

Program EVALPLOT  
(Version 2015-2)

by

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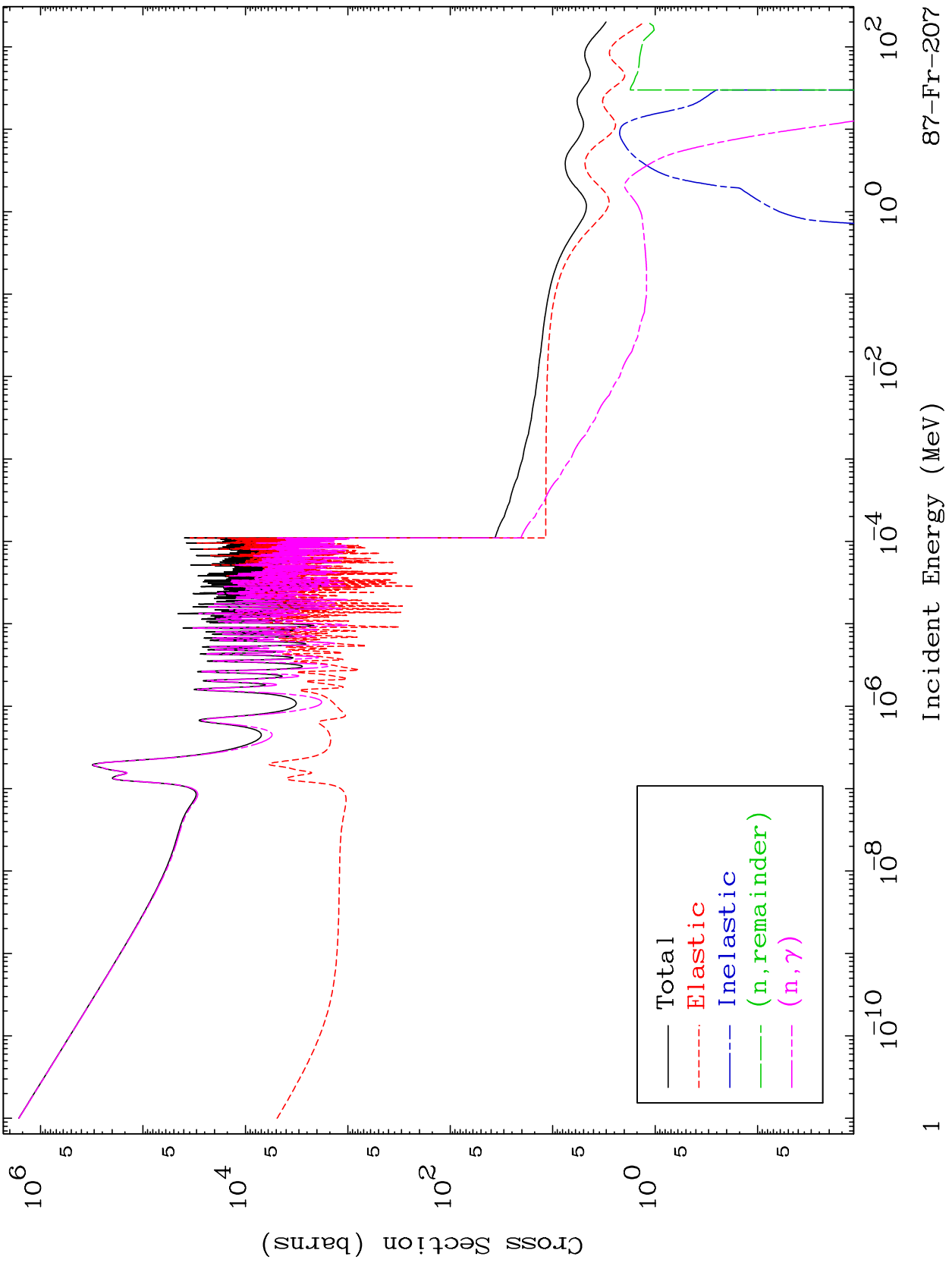
Press Mouse Button to Start

MAT 8710

Major

293 Kelvin Cross Sections

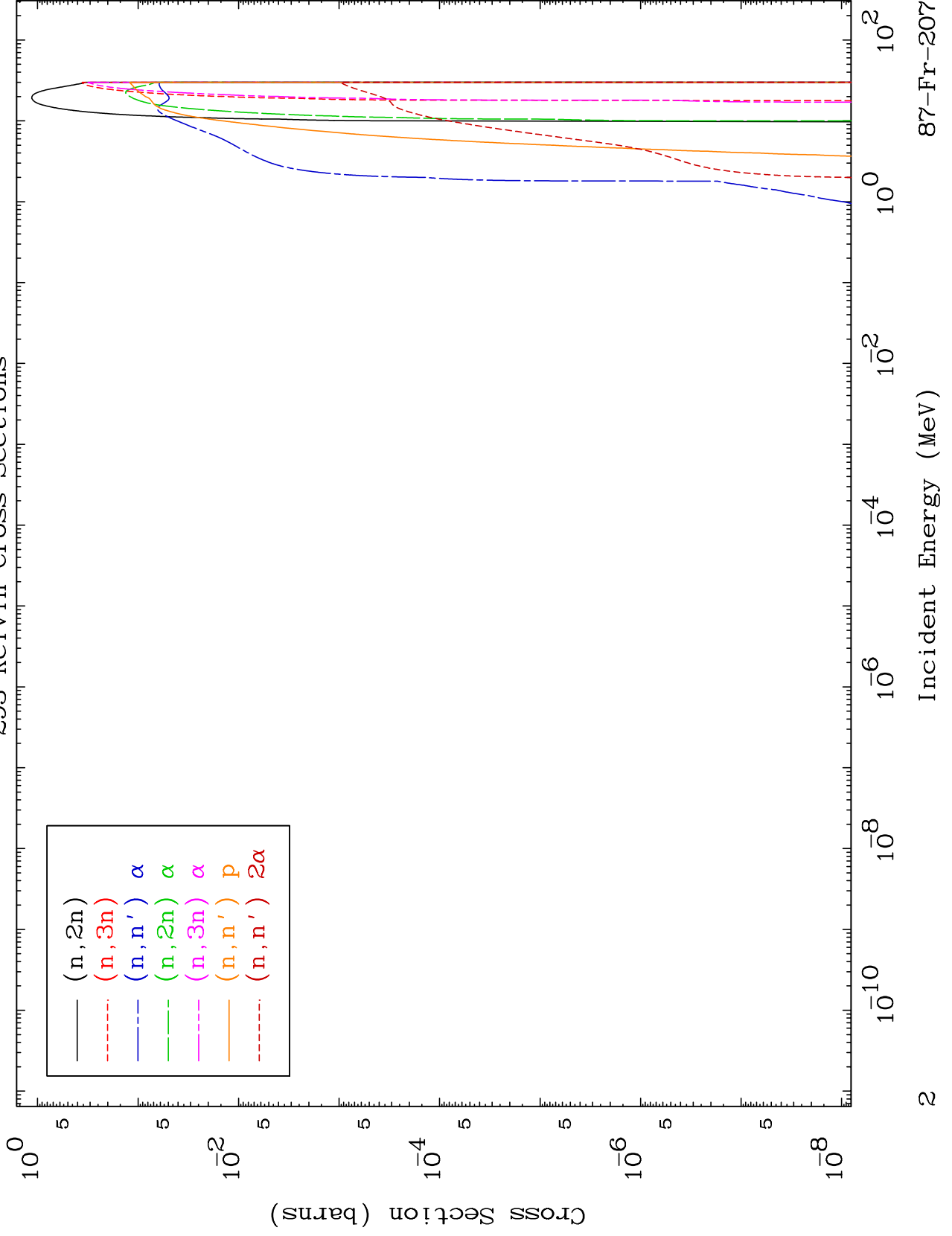
87-Fr-207



MAT 8710

Neutron Production  
293 Kelvin Cross Sections

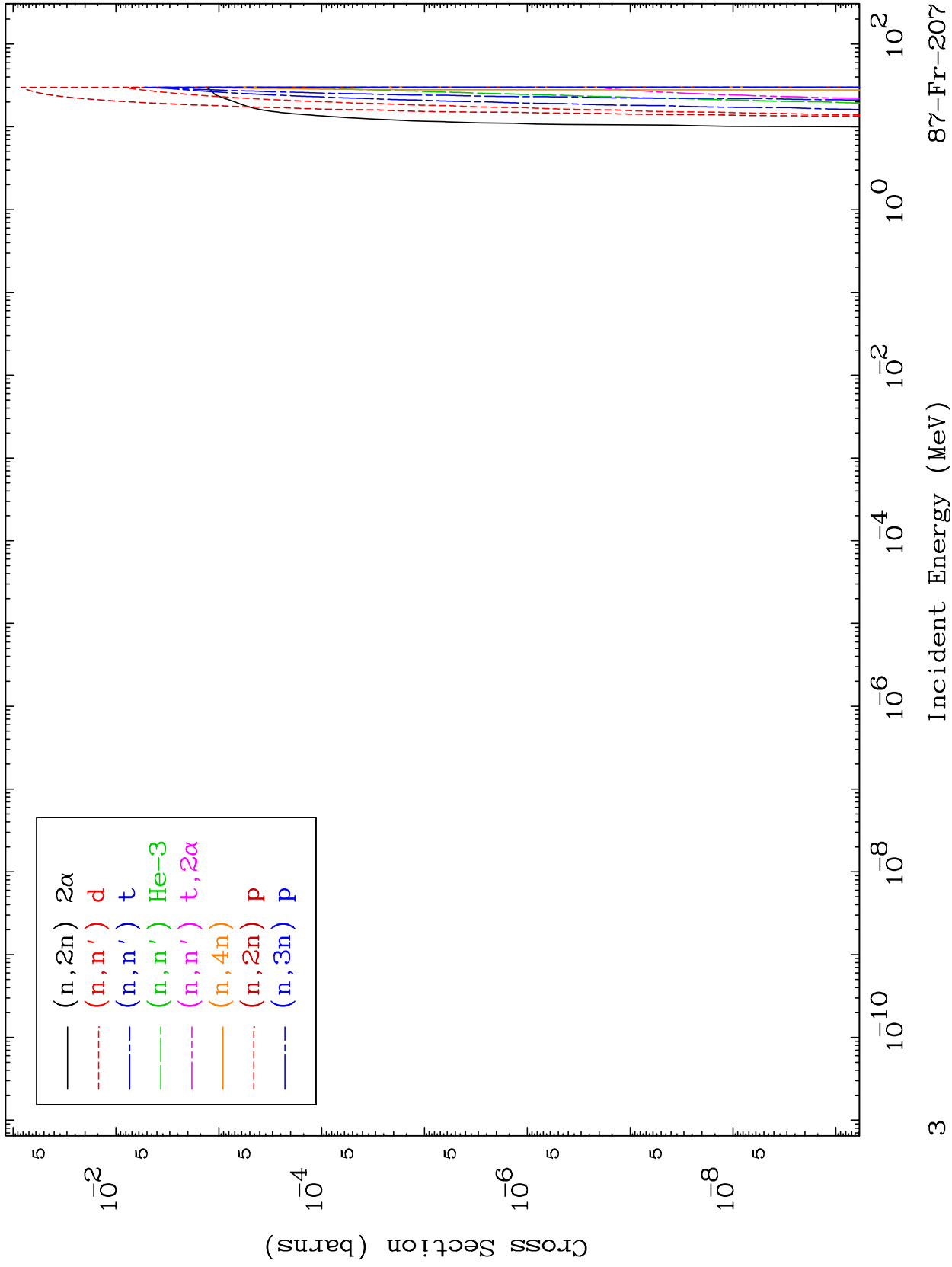
87-Fr-207



MAT 8710

Neutron Production  
293 Kelvin Cross Sections

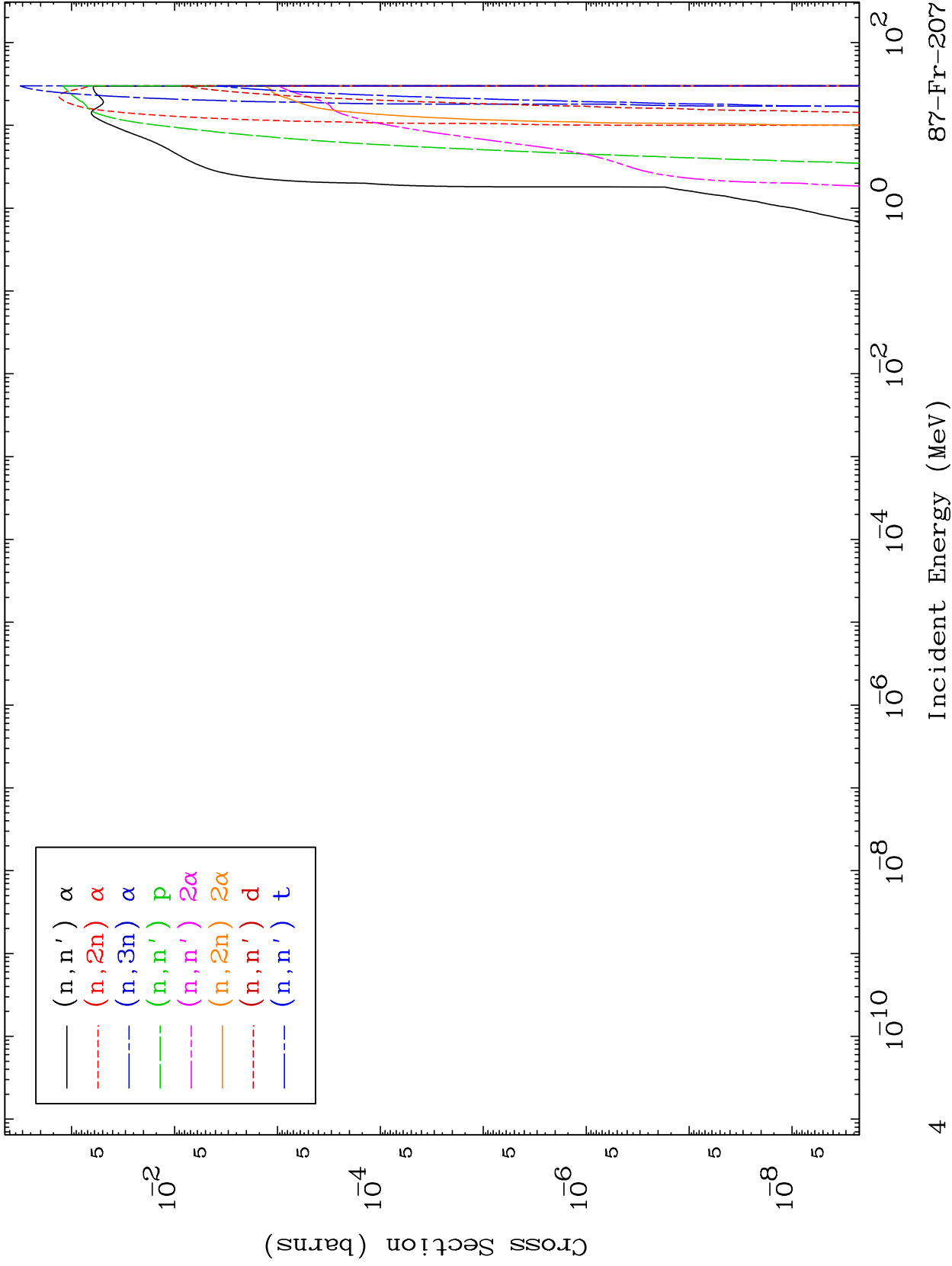
87-Fr-207



MAT 8710

Charged Particle  
293 Kelvin Cross Sections

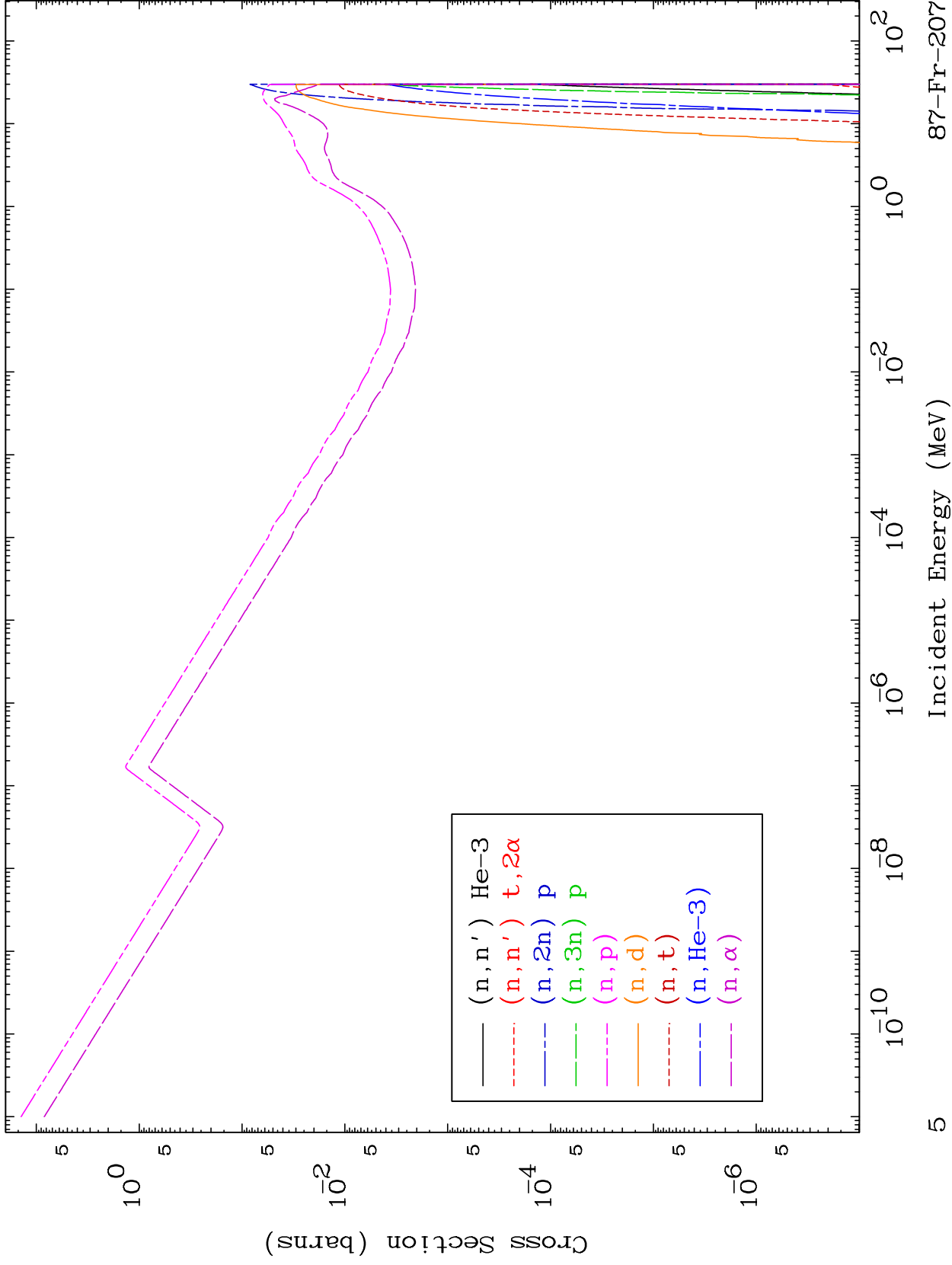
87-Fr-207



MAT 8710

Charged Particle  
293 Kelvin Cross Sections

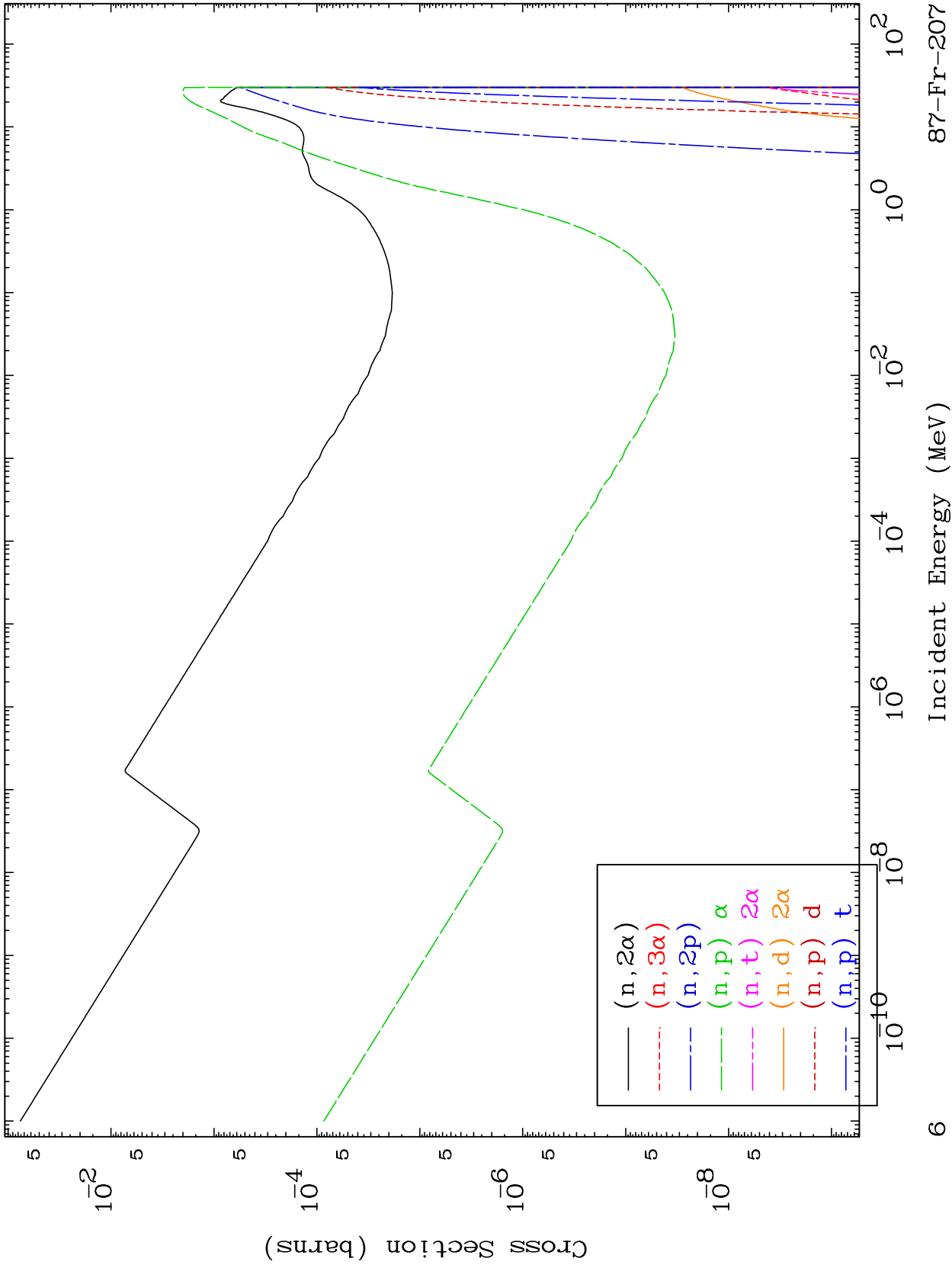
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MAT 8710

Charged Particle  
293 Kelvin Cross Sections

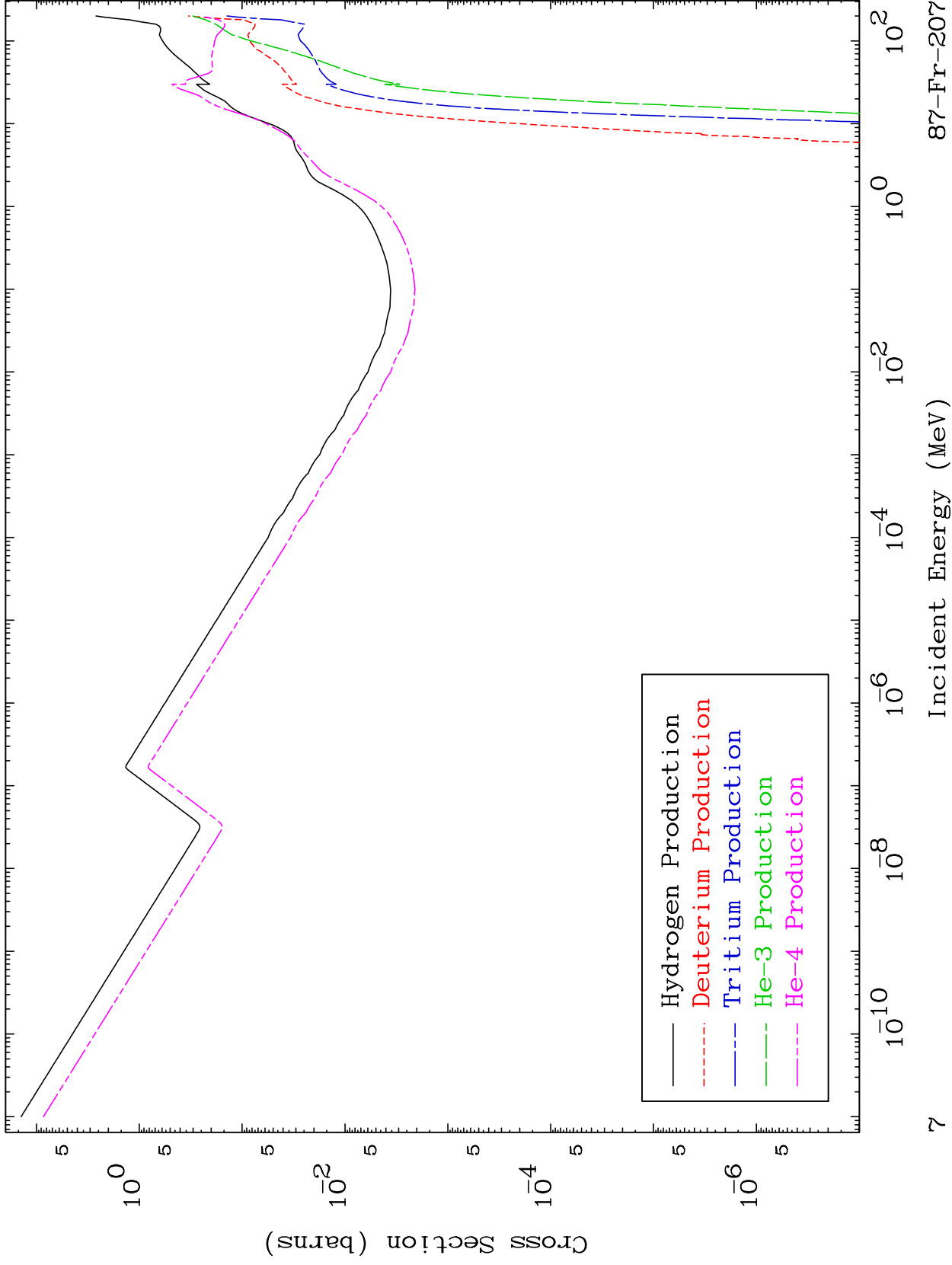
87-Fr-207



MAT 8710

Particle Production  
293 Kelvin Cross Sections

87-Fr-207



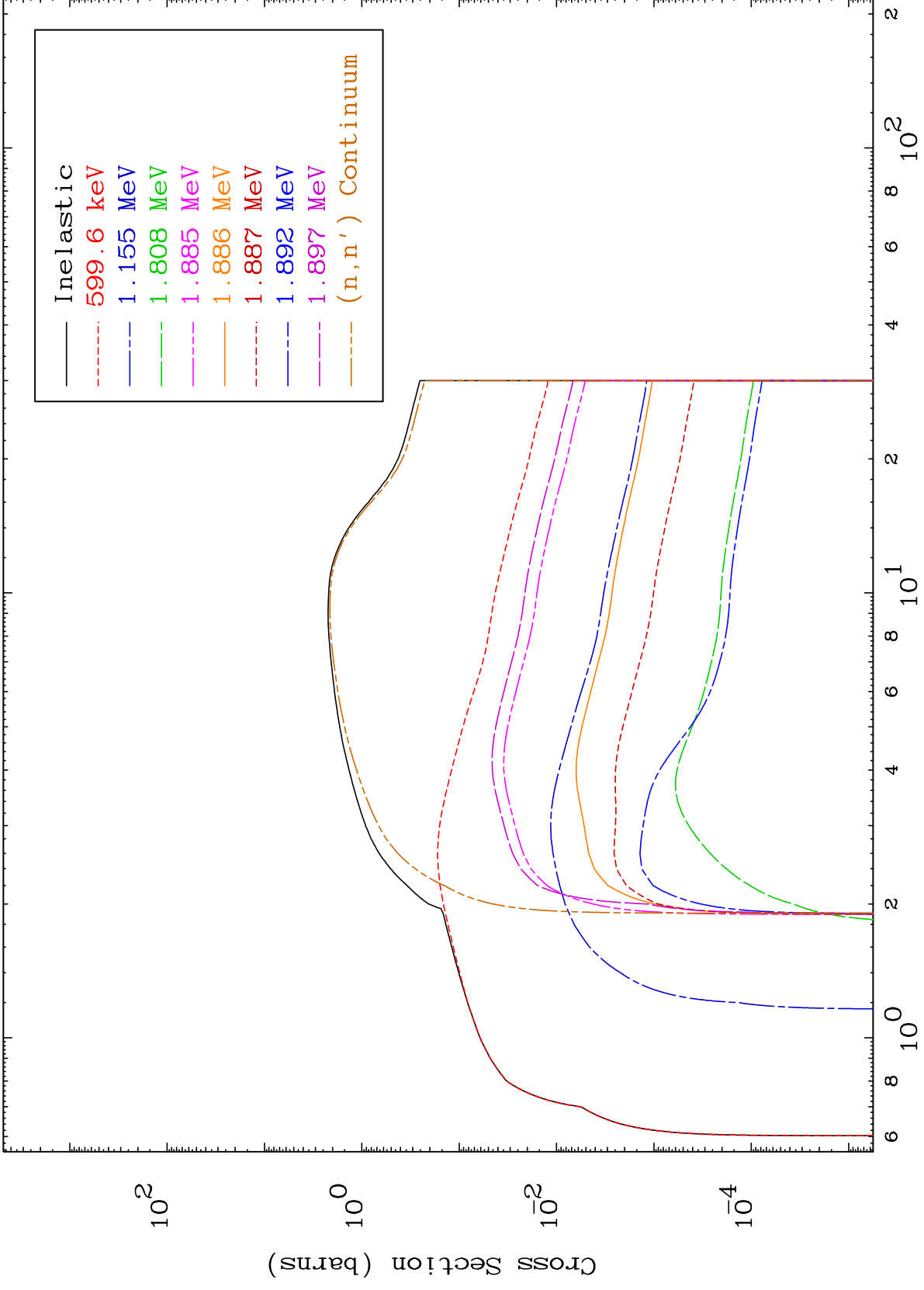


MAT 8710

(n,n') Level

87-Fr-207

293 Kelvin Cross Sections



8

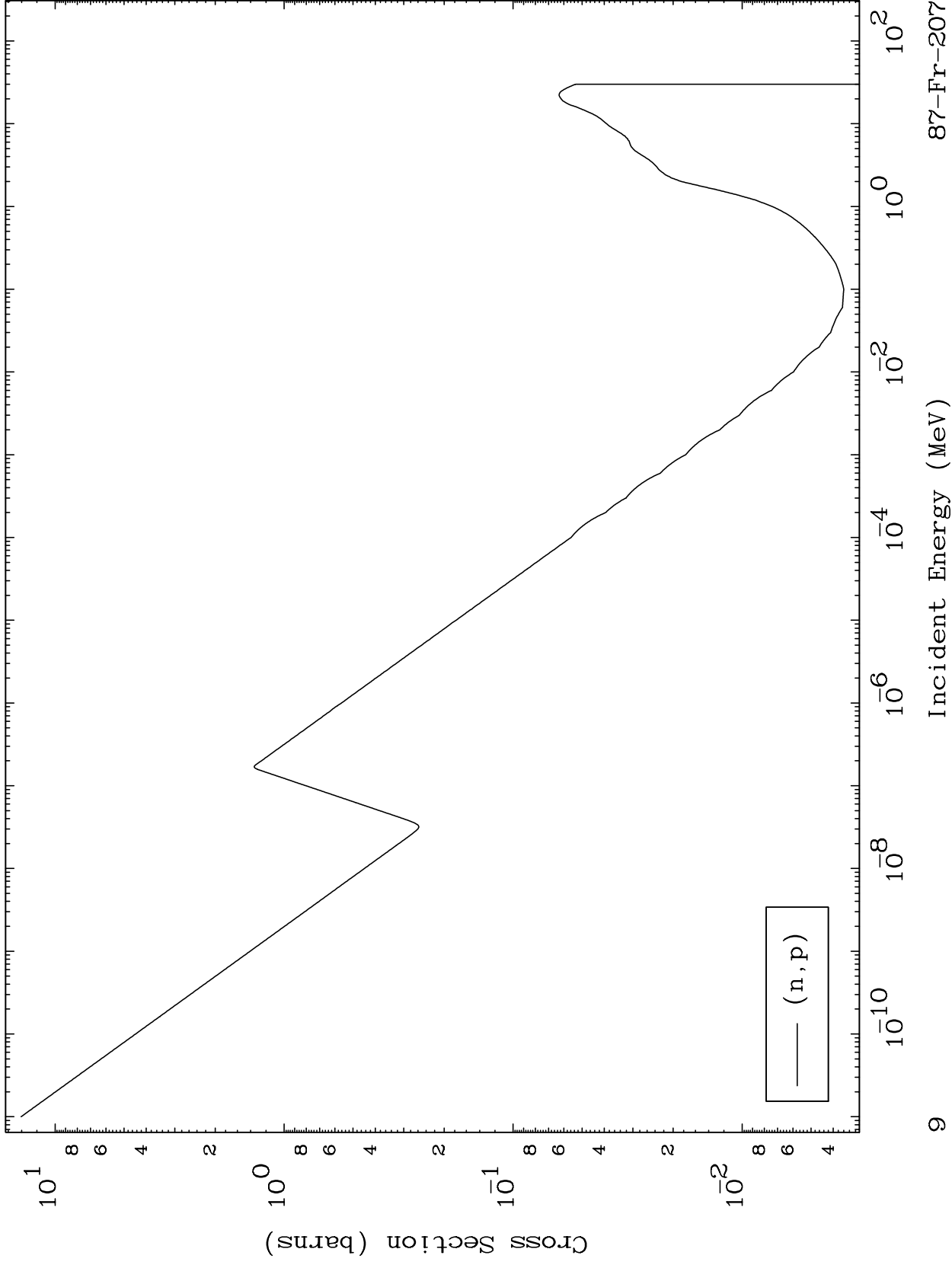
Incident Energy (MeV)

