

Program EVALPLOT  
(Version 2015-2)

by

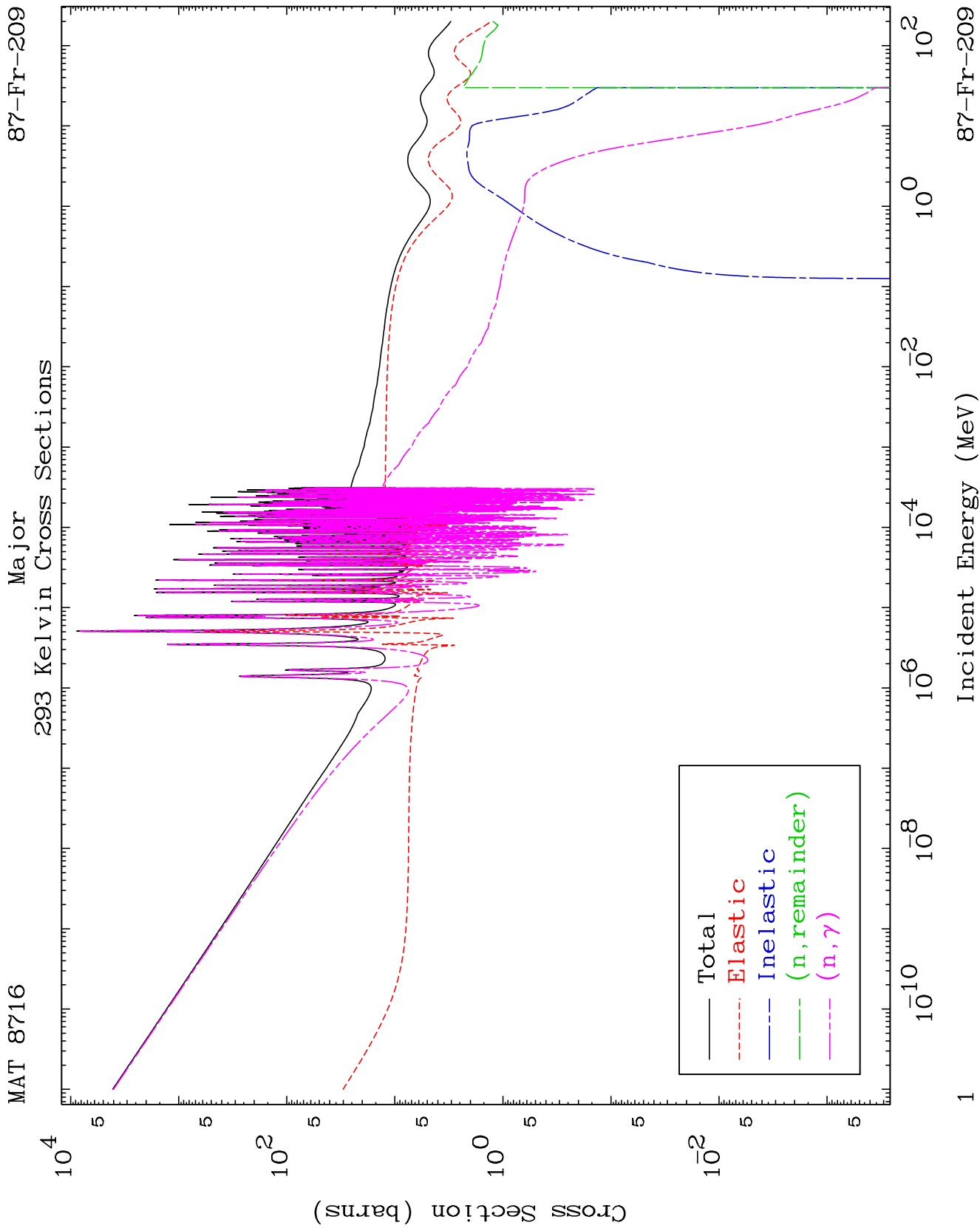
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(Present Contact Information)

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Livermore, CA 94550  
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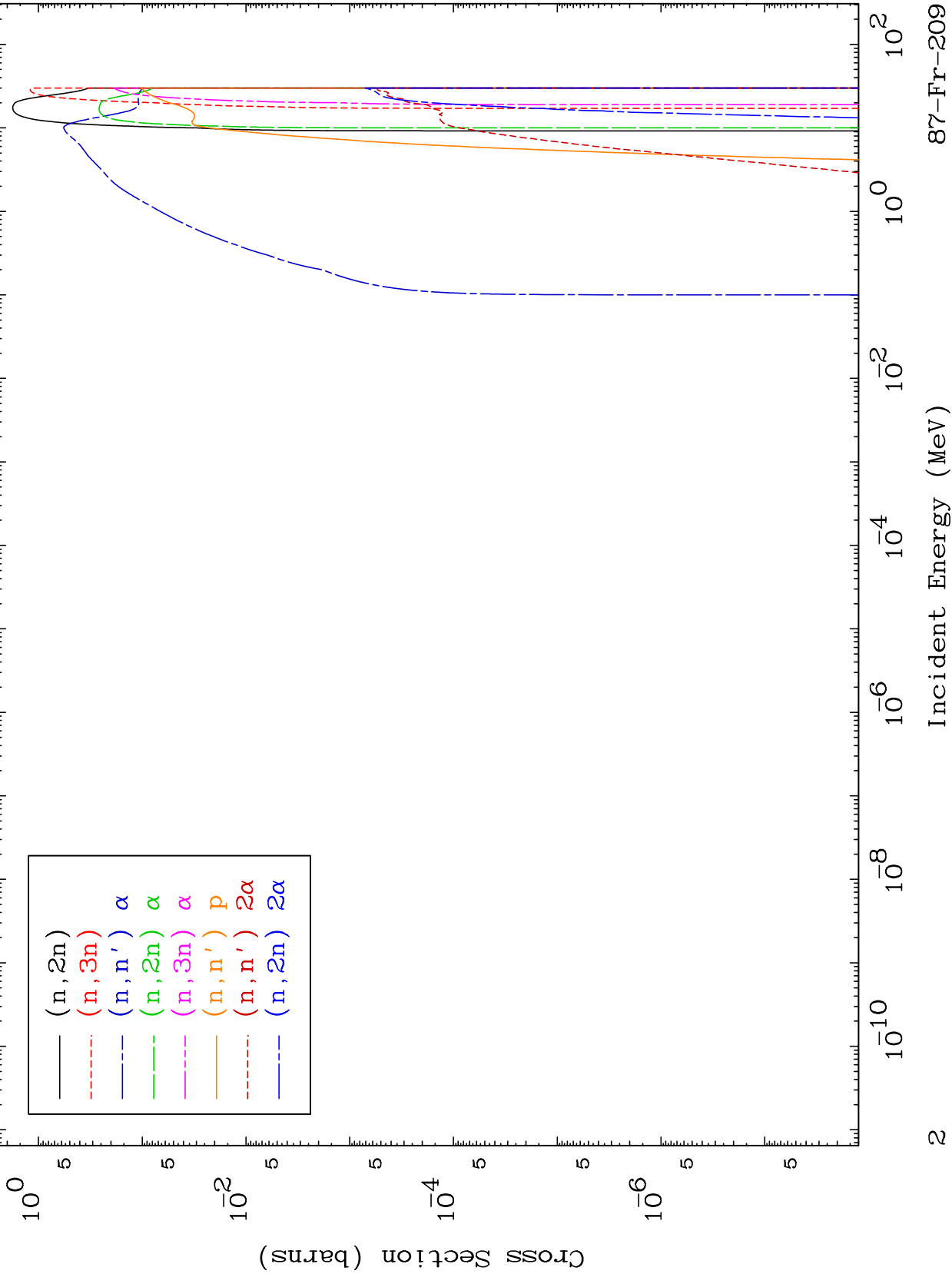
Press Mouse Button to Start



MAT 8716

Neutron Production  
293 Kelvin Cross Sections

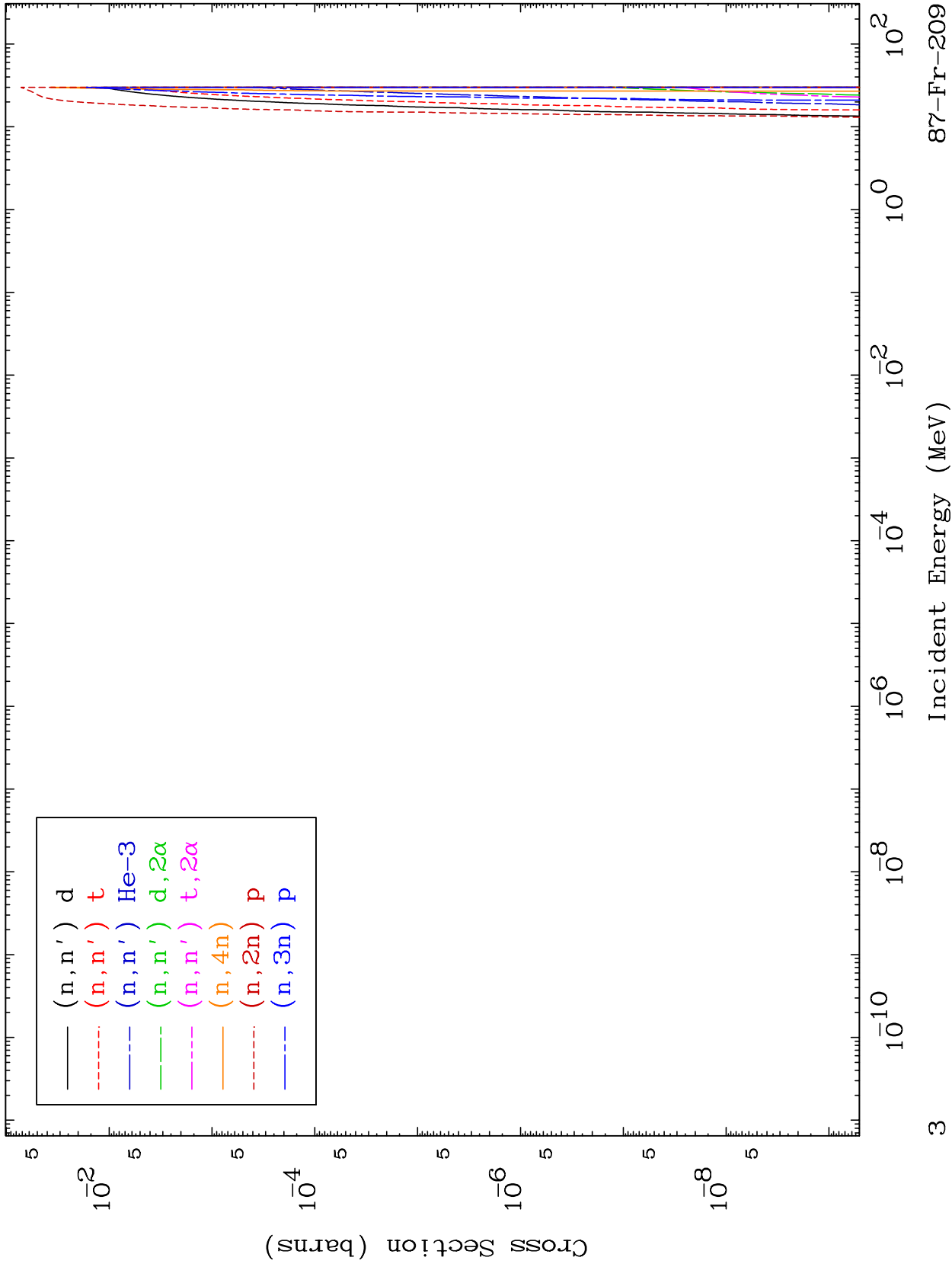
87-Fr-209



MAT 8716

Neutron Production  
293 Kelvin Cross Sections

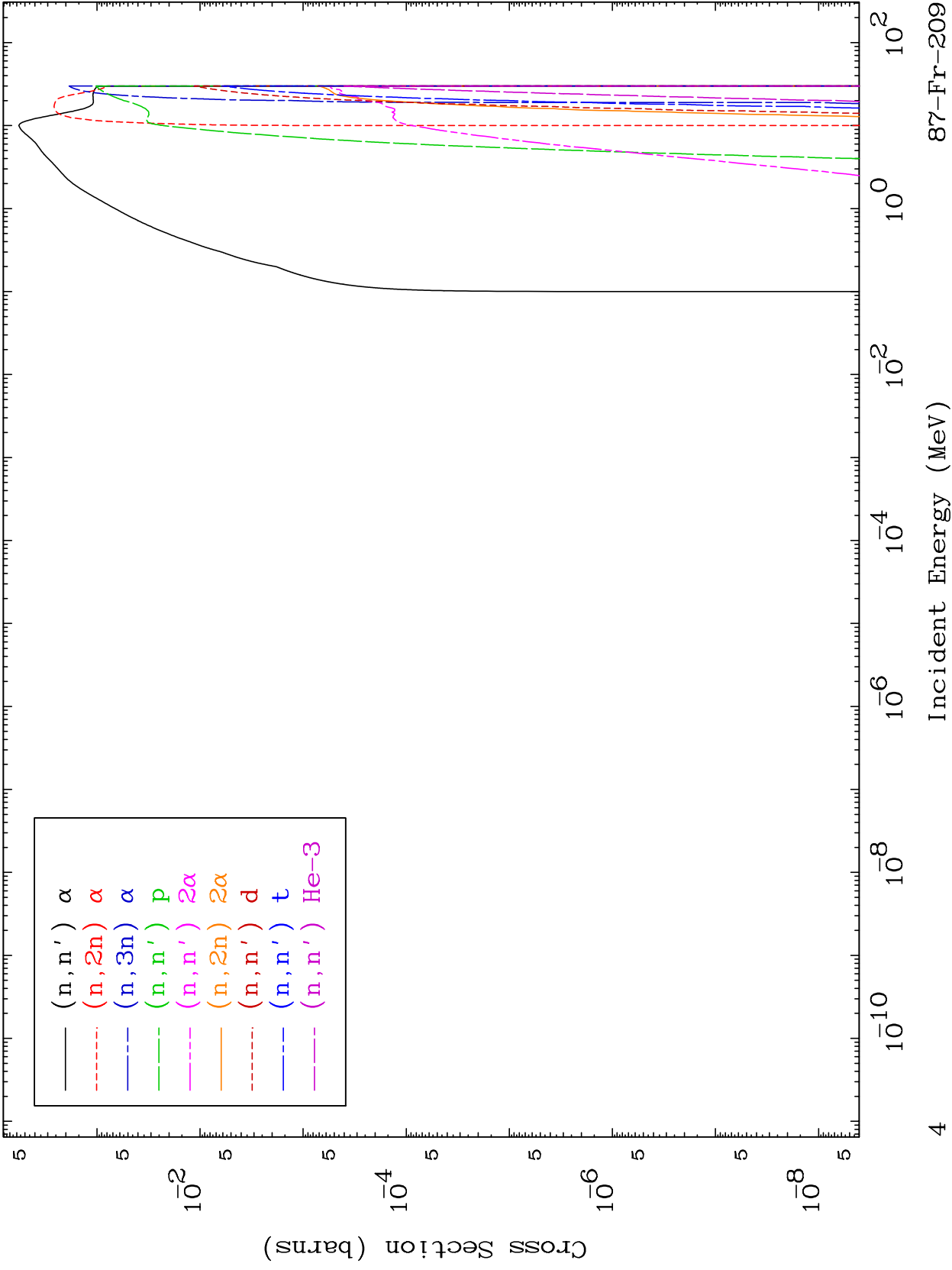
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MAT 8716

