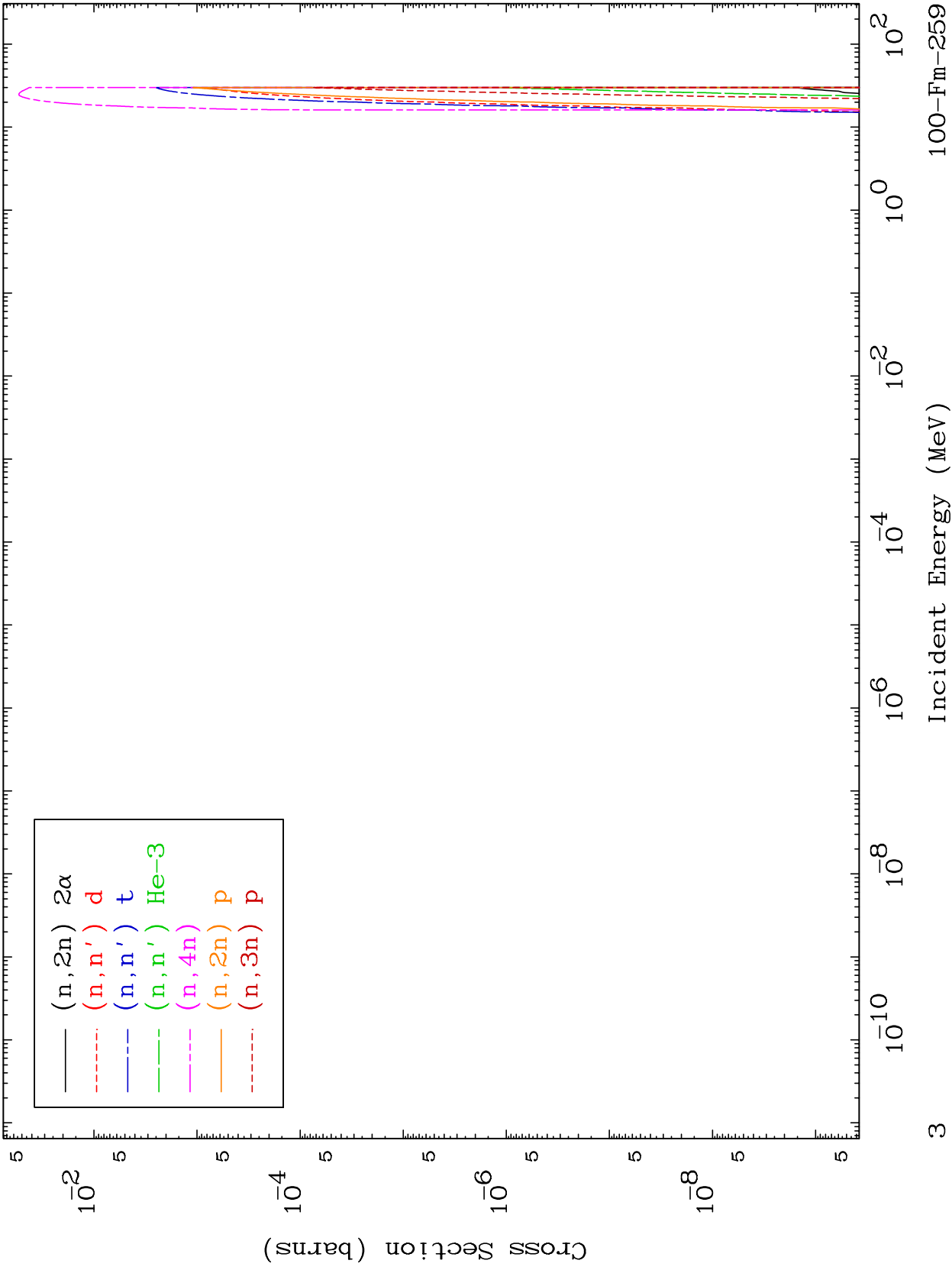
100-Fm-259



MAT 9940

Neutron Production  
293 Kelvin Cross Sections

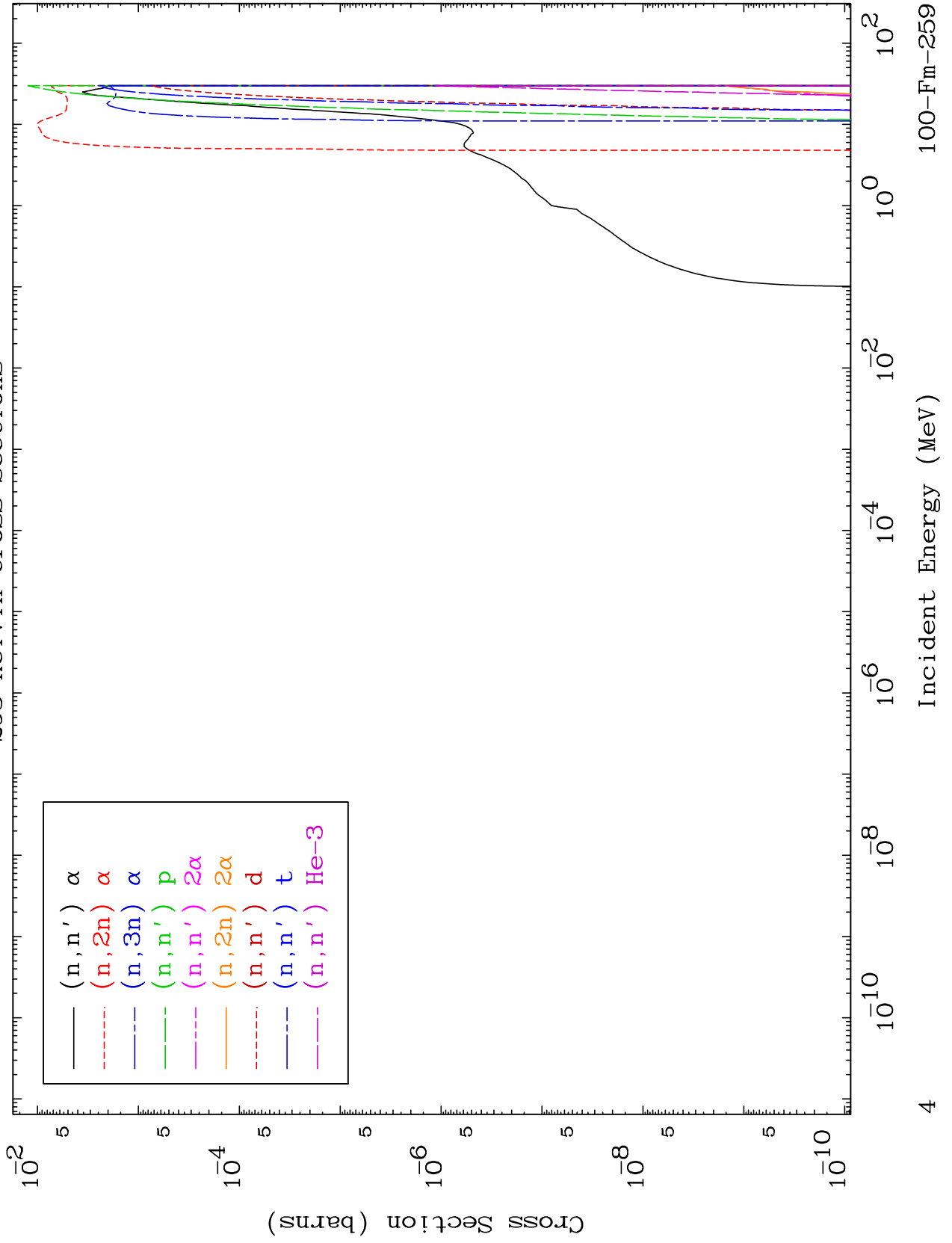
100-Fm-259



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Charged Particle  
293 Kelvin Cross Sections

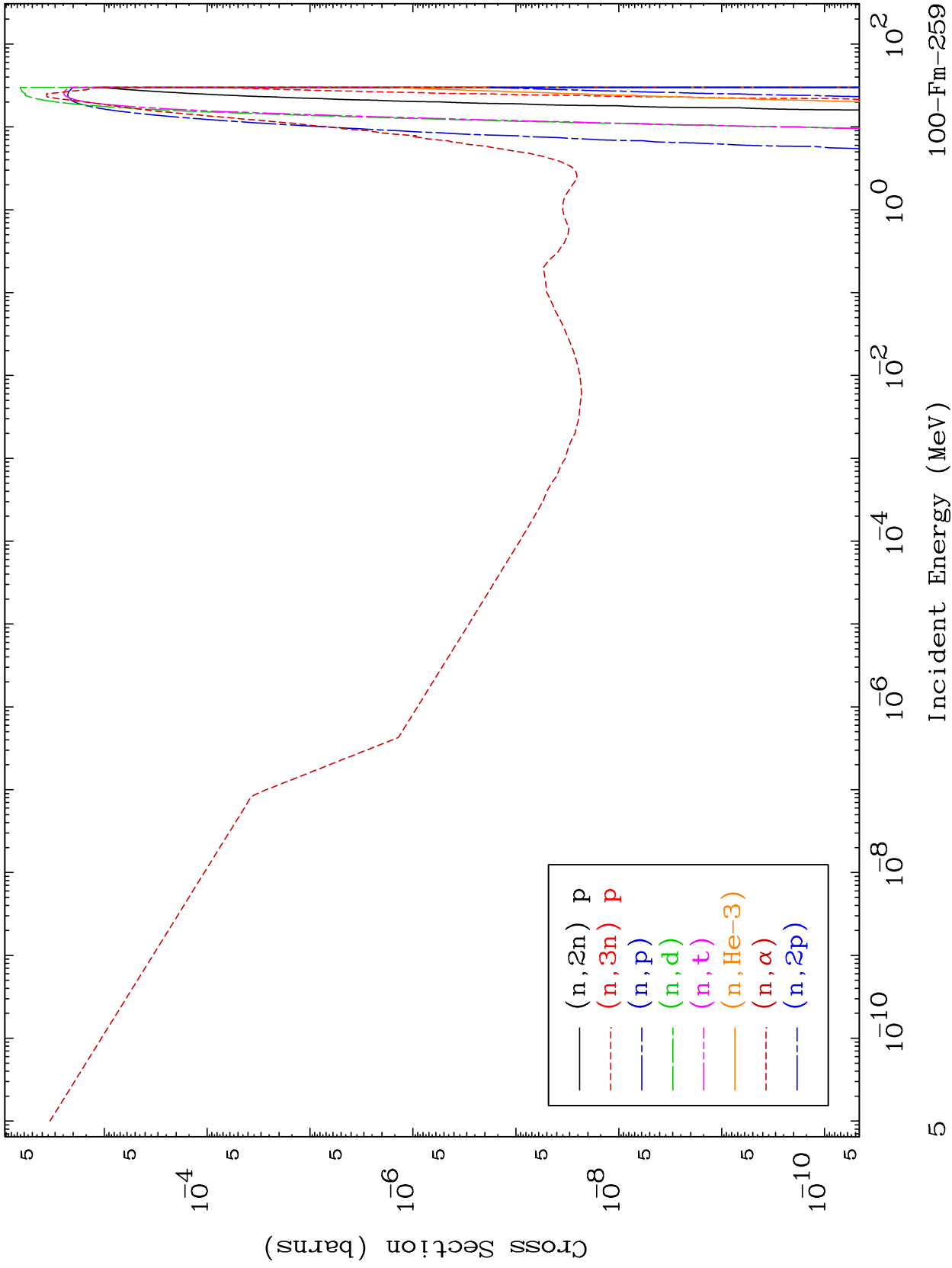
100-Fm-259



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Charged Particle  
293 Kelvin Cross Sections