87-Fr-207

MAT 8710

(n,p) Levels  
293 Kelvin Cross Sections

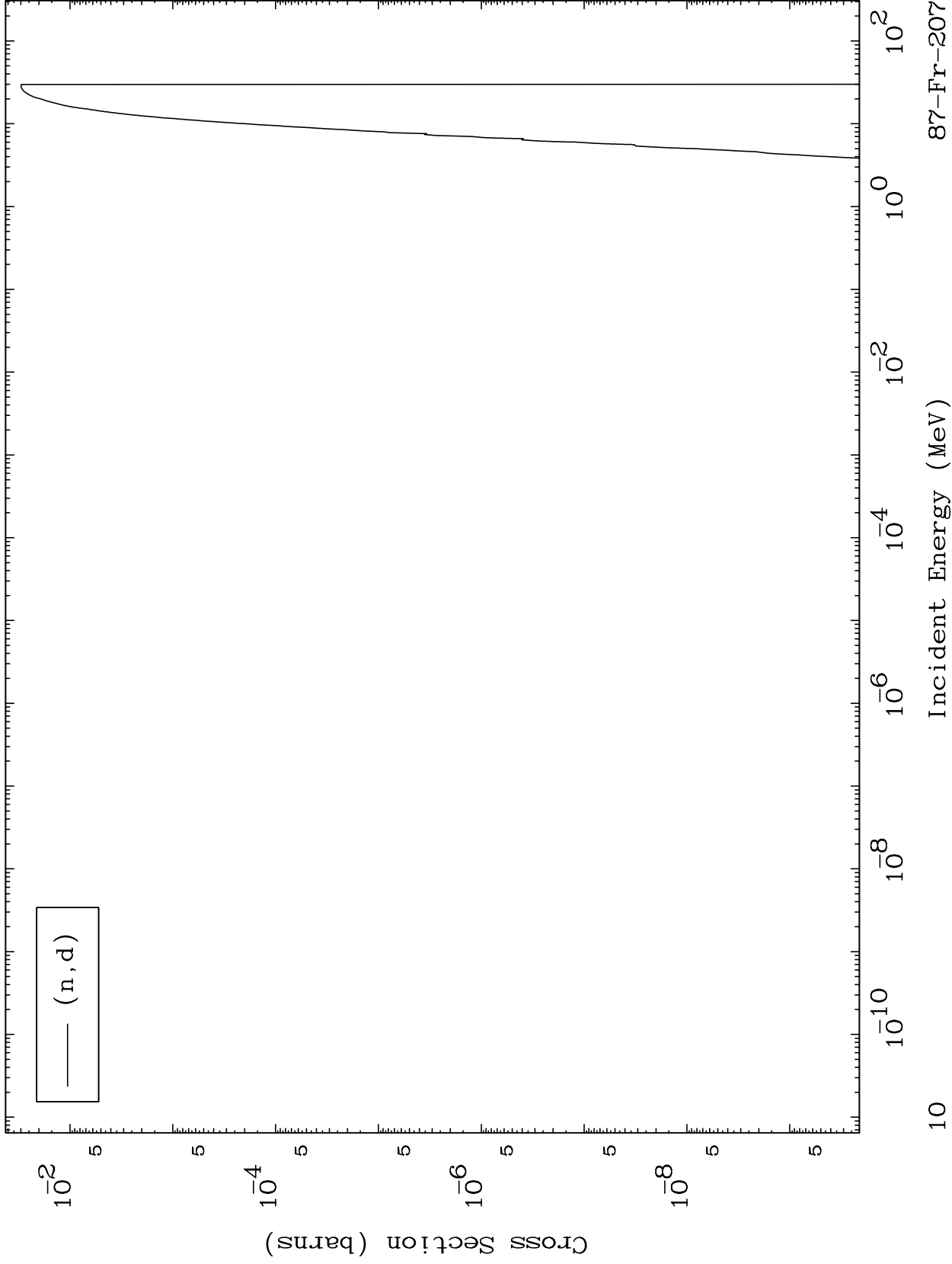
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MAT 8710

(n,d) Levels  
293 Kelvin Cross Sections

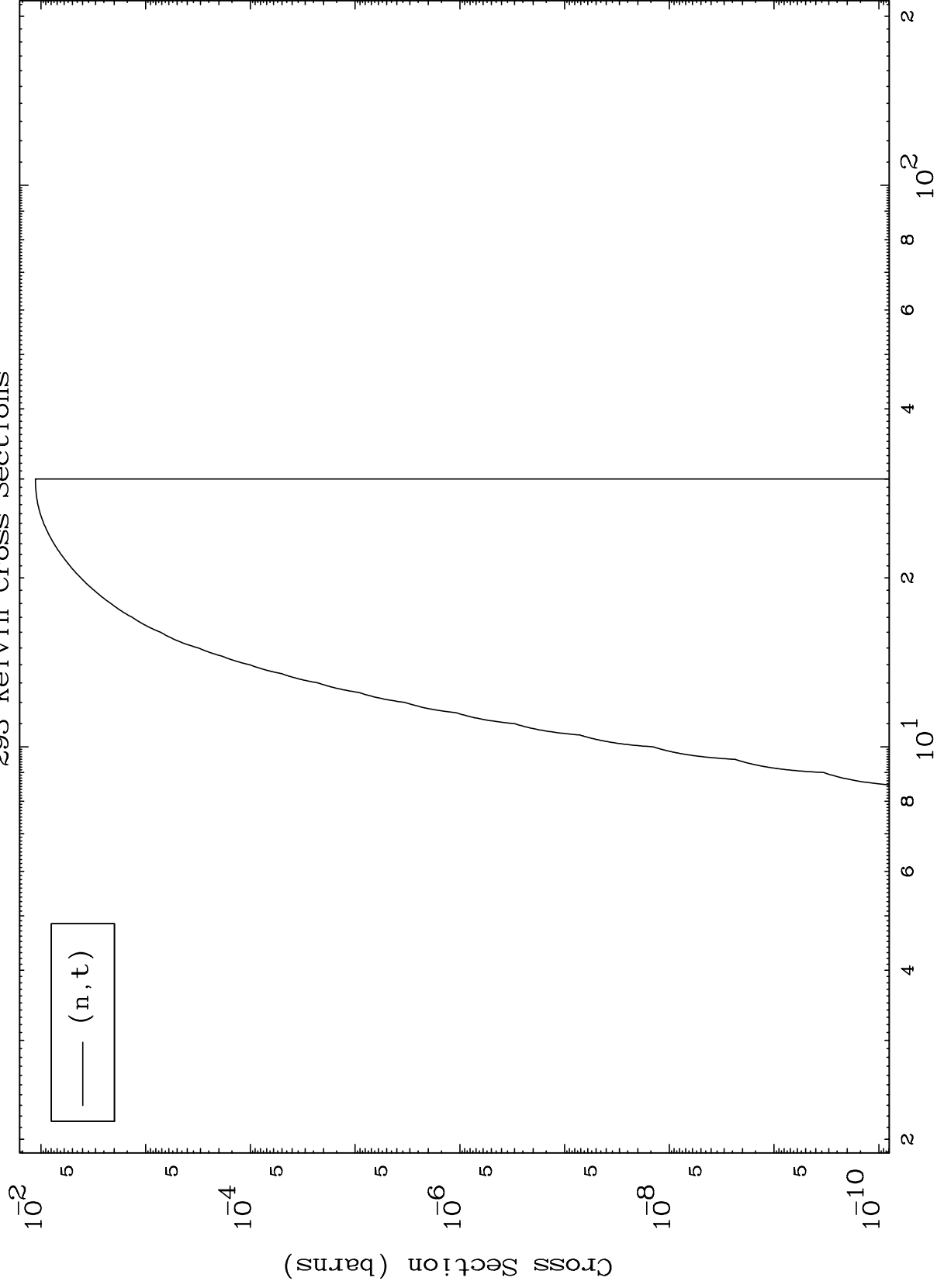
87-Fr-207



MAT 8710

(n,t) Levels  
293 Kelvin Cross Sections

87-Fr-207



11

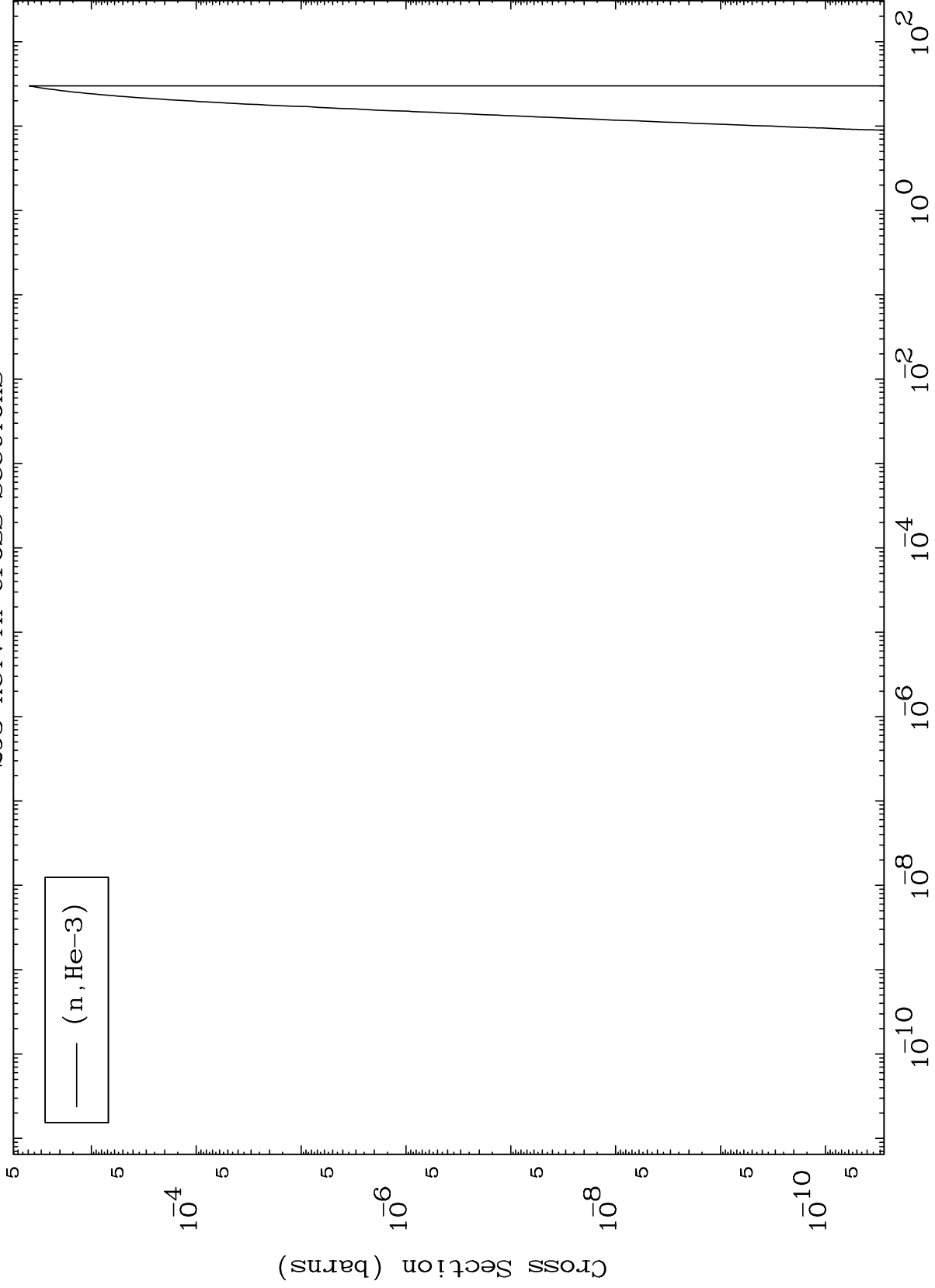
Incident Energy (MeV)

87-Fr-207

MAT 8710

(n,He3) Levels  
293 Kelvin Cross Sections

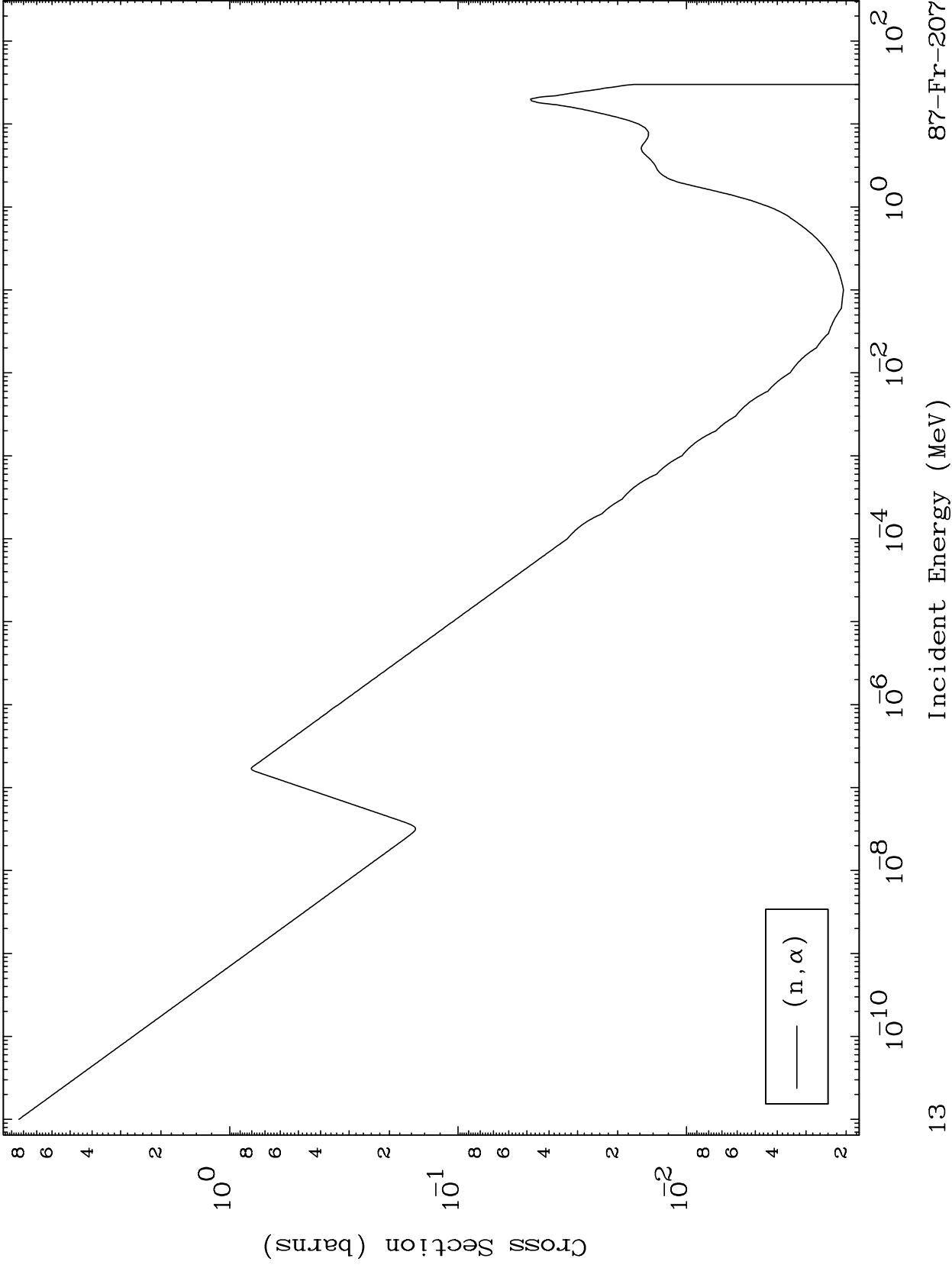
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MAT 8710

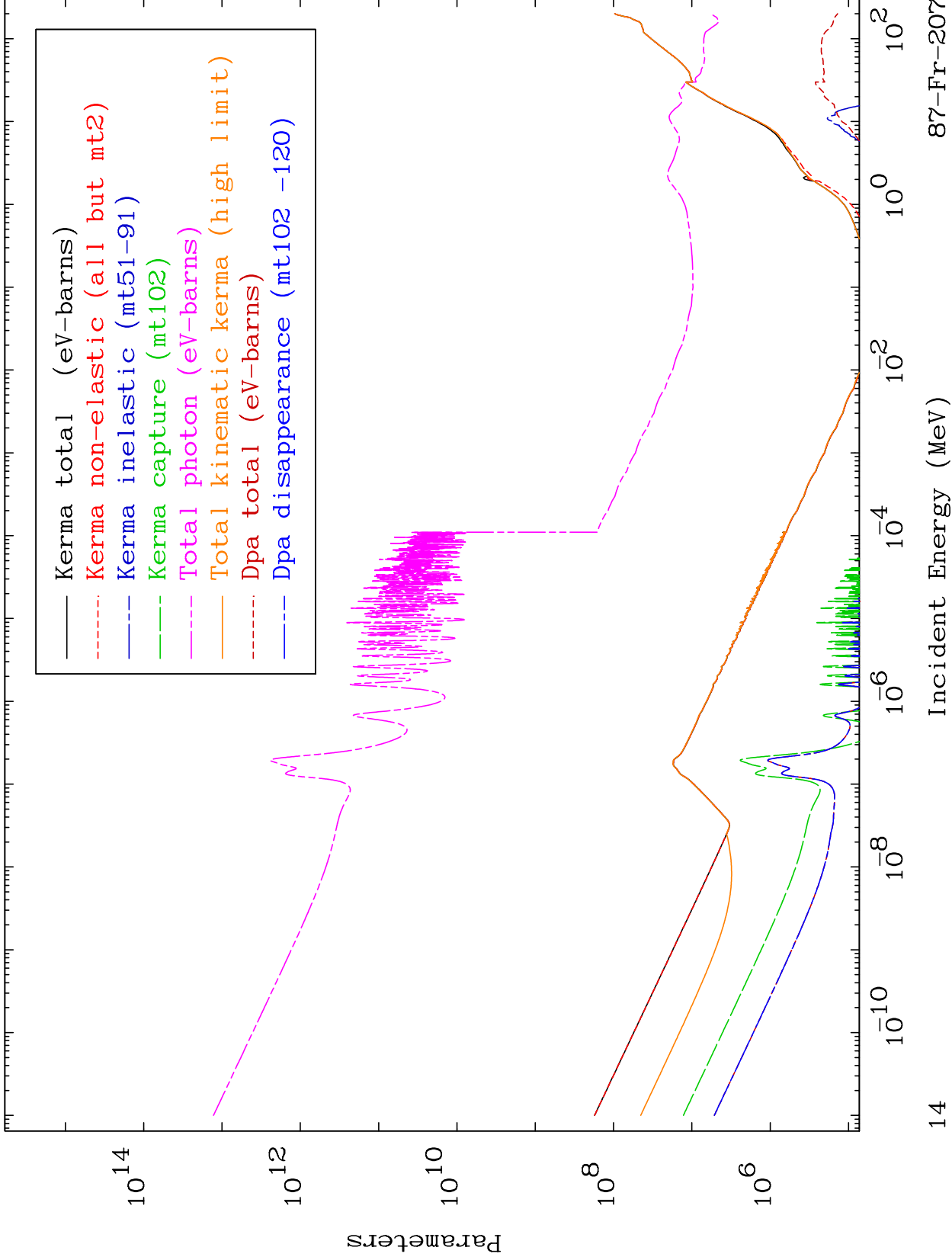
(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

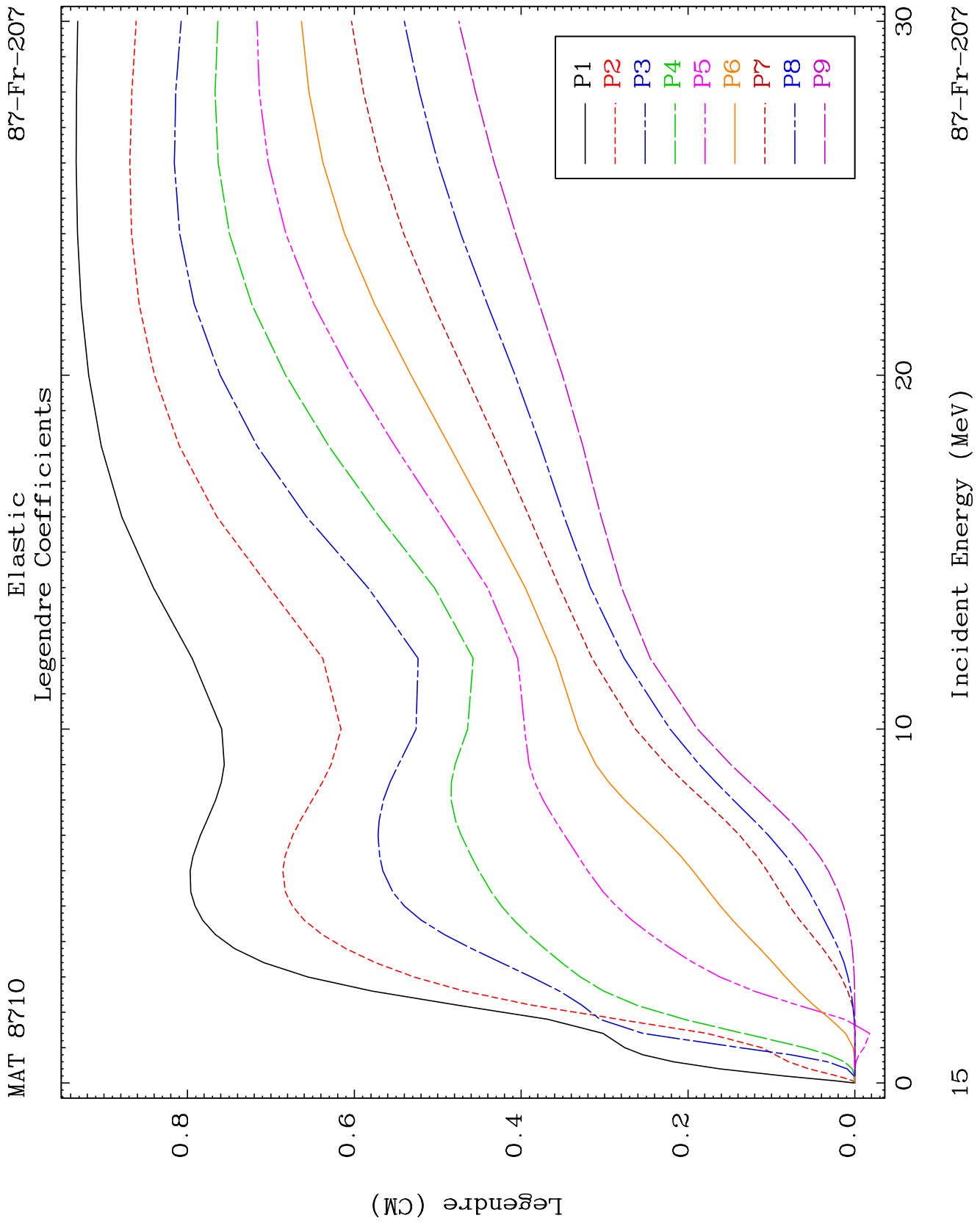
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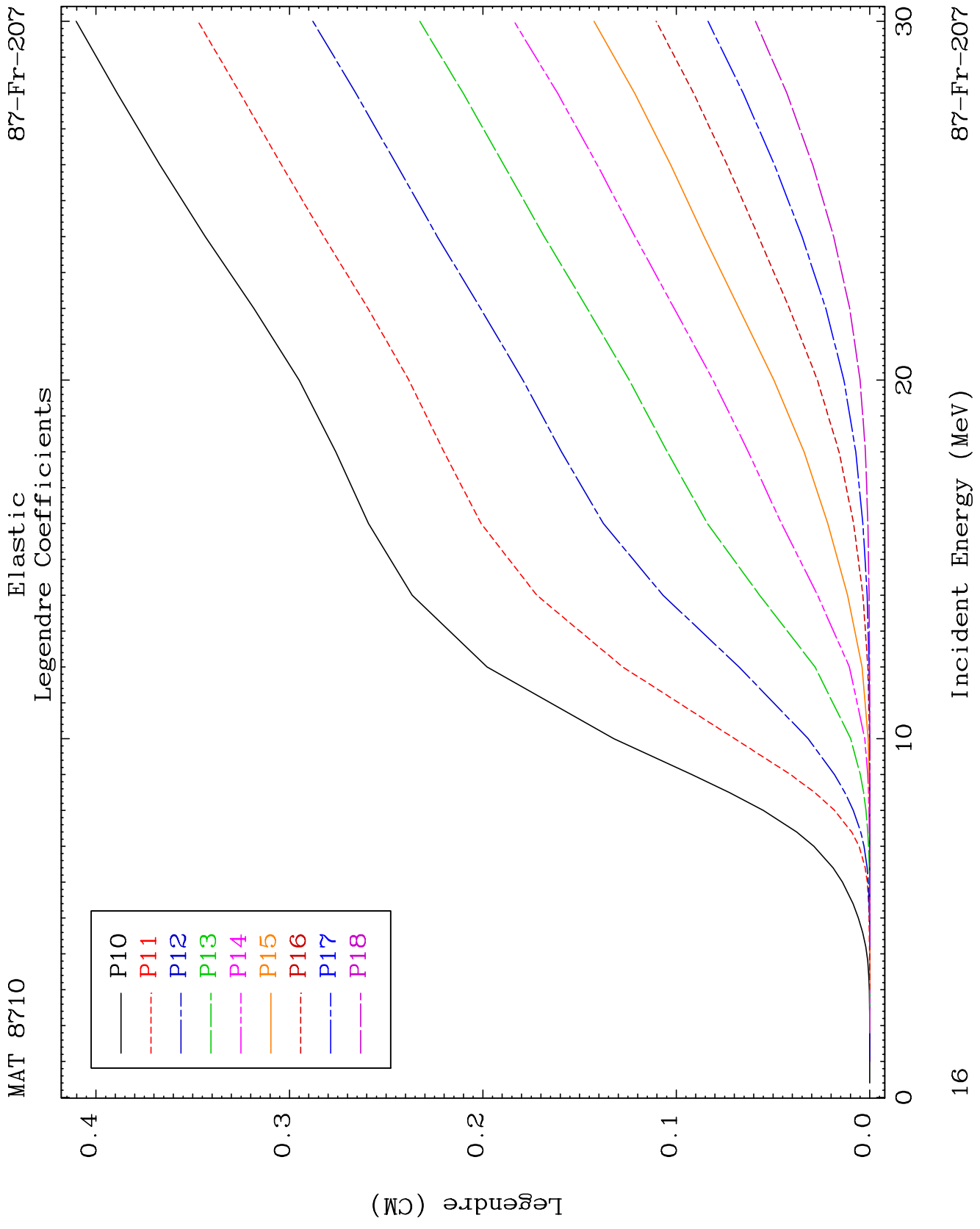
13