Charged Particle  
293 Kelvin Cross Sections

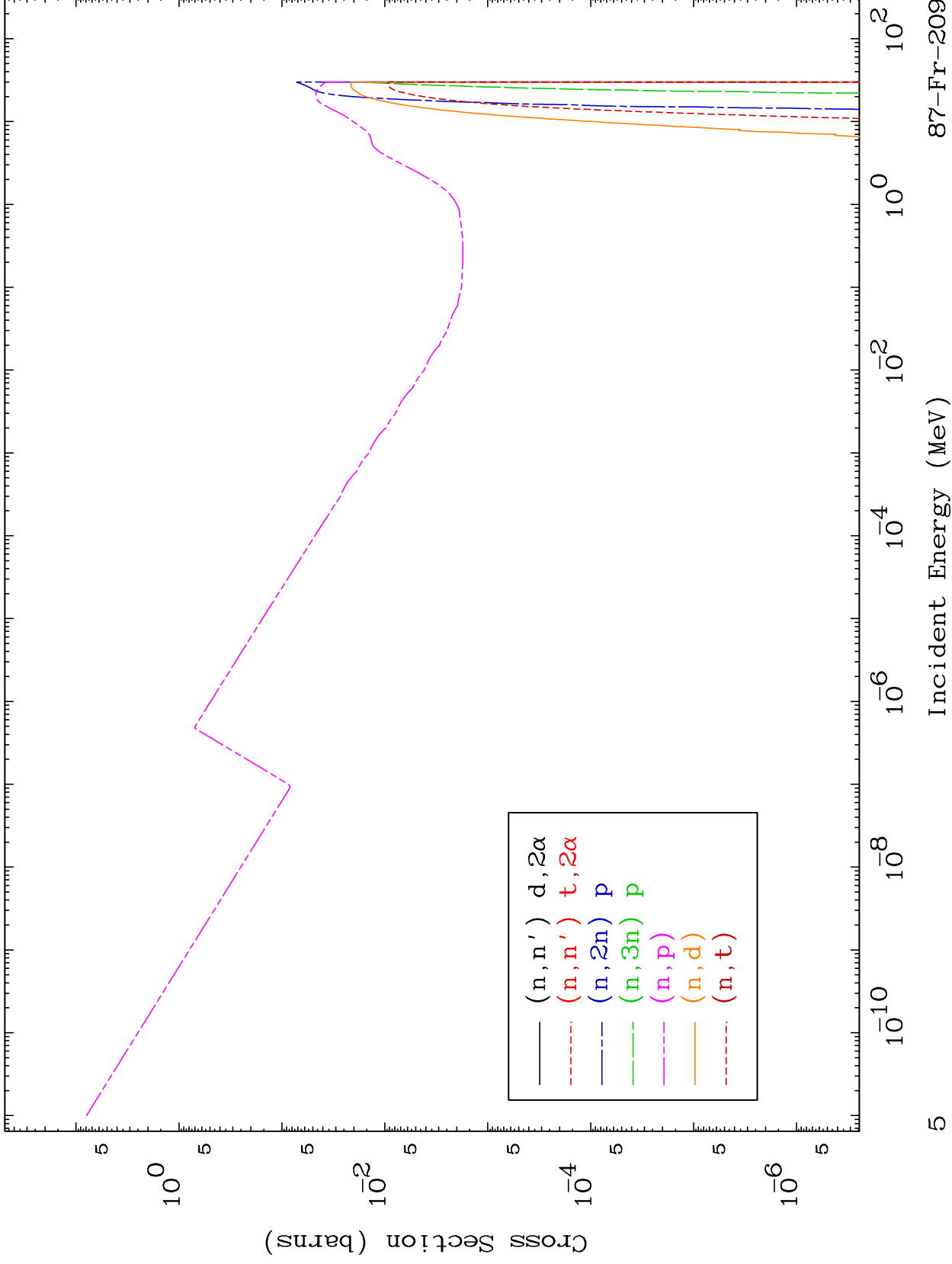
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Charged Particle  
293 Kelvin Cross Sections

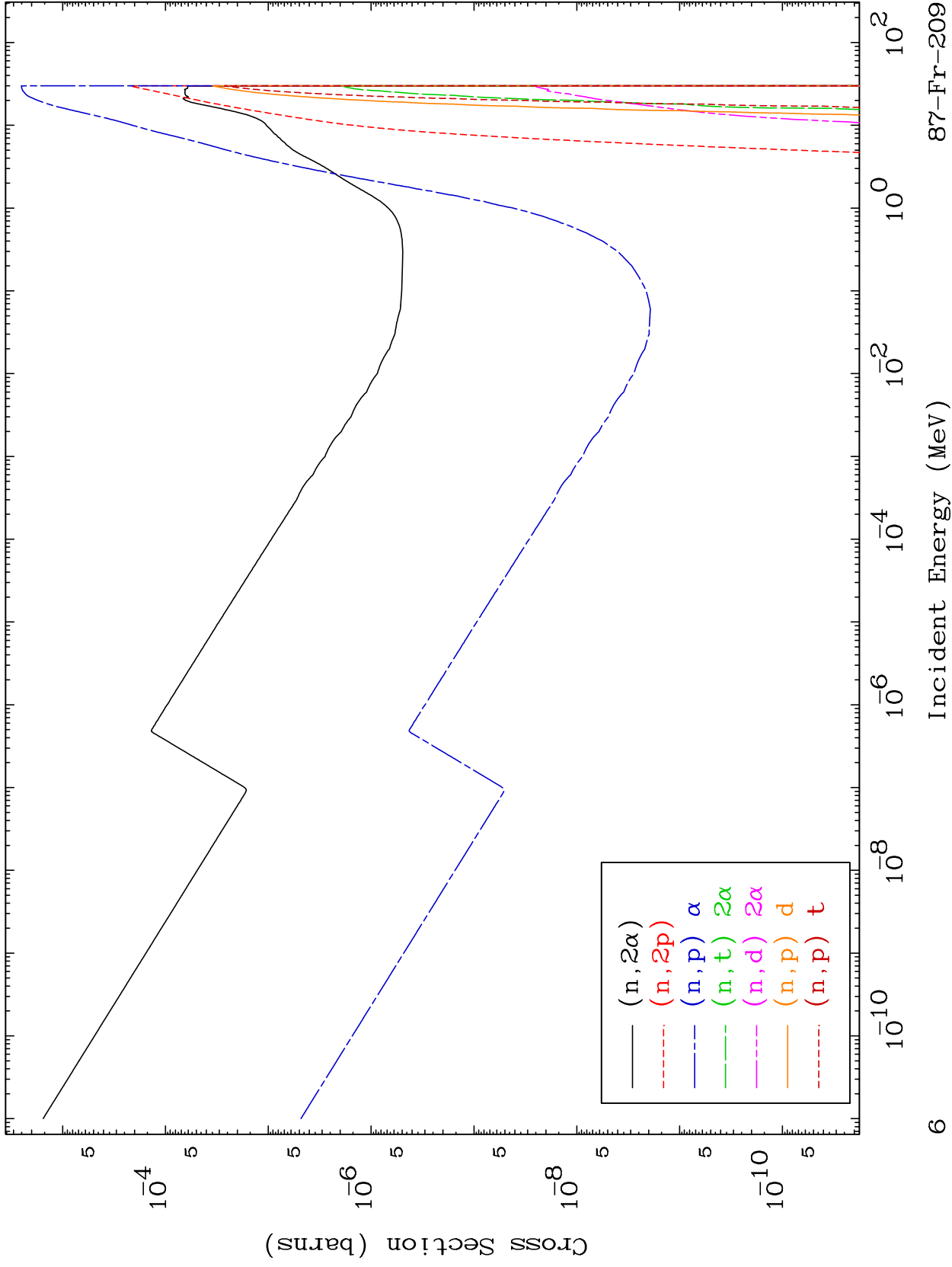
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Charged Particle  
293 Kelvin Cross Sections

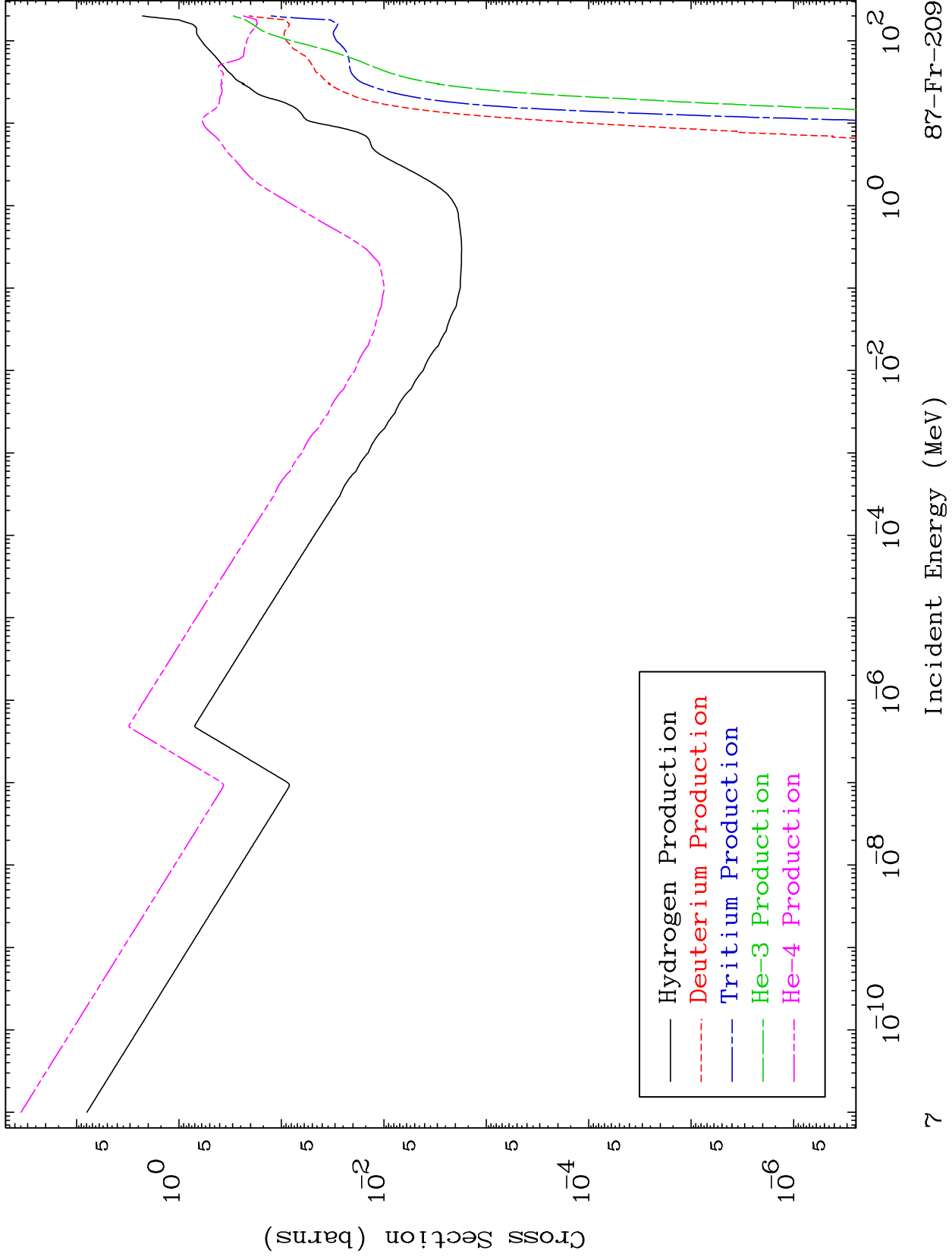
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Particle Production  
293 Kelvin Cross Sections