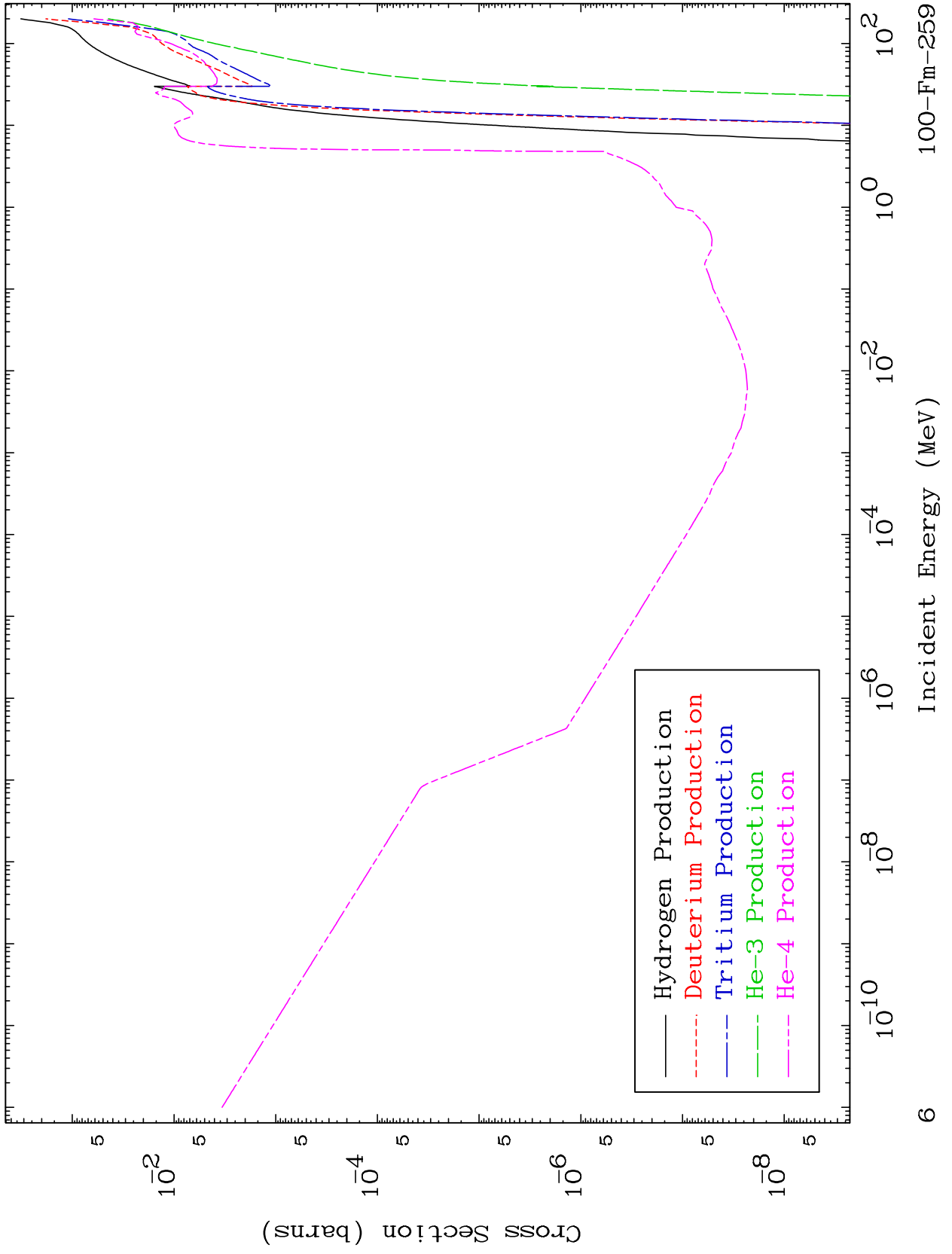
100-Fm-259



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Particle Production  
293 Kelvin Cross Sections

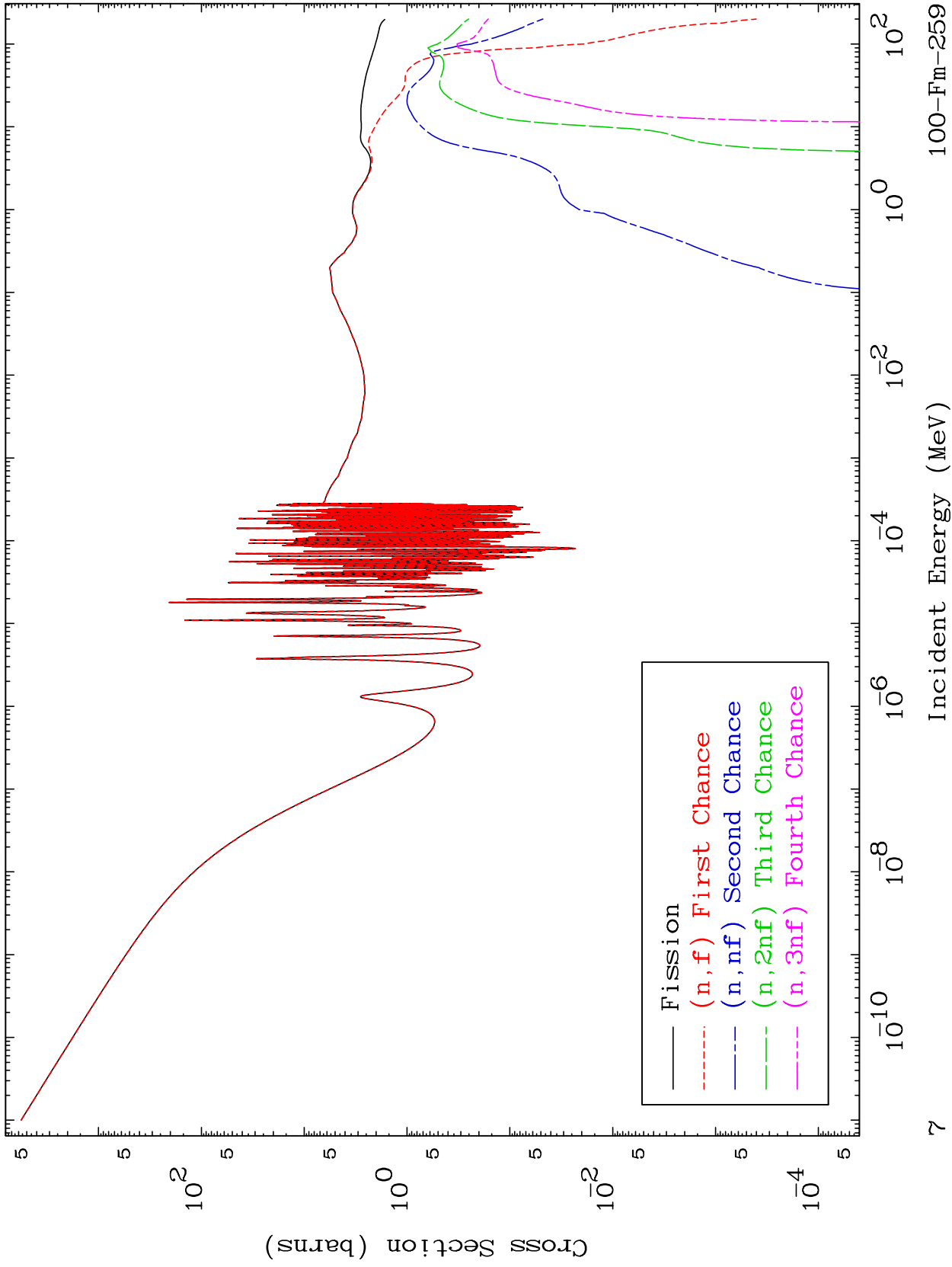
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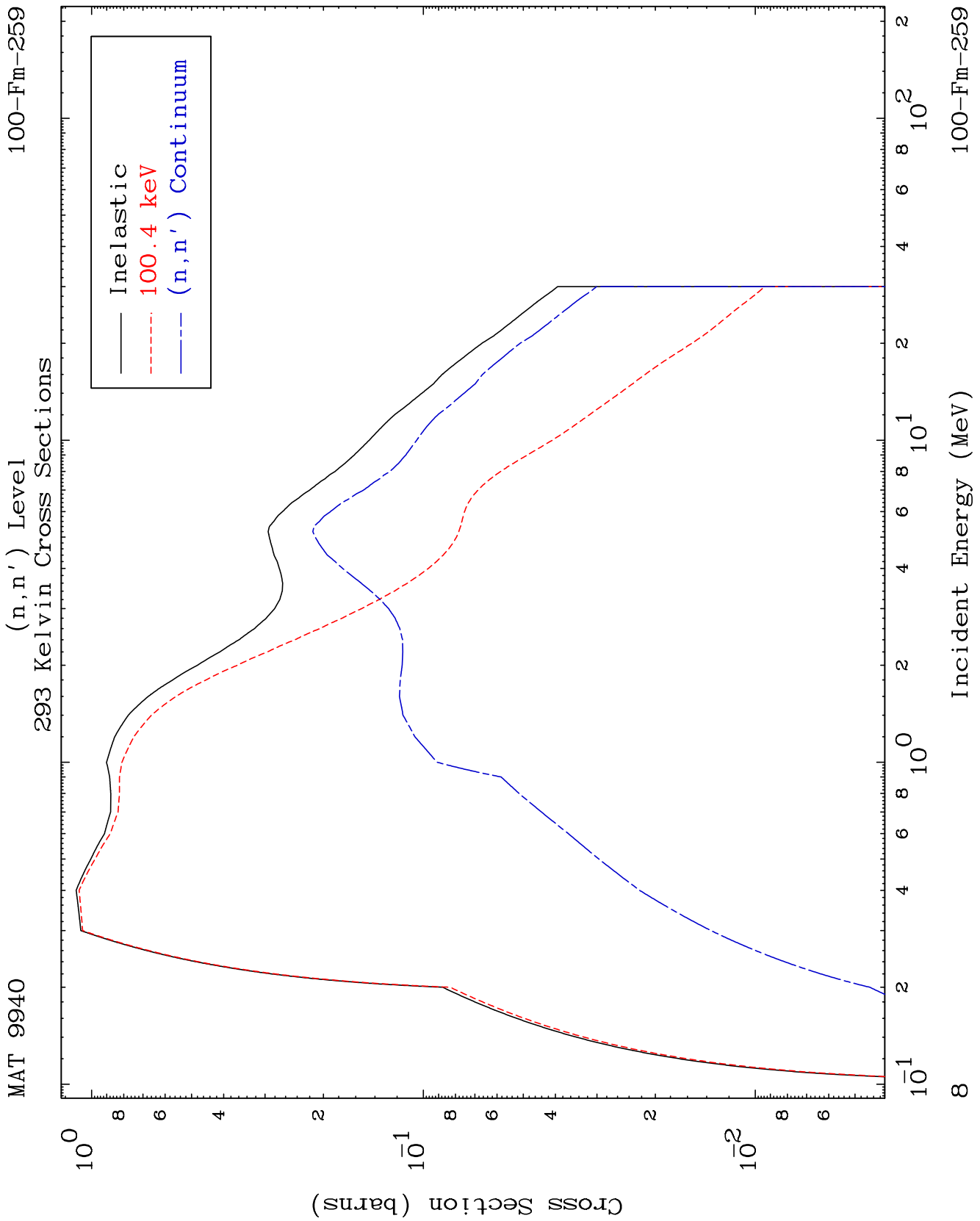
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Fission  
293 Kelvin Cross Sections