87-Fr-207





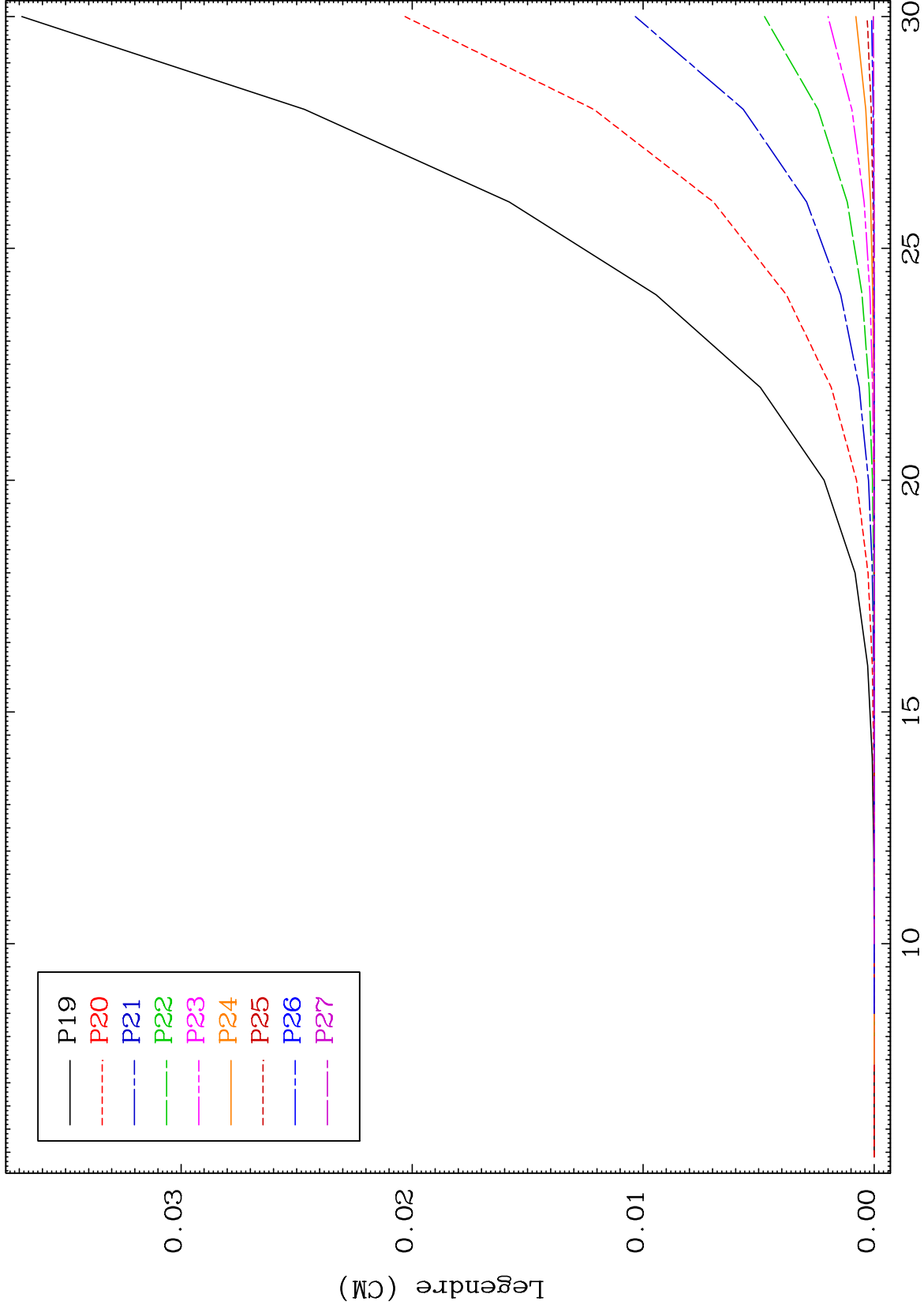




MAT 8710

Elastic  
Legendre Coefficients

87-Fr-207



17

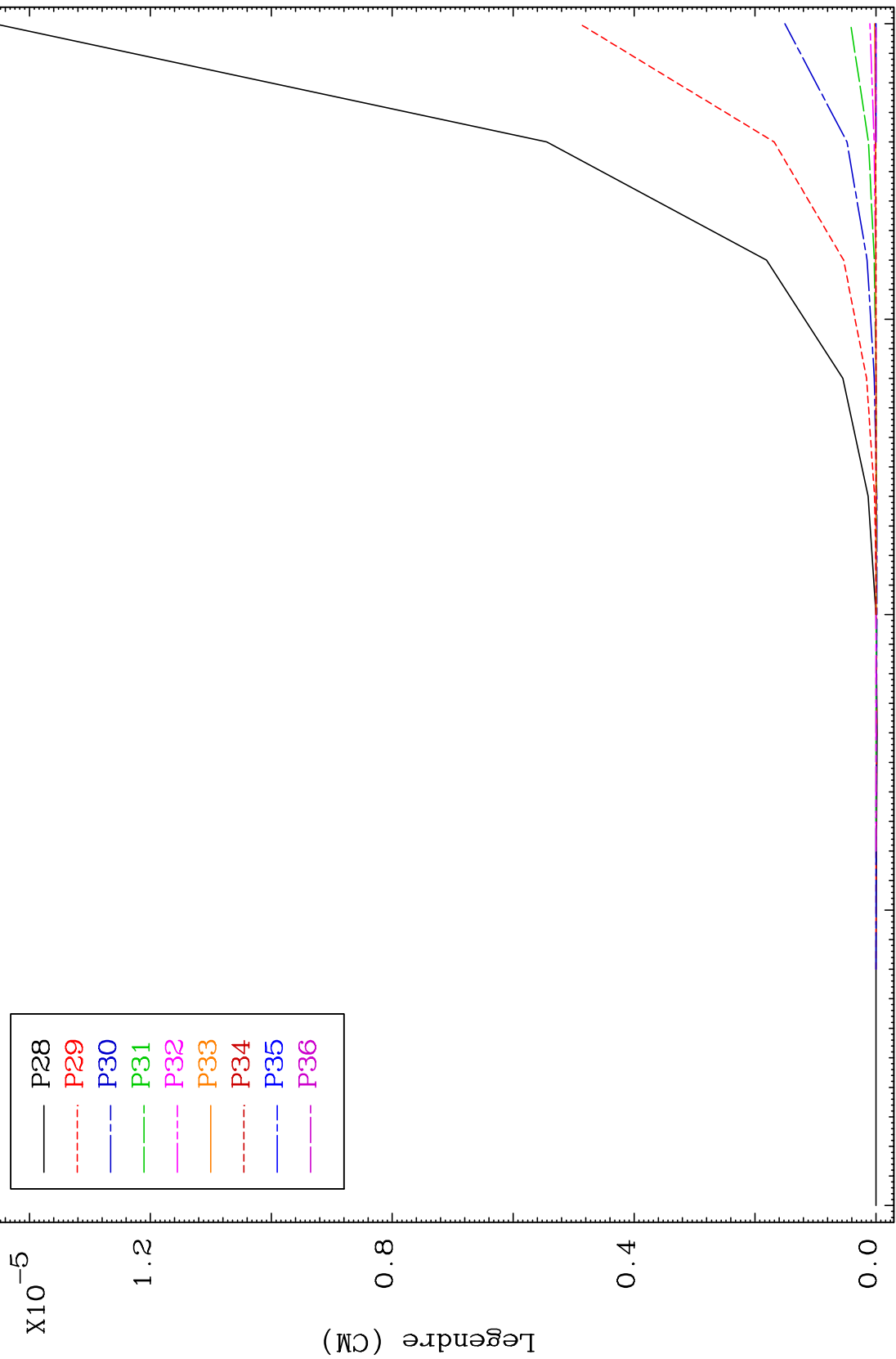
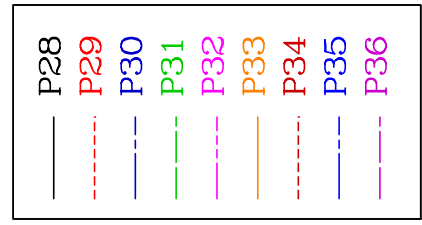
Incident Energy (MeV)

87-Fr-207

MAT 8710

Elastic Legendre Coefficients

87-Fr-207



87-Fr-207

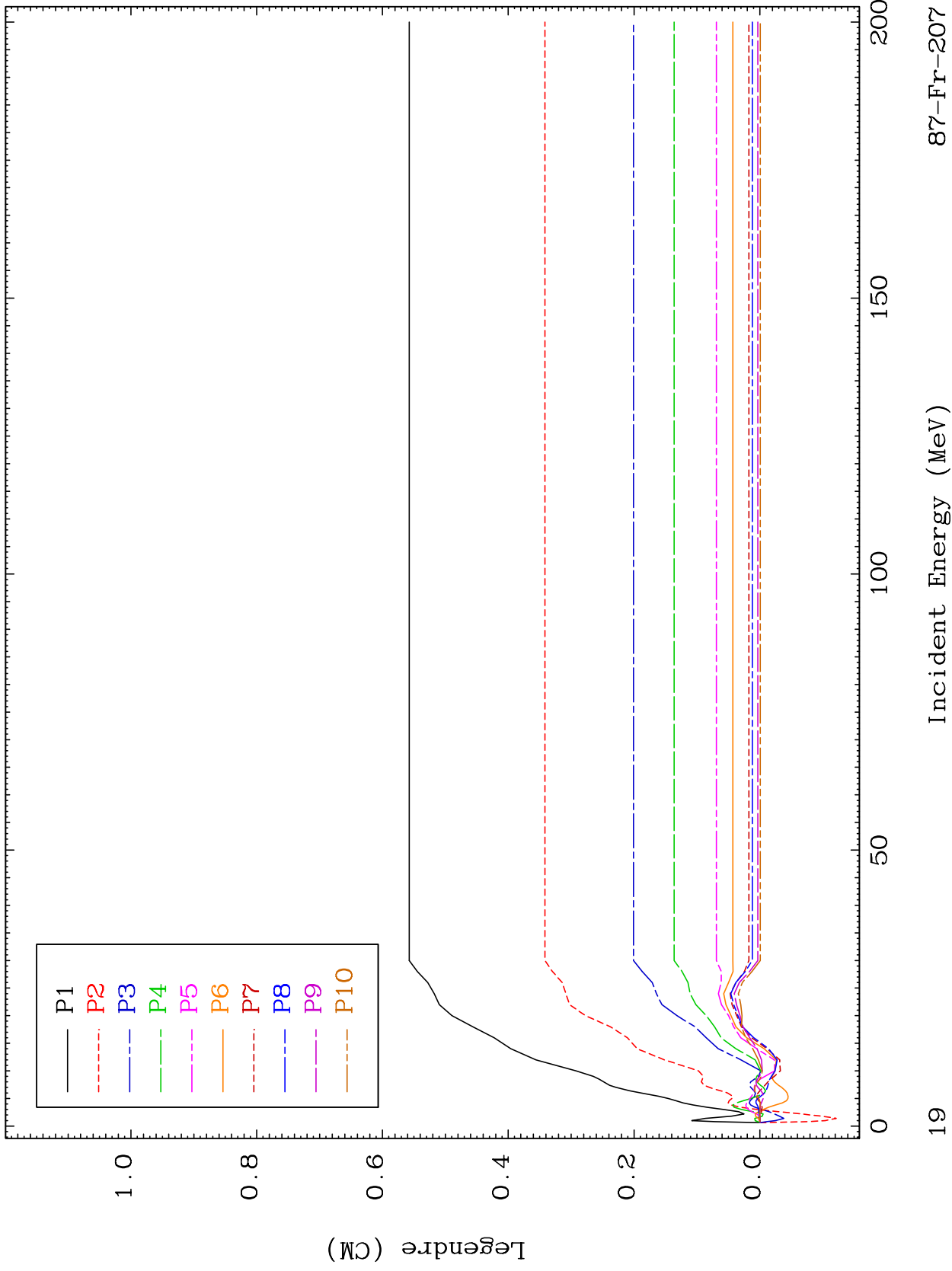
Incident Energy (MeV)

18

MAT 8710

599.6 keV (n,n') Level  
Legendre Coefficients

87-Fr-207

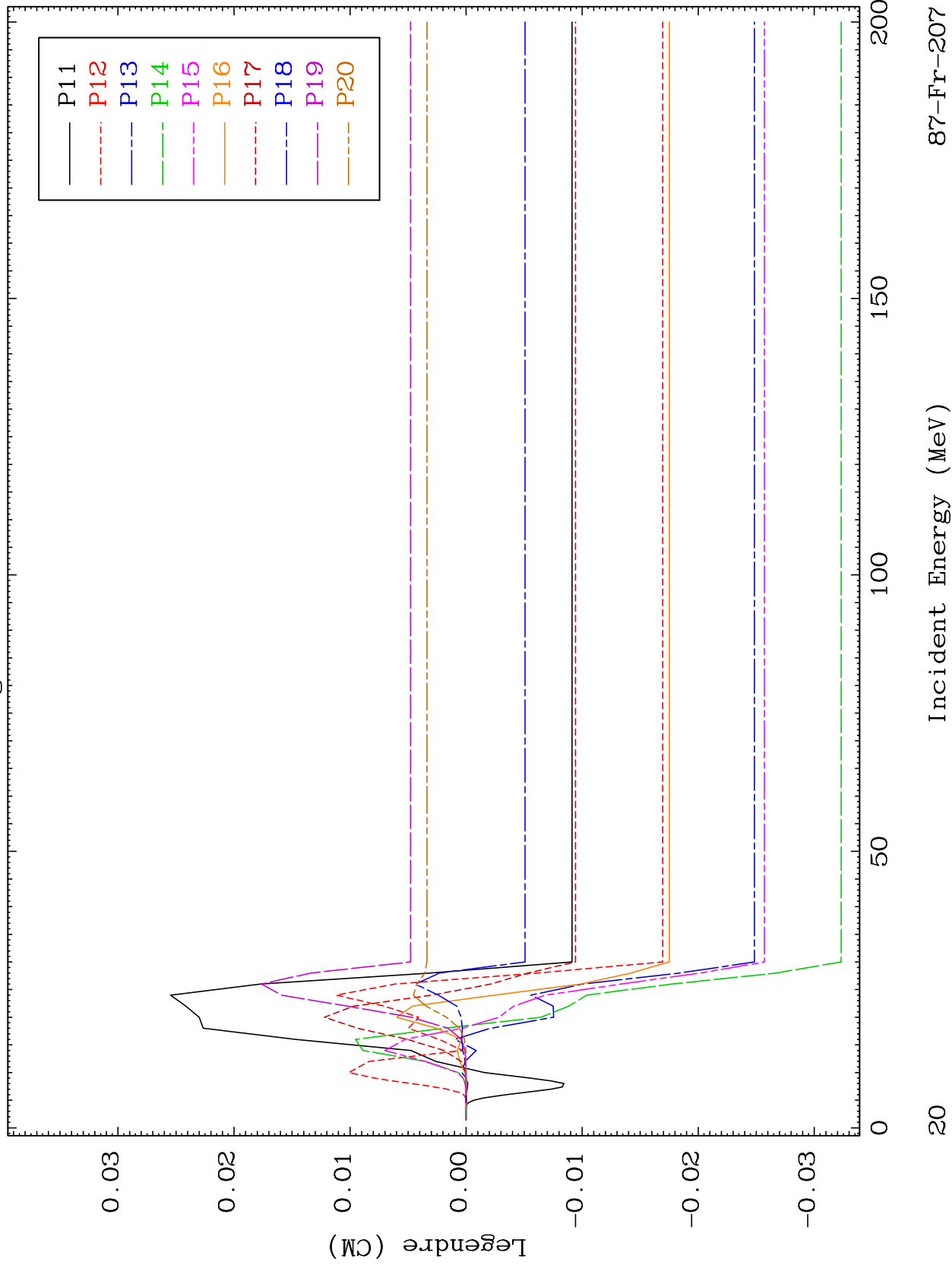


19

MAT 8710

599.6 keV (n,n') Level  
Legendre Coefficients

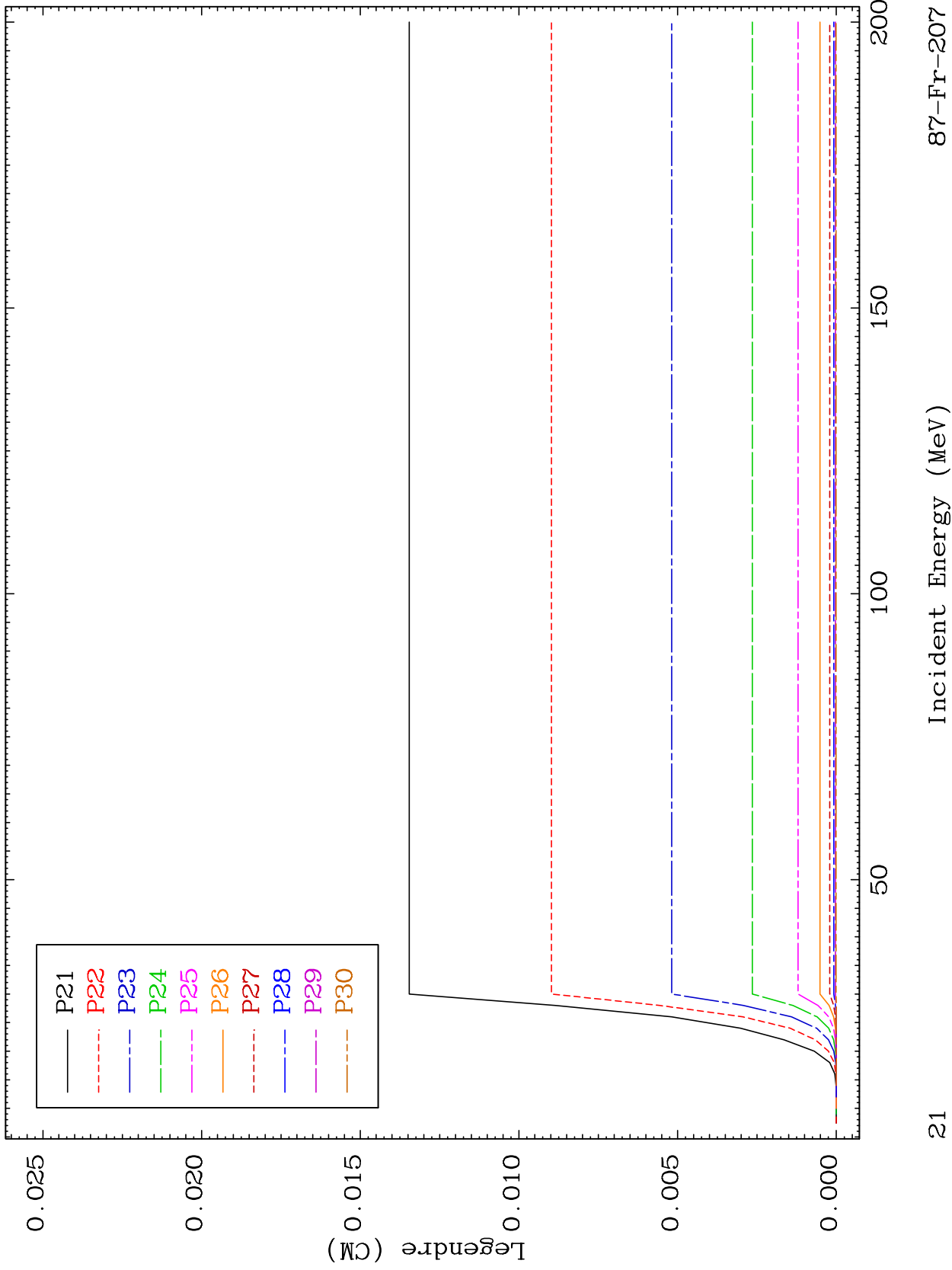
87-Fr-207



87-Fr-207

Incident Energy (MeV)

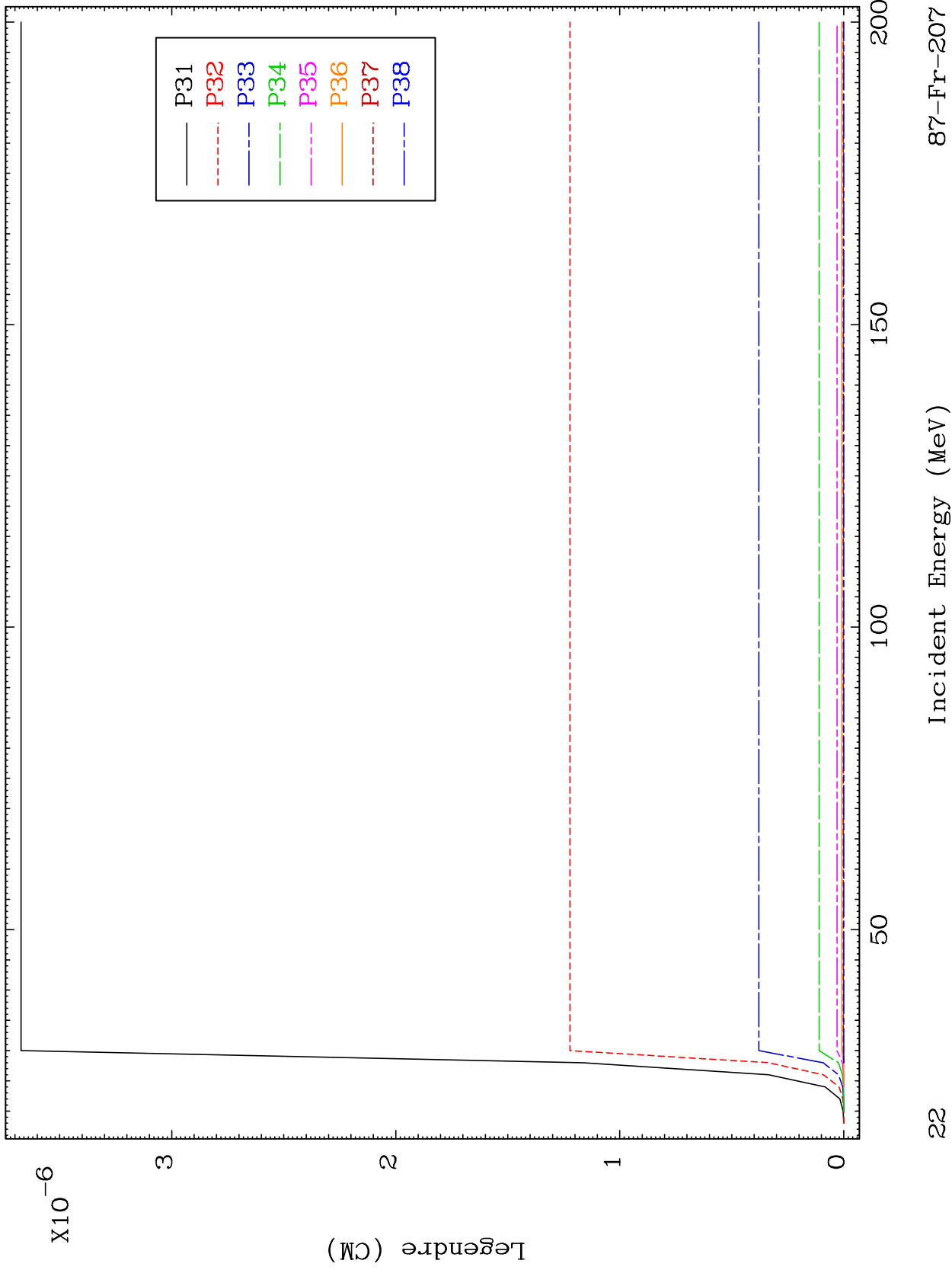
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MAT 8710

599.6 keV (n,n') Level  
Legendre Coefficients

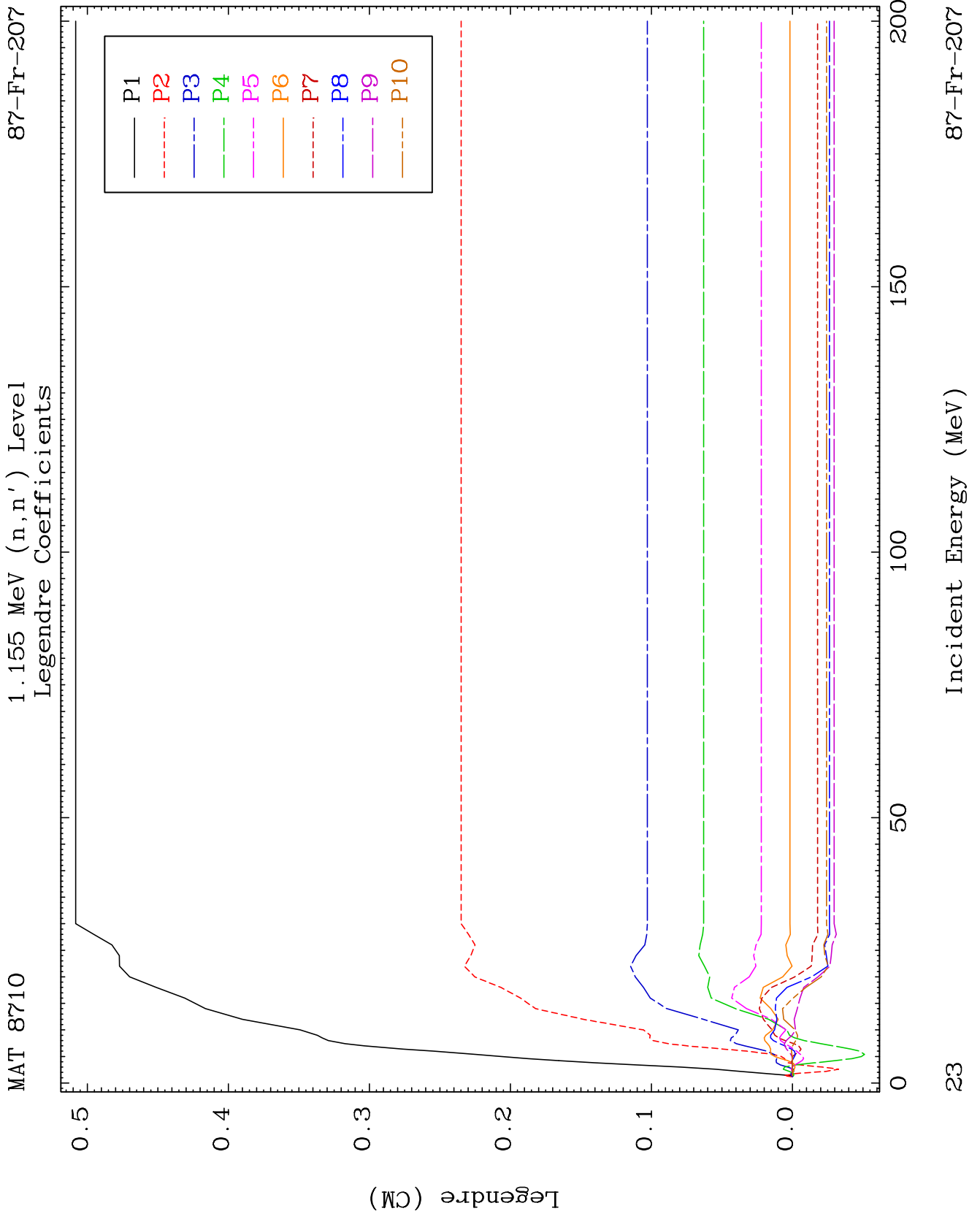
87-Fr-207



22

Incident Energy (MeV)