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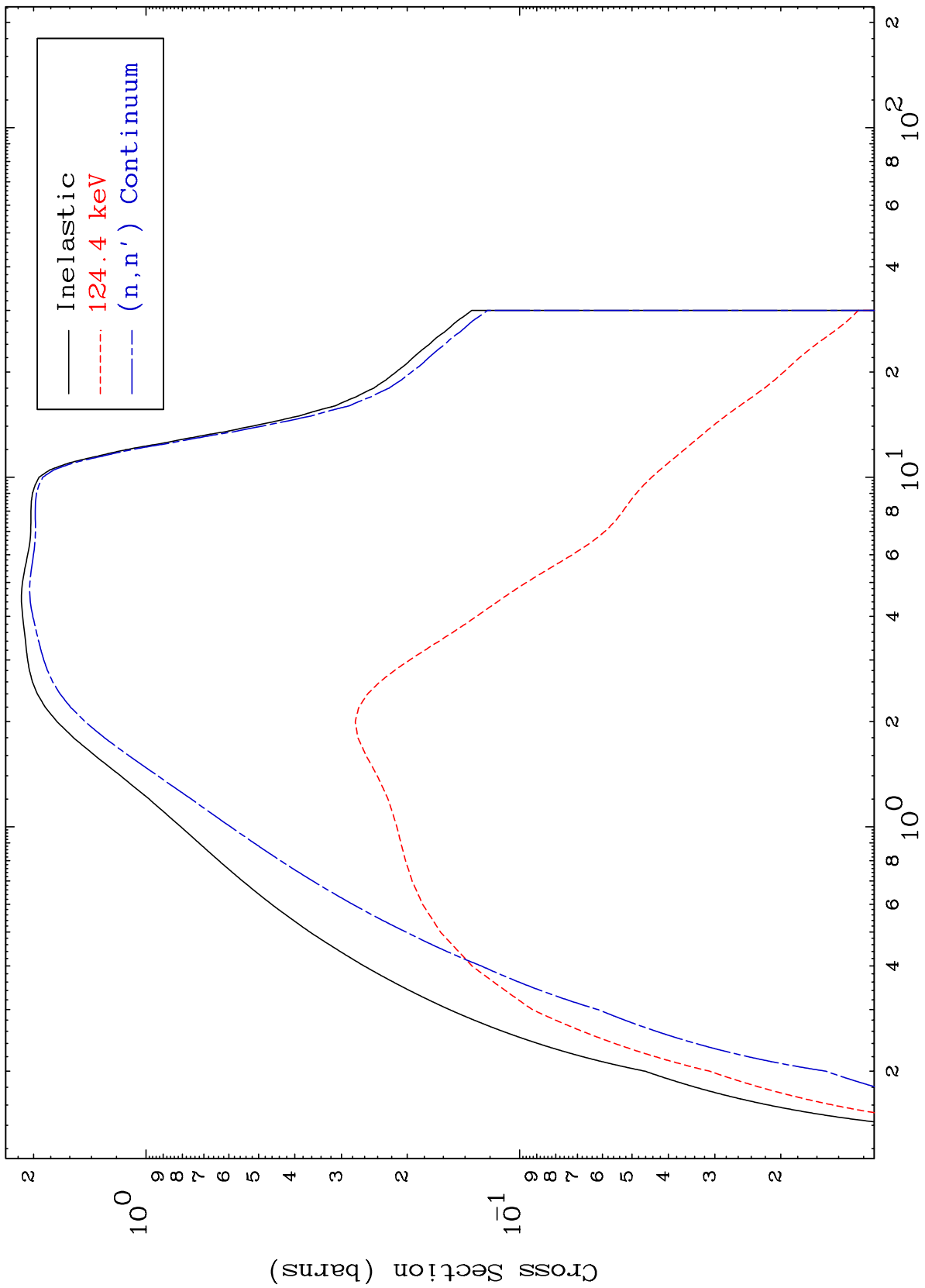


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(n,n') Level

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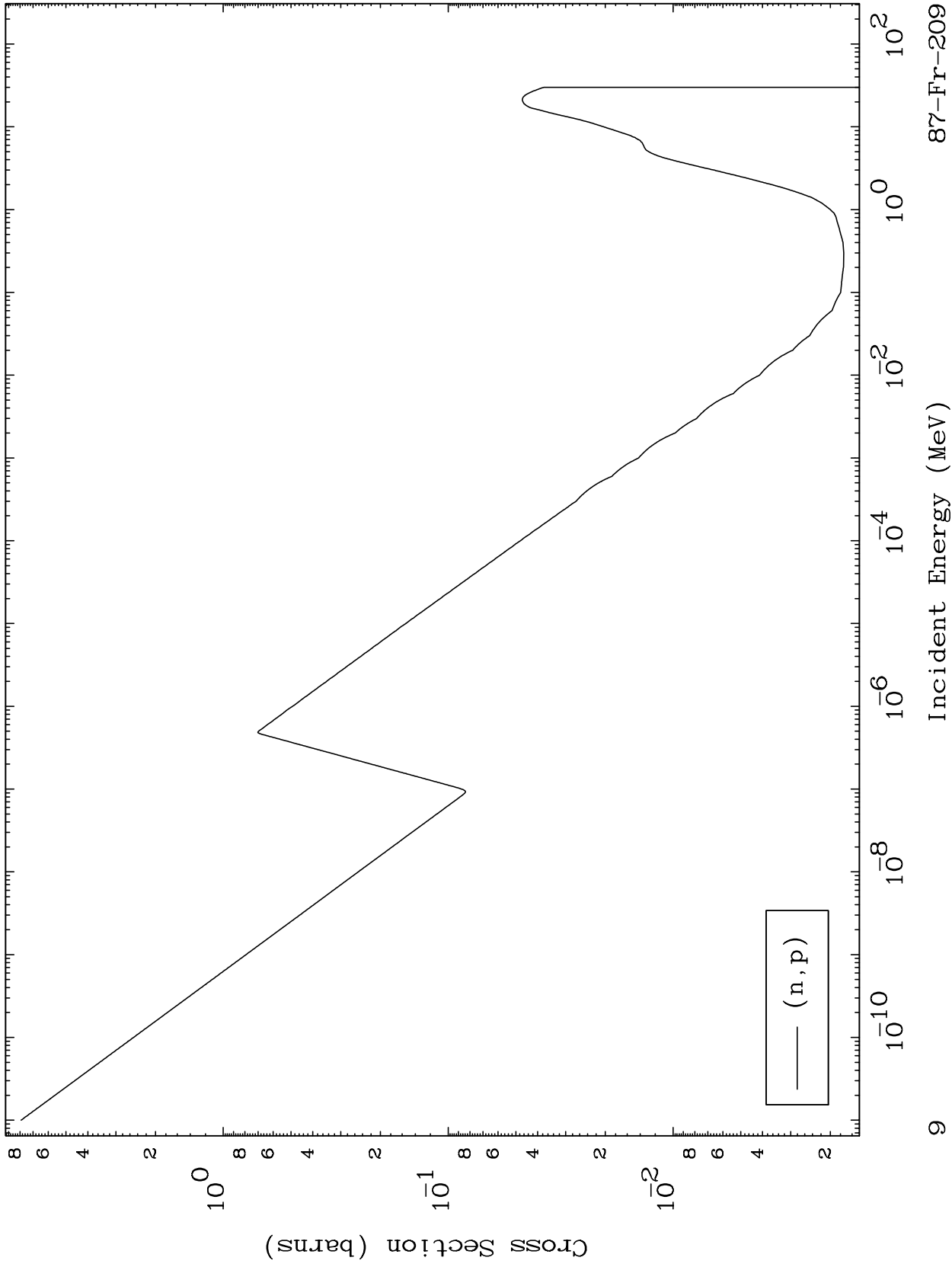
293 Kelvin Cross Sections



MAT 8716

(n,p) Levels  
293 Kelvin Cross Sections

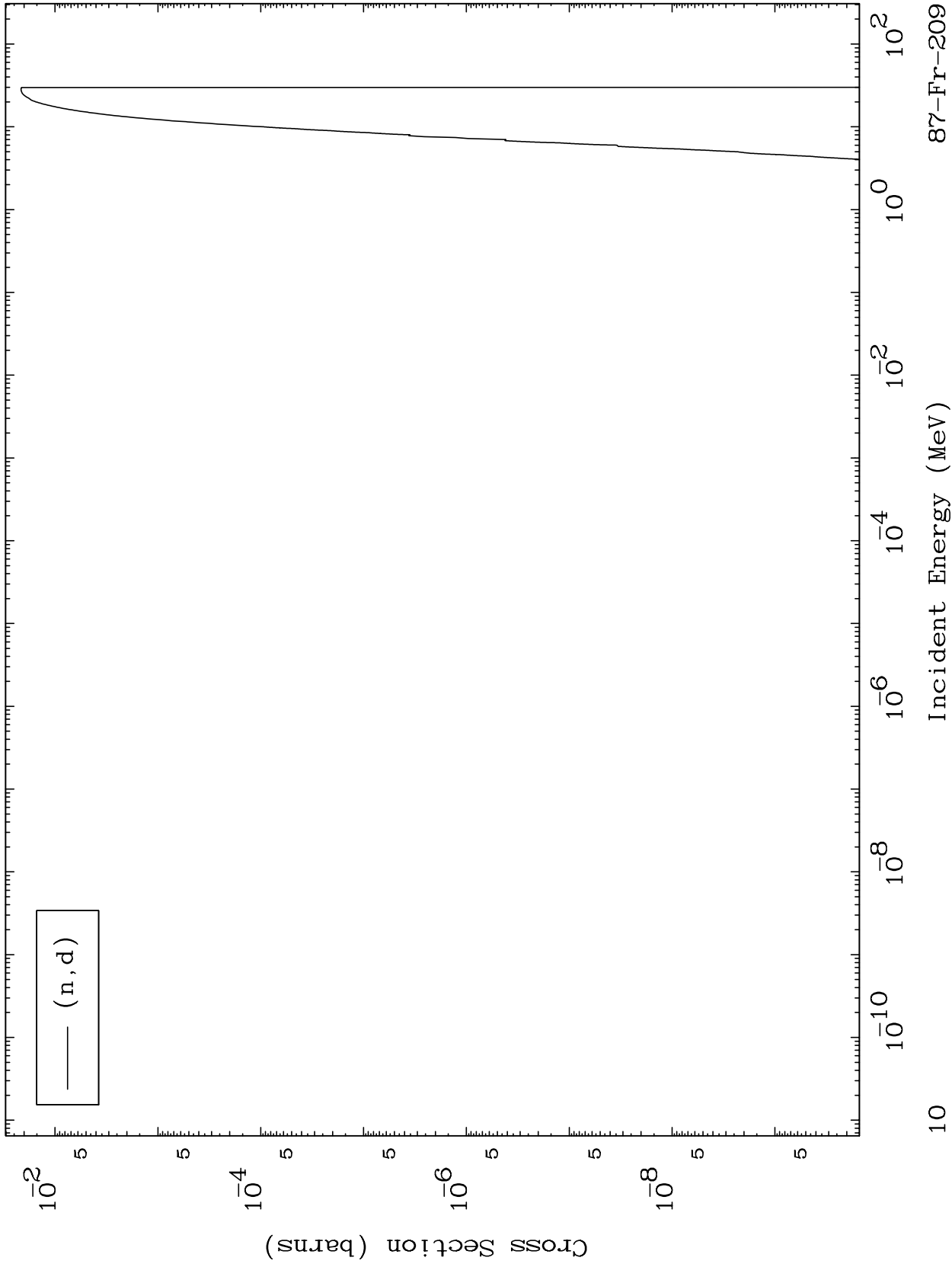
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MAT 8716

(n,d) Levels  
293 Kelvin Cross Sections

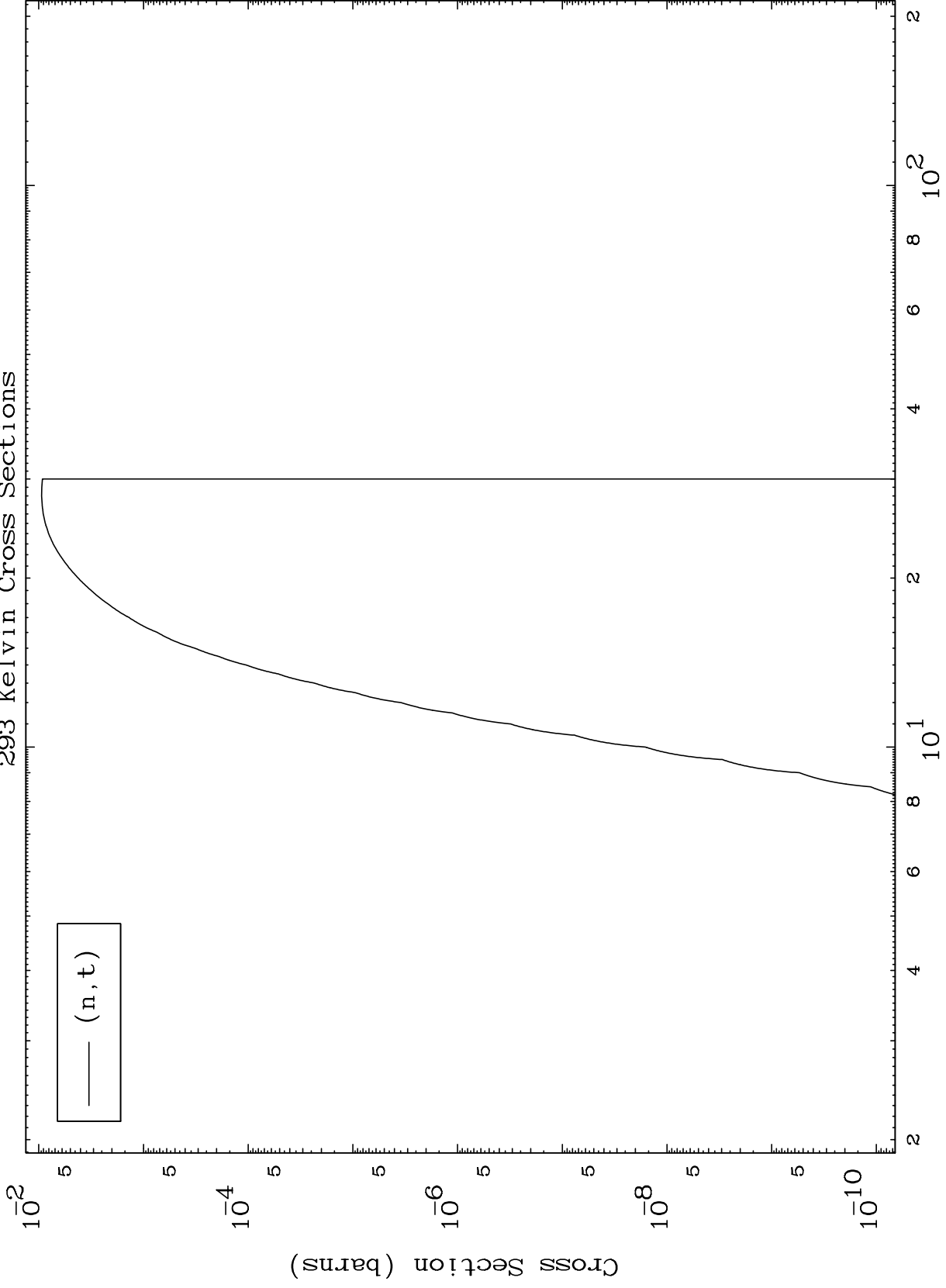
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(n,t) Levels  
293 Kelvin Cross Sections