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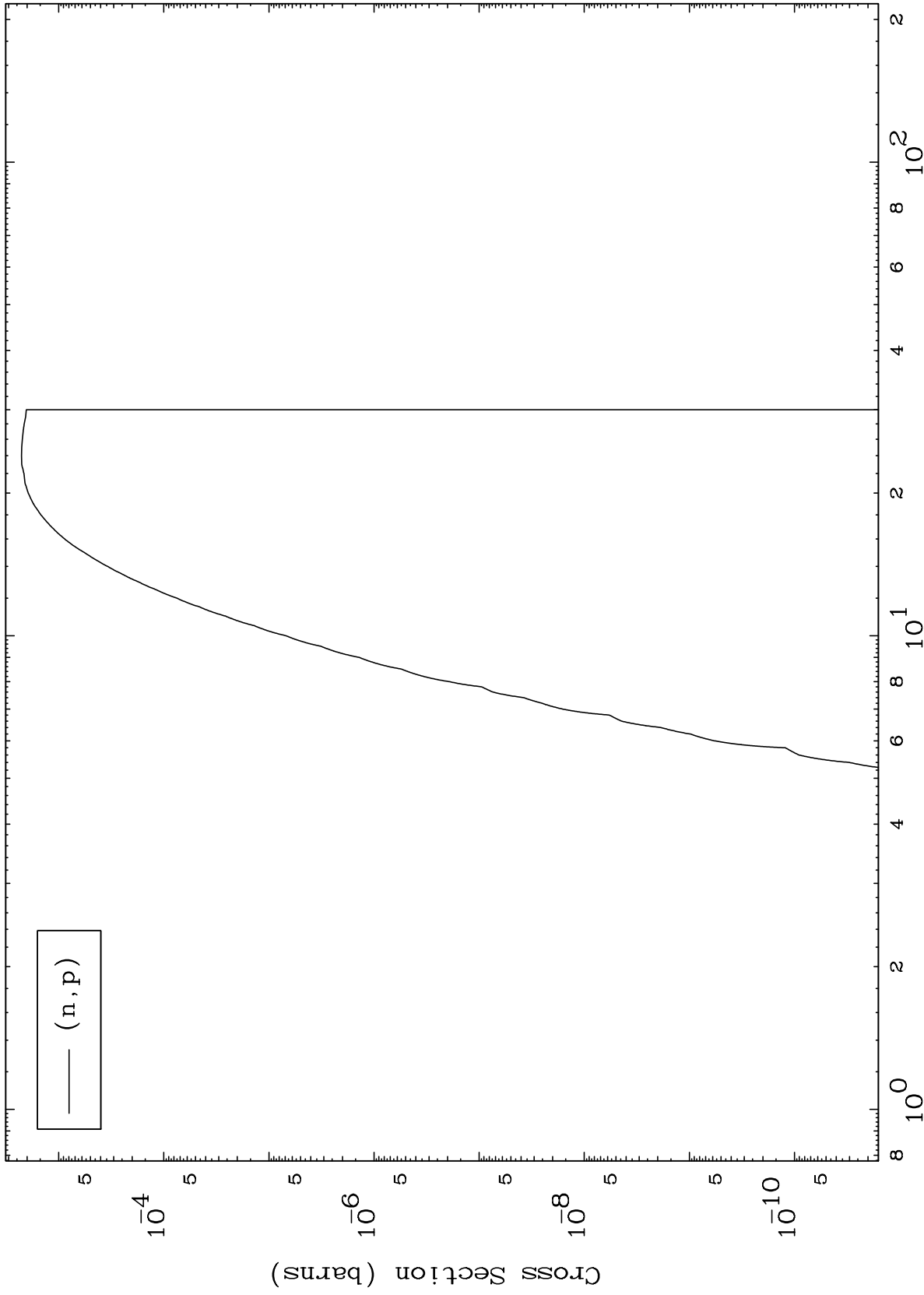




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(n,p) Levels  
293 Kelvin Cross Sections

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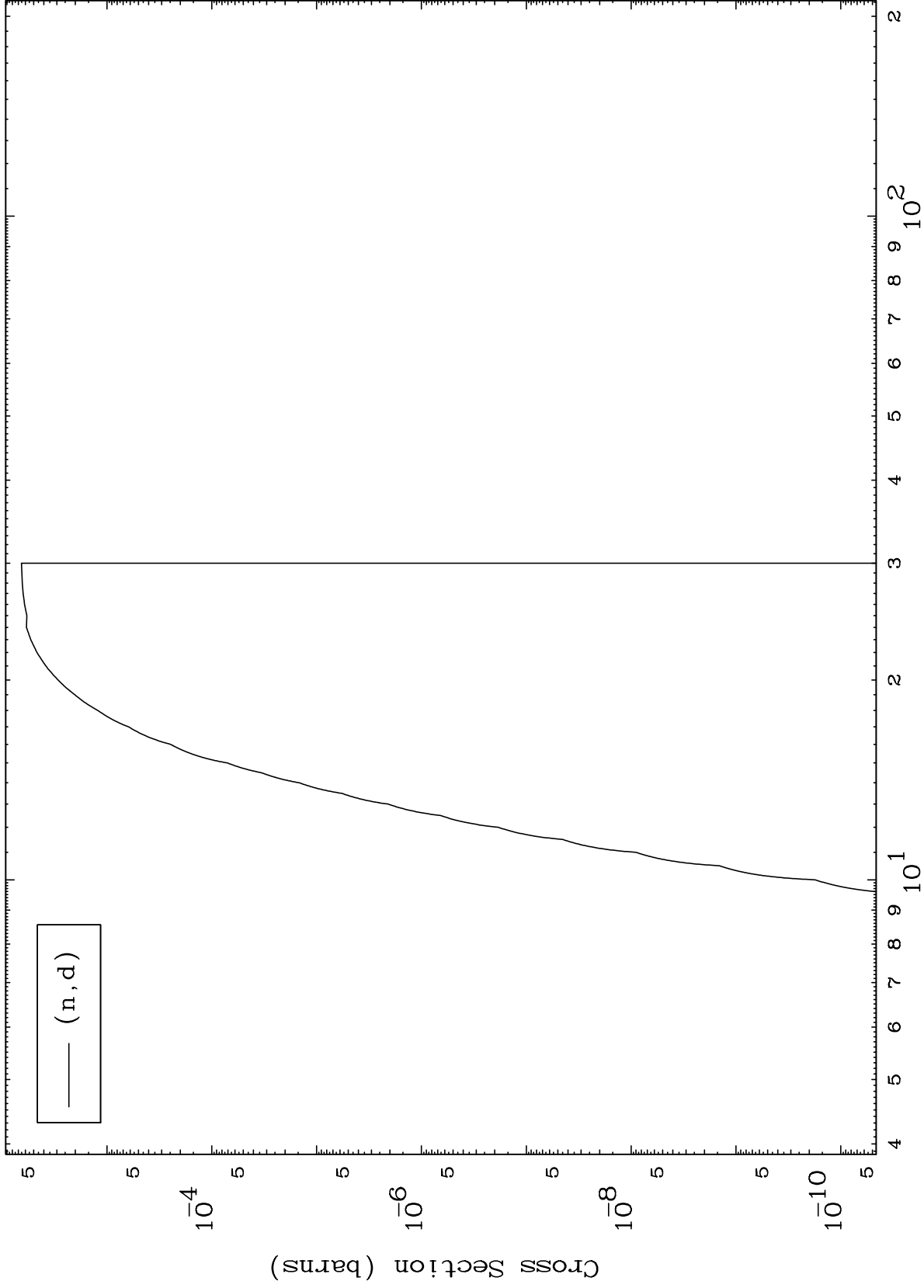
9

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(n,d) Levels  
293 Kelvin Cross Sections

100-Fm-259



(n,d)

Incident Energy (MeV)

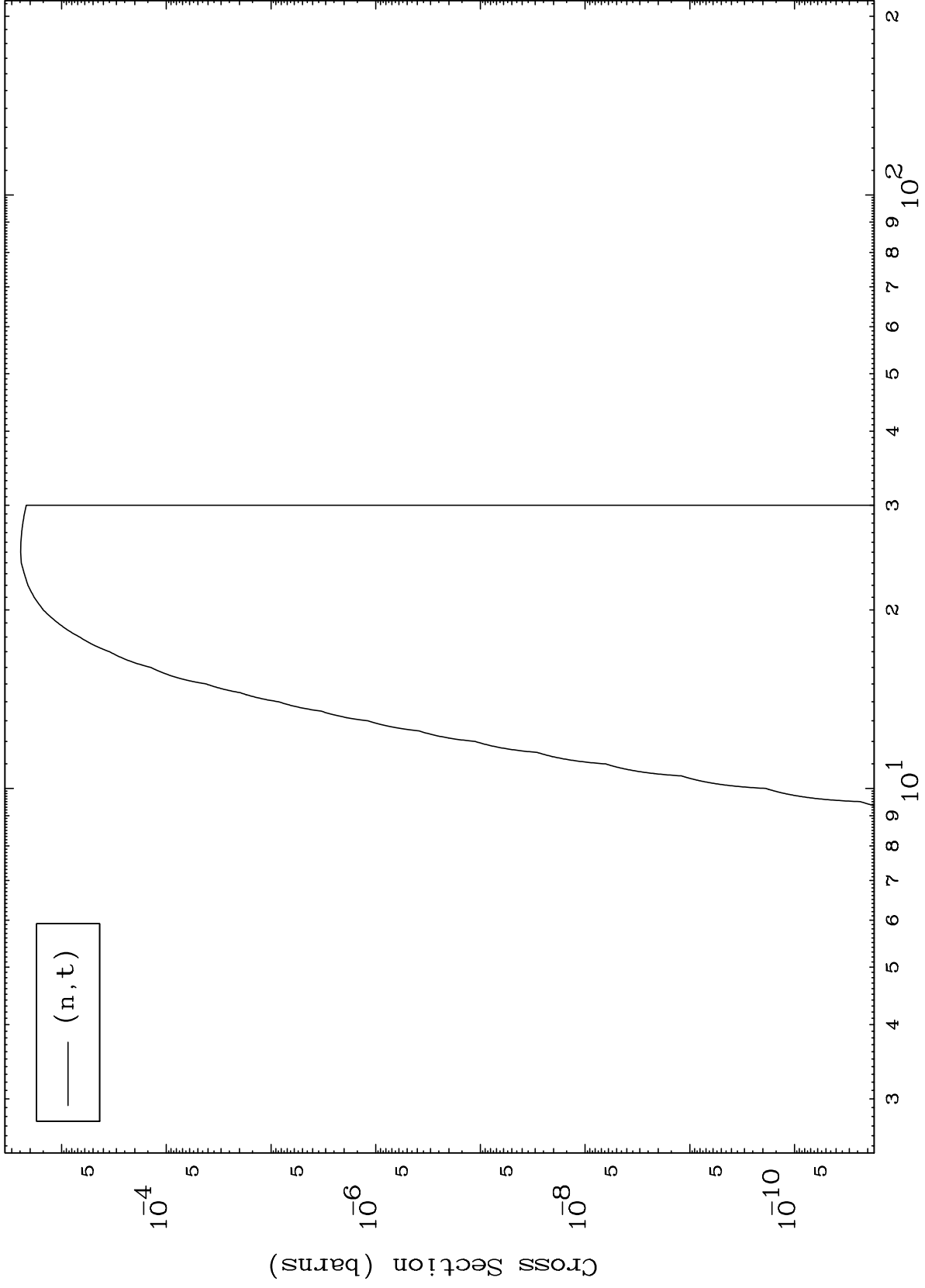
100-Fm-259

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(n,t) Levels  
293 Kelvin Cross Sections