87-Fr-207

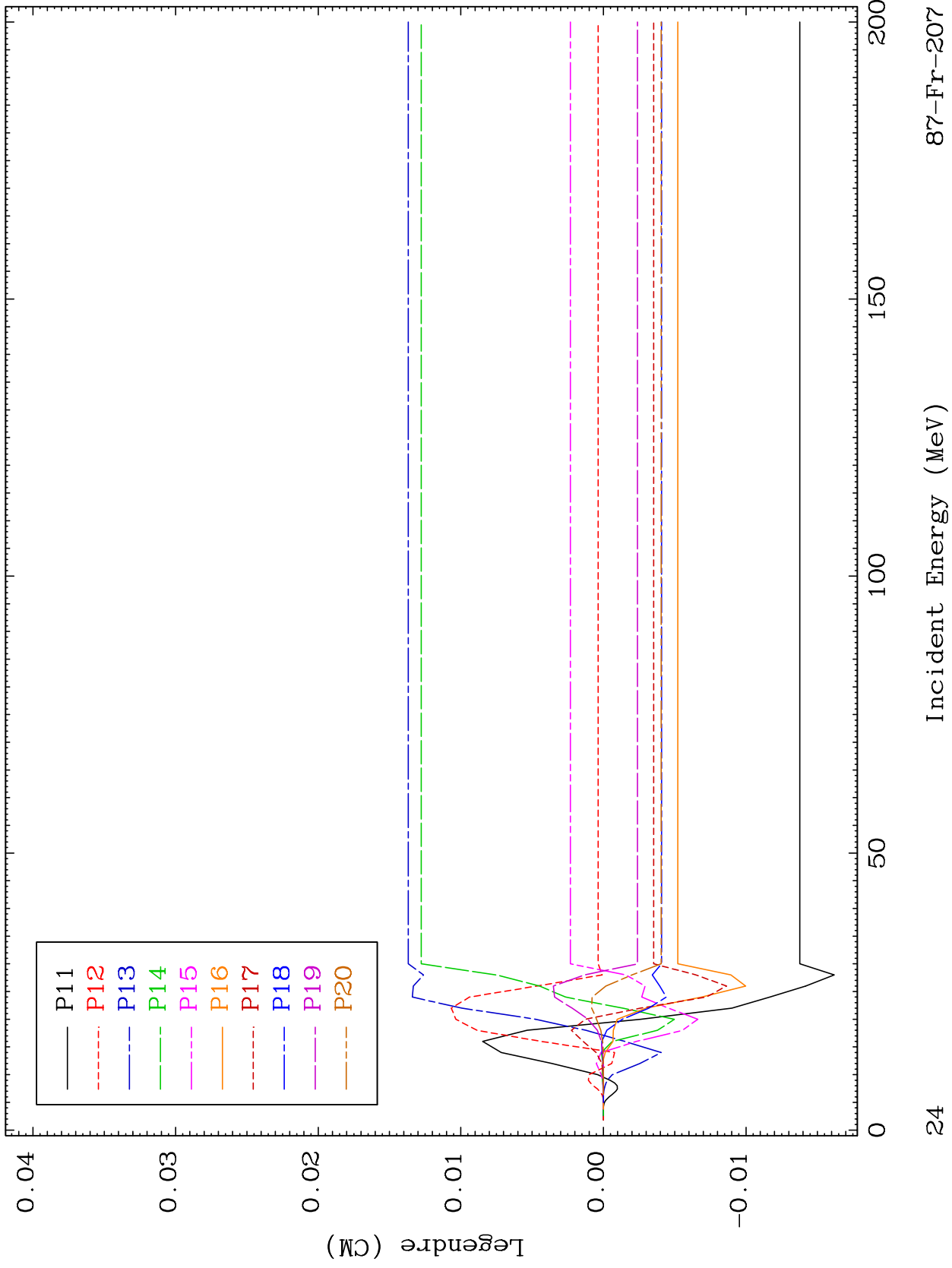




MAT 8710

1.155 MeV (n,n') Level  
Legendre Coefficients

87-Fr-207



24

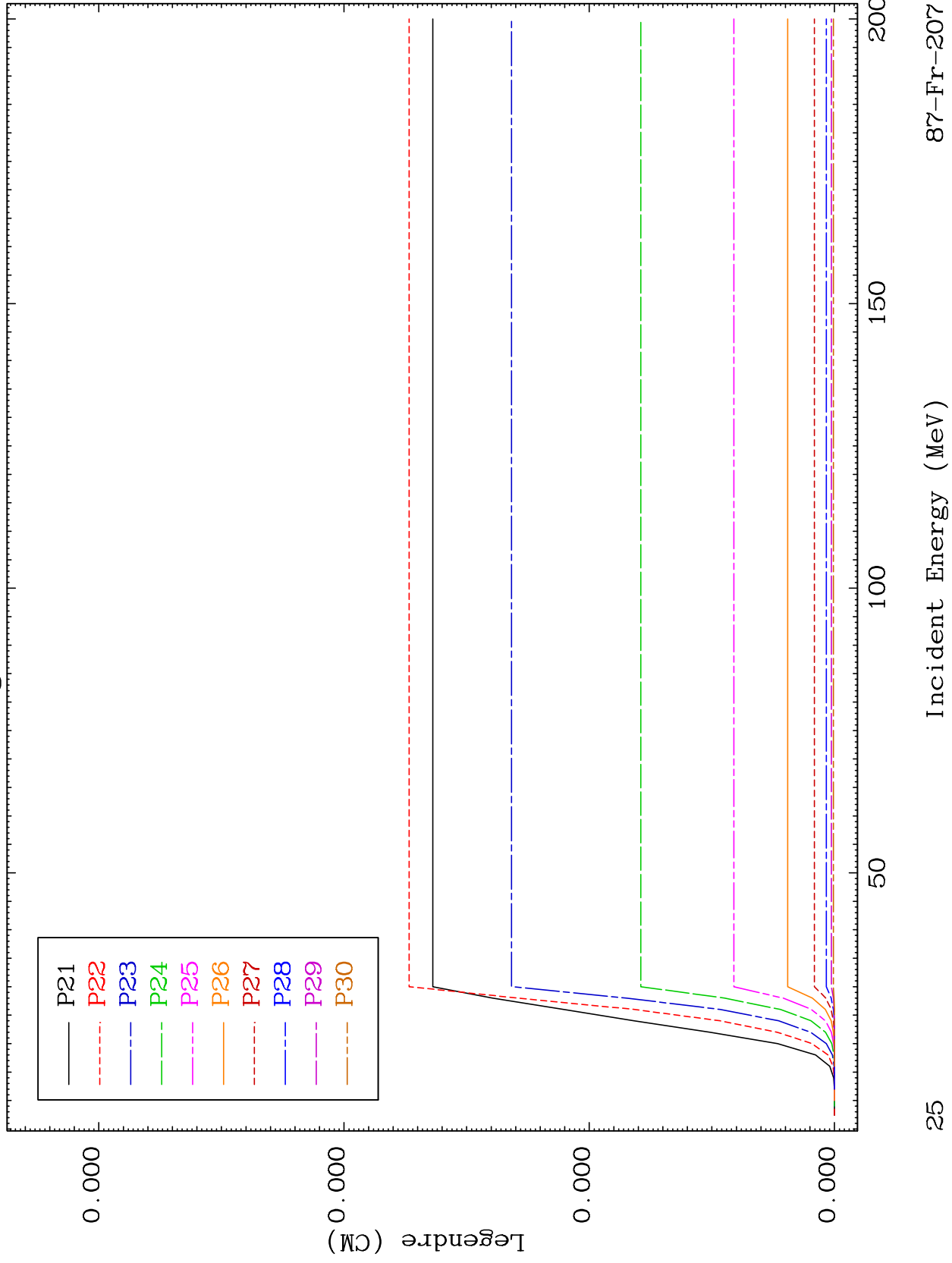
Incident Energy (MeV)

87-Fr-207

MAT 8710

1.155 MeV (n,n') Level  
Legendre Coefficients

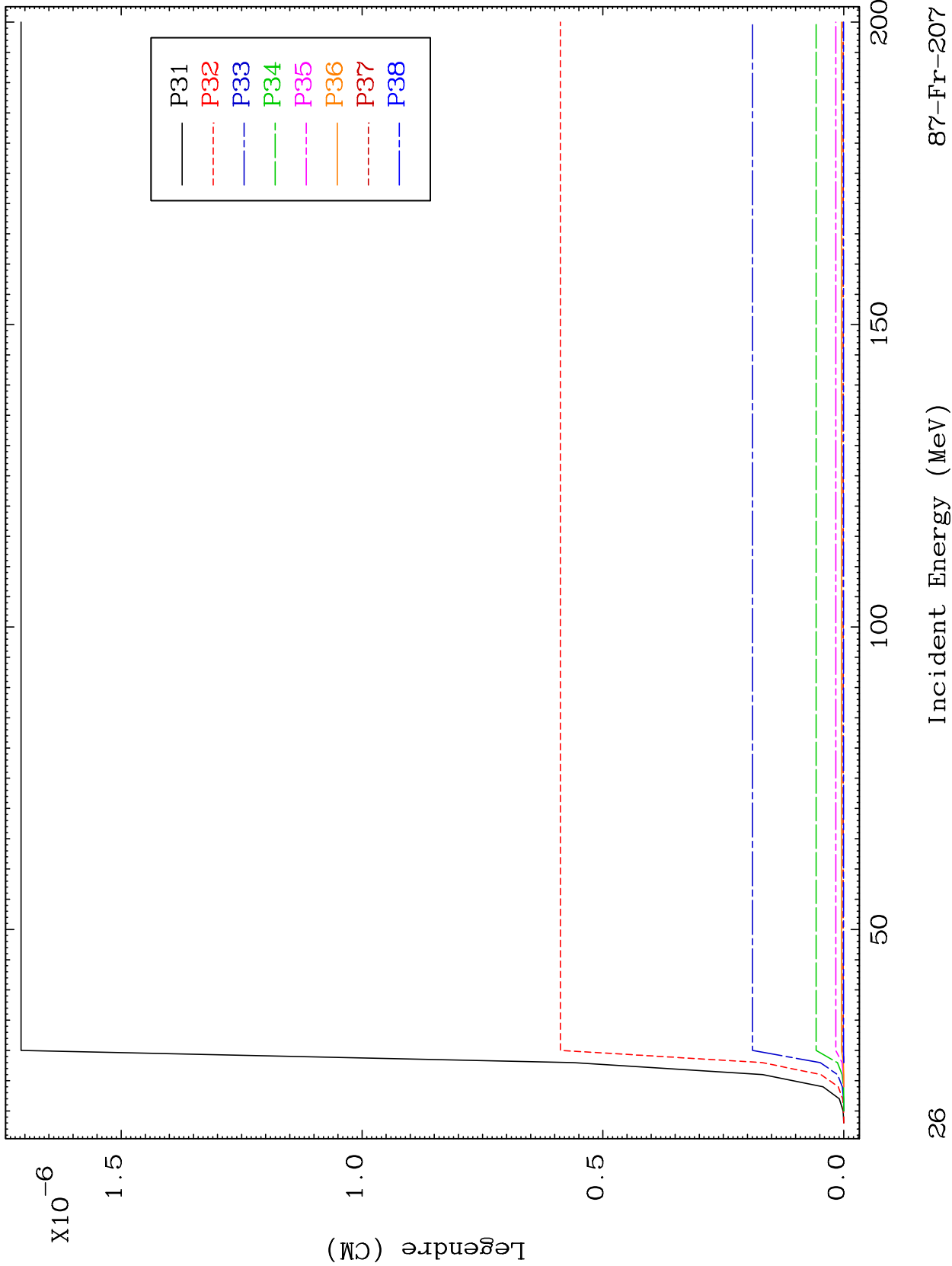
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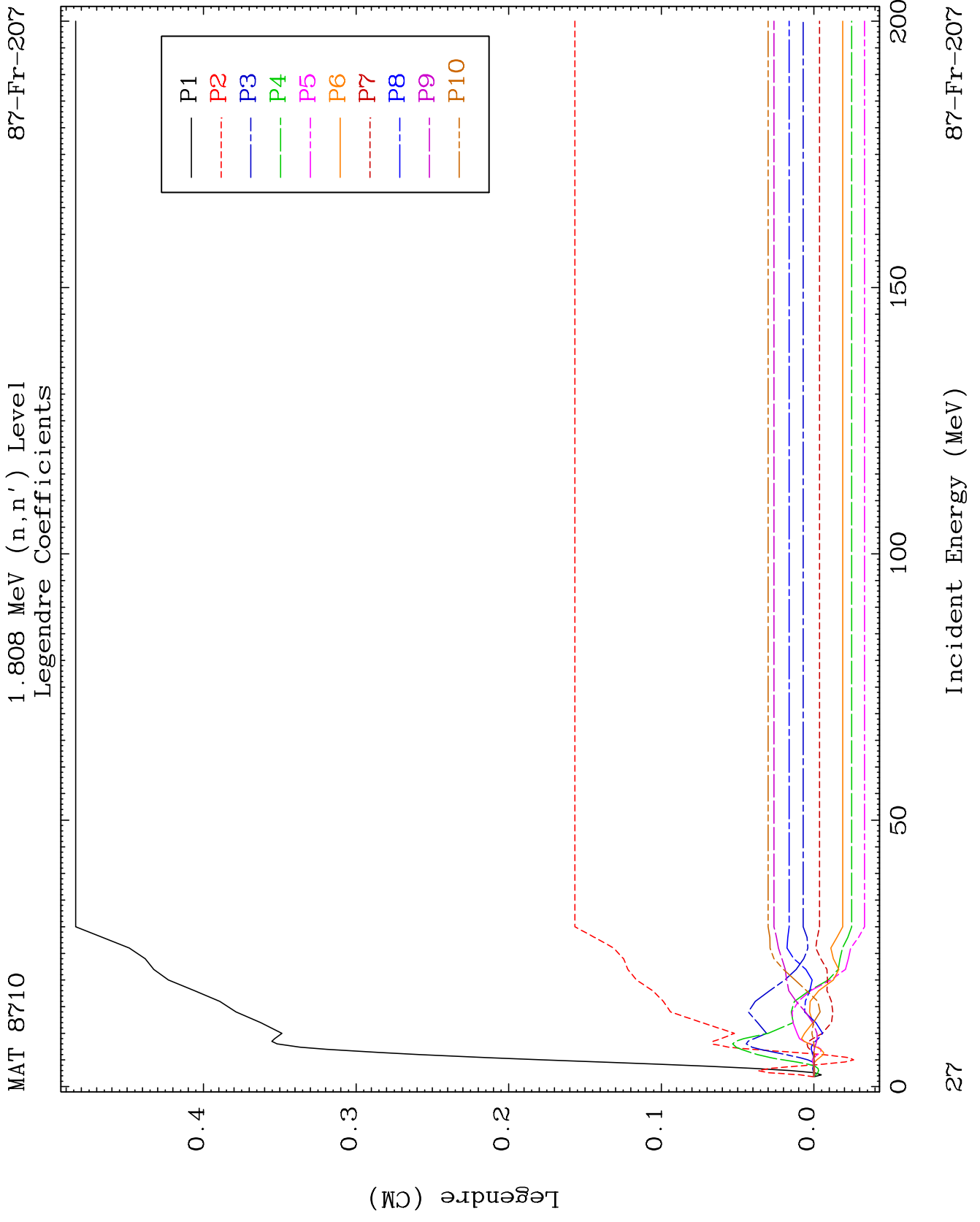


25

Incident Energy (MeV)

87-Fr-207

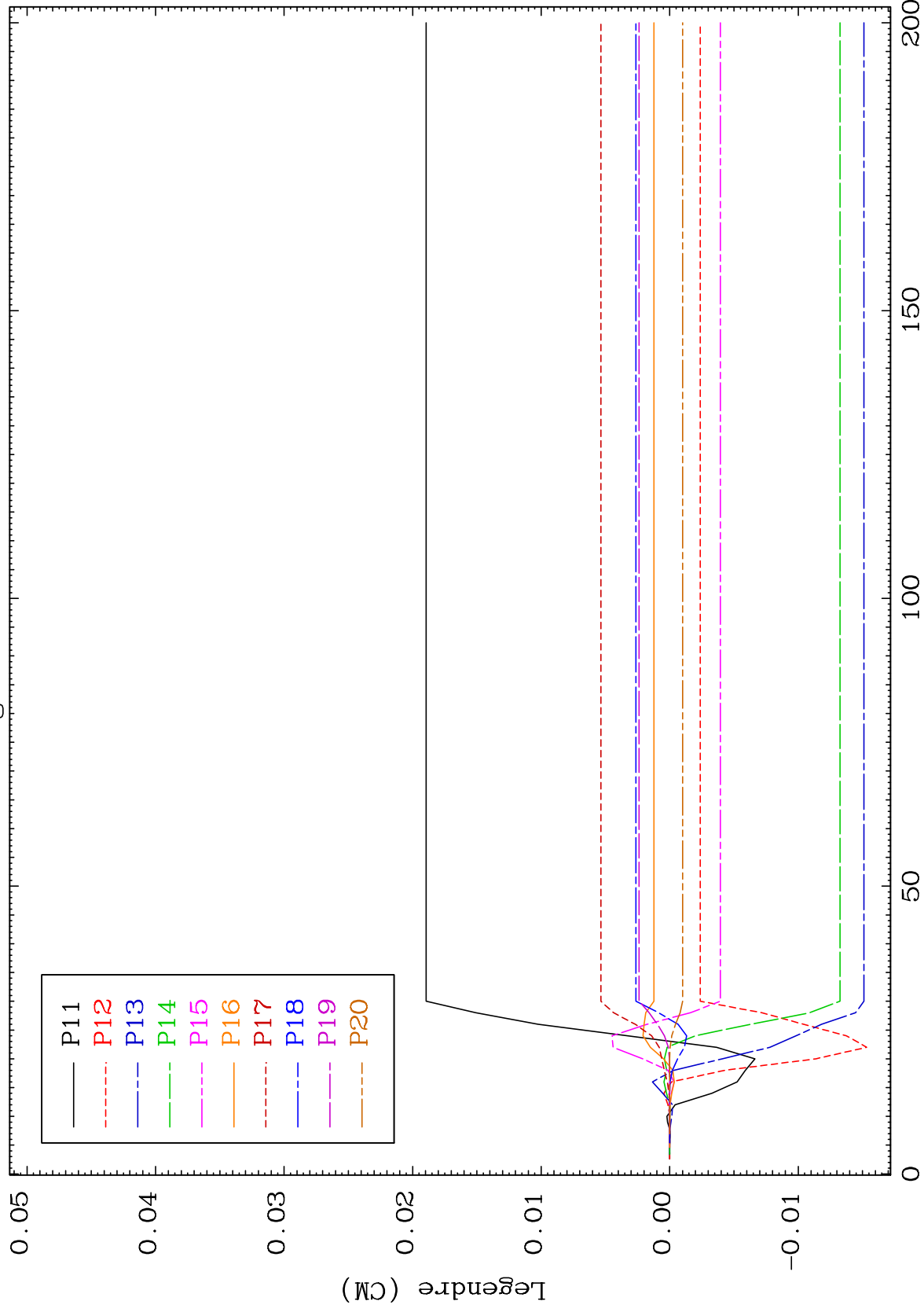




MAT 8710

1.808 MeV (n,n') Level  
Legendre Coefficients

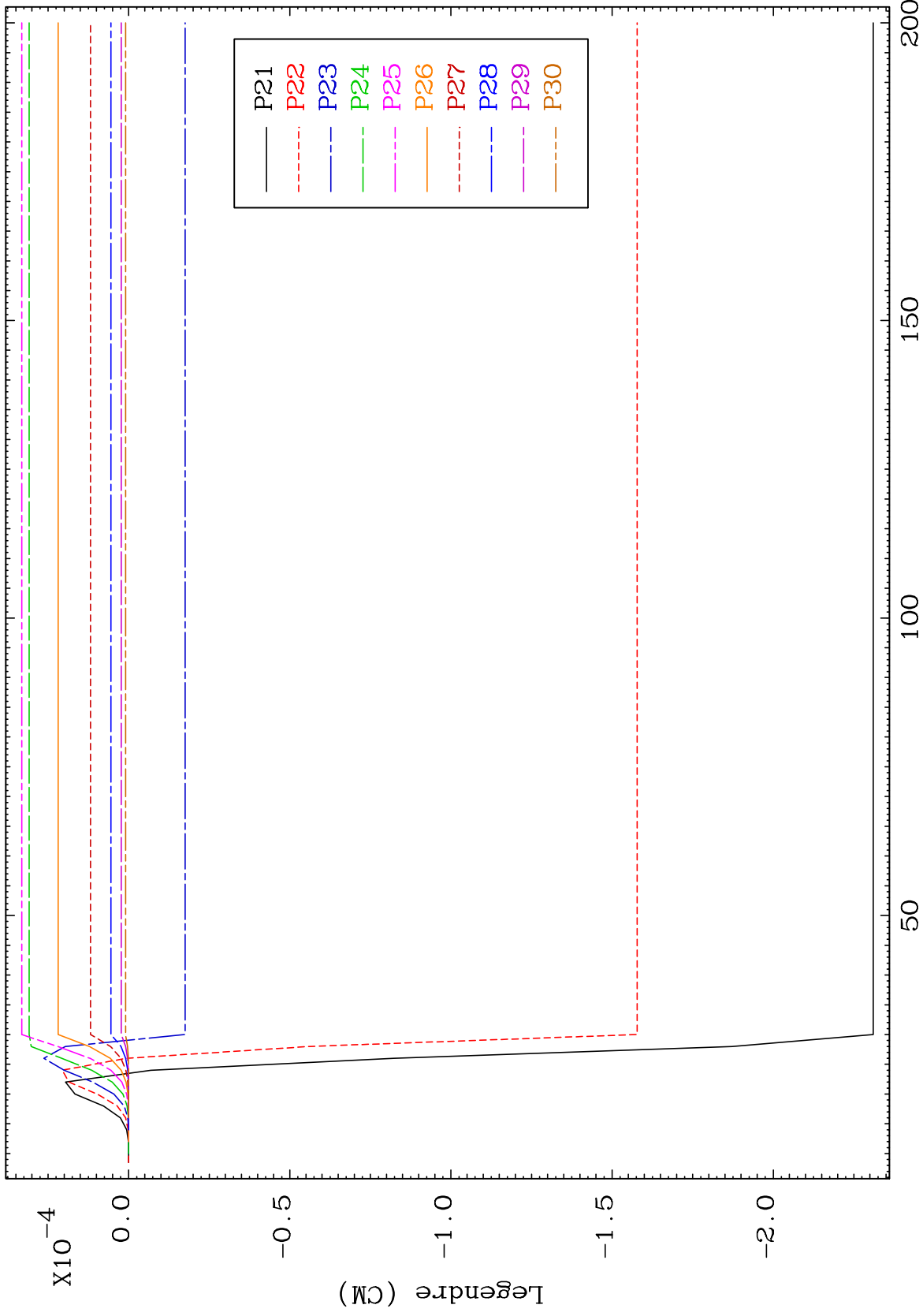
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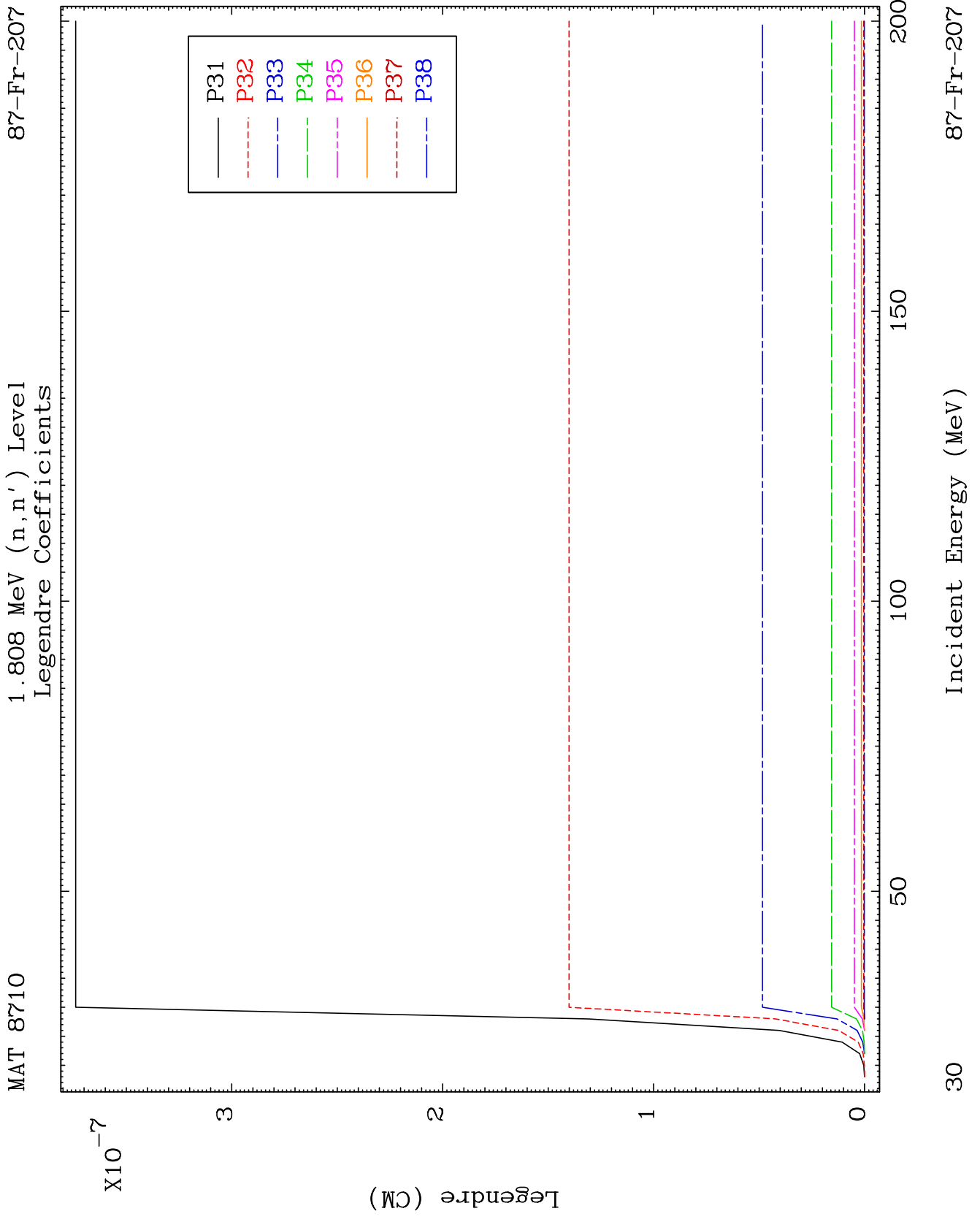


28

Incident Energy (MeV)

87-Fr-207

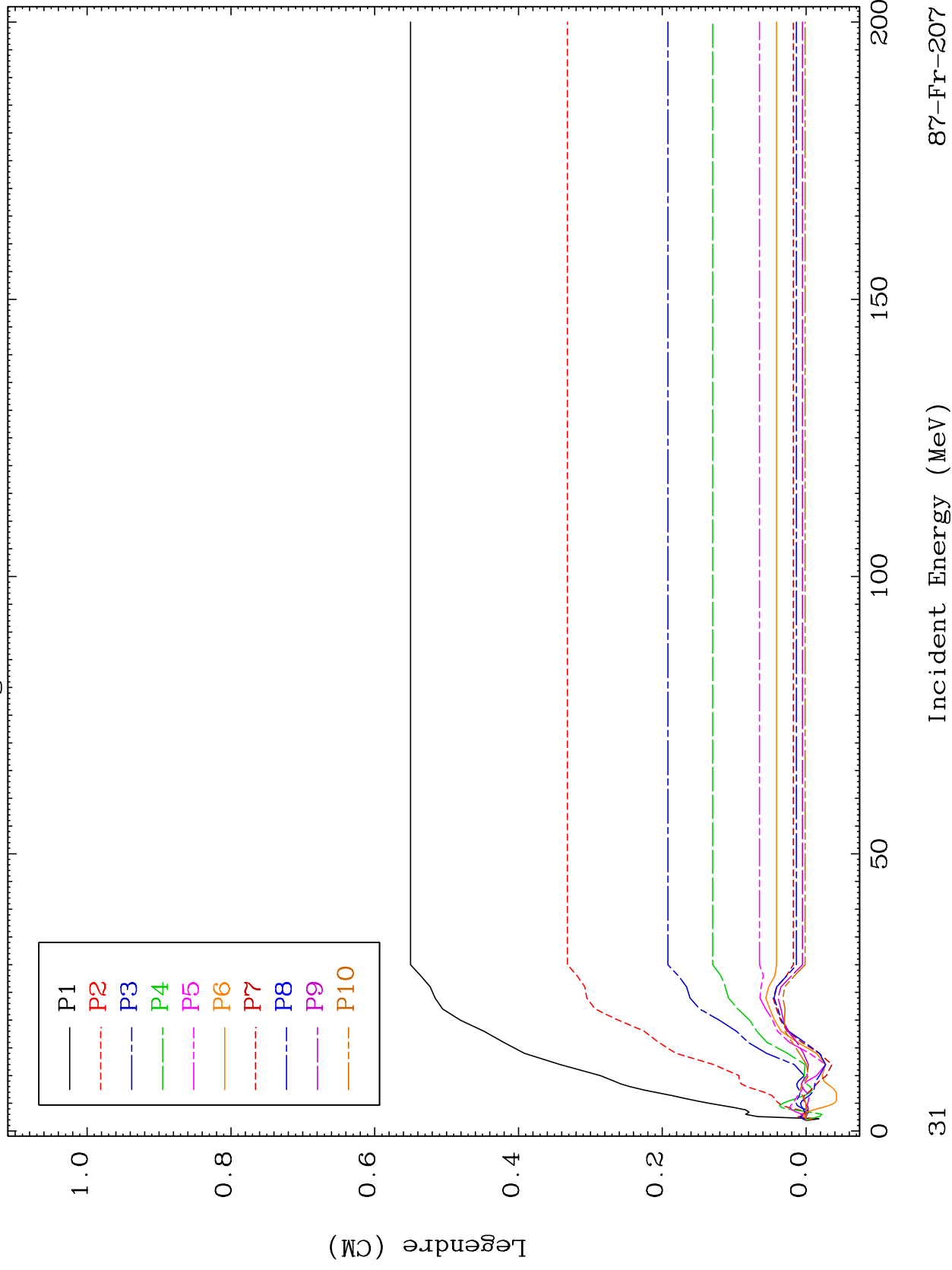




MAT 8710

1.885 MeV (n, n') Level  
Legendre Coefficients

87-Fr-207



31

Incident Energy (MeV)

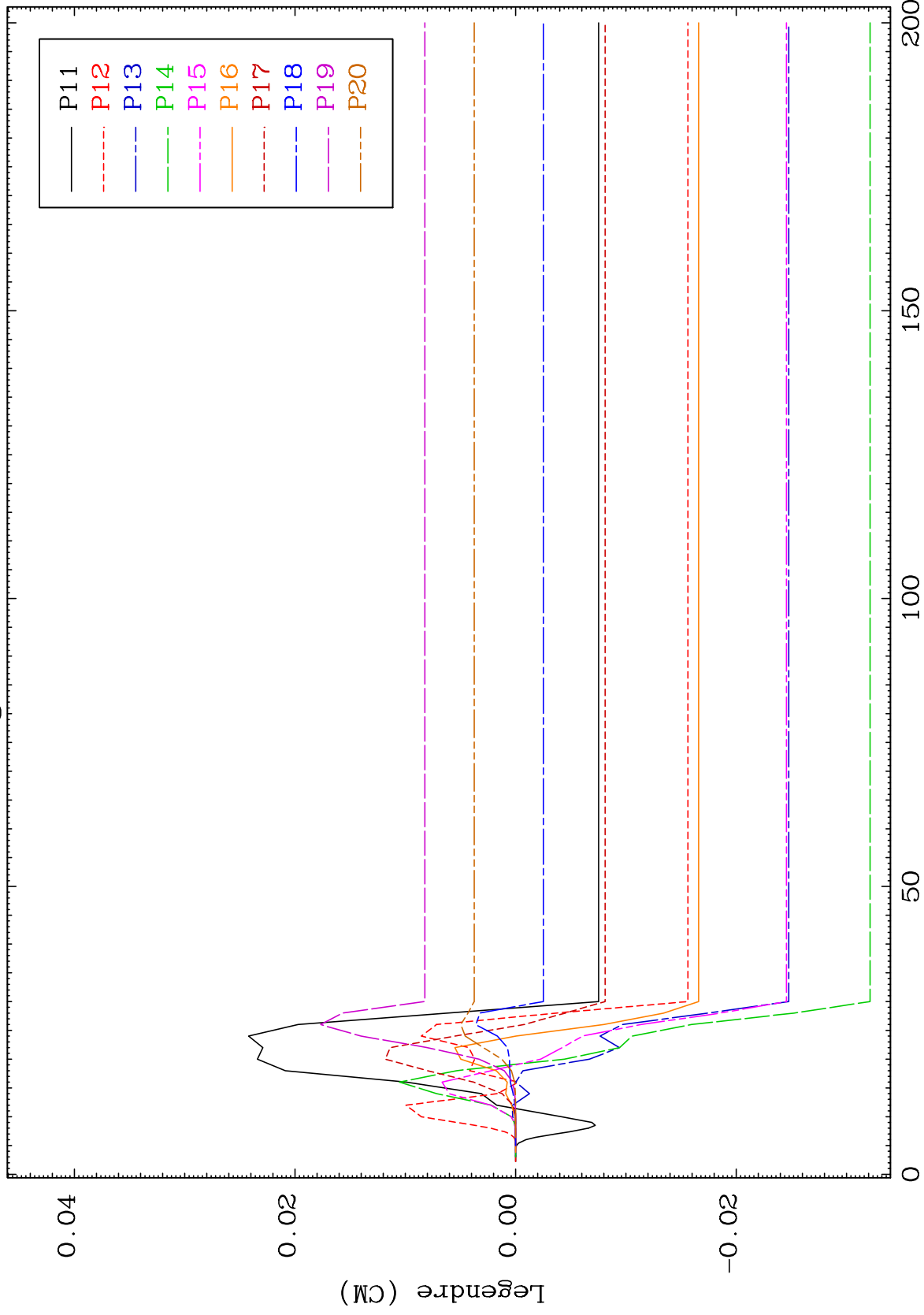
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MAT 8710