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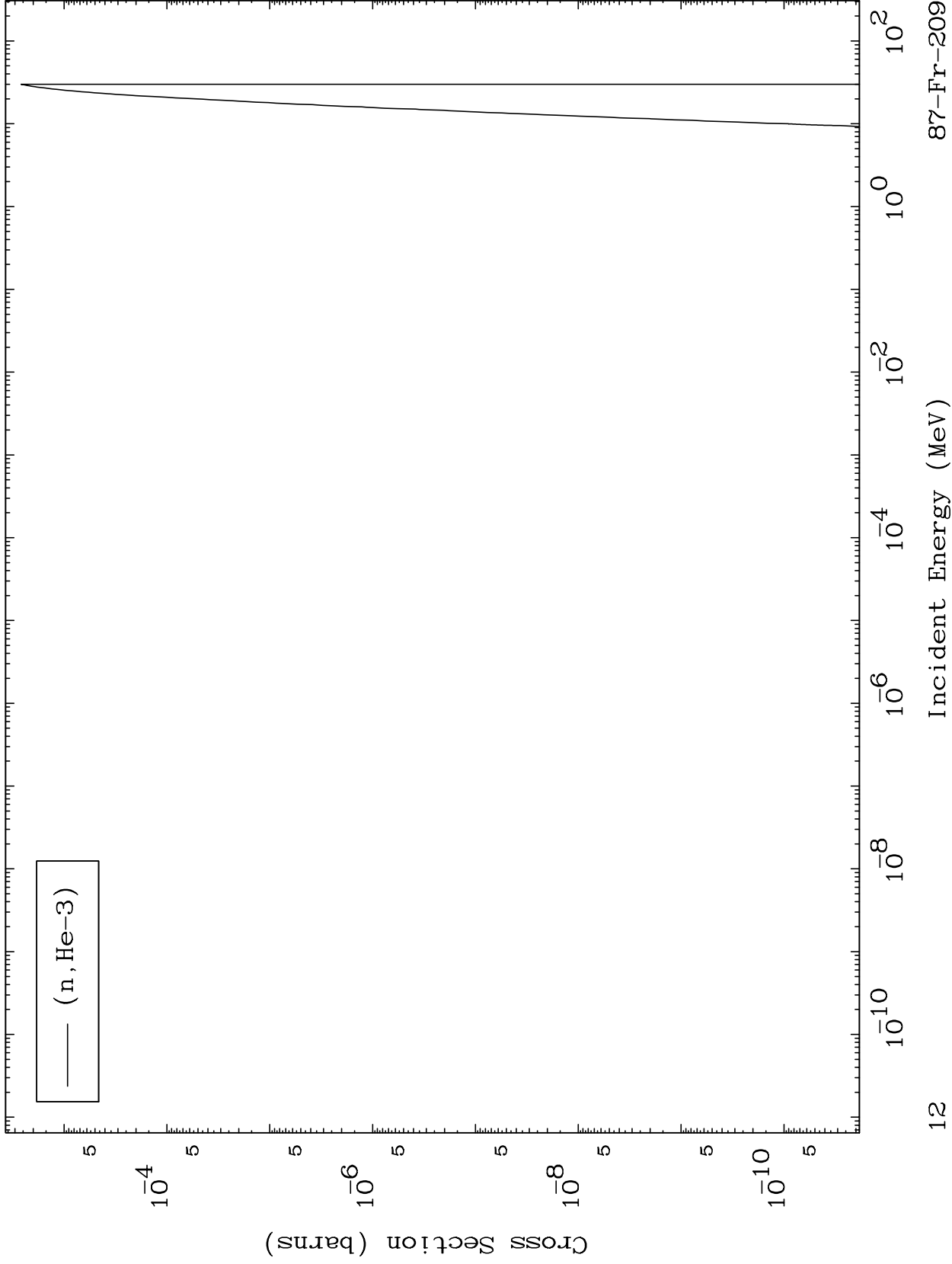


(n,t)

MAT 8716

(n,He3) Levels  
293 Kelvin Cross Sections

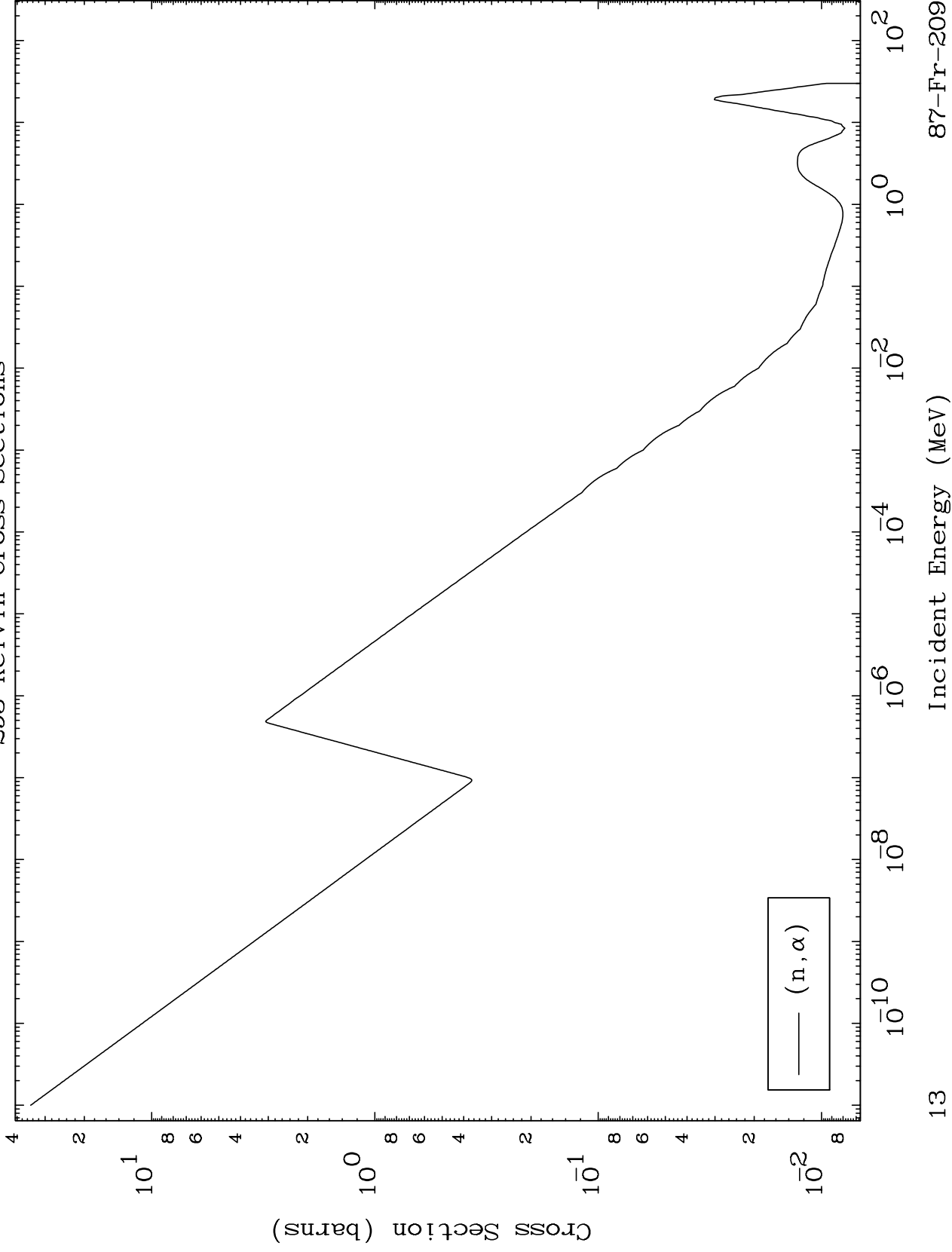
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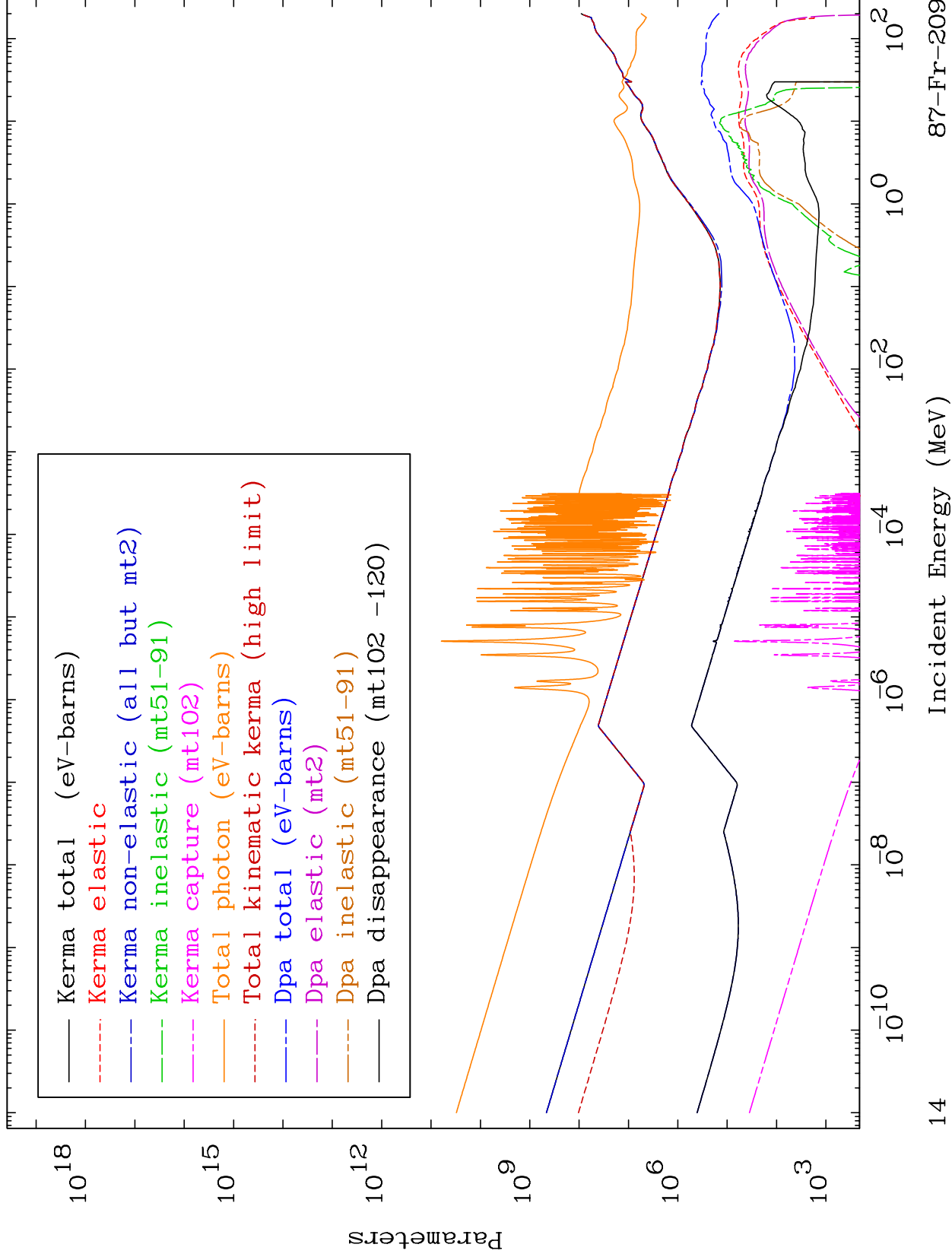


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(n,α) Levels  
293 Kelvin Cross Sections

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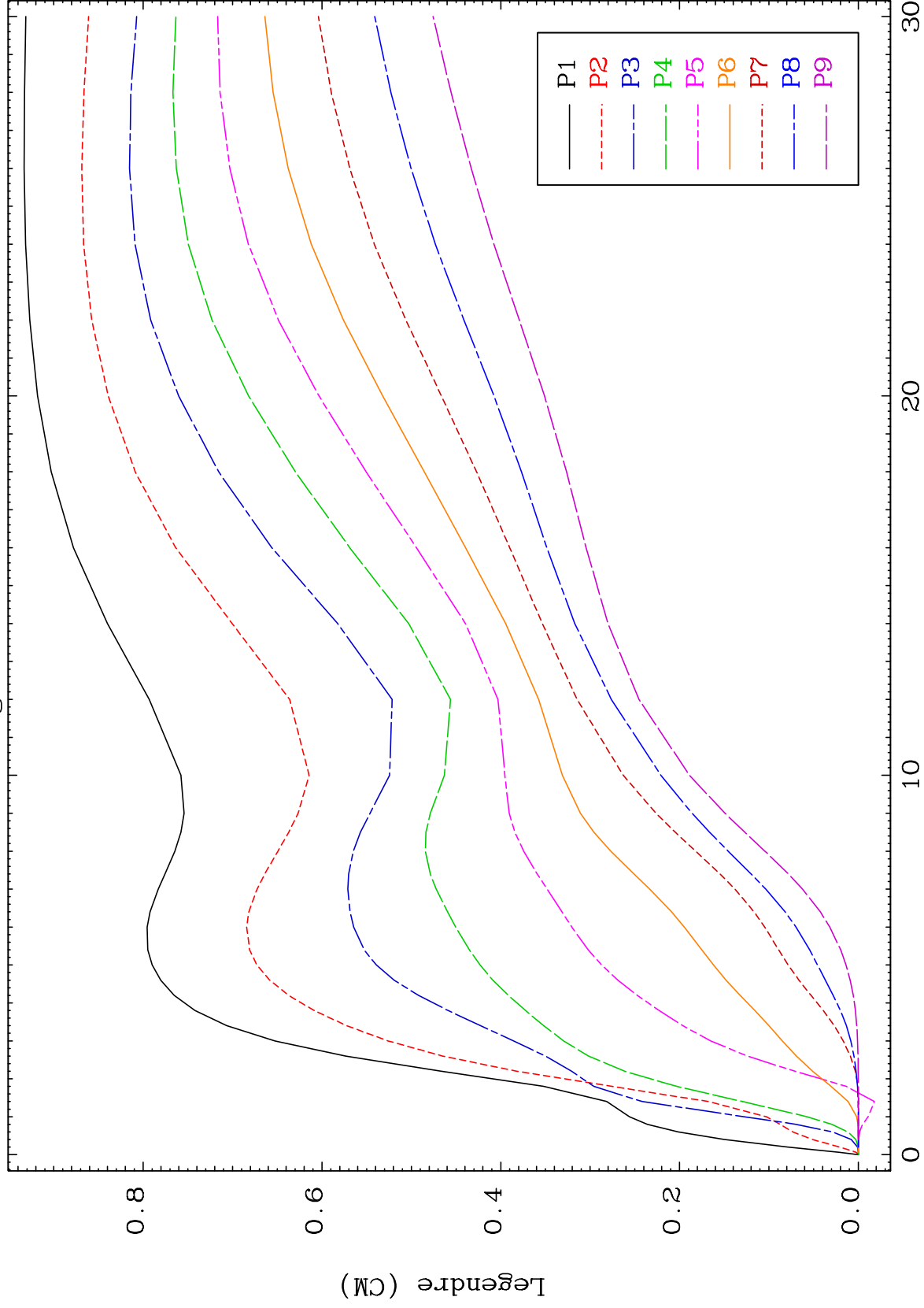