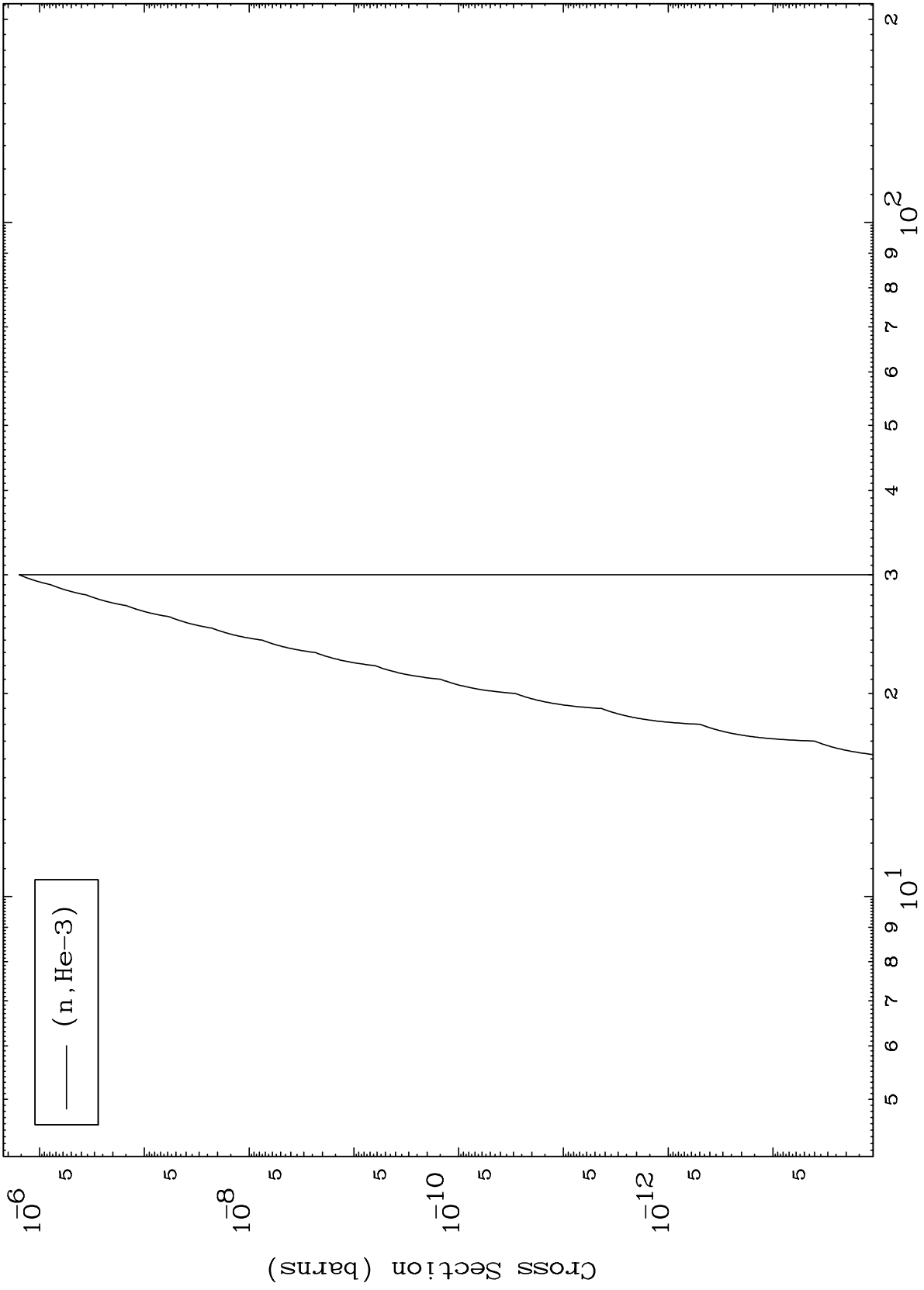
100-Fm-259



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(n,He3) Levels  
293 Kelvin Cross Sections

100-Fm-259



12

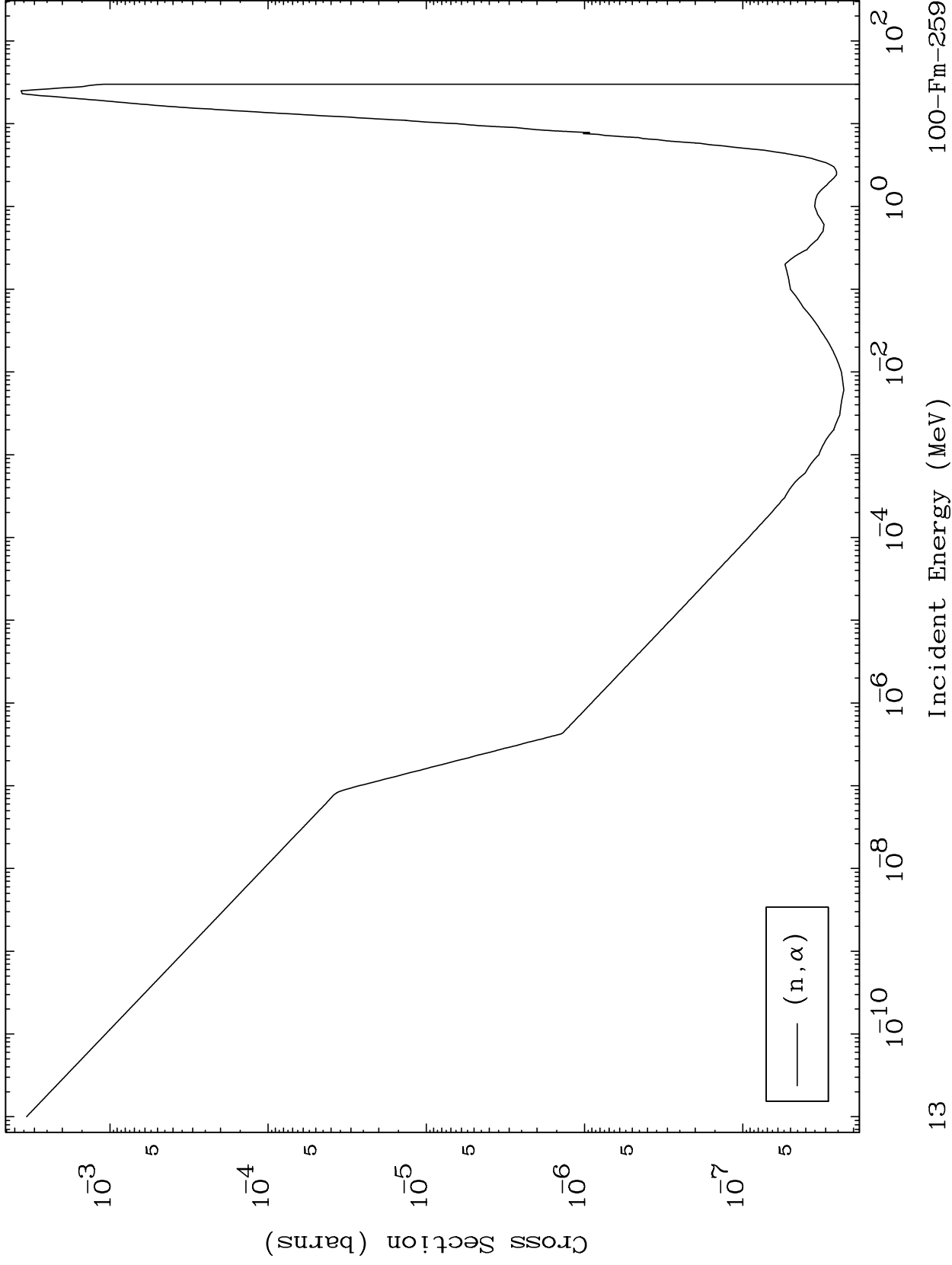
Incident Energy (MeV)