1.885 MeV (n,n') Level Legendre Coefficients

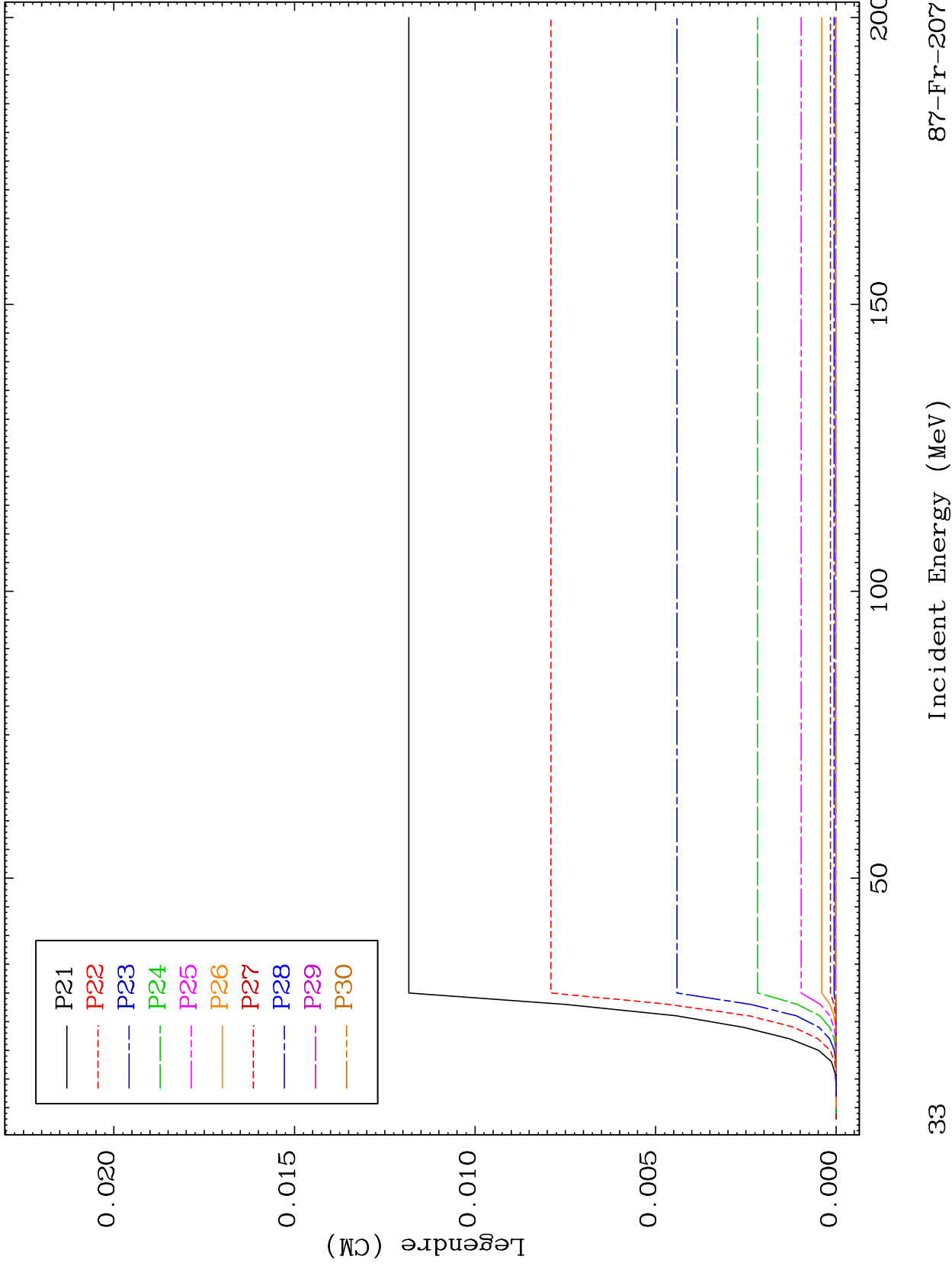
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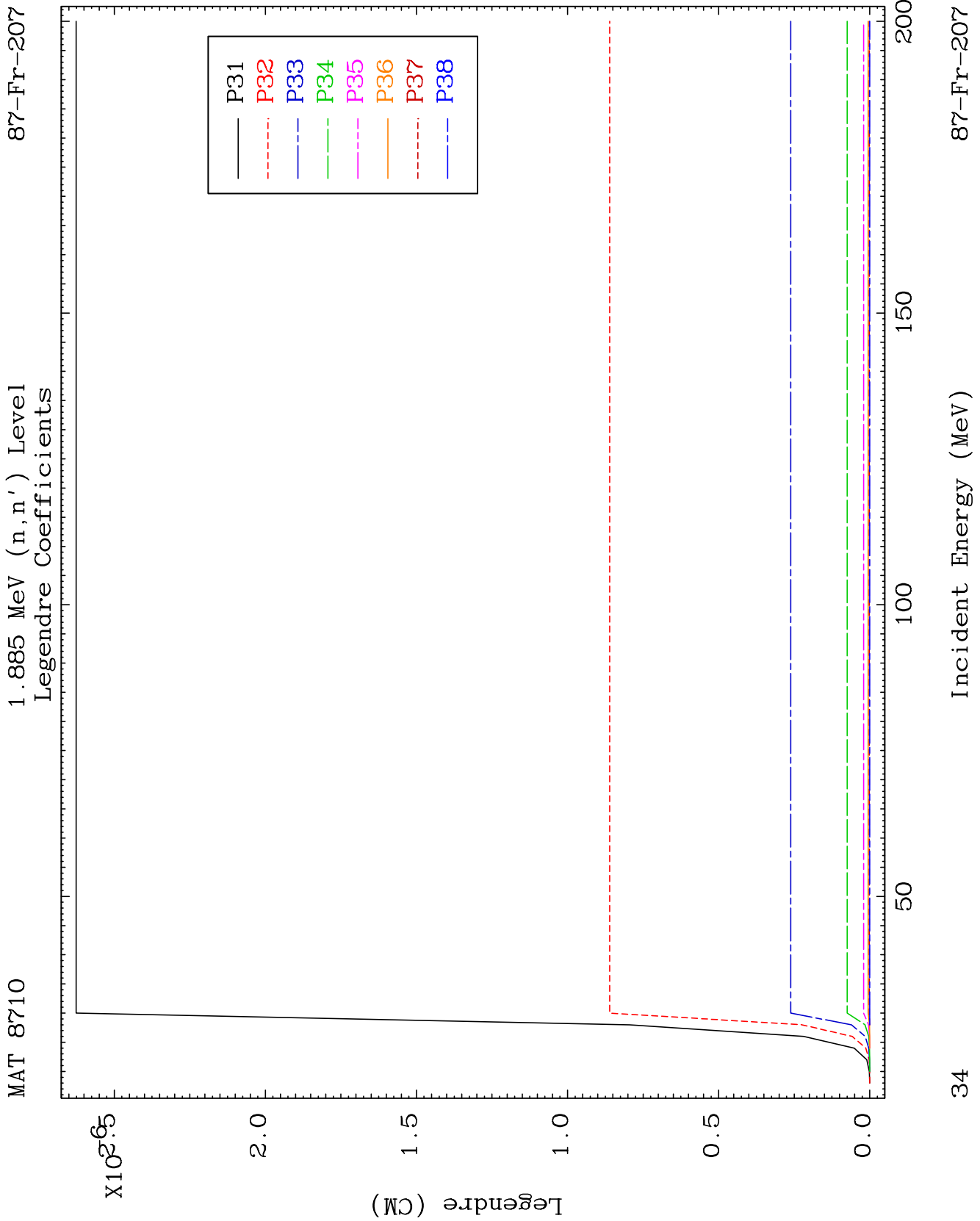


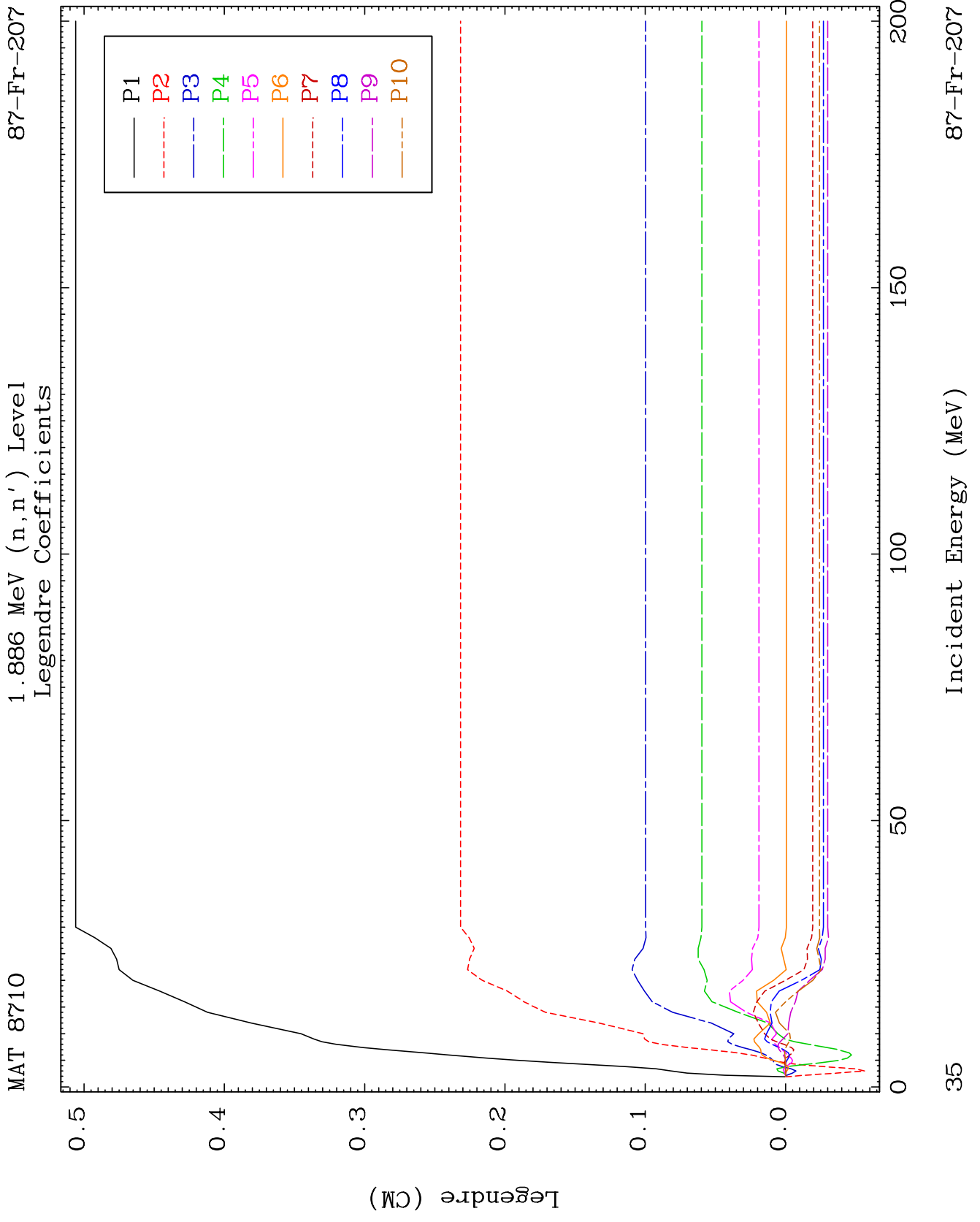
32

Incident Energy (MeV)

87-Fr-207



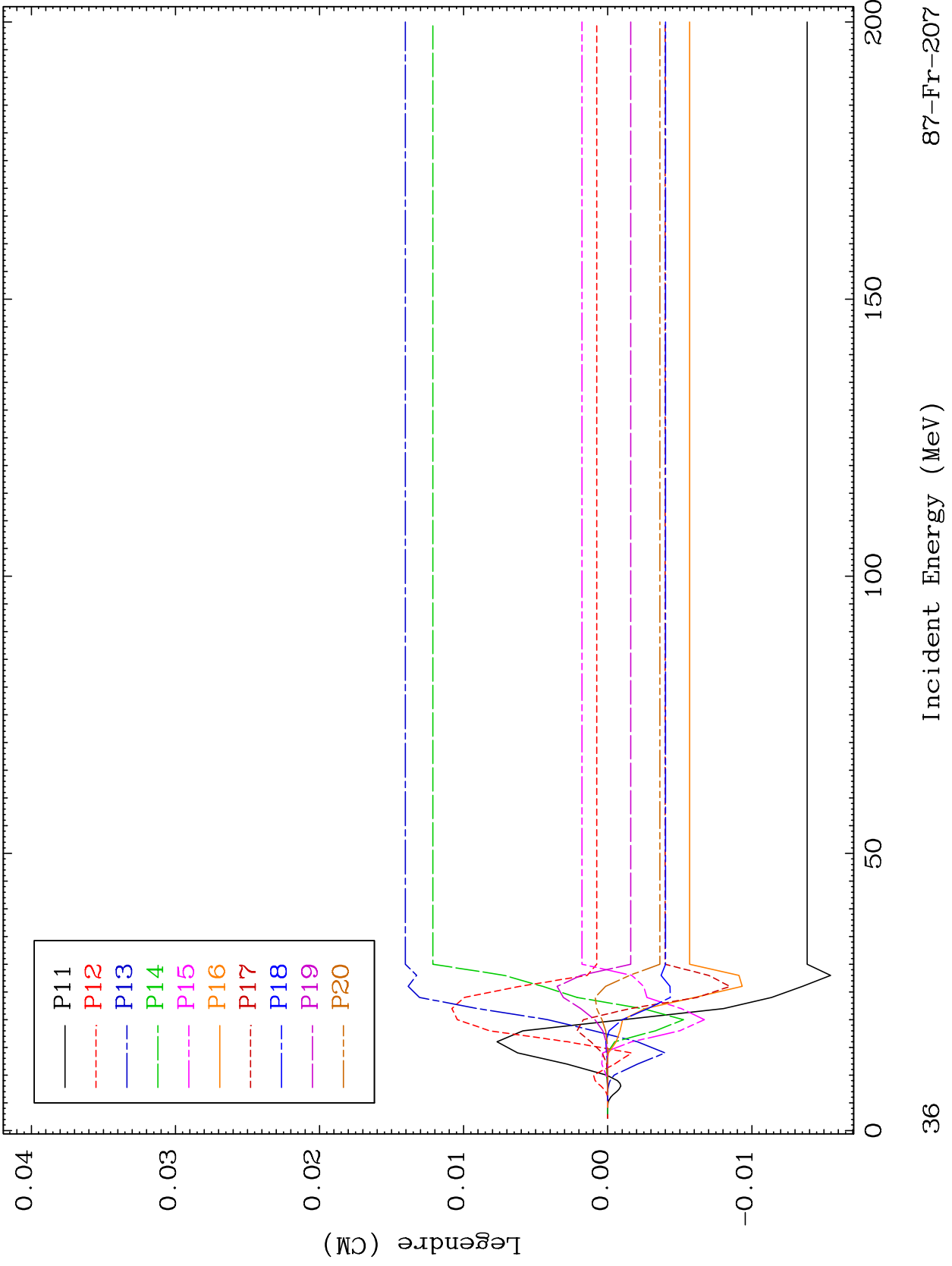




MAT 8710

1.886 MeV (n, n') Level  
Legendre Coefficients

87-Fr-207



36

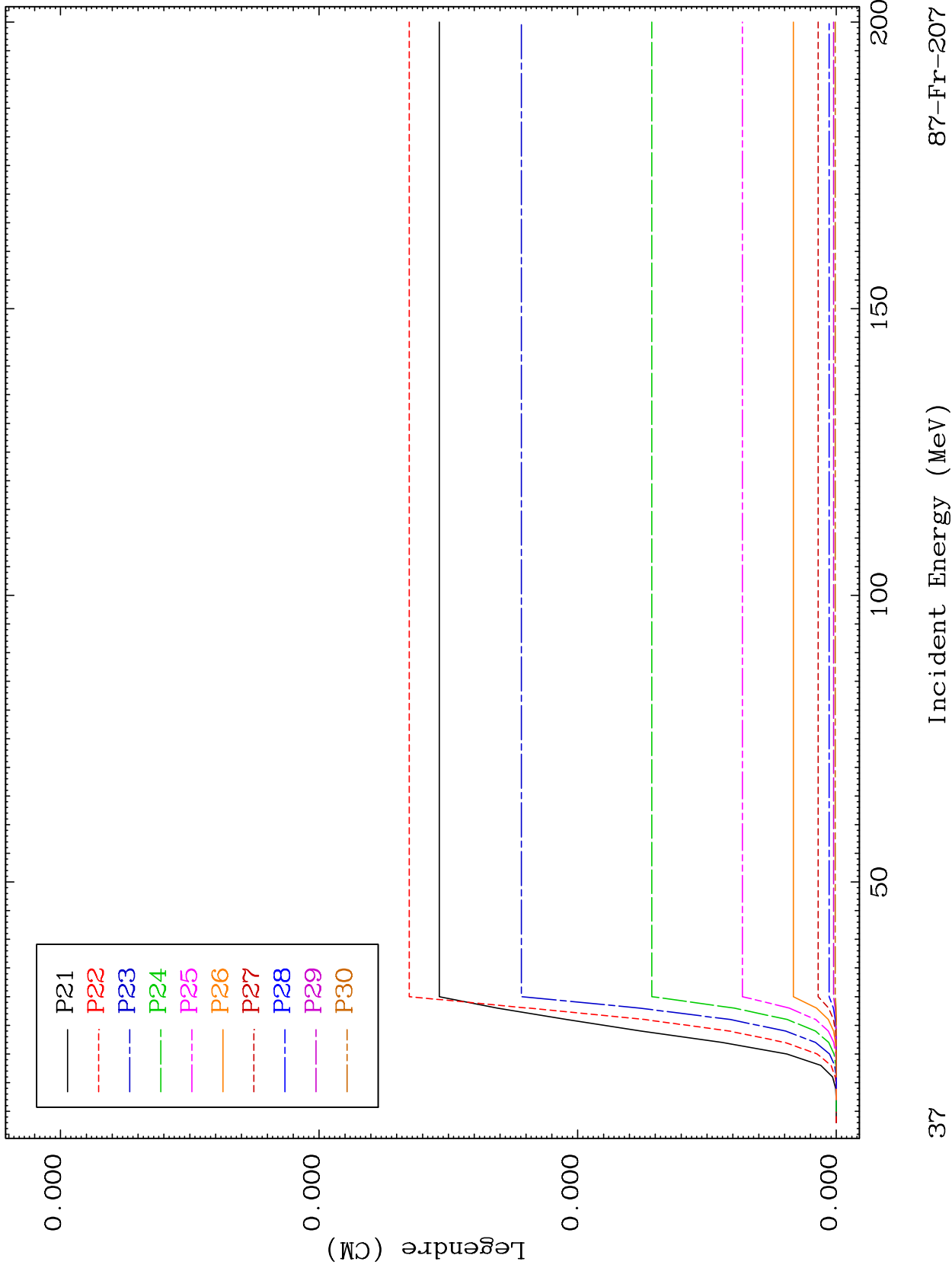
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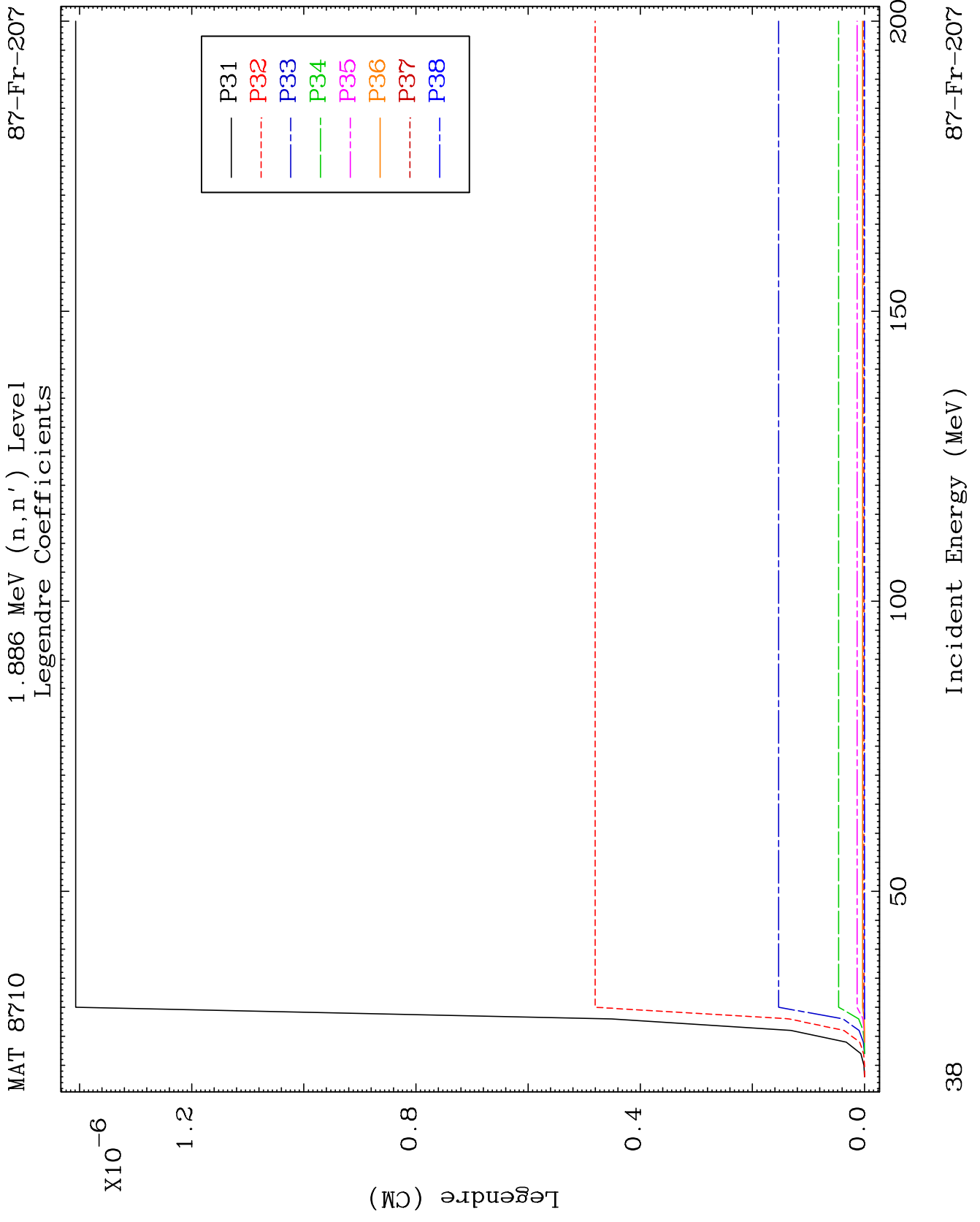
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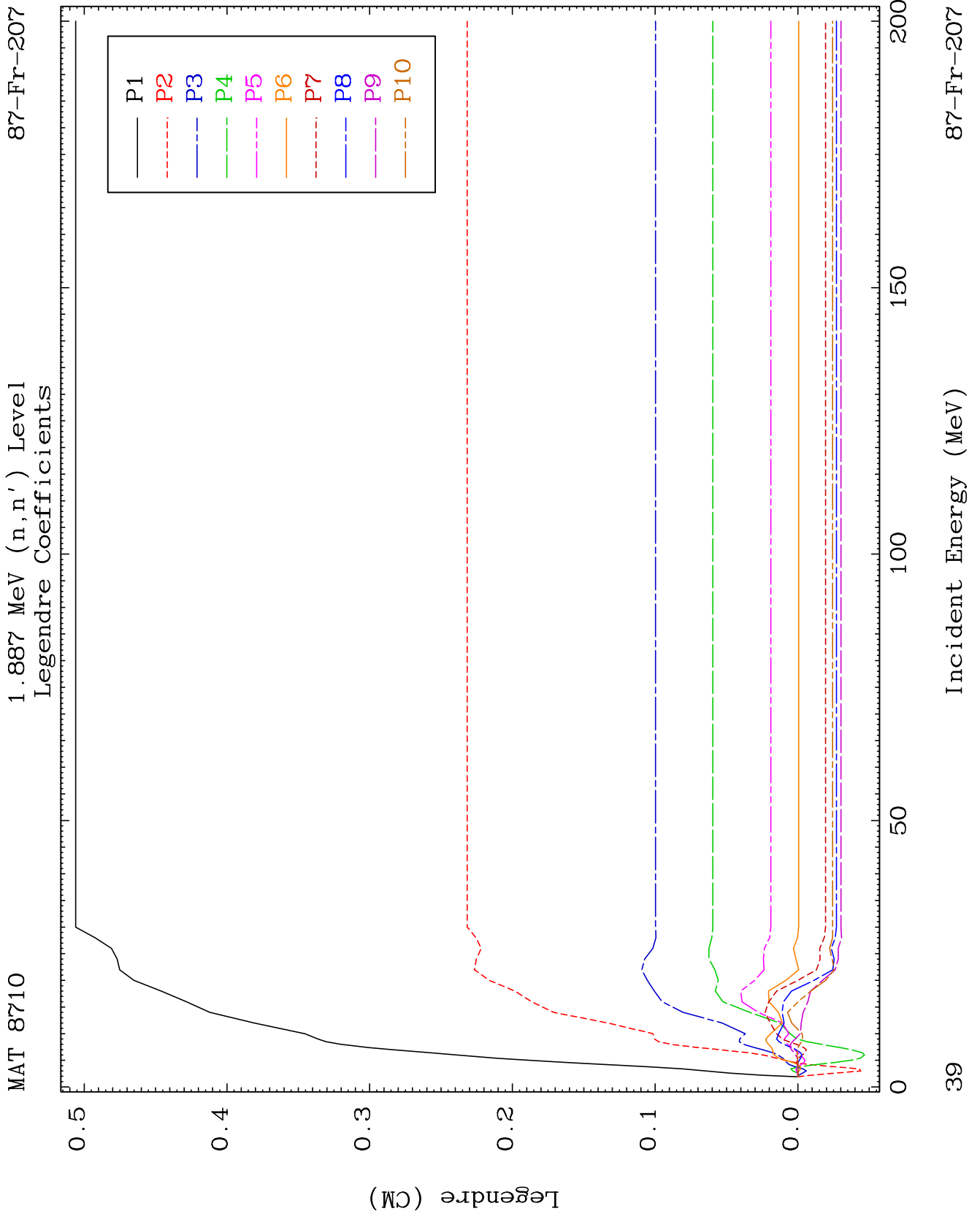
MAT 8710

1.886 MeV (n,n') Level  
Legendre Coefficients

87-Fr-207





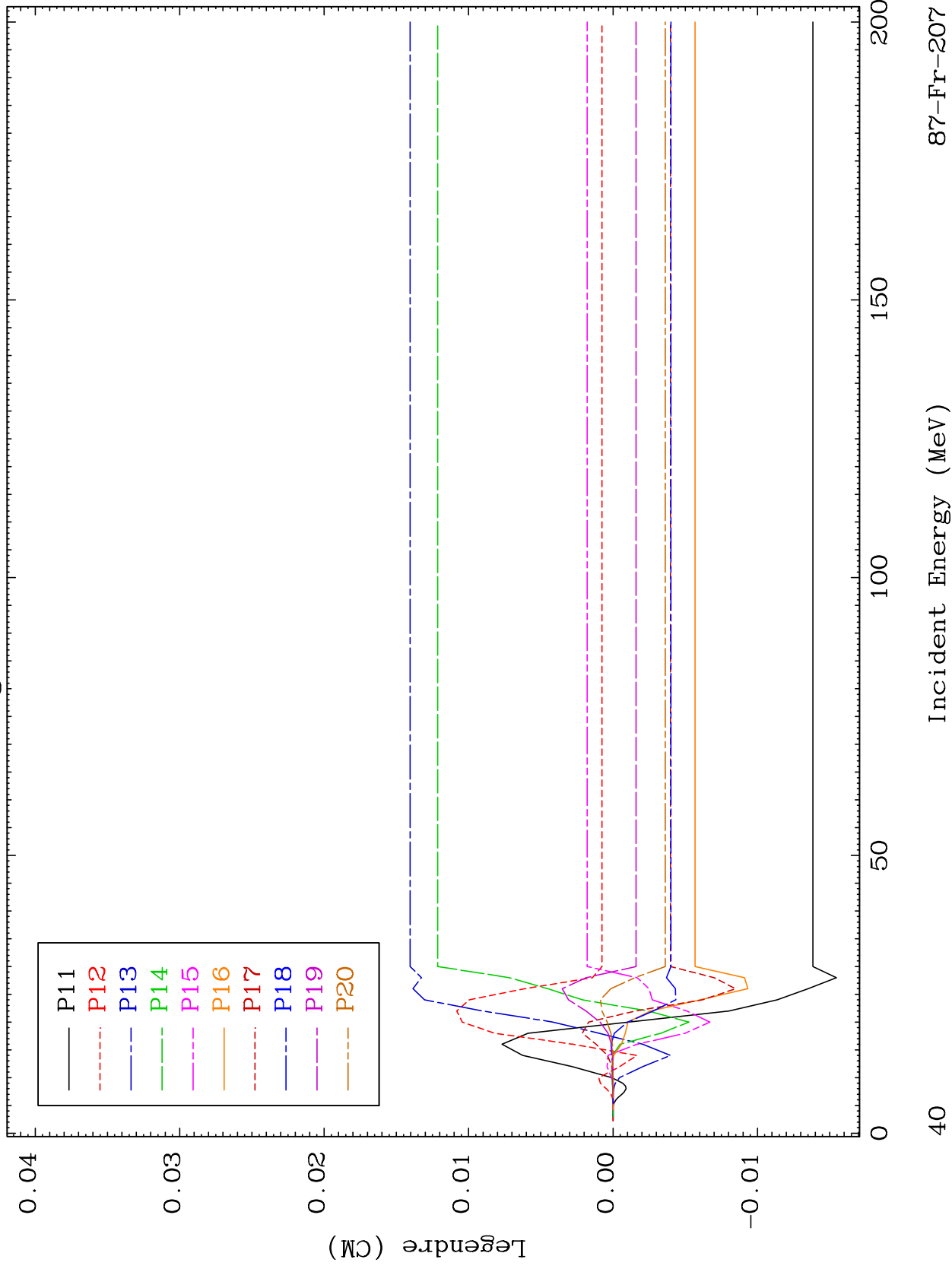




MAT 8710

1.887 MeV (n, n') Level  
Legendre Coefficients

87-Fr-207



87-Fr-207

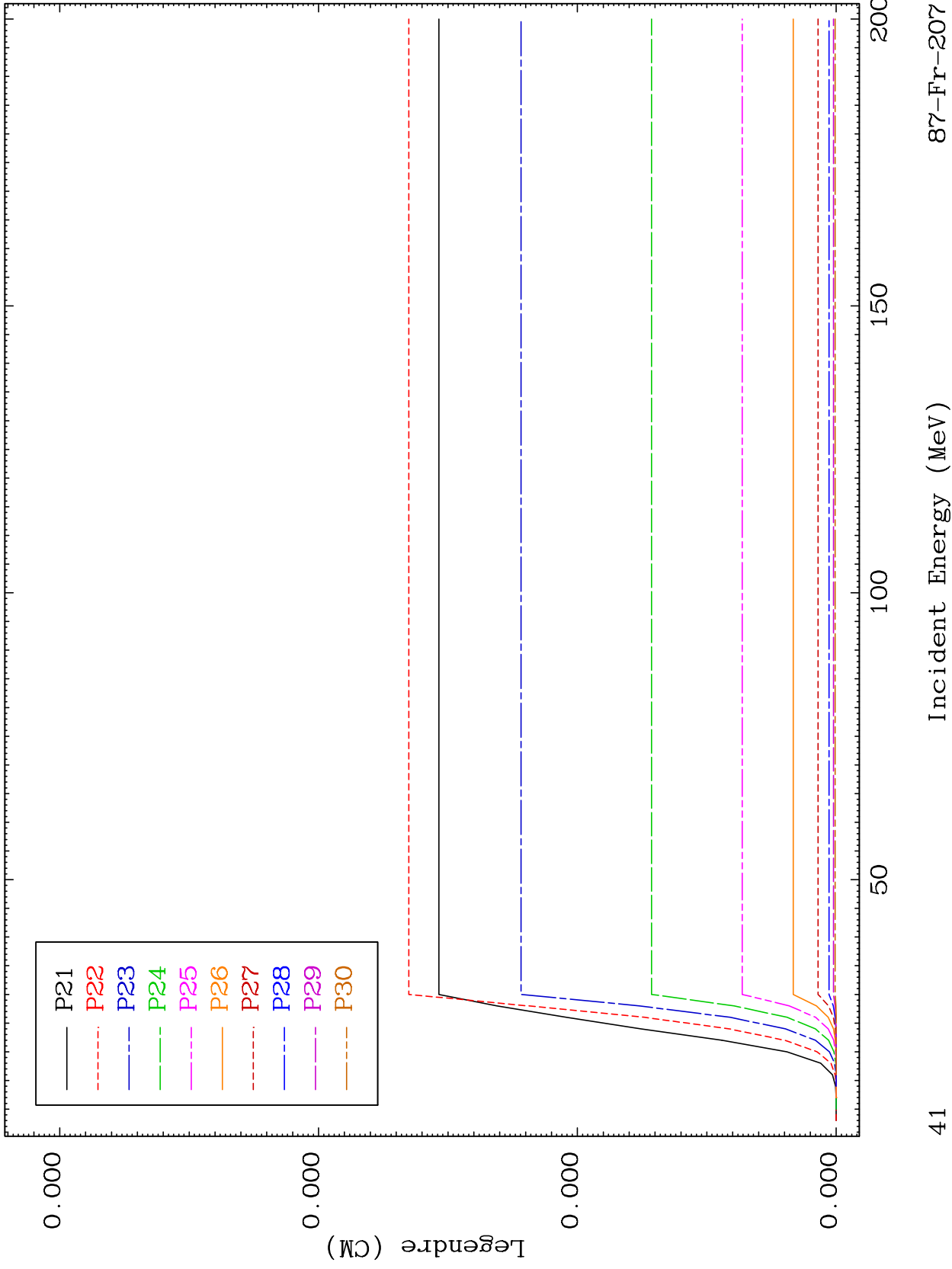
Incident Energy (MeV)