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Elastic Legendre Coefficients

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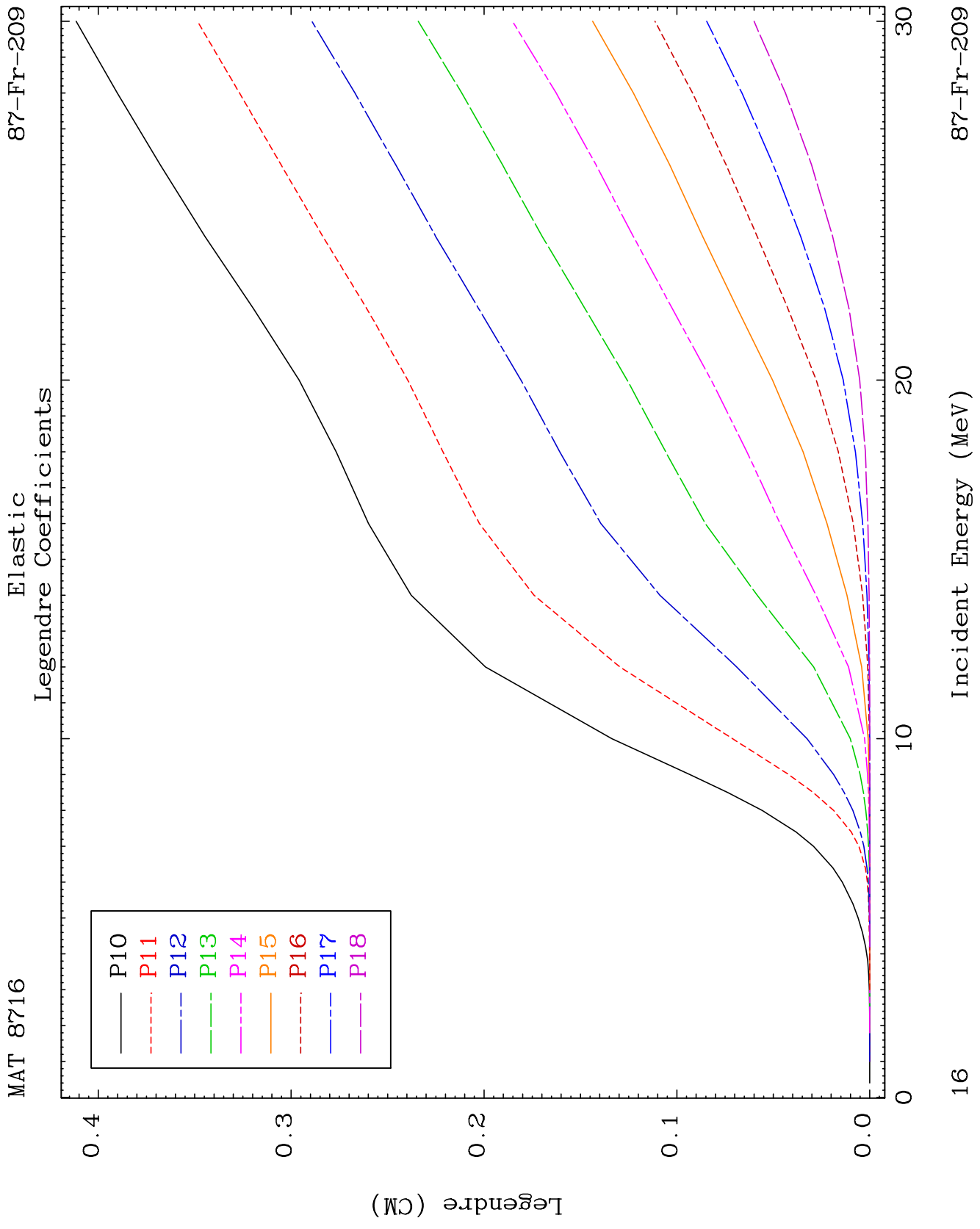


0

Incident Energy (MeV)

87-Fr-209

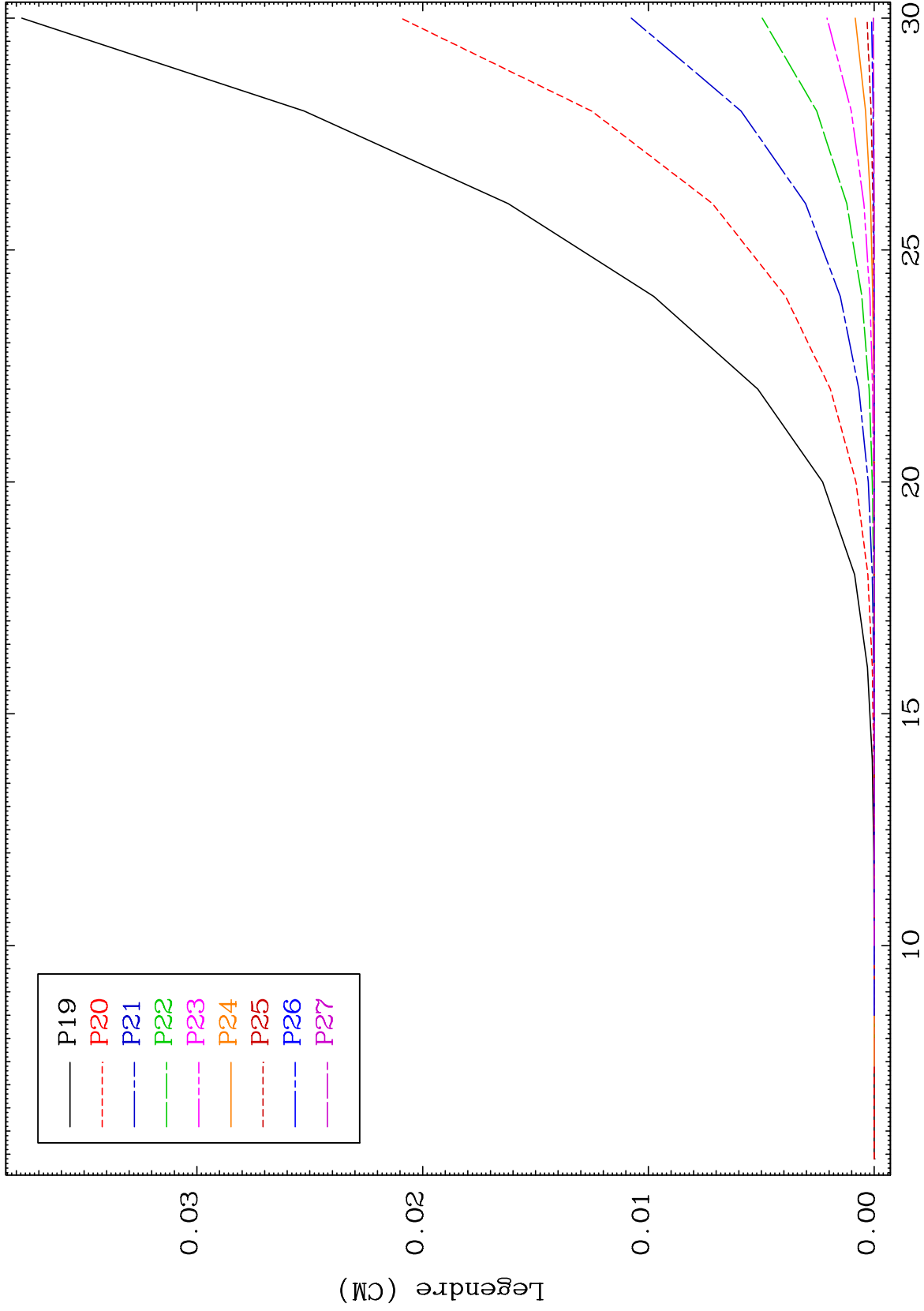




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Elastic Legendre Coefficients

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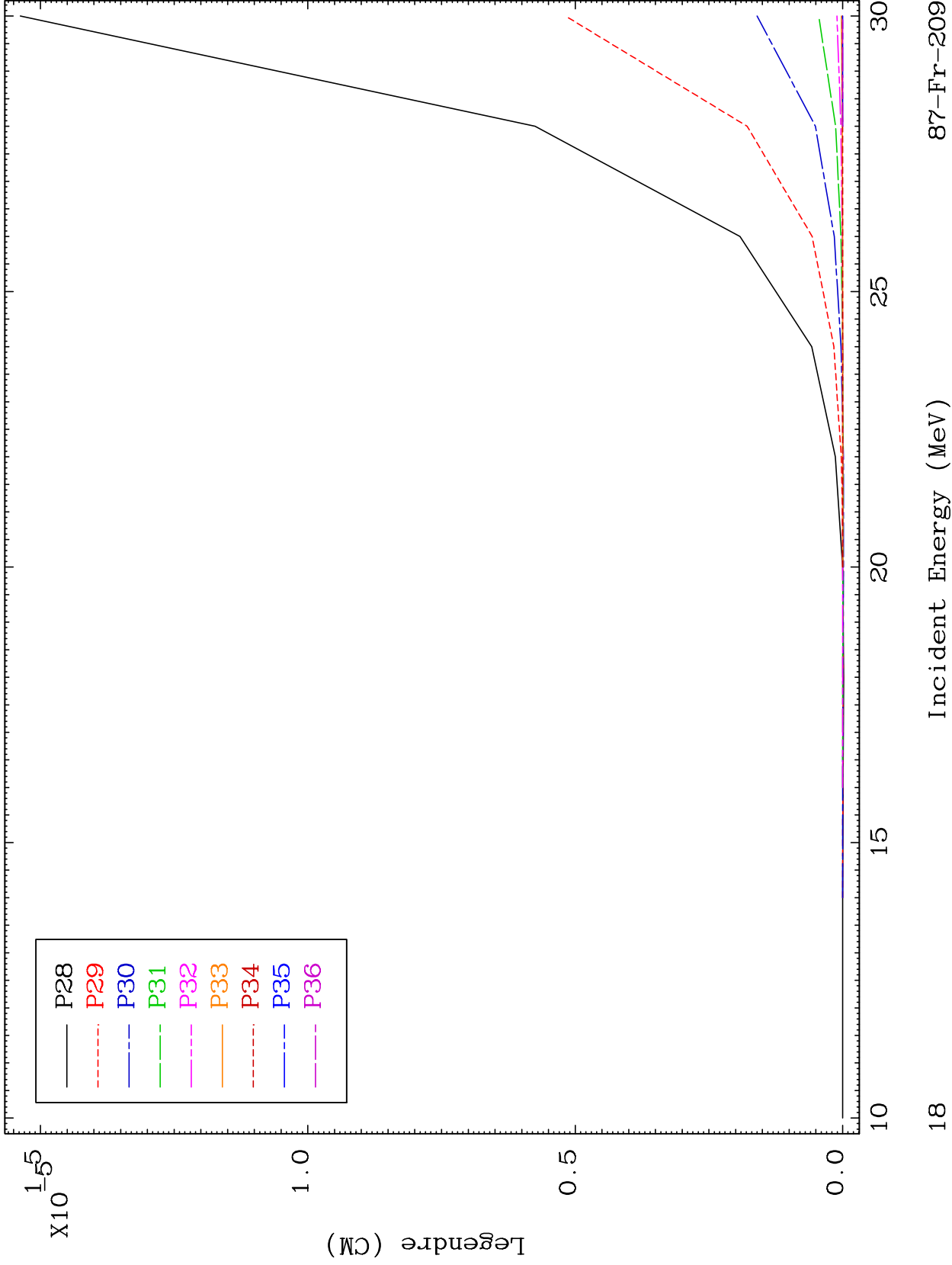
Incident Energy (MeV)