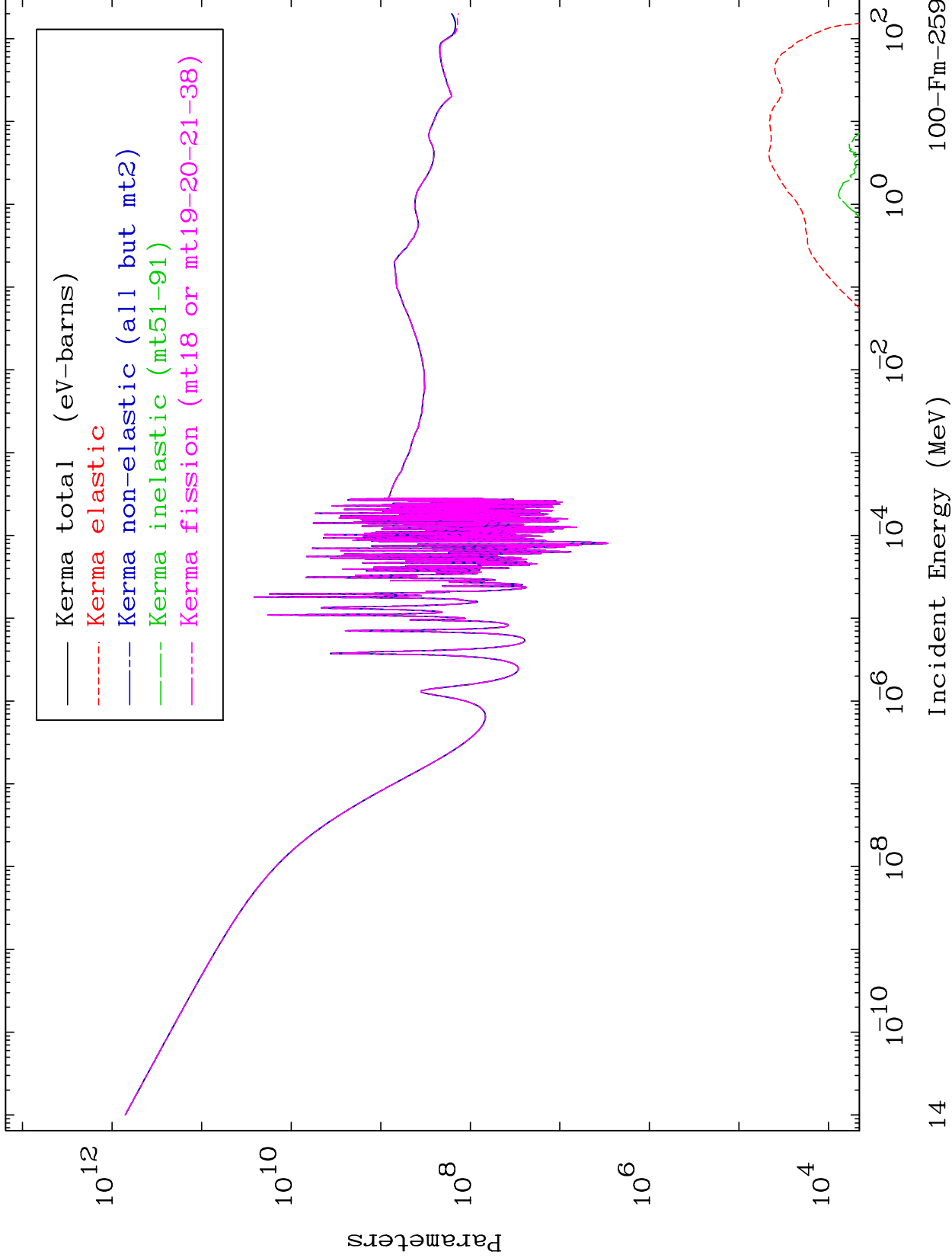
100-Fm-259

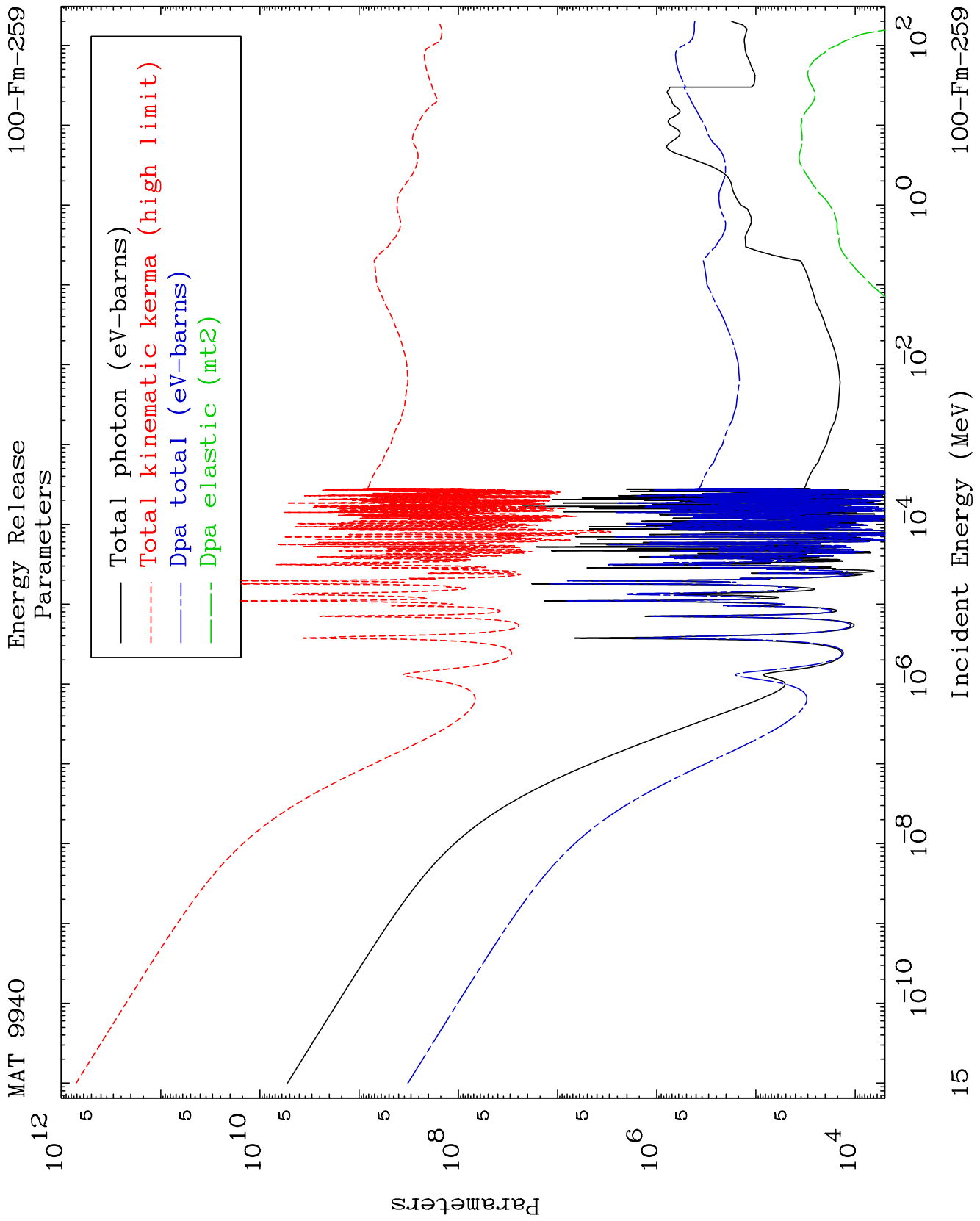
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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

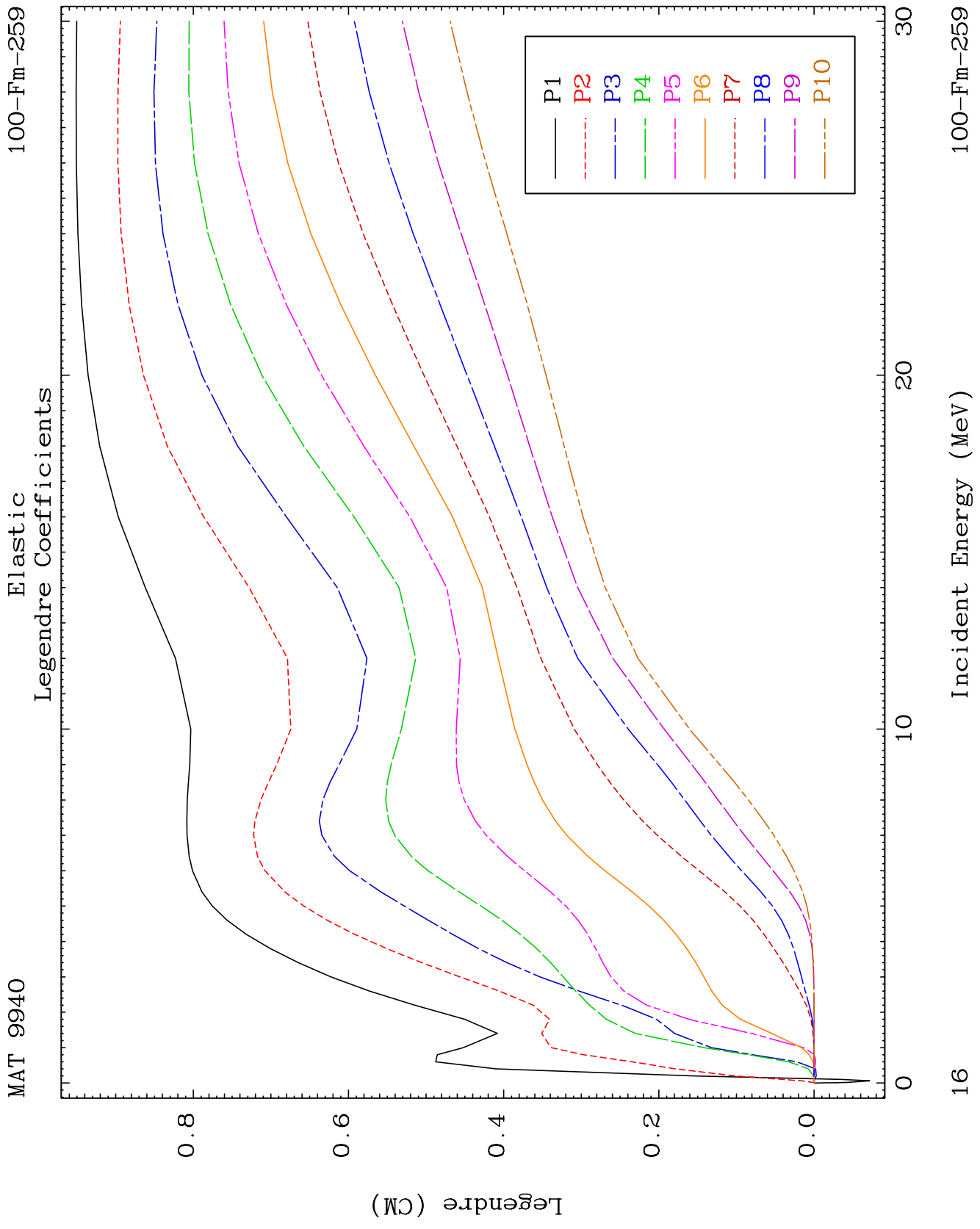
100-Fm-259







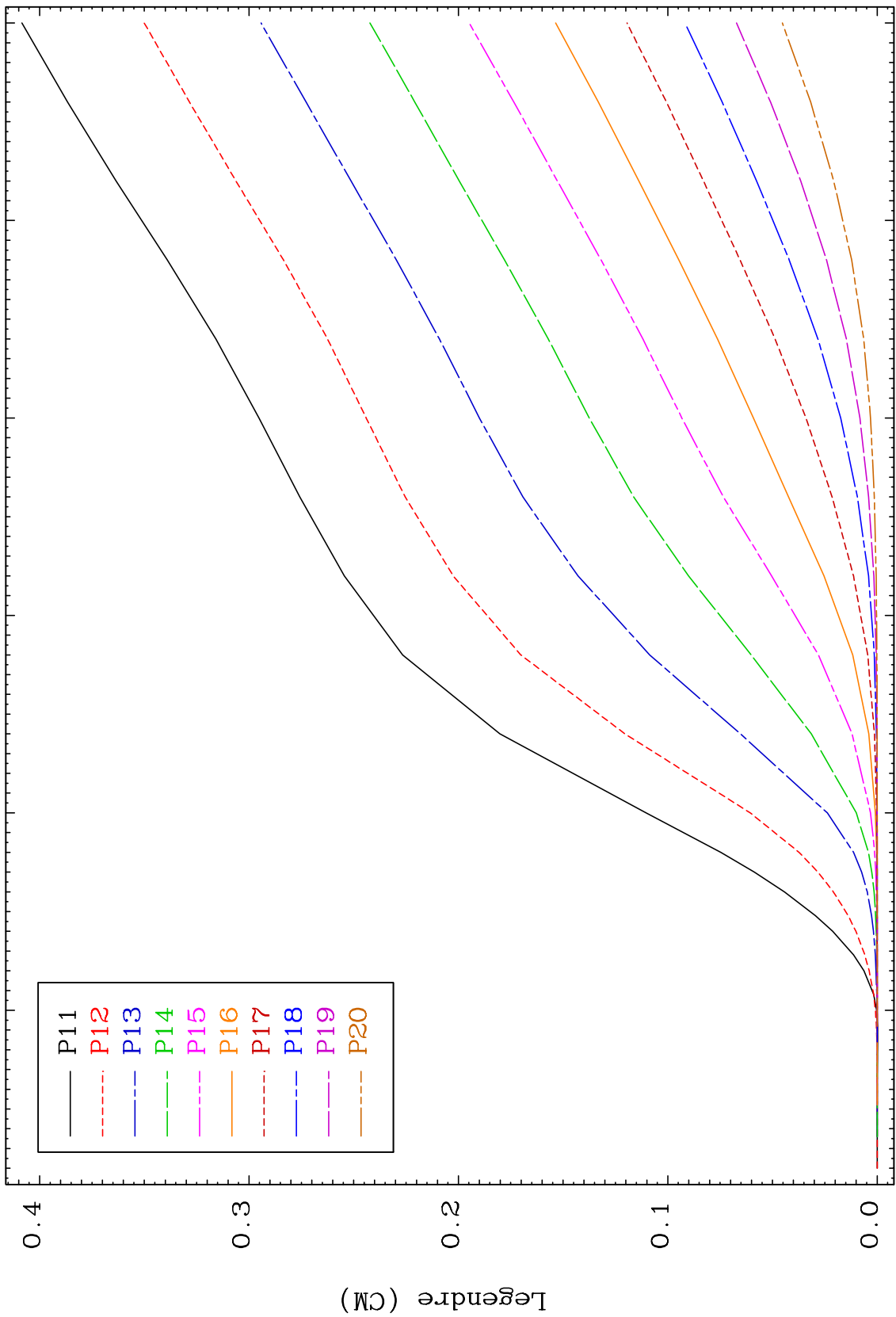




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Elastic Legendre Coefficients