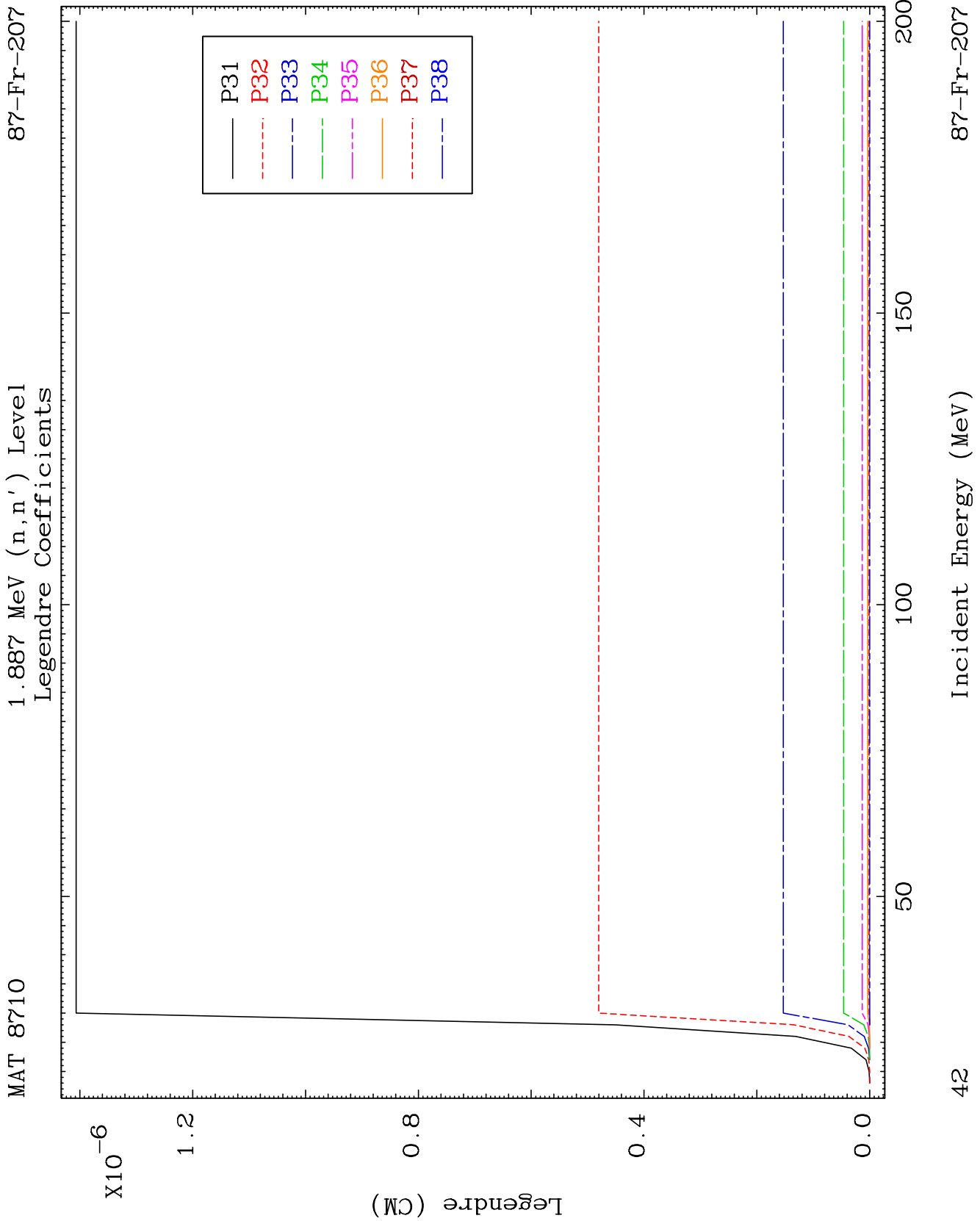
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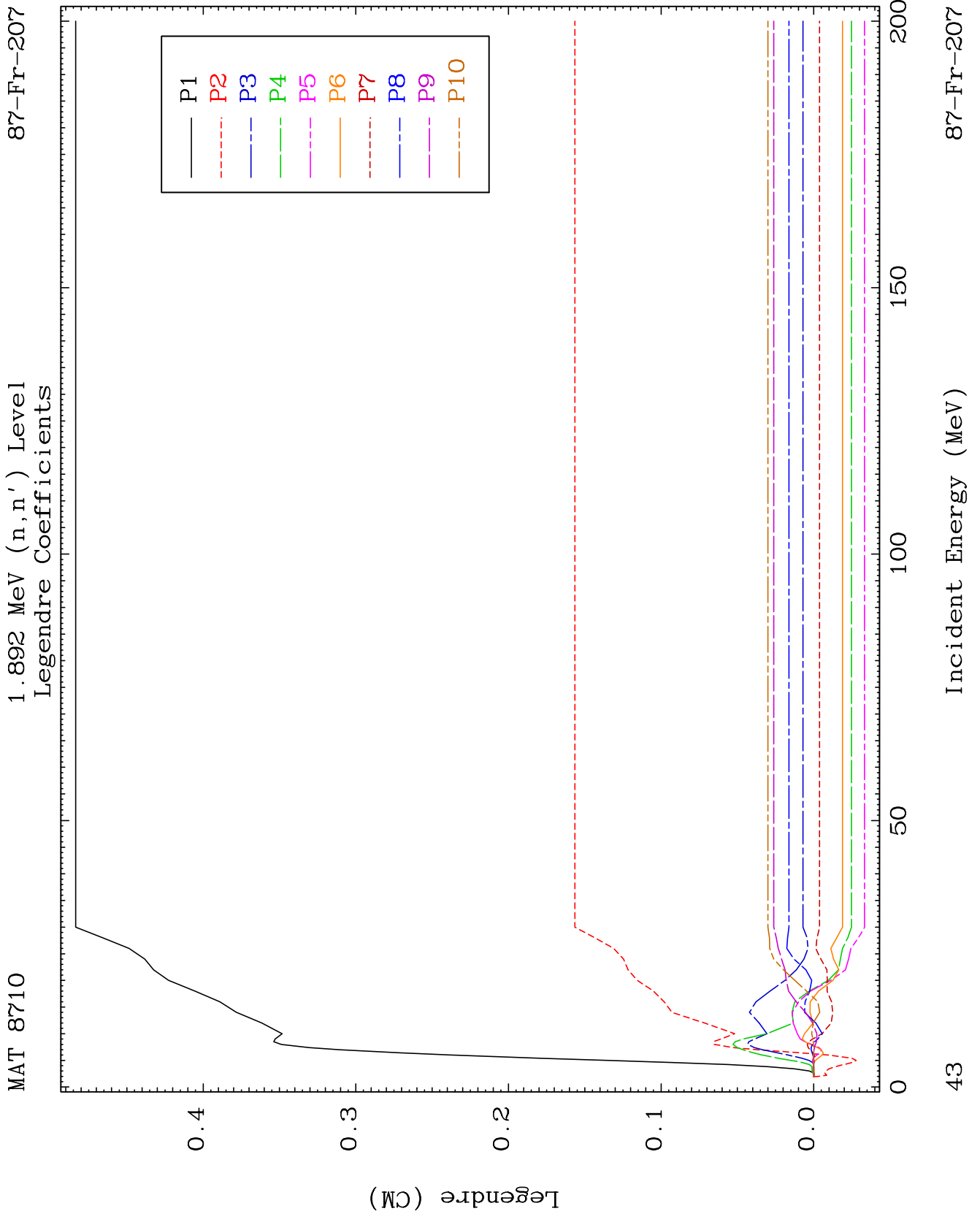
MAT 8710

1.887 MeV (n,n') Level  
Legendre Coefficients

87-Fr-207



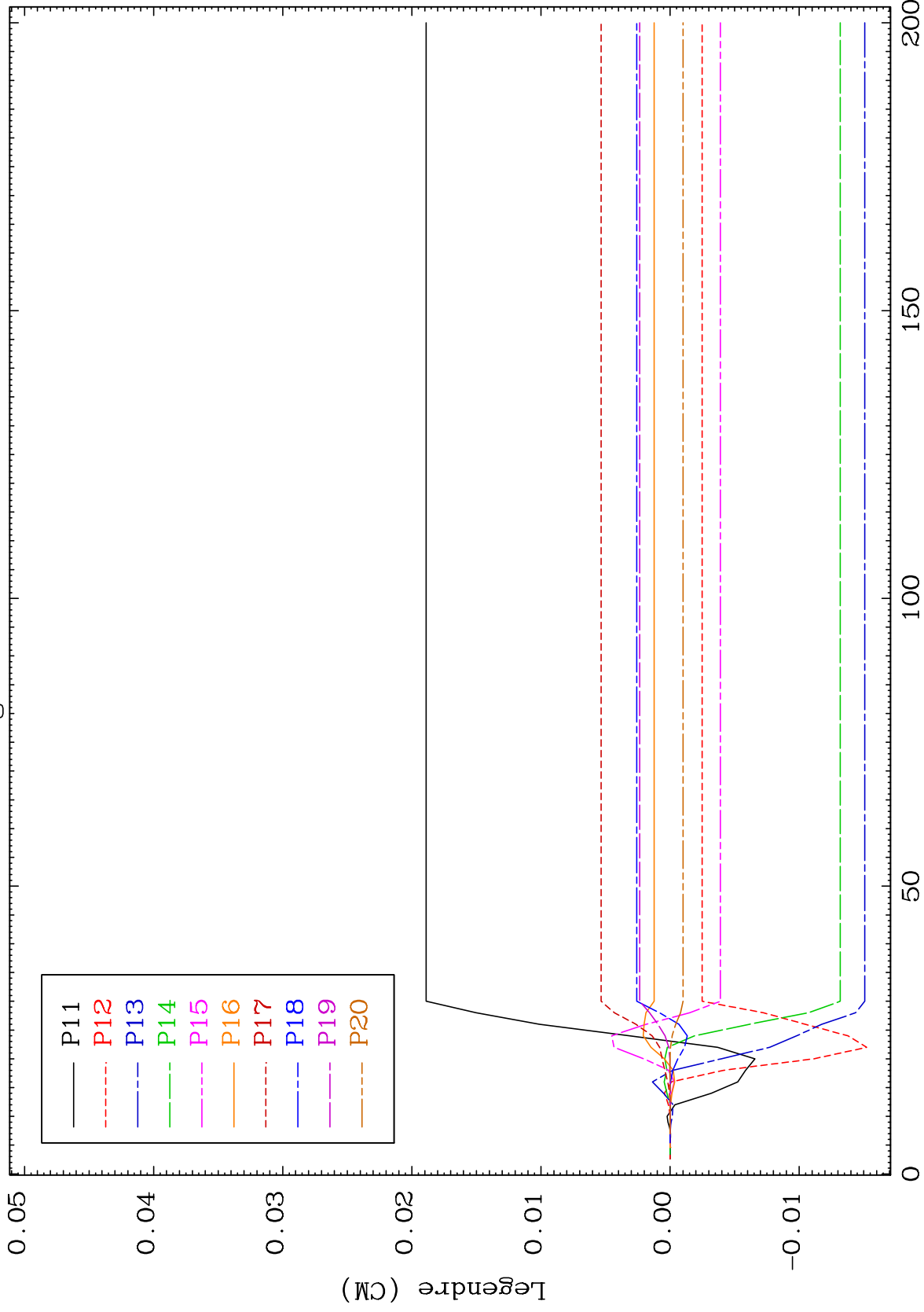


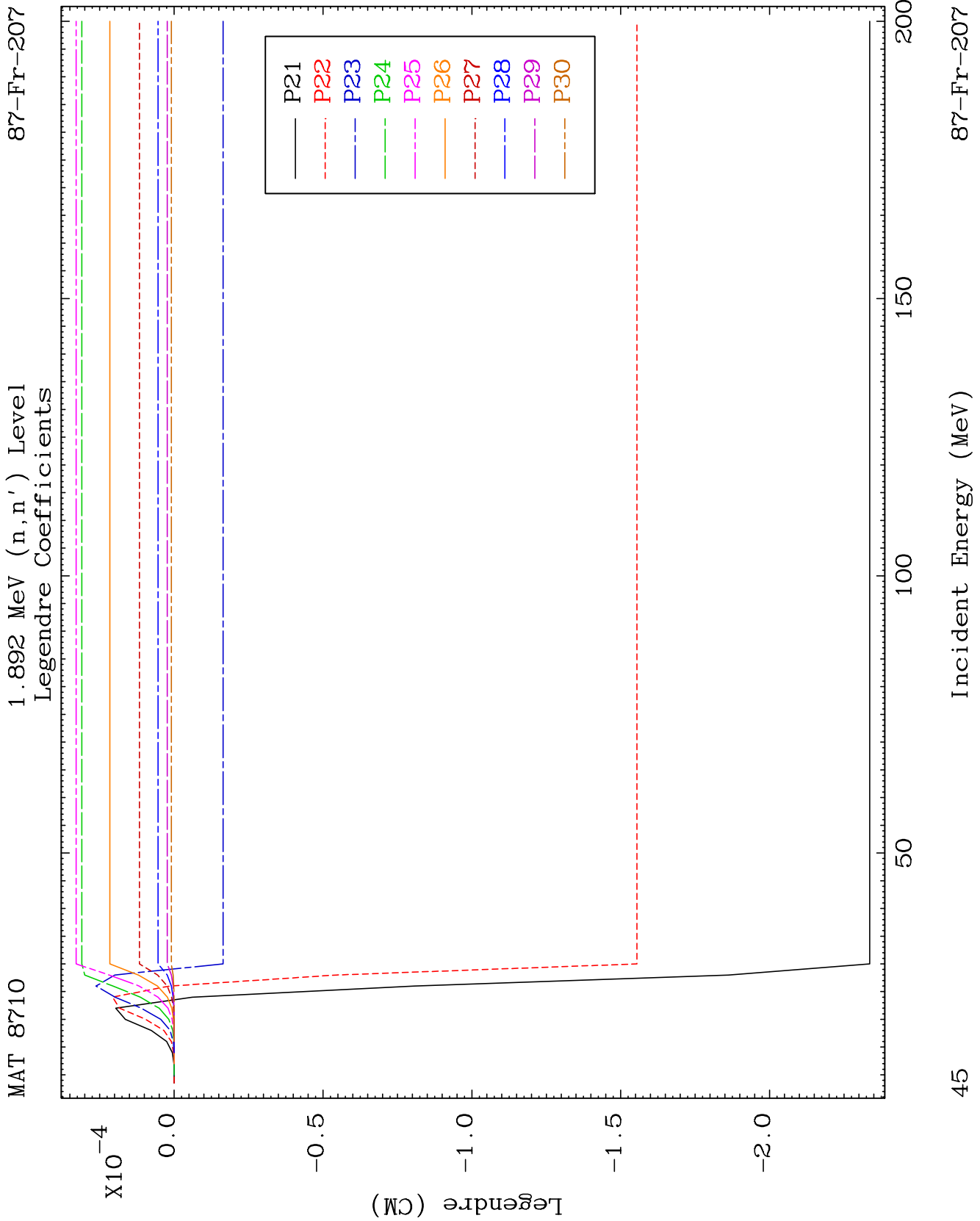


MAT 8710

1.892 MeV (n,n') Level  
Legendre Coefficients

87-Fr-207

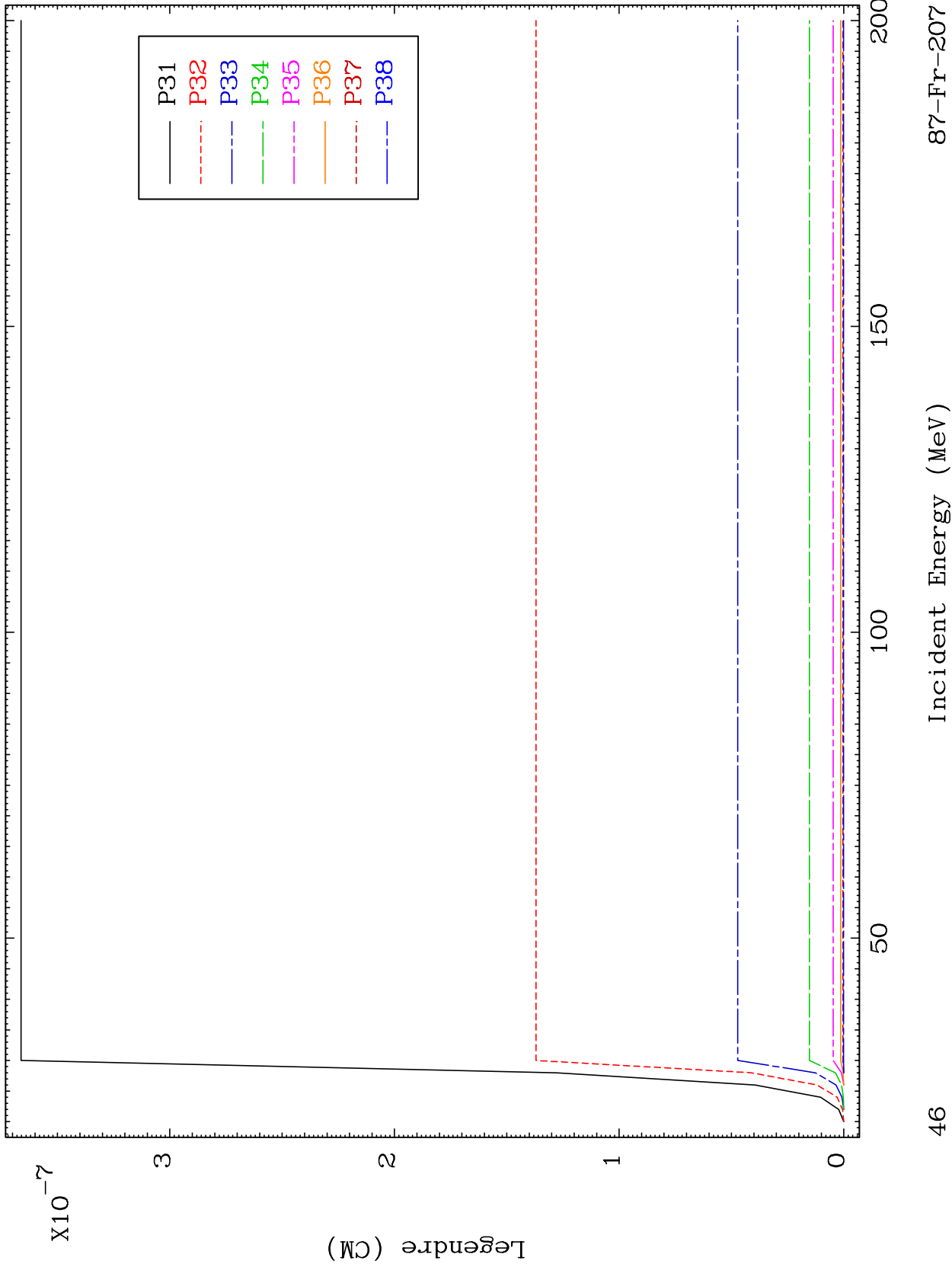




MAT 8710

1.892 MeV (n,n') Level  
Legendre Coefficients

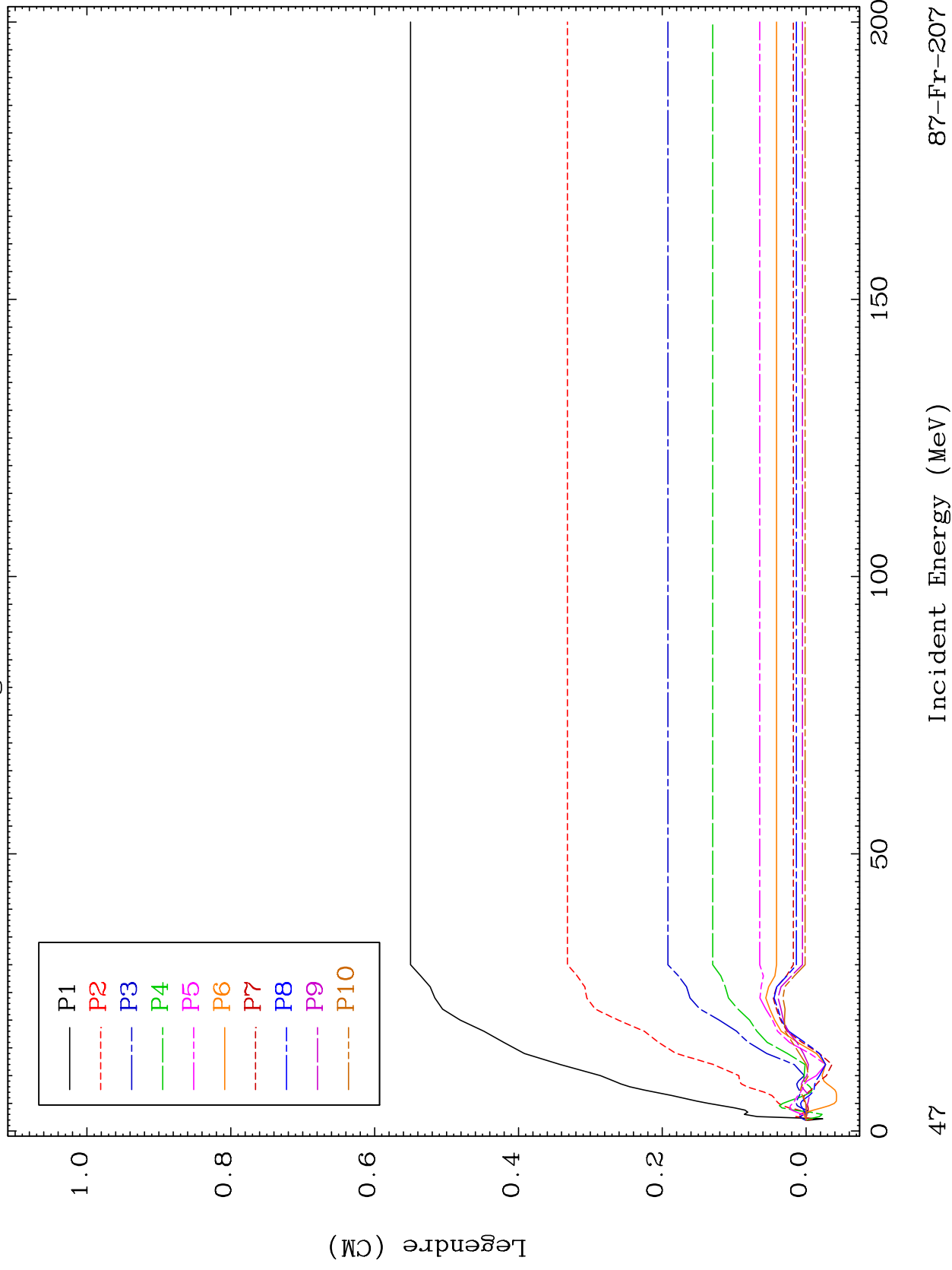
87-Fr-207



MAT 8710

1.897 MeV (n,n') Level  
Legendre Coefficients

87-Fr-207



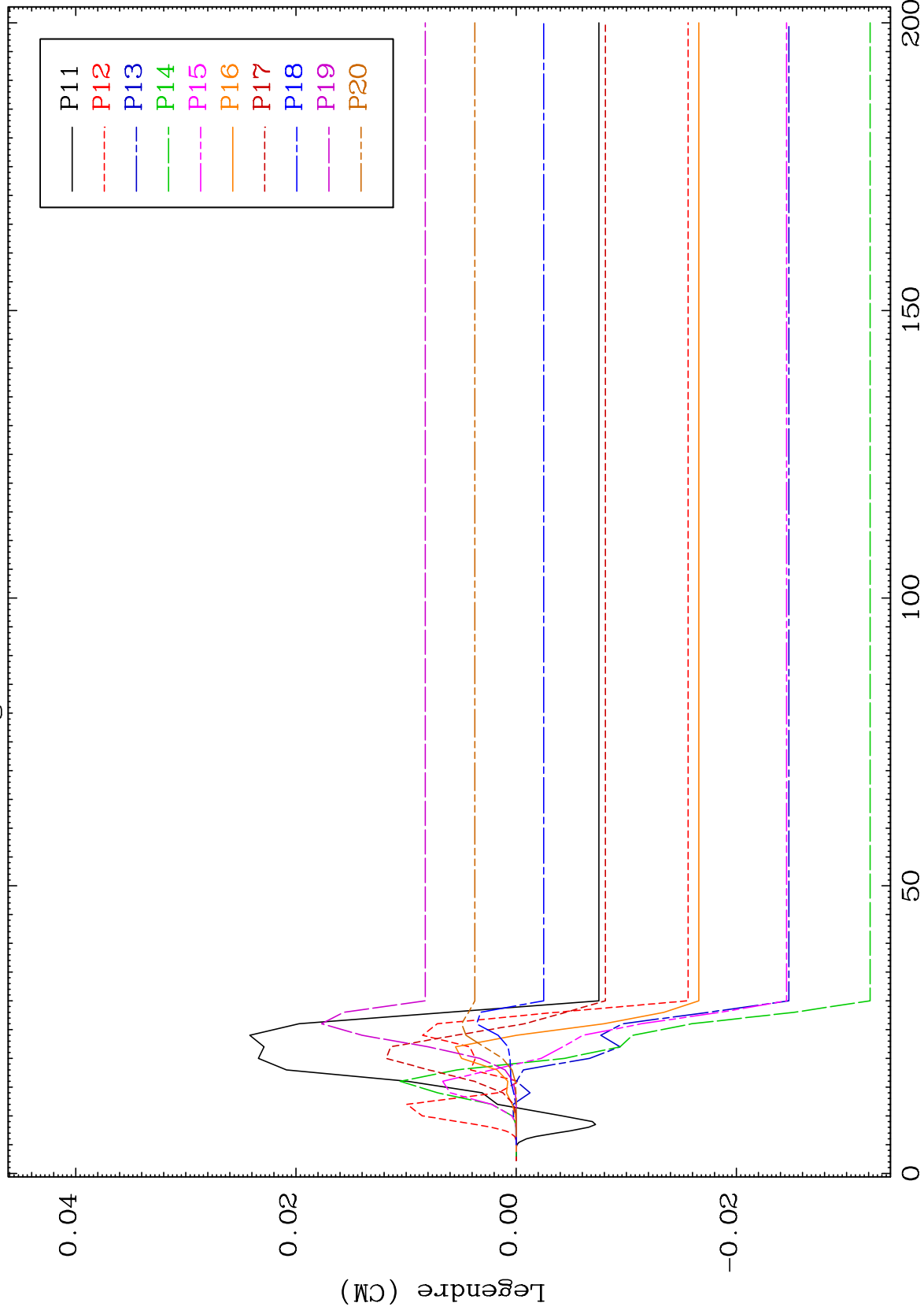
47



MAT 8710

1.897 MeV (n,n') Level  
Legendre Coefficients

87-Fr-207



48

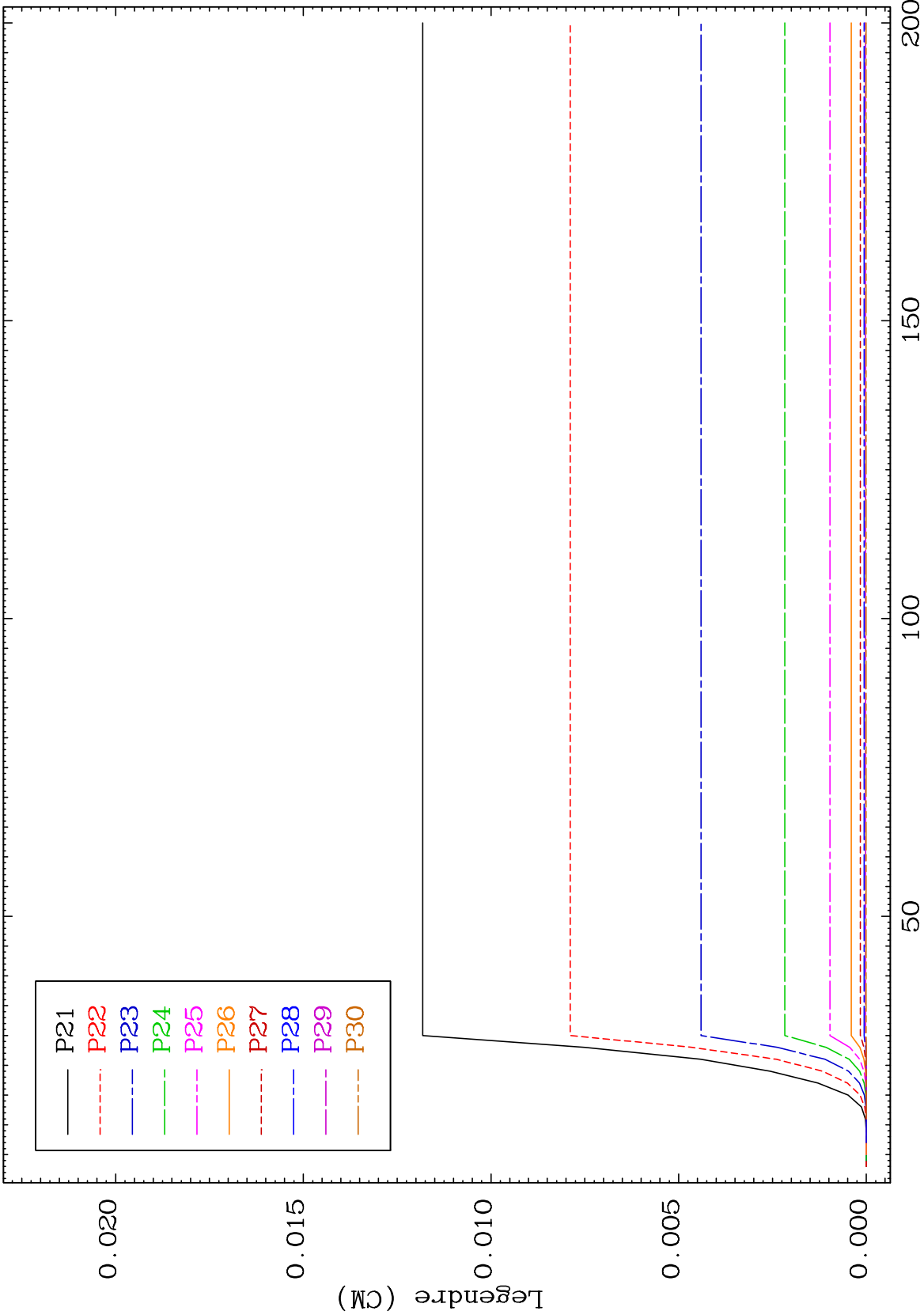
Incident Energy (MeV)