87-Fr-209

MAT 8716

Elastic  
Legendre Coefficients

87-Fr-209



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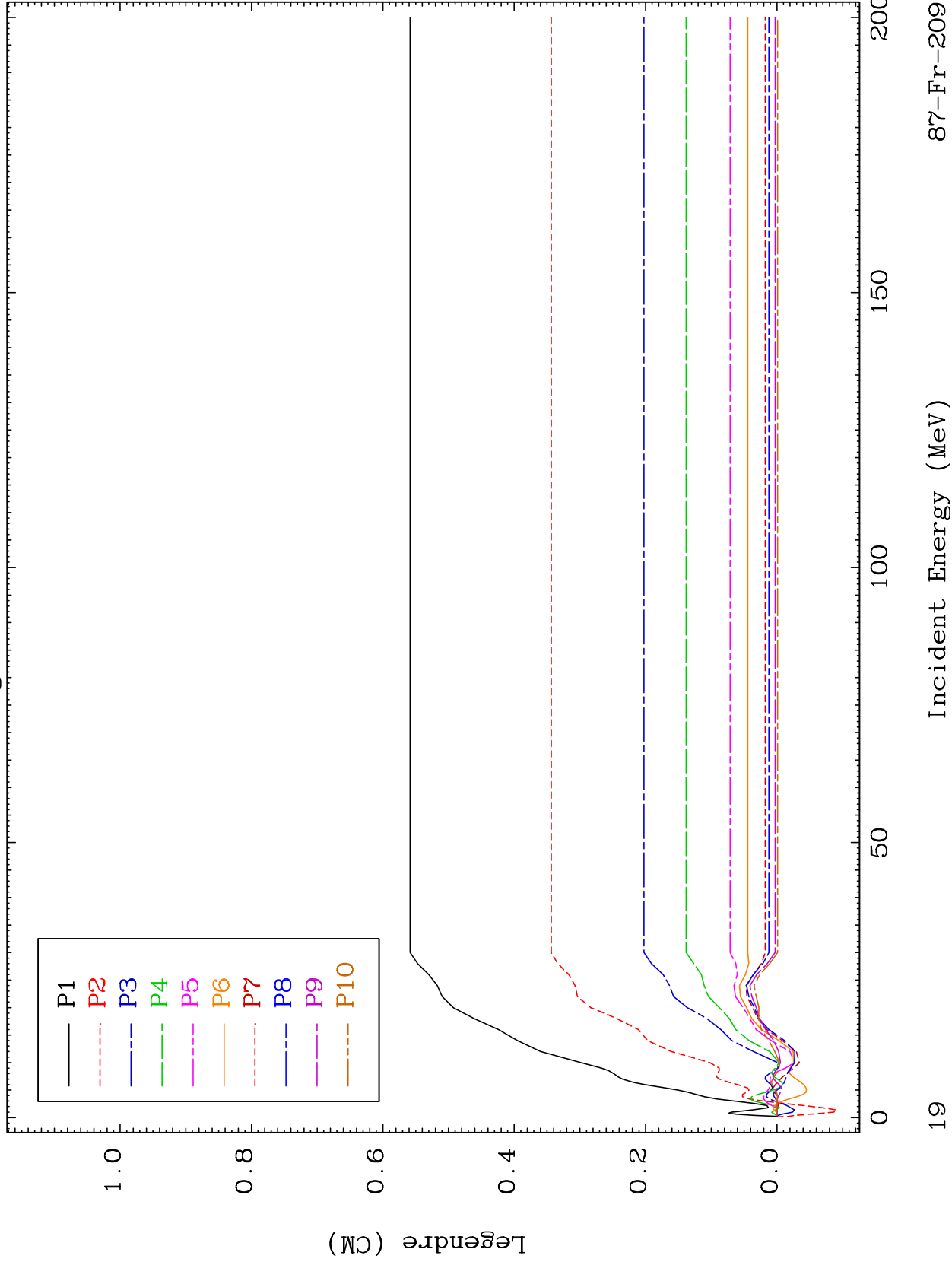
Incident Energy (MeV)

87-Fr-209

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124.4 keV (n, n') Level  
Legendre Coefficients

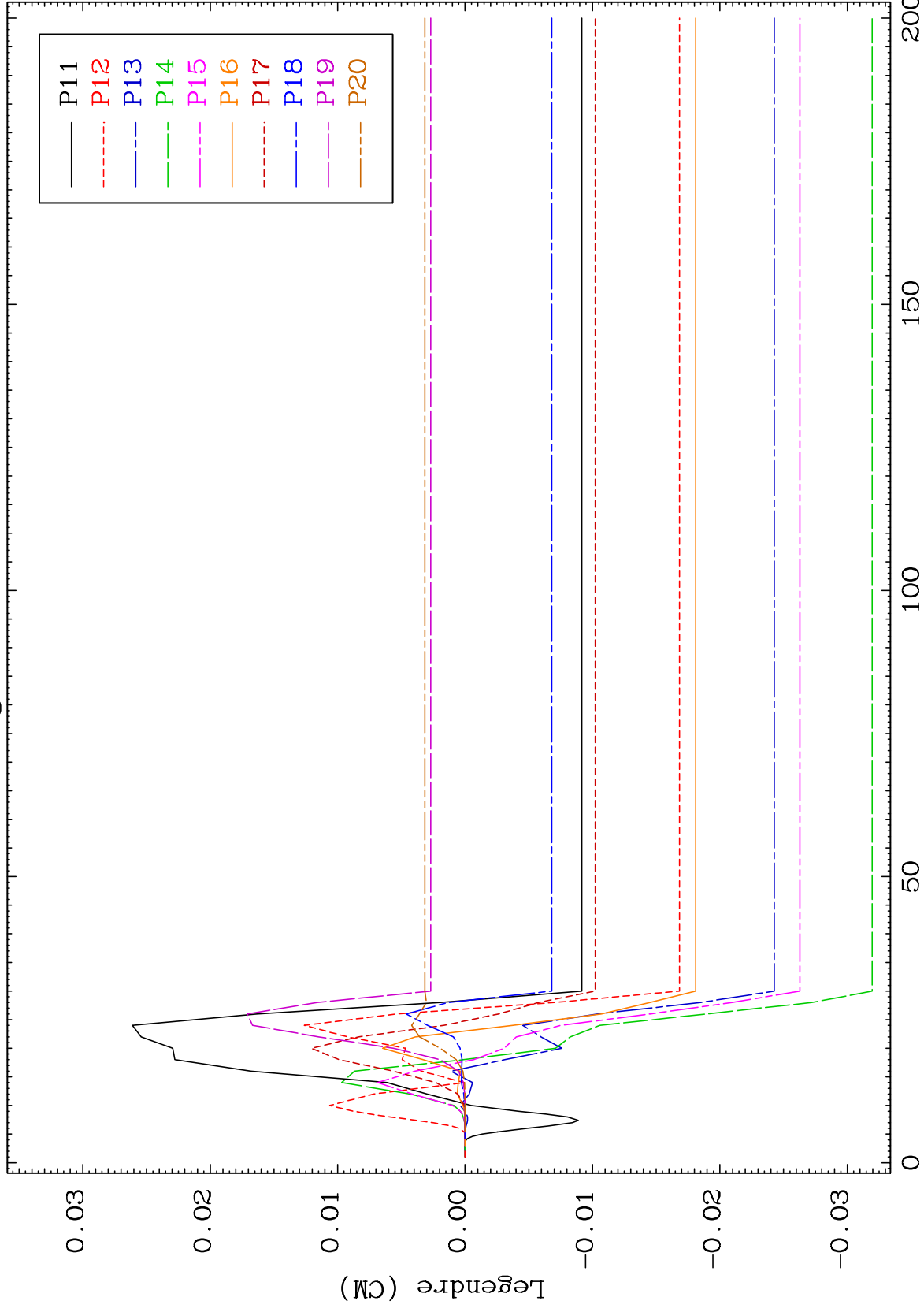
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MAT 8716

124.4 keV (n,n') Level  
Legendre Coefficients

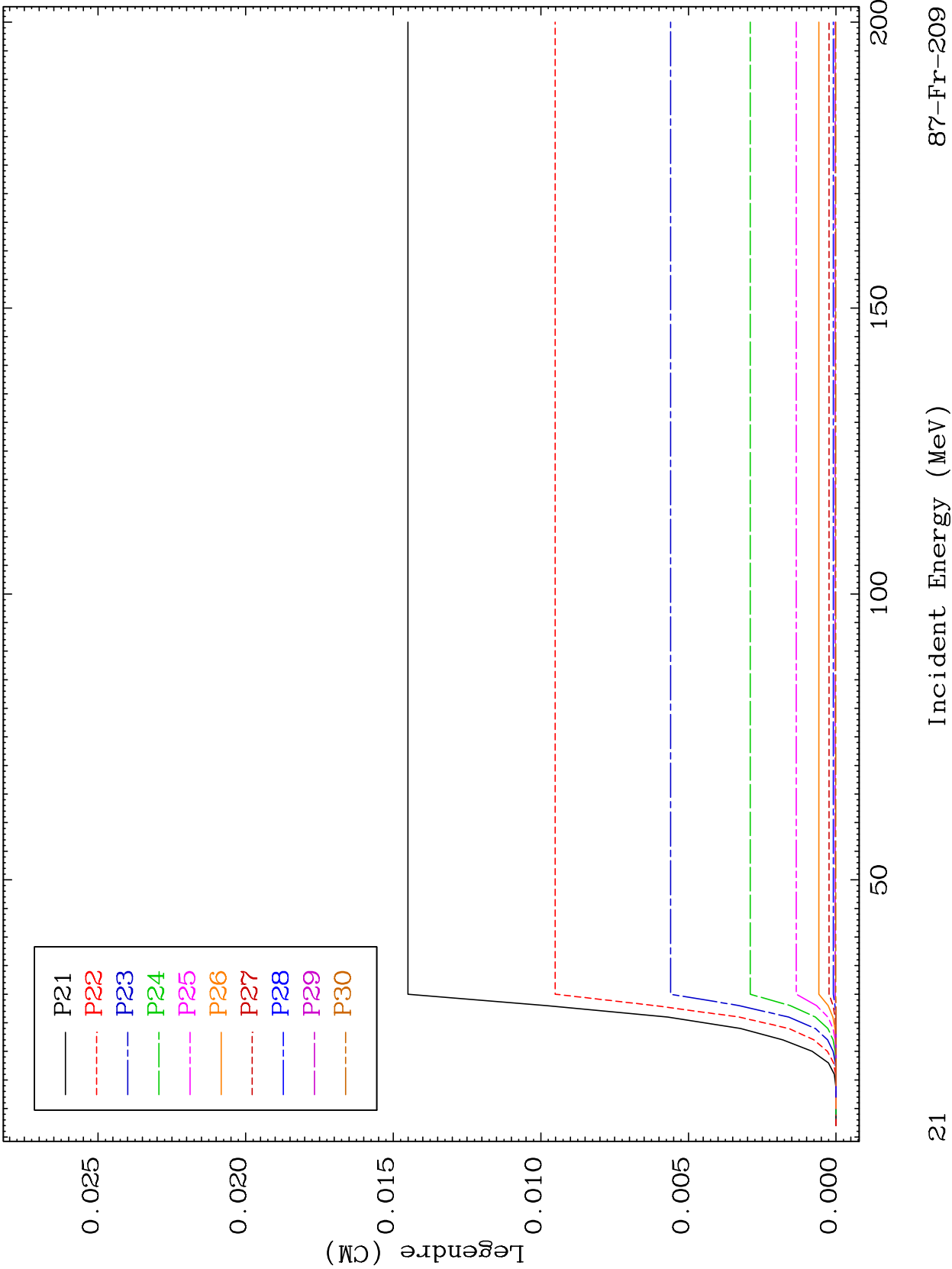
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87-Fr-209

Incident Energy (MeV)