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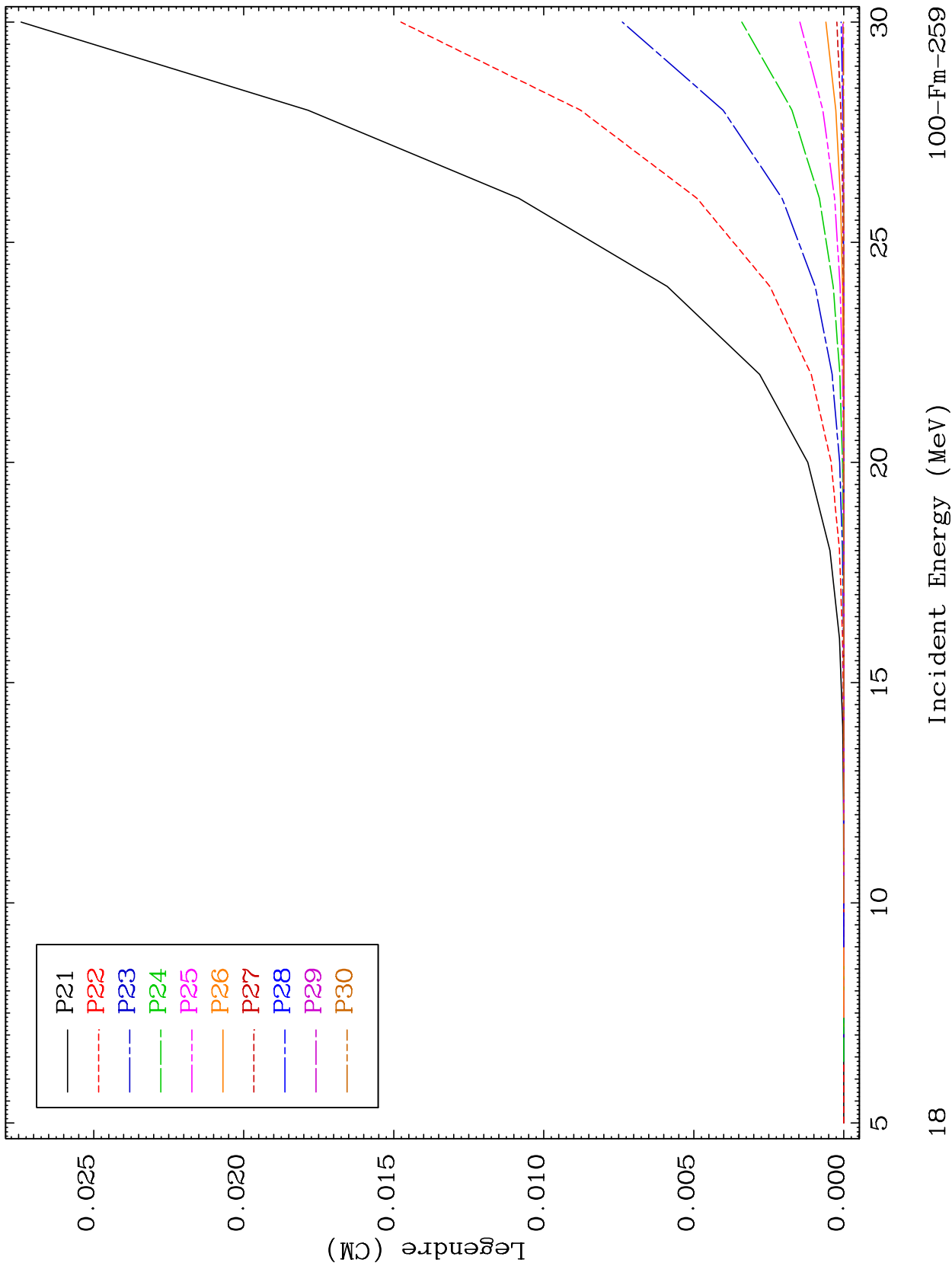
Incident Energy (MeV)

100-Fm-259

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Elastic Legendre Coefficients

100-Fm-259



18

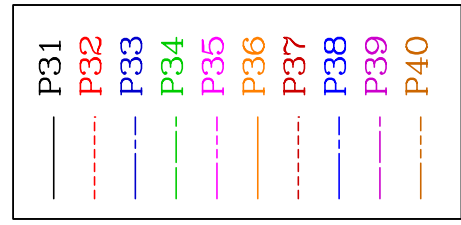
Incident Energy (MeV)

100-Fm-259

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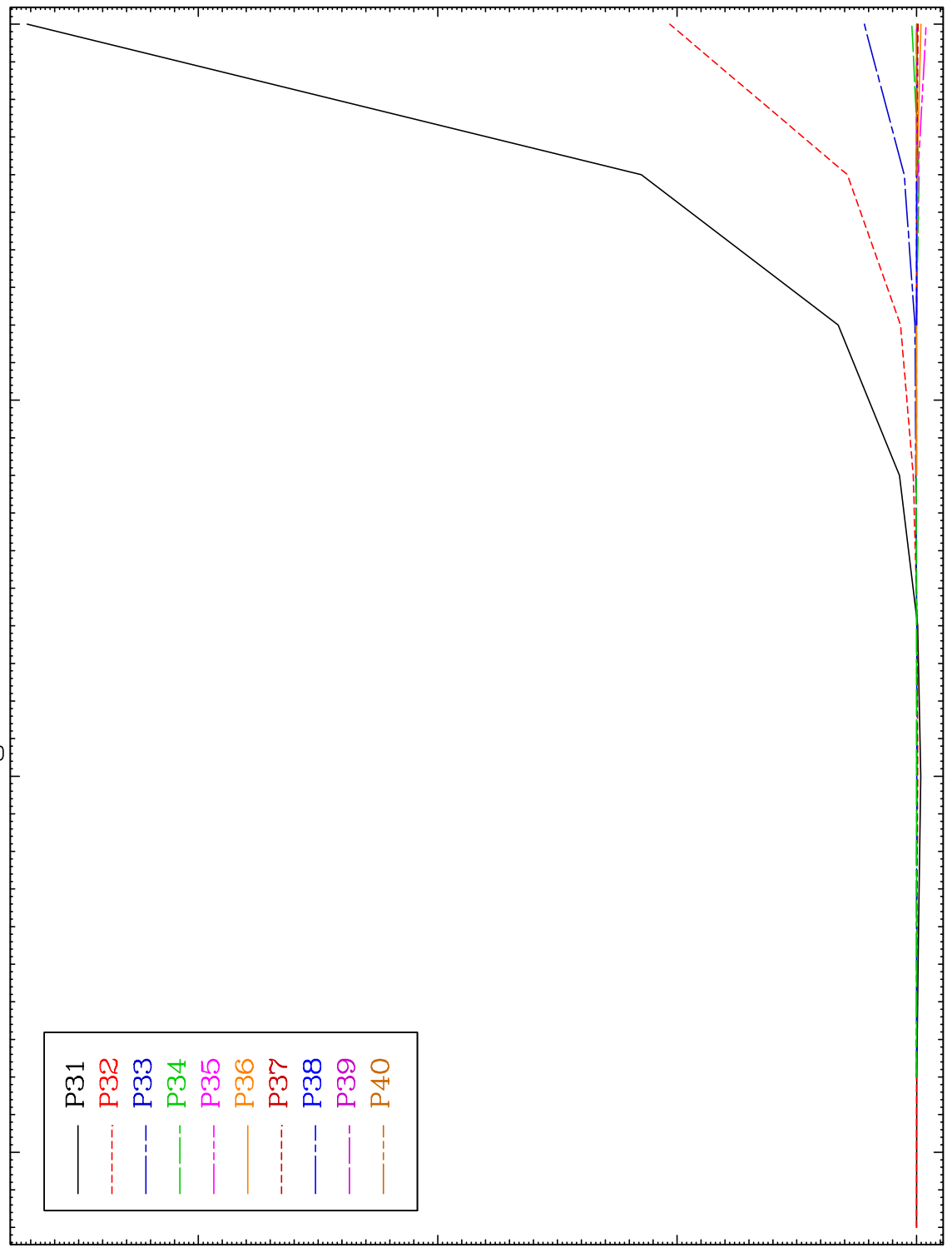
Elastic Legendre Coefficients

100-Fm-259



$\times 10^{-6}$

Legendre (CM)



19

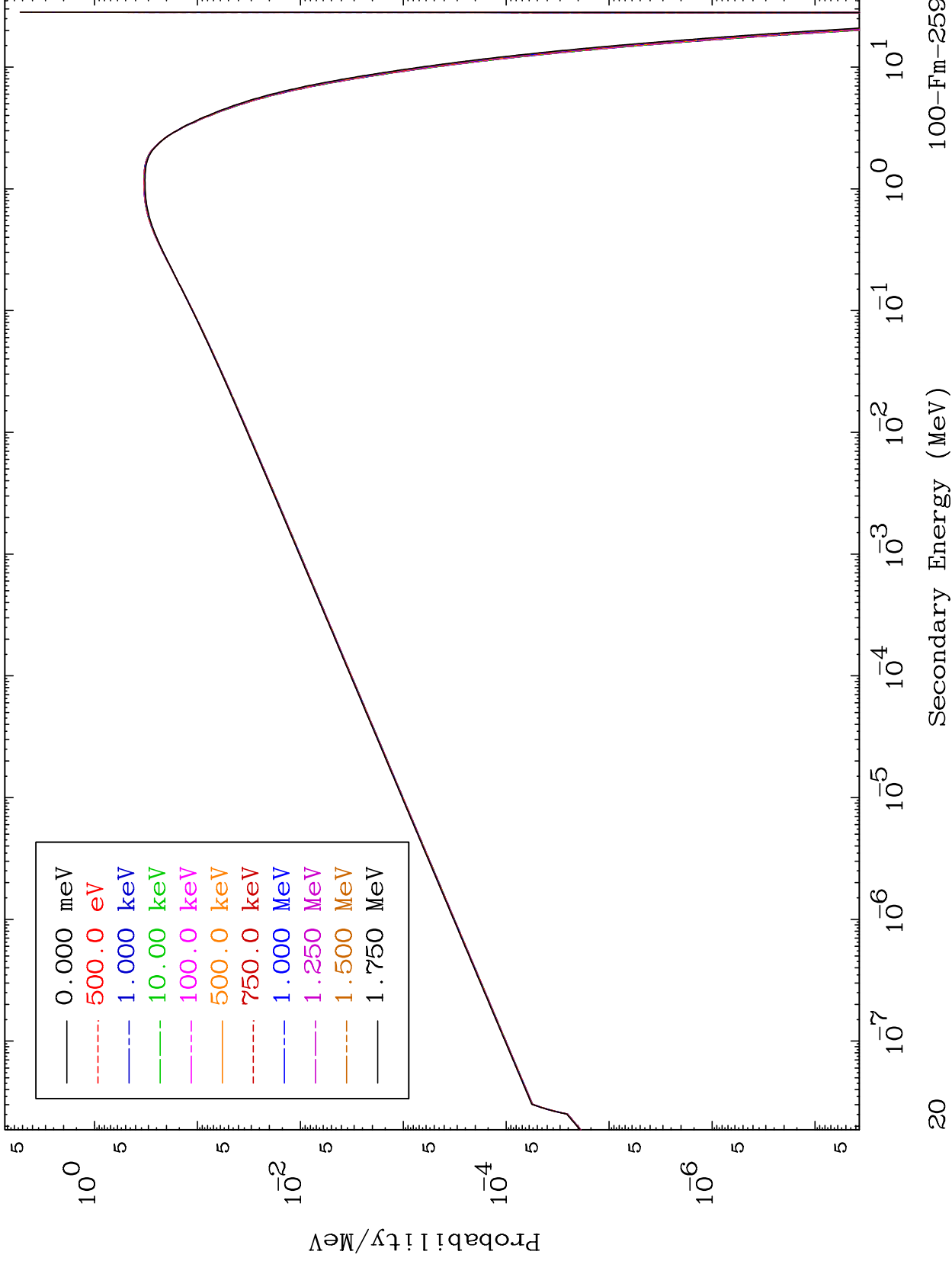
Incident Energy (MeV)

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# Fission Energy Distribution