87-Fr-207

MAT 8710

1.897 MeV (n,n') Level Legendre Coefficients

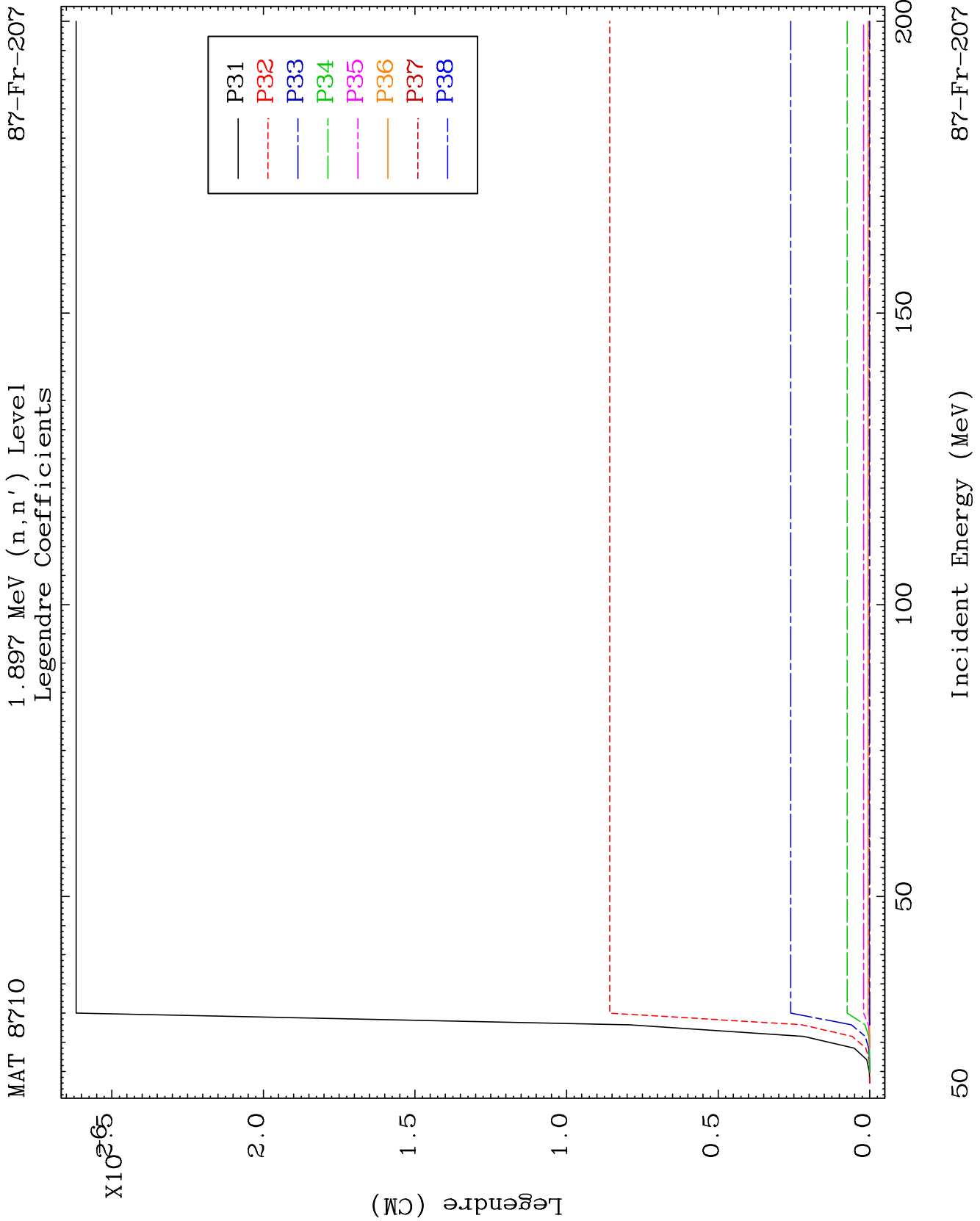
87-Fr-207



49

Incident Energy (MeV)

87-Fr-207

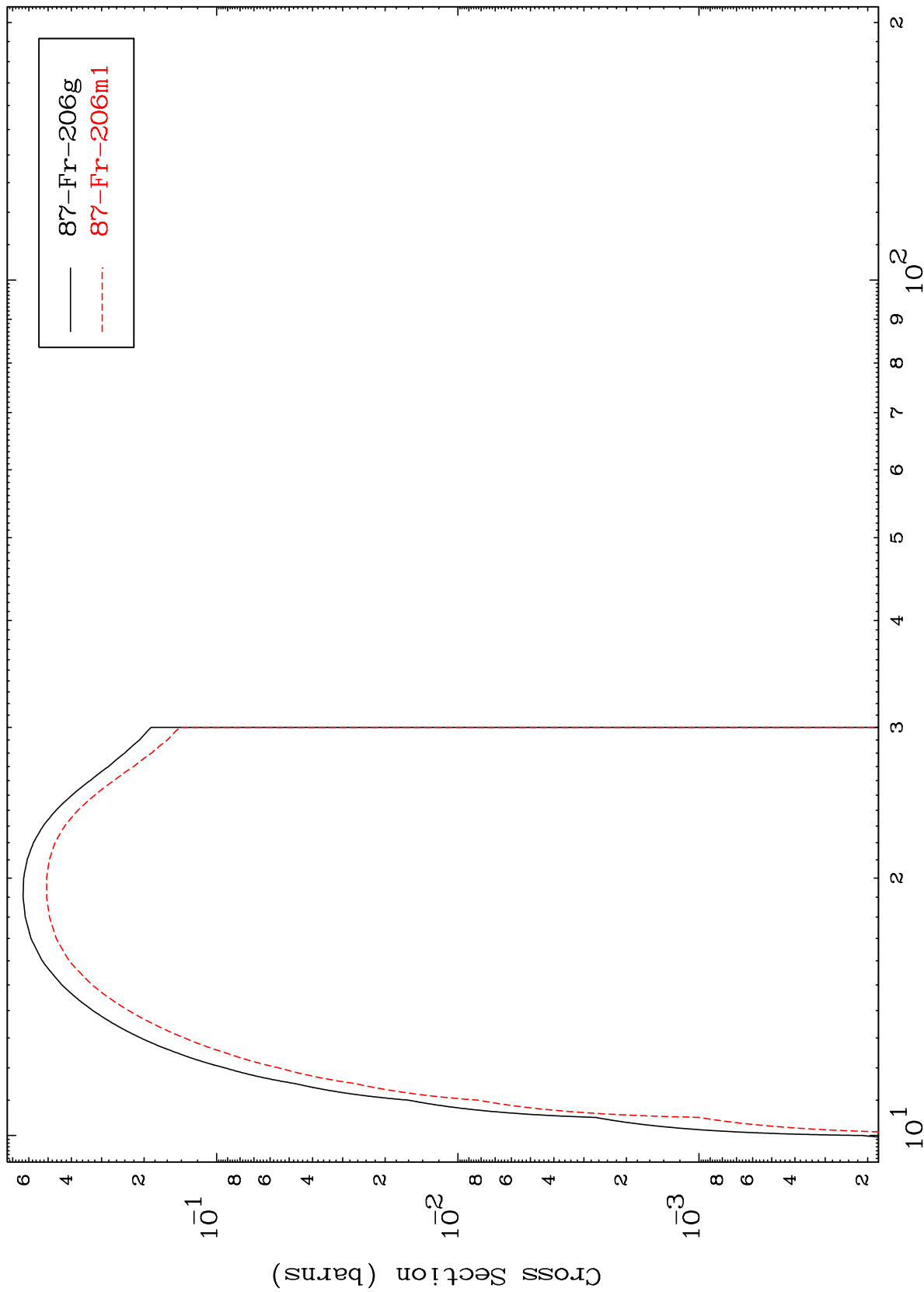


MAT 8710

87-Fr-207

(n,2n)

Radionuclide Production Cross Section



Incident Energy (MeV)

87-Fr-207

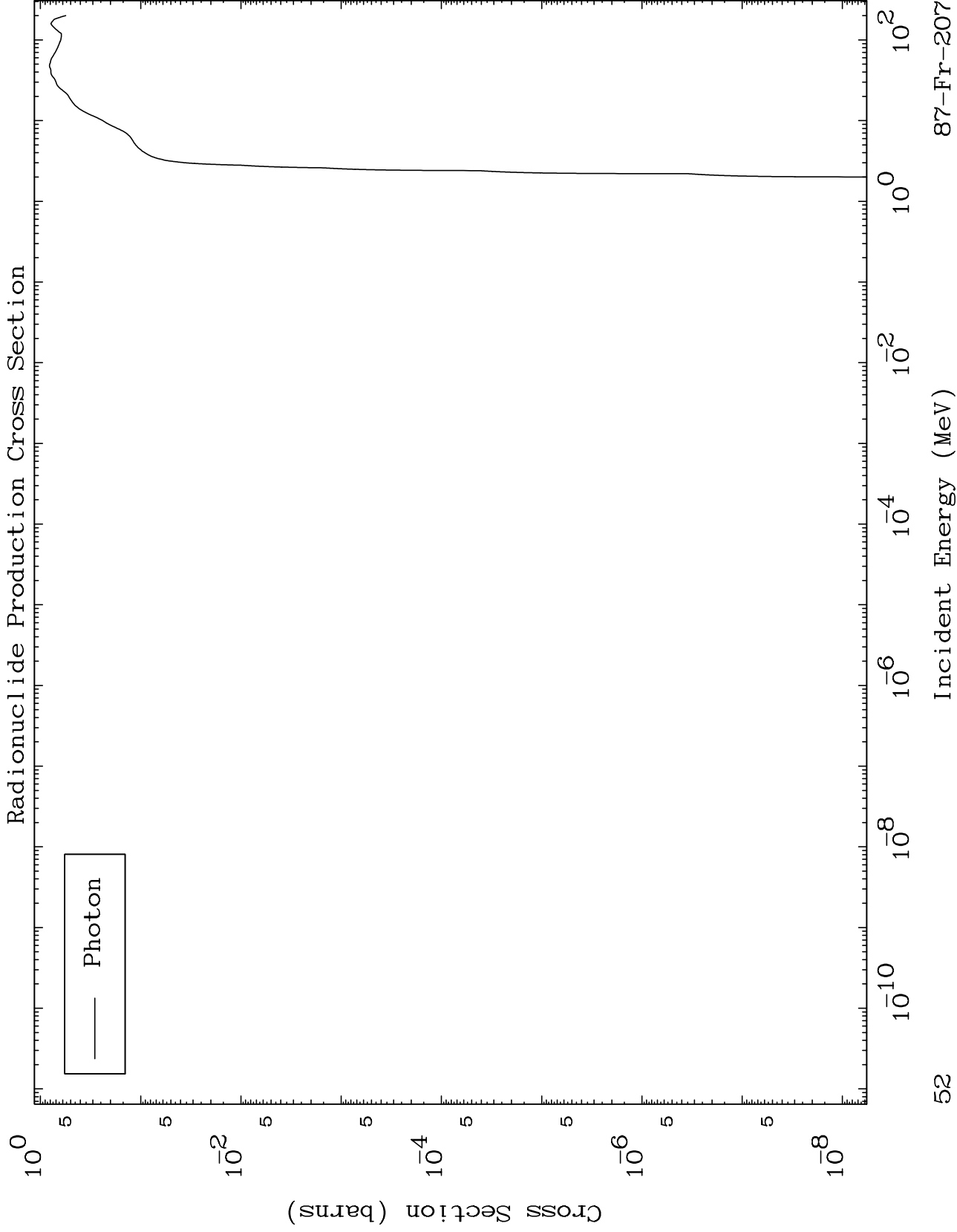
51

MAT 8710

Fission

87-Fr-207

Radionuclide Production Cross Section



52

Incident Energy (MeV)

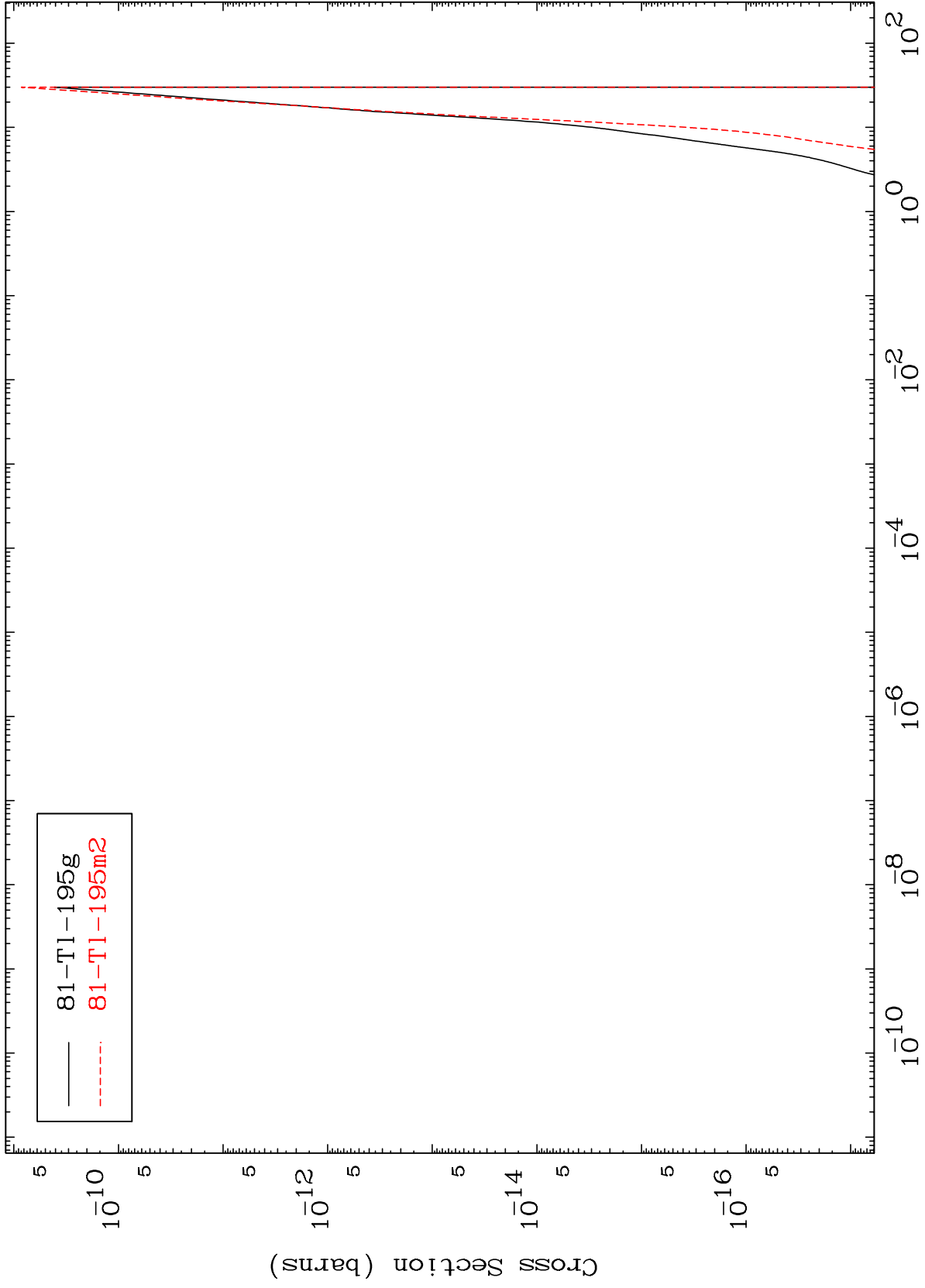
87-Fr-207

MAT 8710

(n,n') 3α

87-Fr-207

Radionuclide Production Cross Section

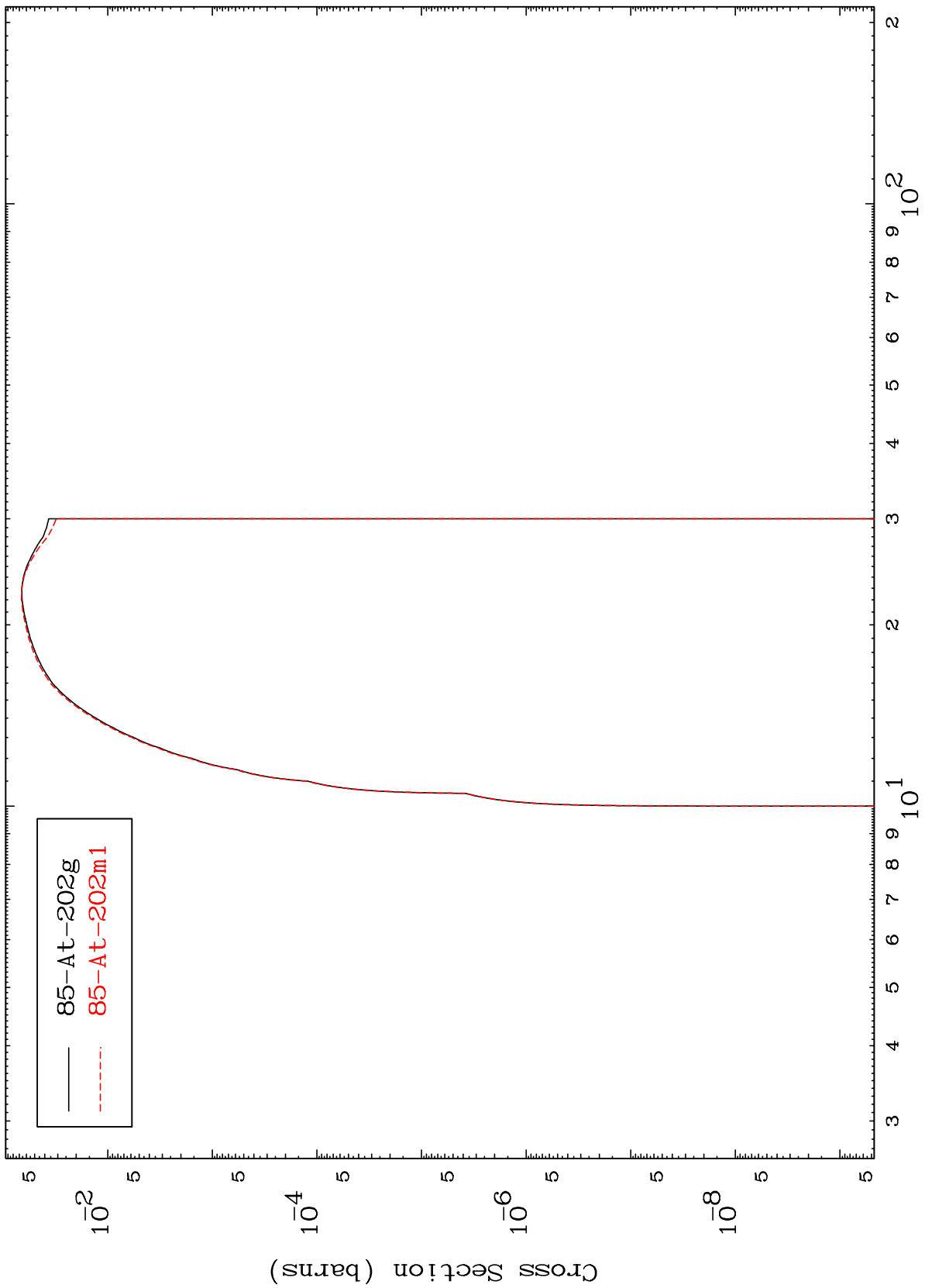


MAT 8710

$(n,2n) \alpha$

87-Fr-207

Radionuclide Production Cross Section



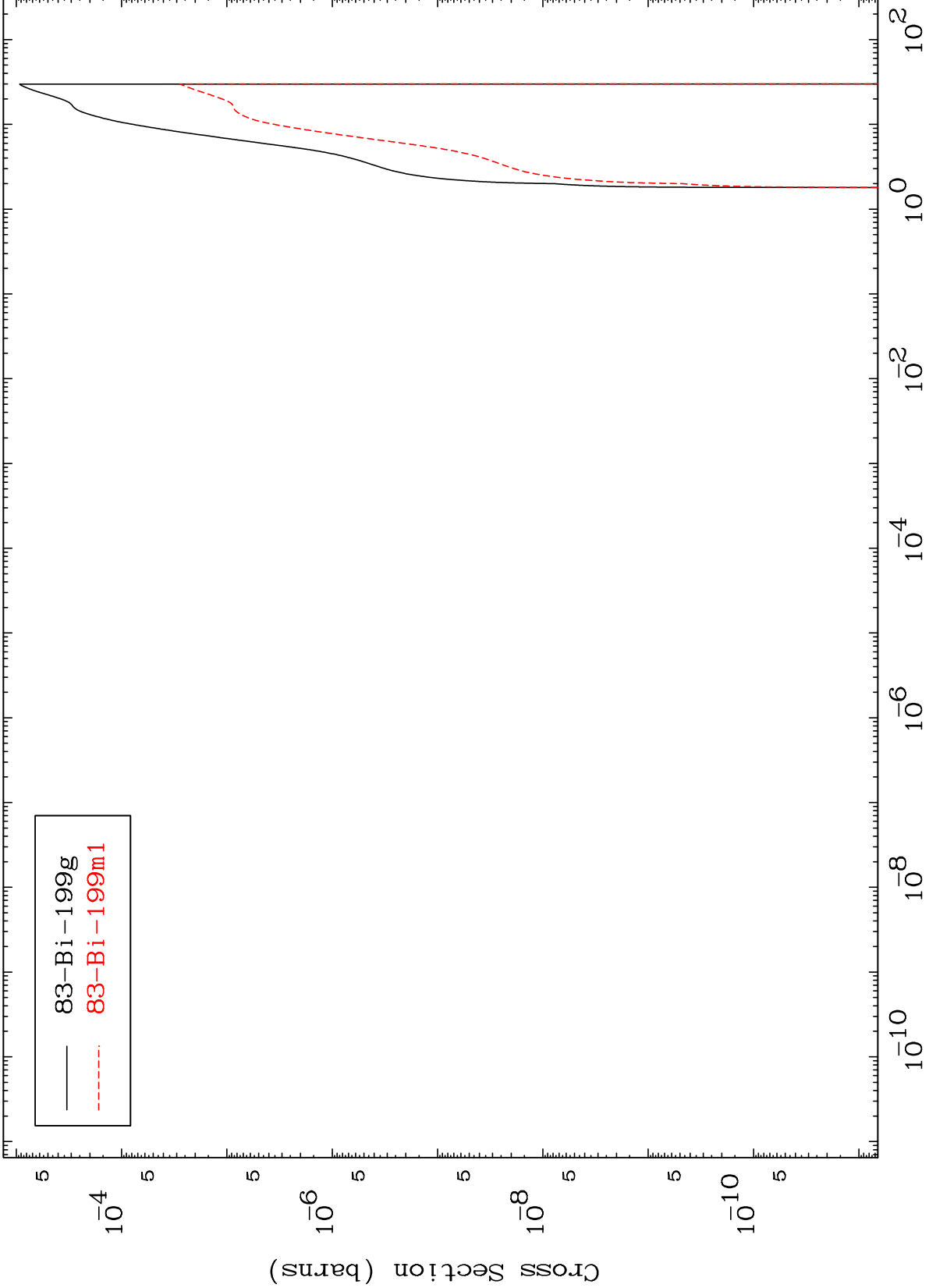
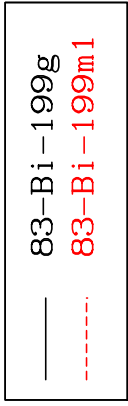
85-At-202g  
85-At-202m1

MAT 8710

(n,n') 2 $\alpha$

87-Fr-207

Radionuclide Production Cross Section



55

Incident Energy (MeV)

87-Fr-207

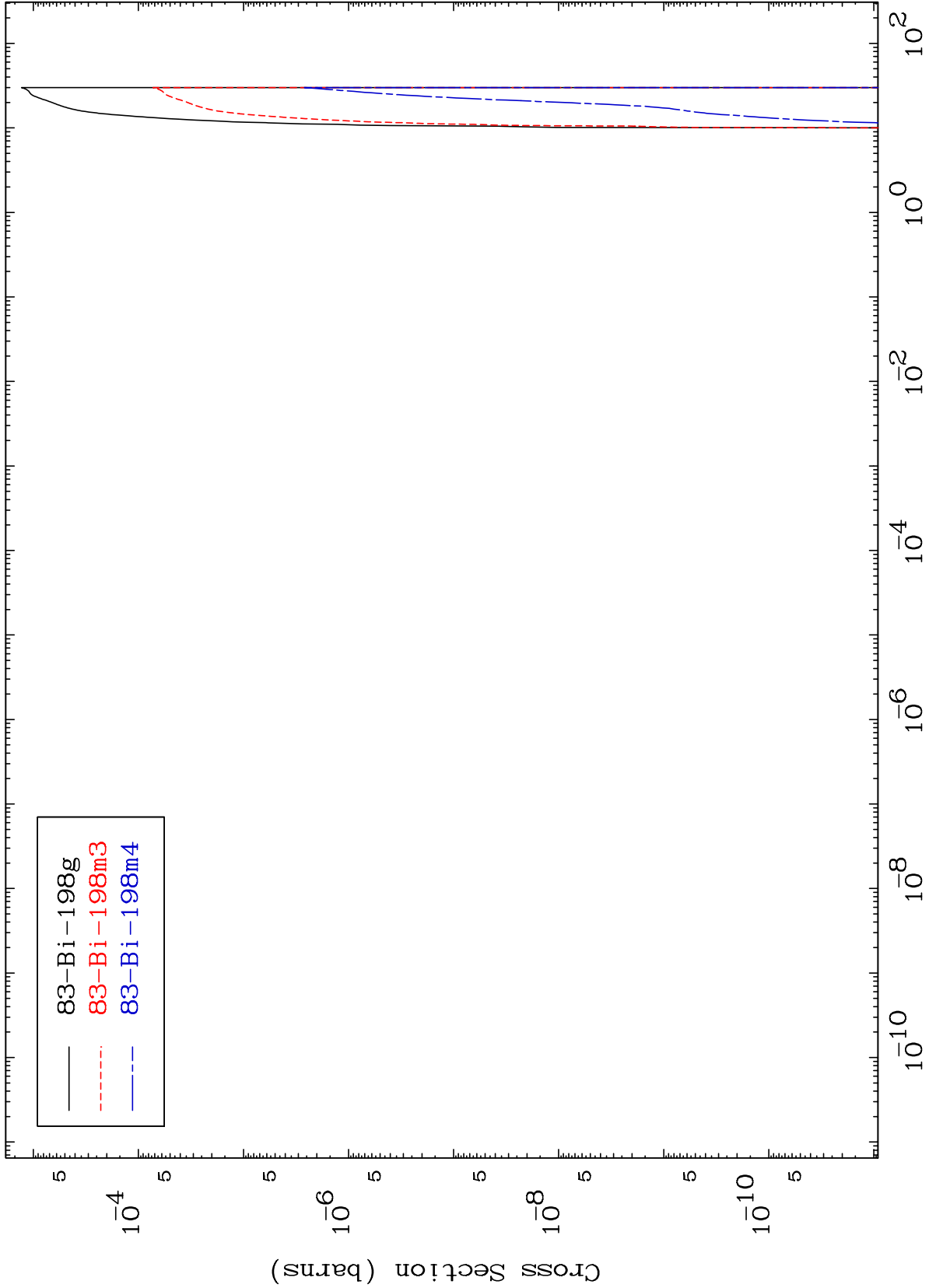


MAT 8710

(n,2n) 2 $\alpha$

87-Fr-207

Radionuclide Production Cross Section



56

Incident Energy (MeV)