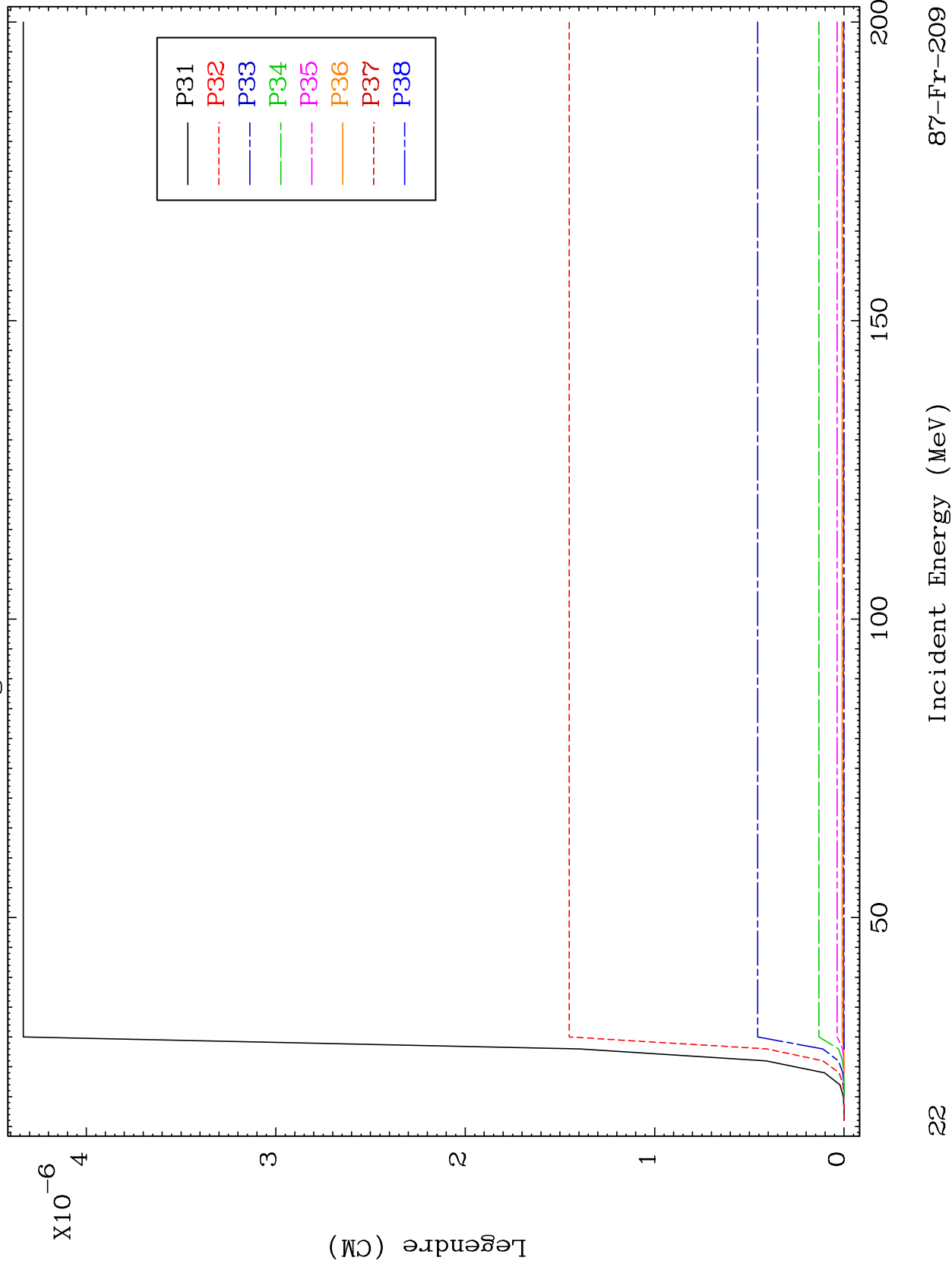
20



MAT 8716

124.4 keV (n,n') Level  
Legendre Coefficients

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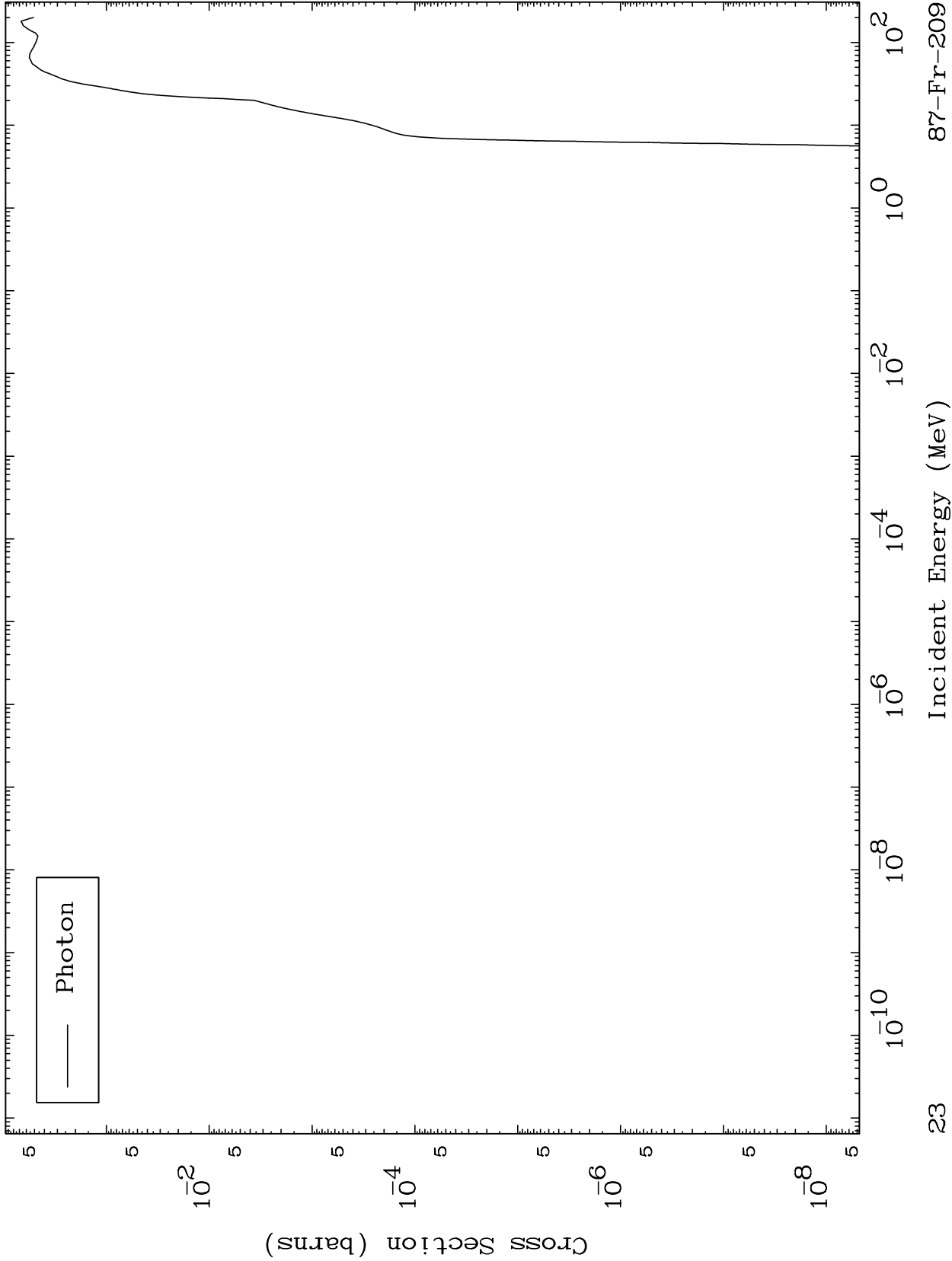


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Fission  
Radionuclide Production Cross Section

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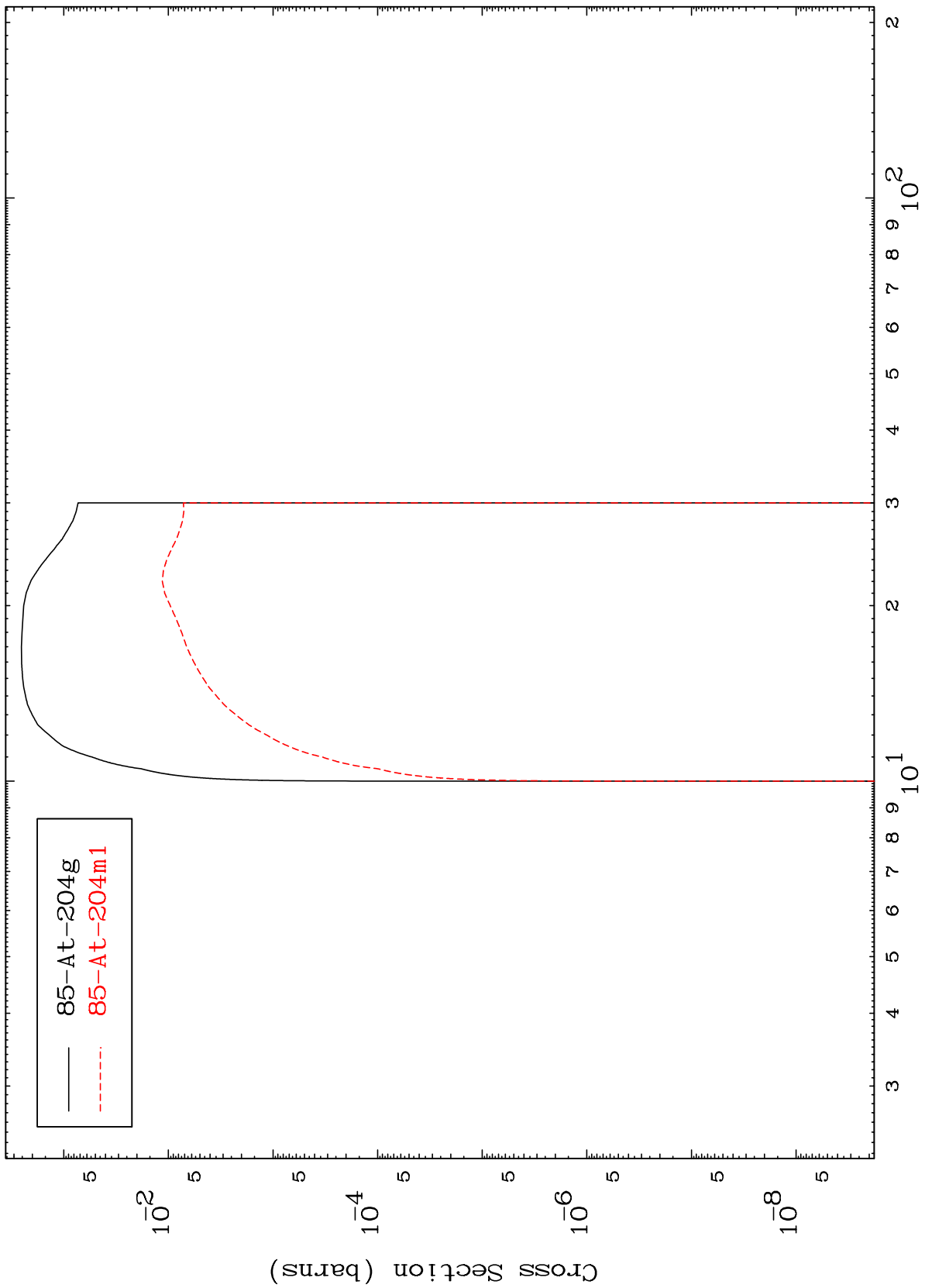


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(n,2n)  $\alpha$

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Radionuclide Production Cross Section



24

Incident Energy (MeV)

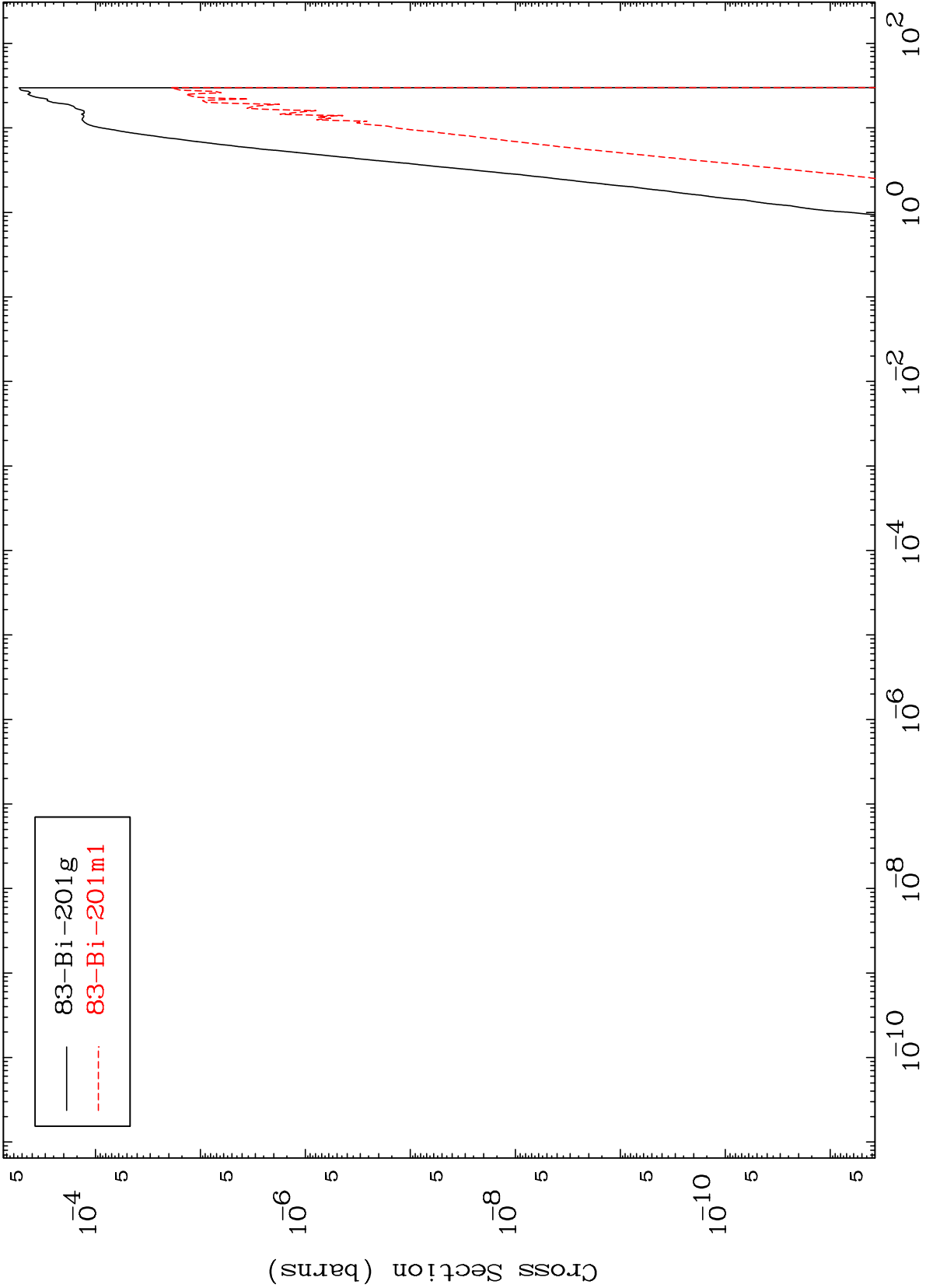
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MAT 8716

(n,n') 2 $\alpha$

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Radionuclide Production Cross Section



25

Incident Energy (MeV)