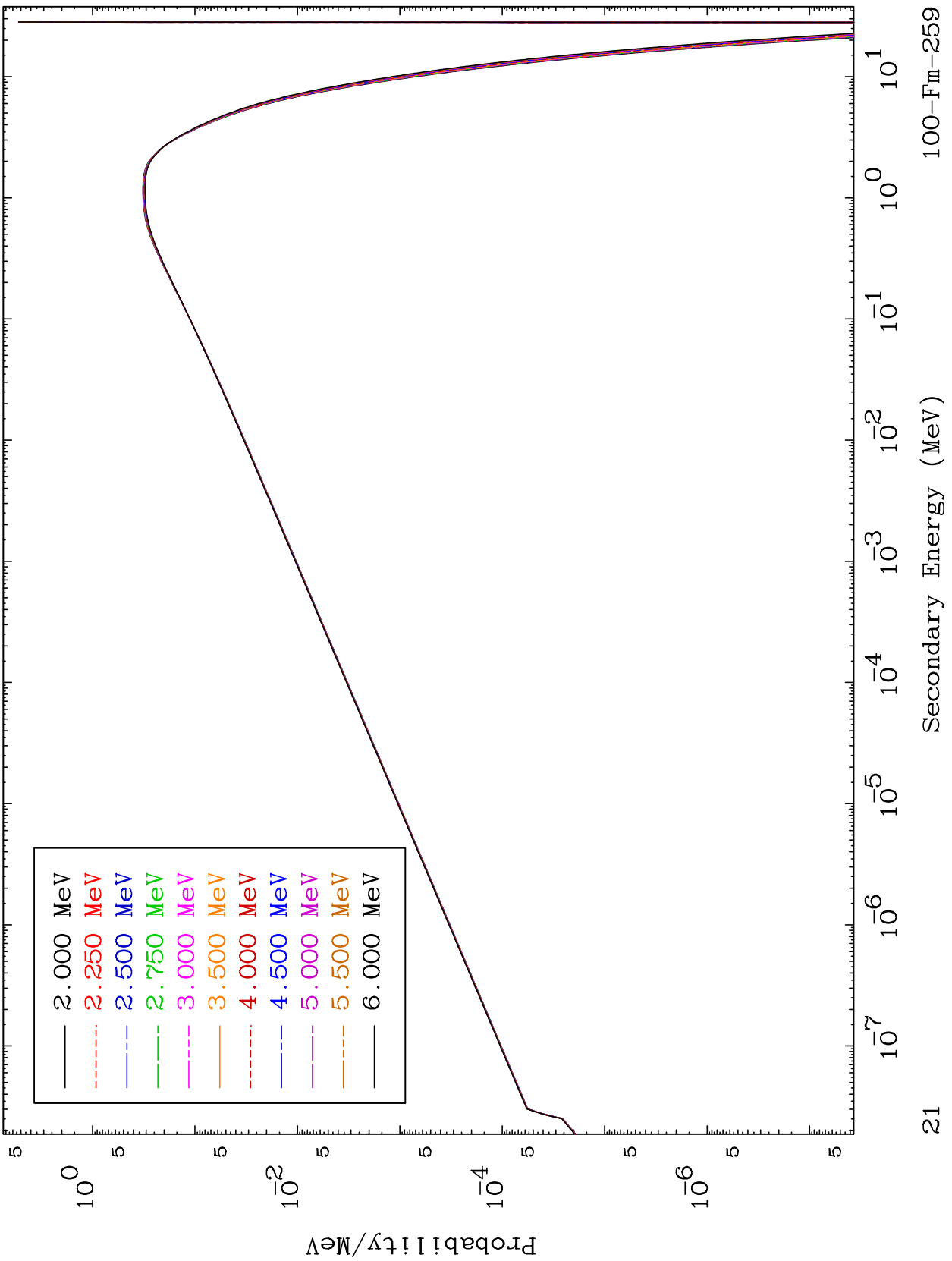
100-Fm-259



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### Fission Energy Distribution

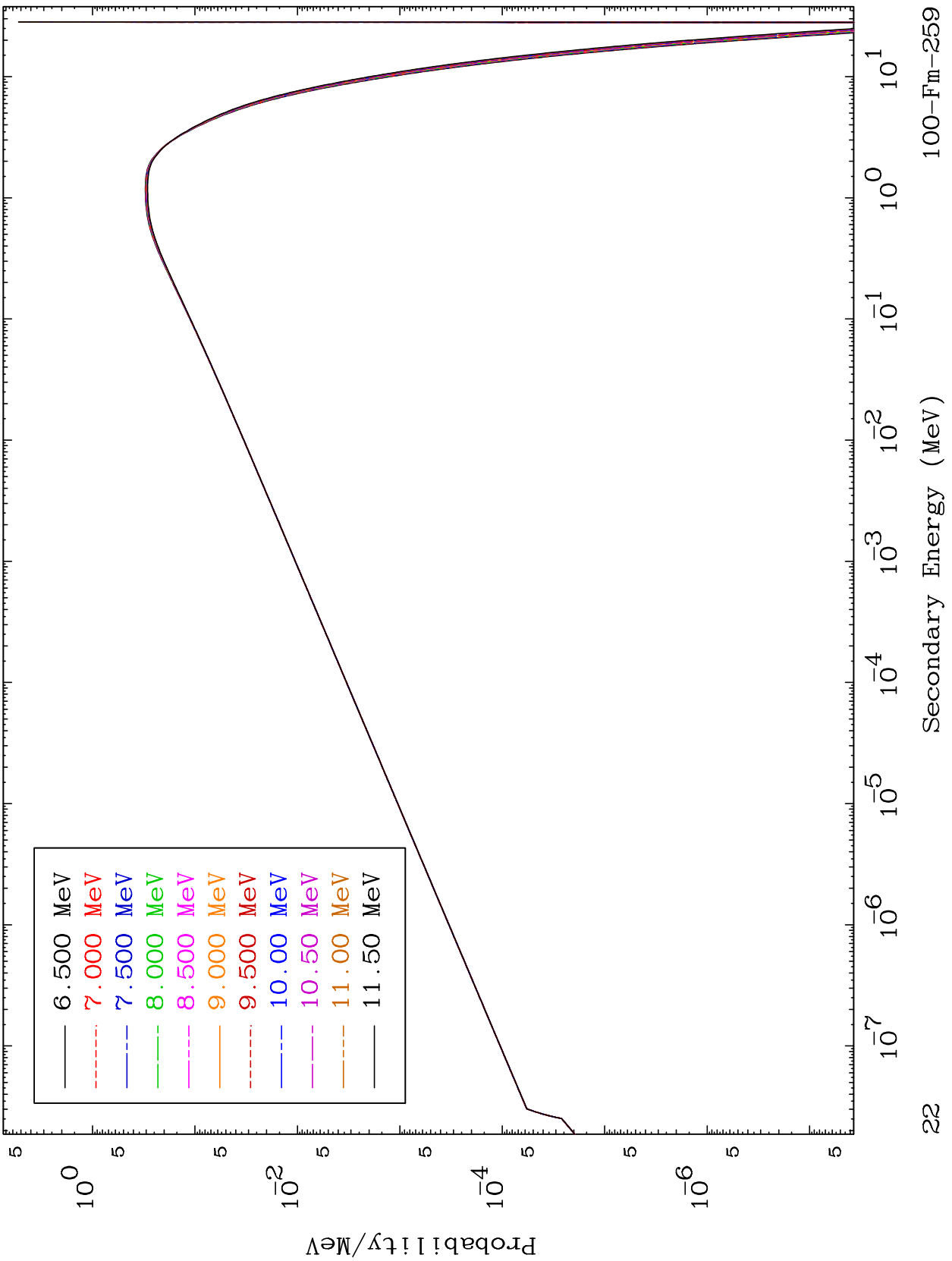
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### Fission Energy Distribution

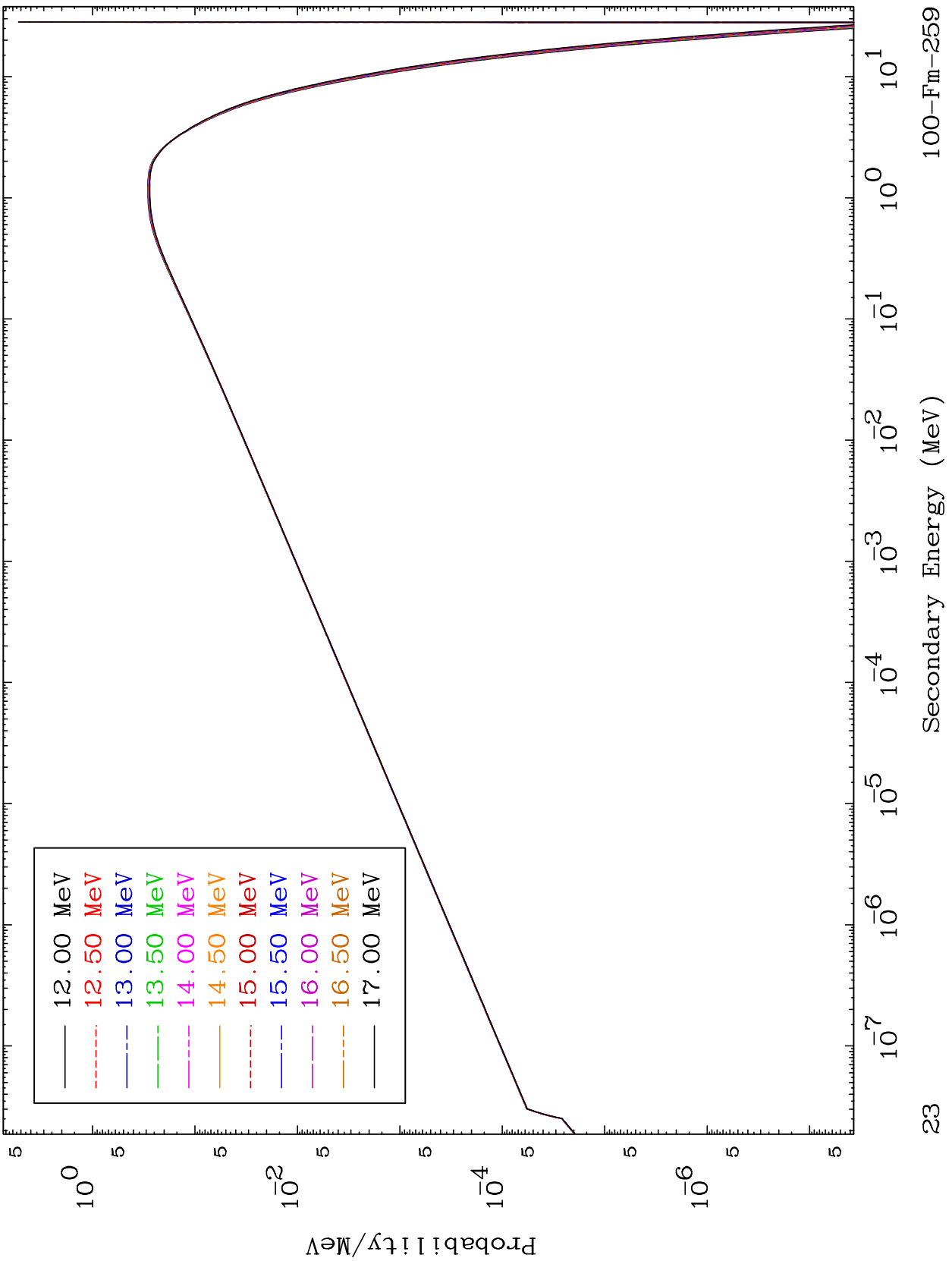
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### Fission Energy Distribution