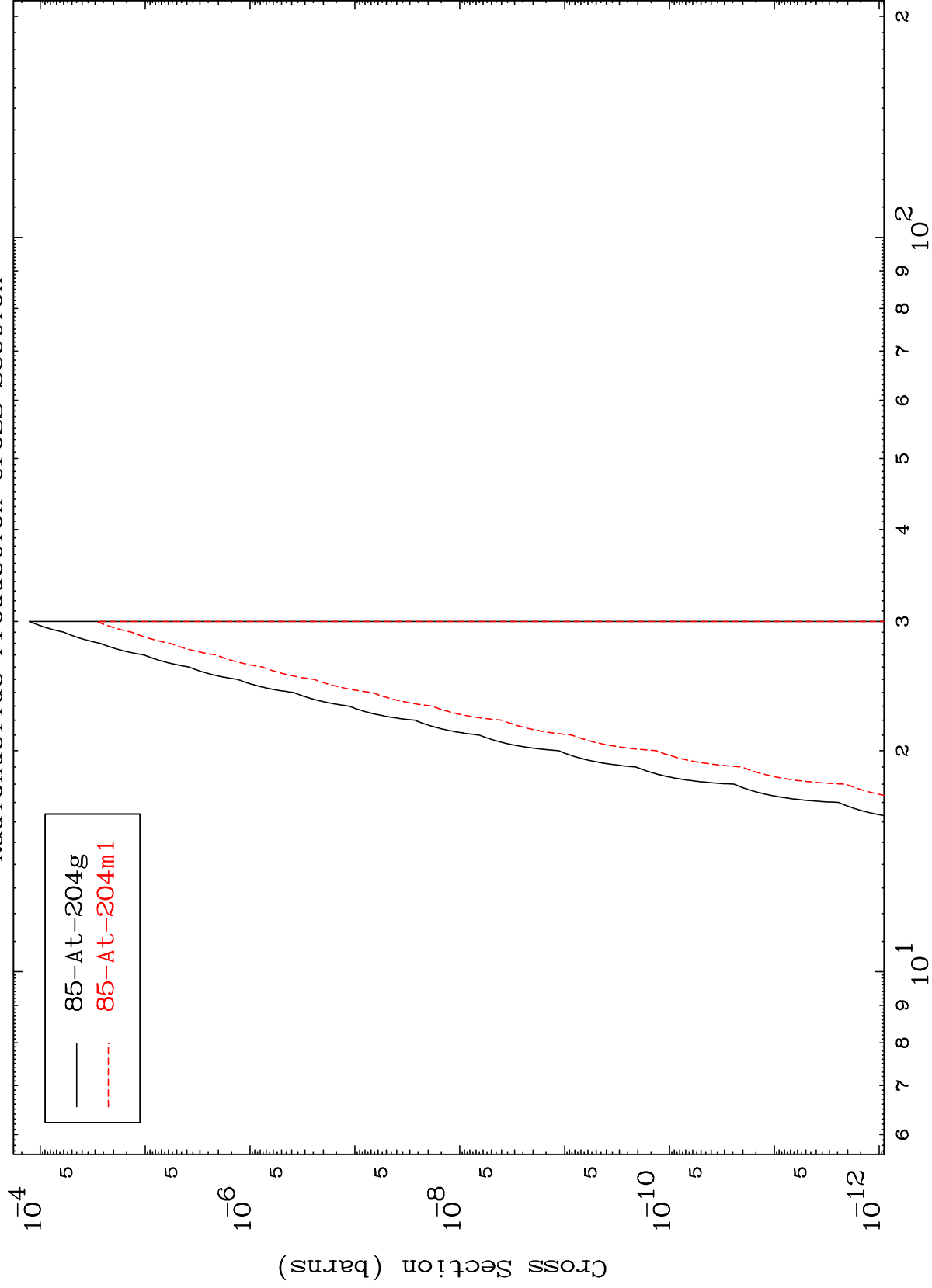
87-Fr-207

MAT 8710

(n,n') He-3

87-Fr-207

Radionuclide Production Cross Section



57

Incident Energy (MeV)

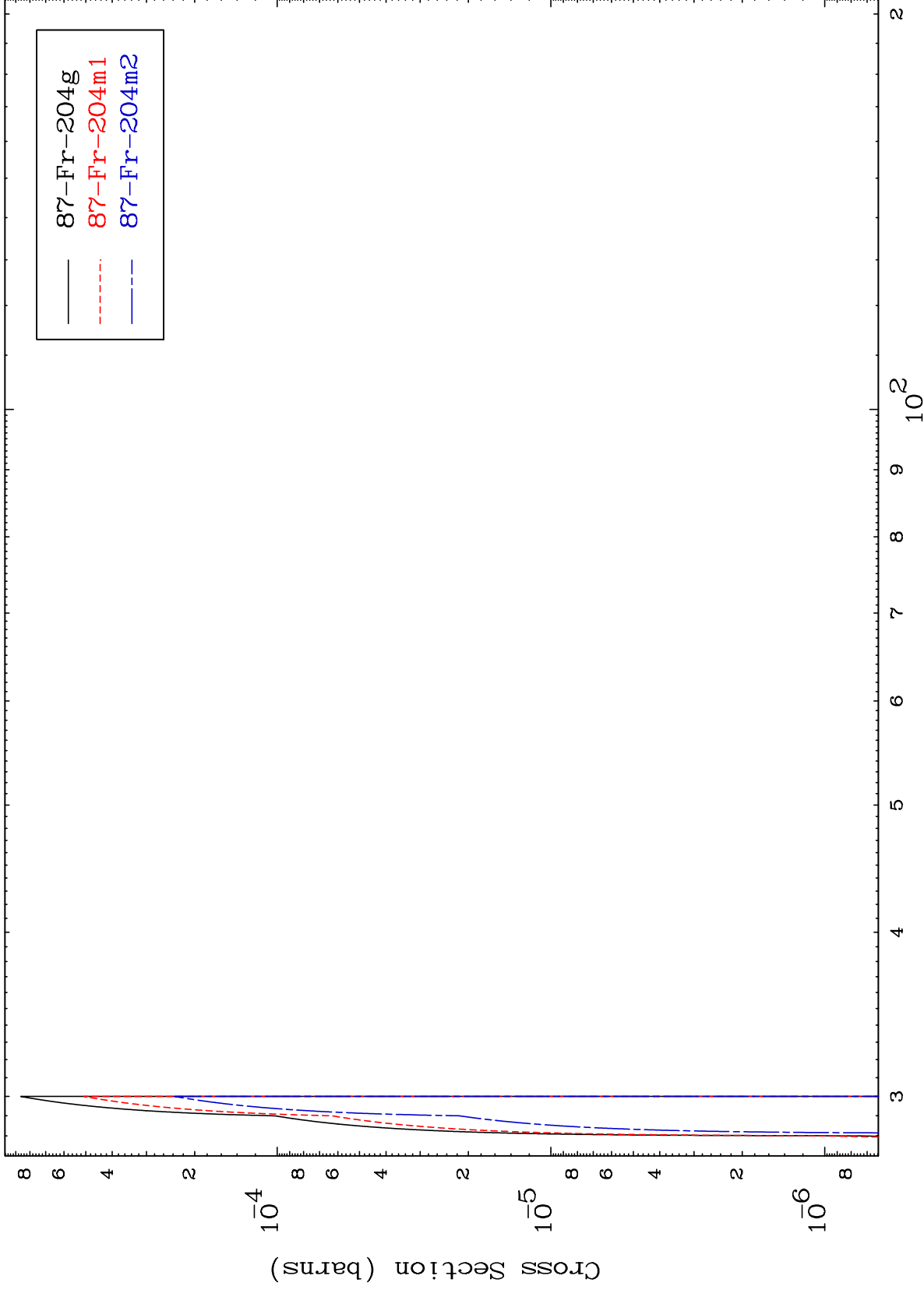
87-Fr-207

MAT 8710

(n,4n)

87-Fr-207

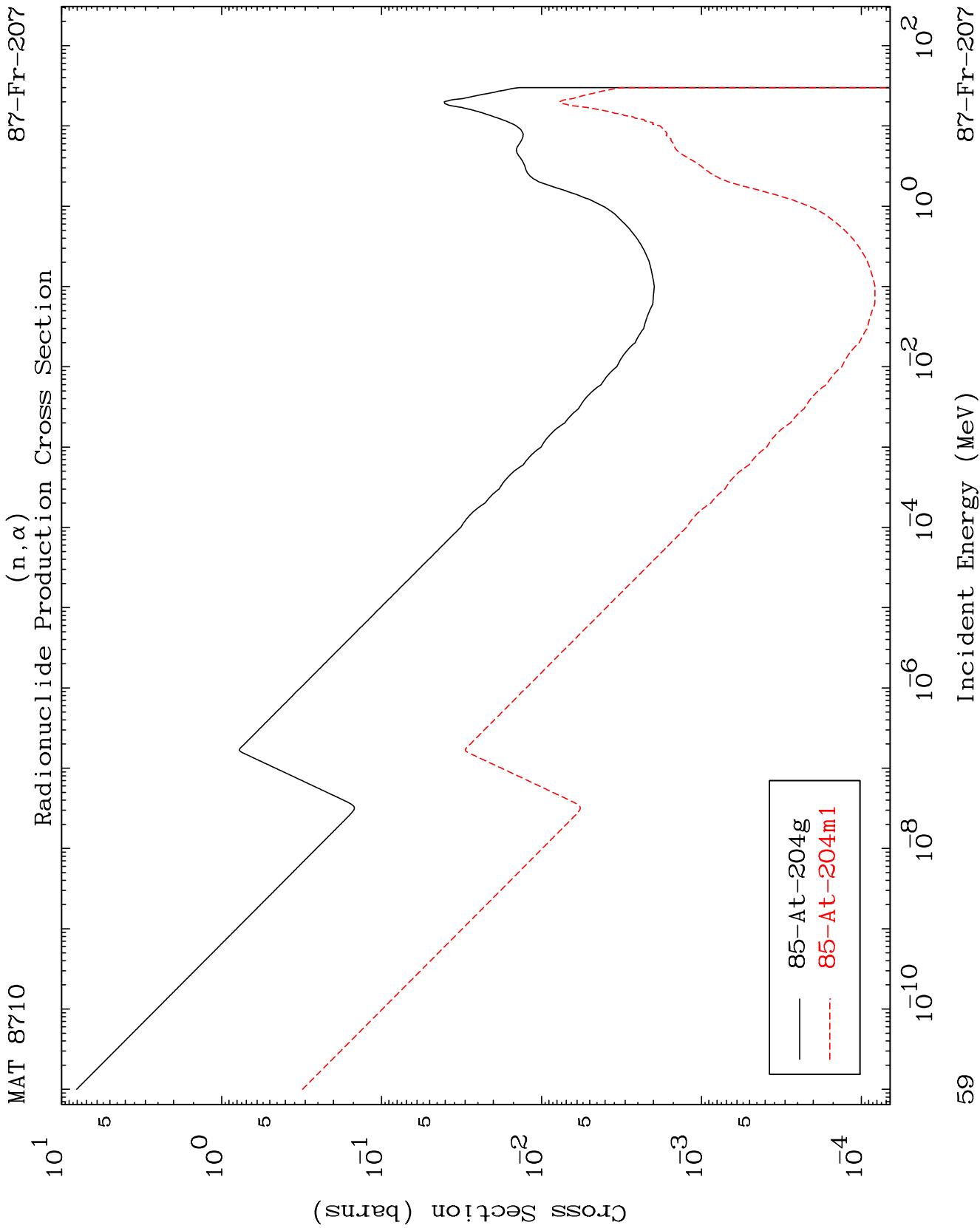
Radionuclide Production Cross Section



58

Incident Energy (MeV)

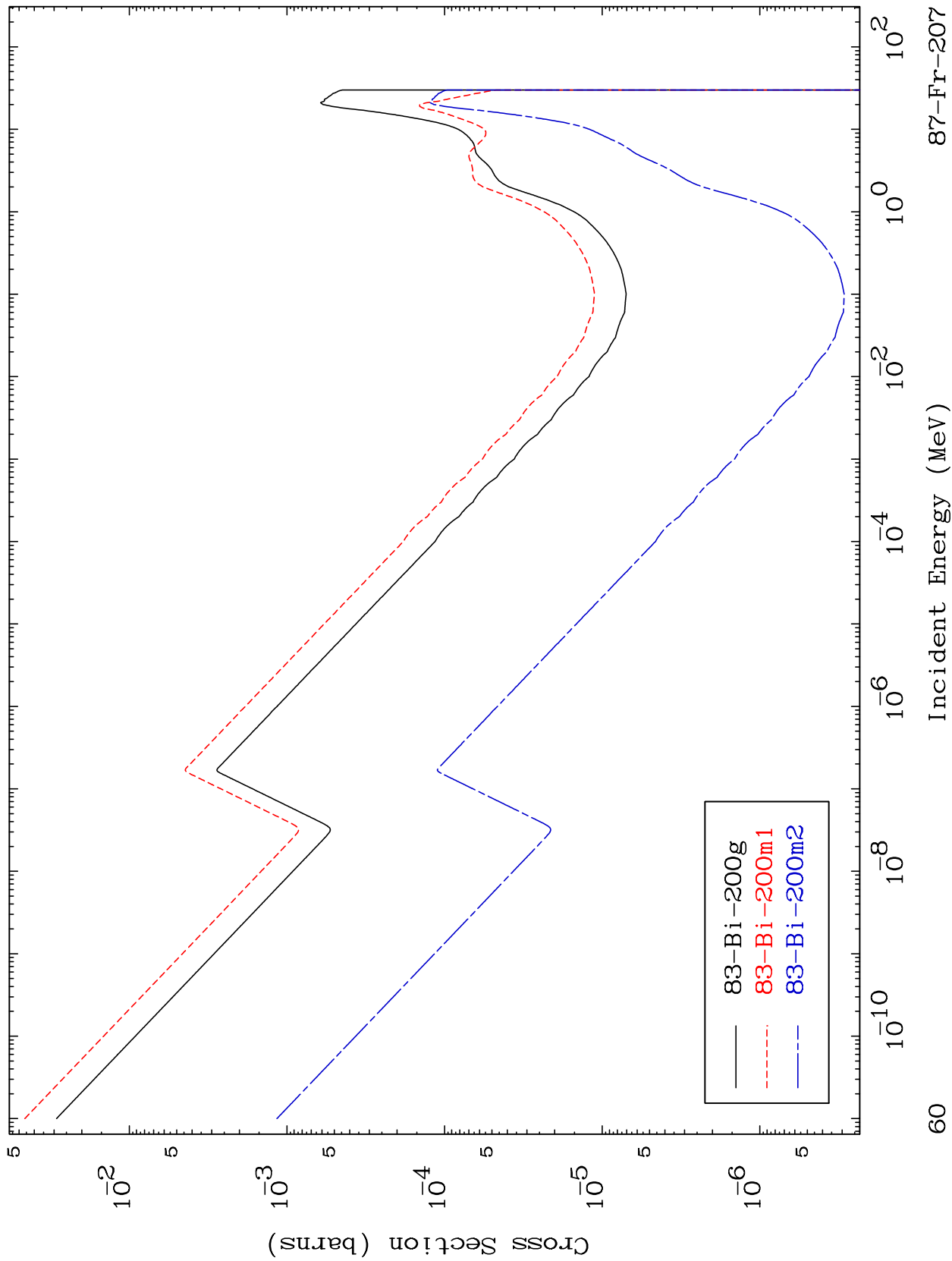
87-Fr-207



MAT 8710

87-Fr-207

Radionuclide Production Cross Section



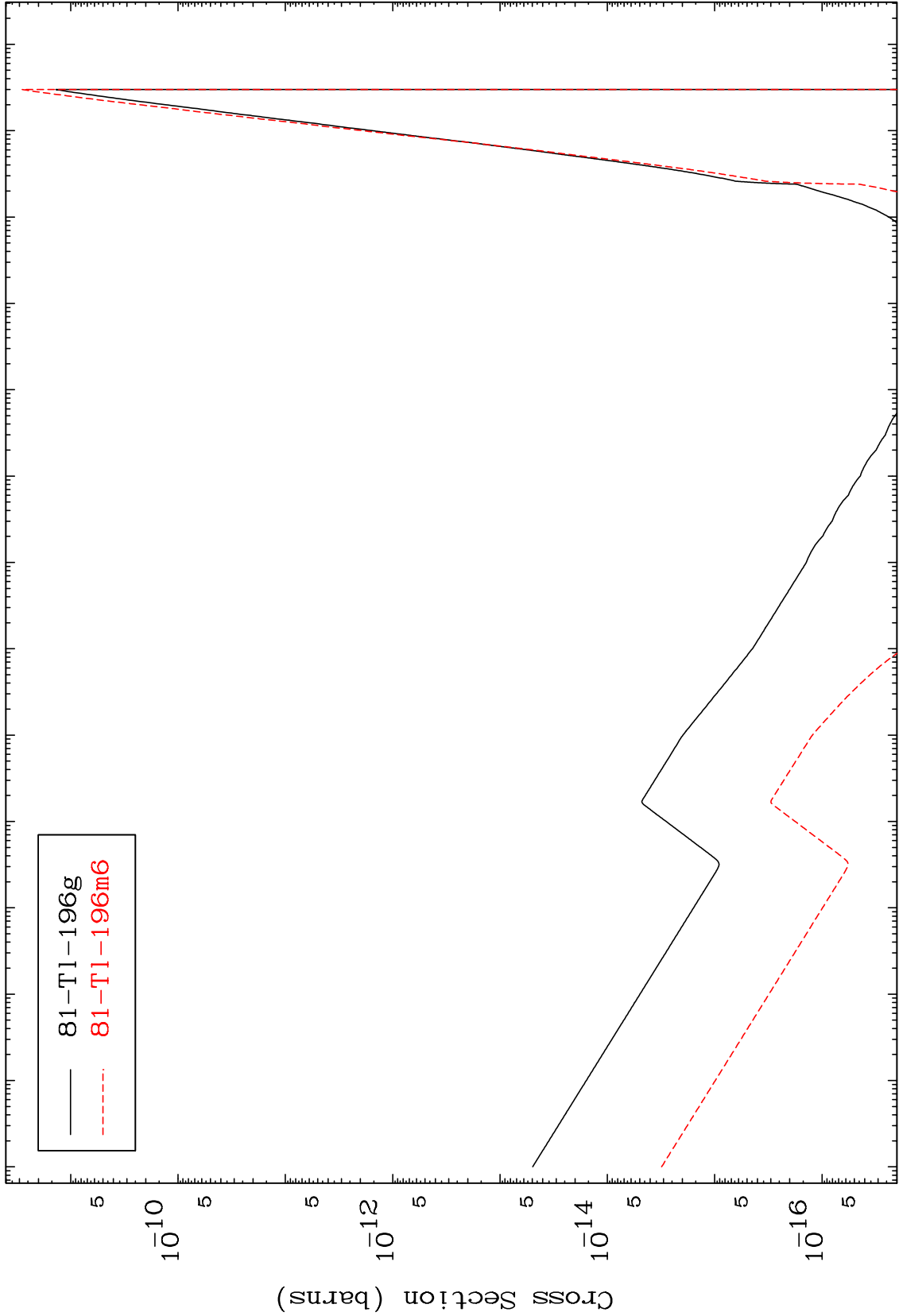
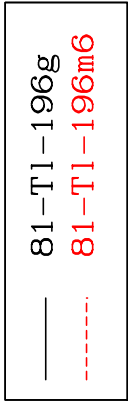
60

MAT 8710

(n, 3α)

87-Fr-207

Radionuclide Production Cross Section



87-Fr-207

10<sup>-2</sup>

10<sup>-4</sup>

10<sup>-6</sup>

10<sup>-8</sup>

10<sup>-10</sup>

10<sup>0</sup>

10<sup>2</sup>

Incident Energy (MeV)

87-Fr-207

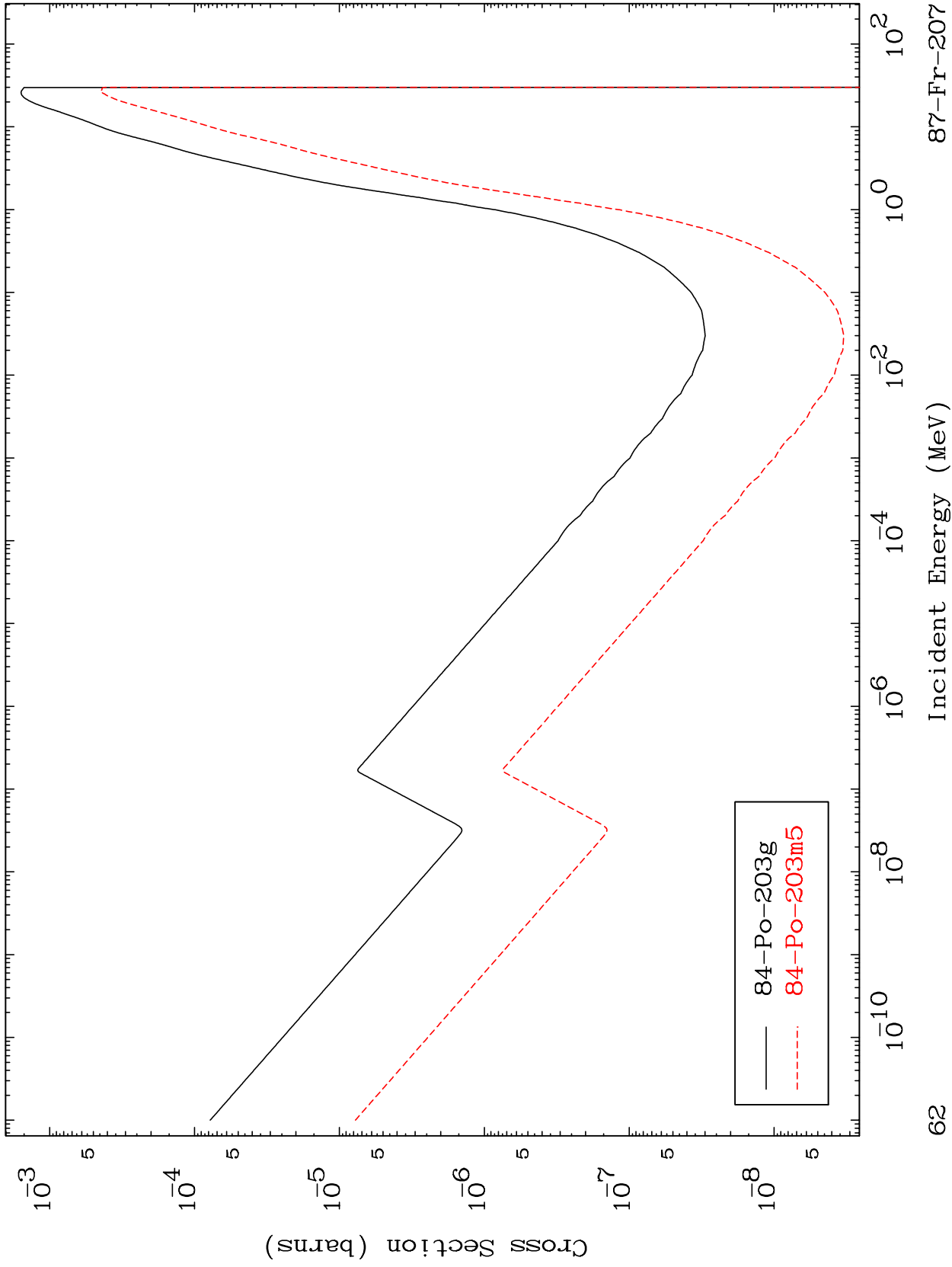
61

MAT 8710

(n,p)  $\alpha$

87-Fr-207

Radionuclide Production Cross Section

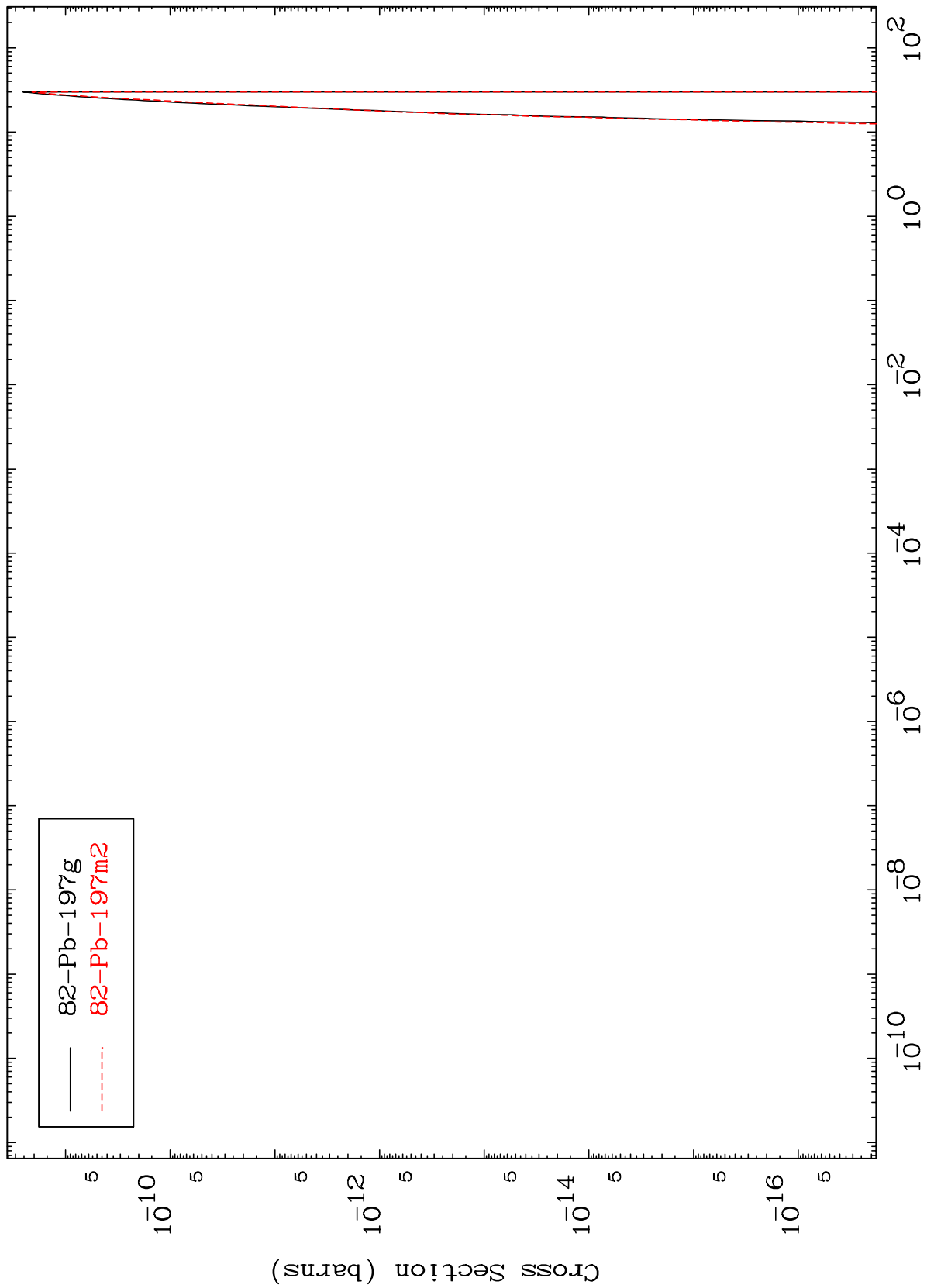


MAT 8710

(n,t) 2 $\alpha$

87-Fr-207

Radionuclide Production Cross Section



63

Incident Energy (MeV)

87-Fr-207

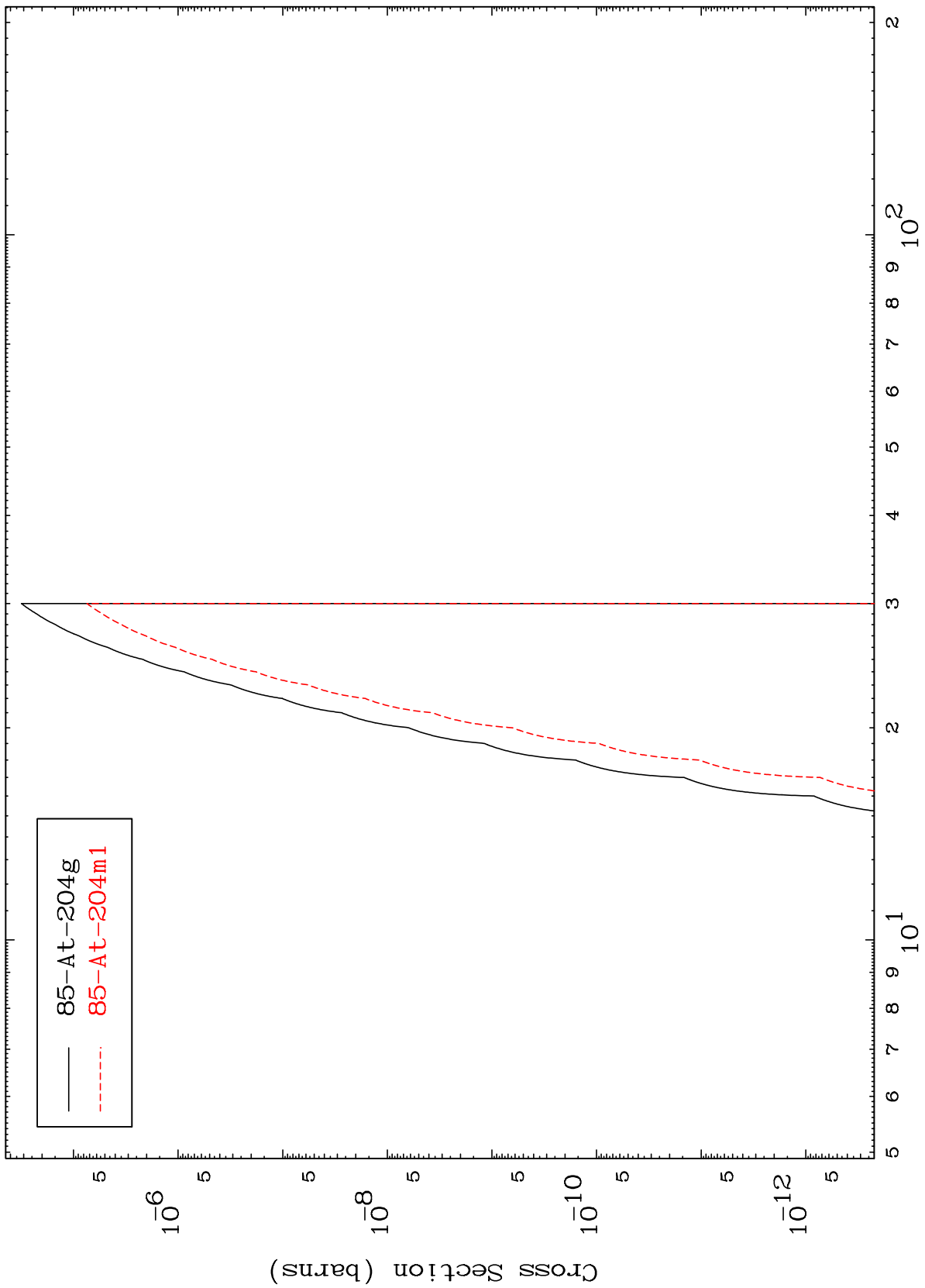


MAT 8710

(n,p) t

87-Fr-207

Radionuclide Production Cross Section



85-At-204g  
85-At-204m1

64

Incident Energy (MeV)

87-Fr-207