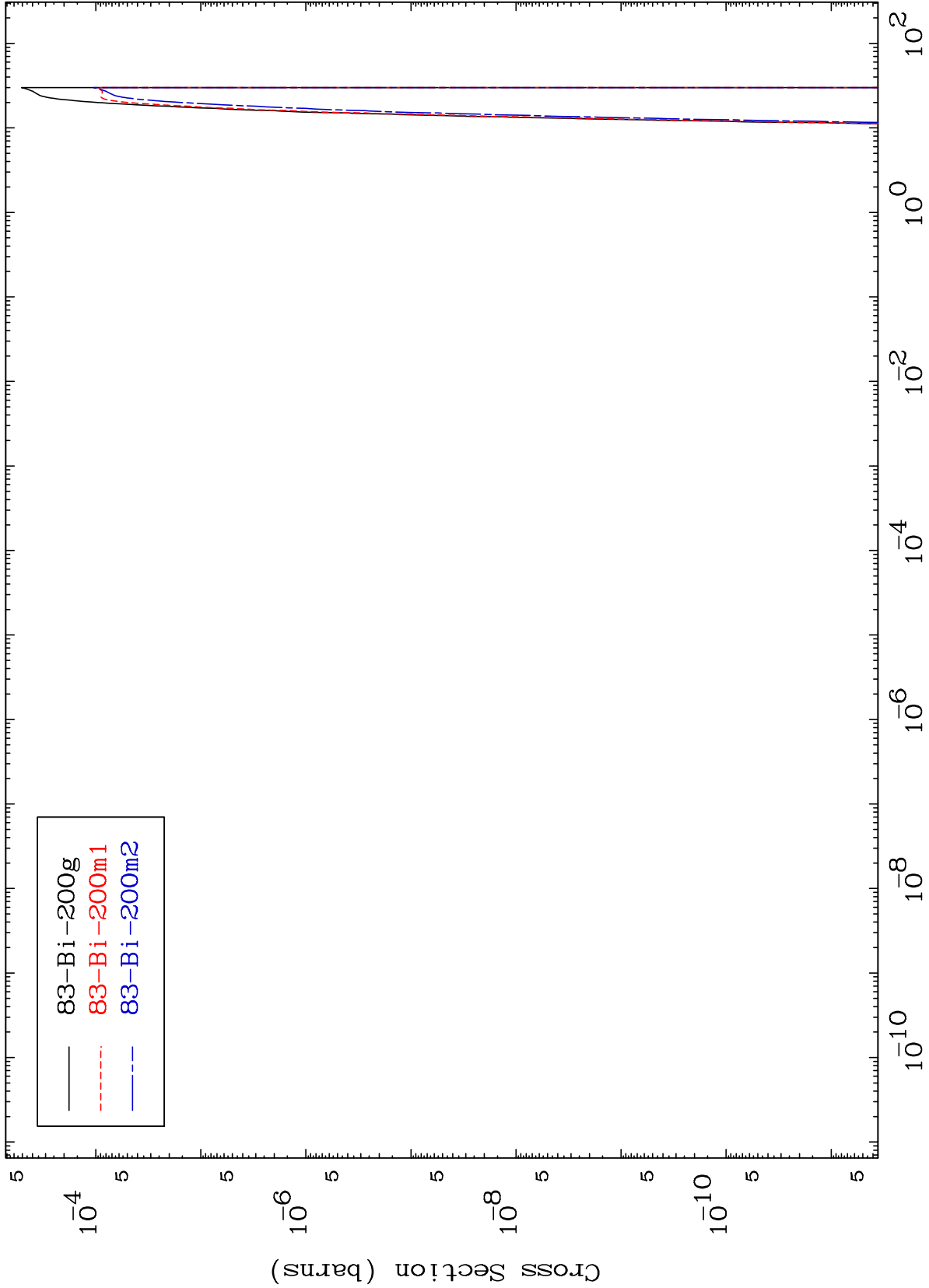
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MAT 8716

(n,2n) 2α

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Radionuclide Production Cross Section



26

Incident Energy (MeV)

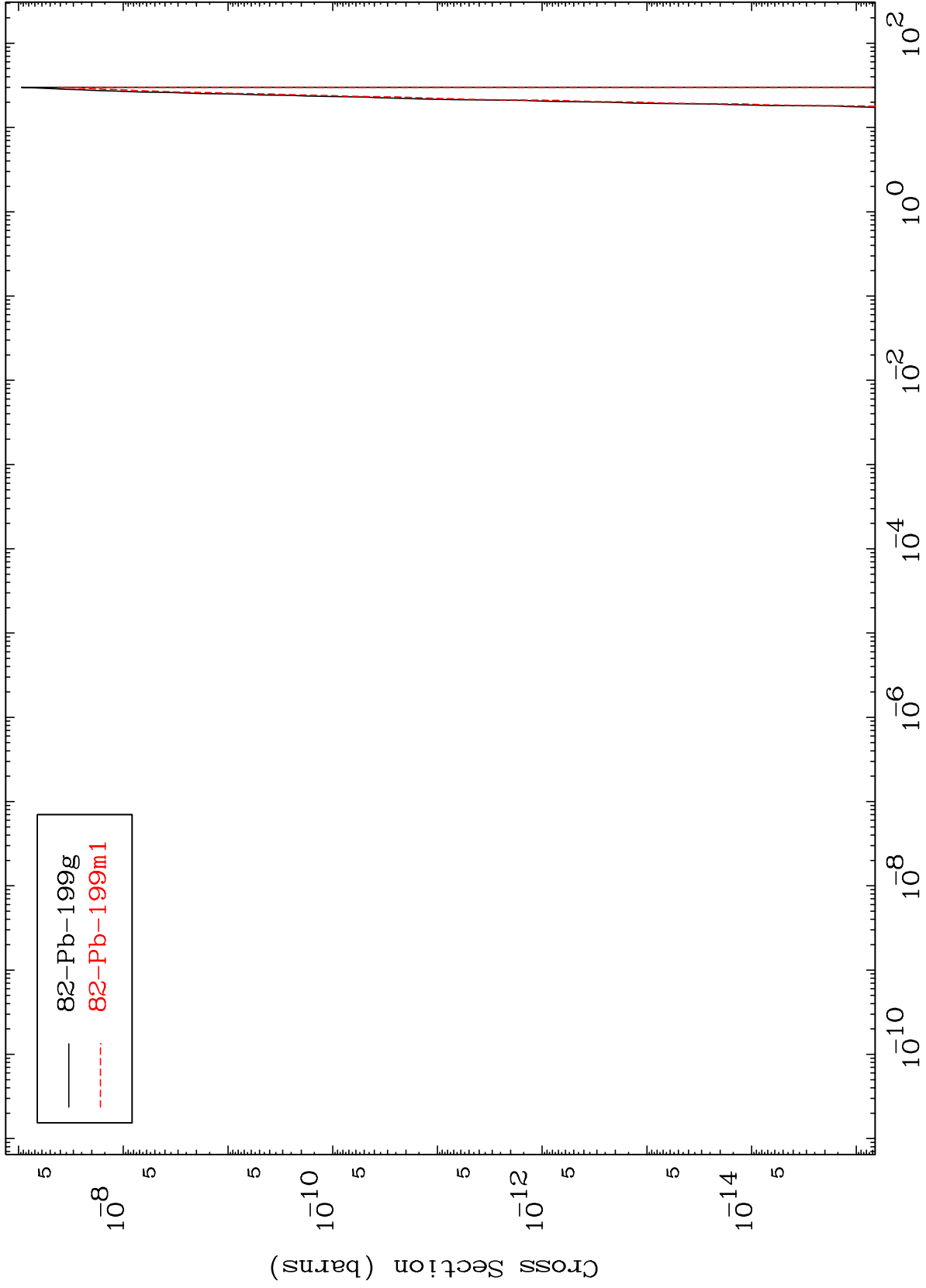
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MAT 8716

(n,n') d,2α

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Radionuclide Production Cross Section



27

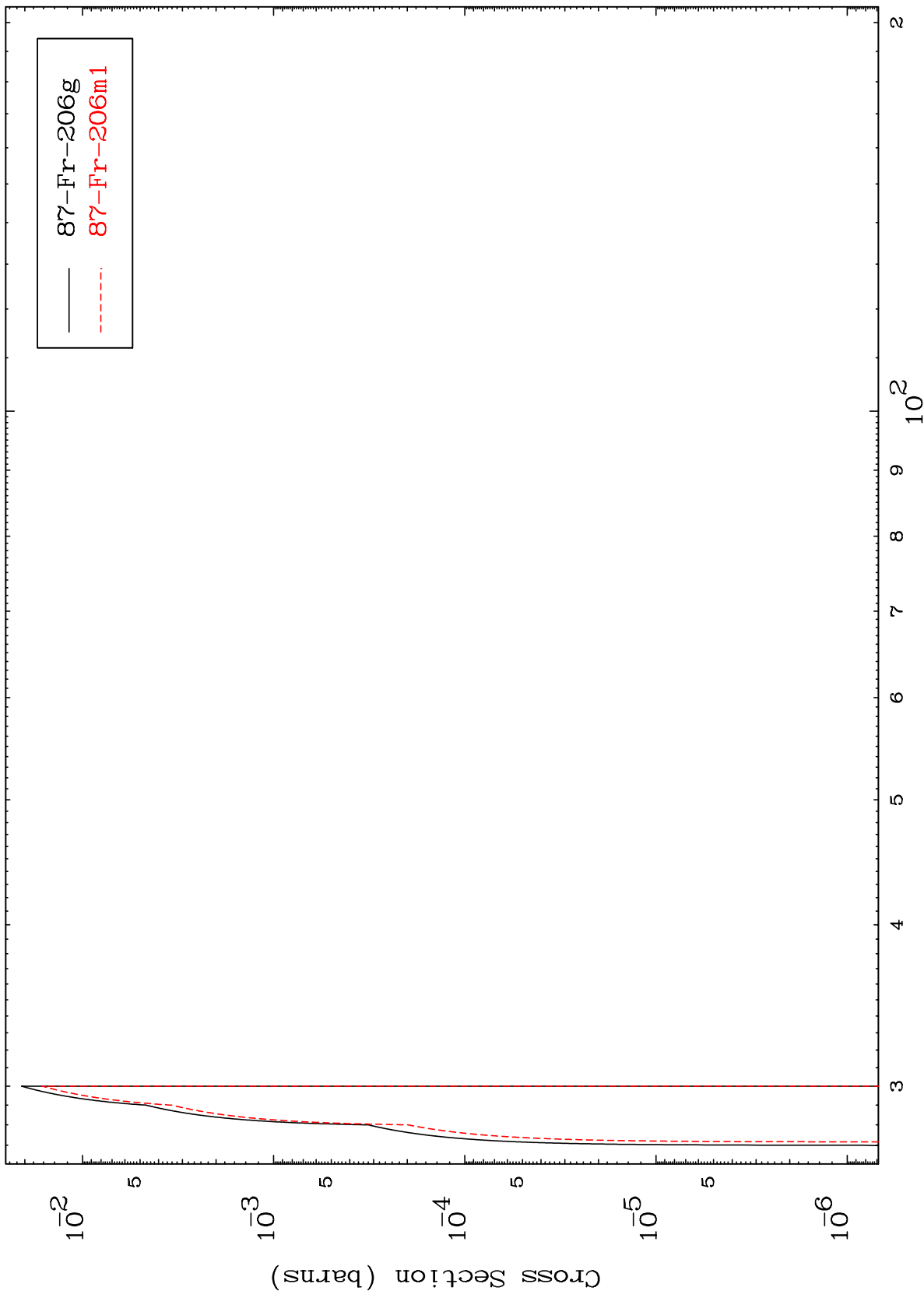
Incident Energy (MeV)

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(n,4n)  
Radionuclide Production Cross Section



28

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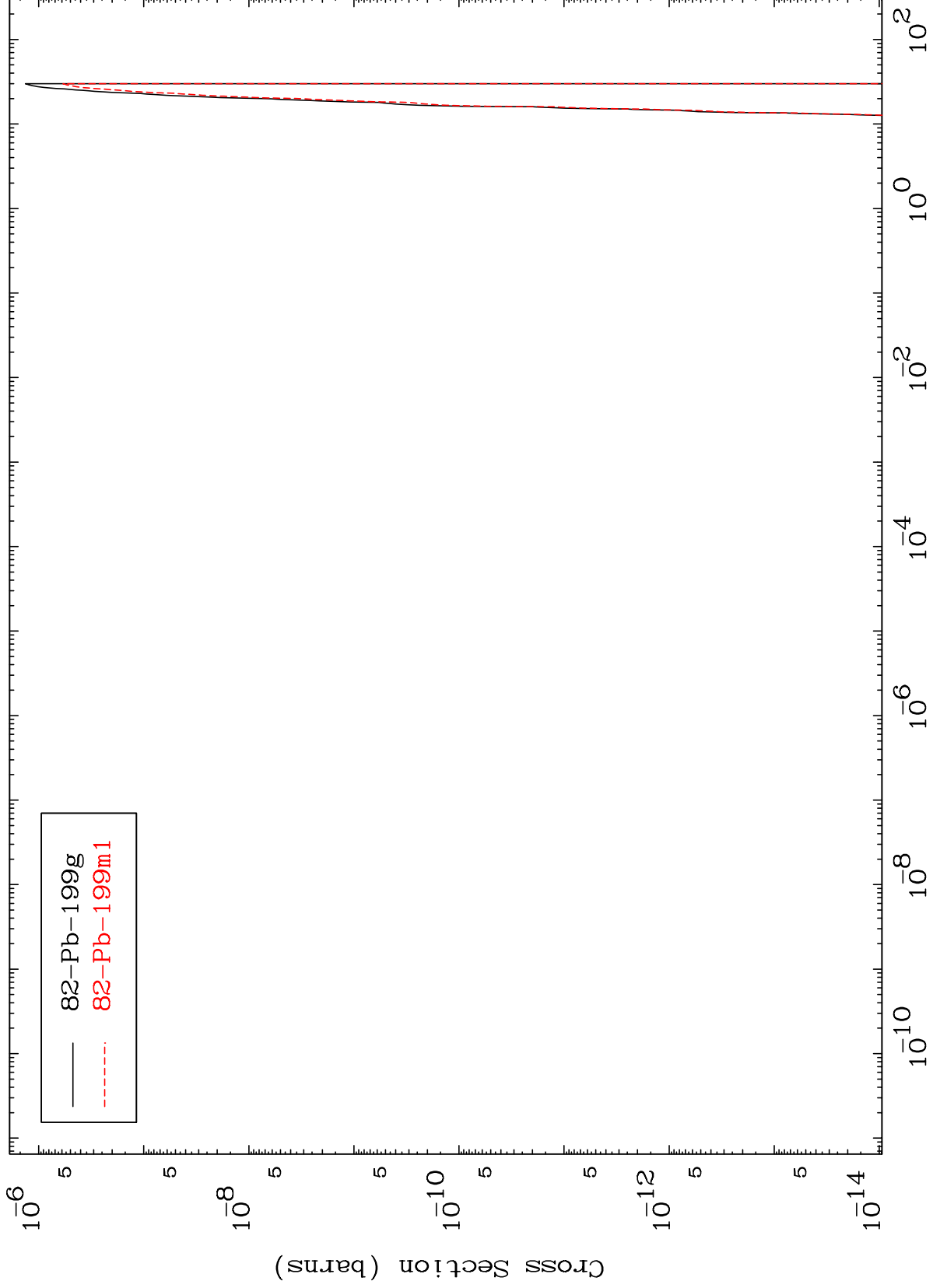
Incident Energy (MeV)

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(n,t) 2 $\alpha$

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Radionuclide Production Cross Section



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Incident Energy (MeV)

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