100-Fm-259

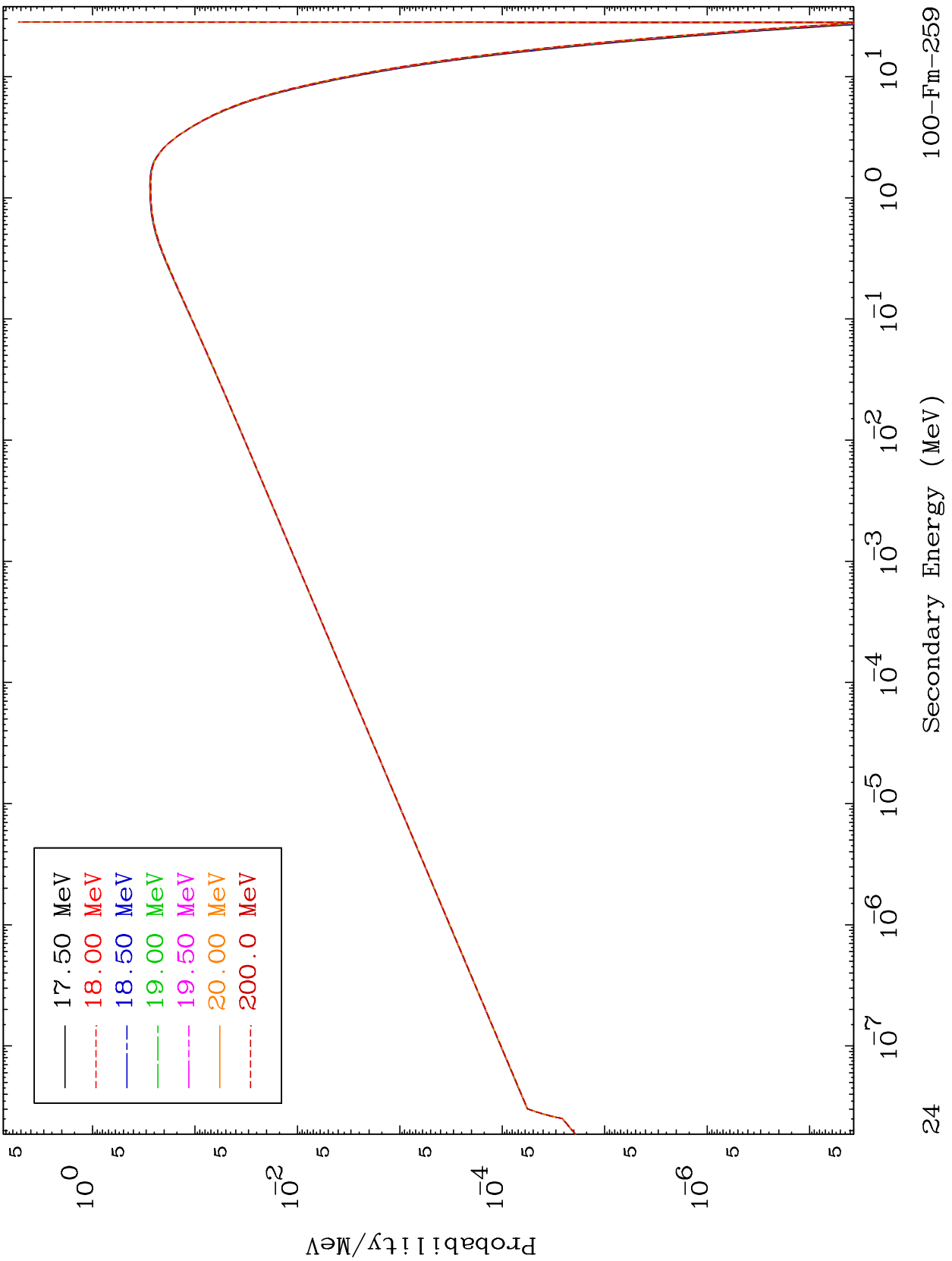




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### Fission Energy Distribution

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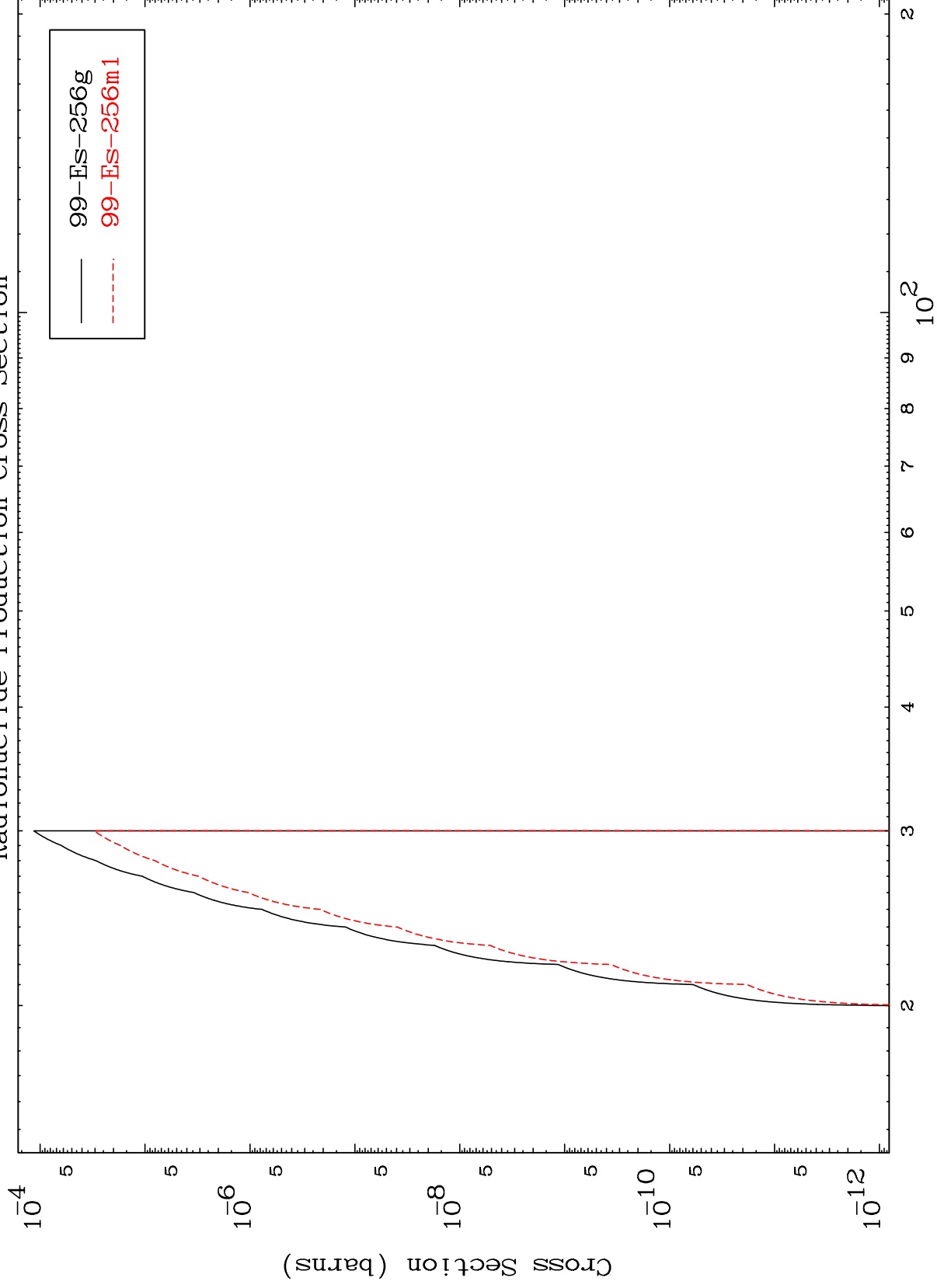


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(n,2n) d

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

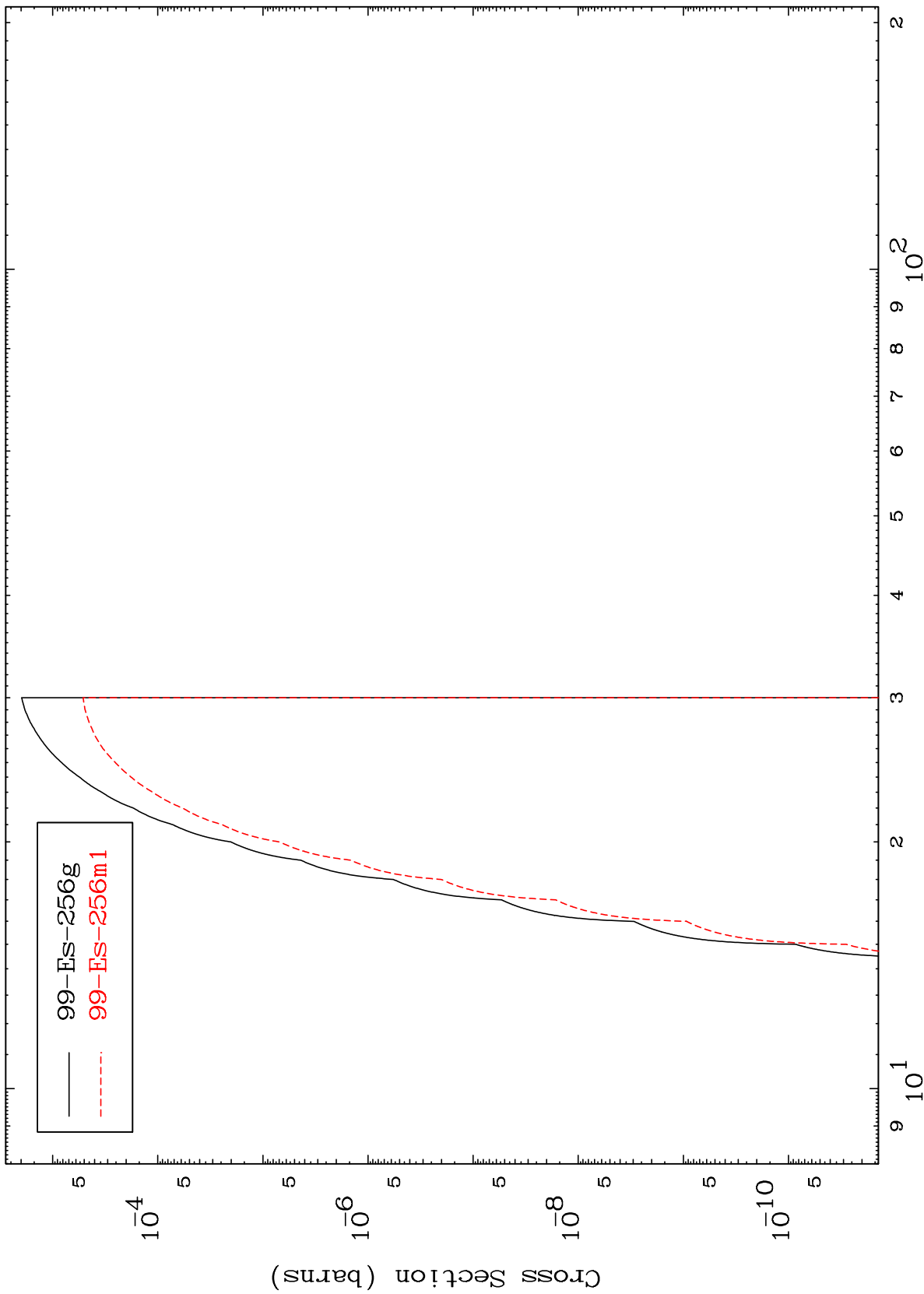
100-Fm-259

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(n,n') t

100-Fm-259

Radionuclide Production Cross Section



26

Incident Energy (MeV)

100-Fm-259

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(n,3n) p

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Radionuclide Production Cross Section

